

Program EVALPLOT  
(Version 2017-1)

by

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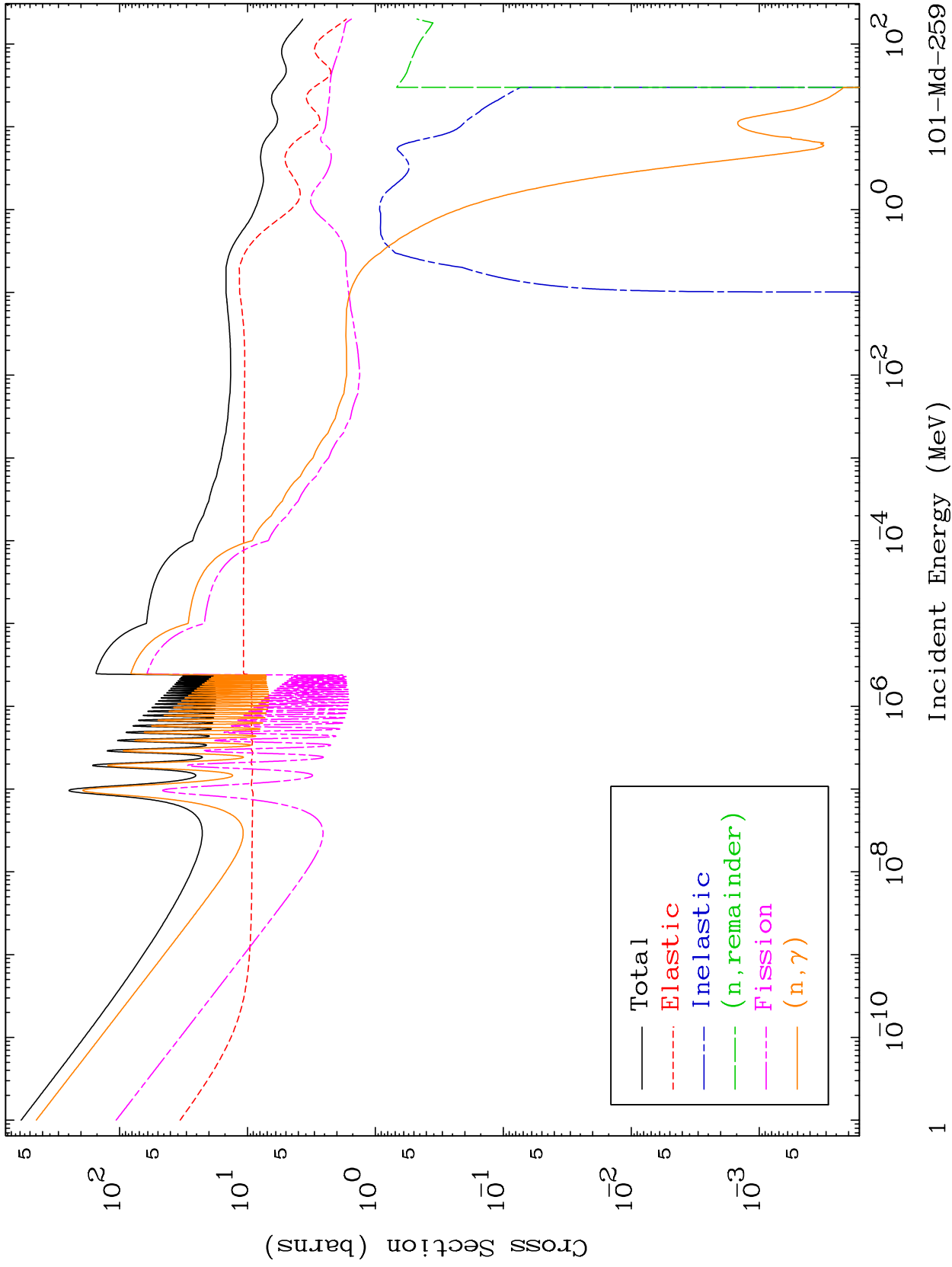
Press Mouse Button to Start

MAT 9960

Major

293 Kelvin Cross Sections

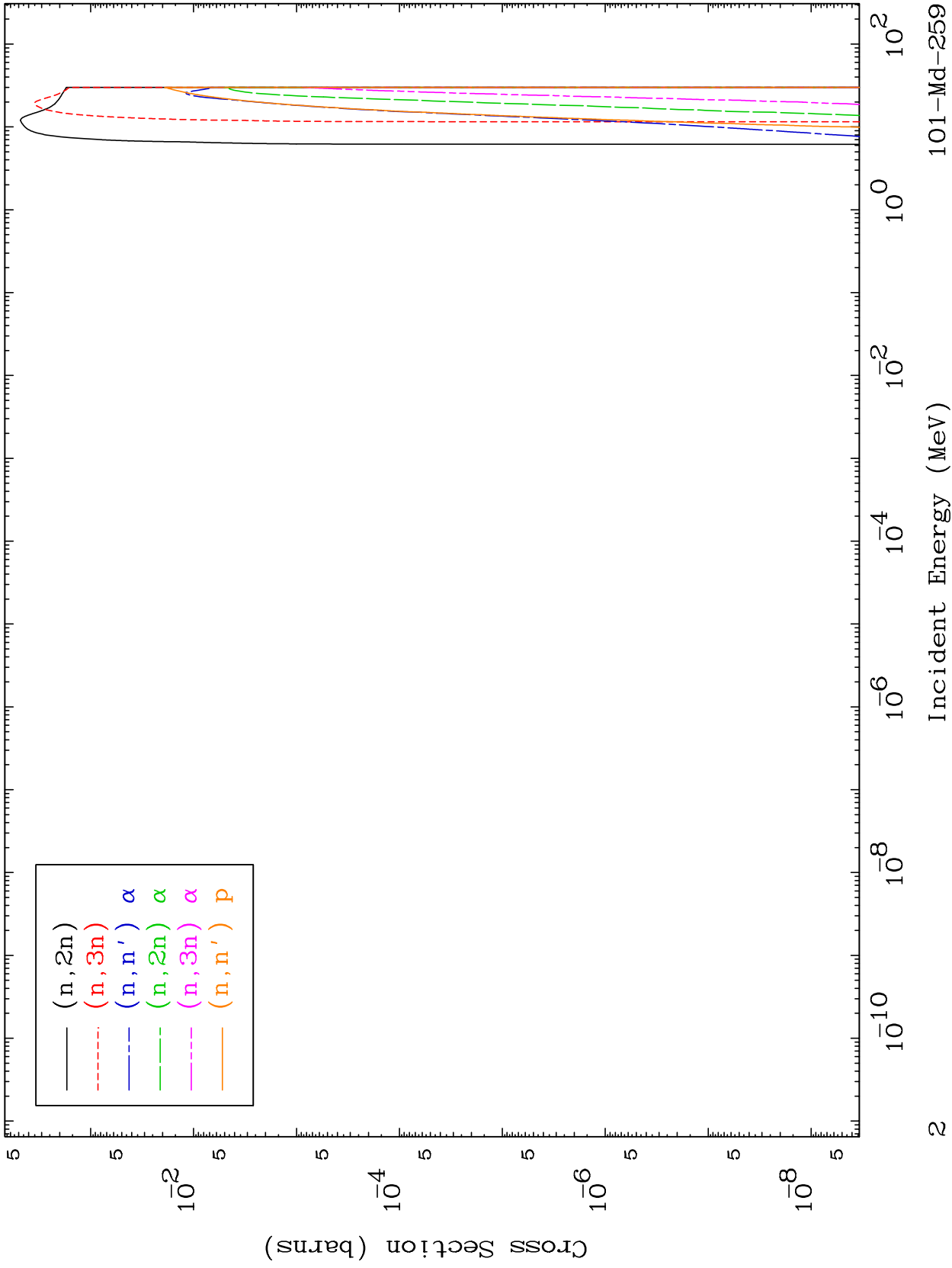
101-Md-259

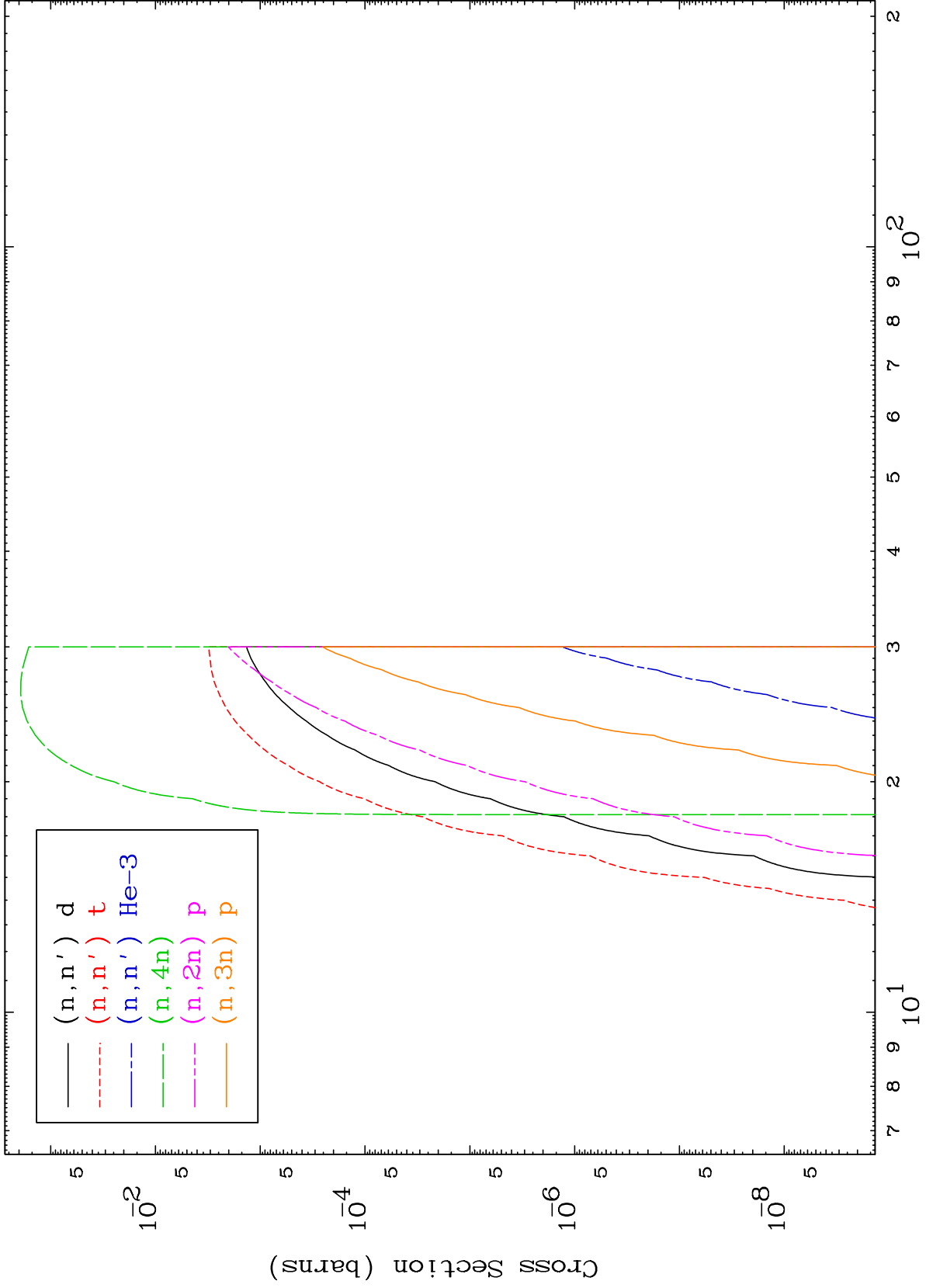


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Neutron Production  
293 Kelvin Cross Sections

101-Md-259

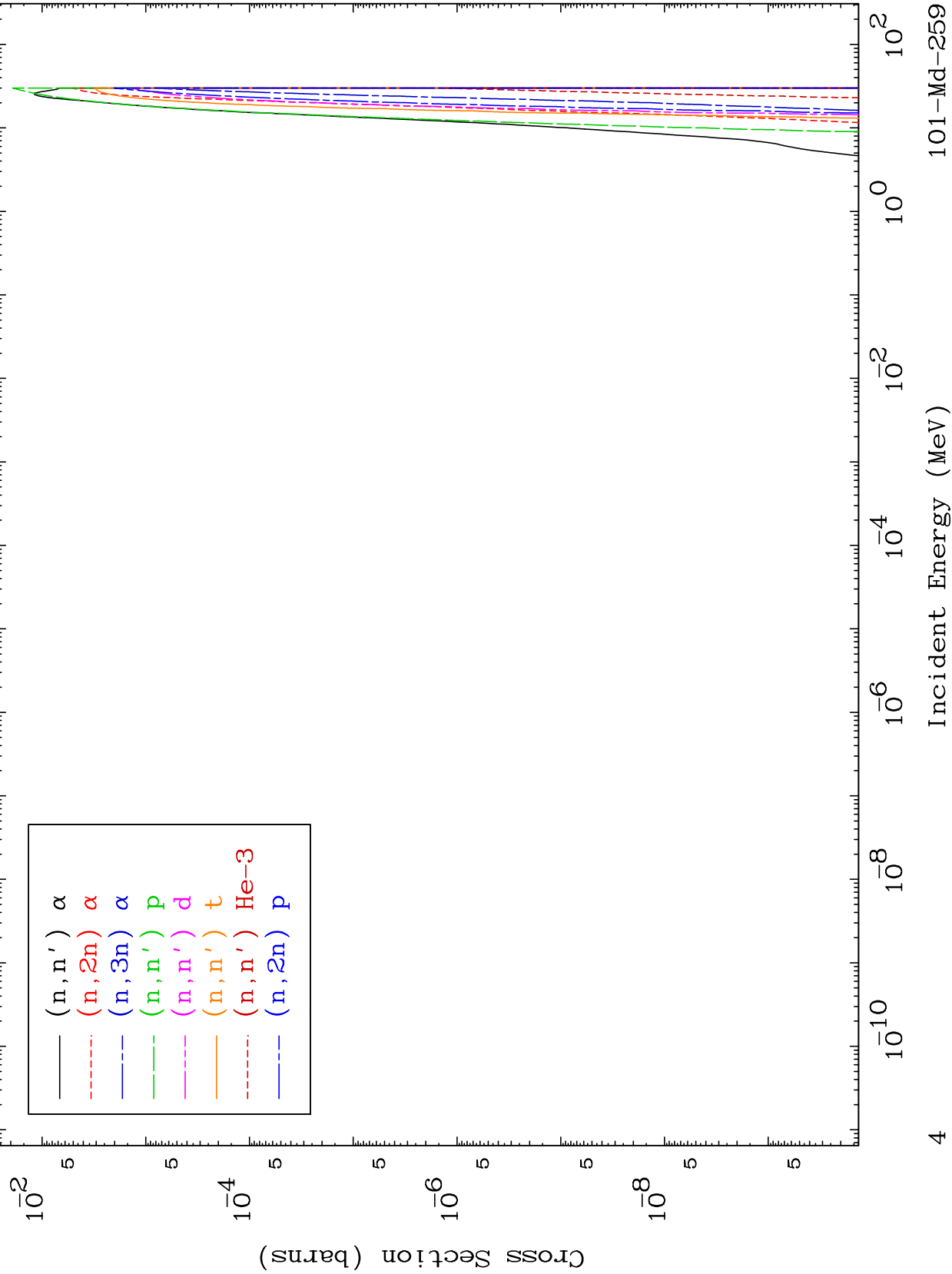


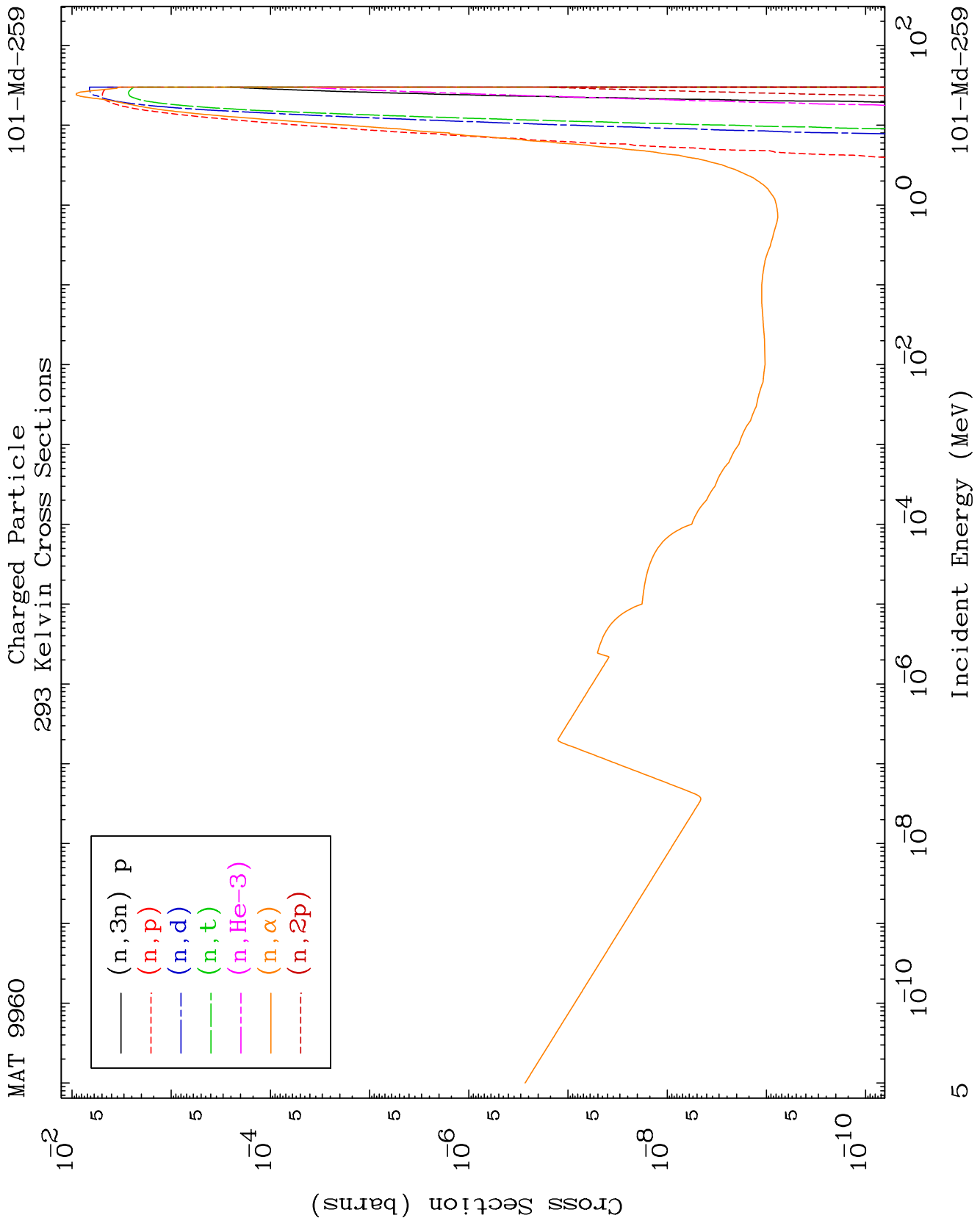


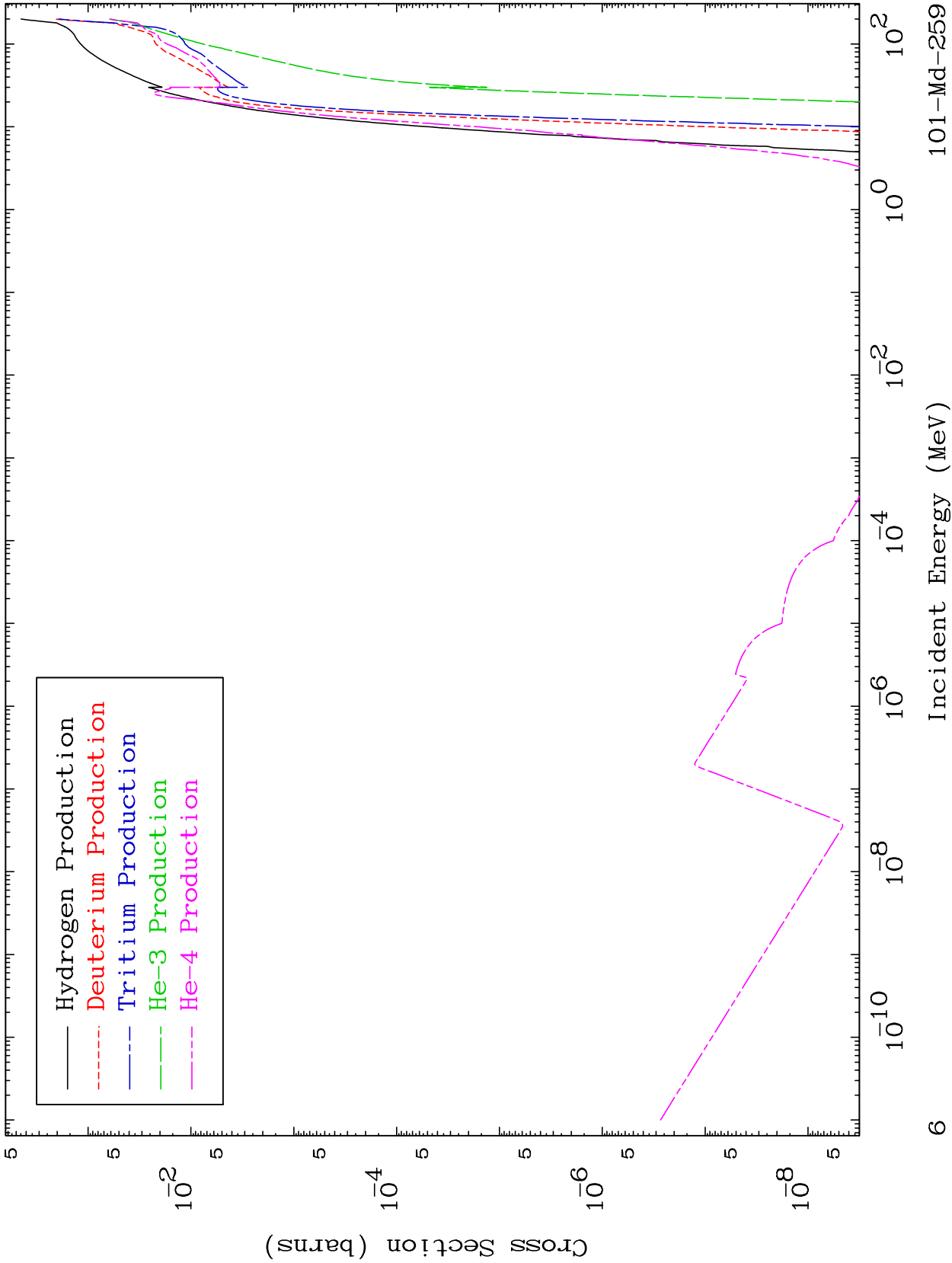
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Charged Particle  
293 Kelvin Cross Sections

101-Md-259



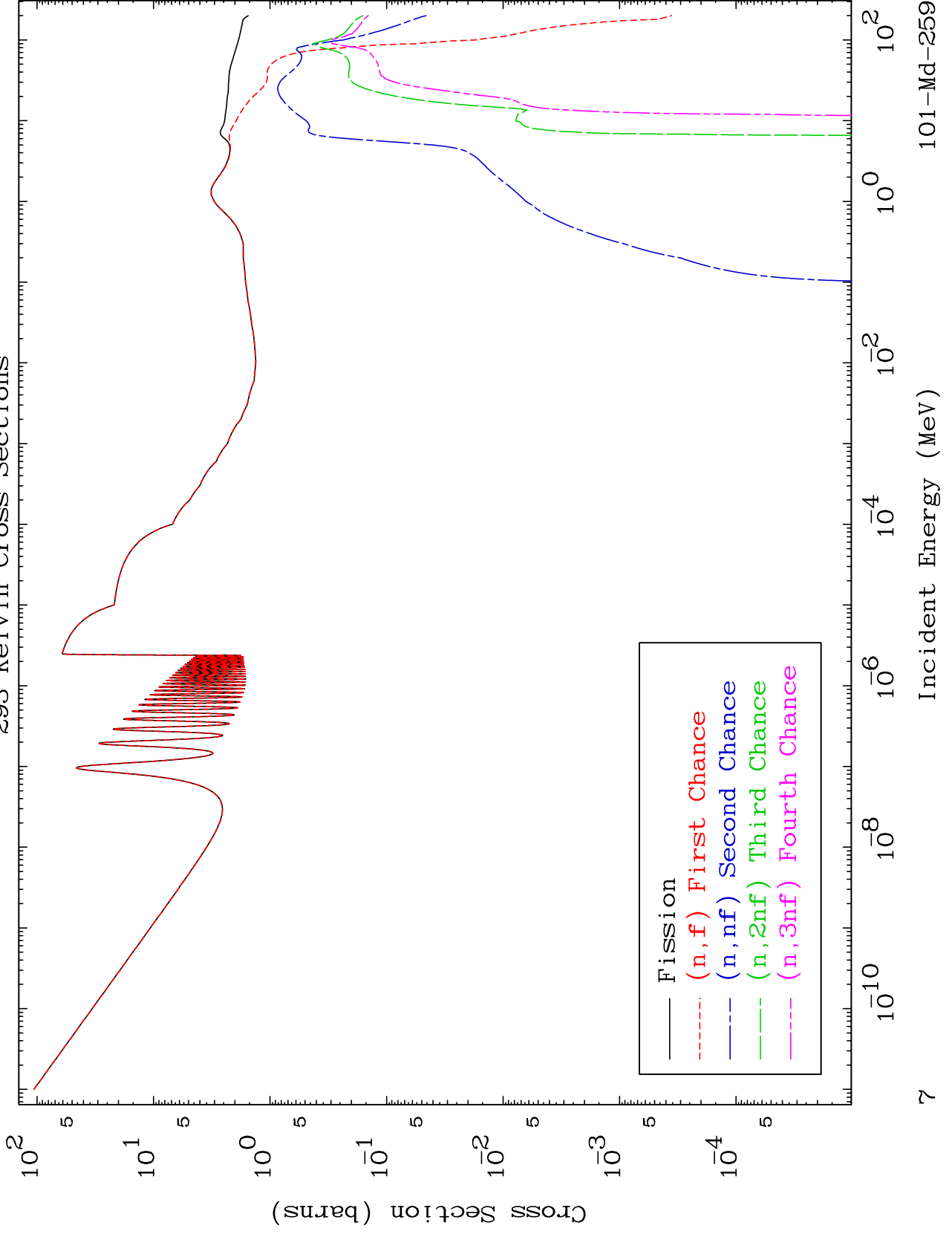




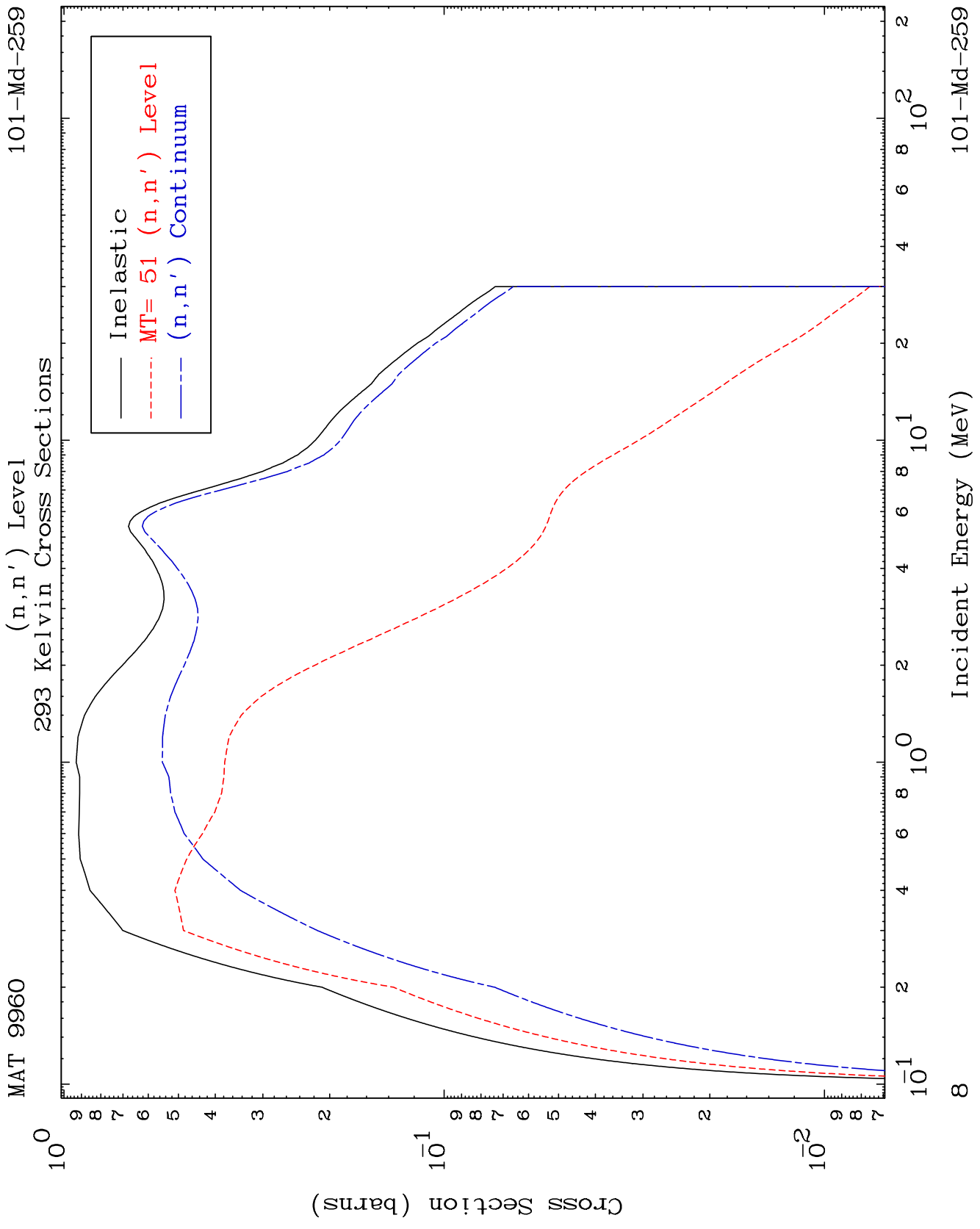
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Fission  
293 Kelvin Cross Sections

101-Md-259



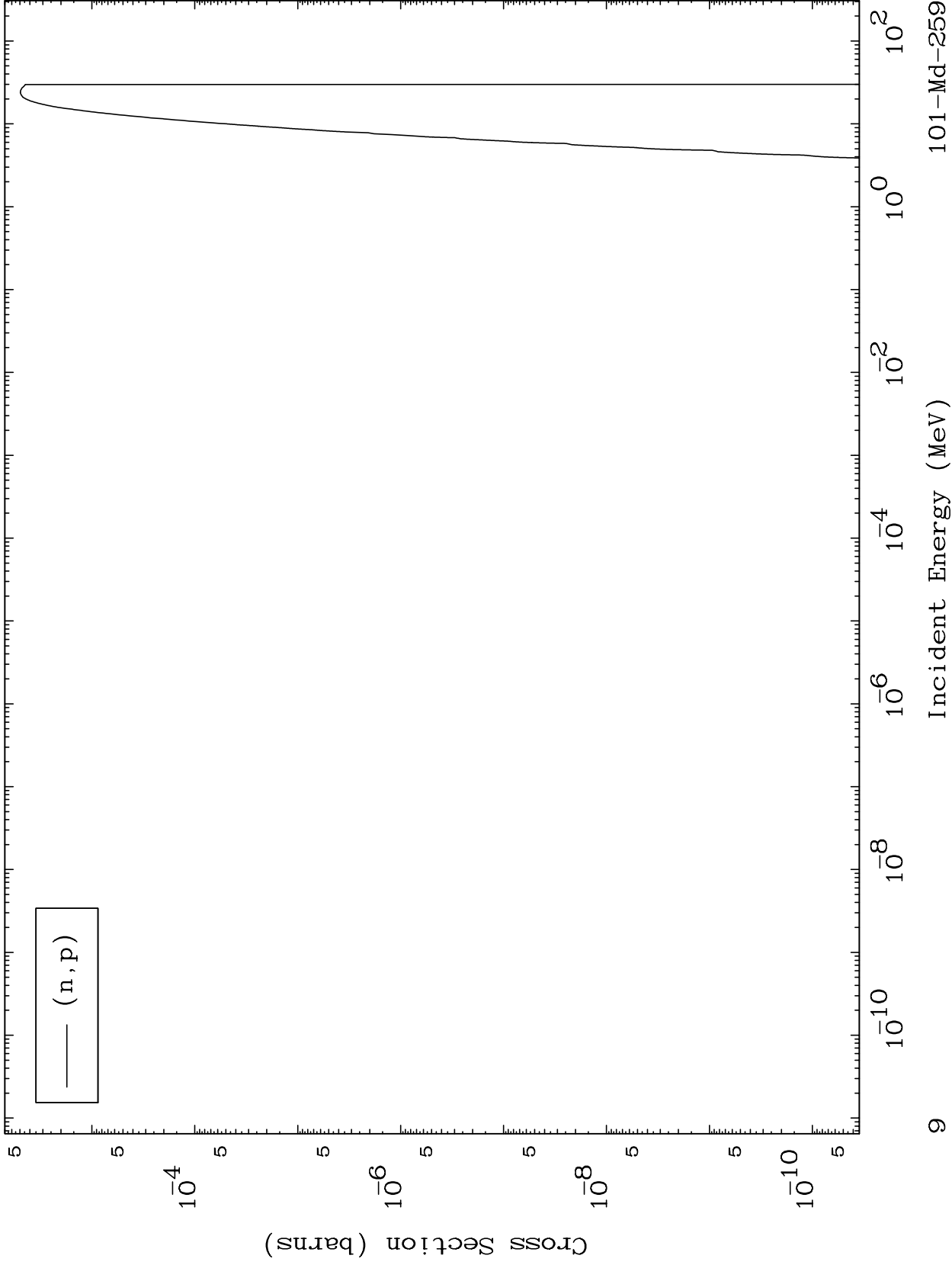




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(n,p) Levels  
293 Kelvin Cross Sections

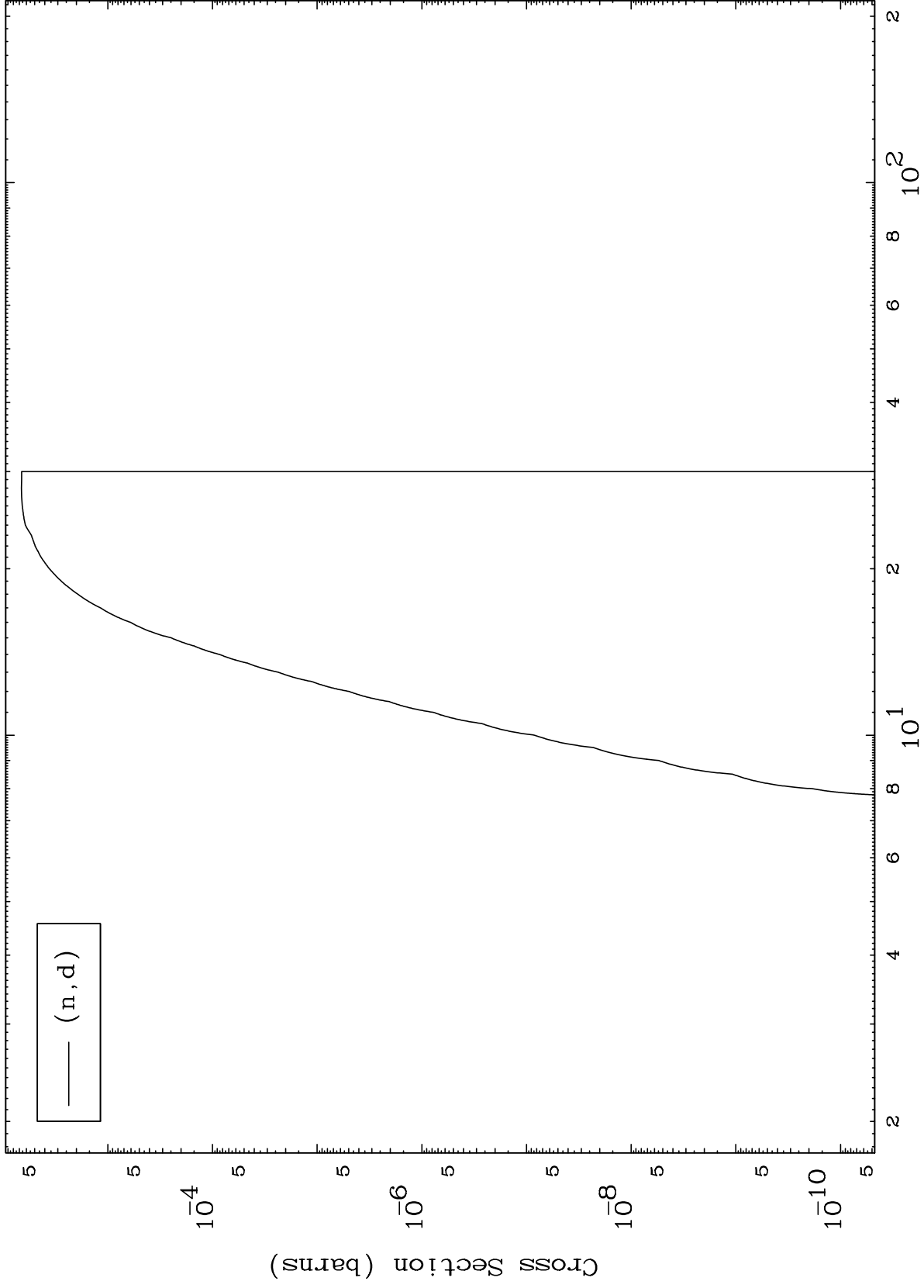
101-Md-259



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(n,d) Levels  
293 Kelvin Cross Sections

101-Md-259



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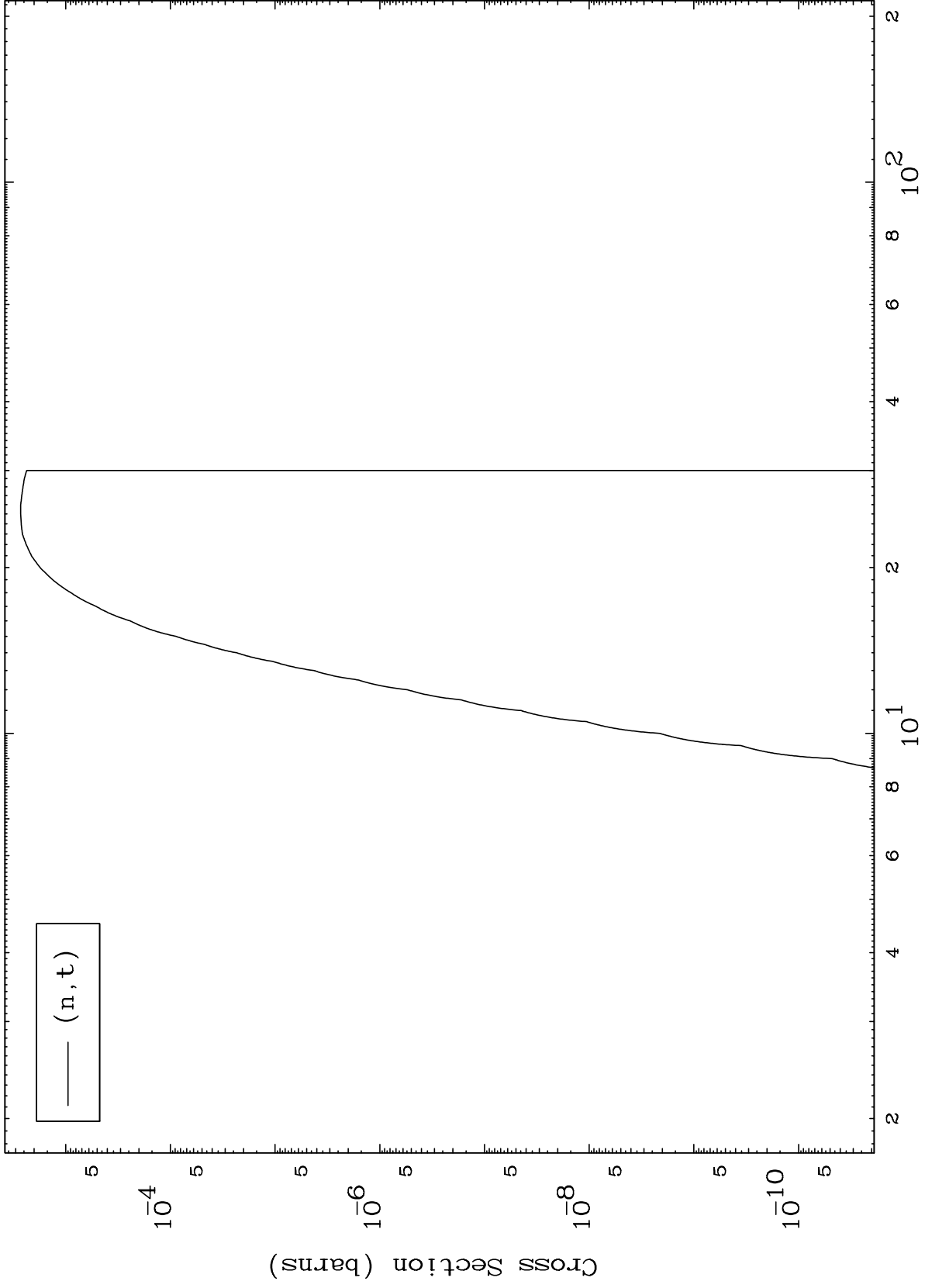
Incident Energy (MeV)

101-Md-259

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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-259



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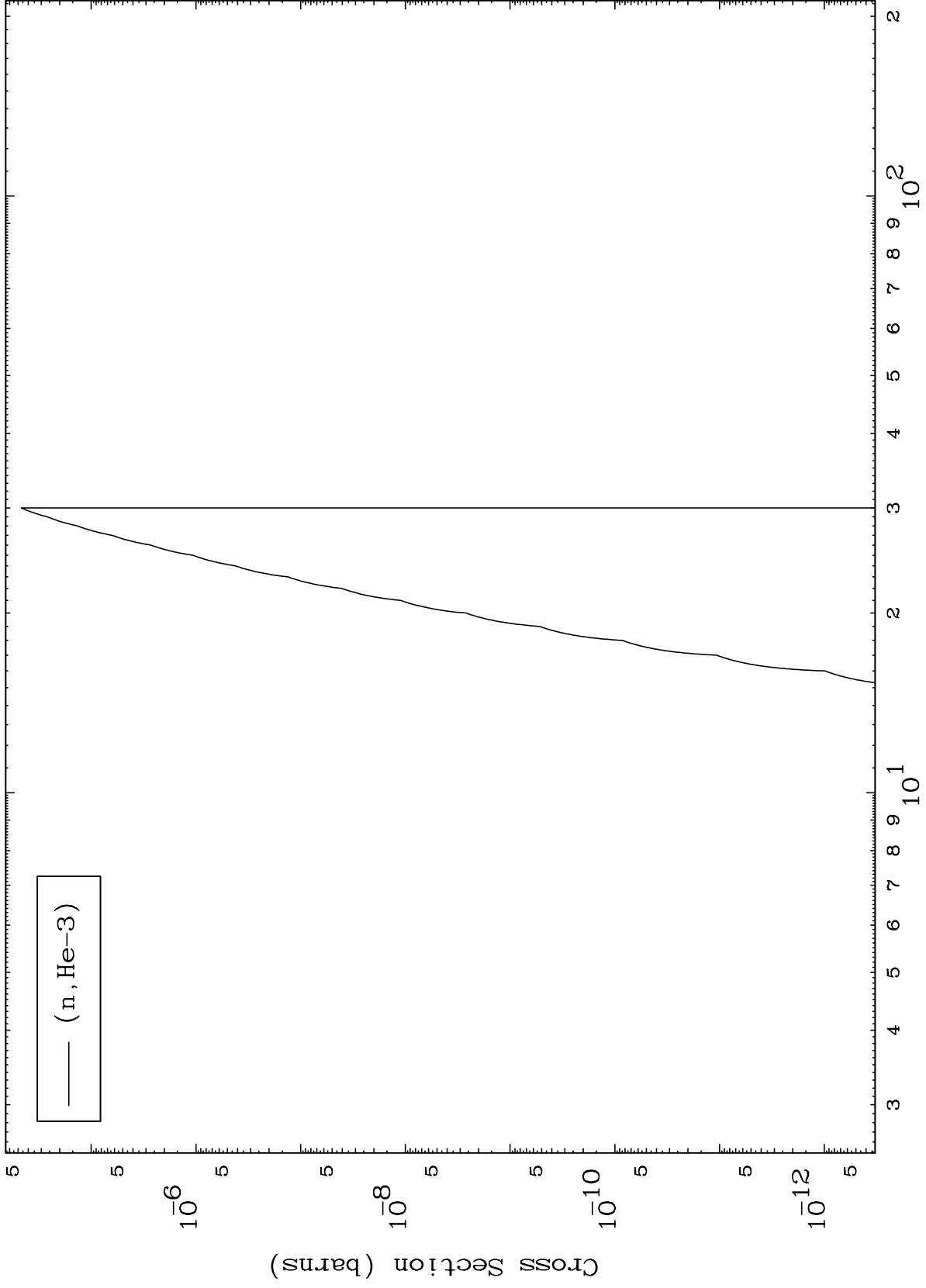
Incident Energy (MeV)

101-Md-259

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(n,He3) Levels  
293 Kelvin Cross Sections

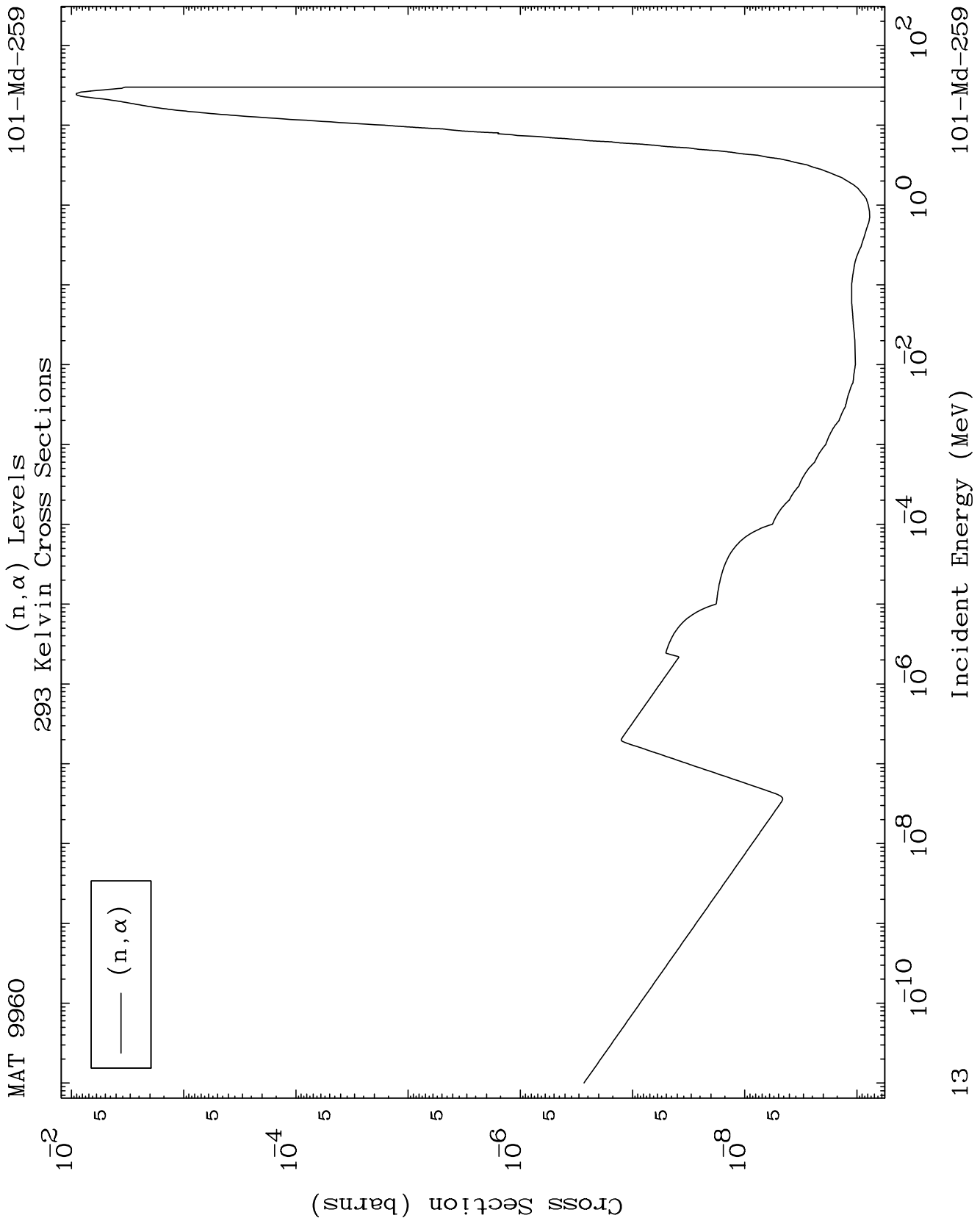
101-Md-259



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Incident Energy (MeV)

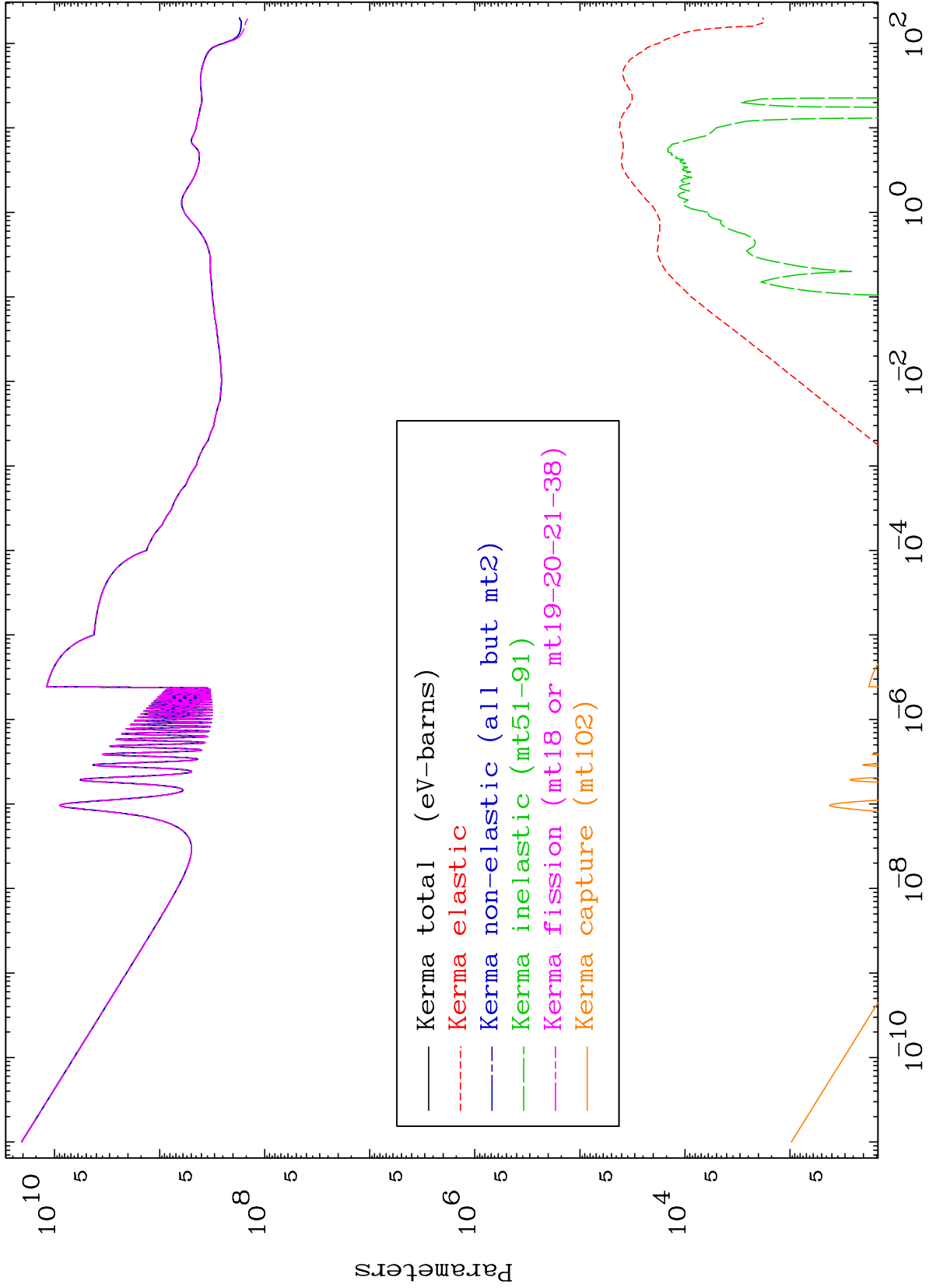
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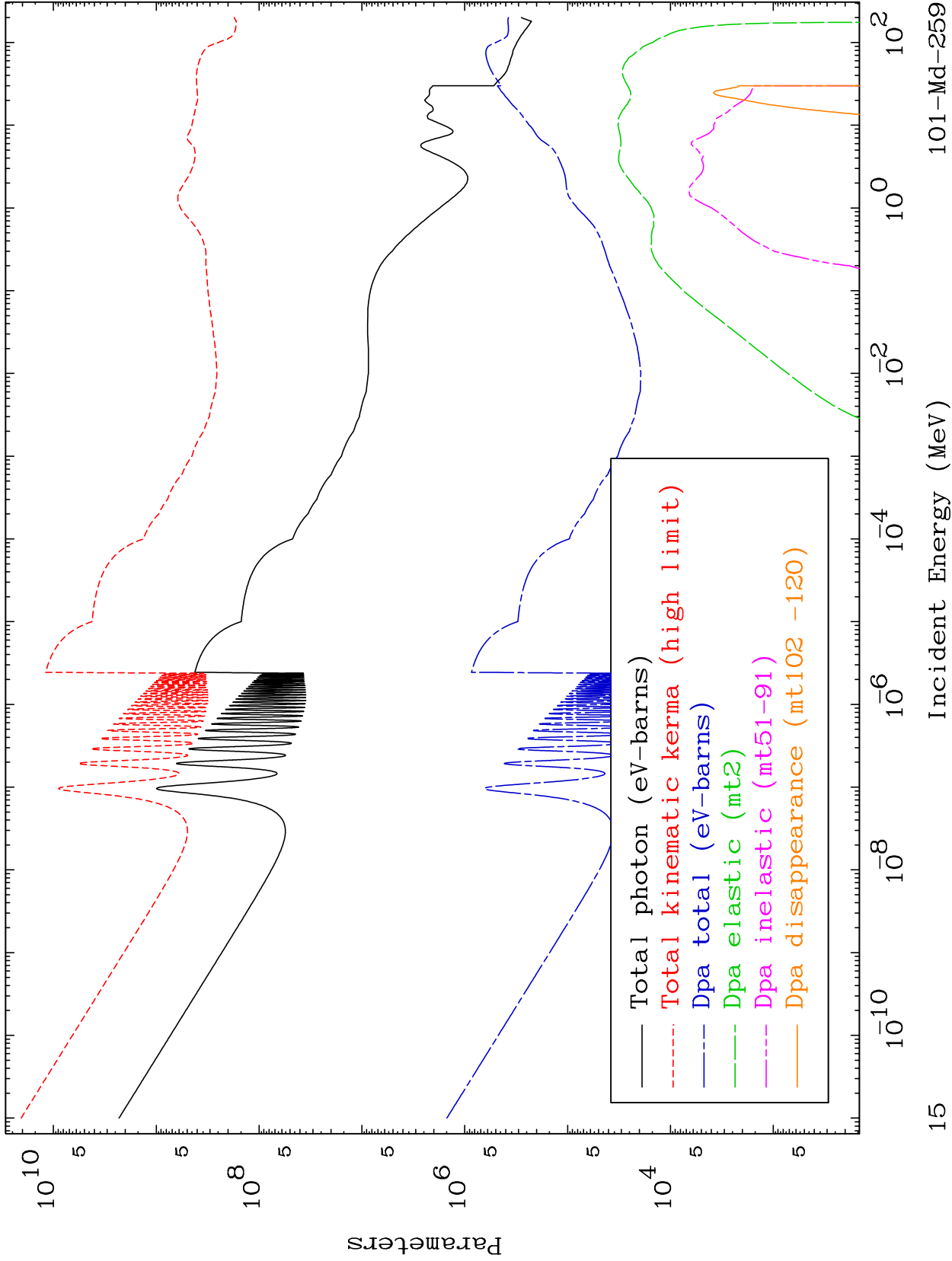


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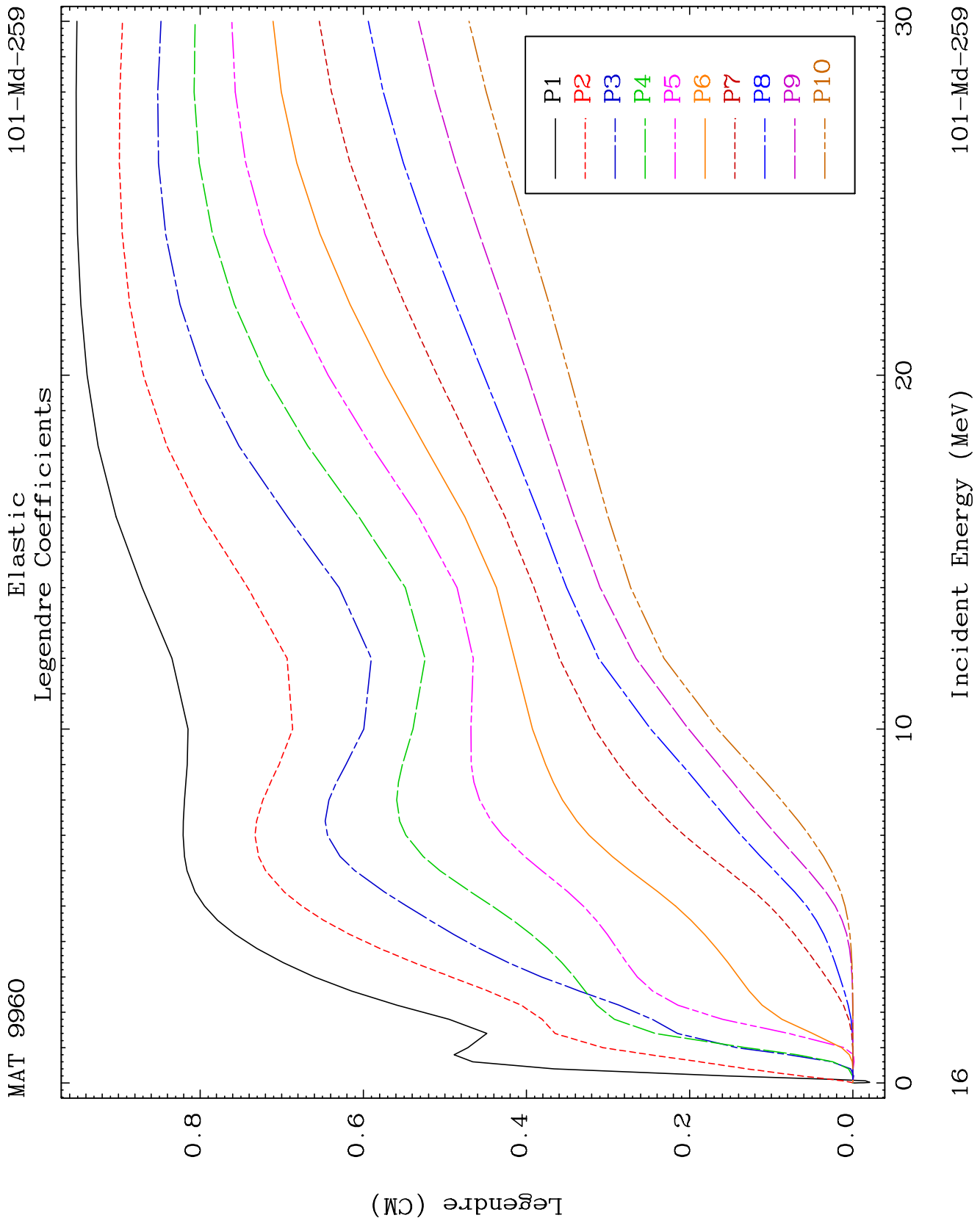
Energy Release  
Parameters

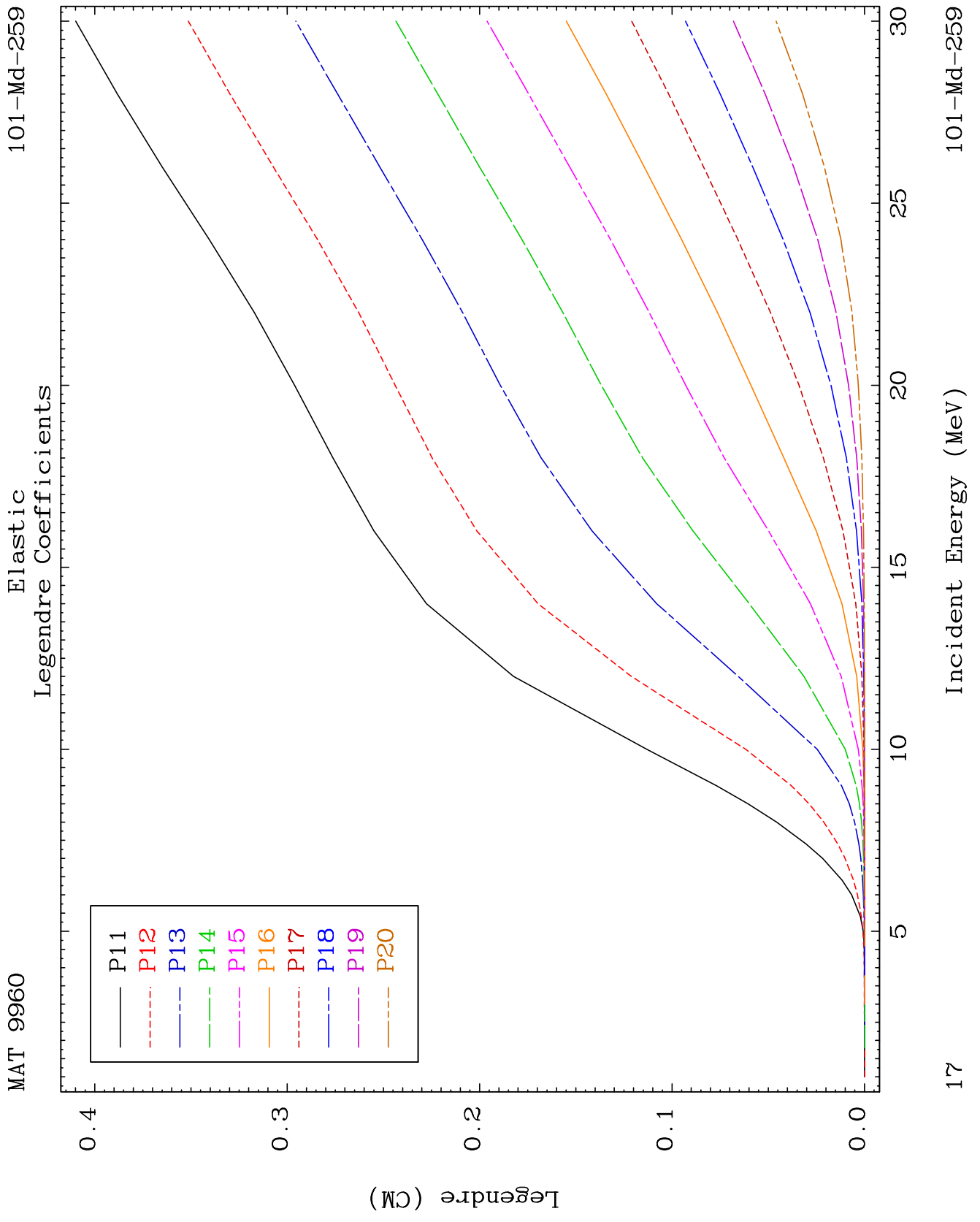
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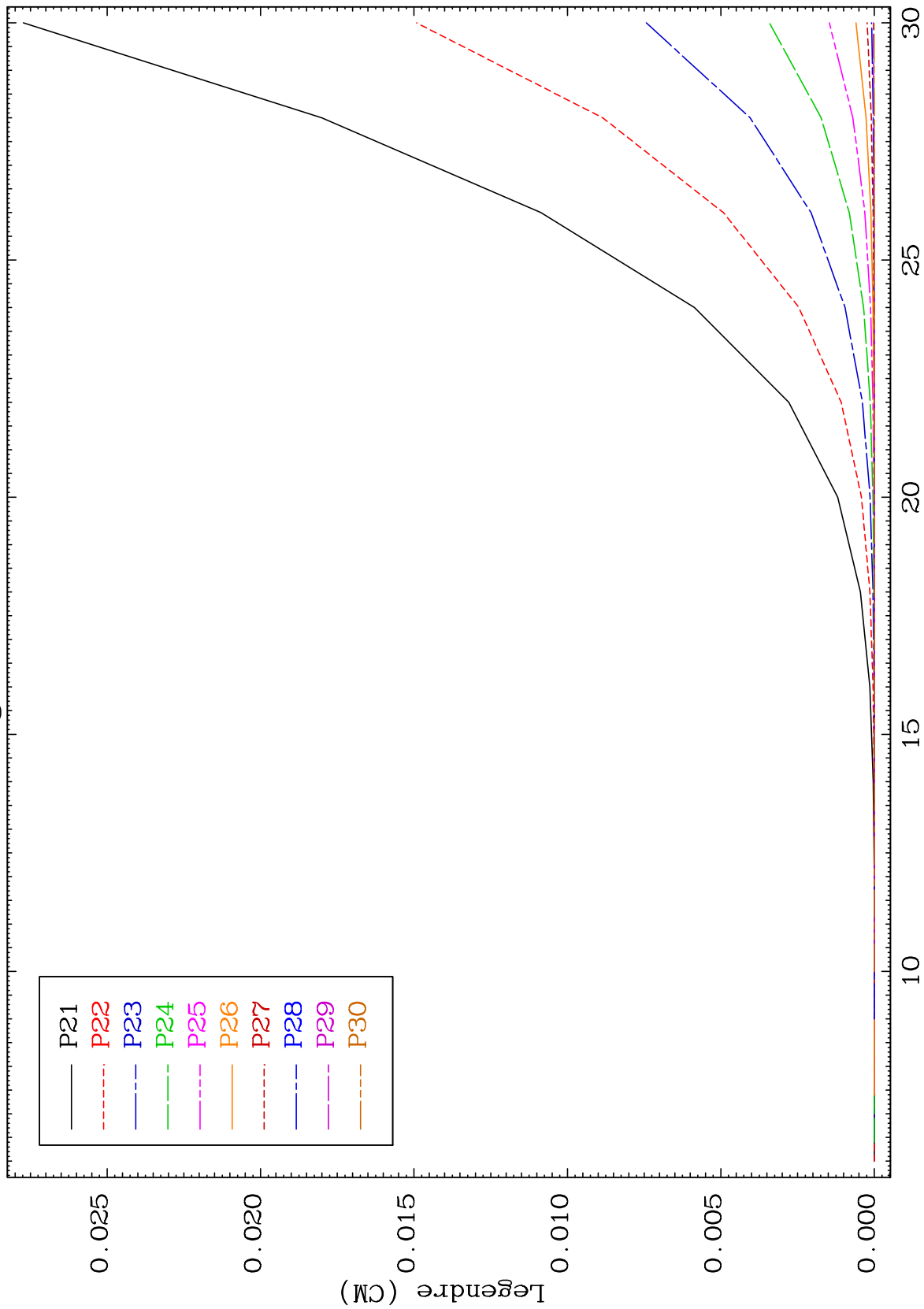




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### Elastic Legendre Coefficients

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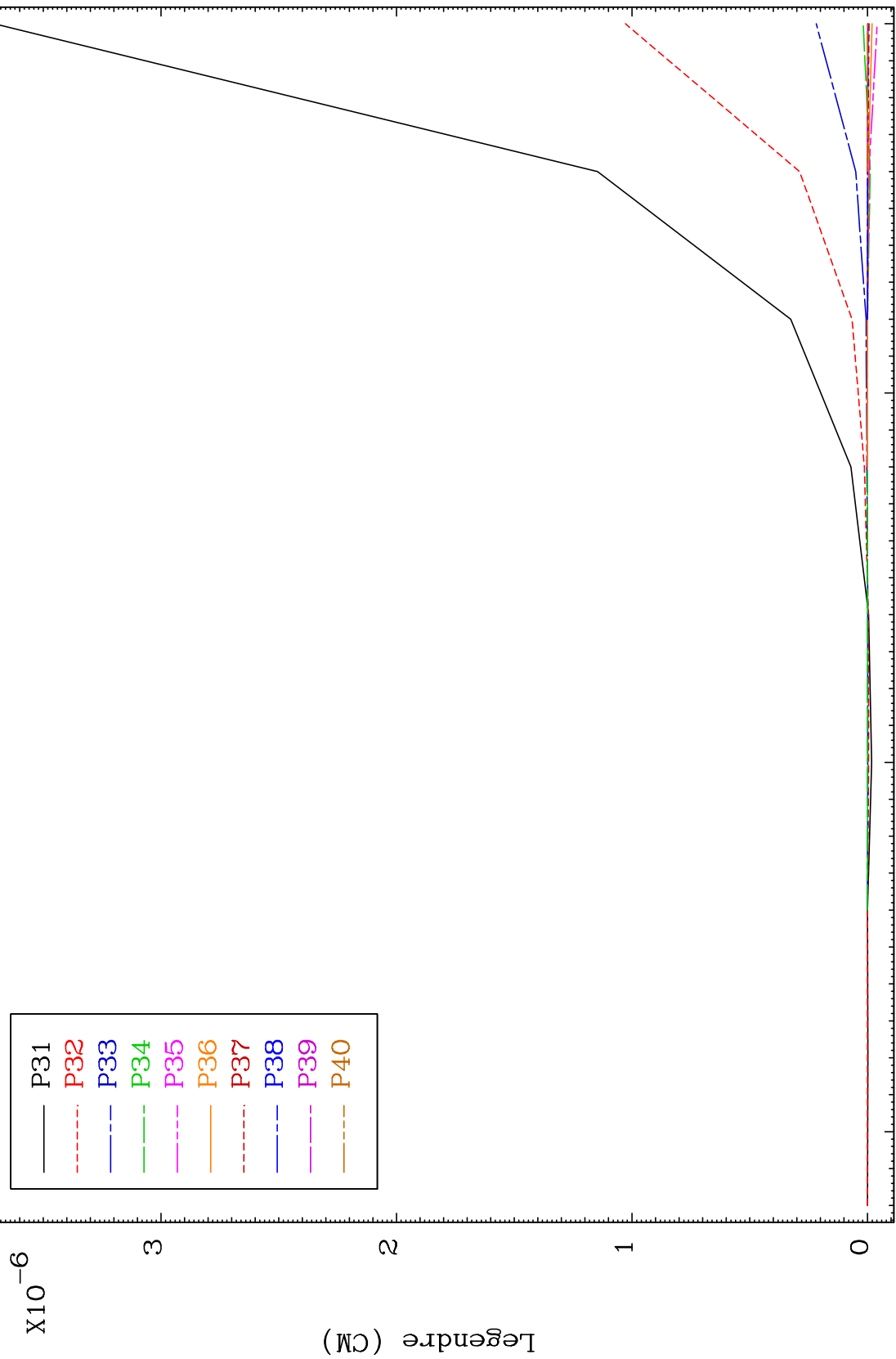
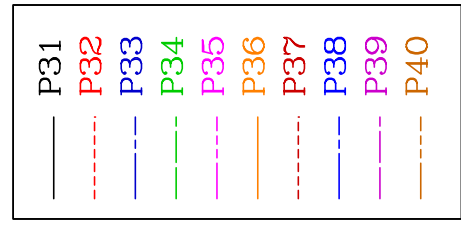
Incident Energy (MeV)

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Elastic Legendre Coefficients

101-Md-259



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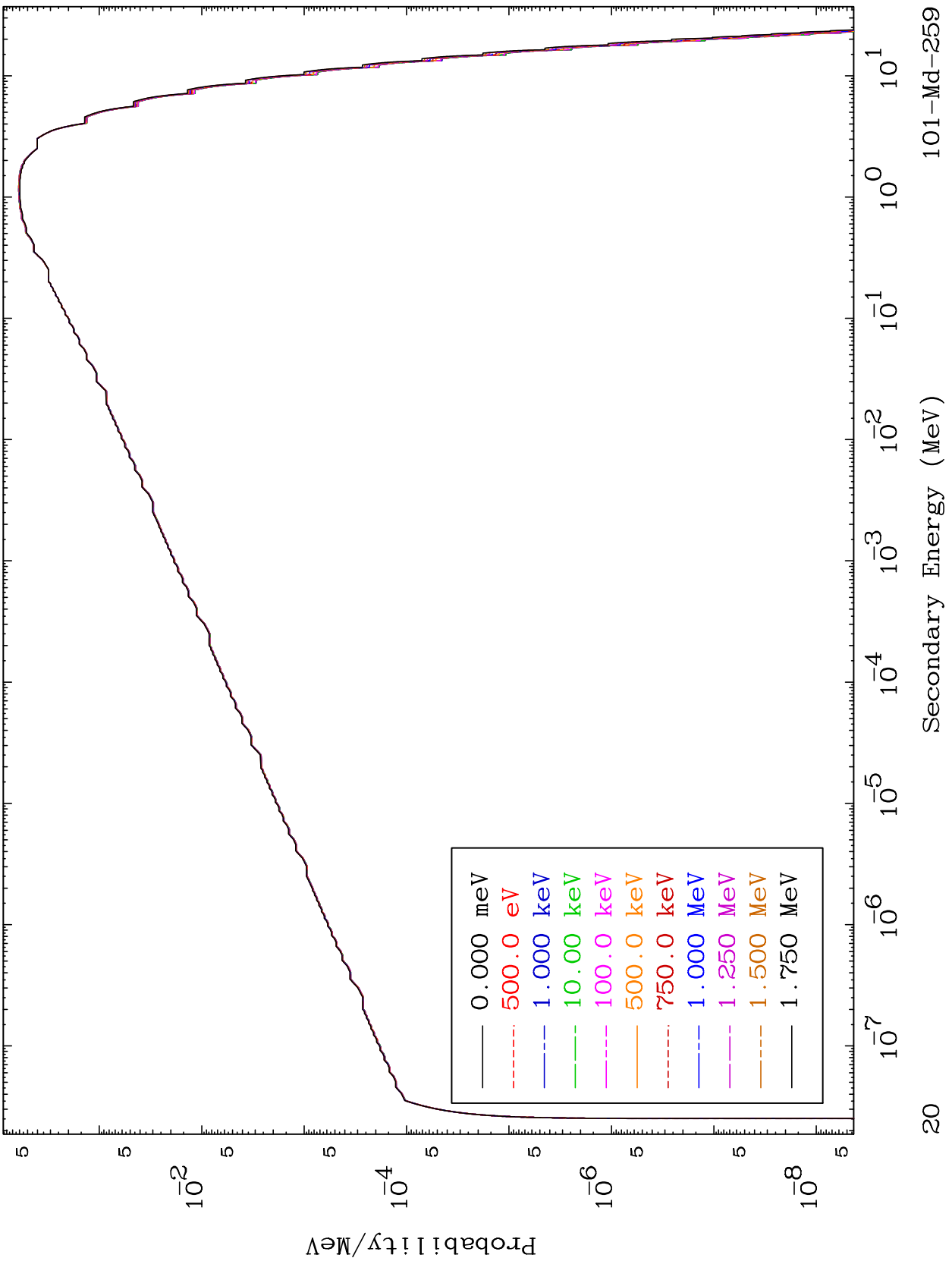
Incident Energy (MeV)

101-Md-259

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### Fission Energy Distribution

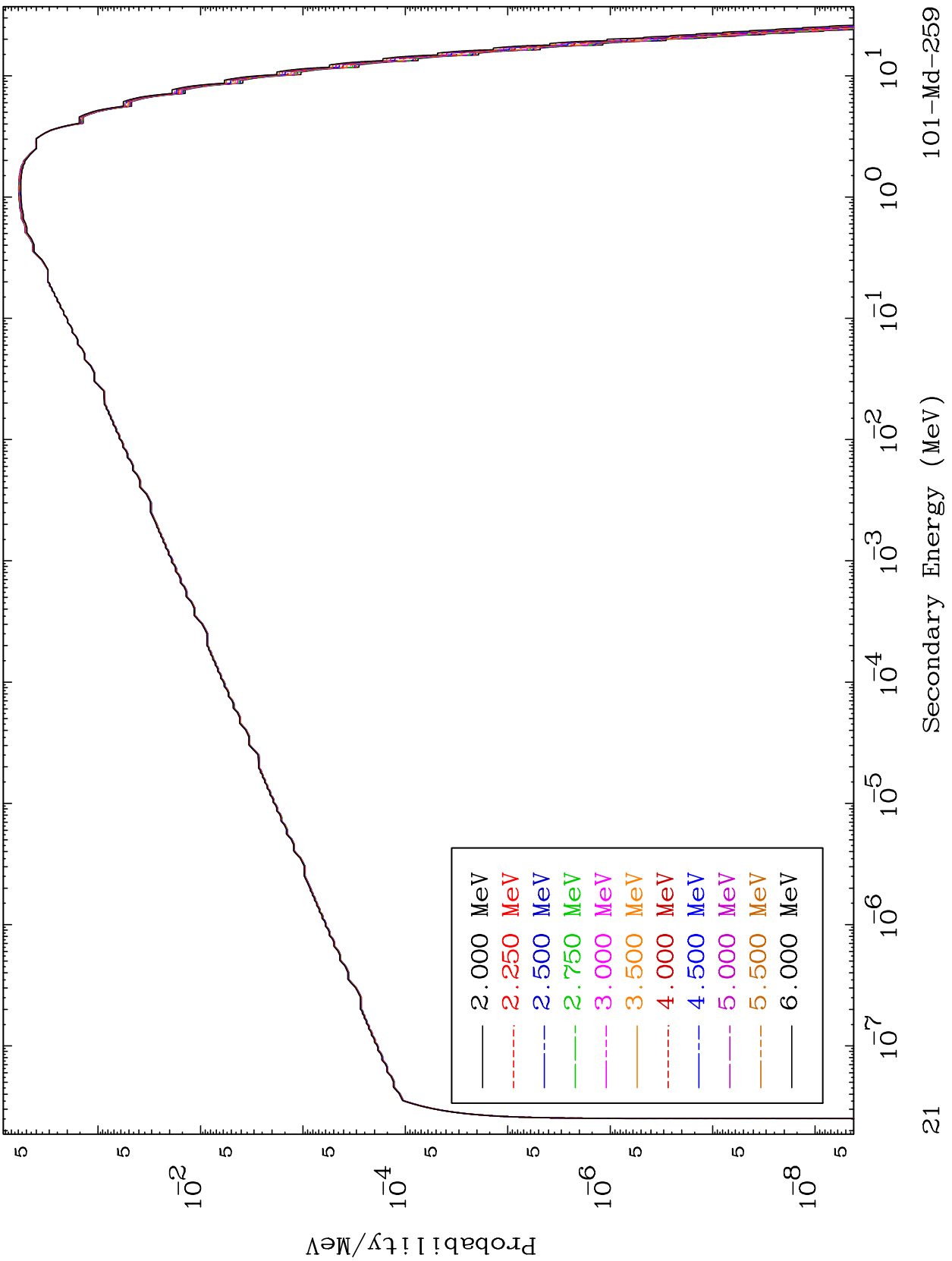
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### Fission Energy Distribution

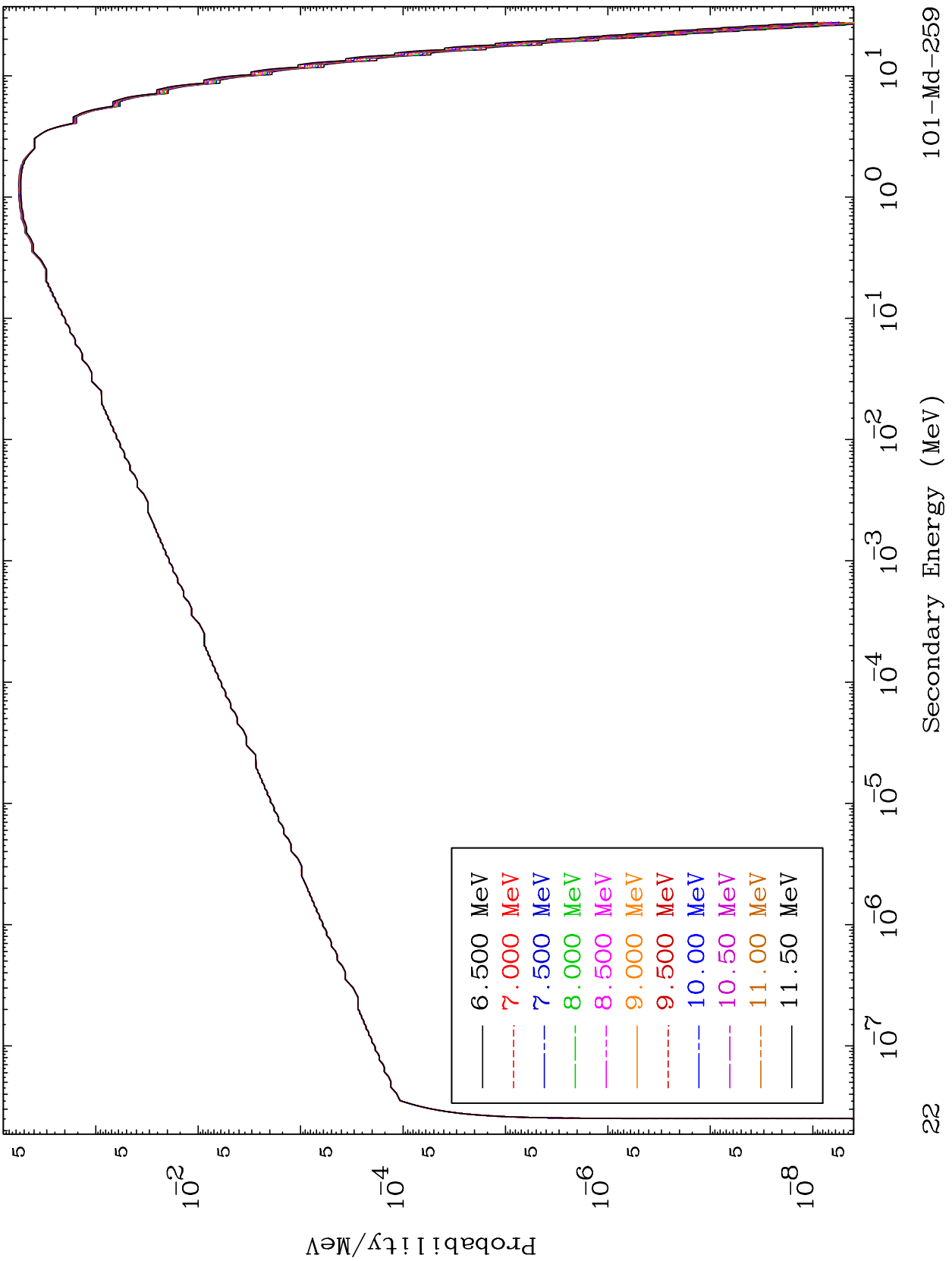
101-Md-259



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### Fission Energy Distribution

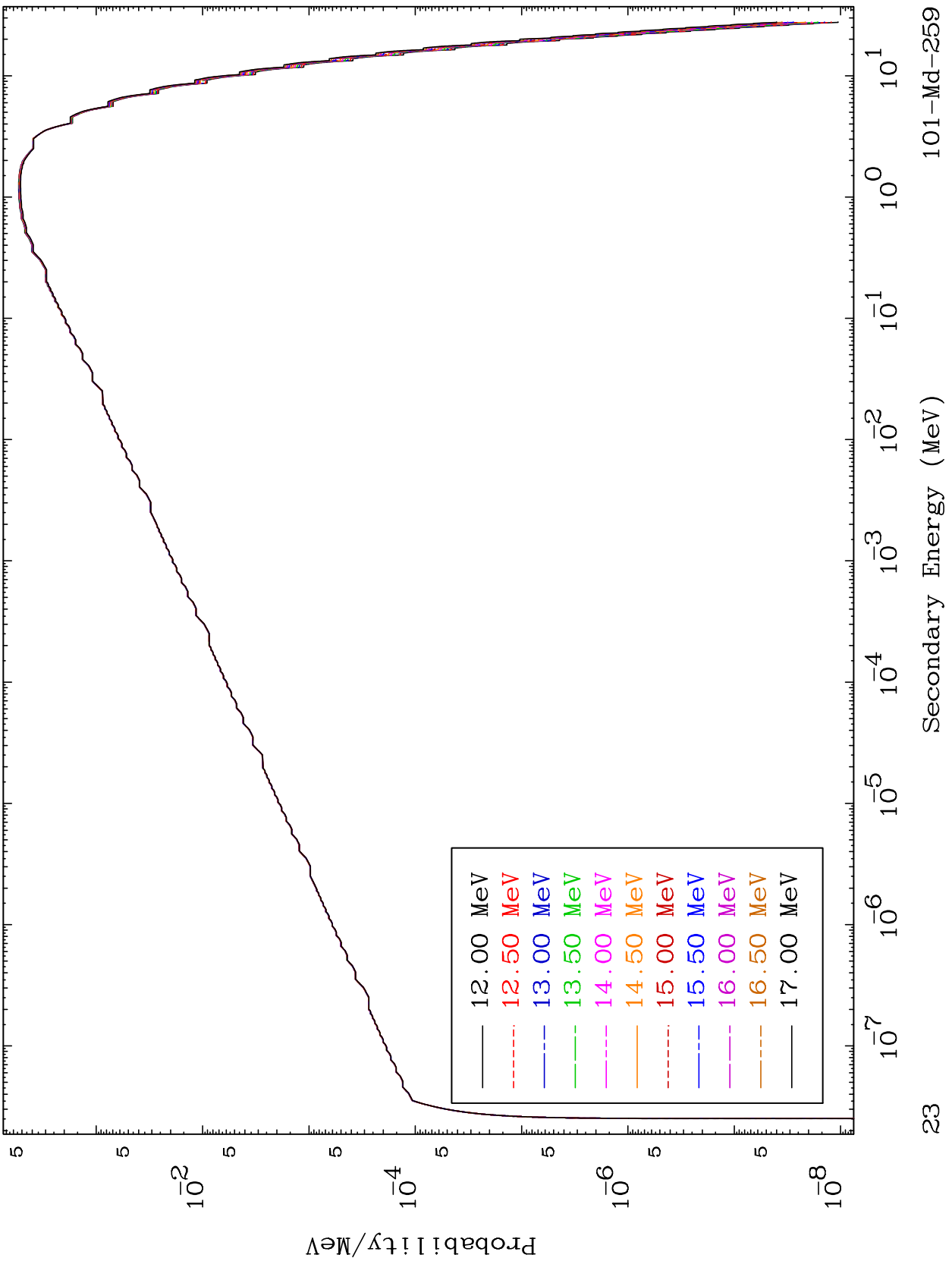
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### Fission Energy Distribution

101-Md-259

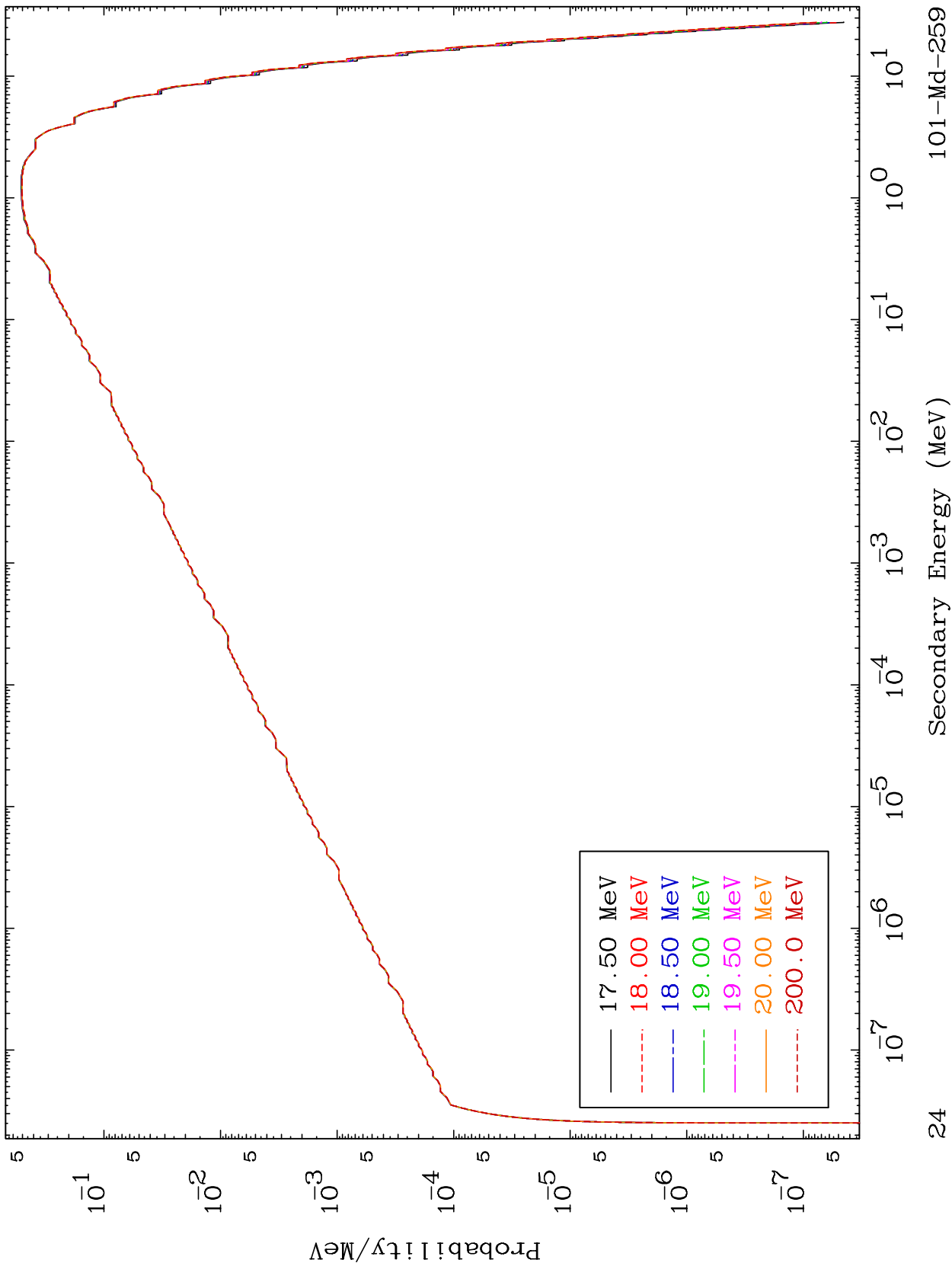




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### Fission Energy Distribution

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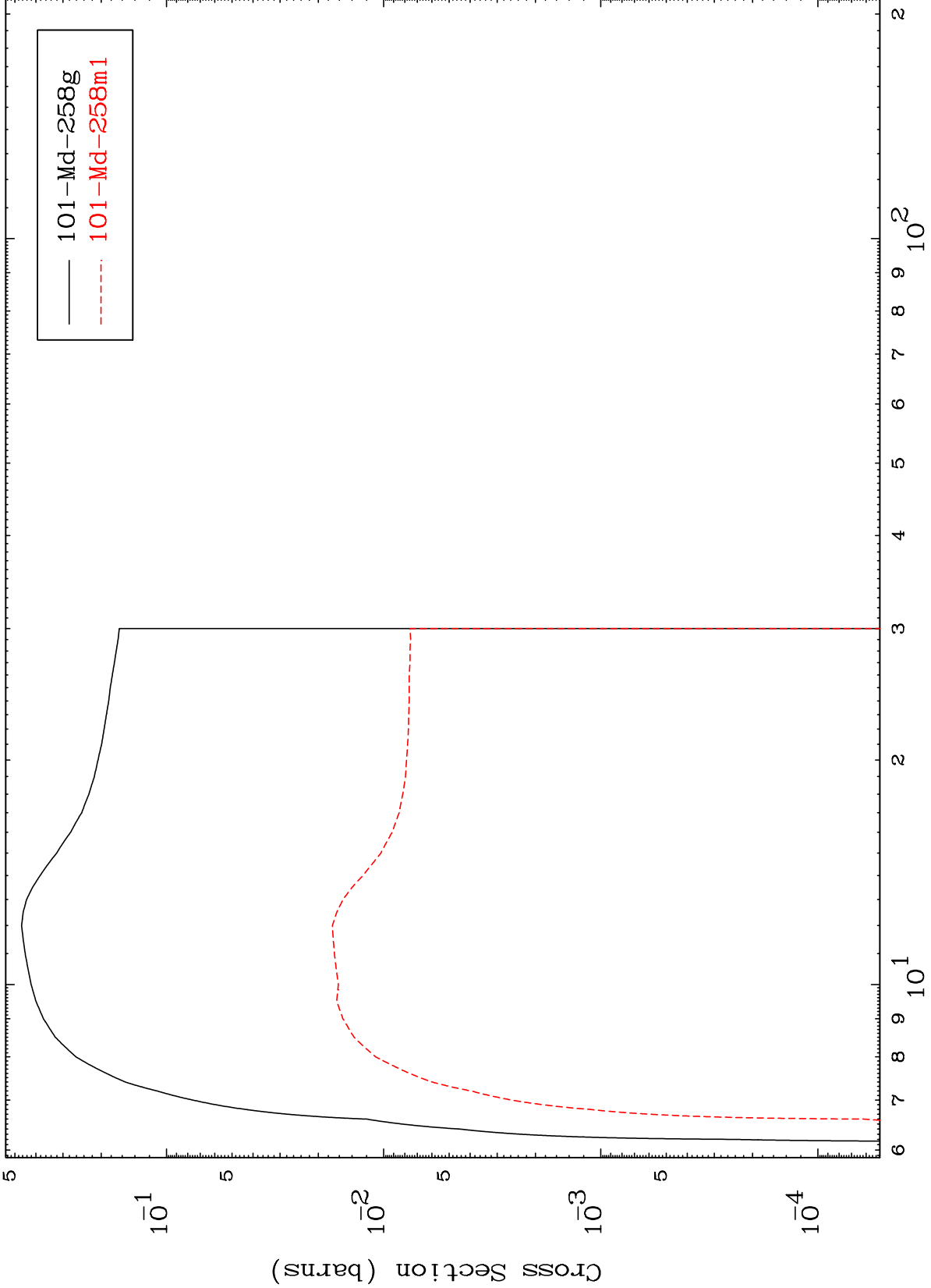


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(n,2n)

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Radionuclide Production Cross Section



Incident Energy (MeV)

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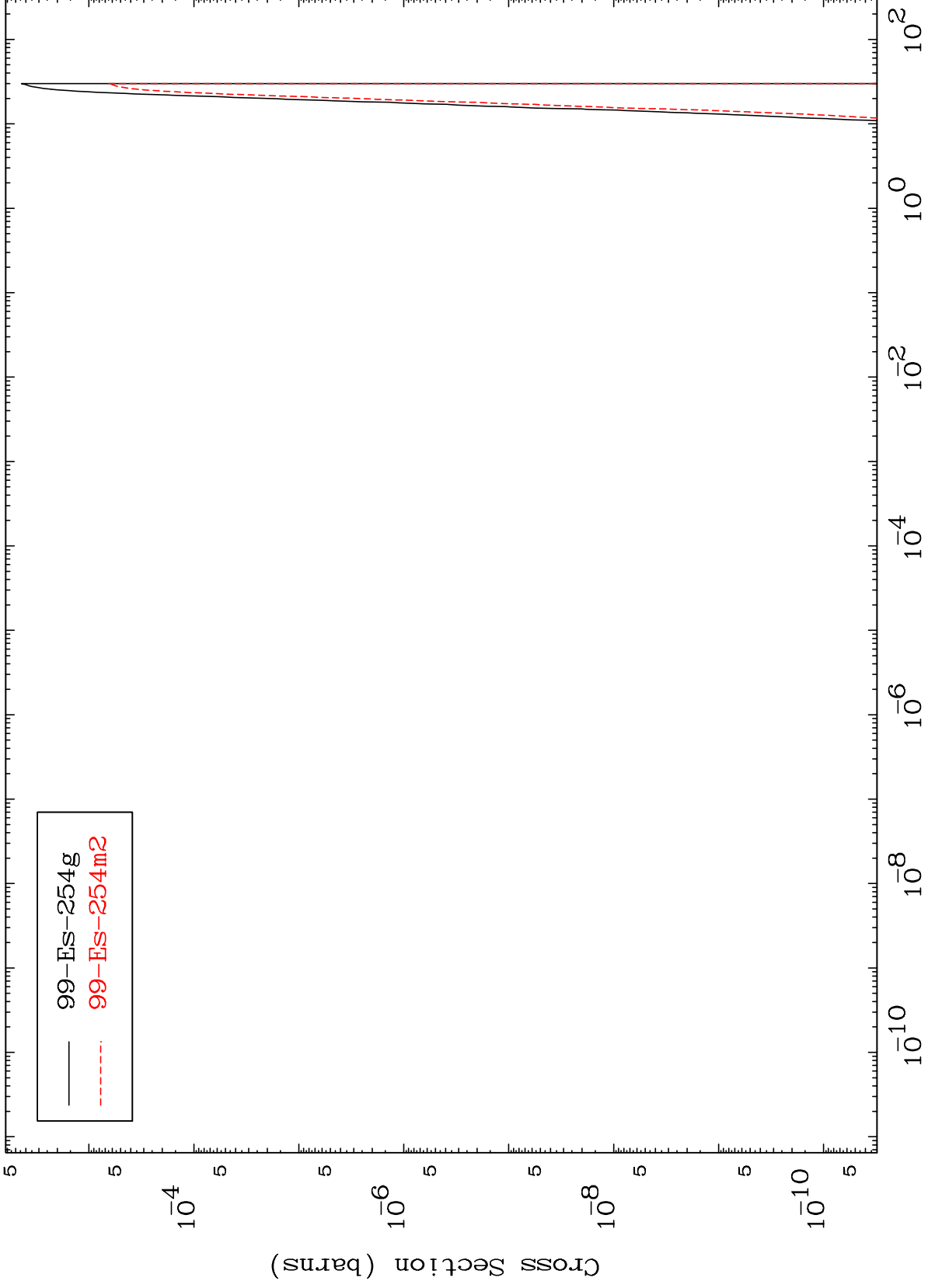
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(n,2n)  $\alpha$

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Radionuclide Production Cross Section

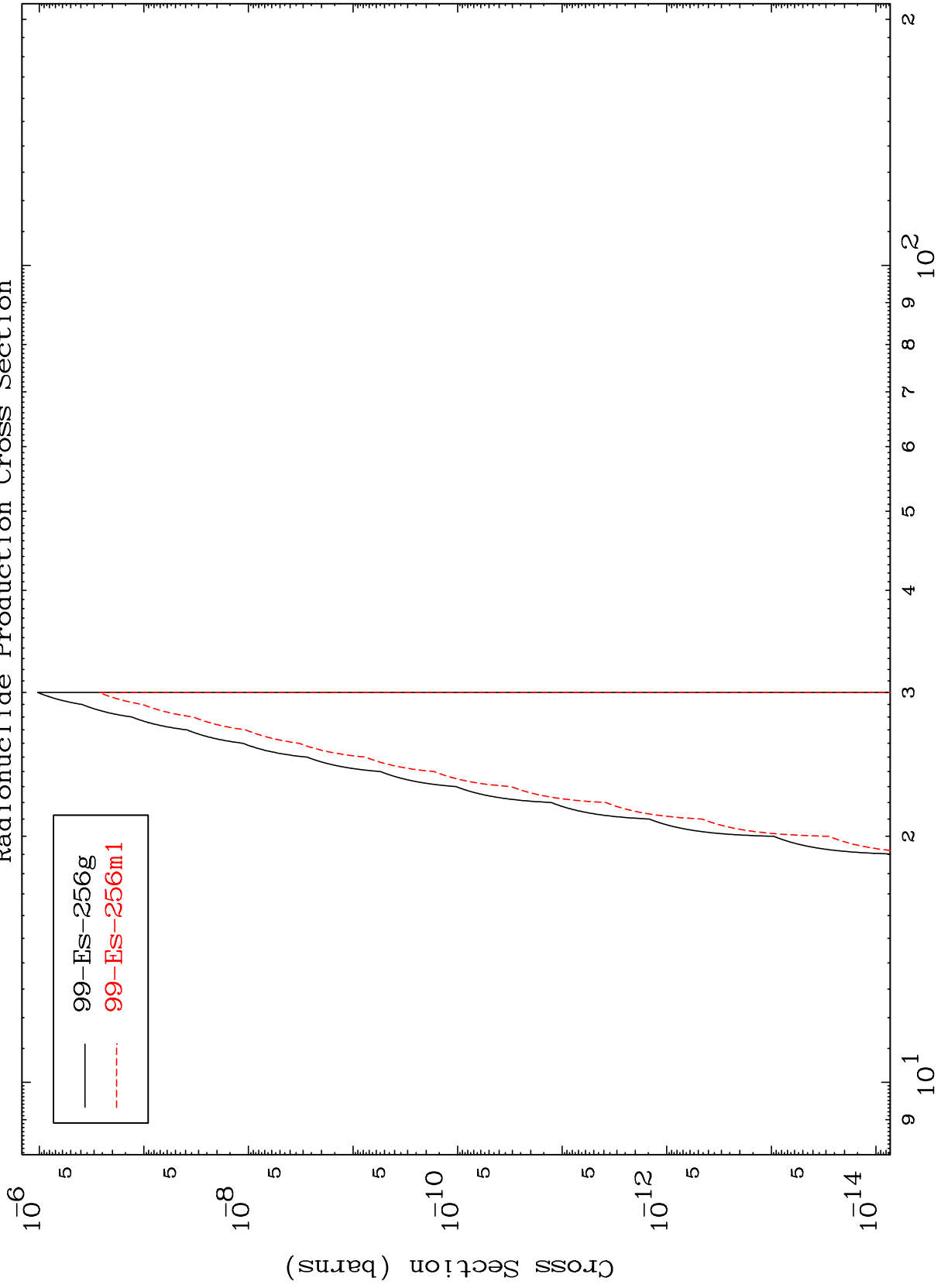


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(n,n') He-3

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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101-Md-259

Radionuclide Production Cross Section  
(n,  $\alpha$ )

