

Program EVALPLOT
(Version 2017-1)

by

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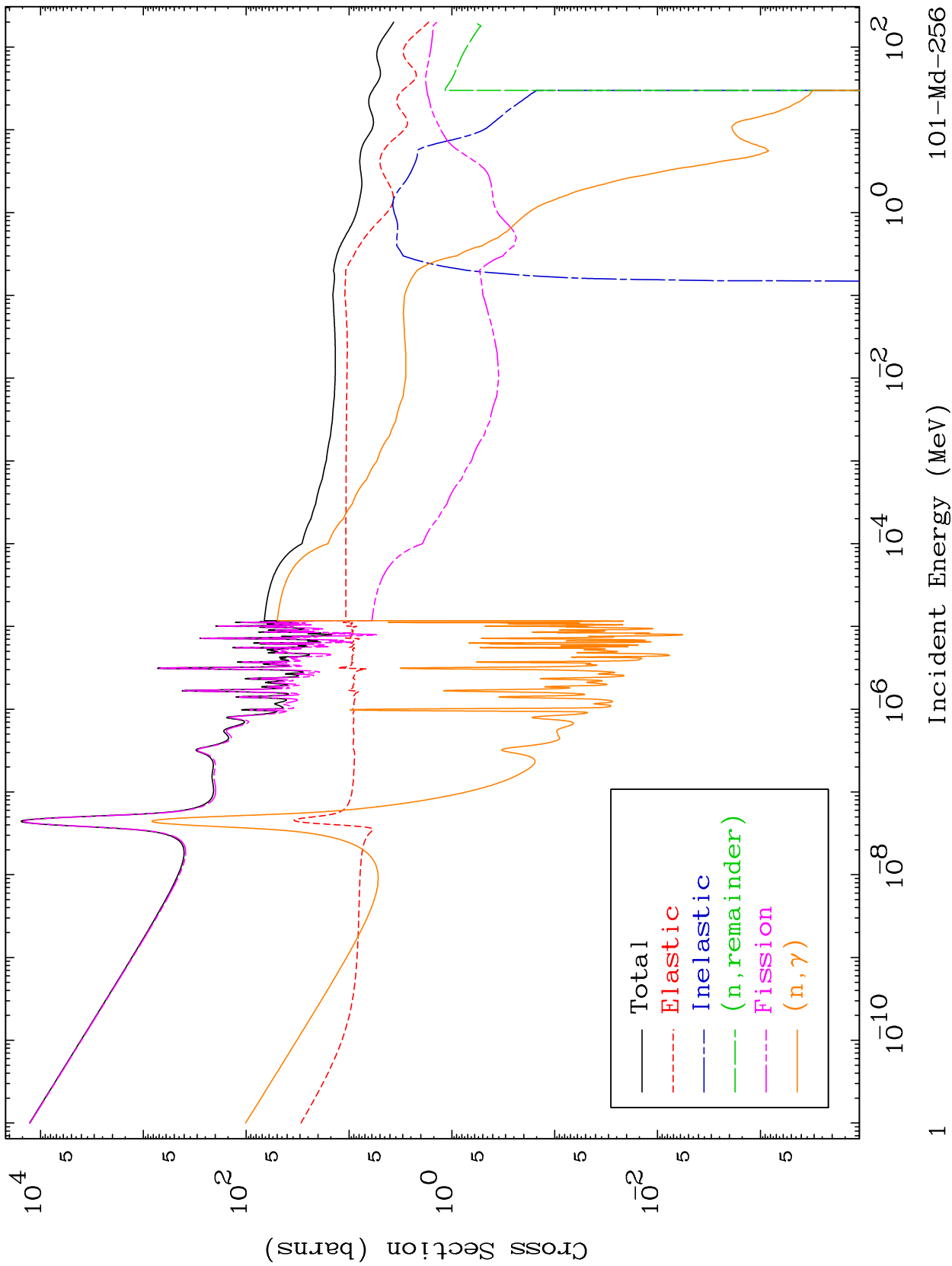
Press Mouse Button to Start

MAT 9957

Major

293 Kelvin Cross Sections

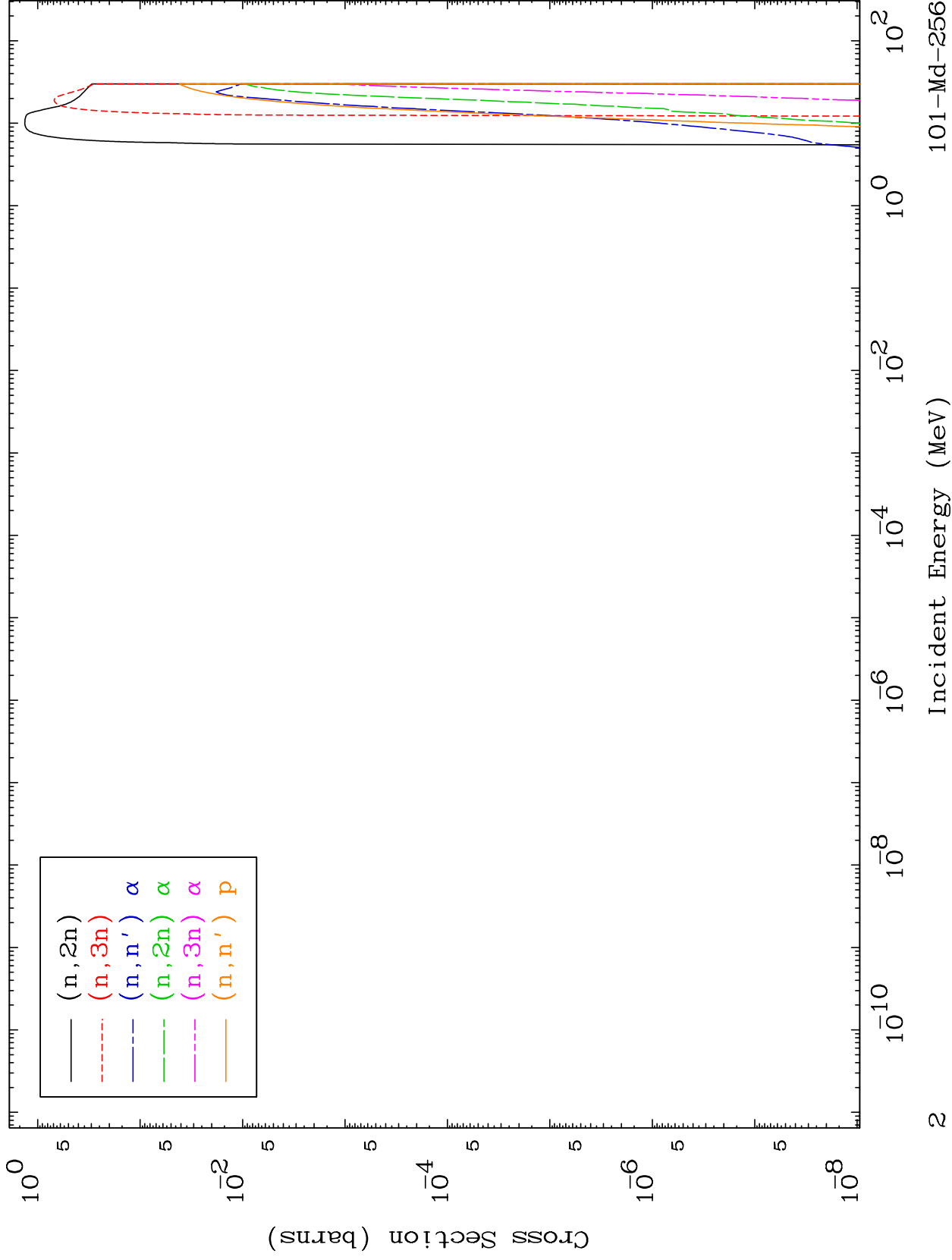
101-Md-256

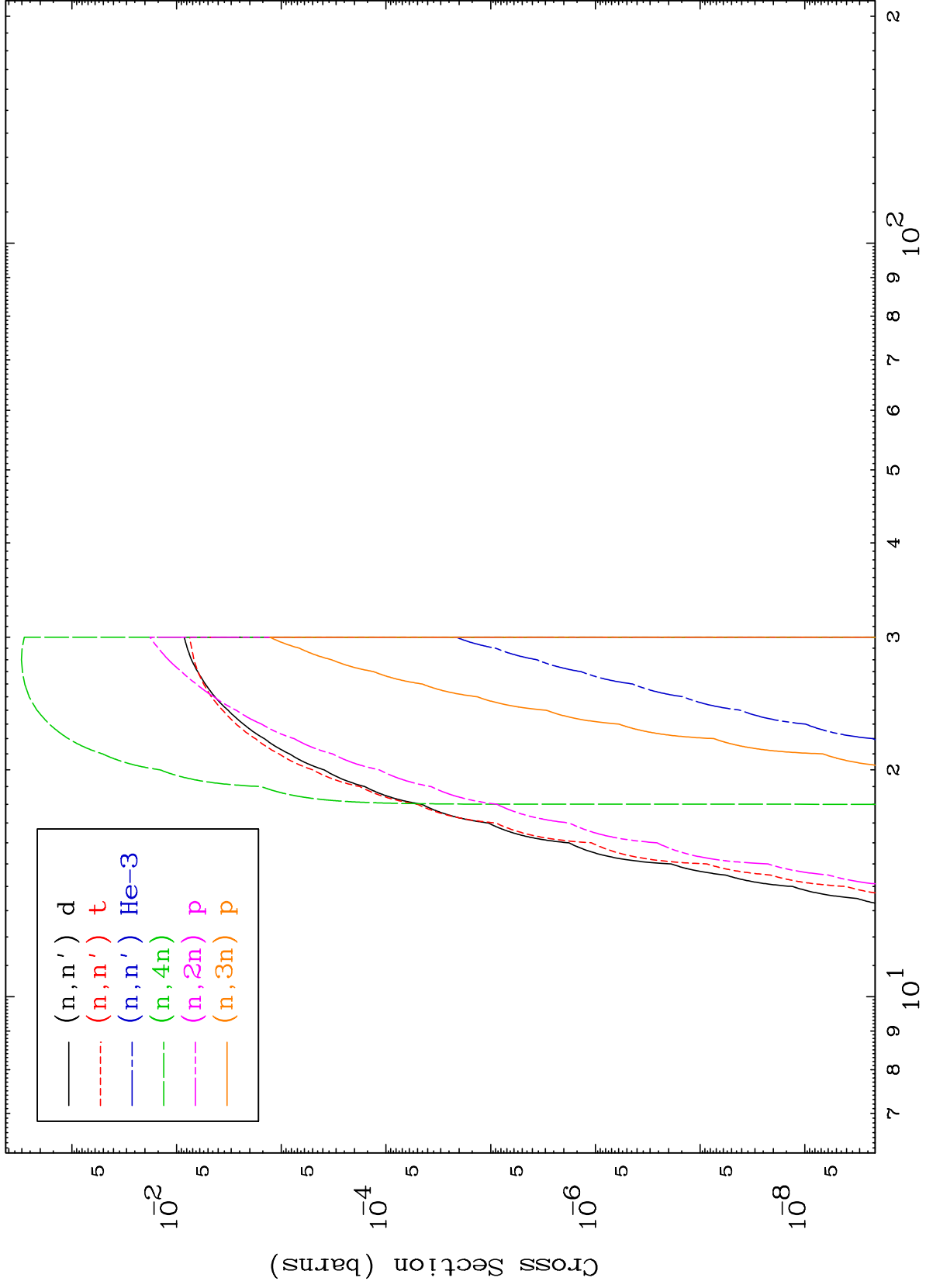


MAT 9957

Neutron Production
293 Kelvin Cross Sections

101-Md-256

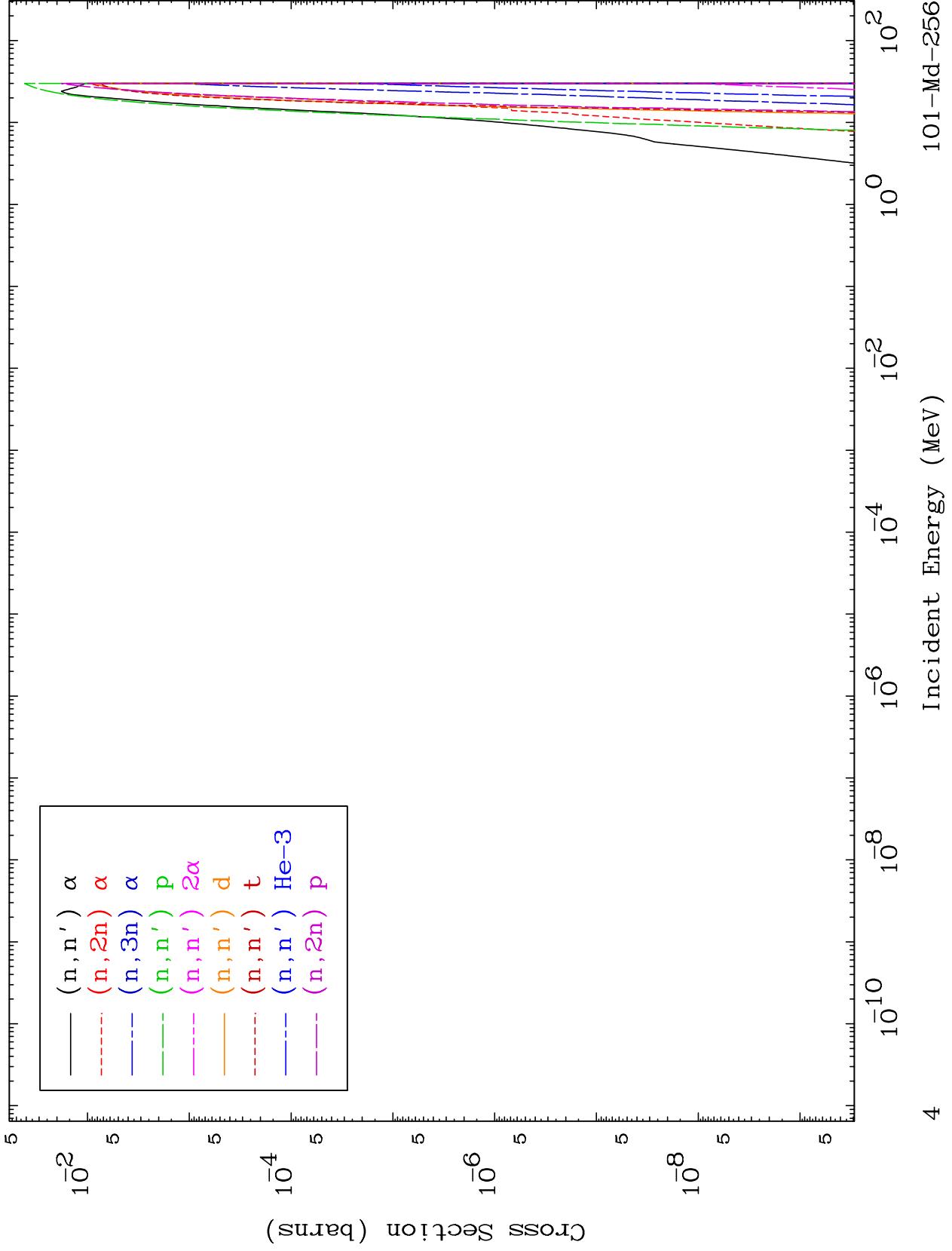




MAT 9957

Charged Particle
293 Kelvin Cross Sections

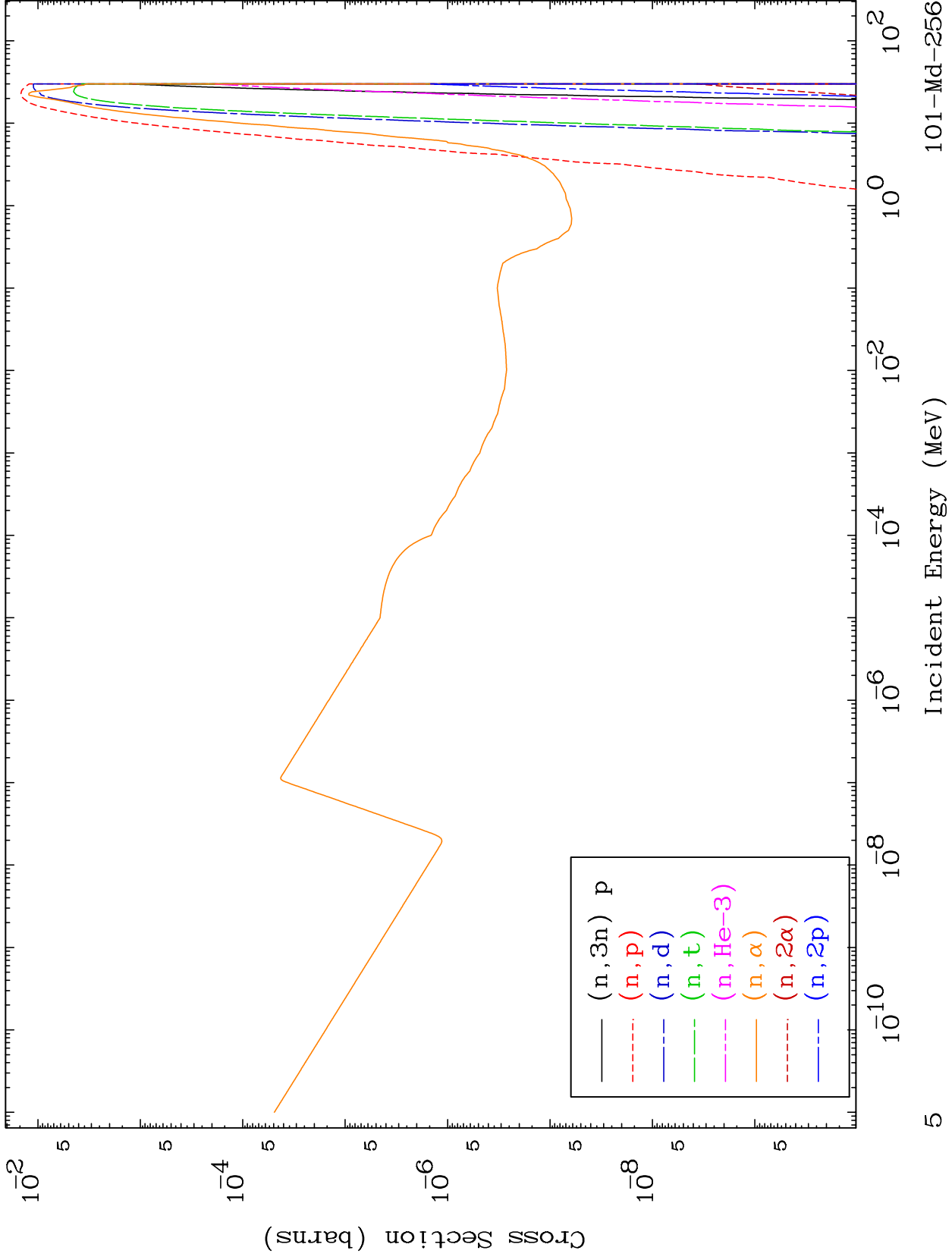
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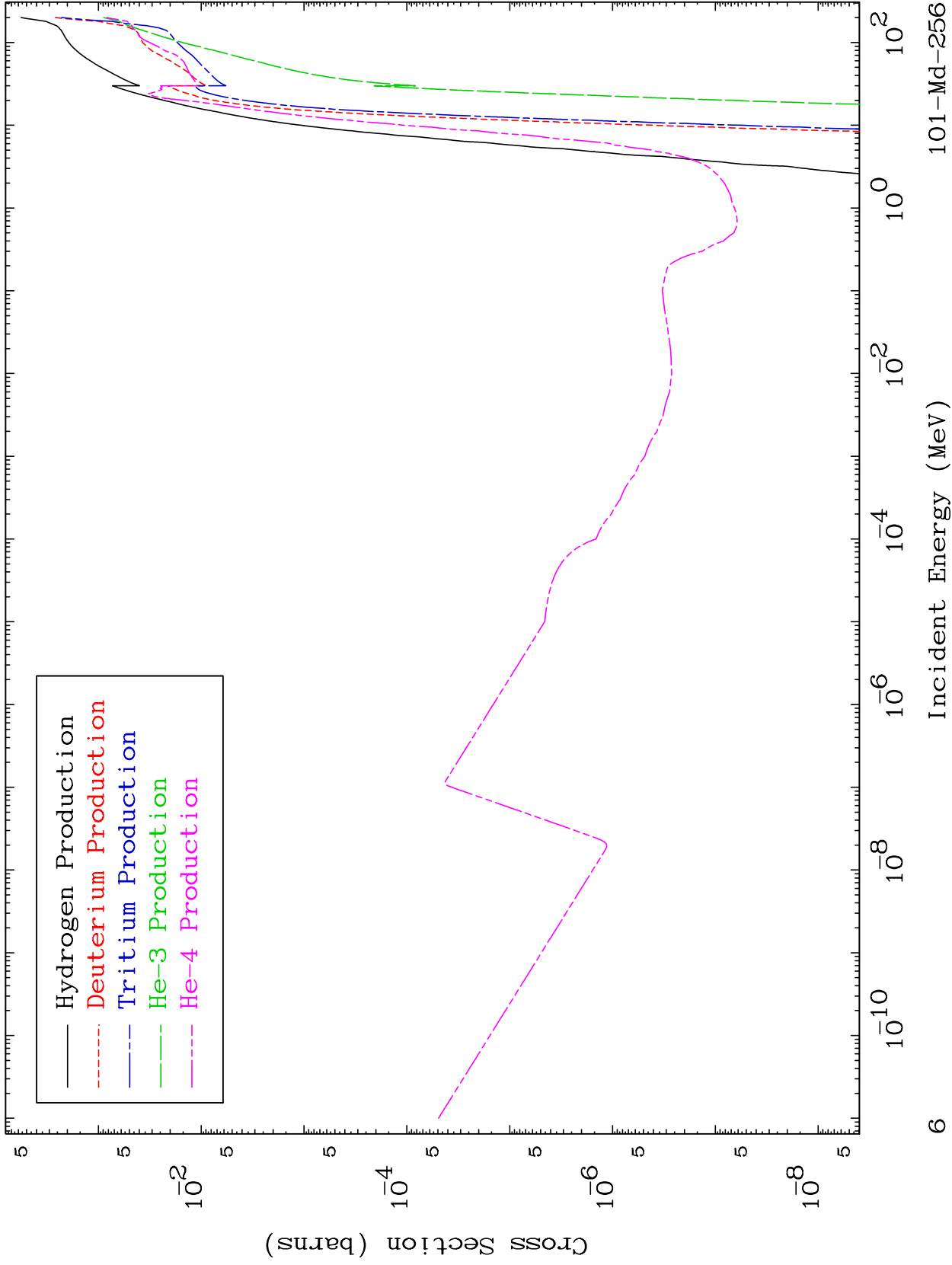


MAT 9957

Charged Particle
293 Kelvin Cross Sections

101-Md-256

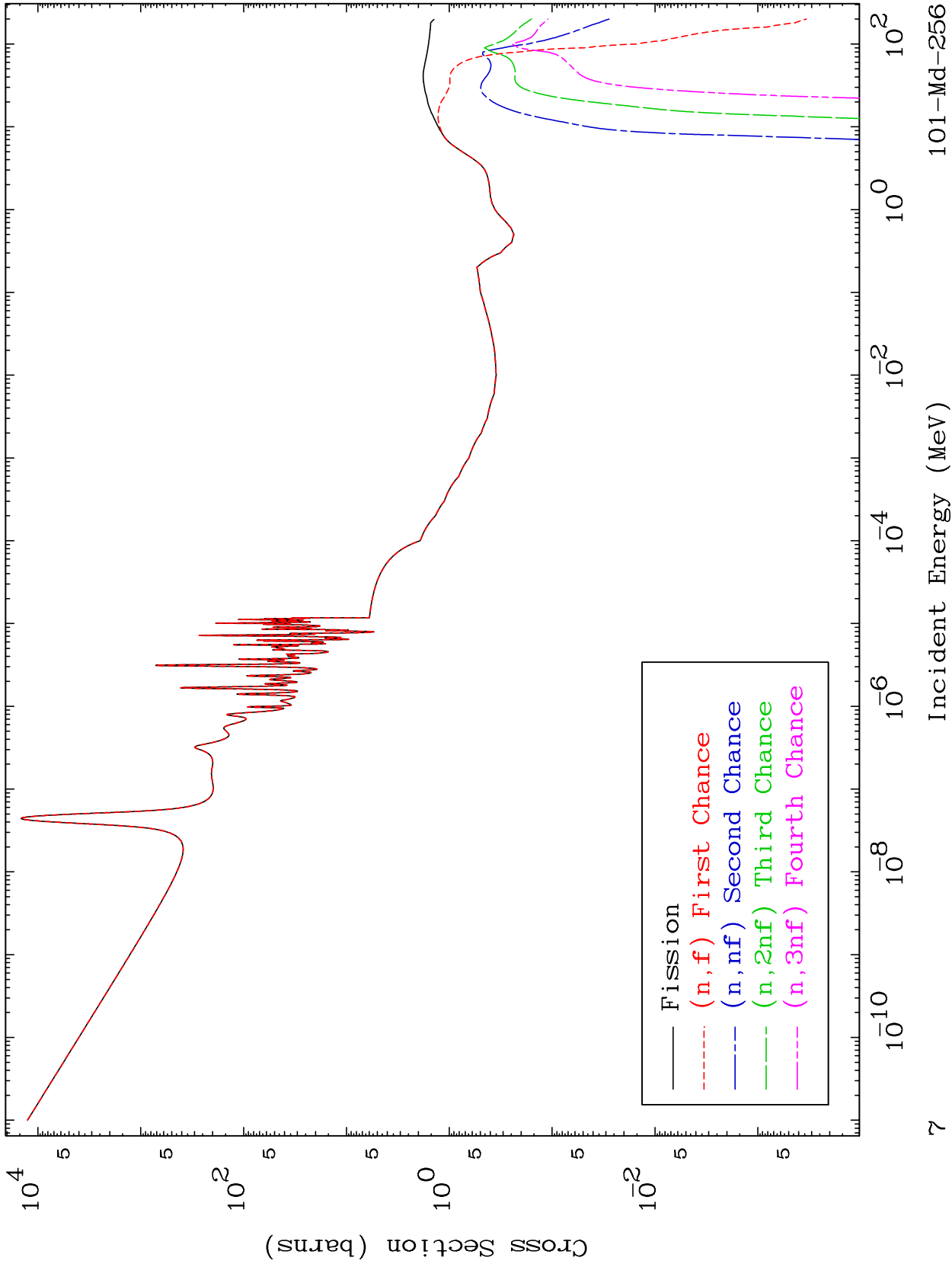




MAT 9957

Fission
293 Kelvin Cross Sections

101-Md-256

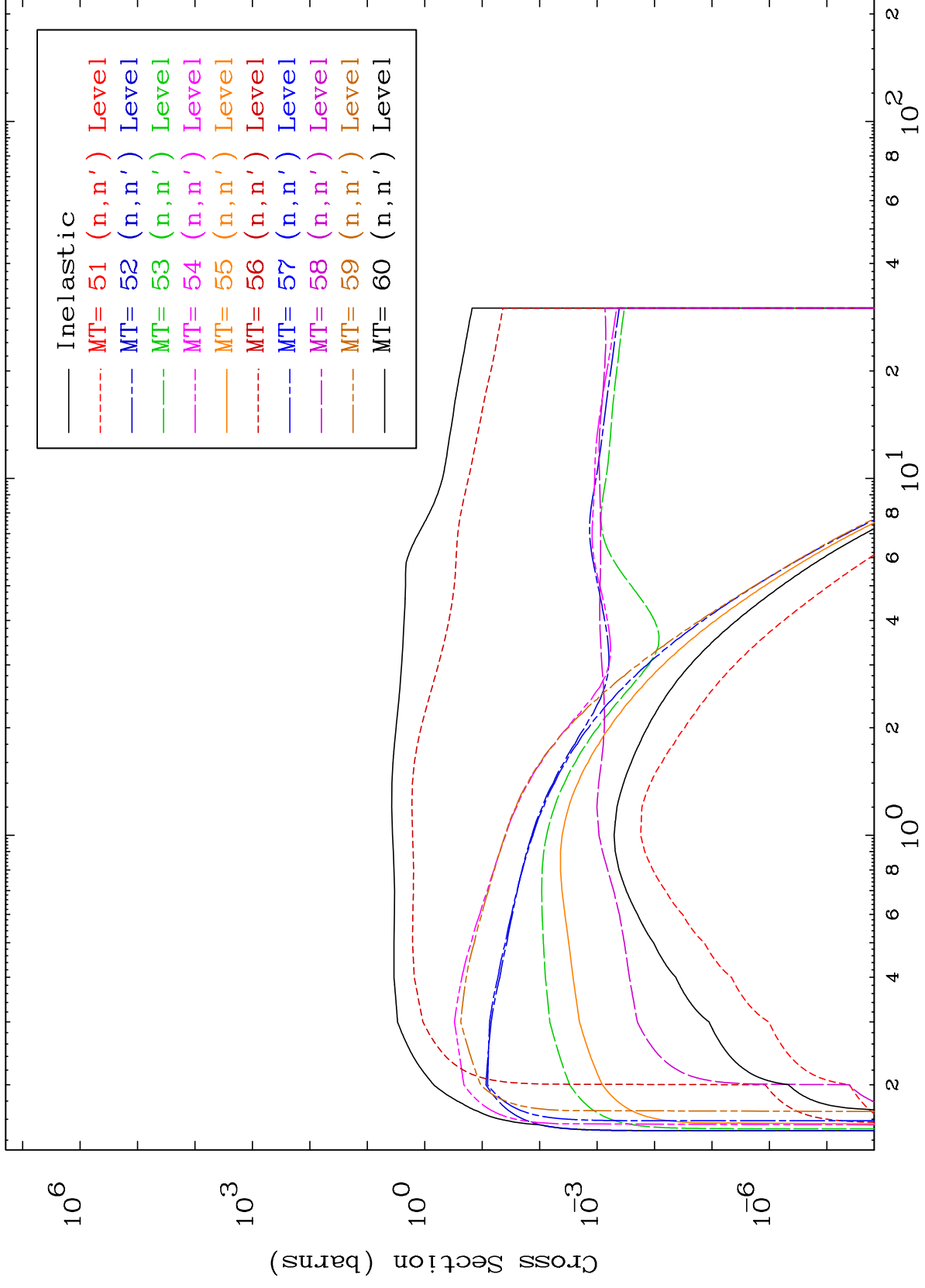


MAT 9957

(n,n') Level

101-Md-256

293 Kelvin Cross Sections

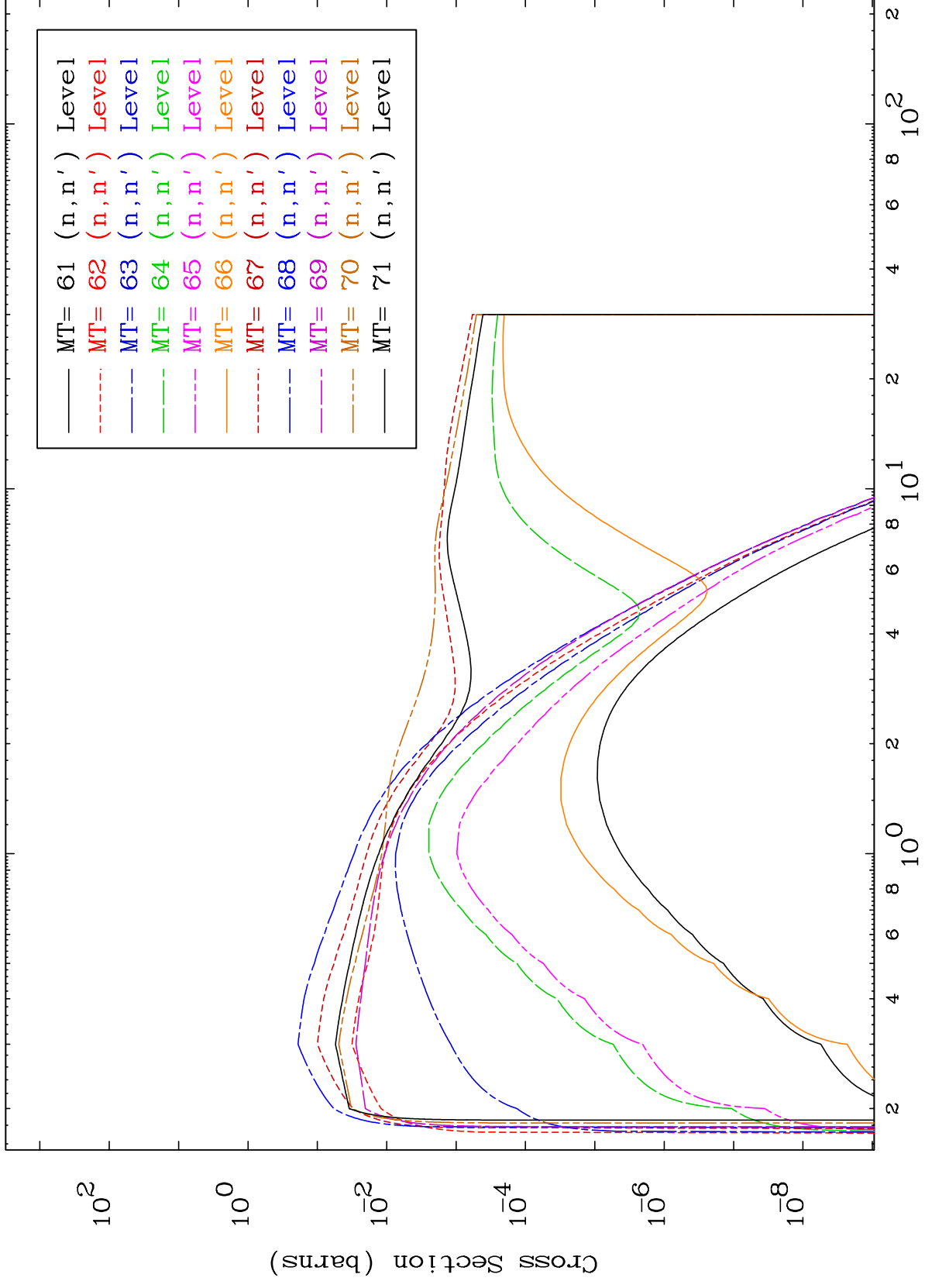


MAT 9957

(n,n') Level

101-Md-256

293 Kelvin Cross Sections

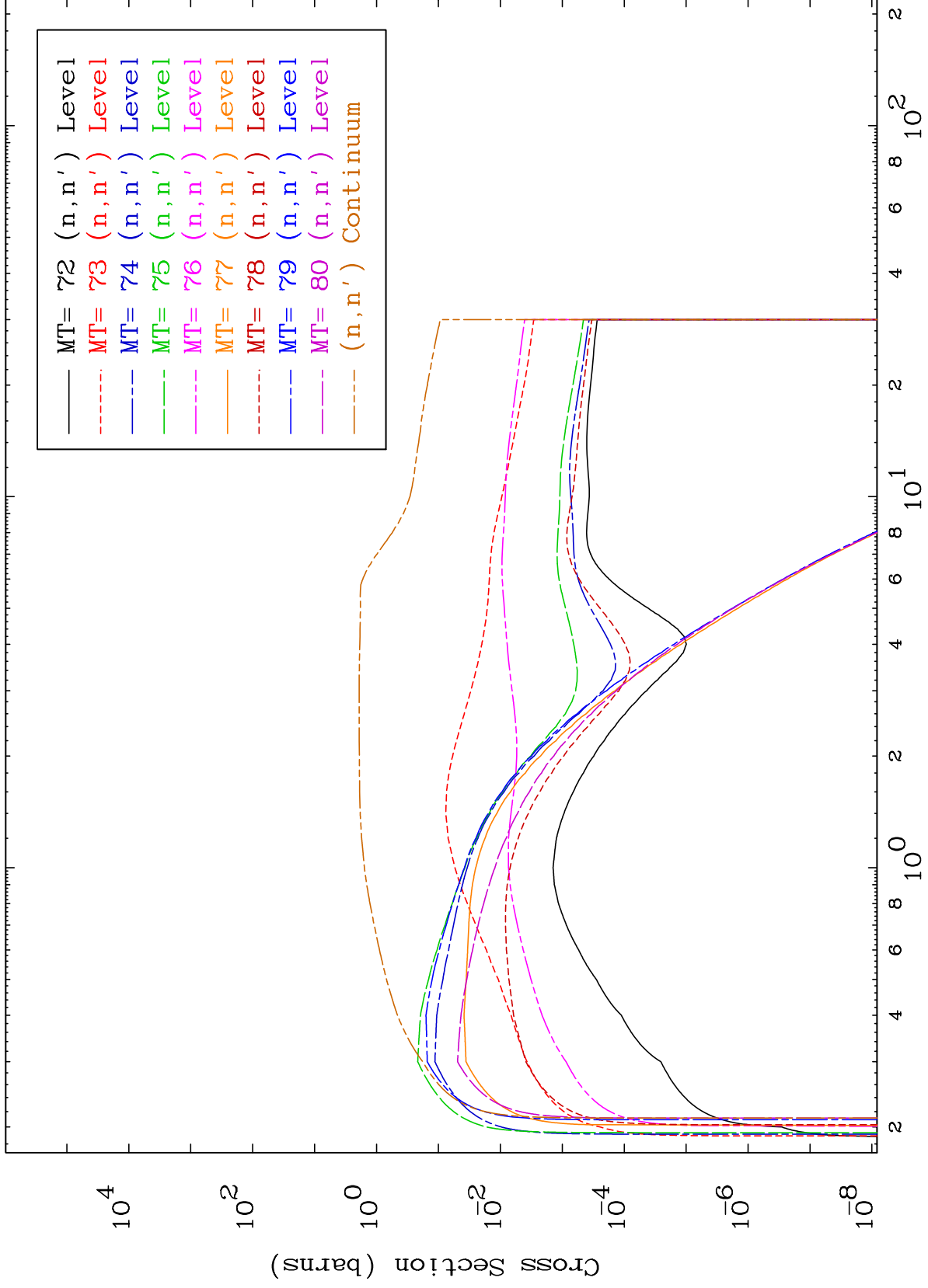


MAT 9957

(n,n') Level

293 Kelvin Cross Sections

101-Md-256



10

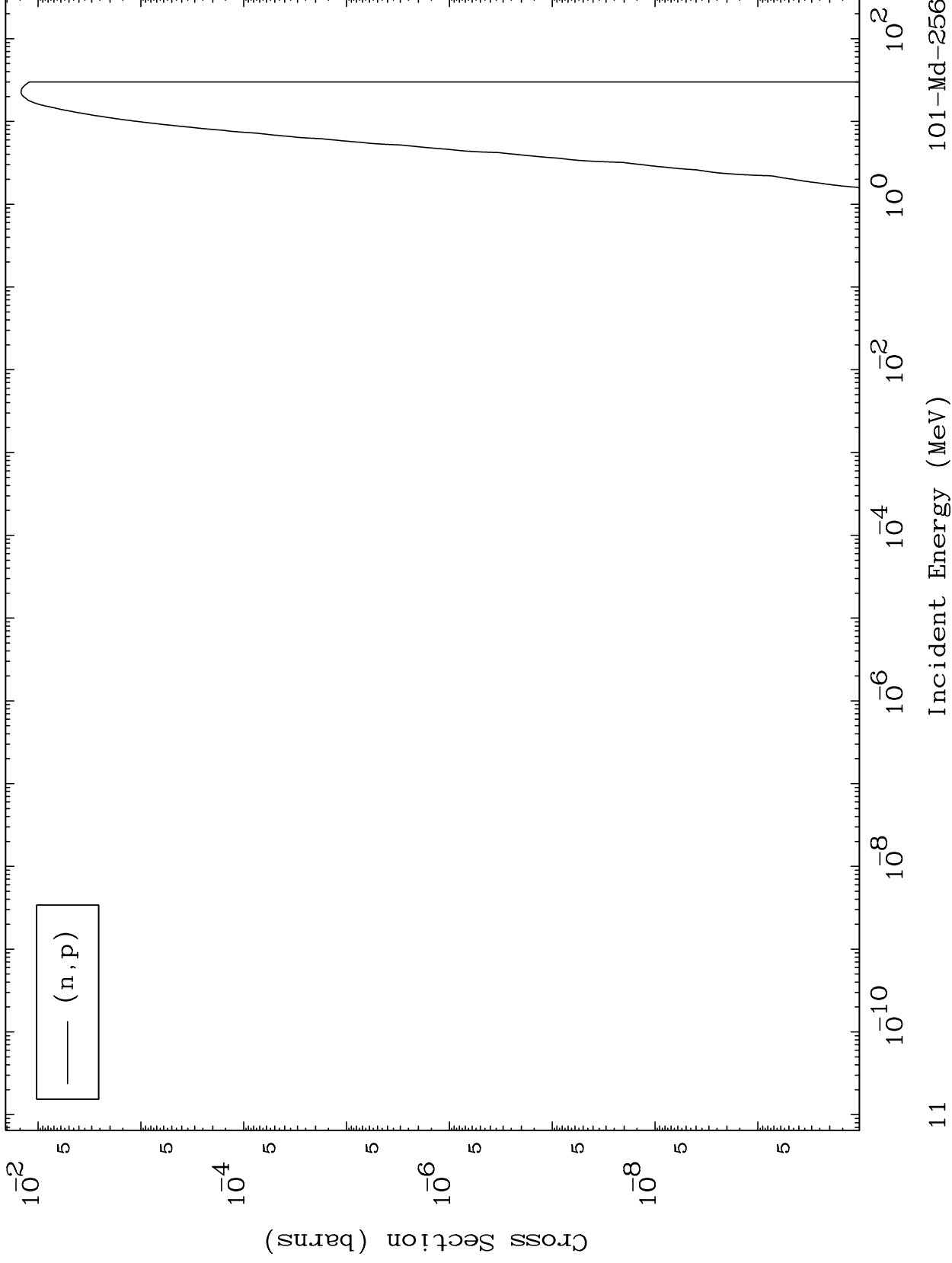
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,p) Levels
293 Kelvin Cross Sections

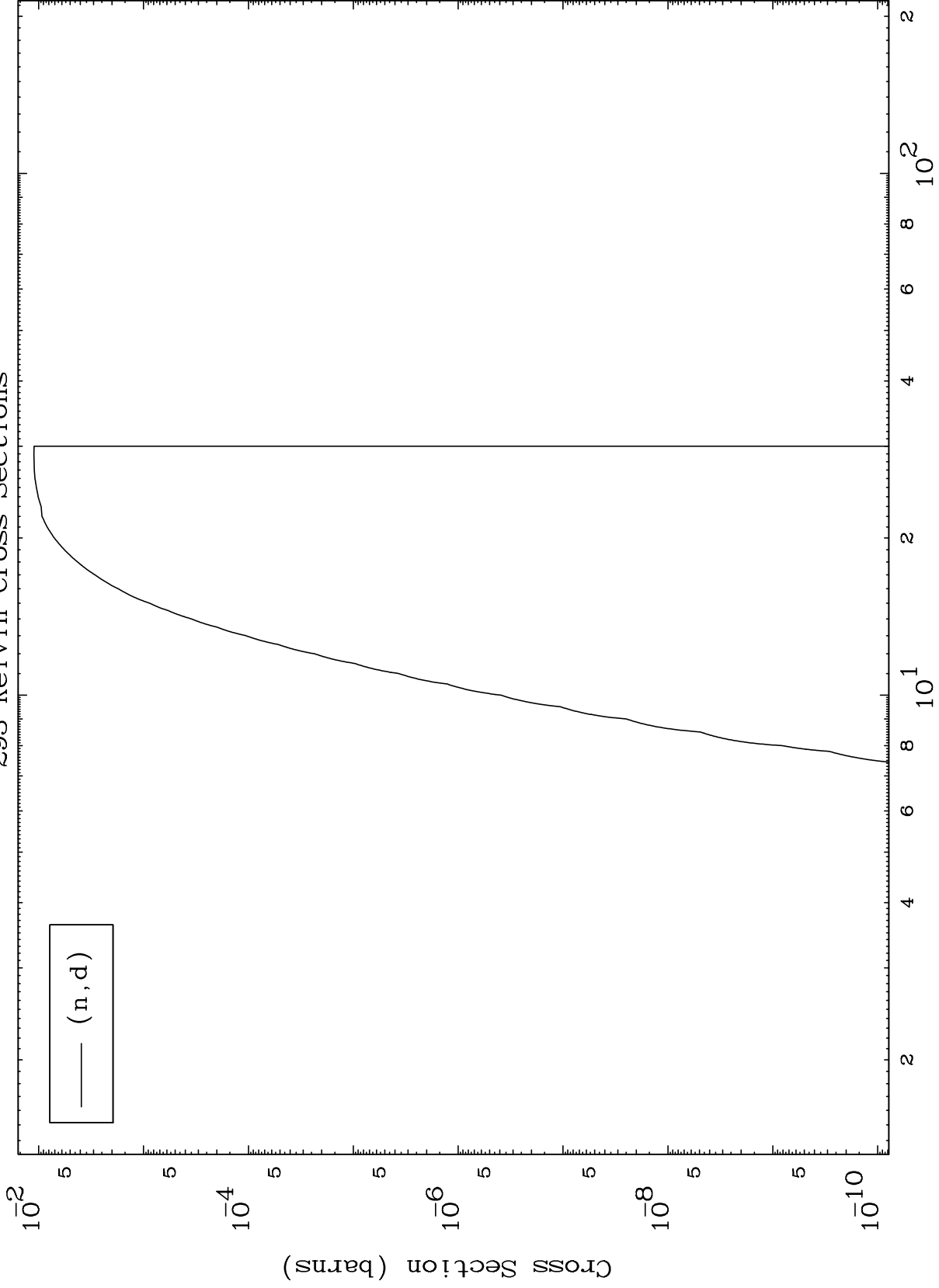
101-Md-256



MAT 9957

(n,d) Levels
293 Kelvin Cross Sections

101-Md-256



12

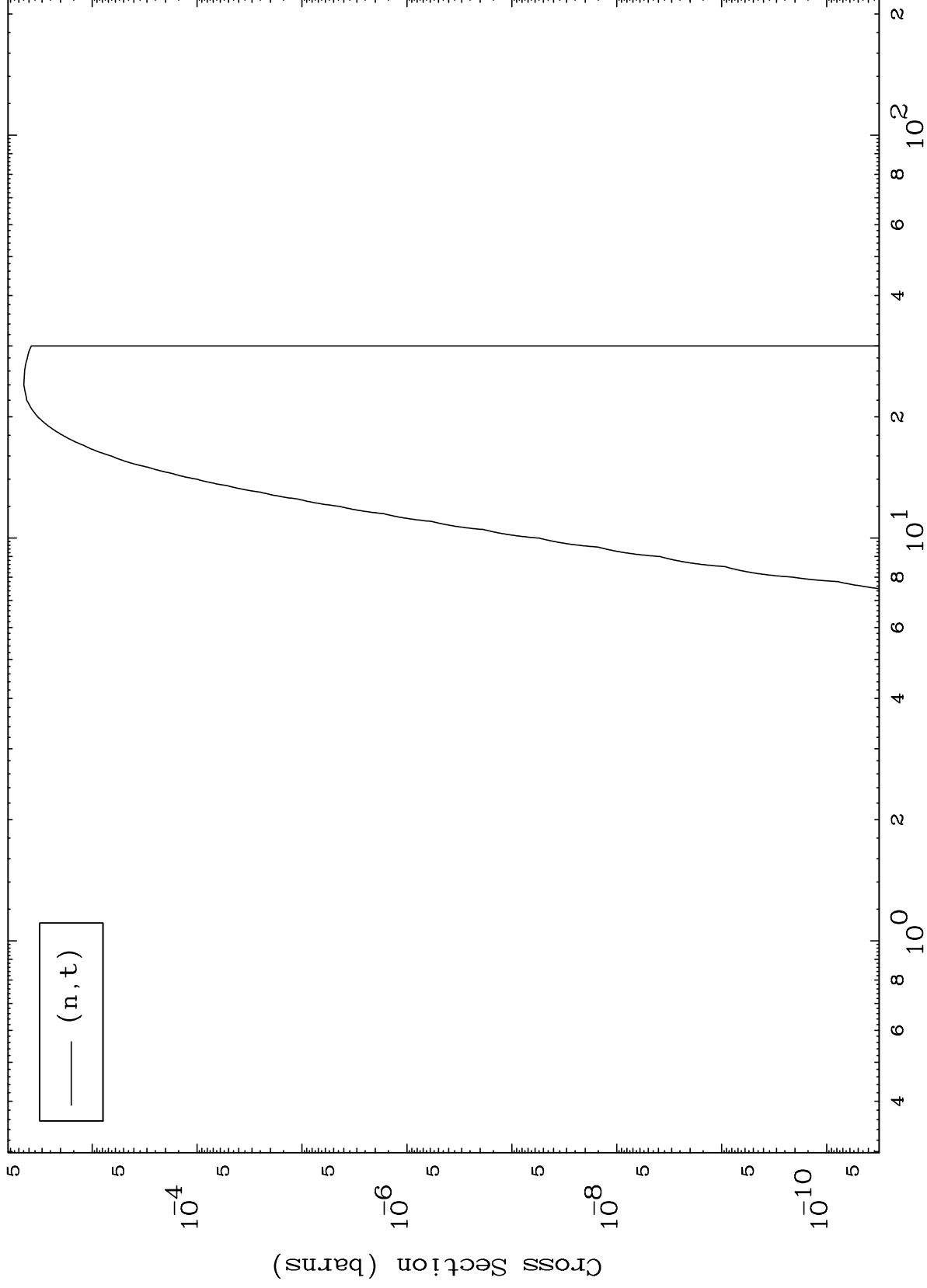
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,t) Levels
293 Kelvin Cross Sections

101-Md-256



13

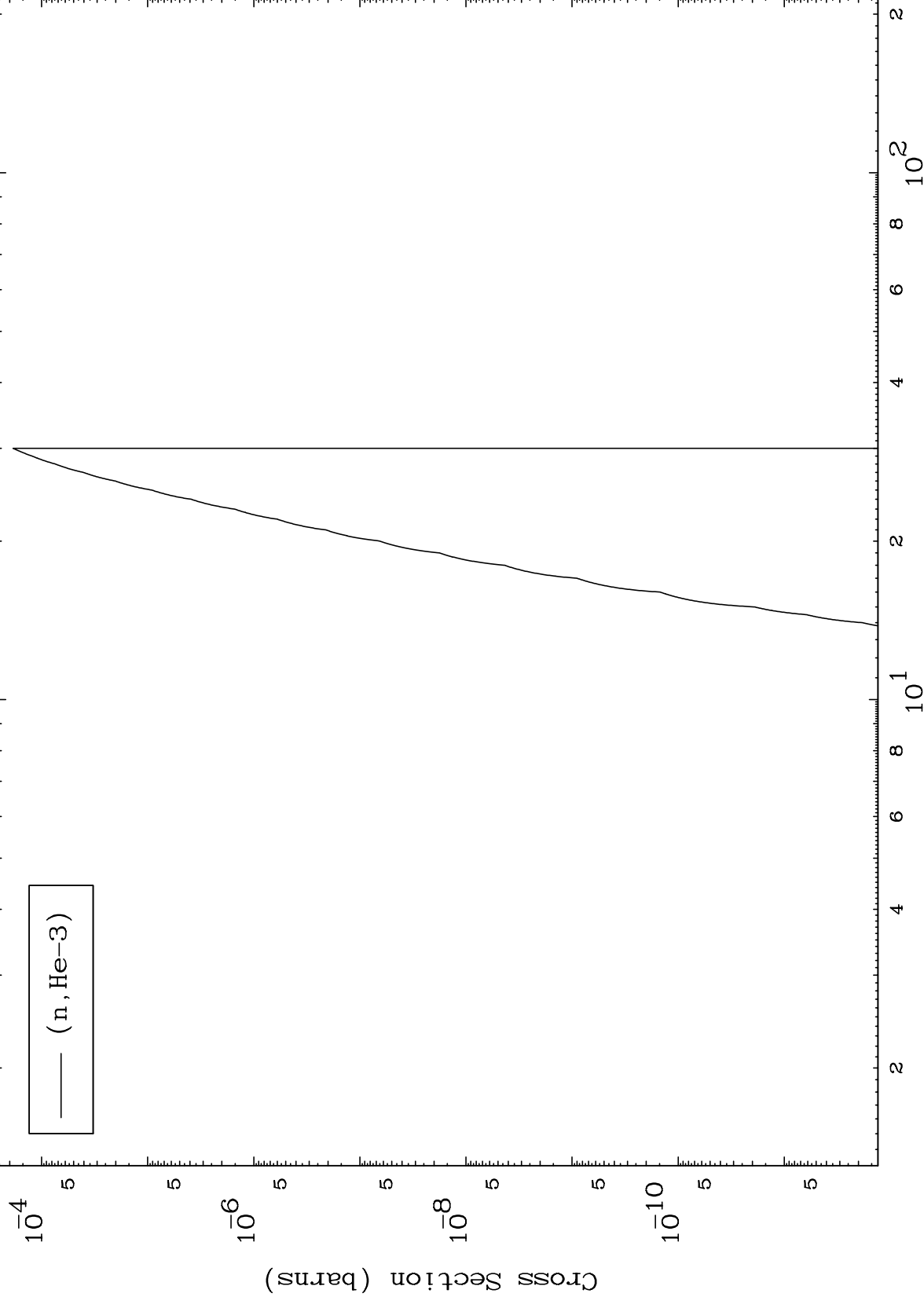
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,He3) Levels
293 Kelvin Cross Sections

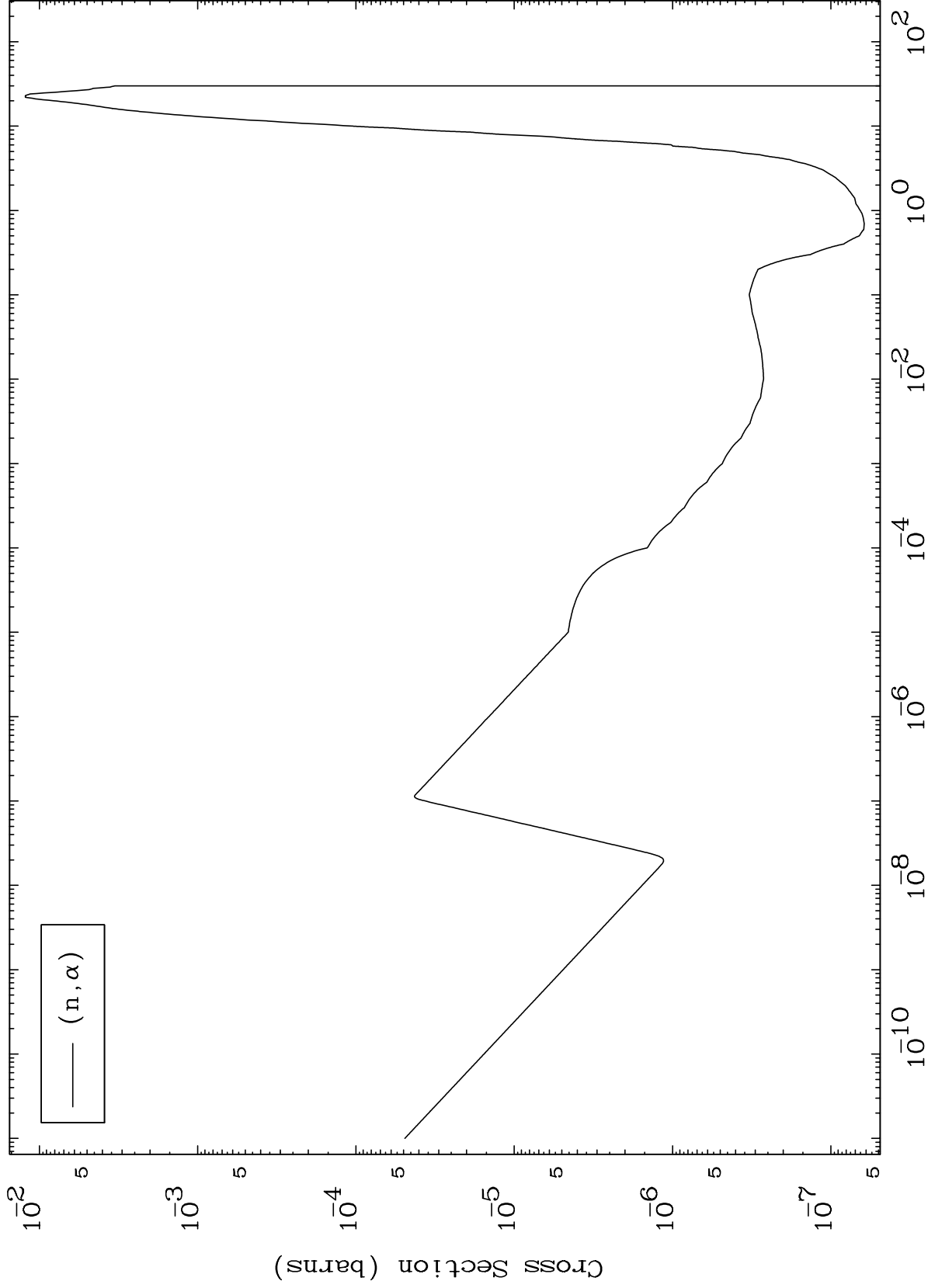
101-Md-256



MAT 9957

(n, α) Levels
293 Kelvin Cross Sections

101-Md-256

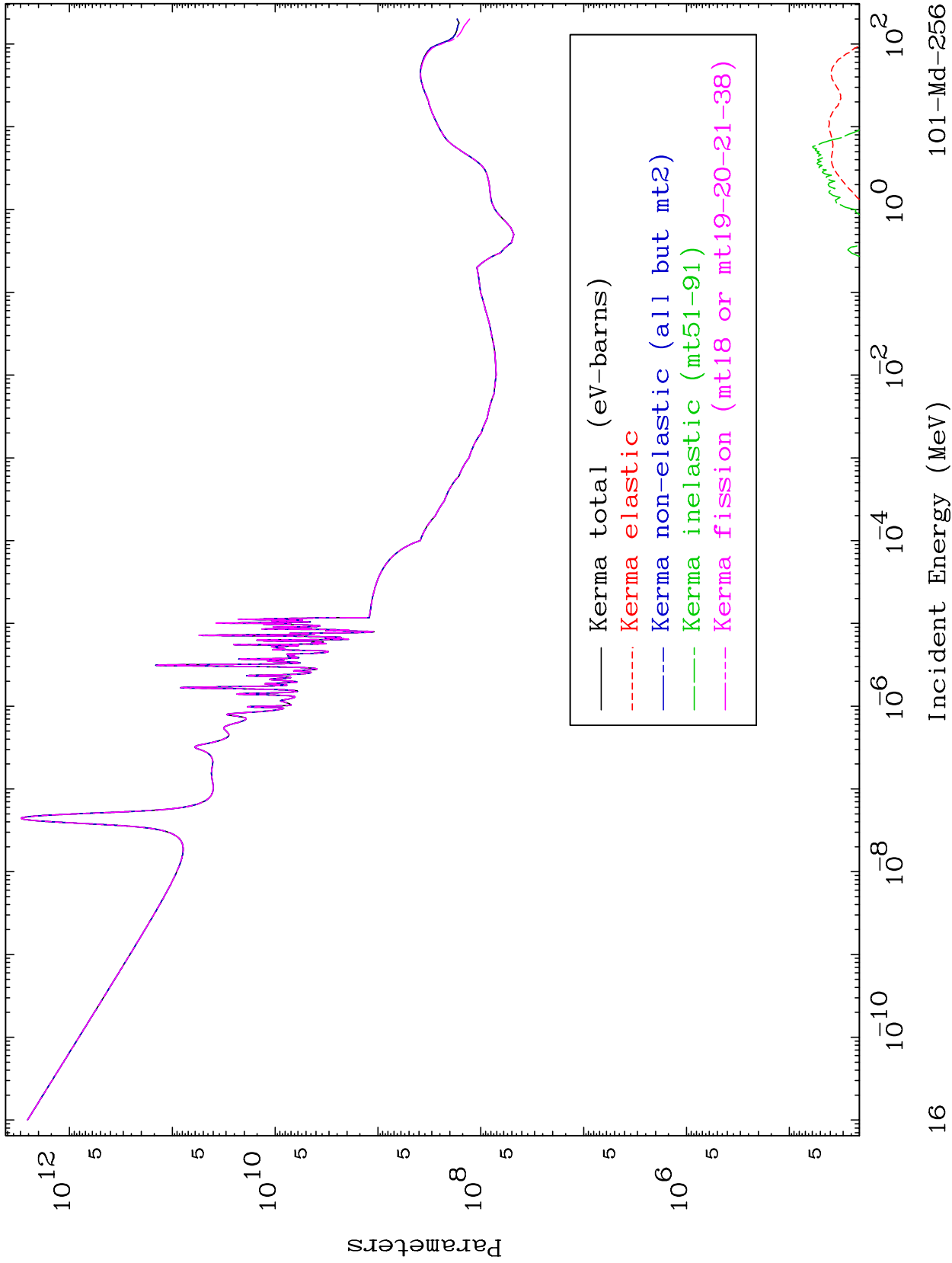


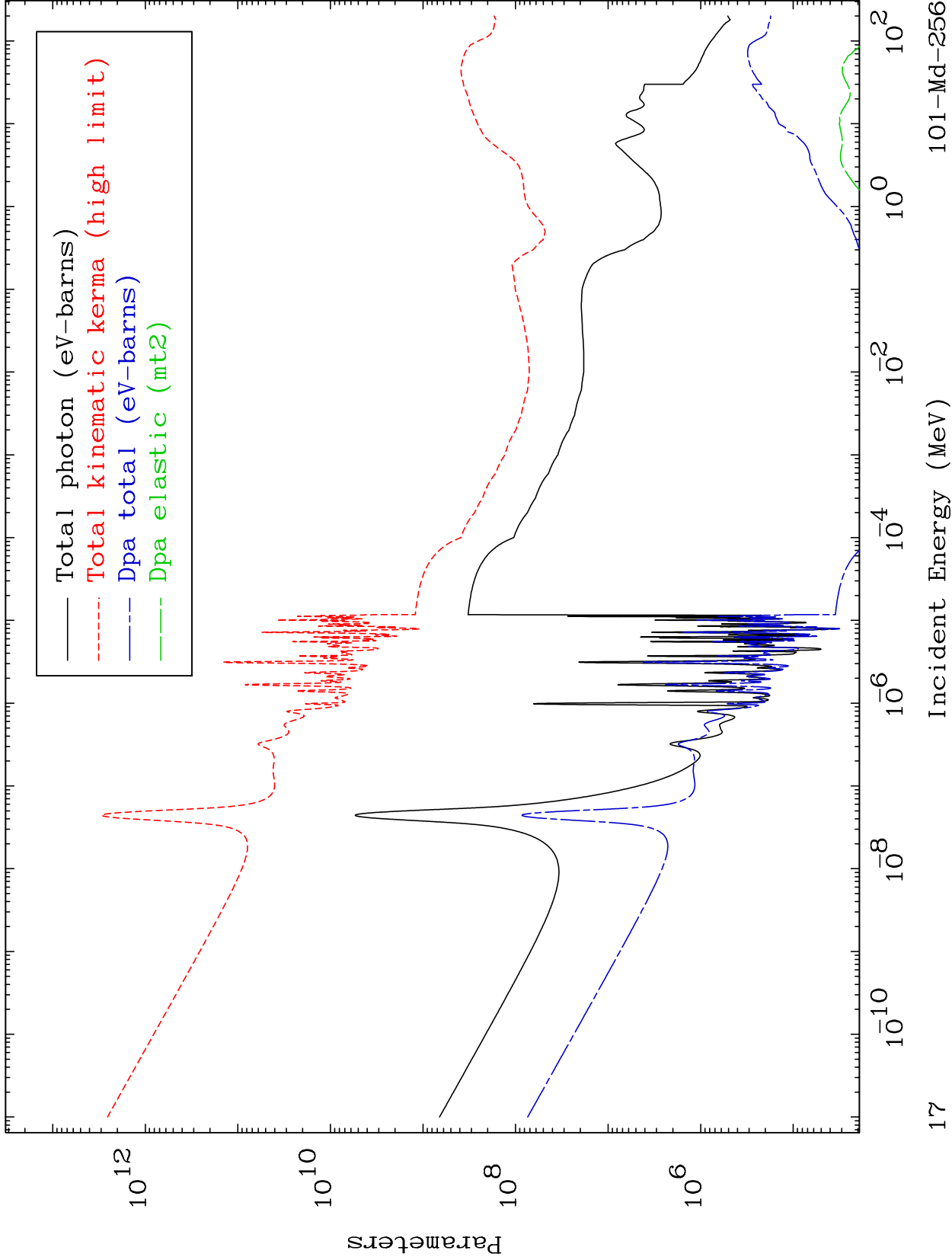
(n, α)

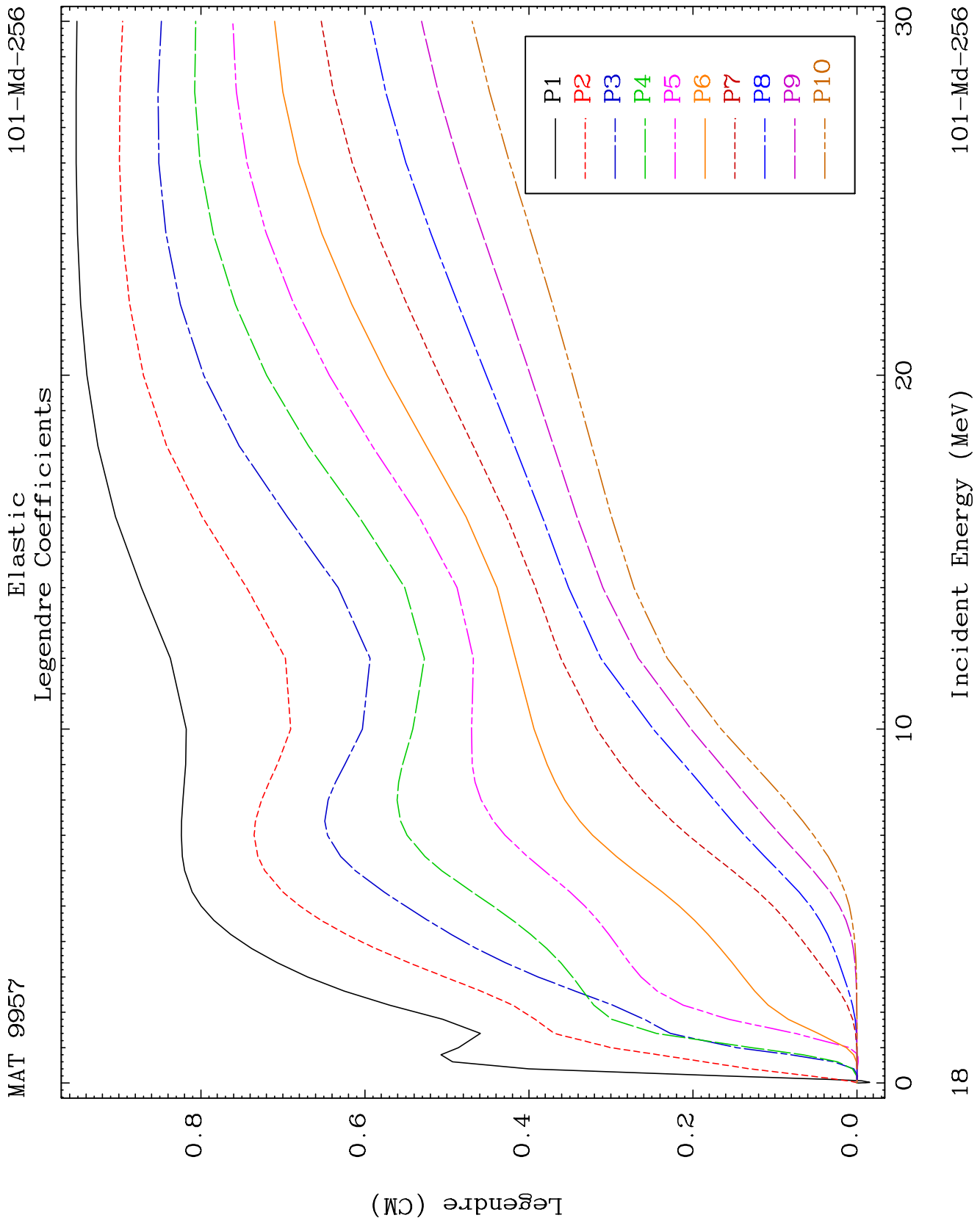
101-Md-256

Incident Energy (MeV)

15



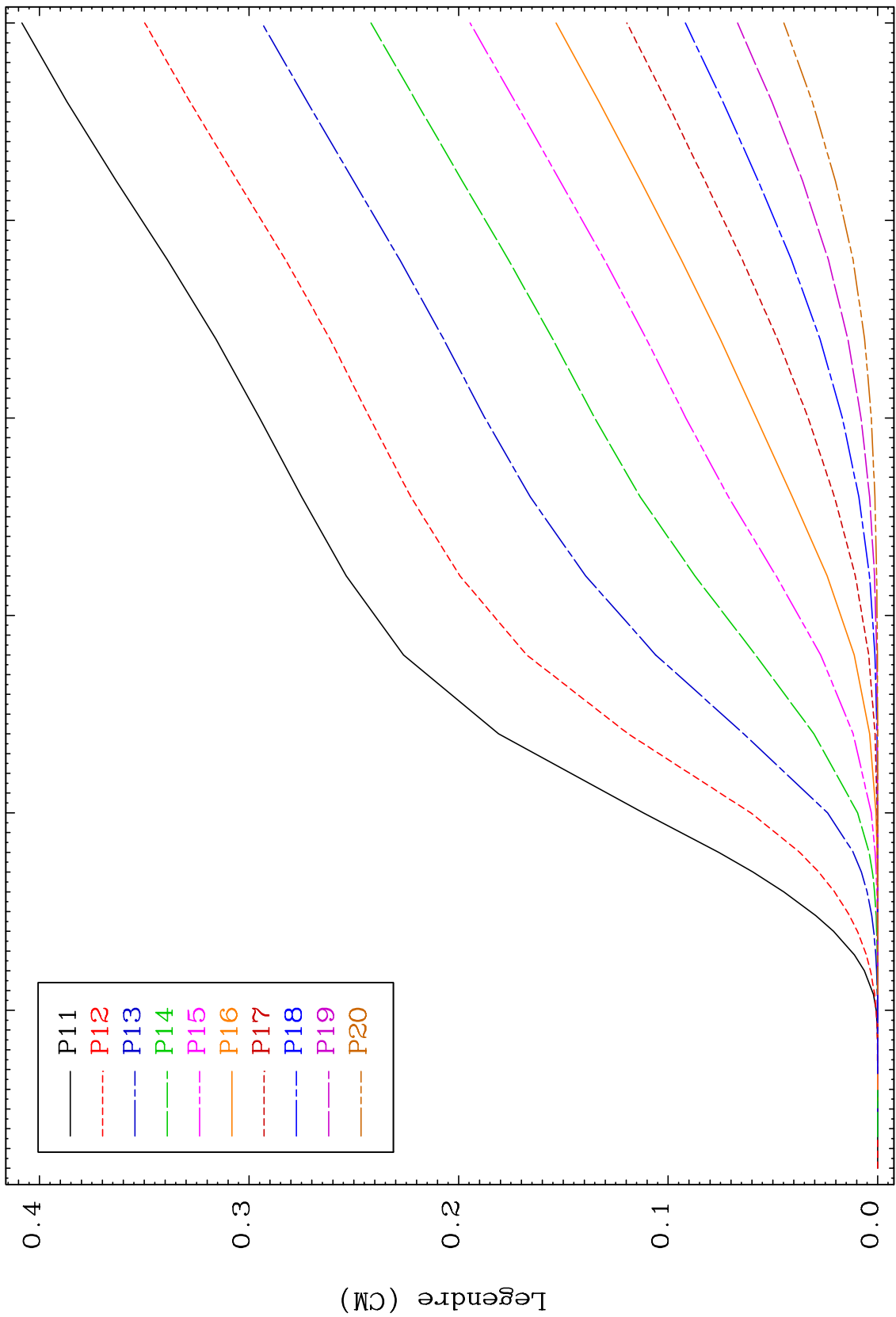




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Elastic Legendre Coefficients

101-Md-256



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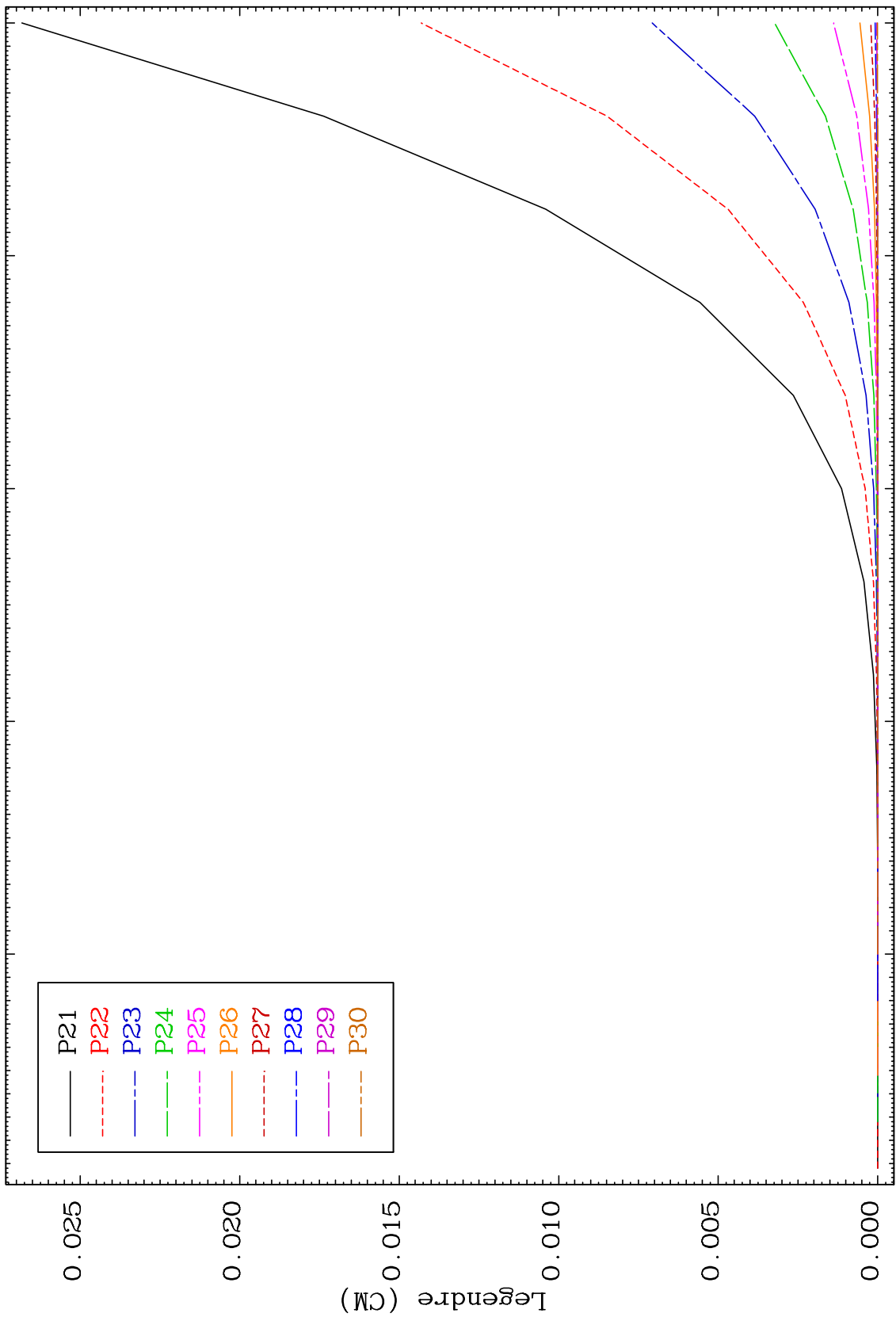
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



20

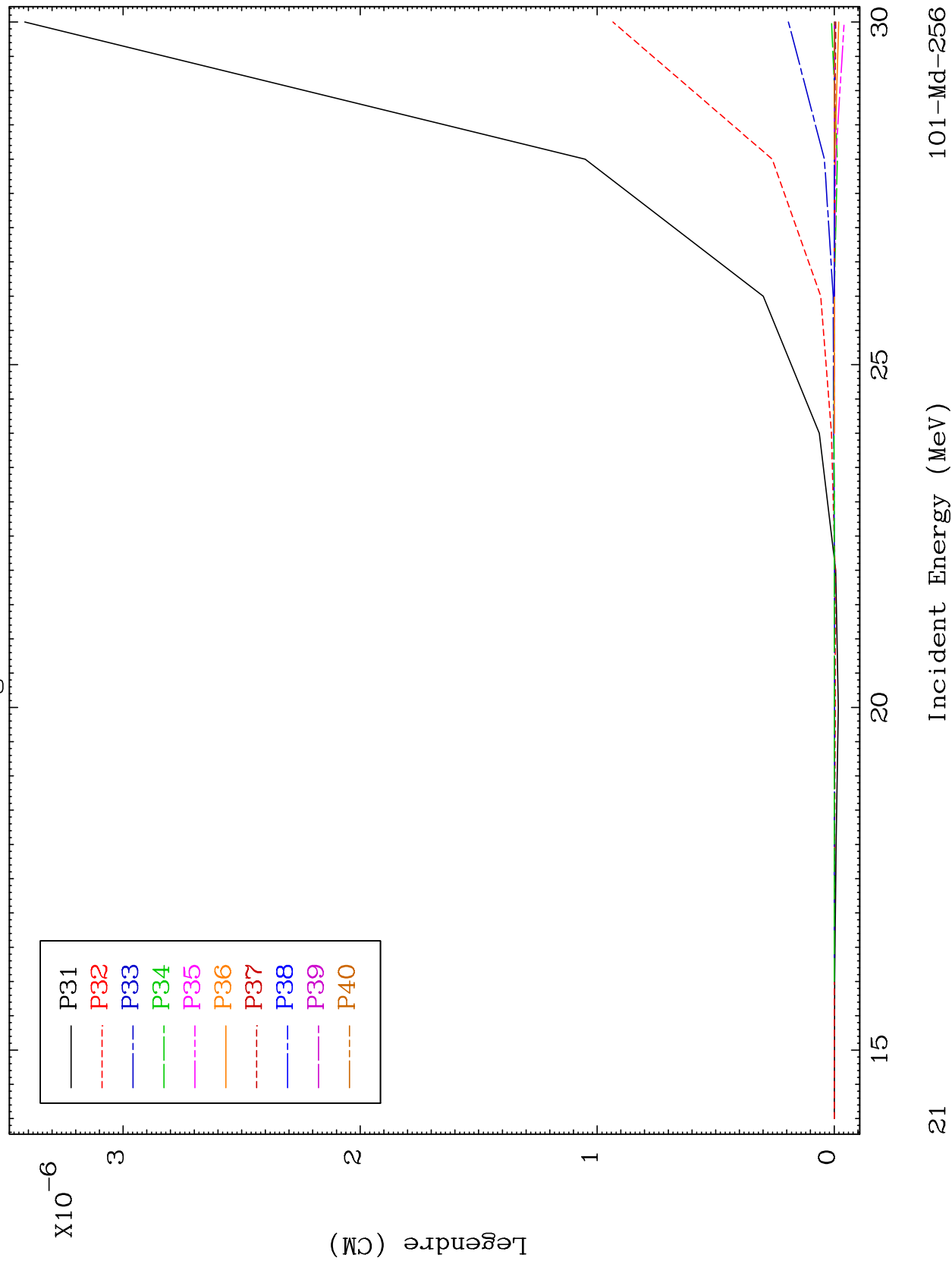
Incident Energy (MeV)

101-Md-256

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Elastic
Legendre Coefficients

101-Md-256



21

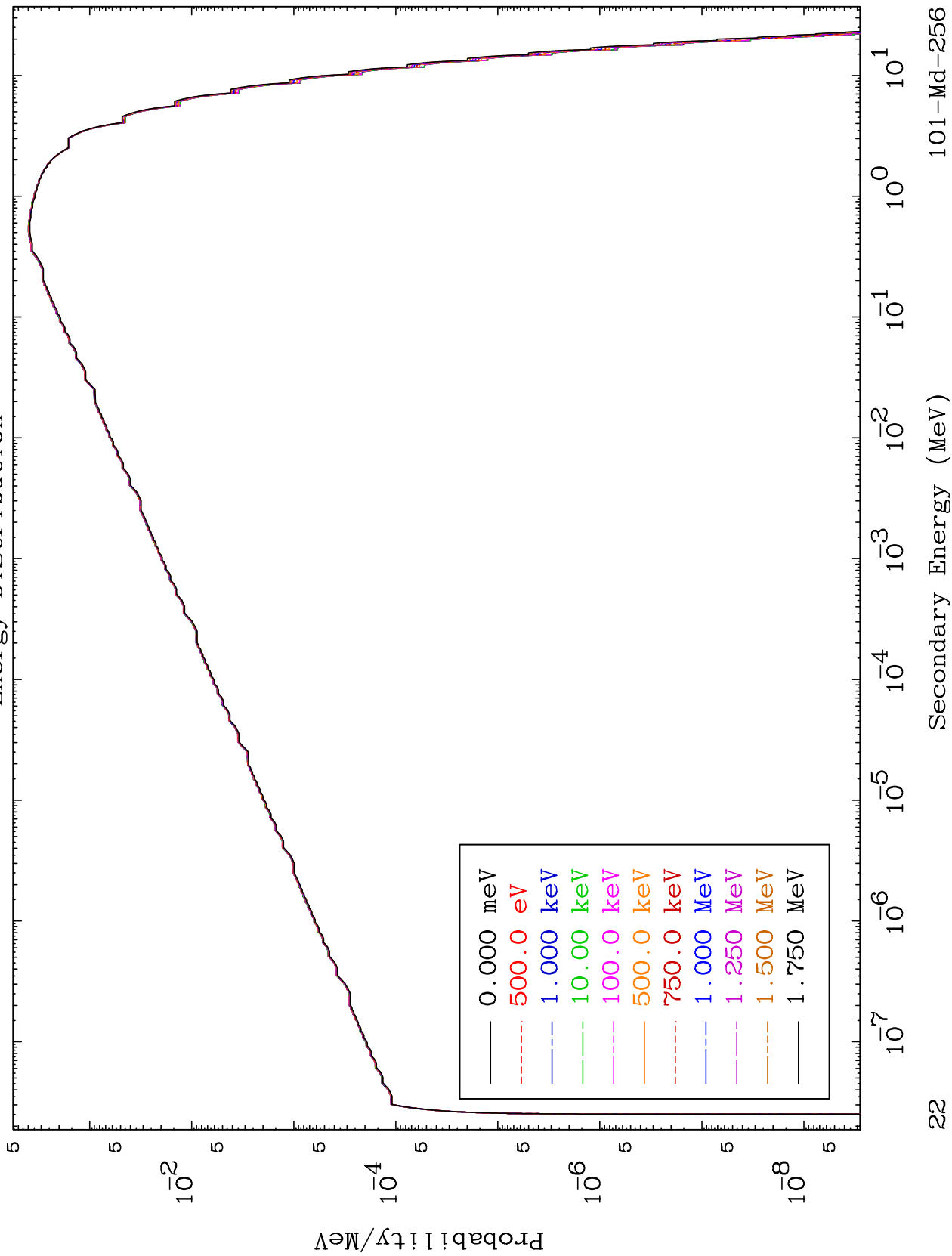
Incident Energy (MeV)

101-Md-256

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Fission Energy Distribution

101-Md-256

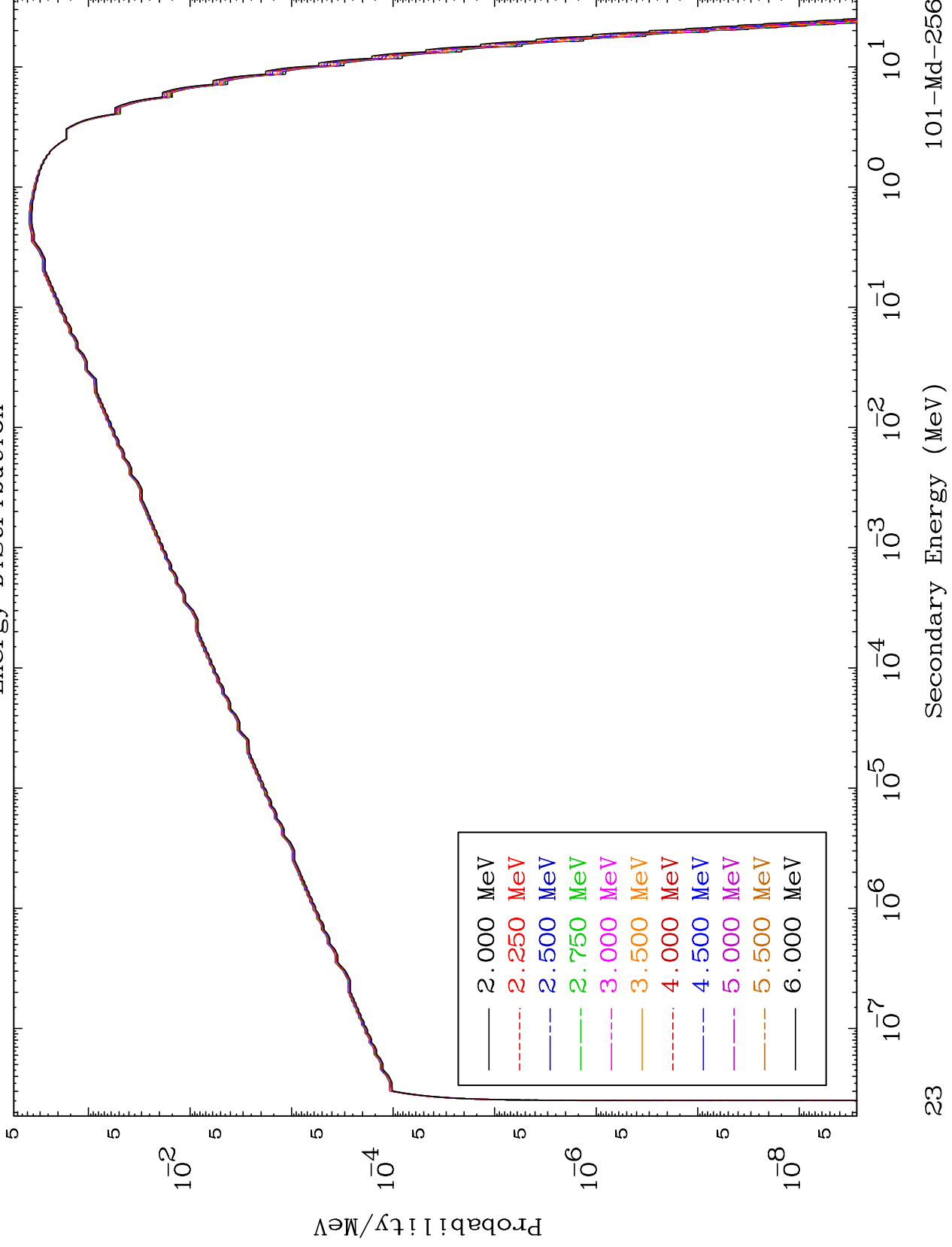


101-Md-256

MAT 9957

Fission Energy Distribution

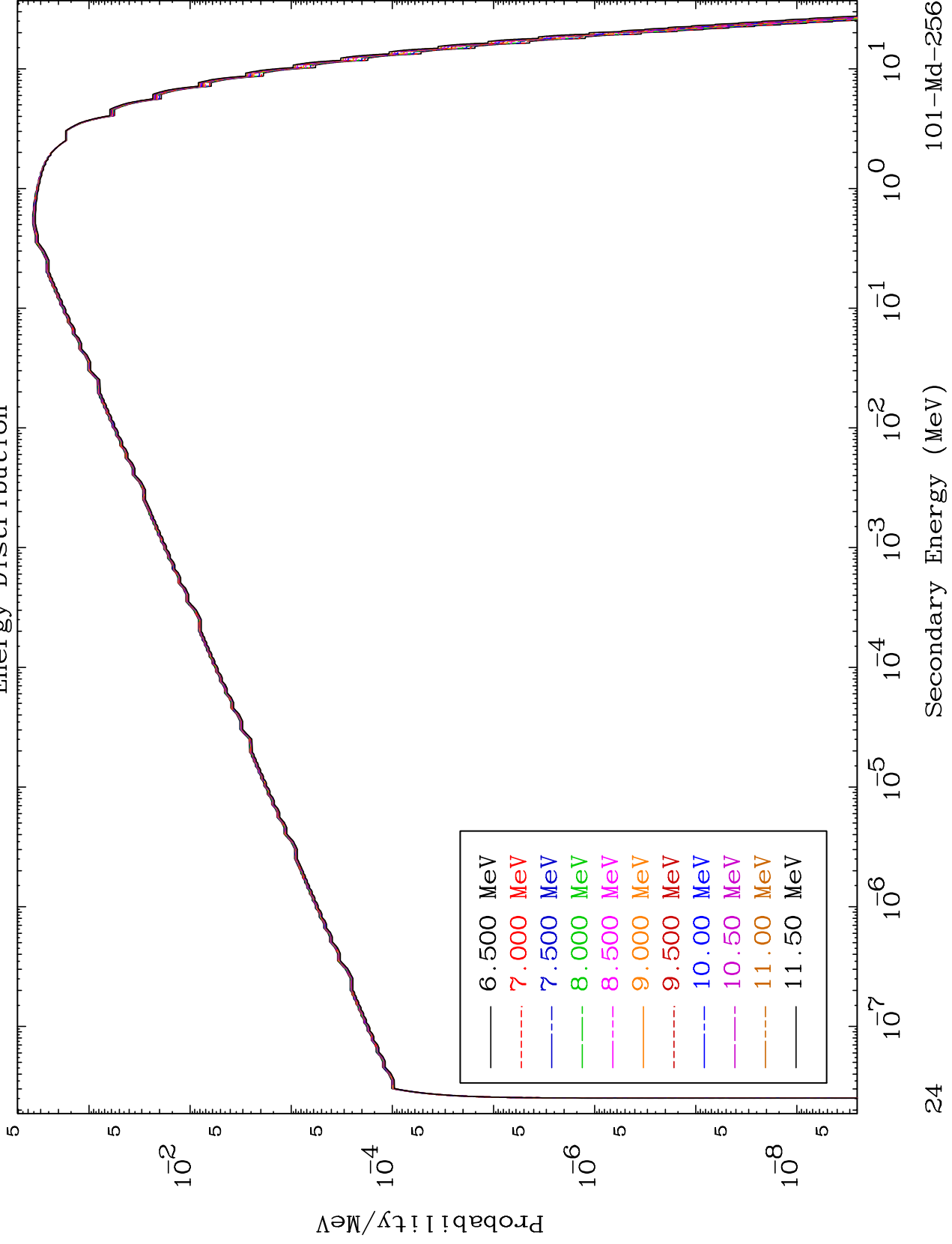
101-Md-256



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Fission
Energy Distribution

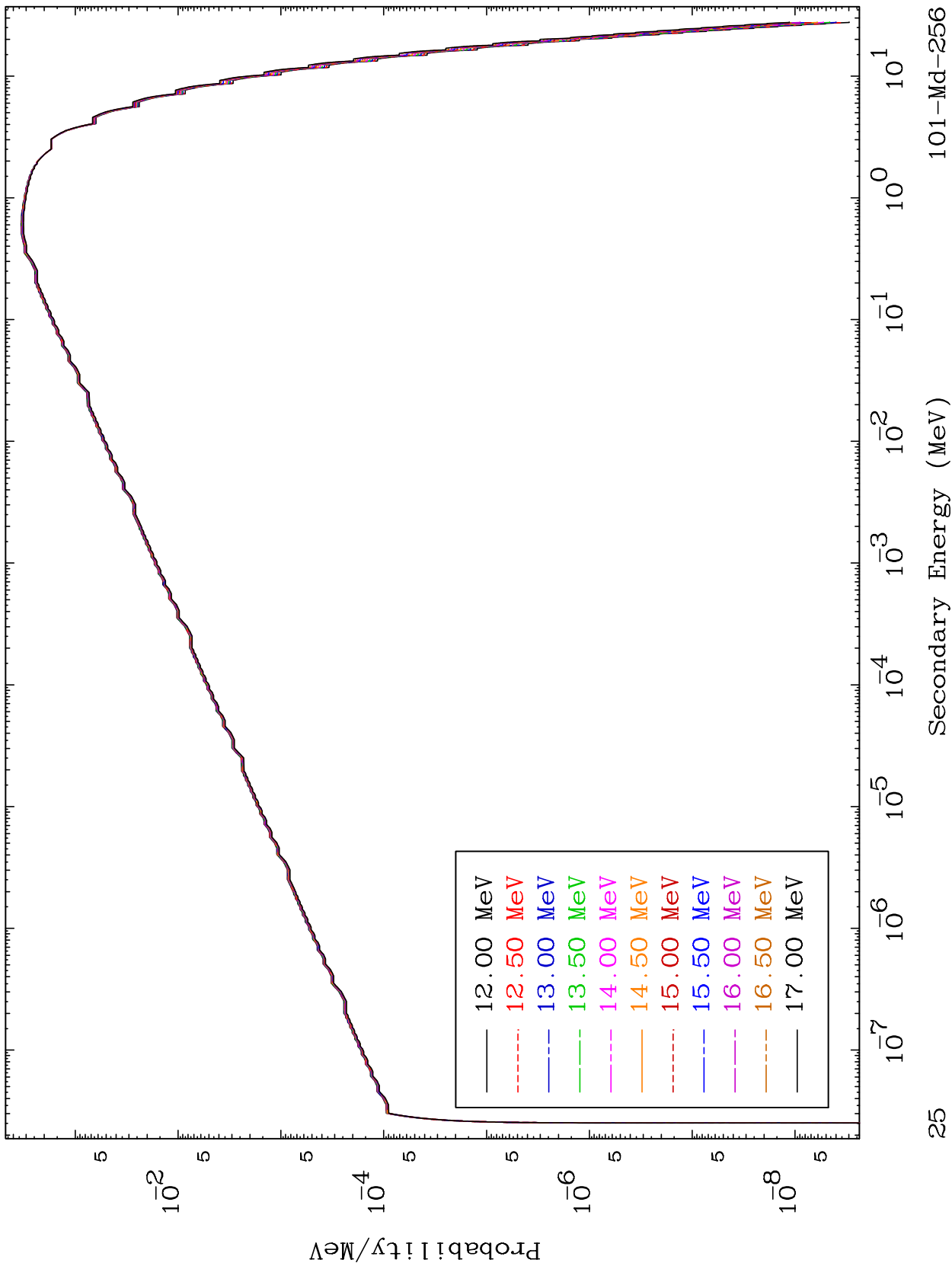
101-Md-256



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Fission
Energy Distribution

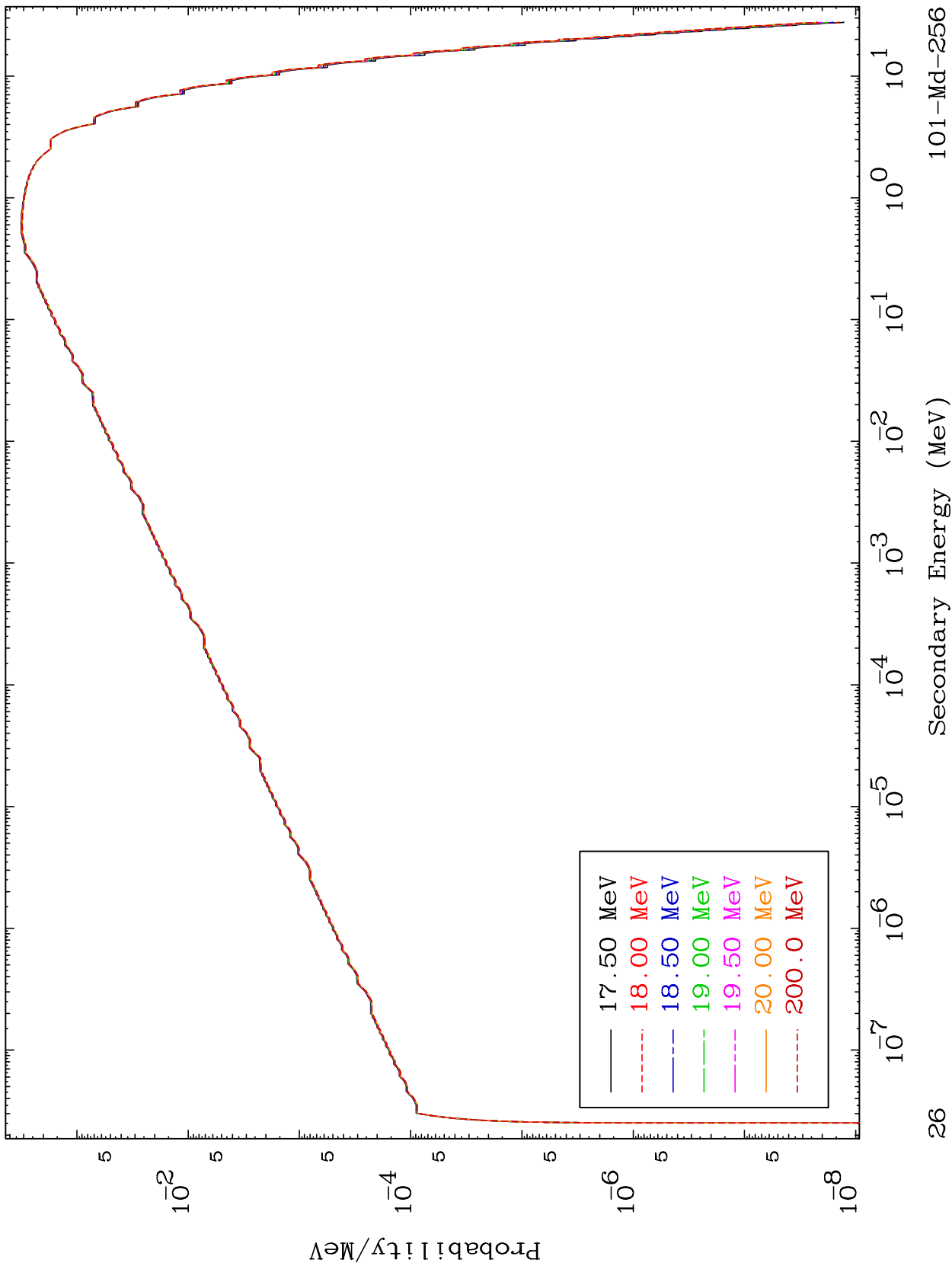
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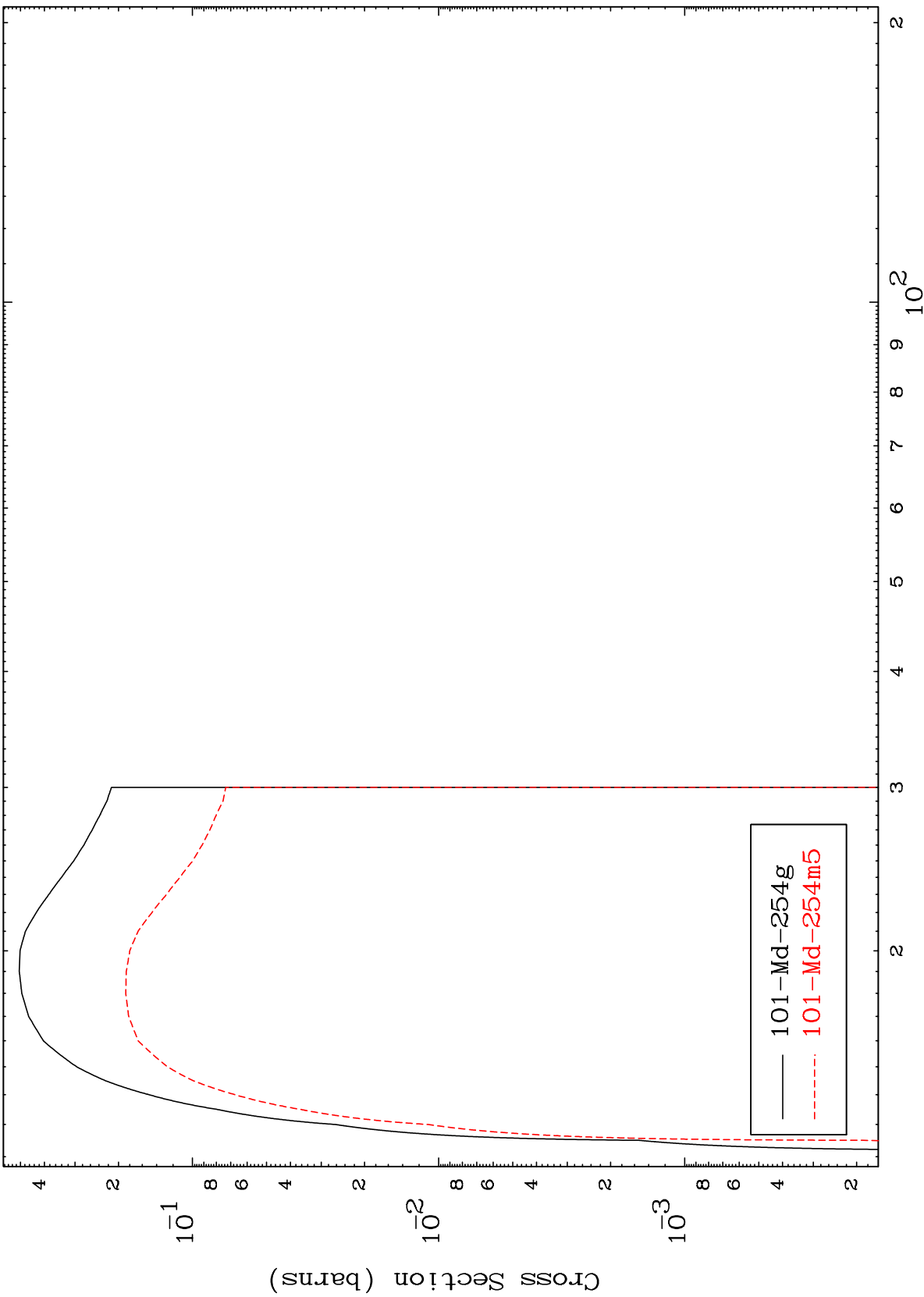
101-Md-256

Secondary Energy (MeV)

Fission
Energy Distribution



Radionuclide Production Cross Section (n,3n)

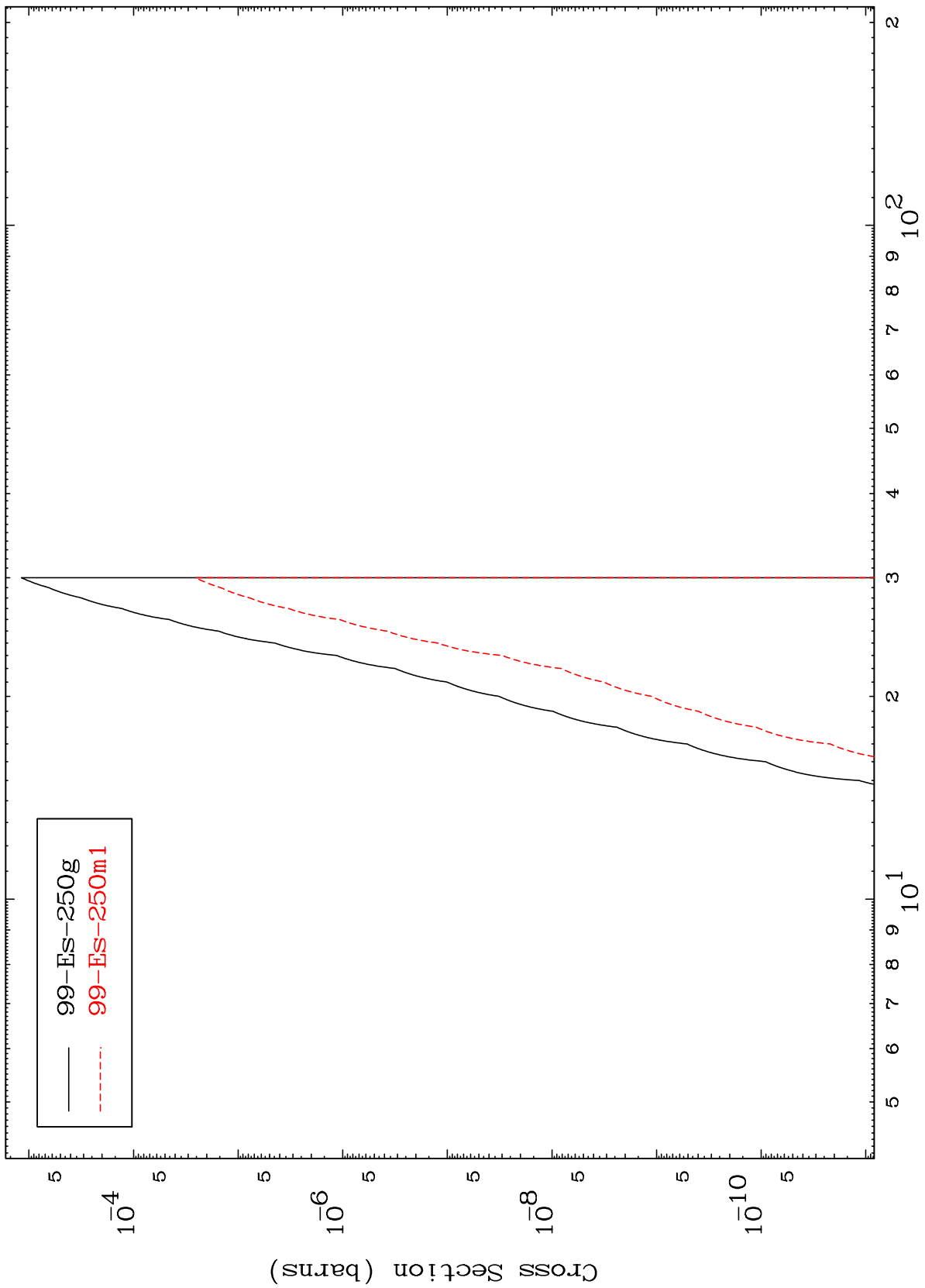


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(n,3n) α

101-Md-256

Radionuclide Production Cross Section



28

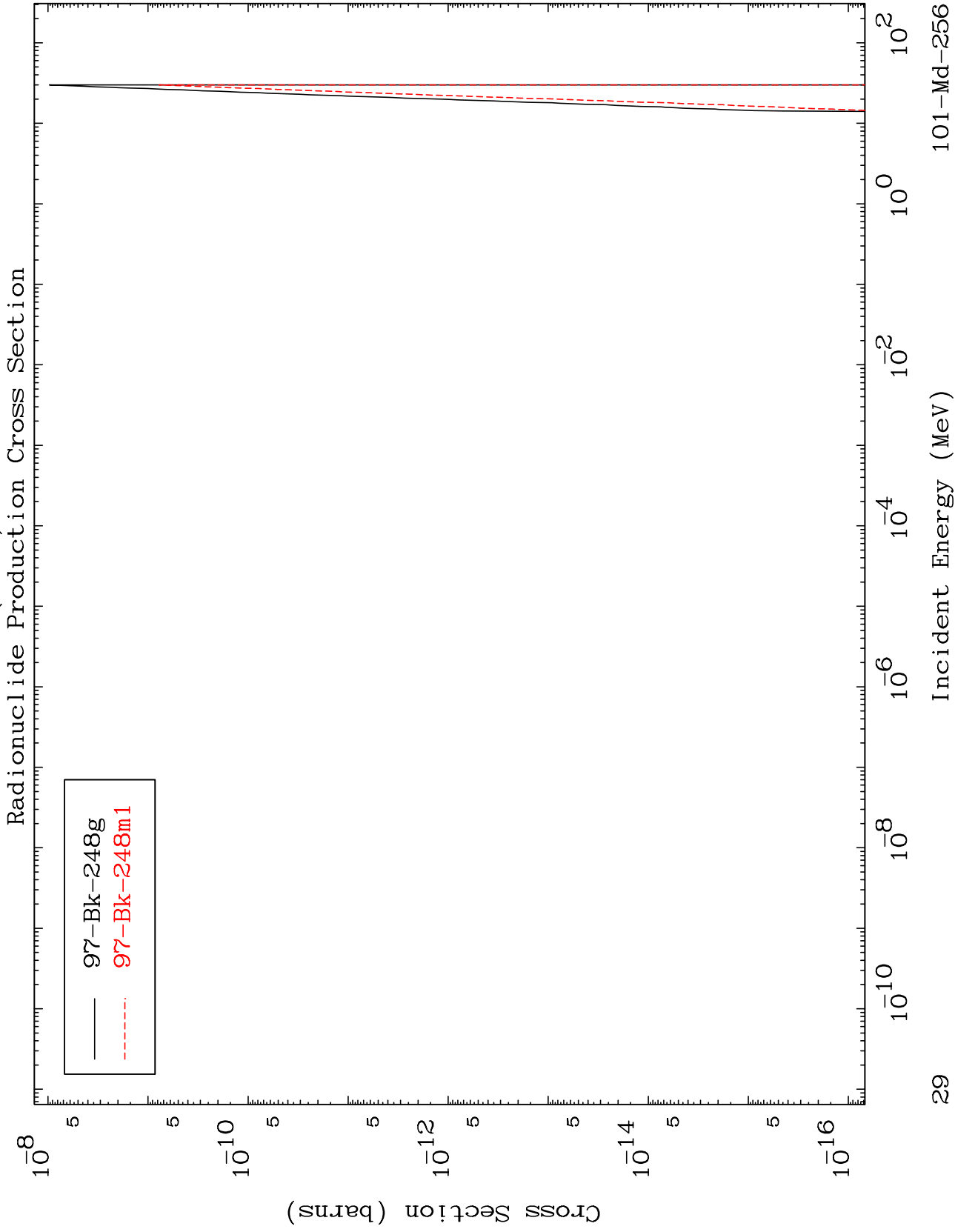
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') 2 α

101-Md-256

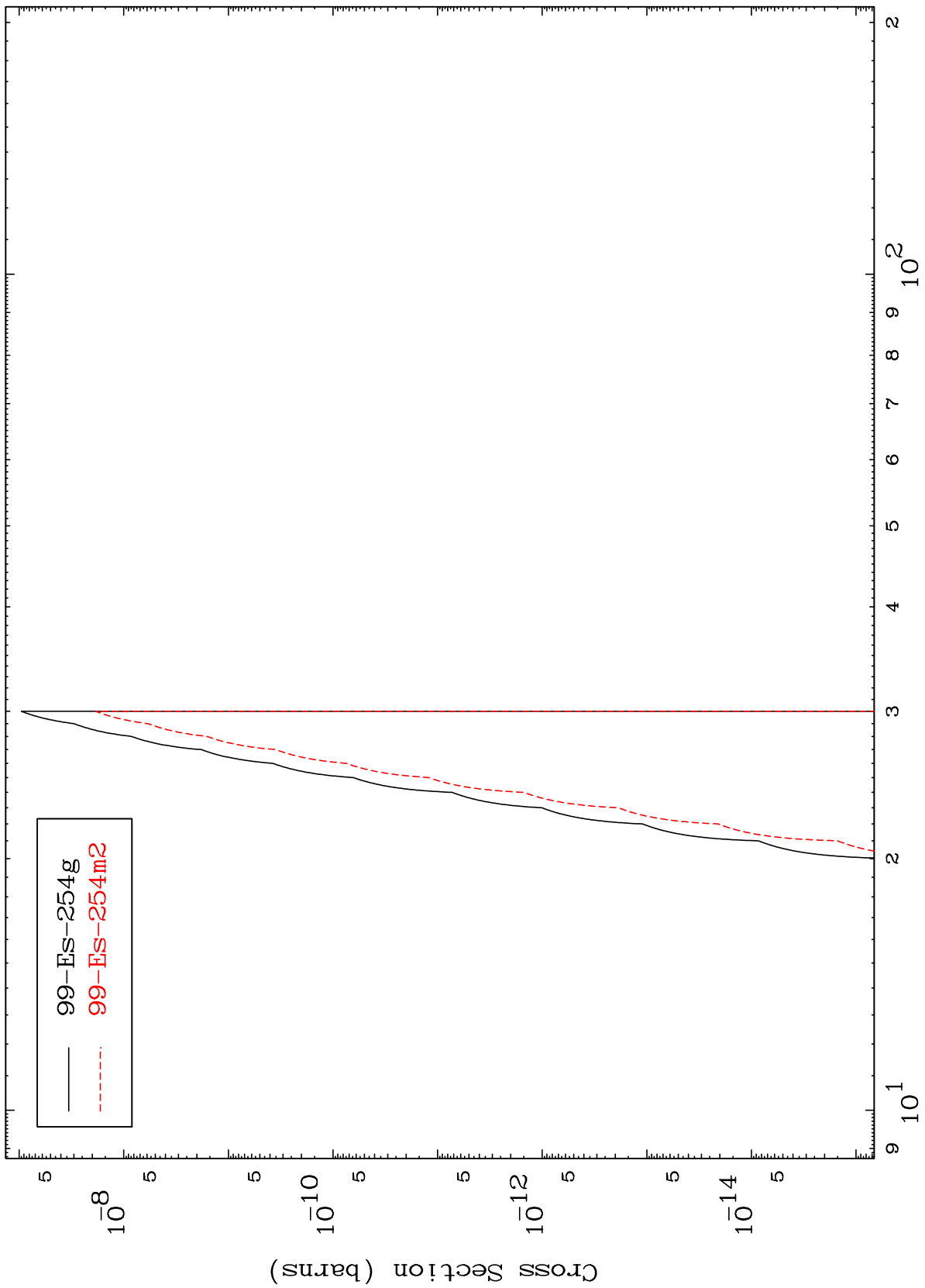


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(n,2n) p

101-Md-256

Radionuclide Production Cross Section



— 99-Es-254g
- - - 99-Es-254m2

30

Incident Energy (MeV)

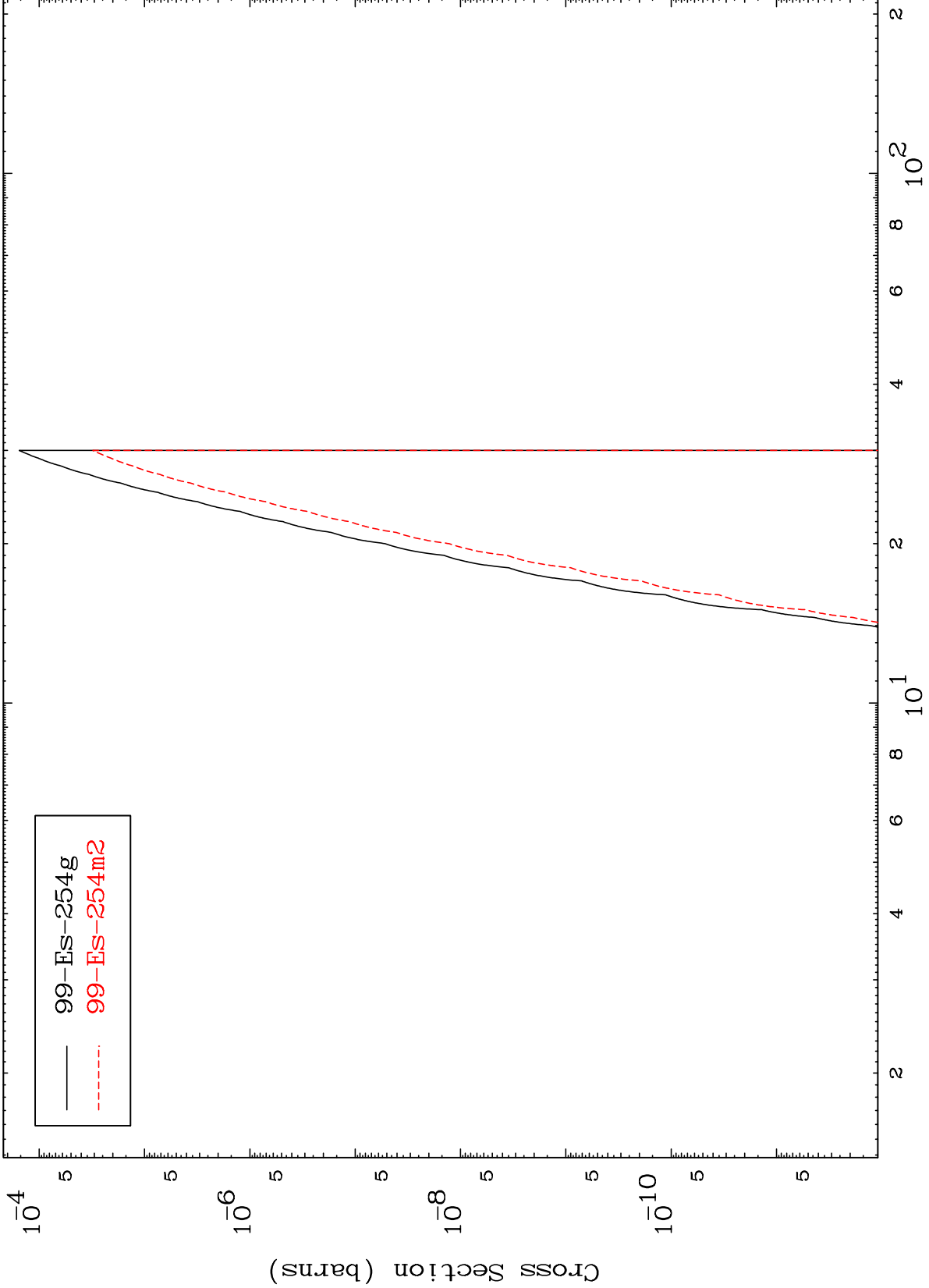
101-Md-256

MAT 9957

(n,He-3)

101-Md-256

Radionuclide Production Cross Section



99-Es-254g
99-Es-254m2