

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

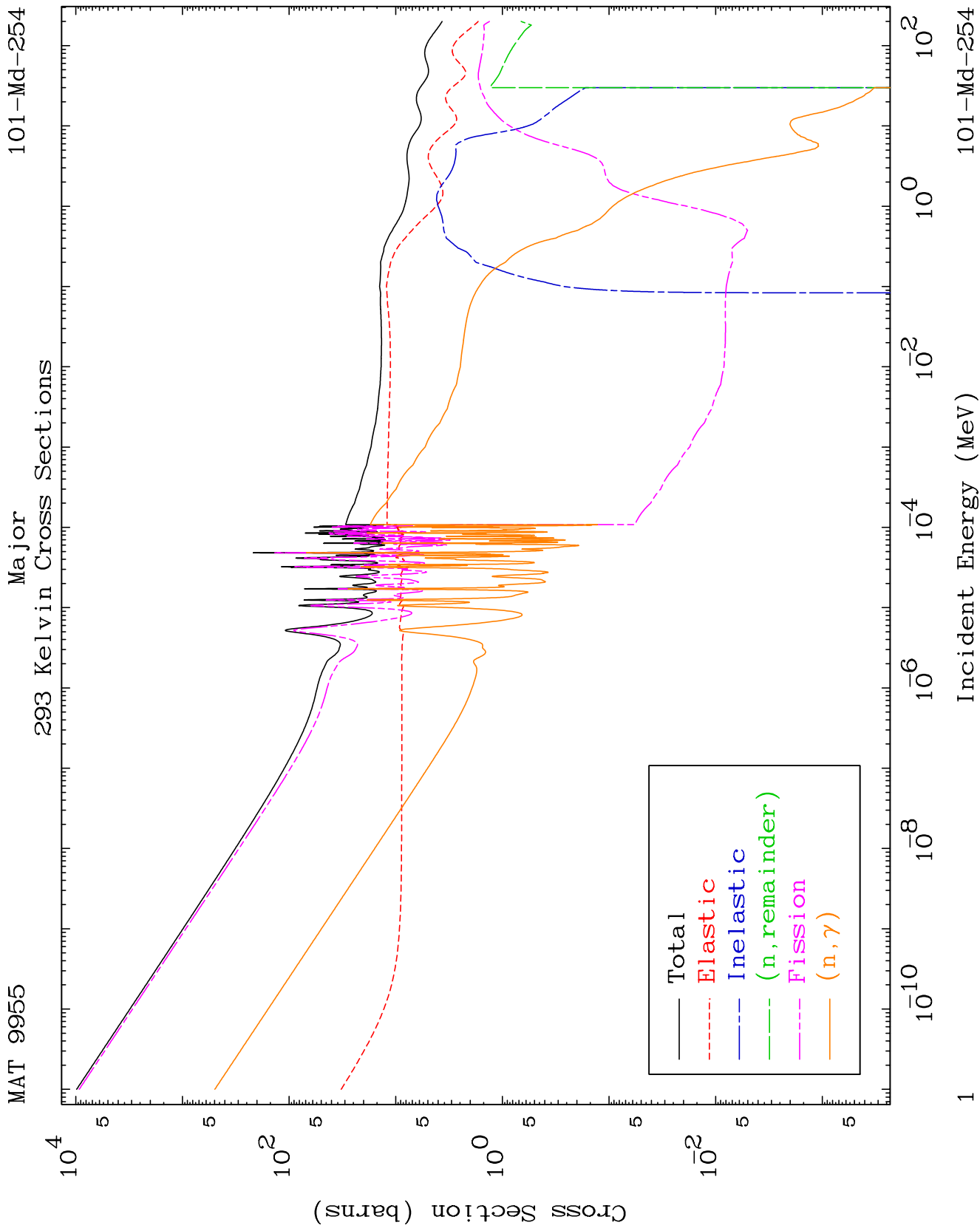
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Web:redcullen1.net/HOMEPAGE.NEW

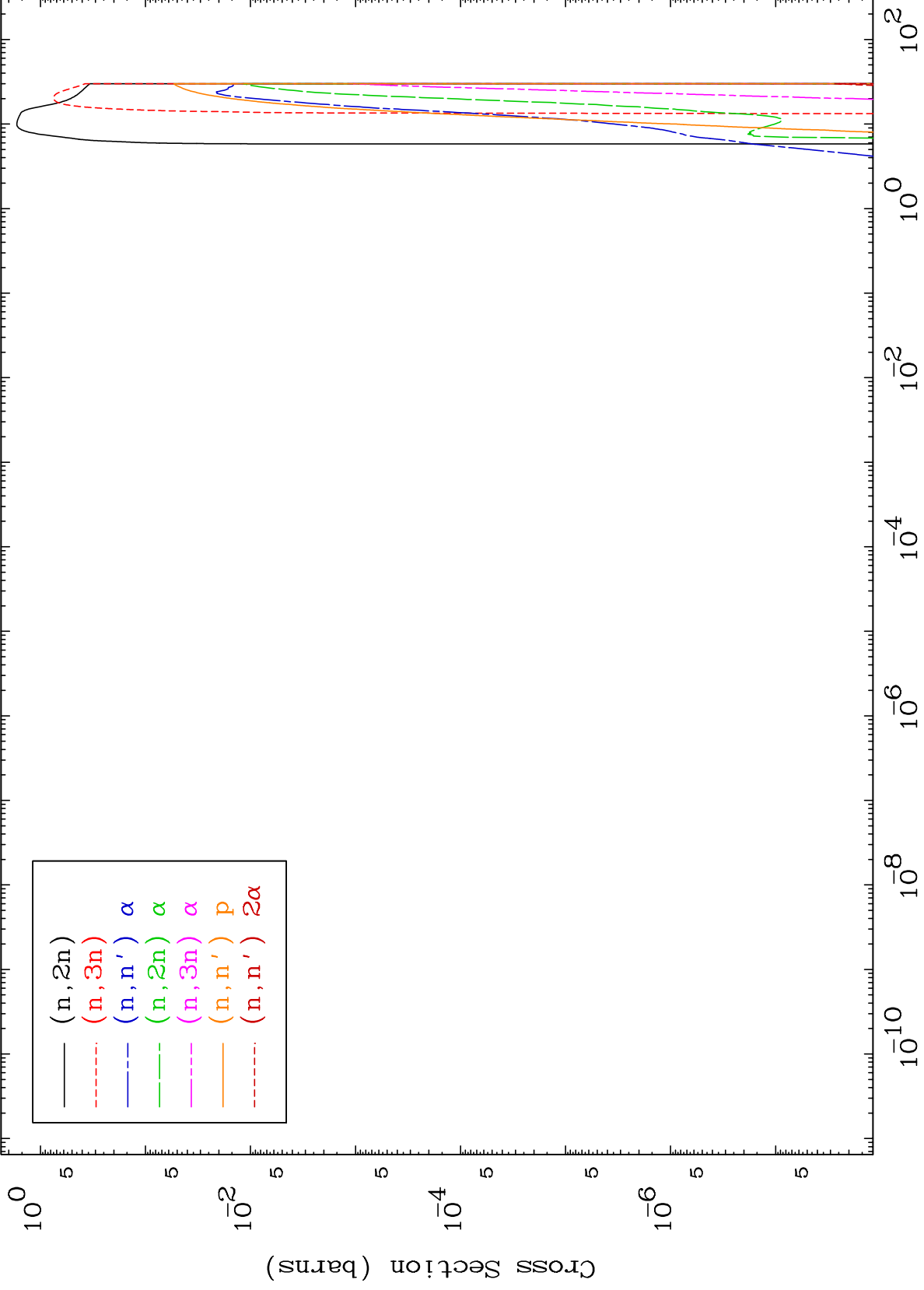
Press Mouse Button to Start

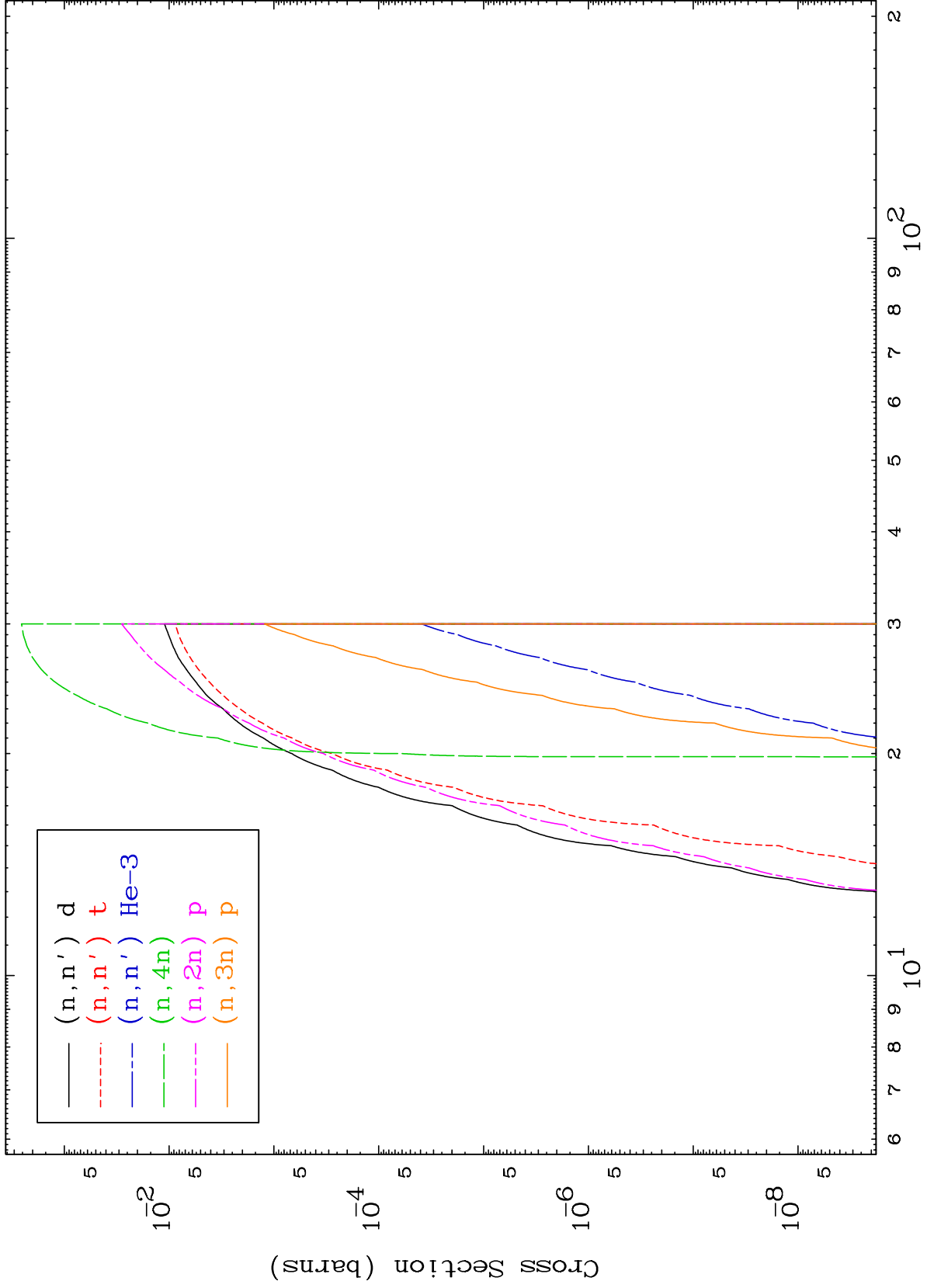


MAT 9955

Neutron Production
293 Kelvin Cross Sections

101-Md-254

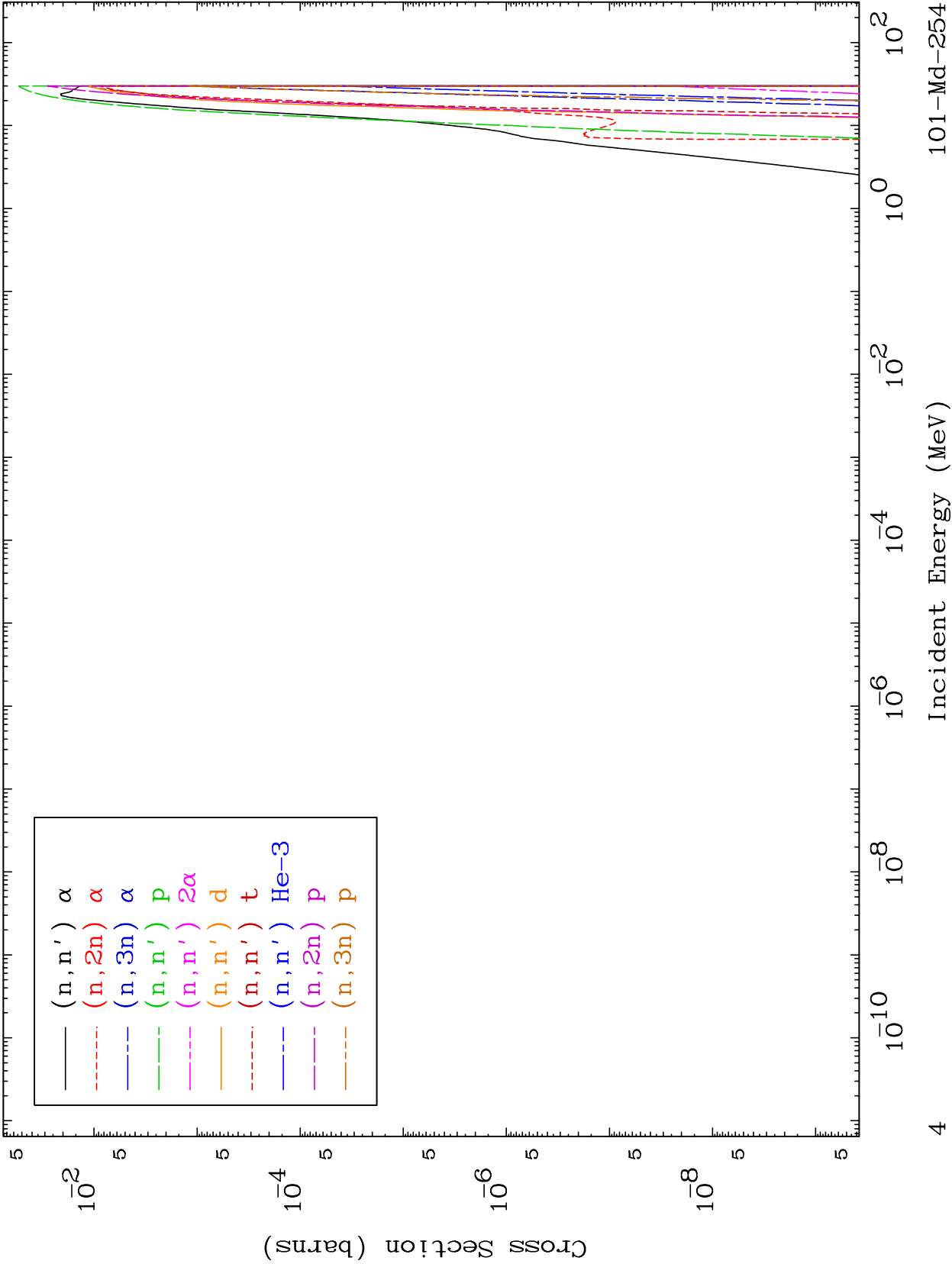




MAT 9955

Charged Particle
293 Kelvin Cross Sections

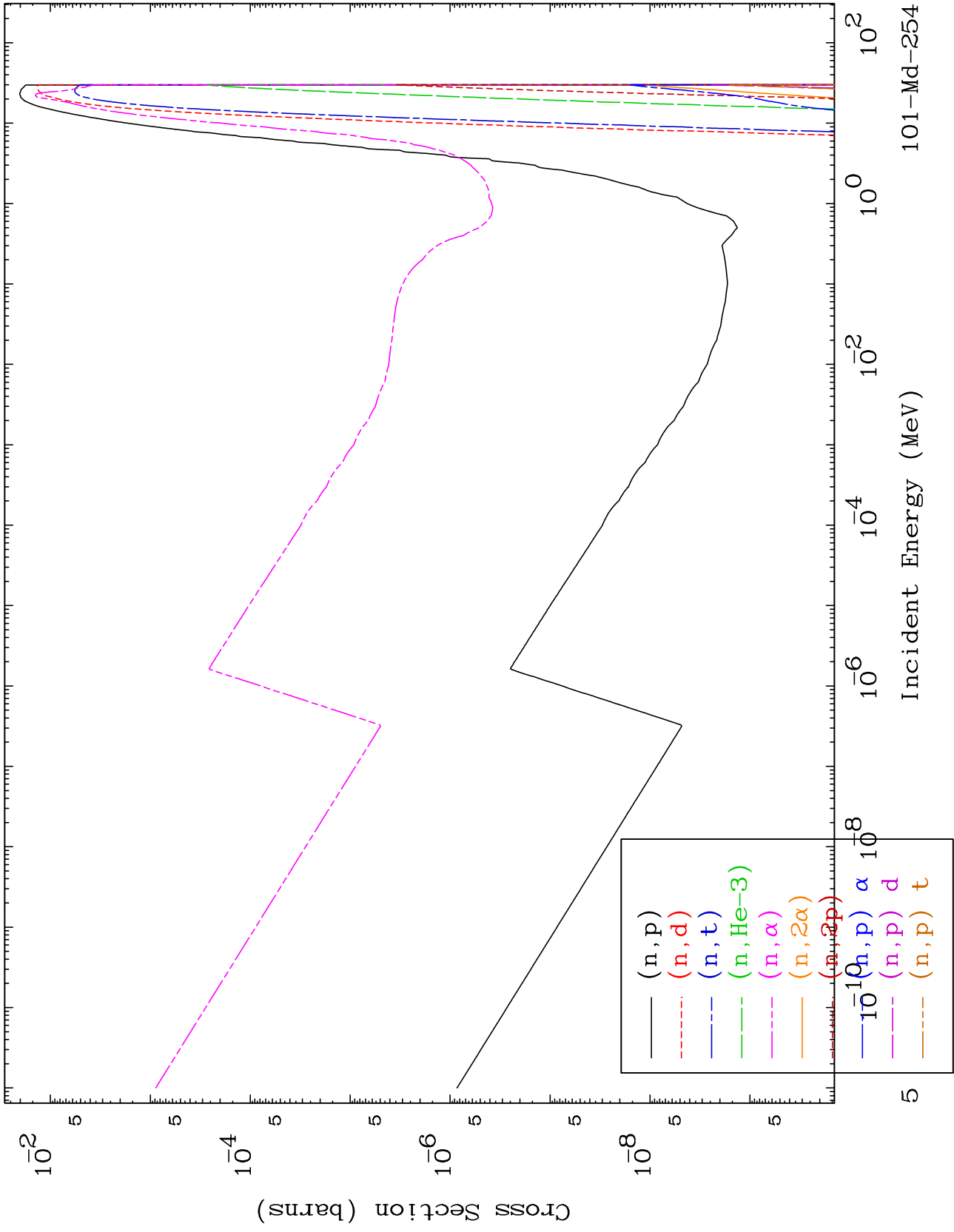
101-Md-254



MAT 9955

Charged Particle
293 Kelvin Cross Sections

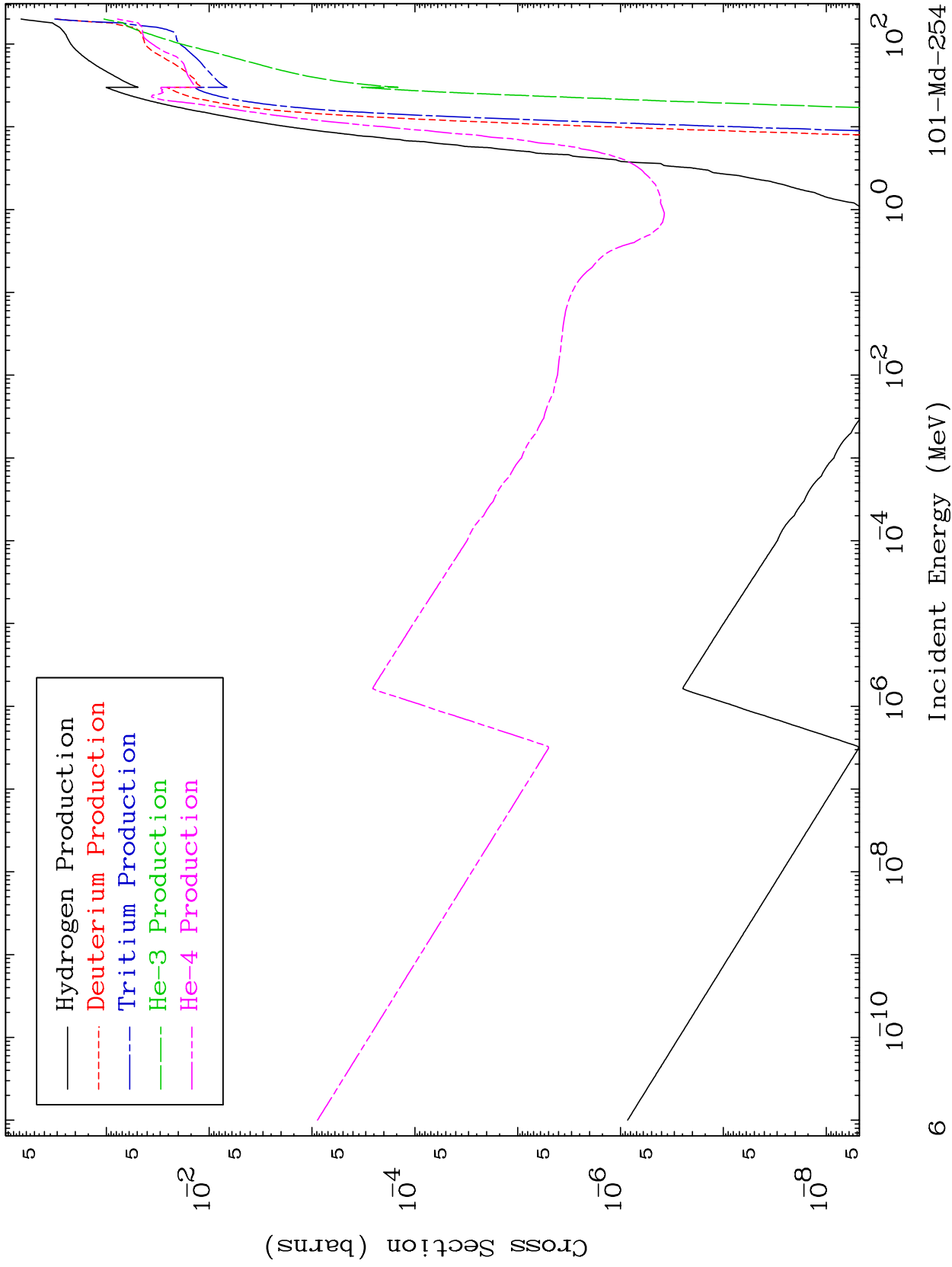
101-Md-254



MAT 9955

Particle Production
293 Kelvin Cross Sections

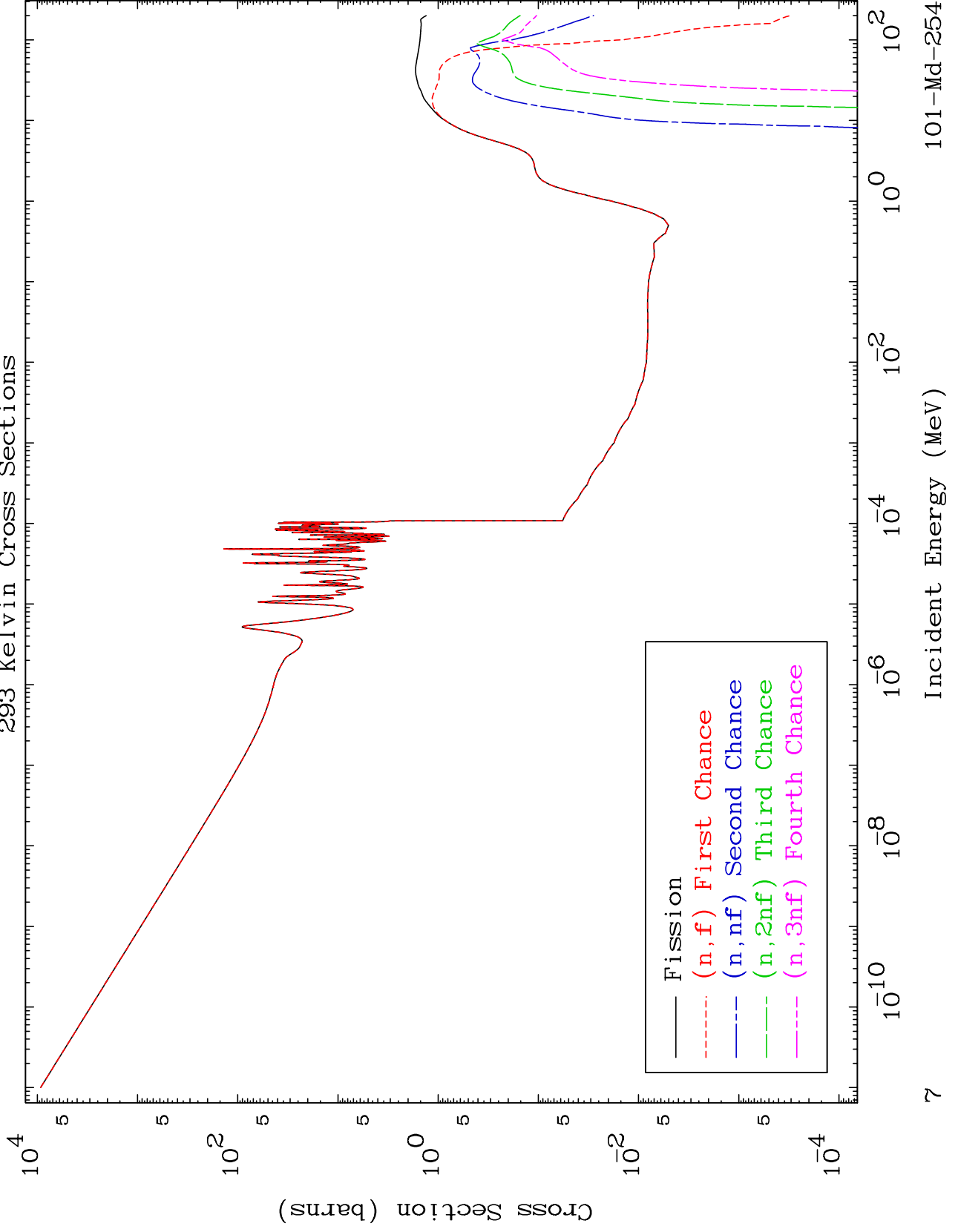
101-Md-254



MAT 9955

Fission
293 Kelvin Cross Sections

101-Md-254

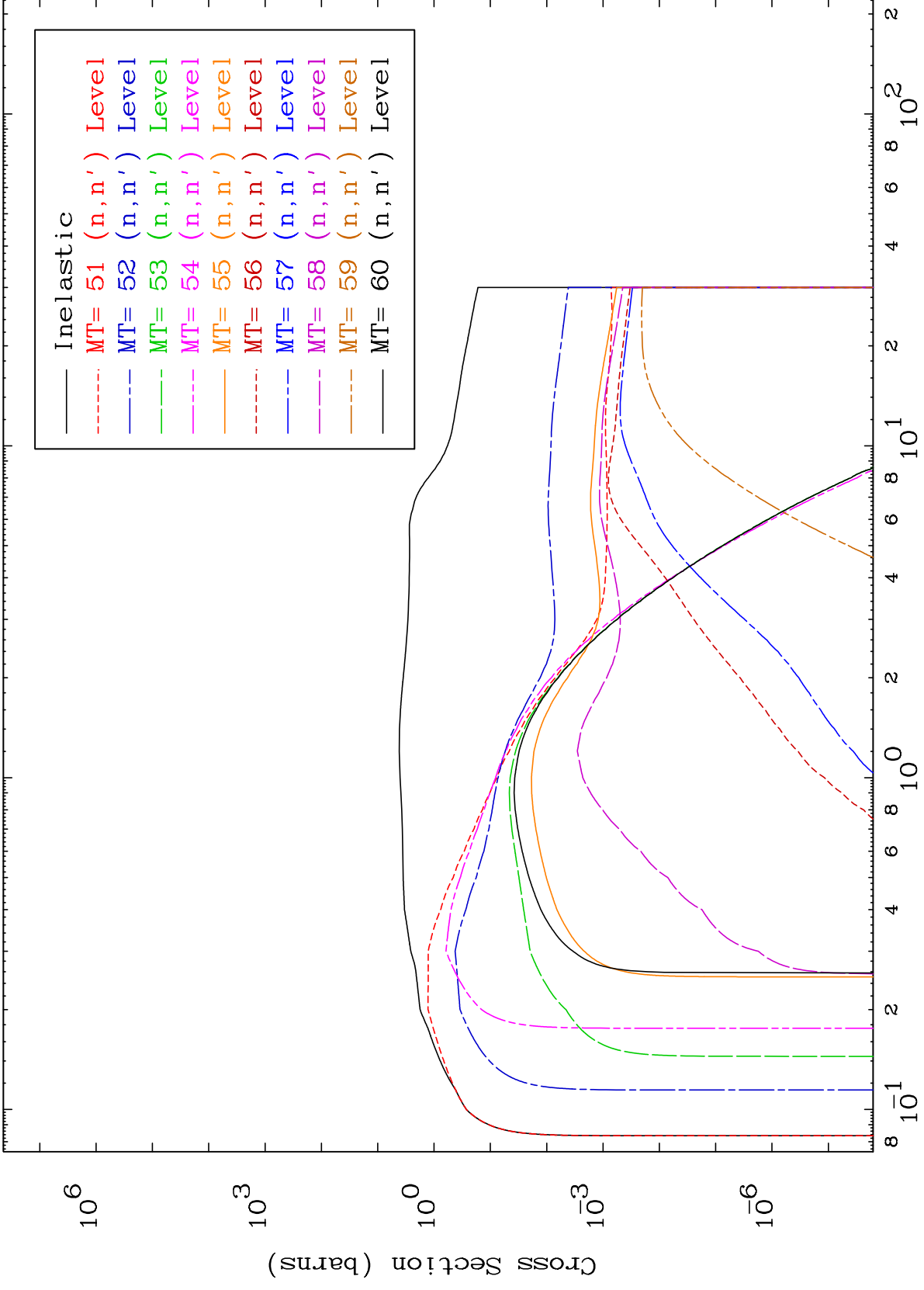


MAT 9955

(n,n') Level

101-Md-254

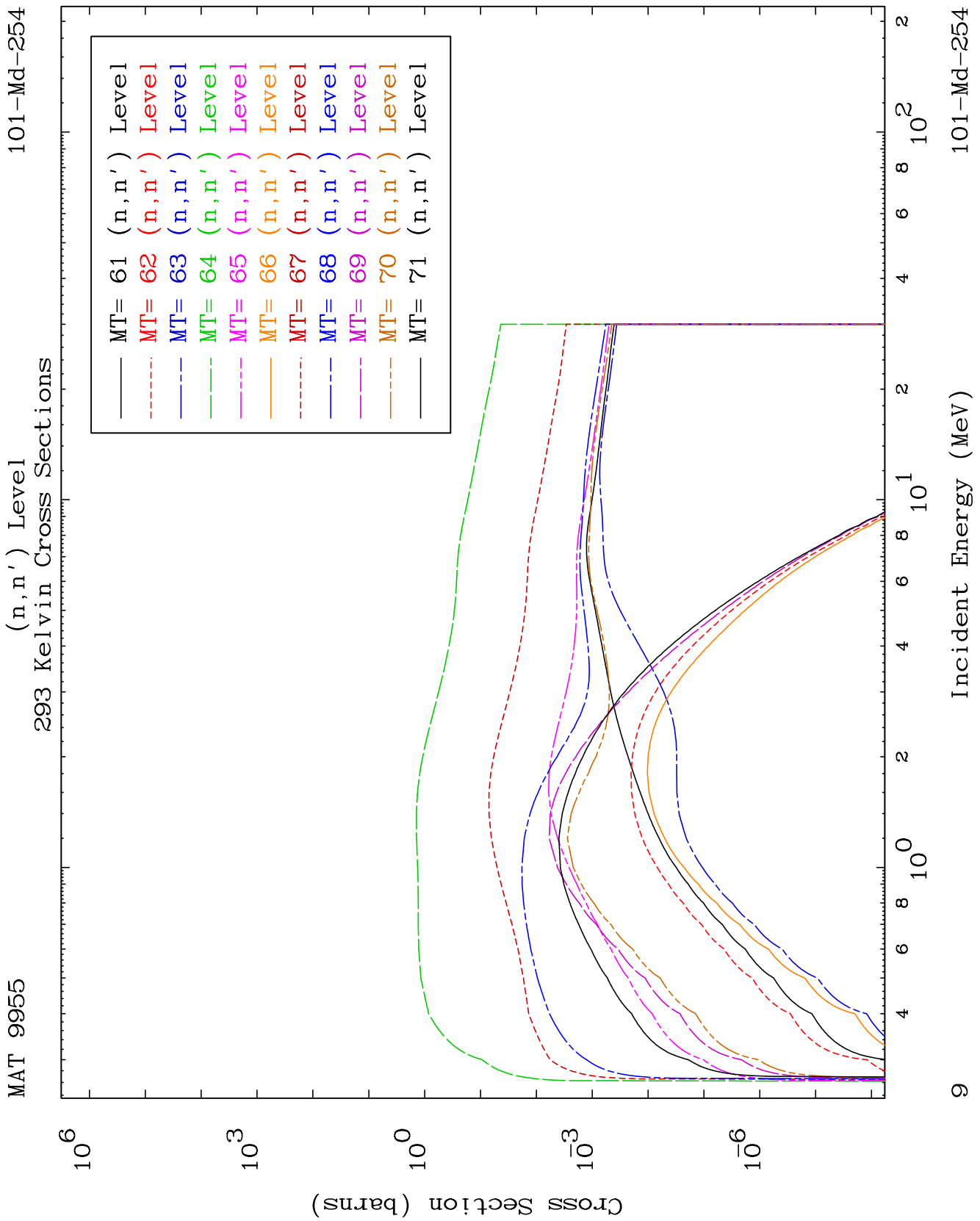
293 Kelvin Cross Sections



8

Incident Energy (MeV)

101-Md-254

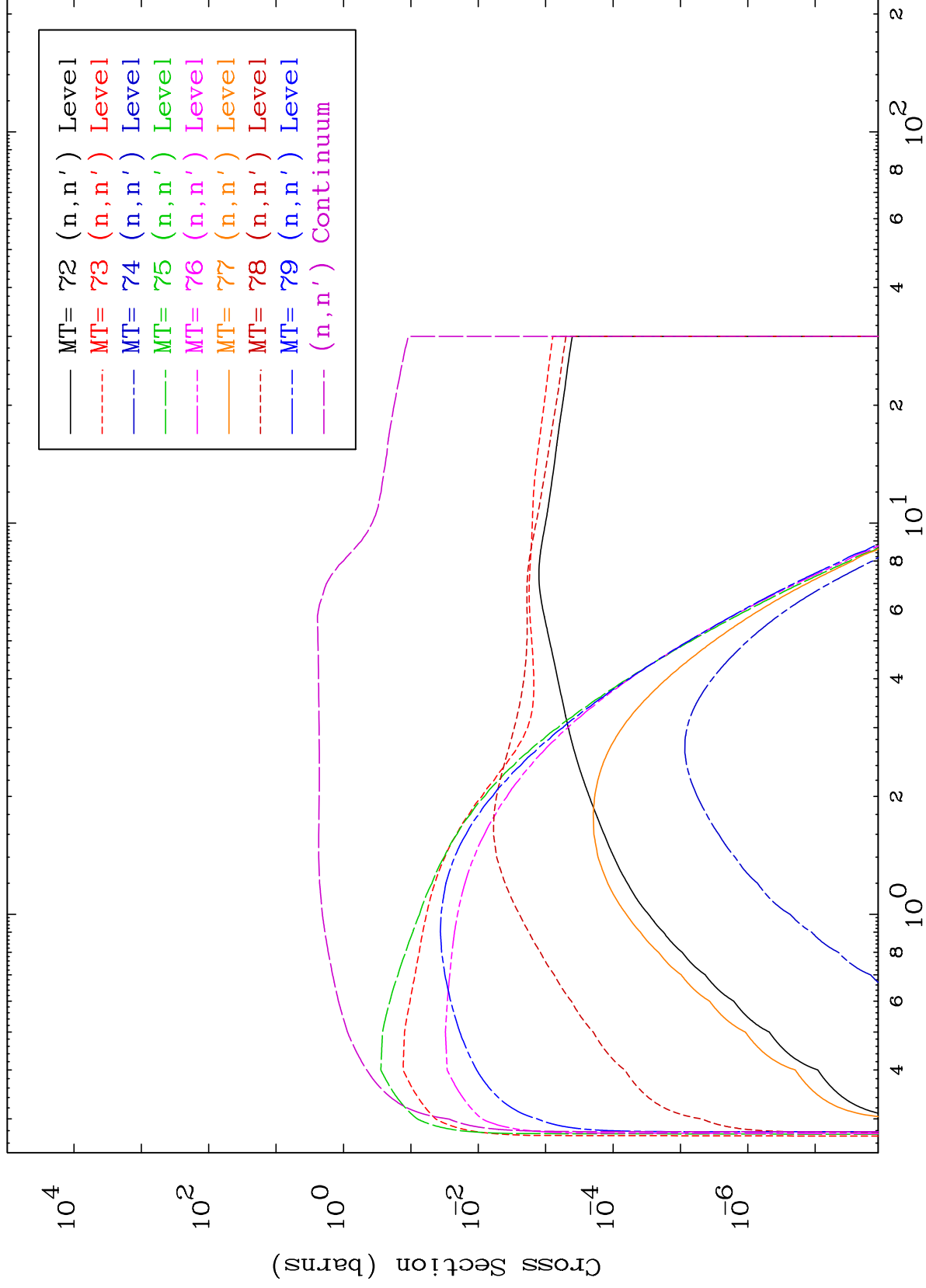


MAT 9955

(n,n') Level

101-Md-254

293 Kelvin Cross Sections



10

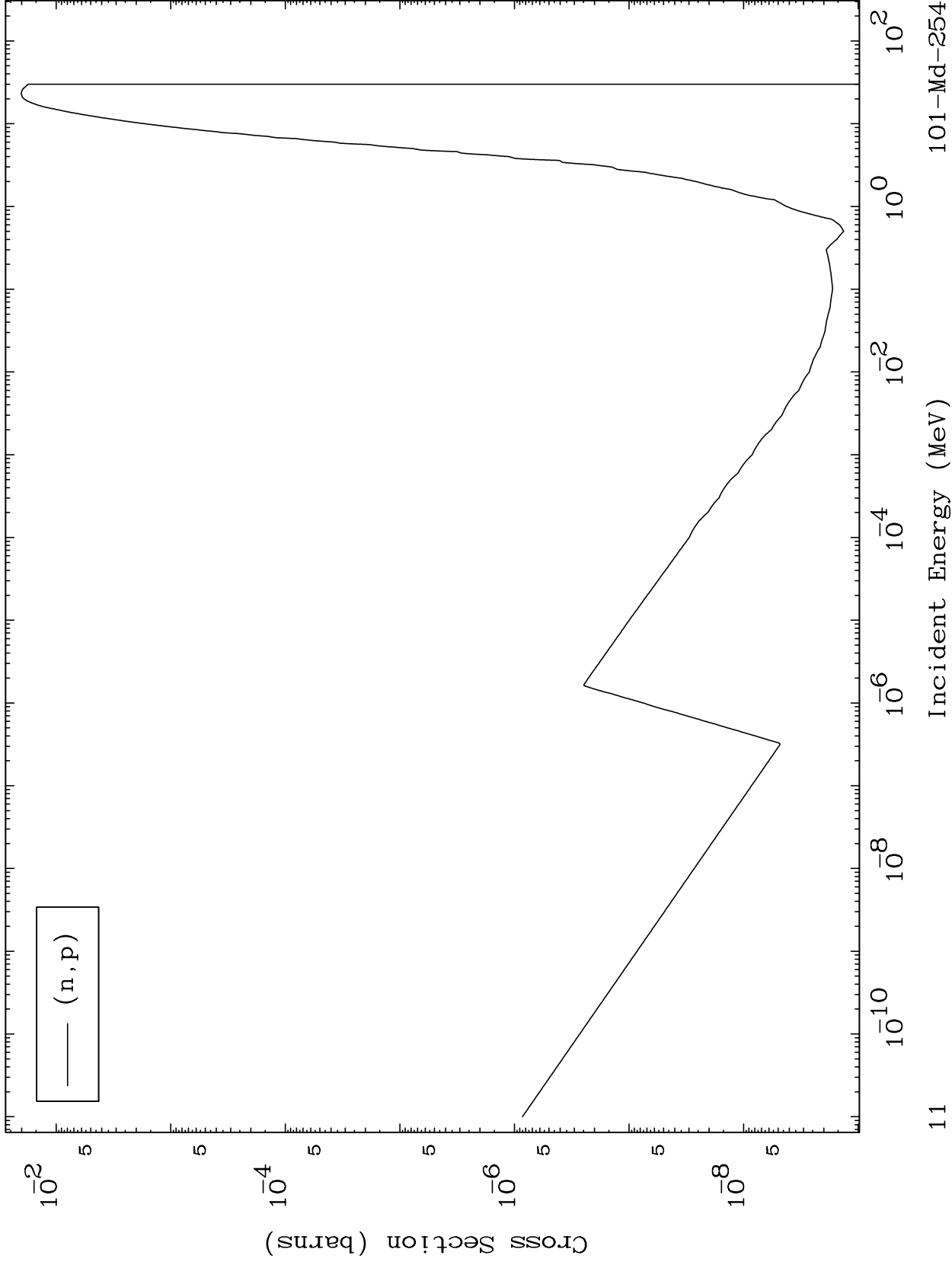
Incident Energy (MeV)

101-Md-254

MAT 9955

(n,p) Levels
293 Kelvin Cross Sections

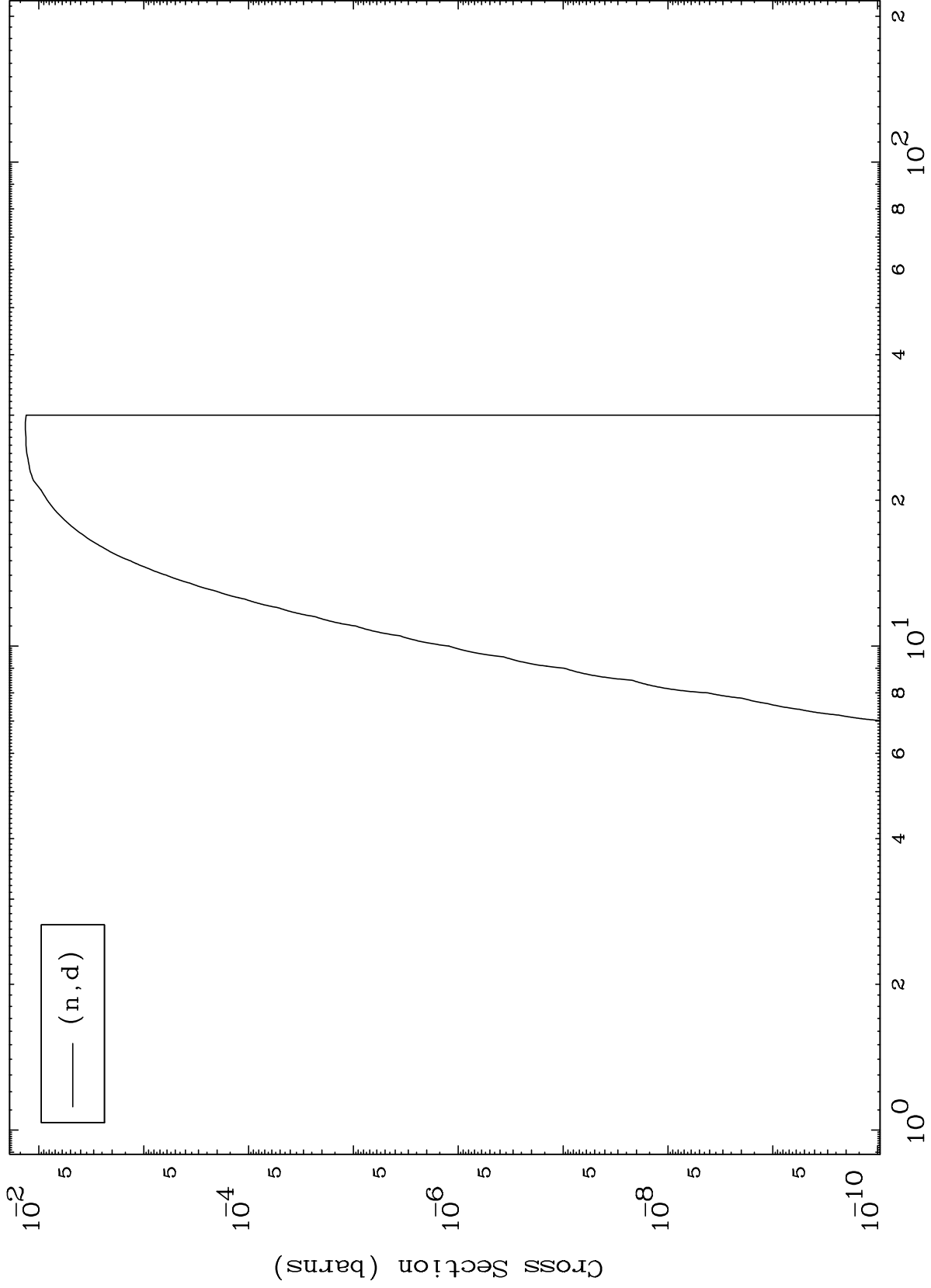
101-Md-254



MAT 9955

(n,d) Levels
293 Kelvin Cross Sections

101-Md-254



12

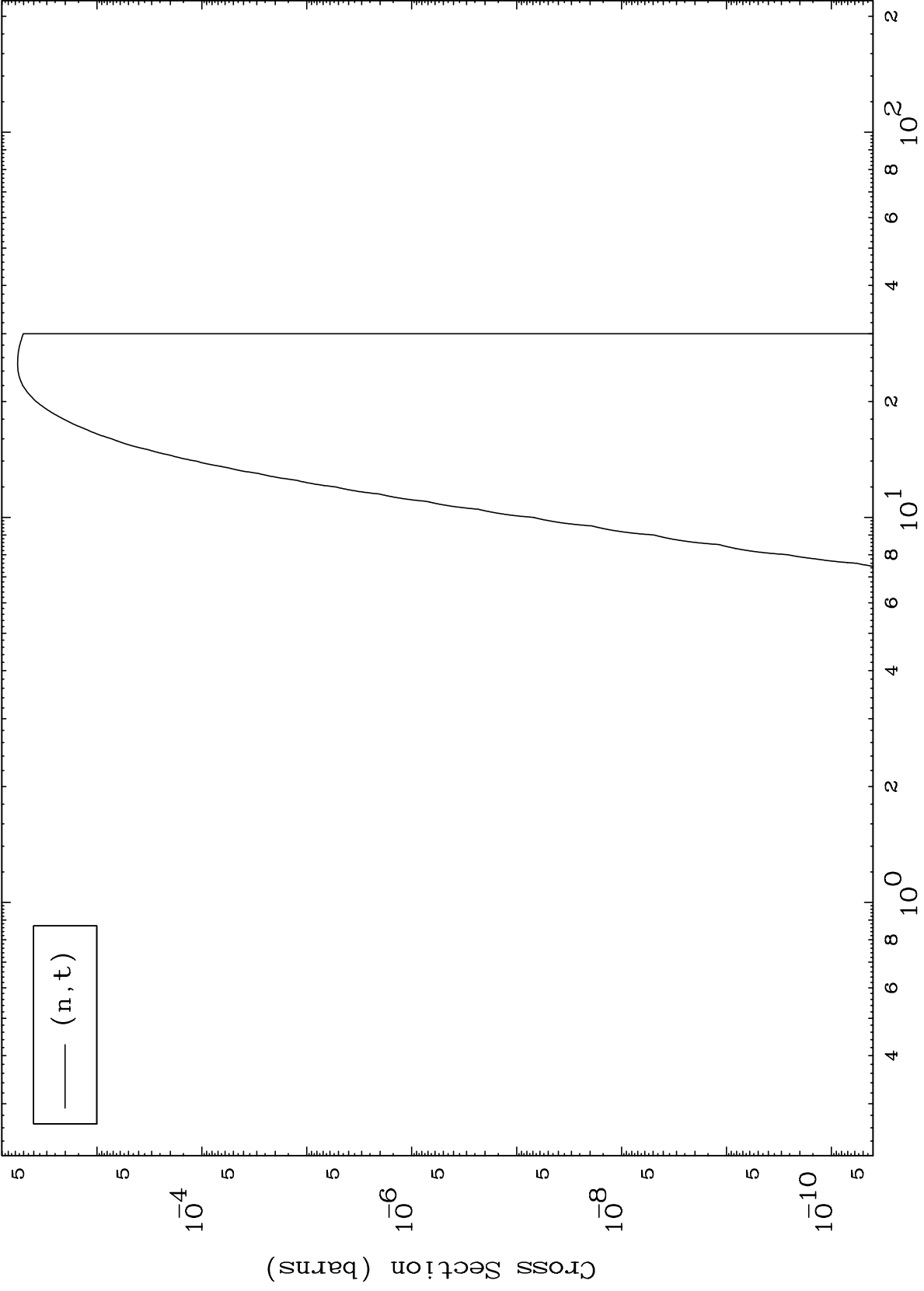
Incident Energy (MeV)

101-Md-254

MAT 9955

(n,t) Levels
293 Kelvin Cross Sections

101-Md-254



13

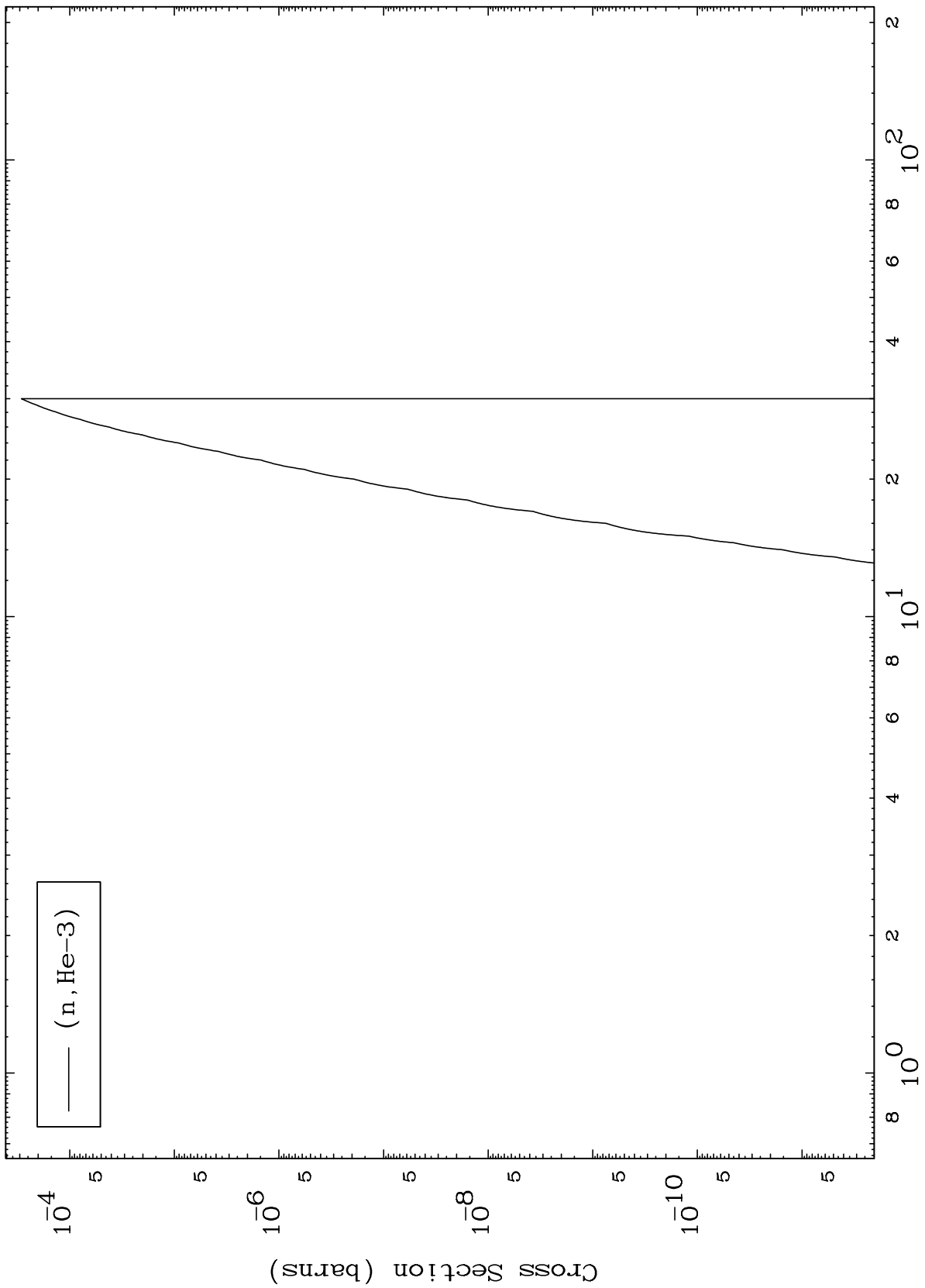
Incident Energy (MeV)

101-Md-254

MAT 9955

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-254



14

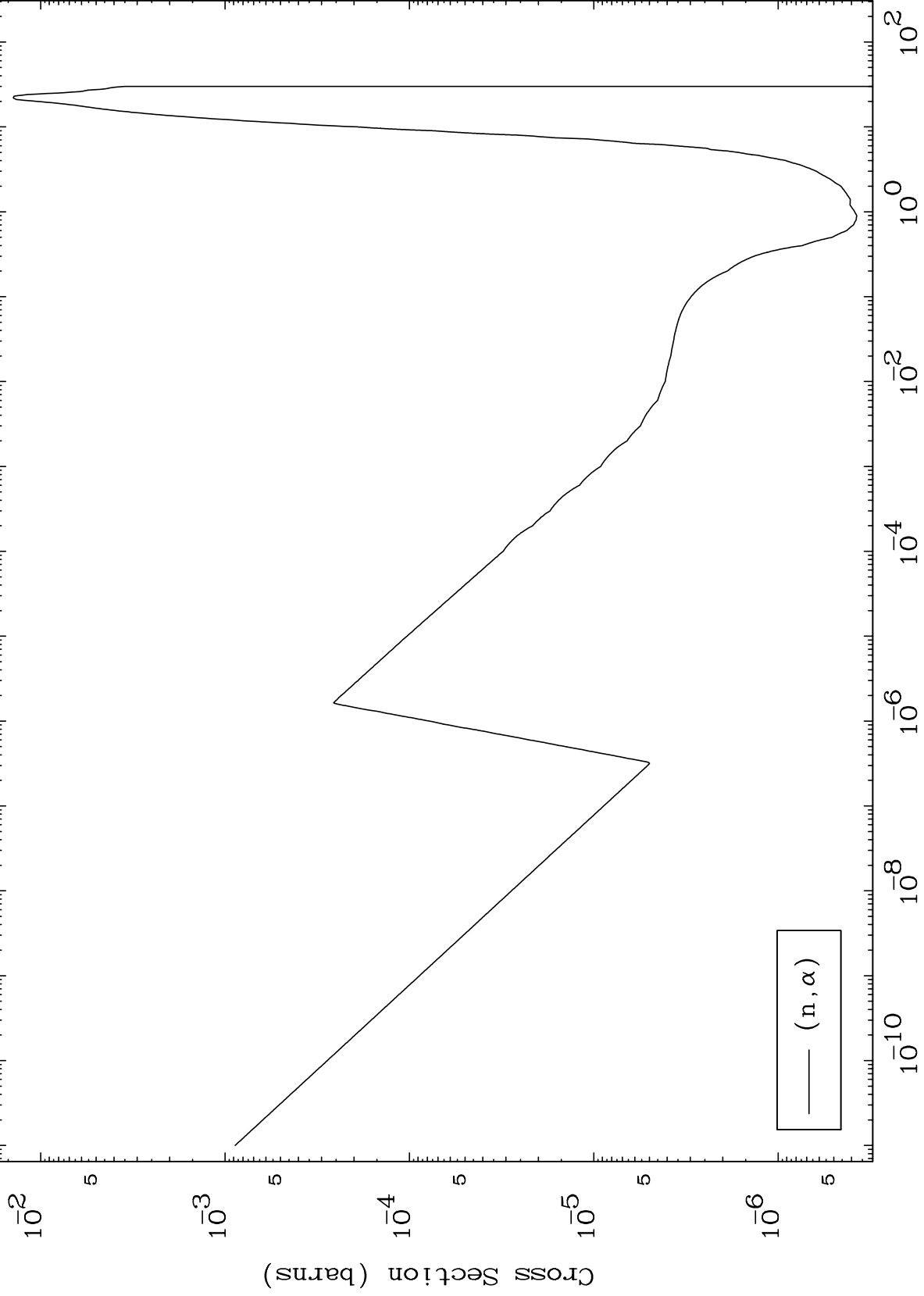
Incident Energy (MeV)

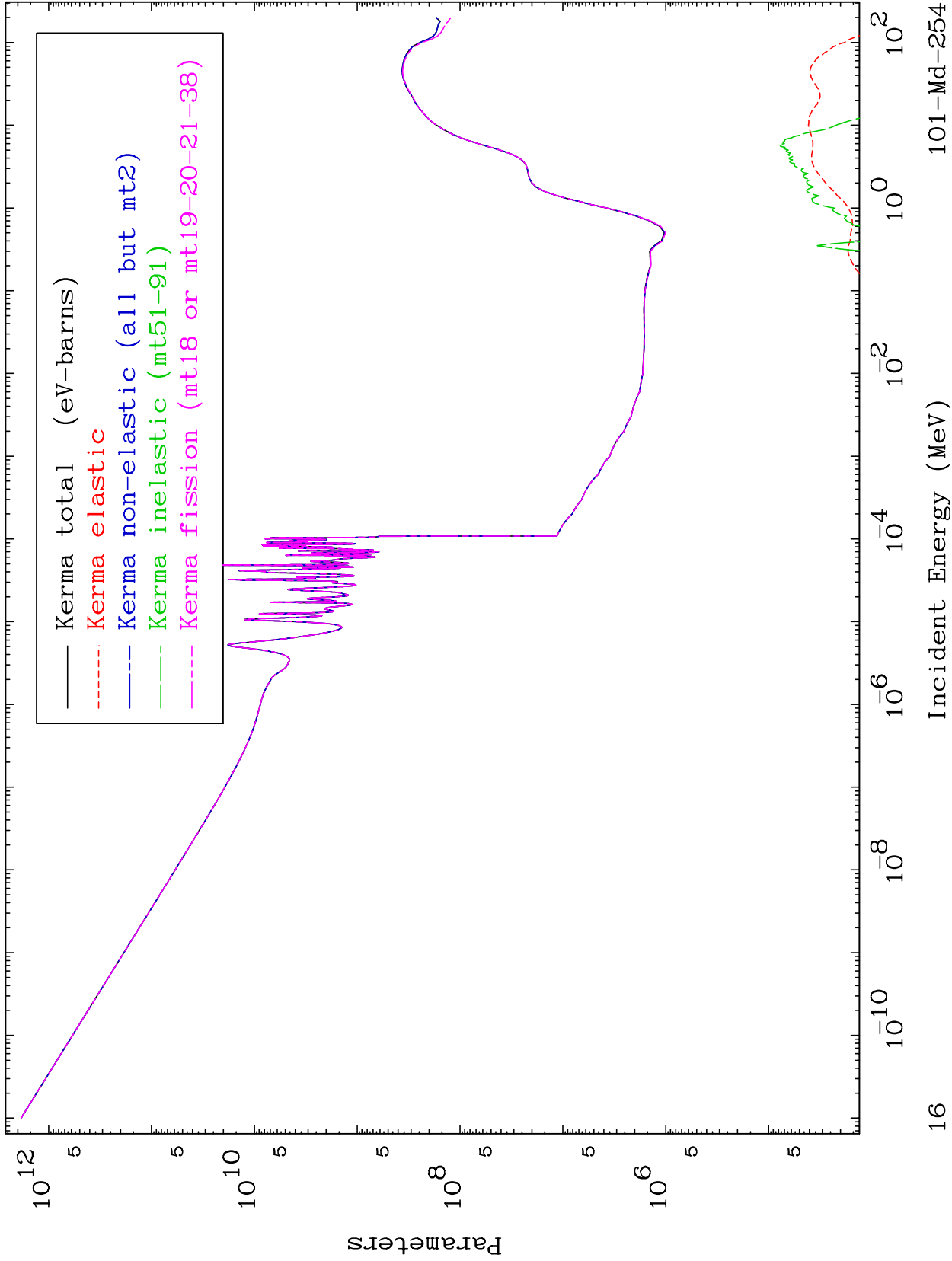
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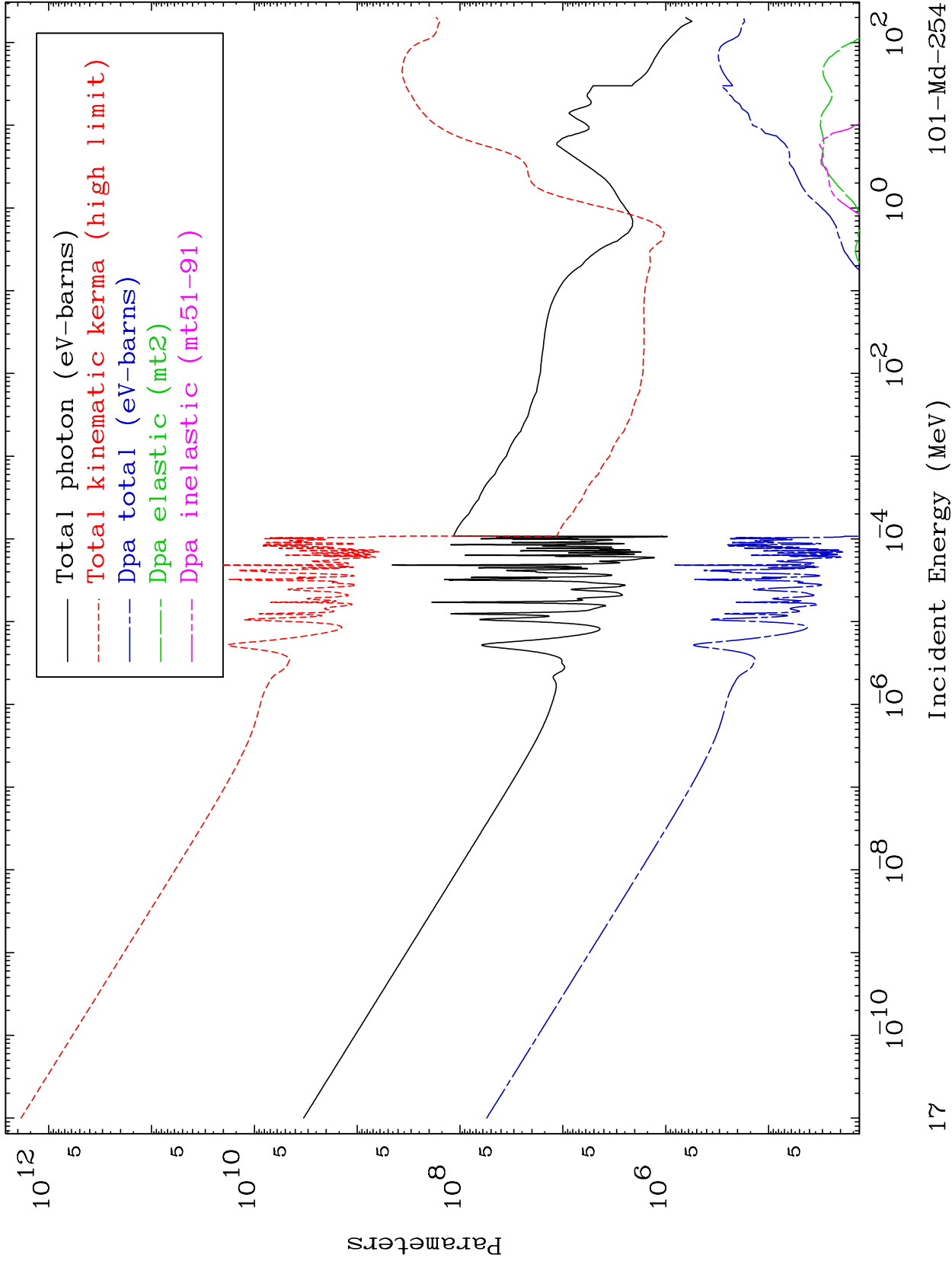
MAT 9955

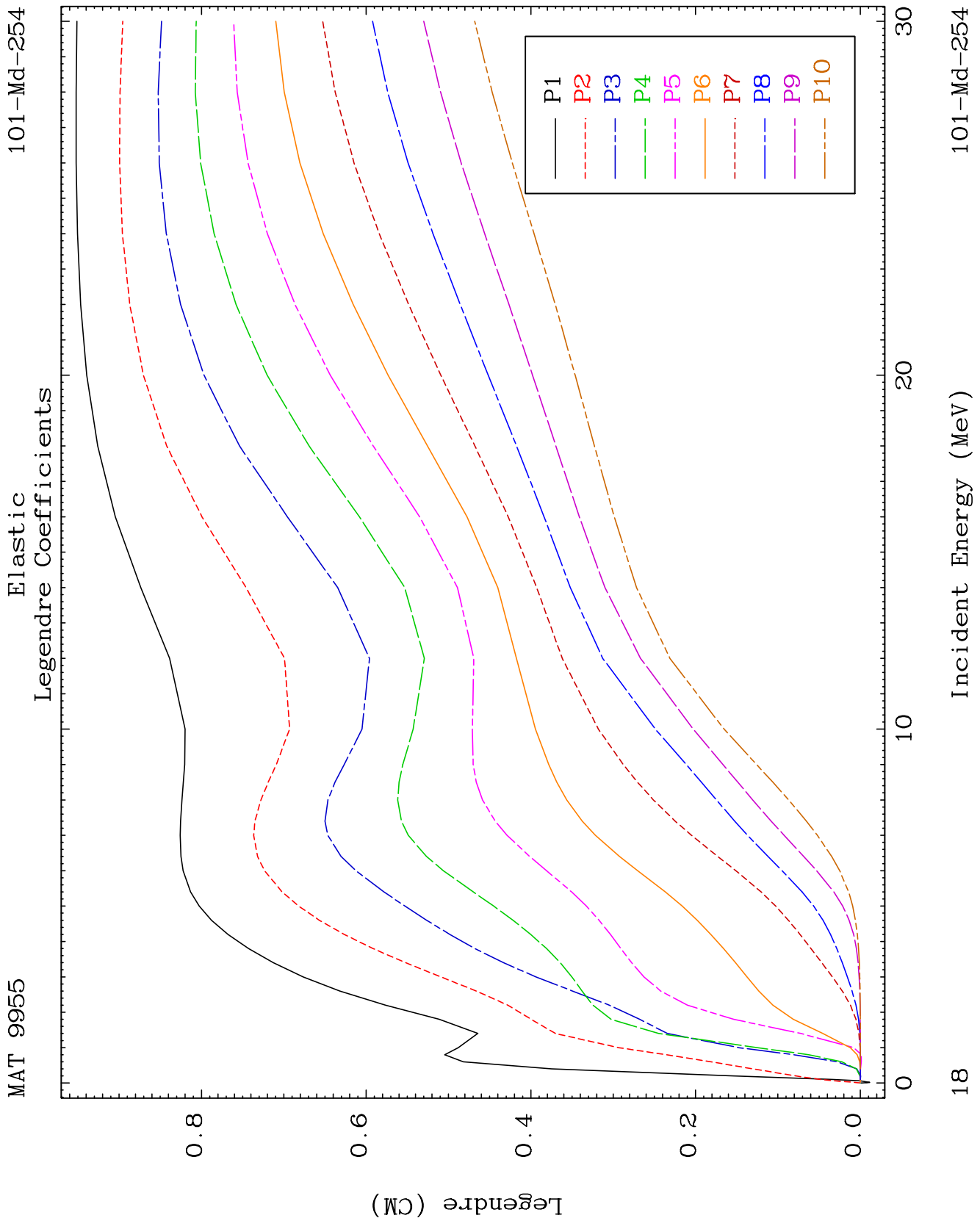
(n, α) Levels
293 Kelvin Cross Sections

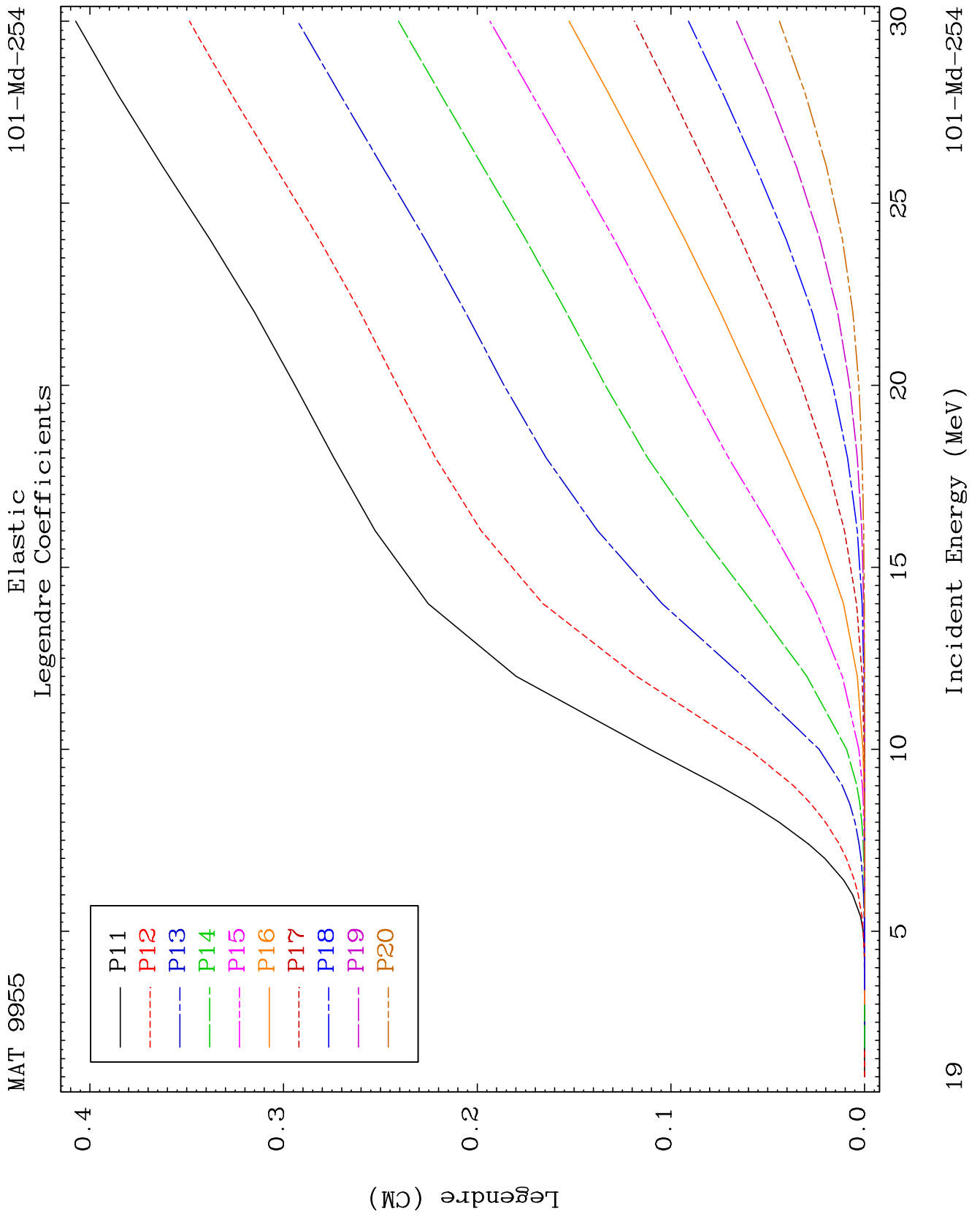
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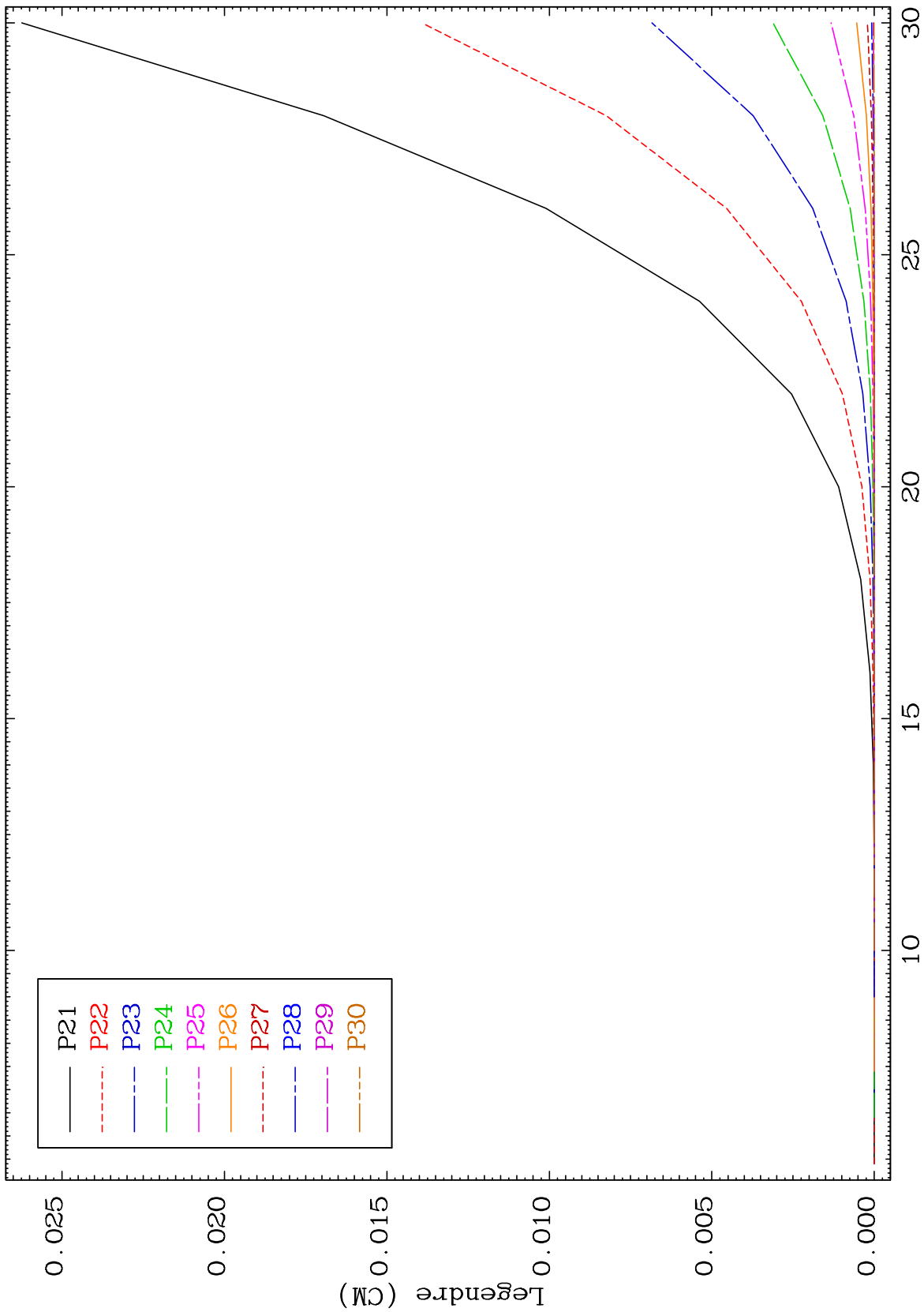




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Elastic
Legendre Coefficients

101-Md-254



20

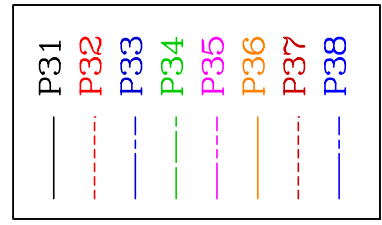
Incident Energy (MeV)

101-Md-254

MAT 9955

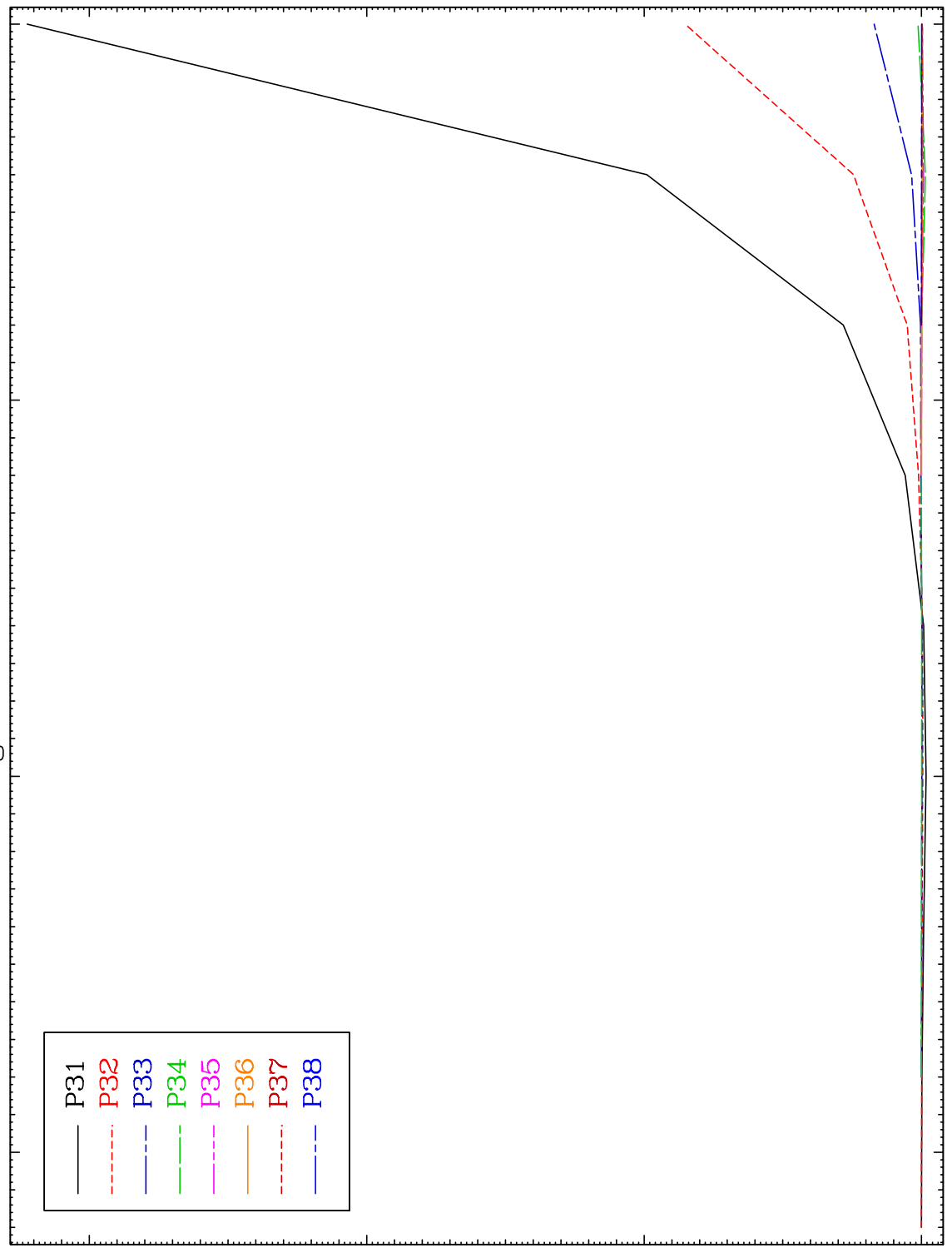
Elastic Legendre Coefficients

101-Md-254



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

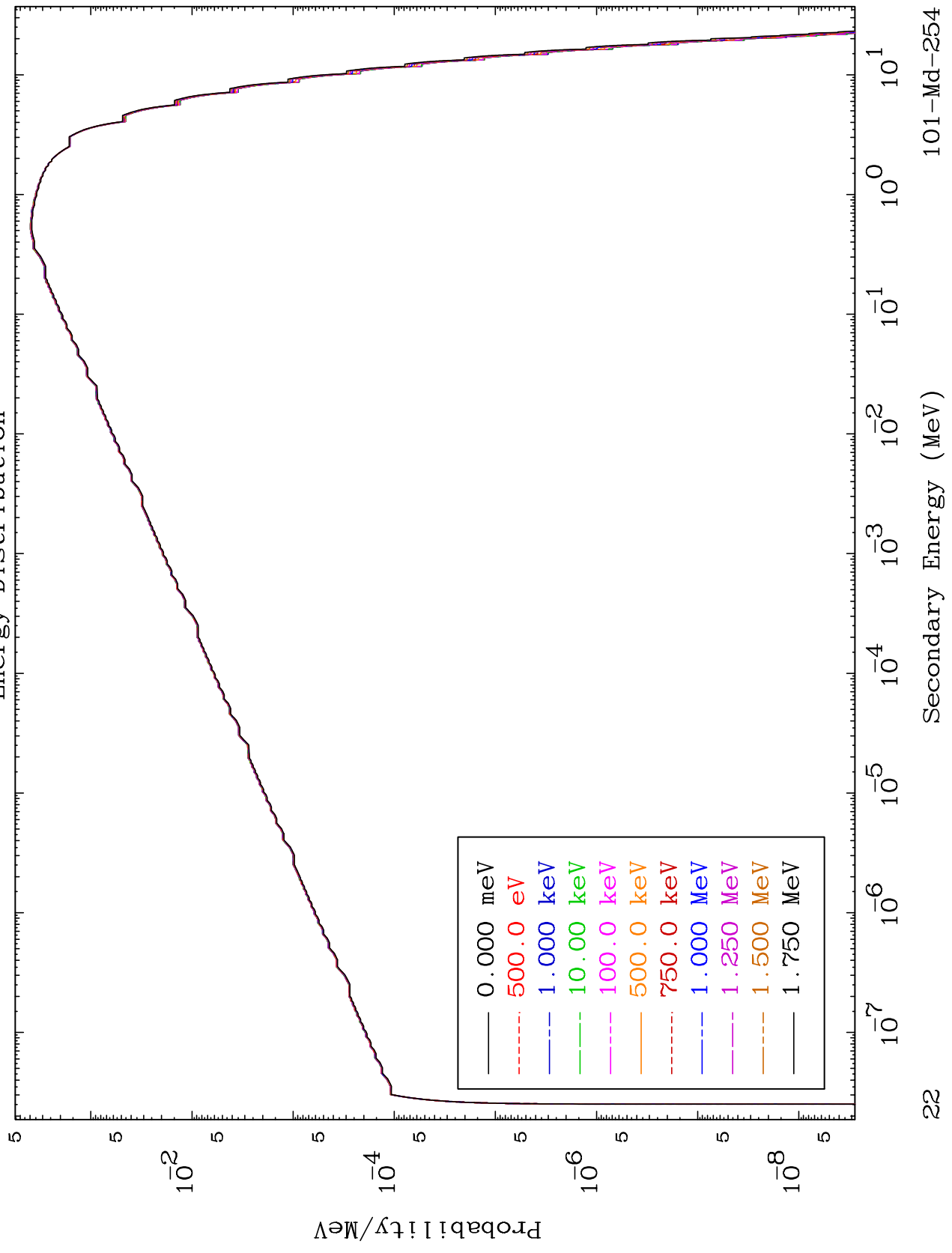
Incident Energy (MeV)

101-Md-254

MAT 9955

Fission Energy Distribution

101-Md-254

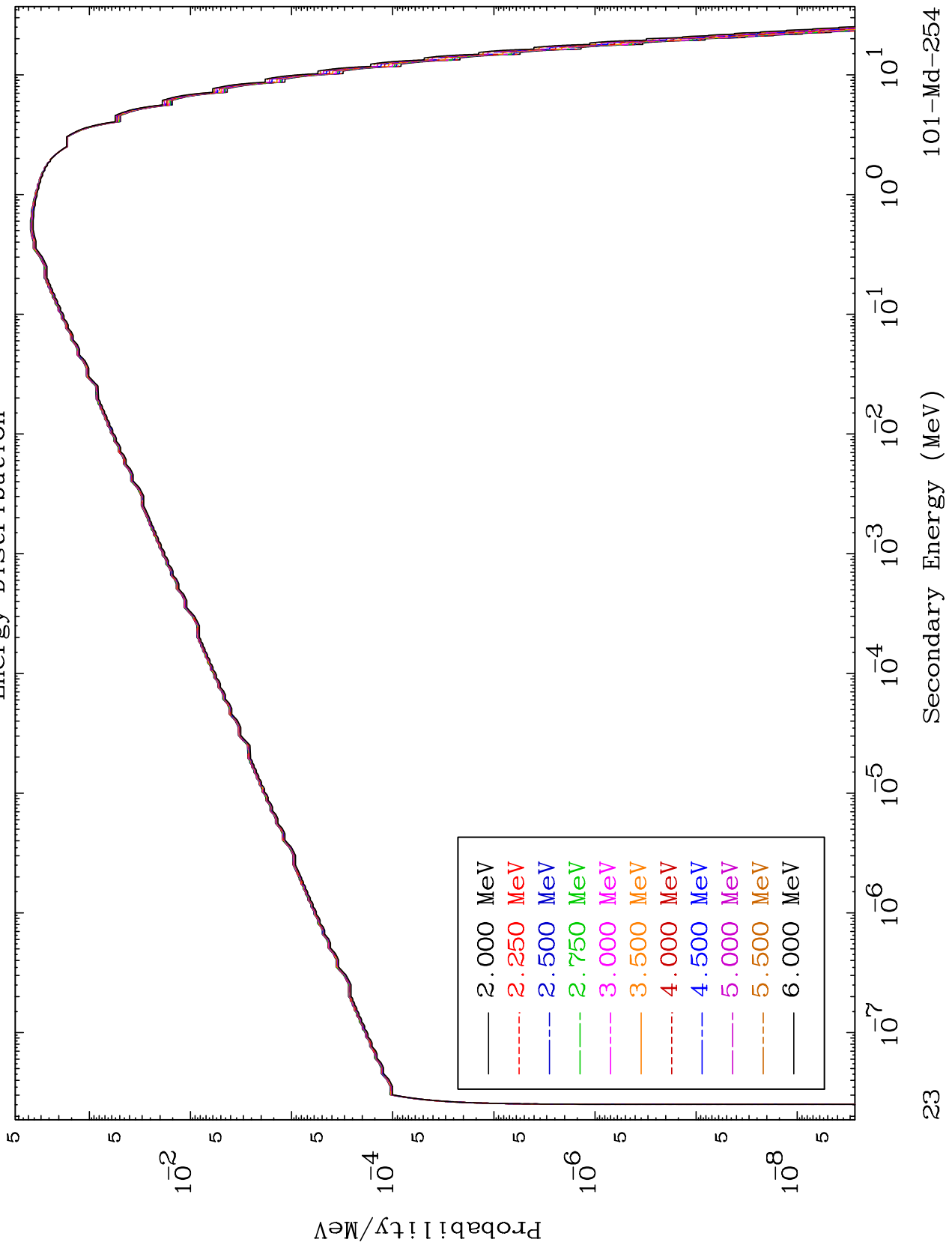


101-Md-254

MAT 9955

Fission Energy Distribution

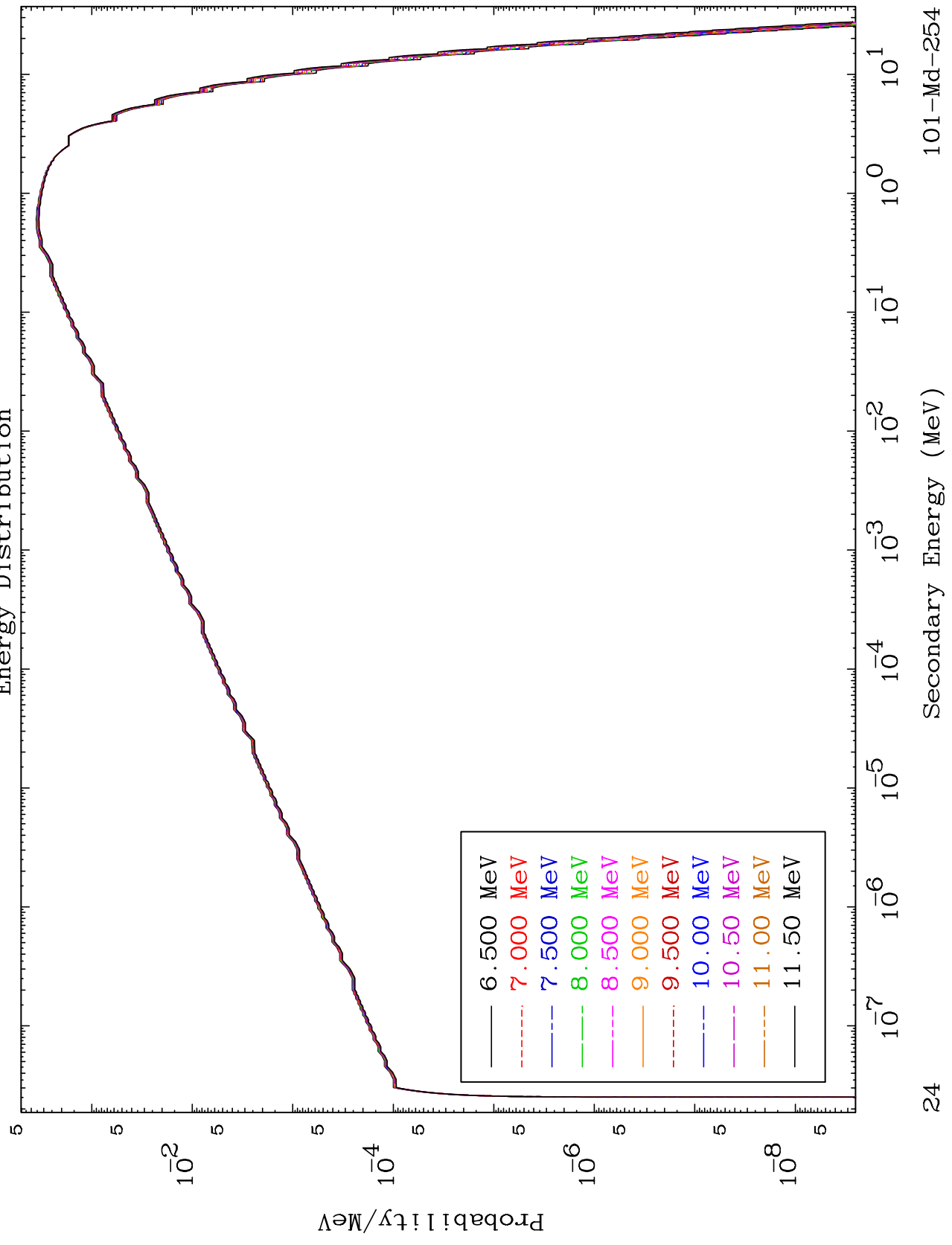
101-Md-254



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Fission Energy Distribution

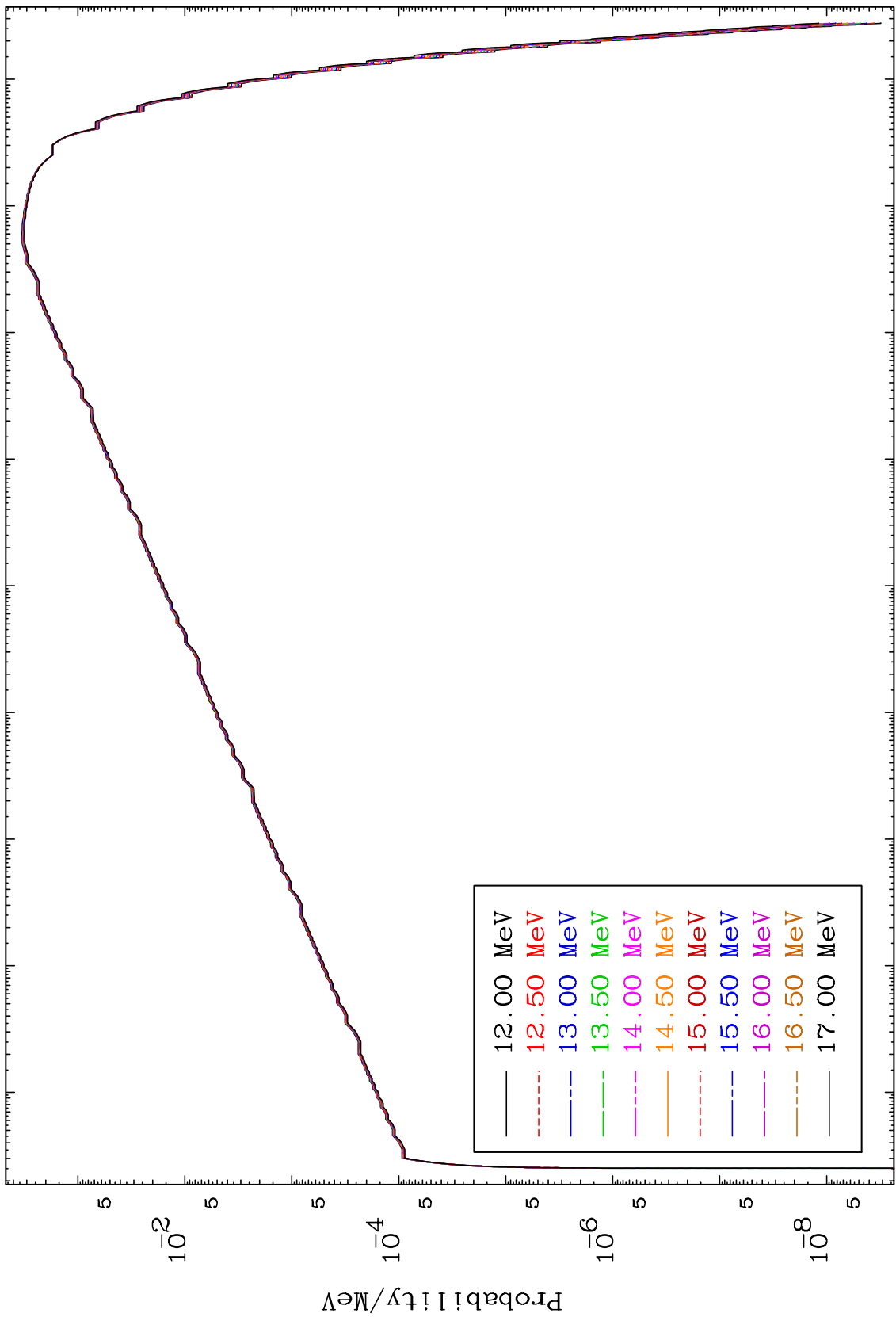
101-Md-254



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Fission Energy Distribution

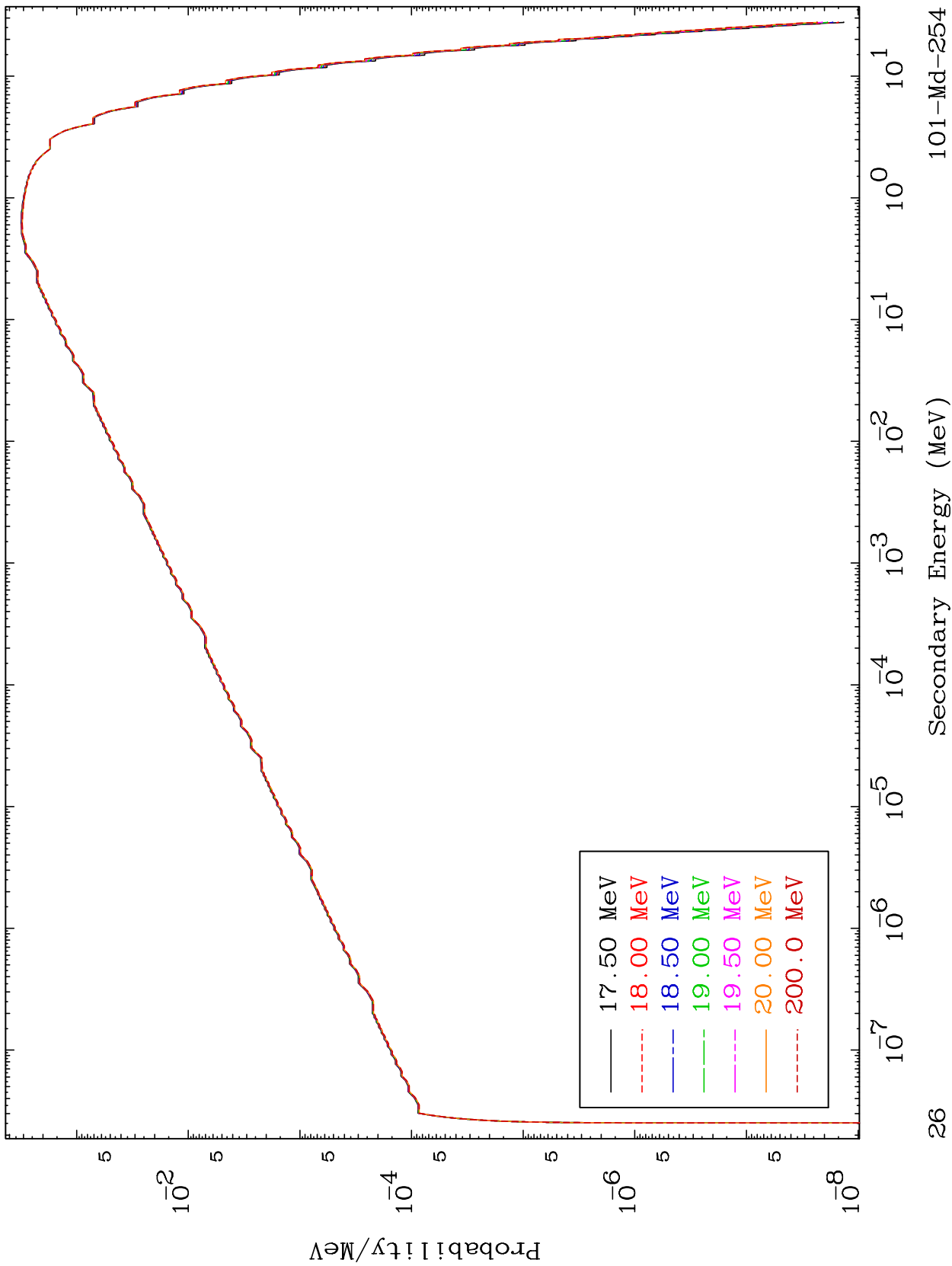
101-Md-254



101-Md-254

Secondary Energy (MeV)

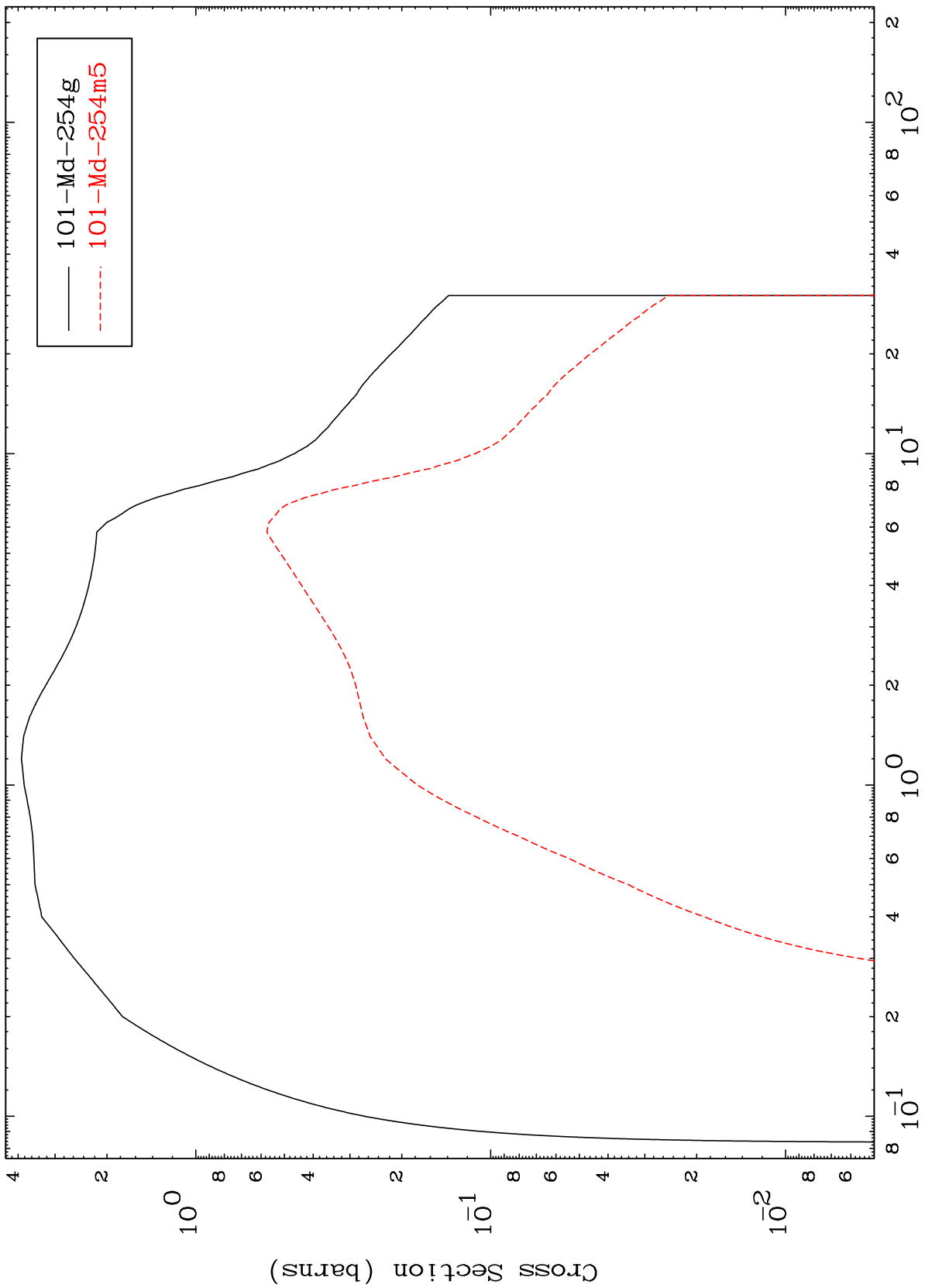
Fission
Energy Distribution



MAT 9955

101-Md-254

Inelastic
Radionuclide Production Cross Section



101-Md-254g
101-Md-254m5

27

Incident Energy (MeV)

101-Md-254

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(n,n') α

101-Md-254

Radionuclide Production Cross Section

