

Program EVALPLOT
(Version 2017-1)

by

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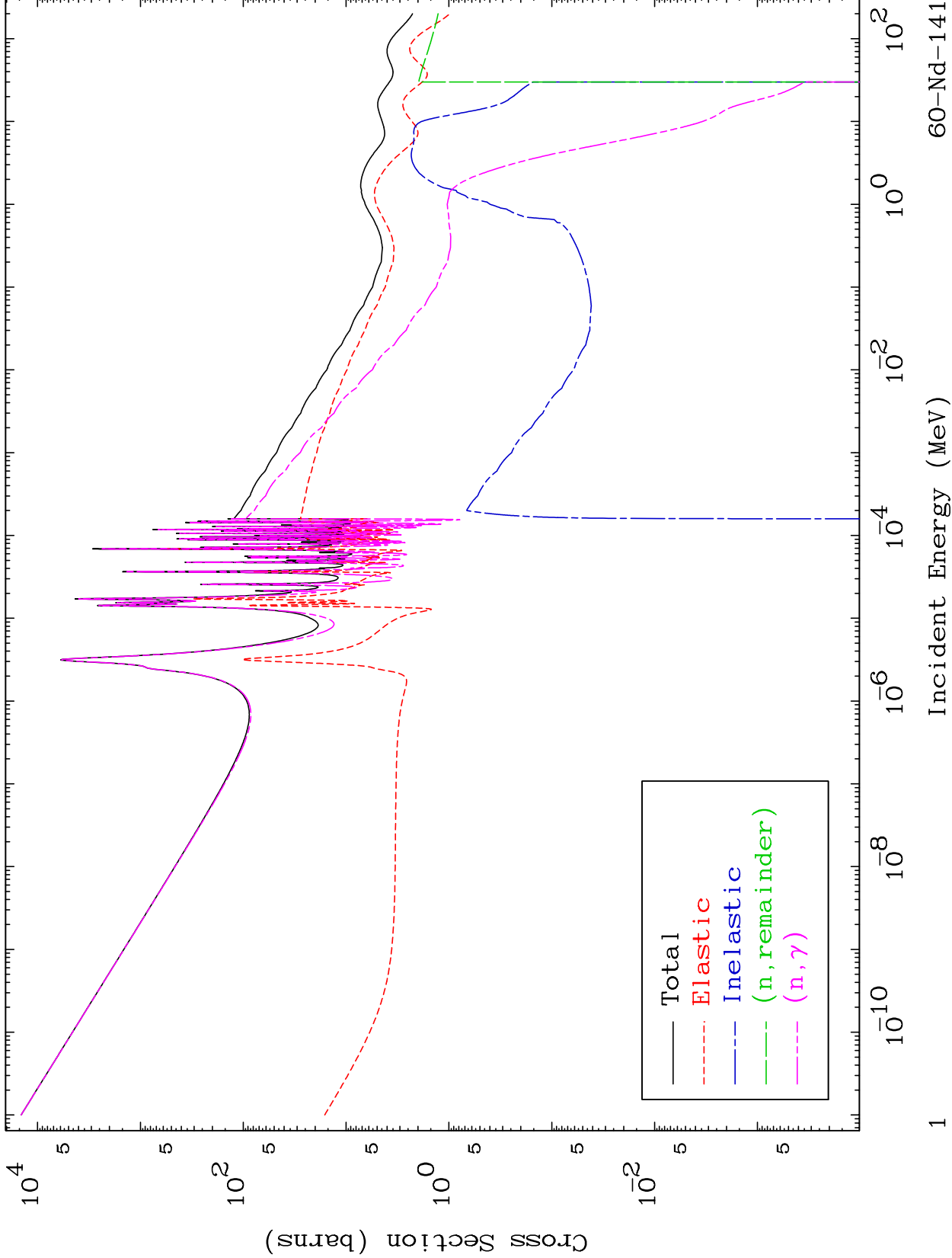
Press Mouse Button to Start

MAT 6023

Major

293 Kelvin Cross Sections

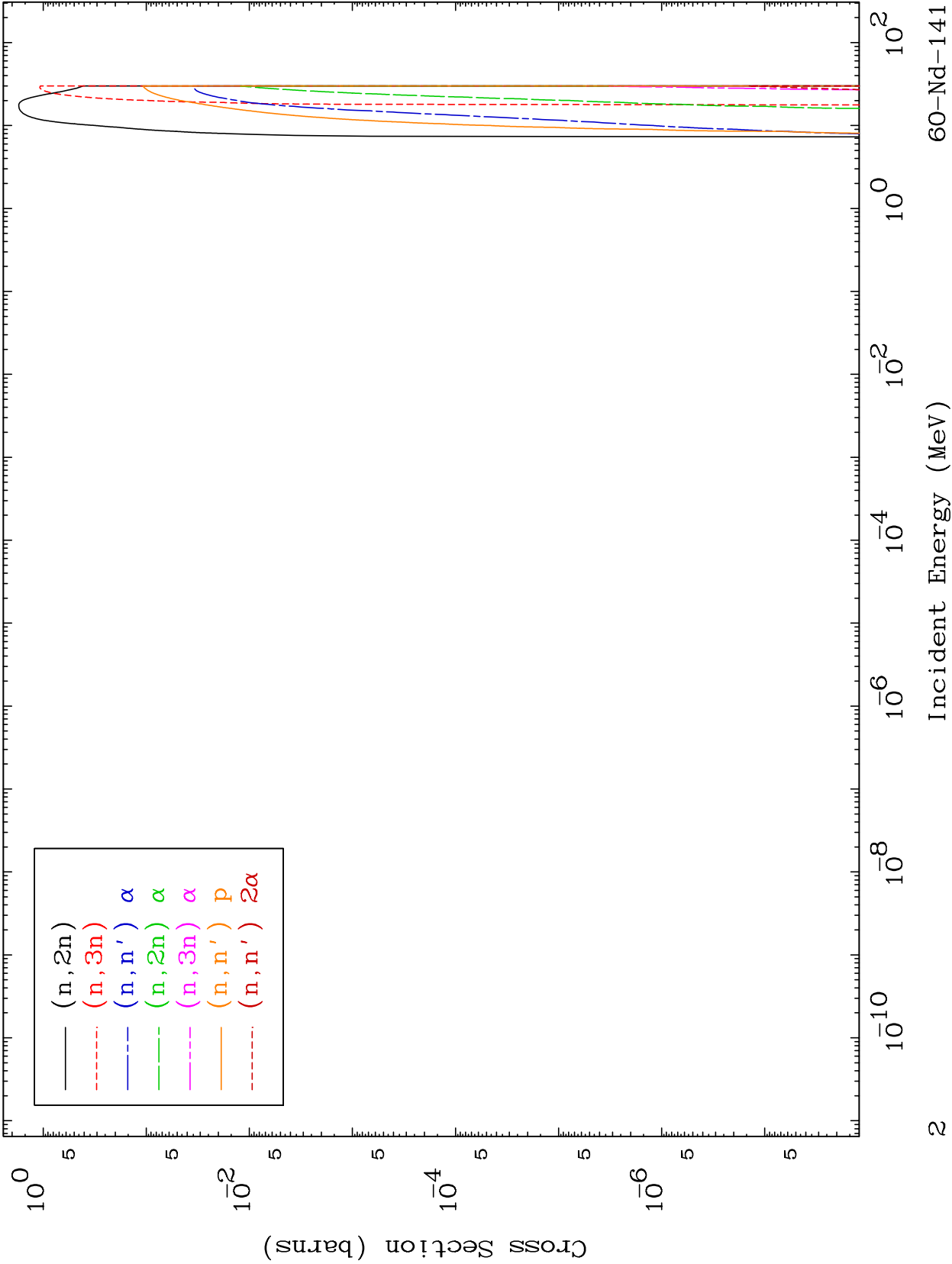
60-Nd-141



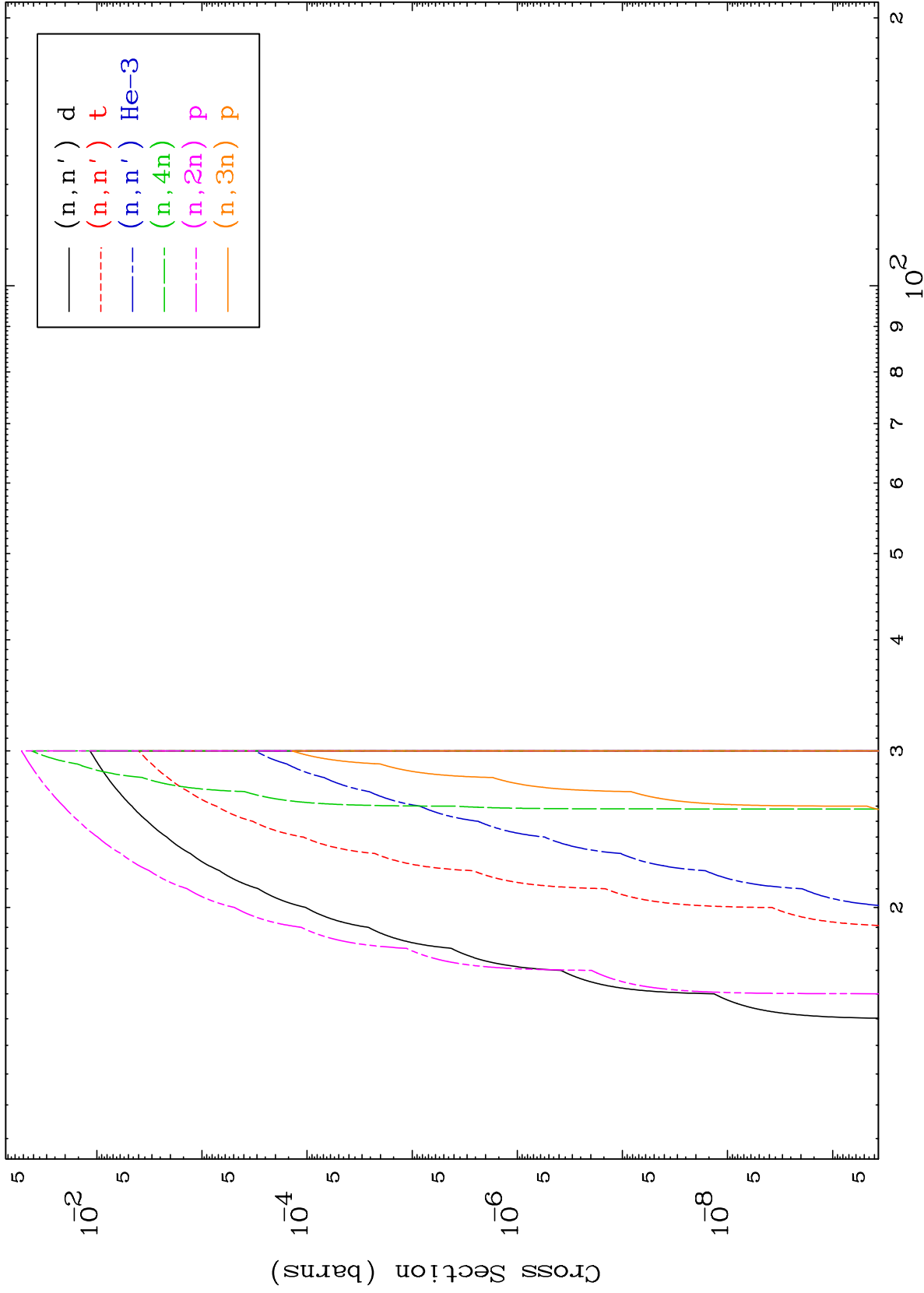
MAT 6023

Neutron Production
293 Kelvin Cross Sections

60-Nd-141



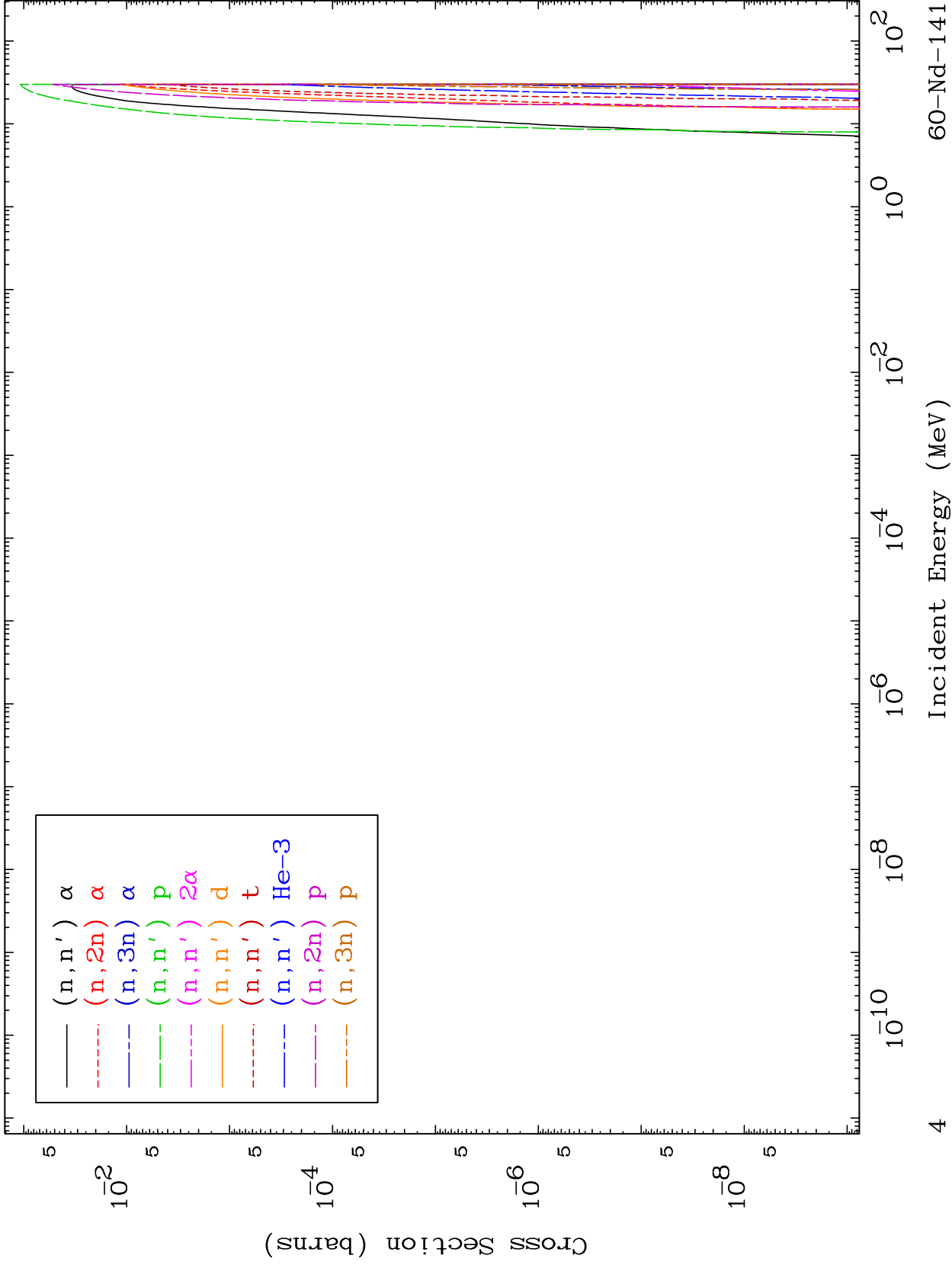
Neutron Production
293 Kelvin Cross Sections



MAT 6023

Charged Particle
293 Kelvin Cross Sections

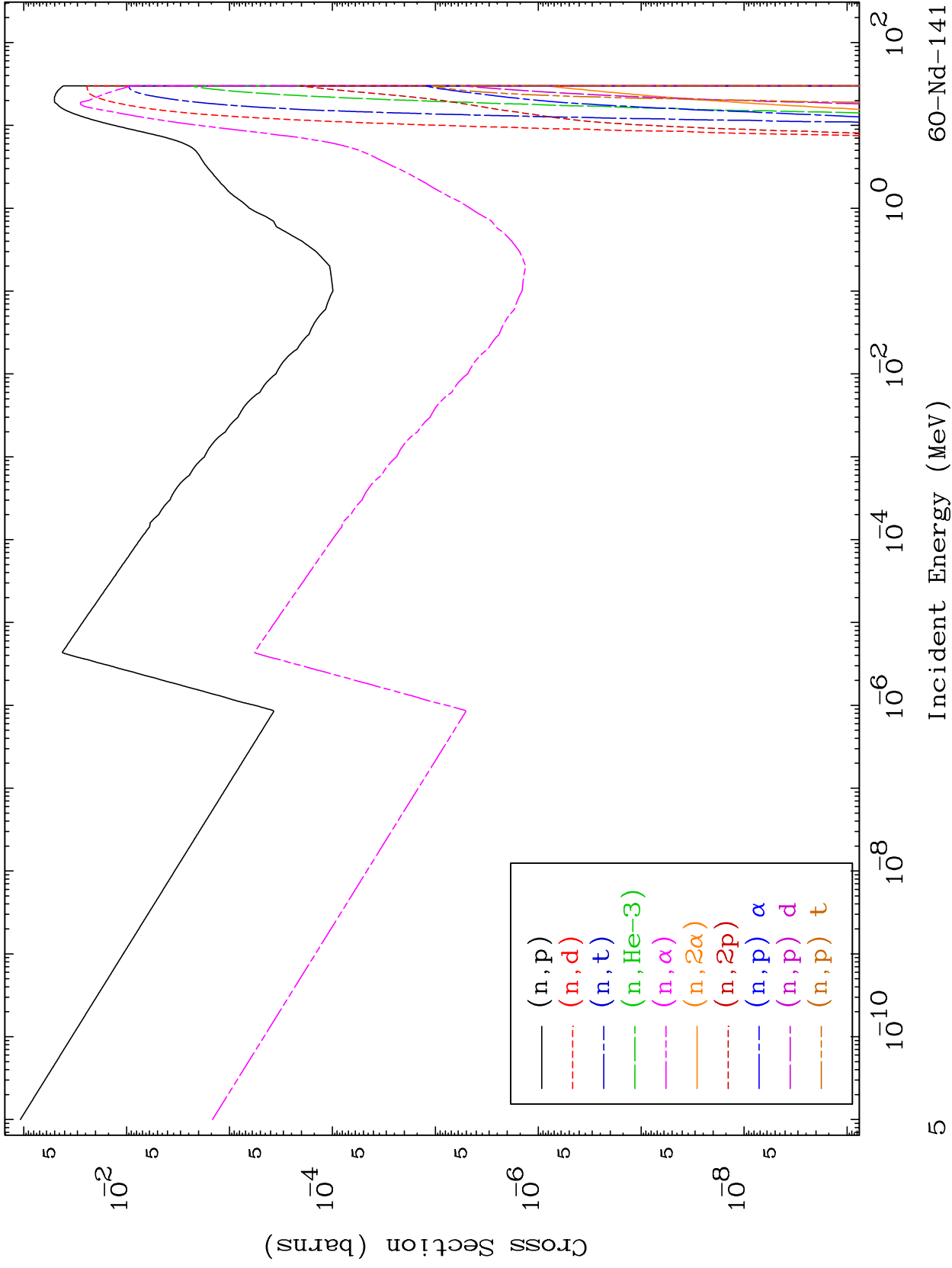
60-Nd-141

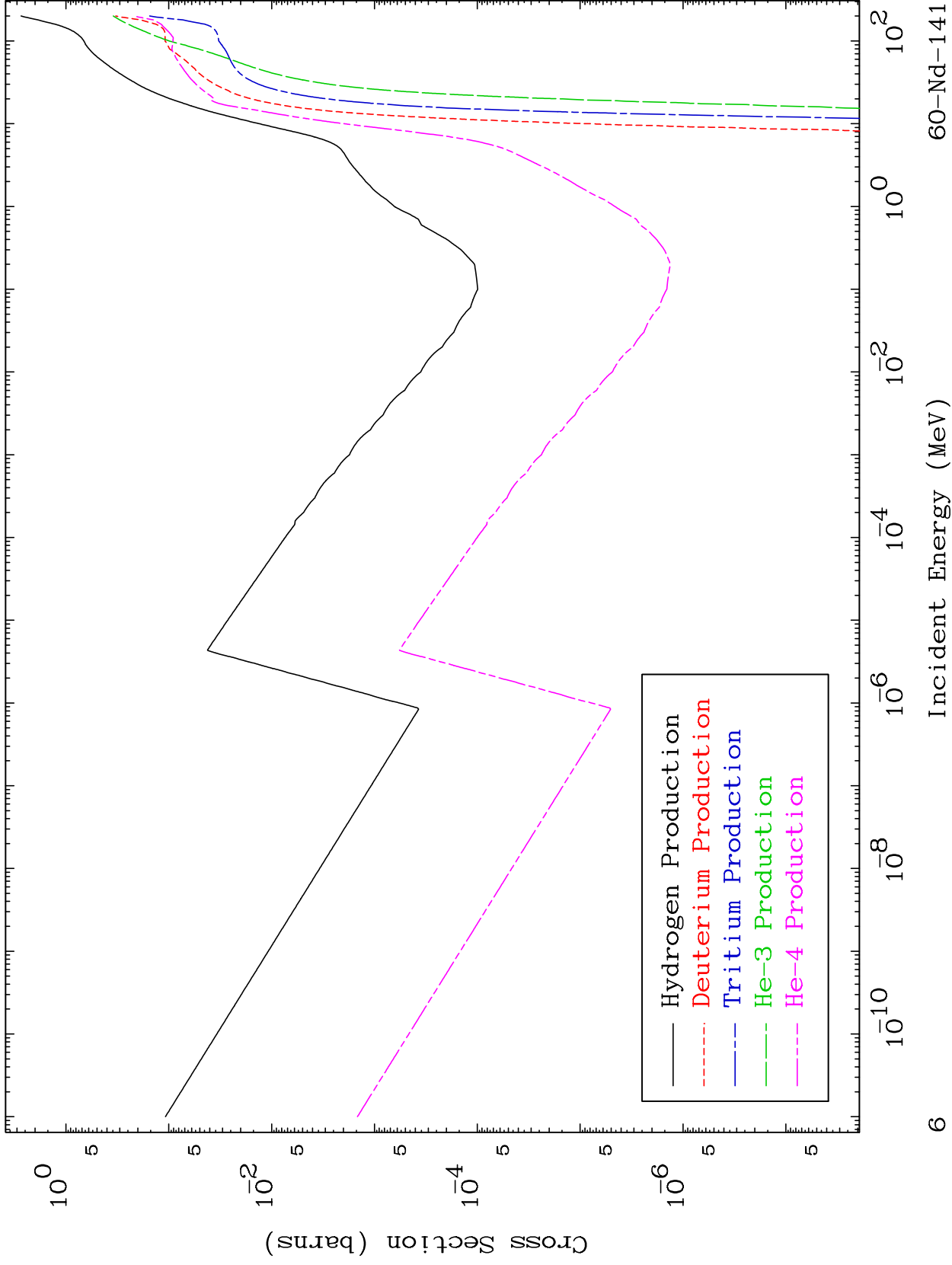


MAT 6023

Charged Particle
293 Kelvin Cross Sections

60-Nd-141

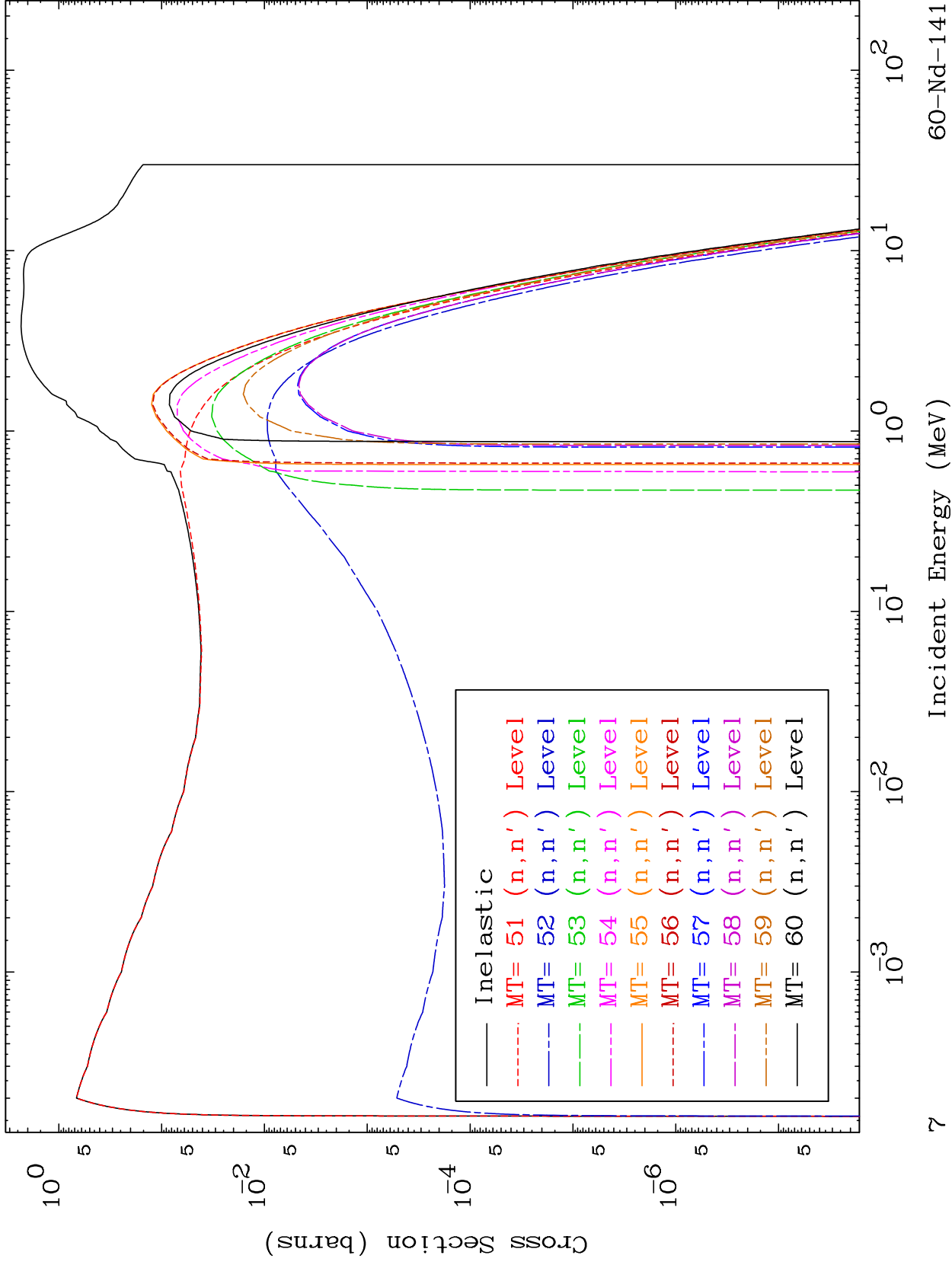




MAT 6023

(n,n') Level
293 Kelvin Cross Sections

60-Nd-141

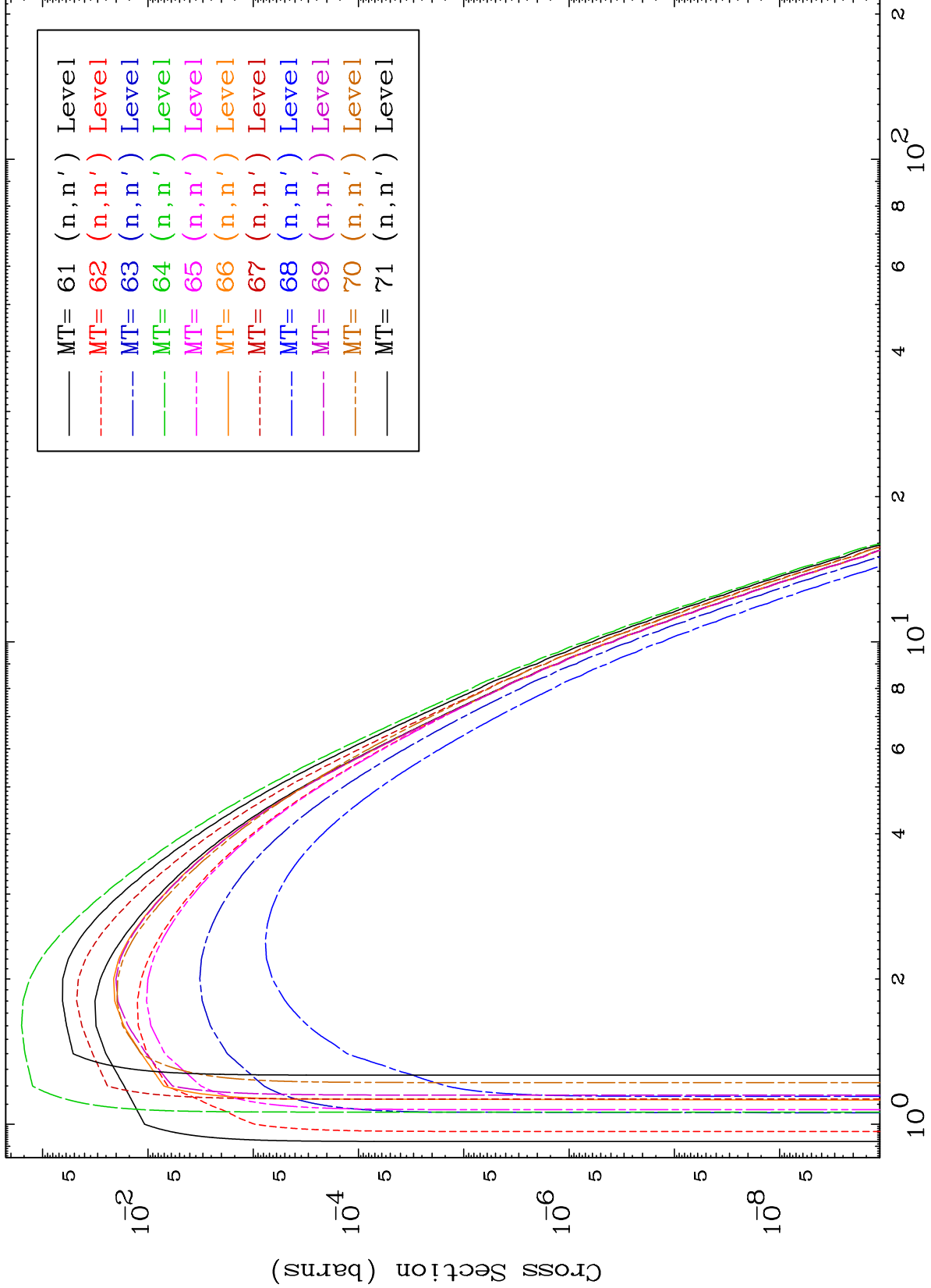


MAT 6023

(n,n') Level

60-Nd-141

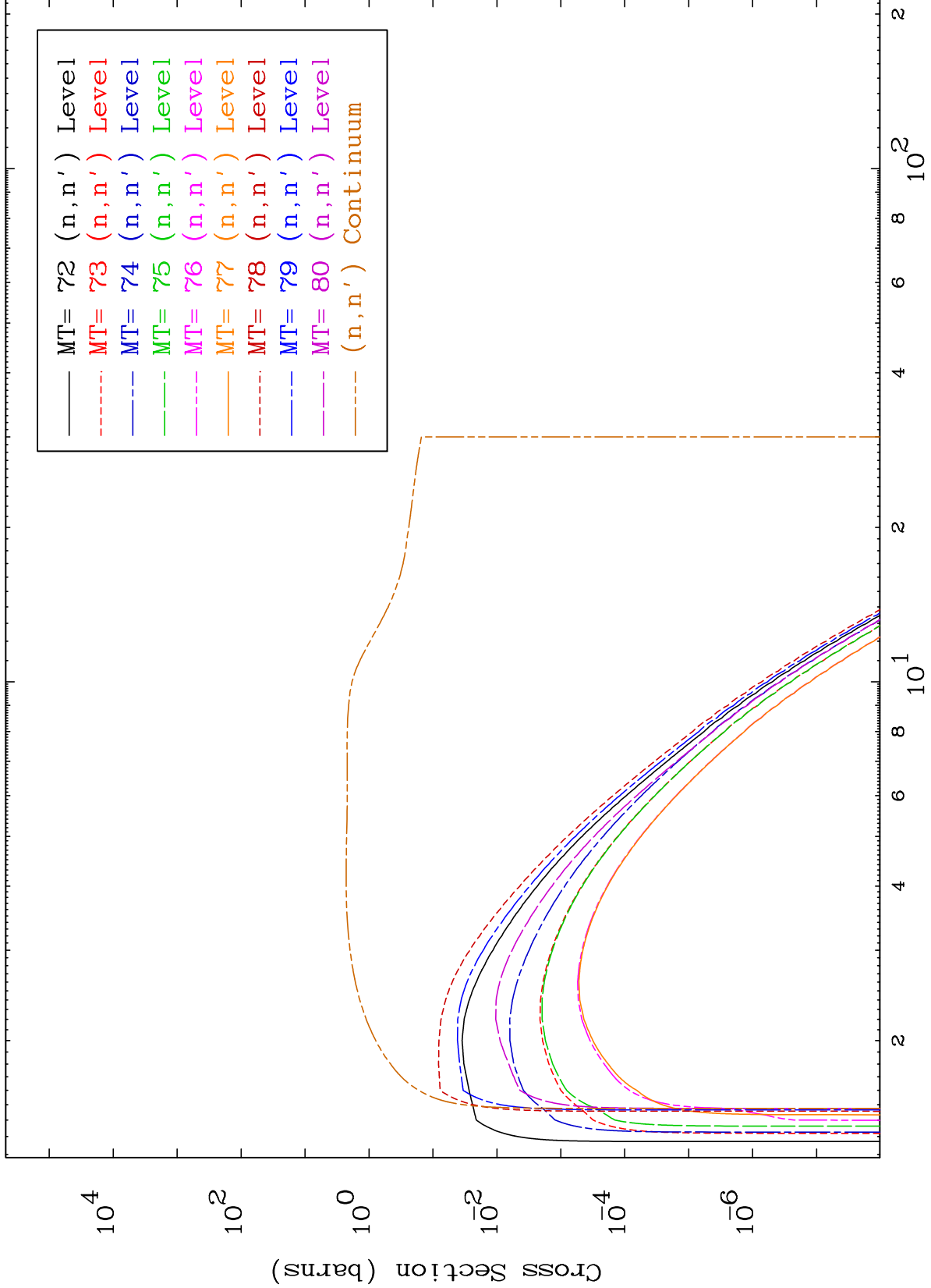
293 Kelvin Cross Sections



Incident Energy (MeV)

60-Nd-141

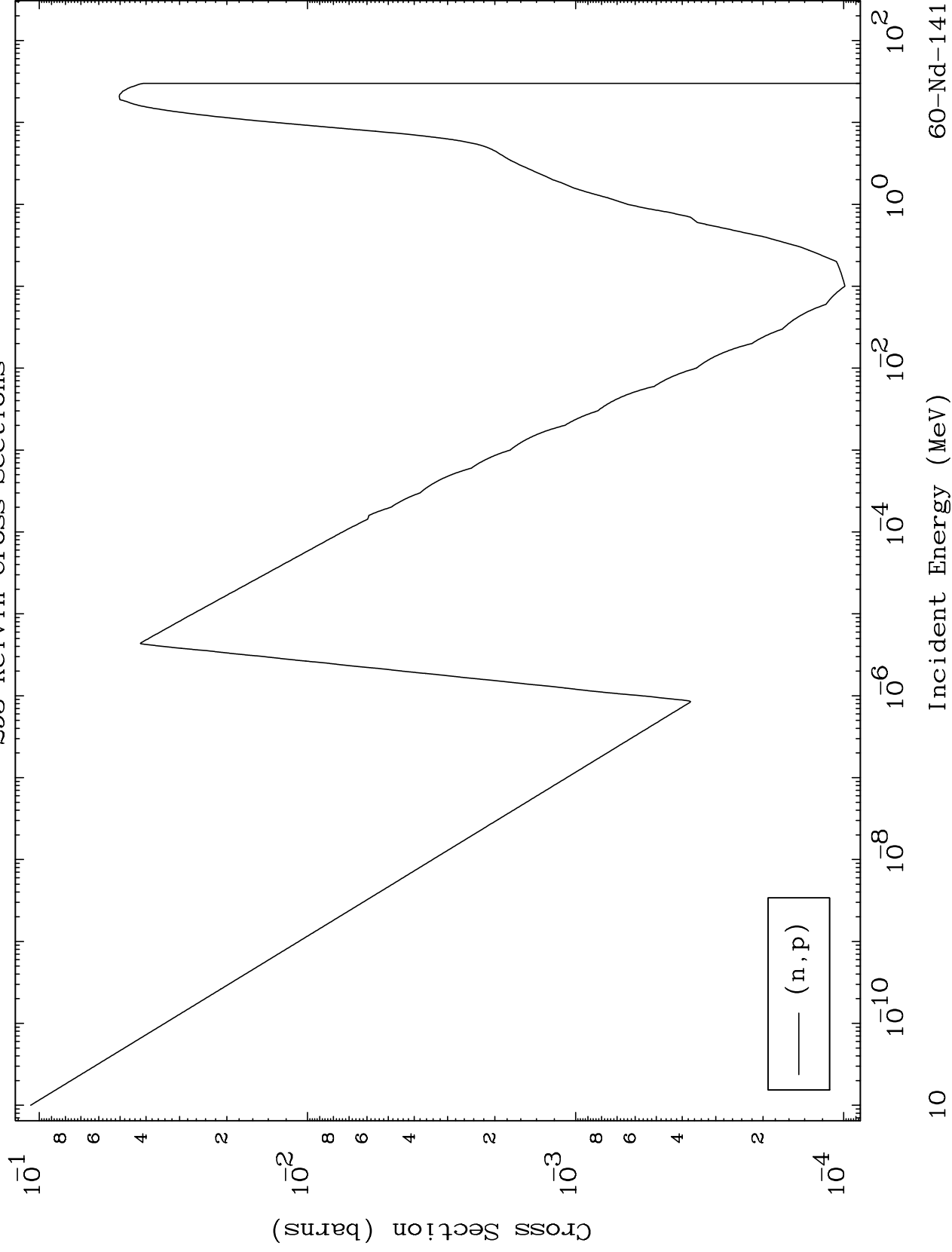
293 Kelvin Cross Sections



MAT 6023

(n,p) Levels
293 Kelvin Cross Sections

60-Nd-141



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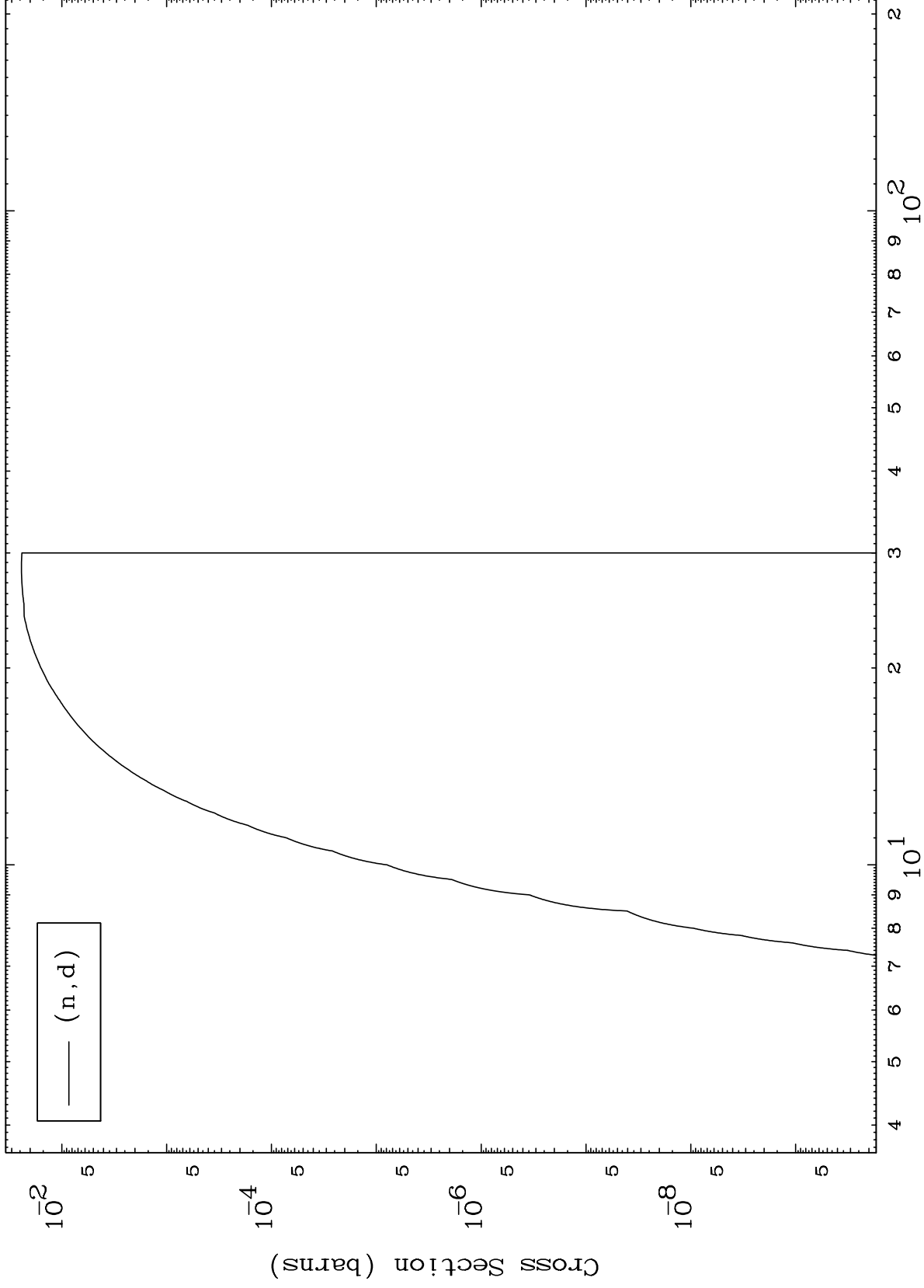
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,d) Levels
293 Kelvin Cross Sections

60-Nd-141



11

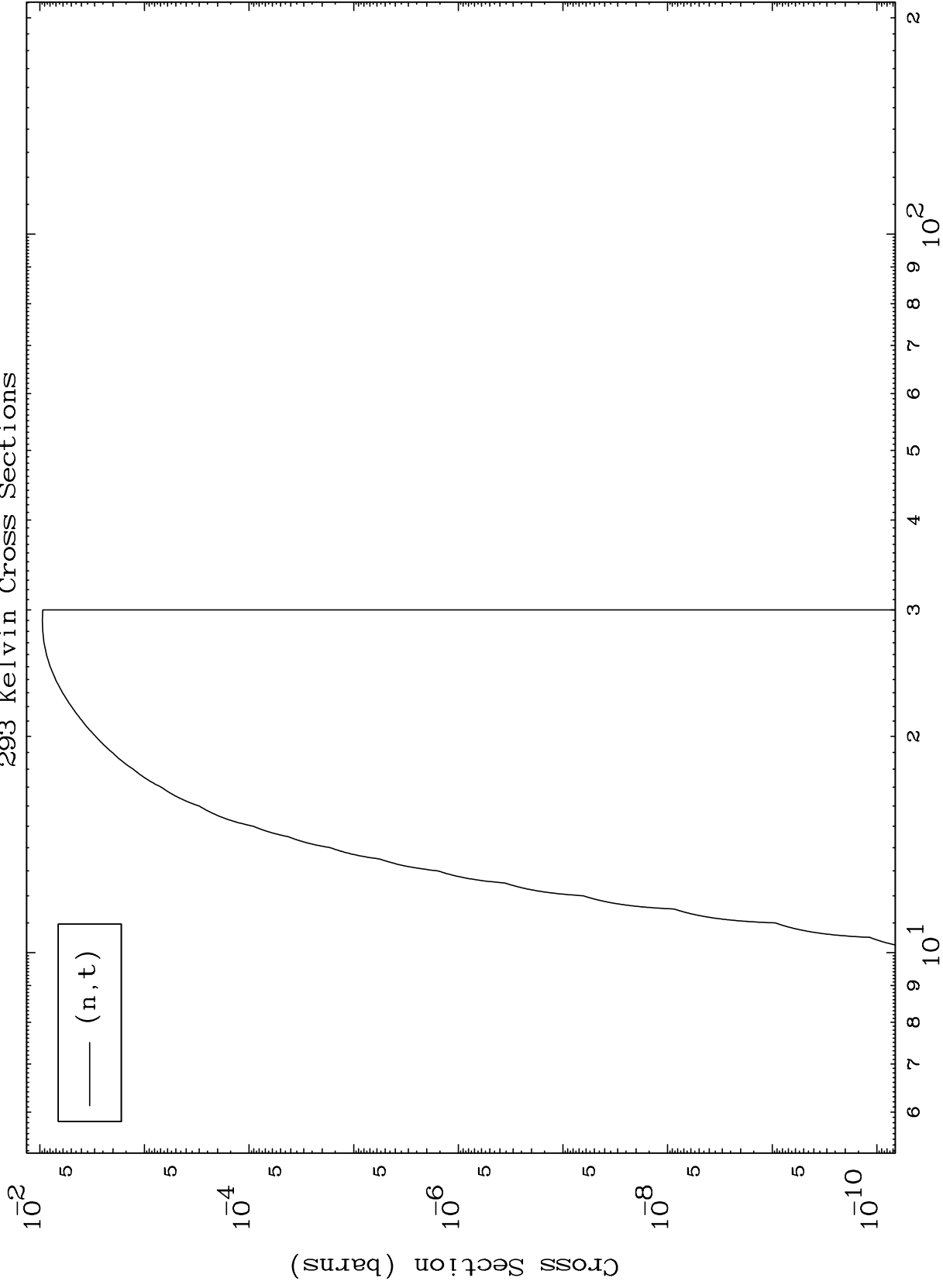
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,t) Levels
293 Kelvin Cross Sections

60-Nd-141



12

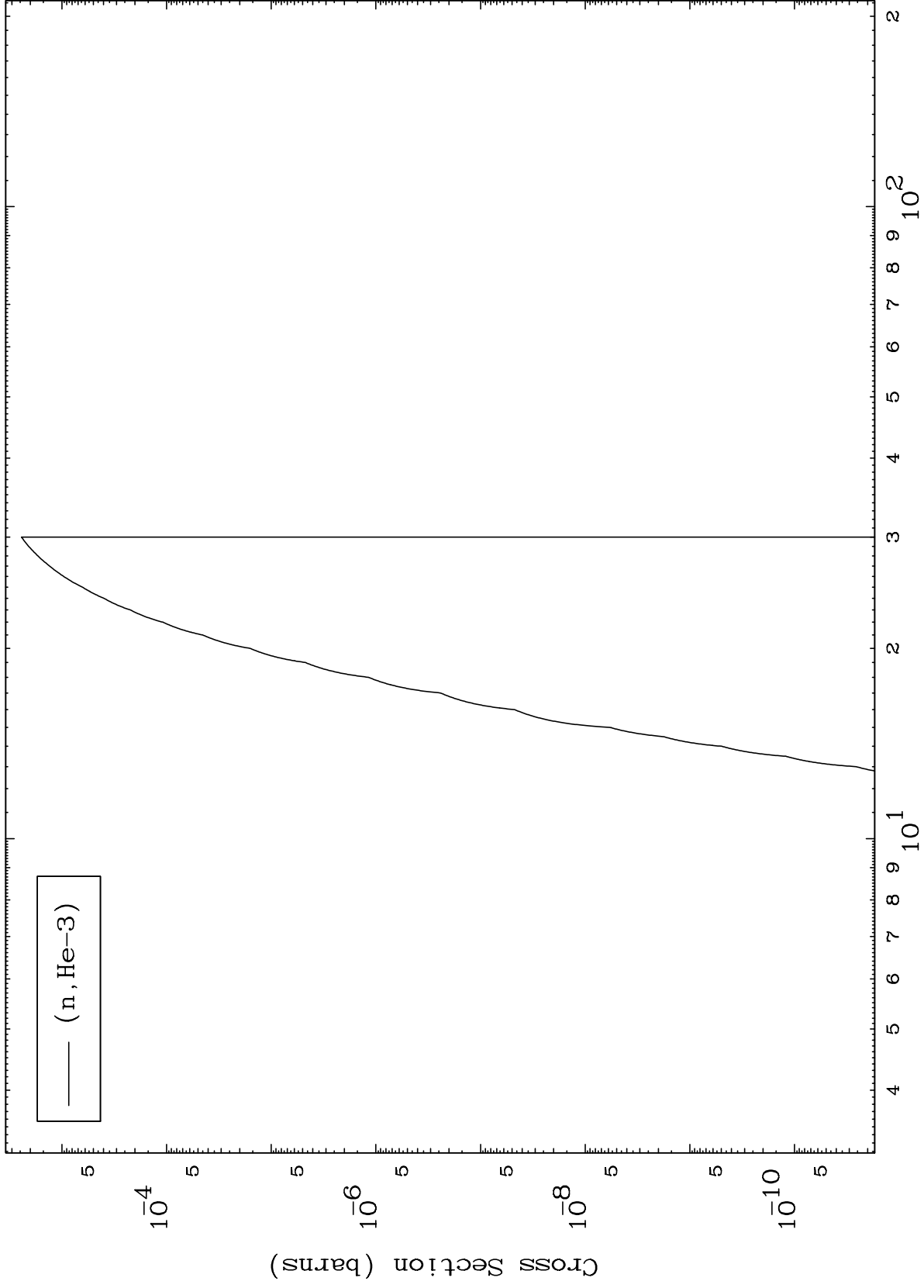
Incident Energy (MeV)

60-Nd-141

MAT 6023

(n,He3) Levels
293 Kelvin Cross Sections

60-Nd-141



13

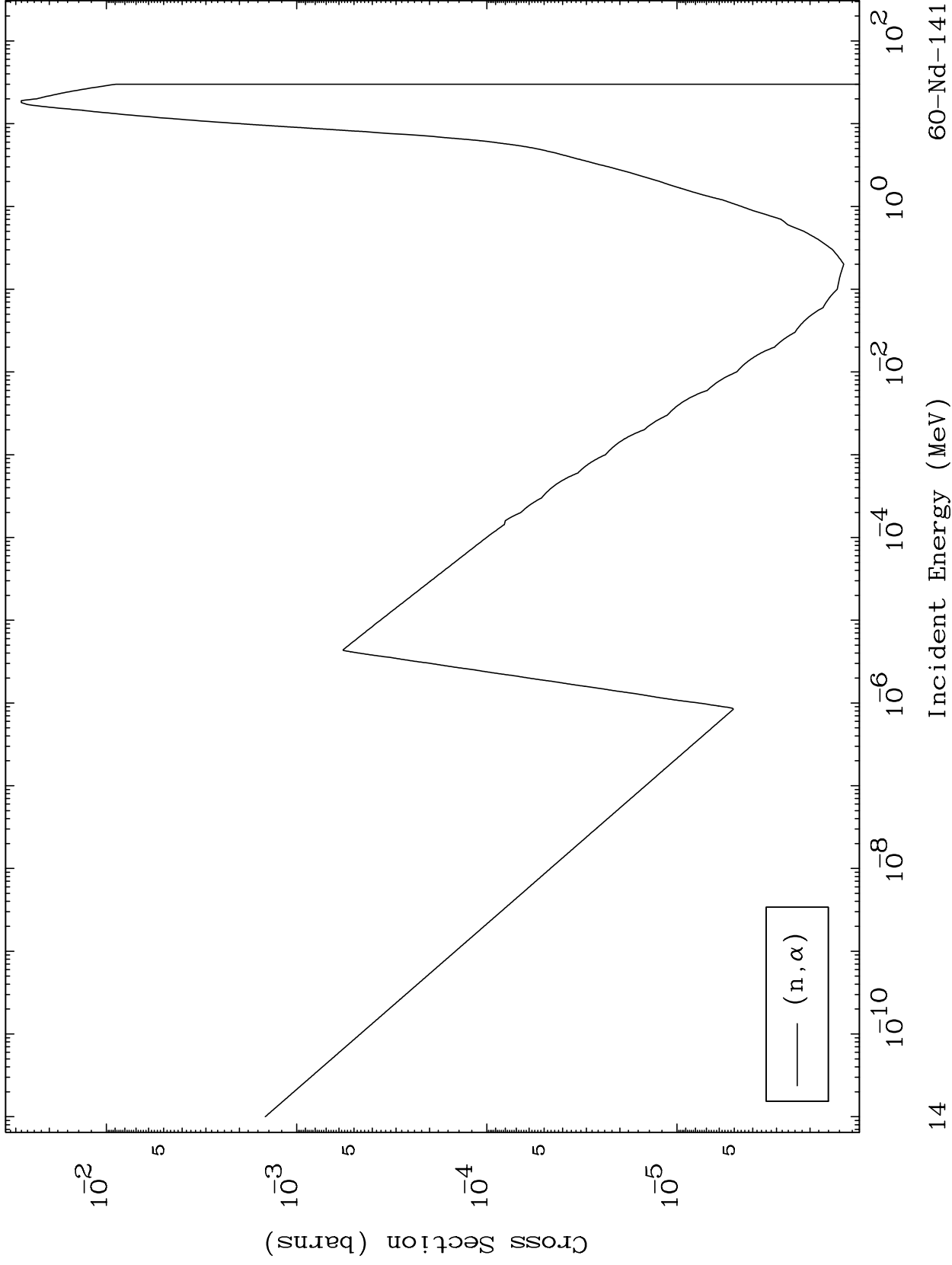
Incident Energy (MeV)

60-Nd-141

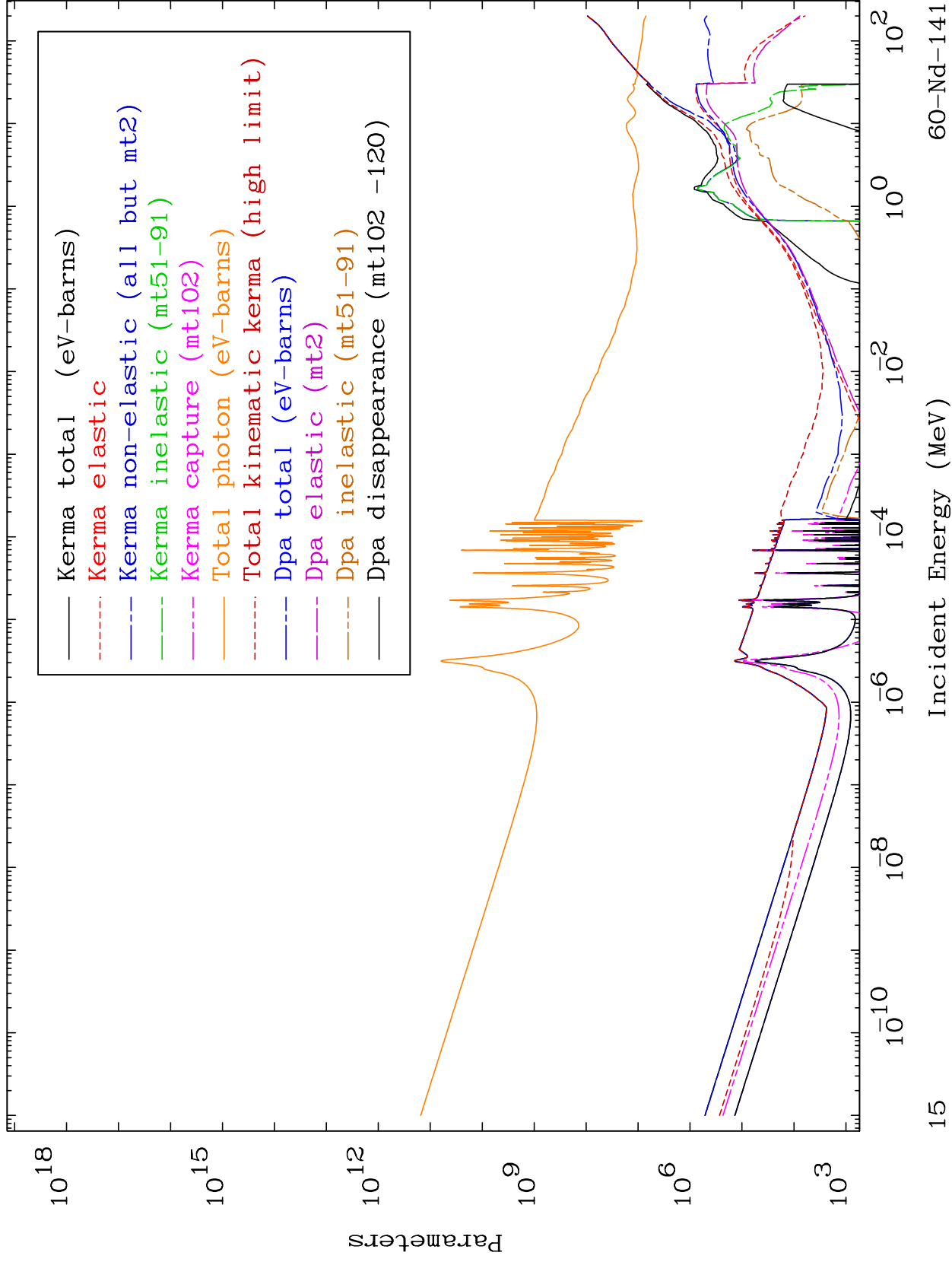
MAT 6023

(n, α) Levels
293 Kelvin Cross Sections

60-Nd-141



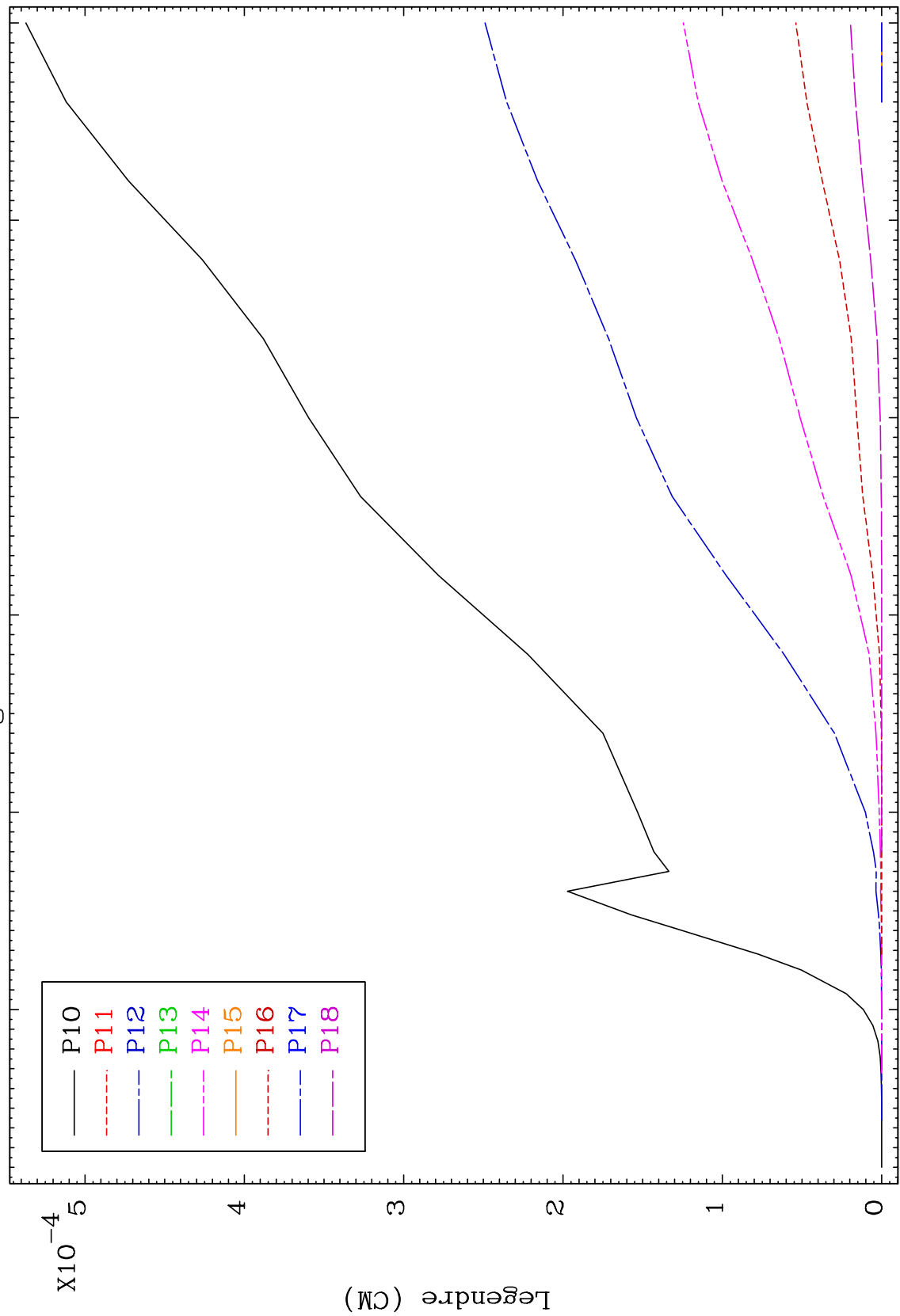
14



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Elastic Legendre Coefficients

⁶⁰Nd-141



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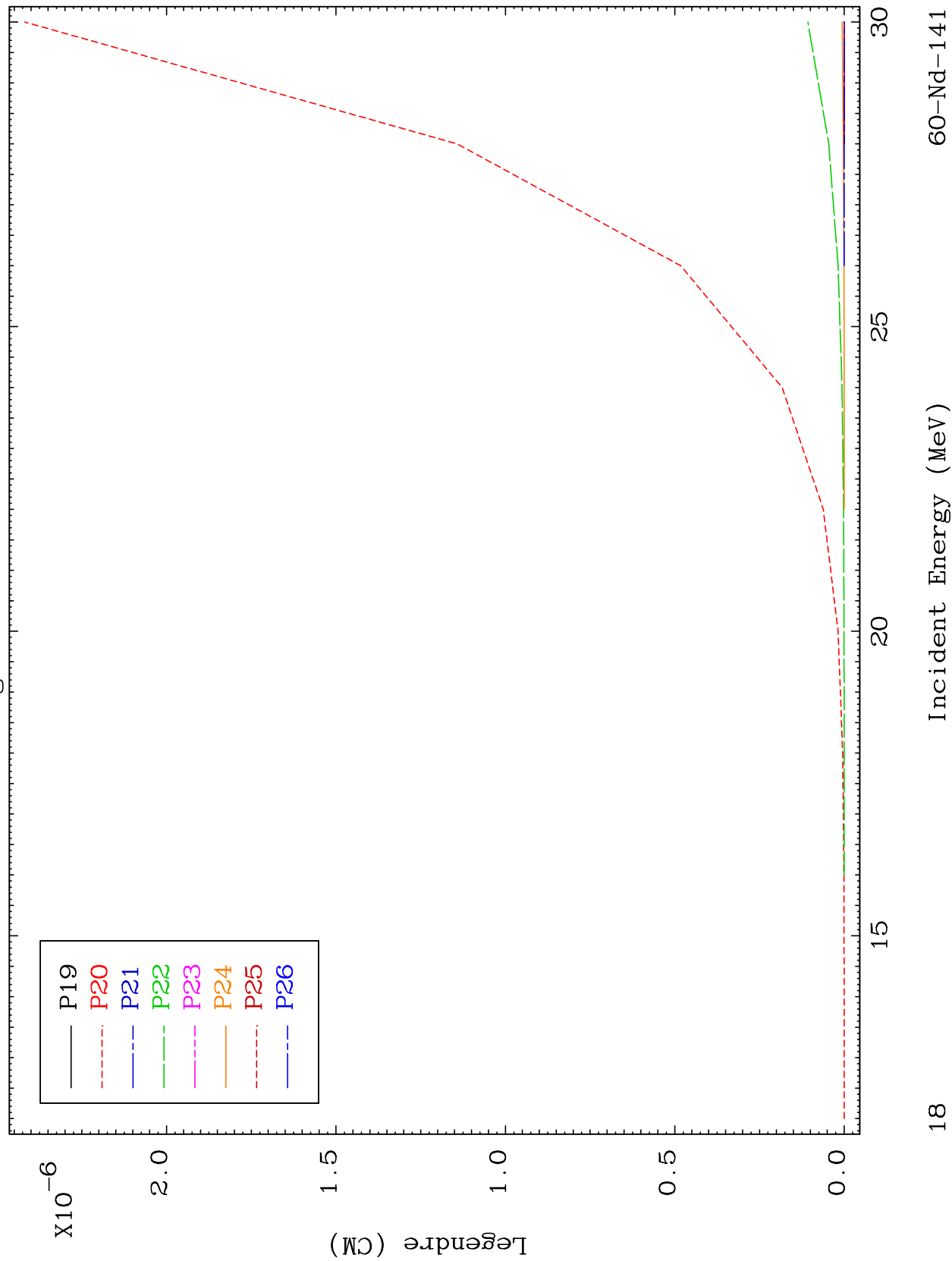
⁶⁰Nd-141

Incident Energy (MeV)

MAT 6023

Elastic Legendre Coefficients

⁶⁰Nd-141



18

Incident Energy (MeV)

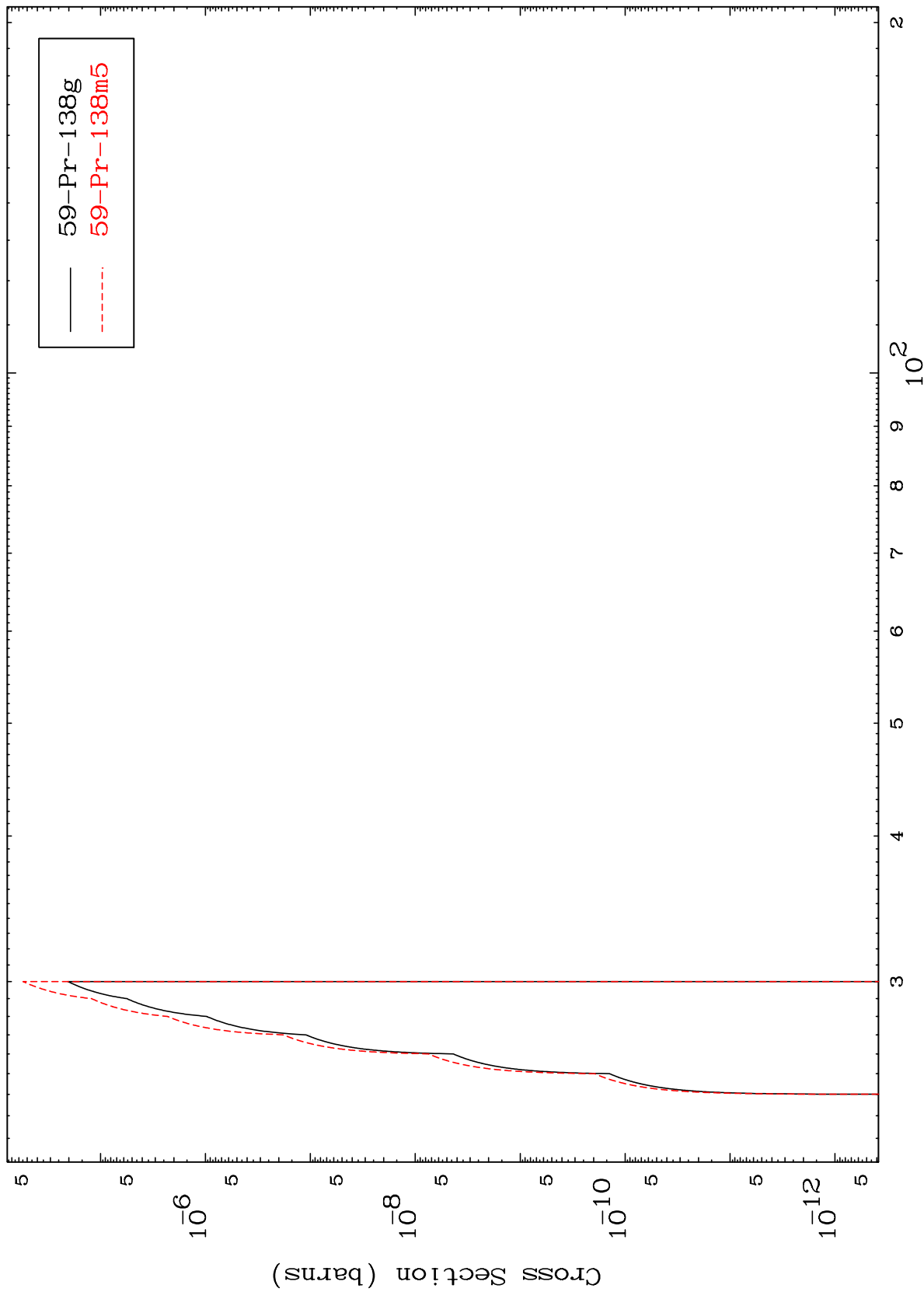
⁶⁰Nd-141

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(n,2n) d

60-Nd-141

Radionuclide Production Cross Section



20

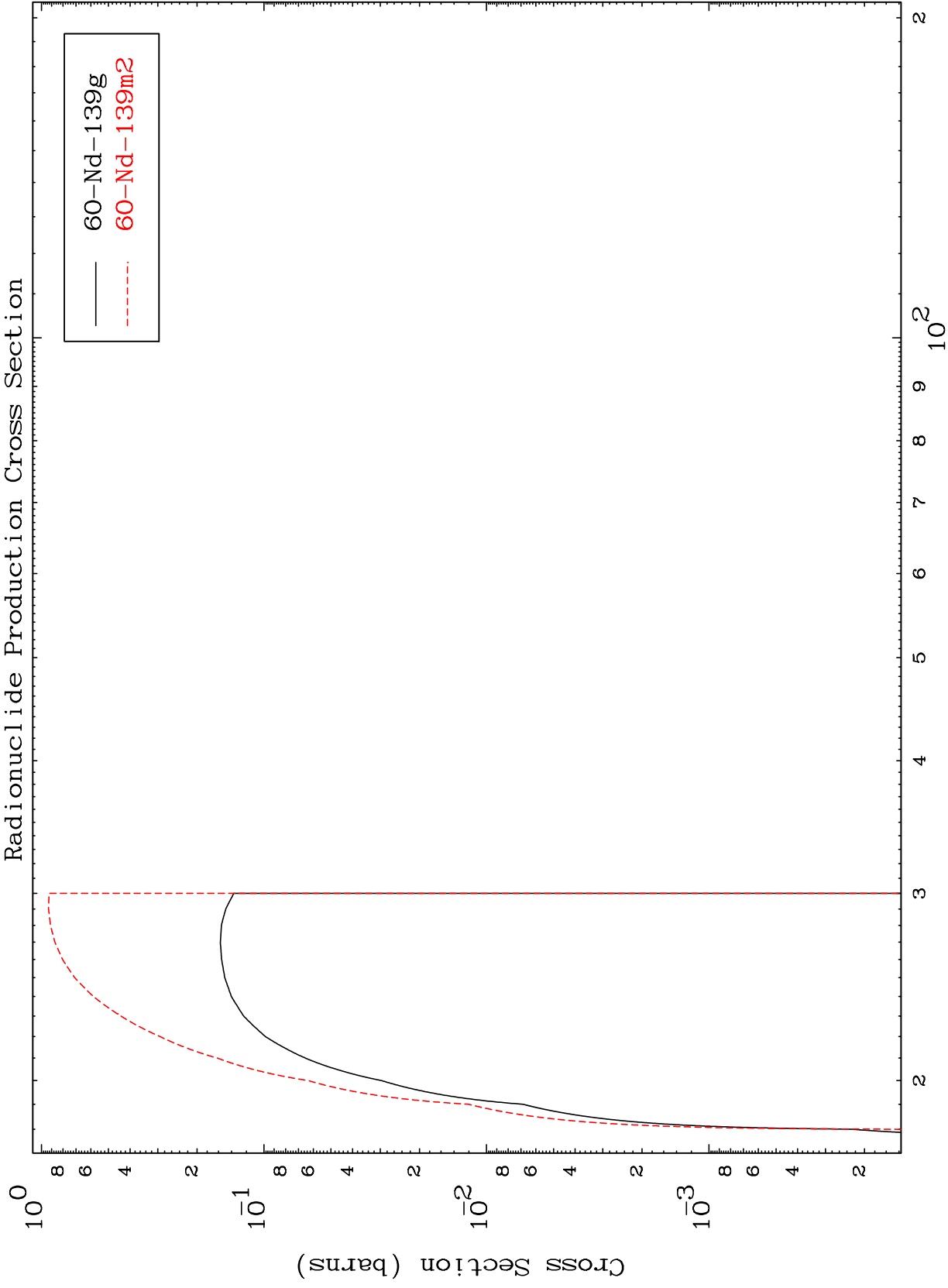
Incident Energy (MeV)

60-Nd-141

MAT 6023

60-Nd-141

(n,3n)
Radionuclide Production Cross Section



21

Incident Energy (MeV)

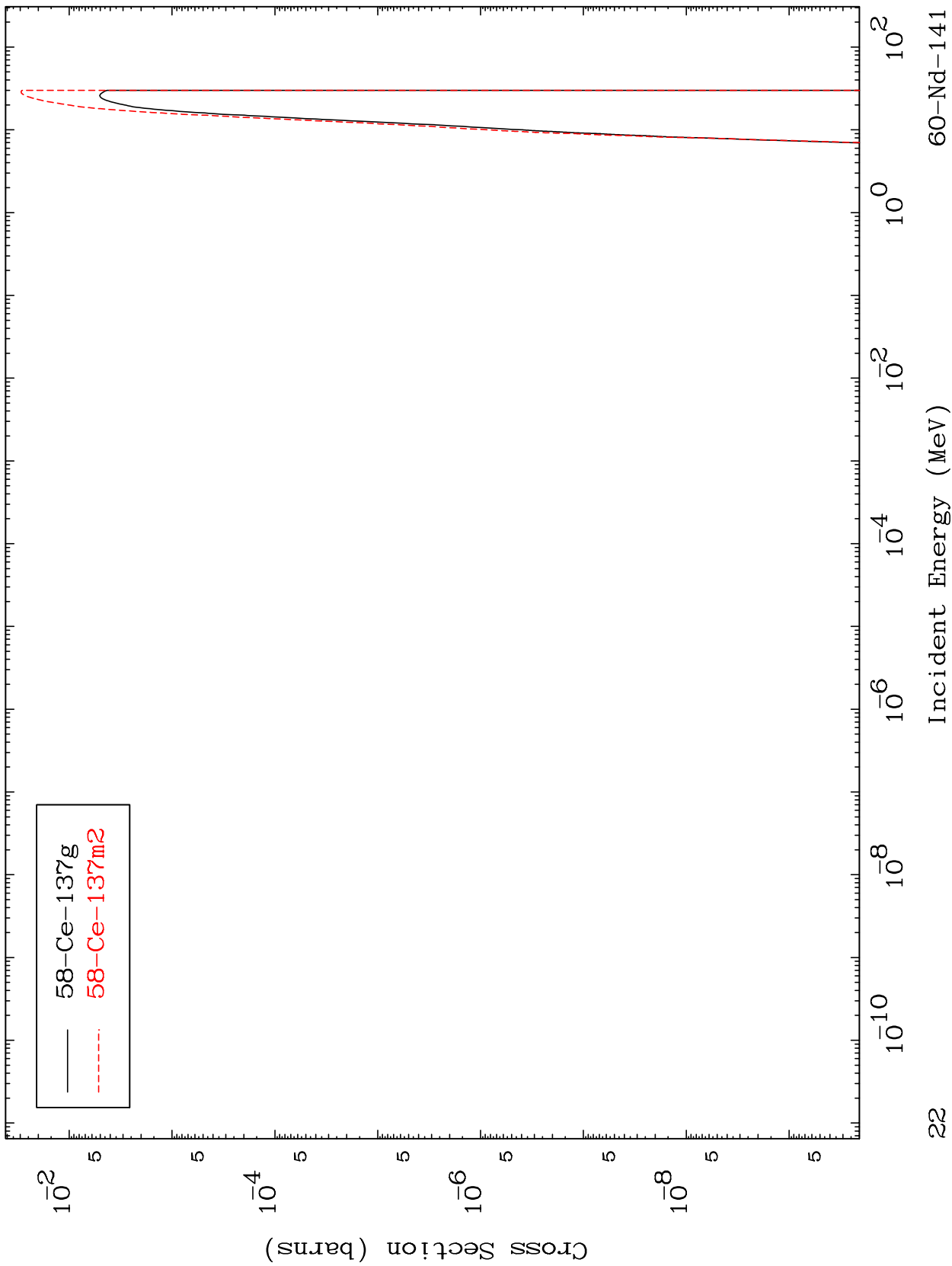
60-Nd-141

MAT 6023

$(n, n') \alpha$

60-Nd-141

Radionuclide Production Cross Section



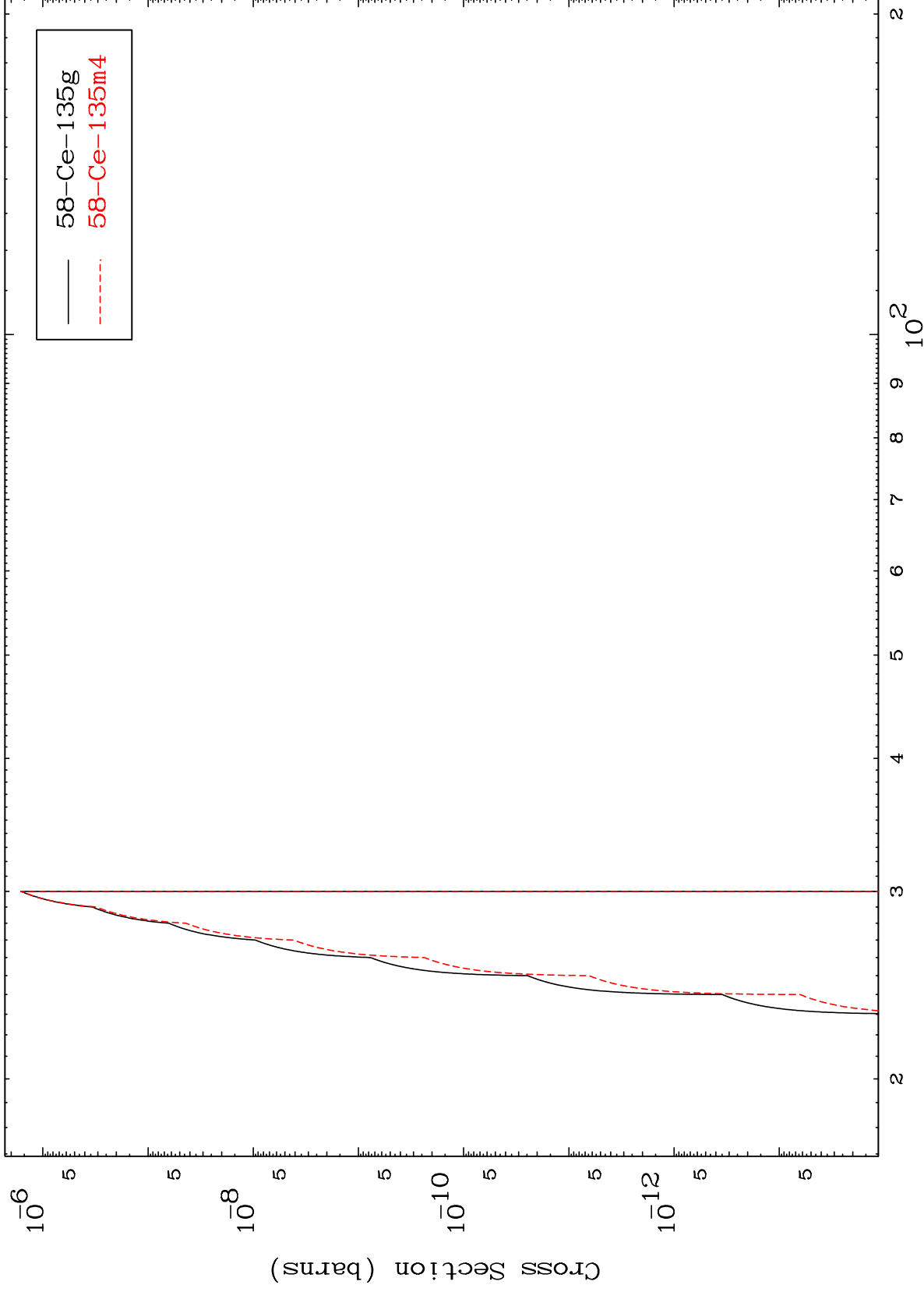
— 58-Ce-137g
- - - 58-Ce-137m2

MAT 6023

(n,3n) α

60-Nd-141

Radionuclide Production Cross Section



23

Incident Energy (MeV)

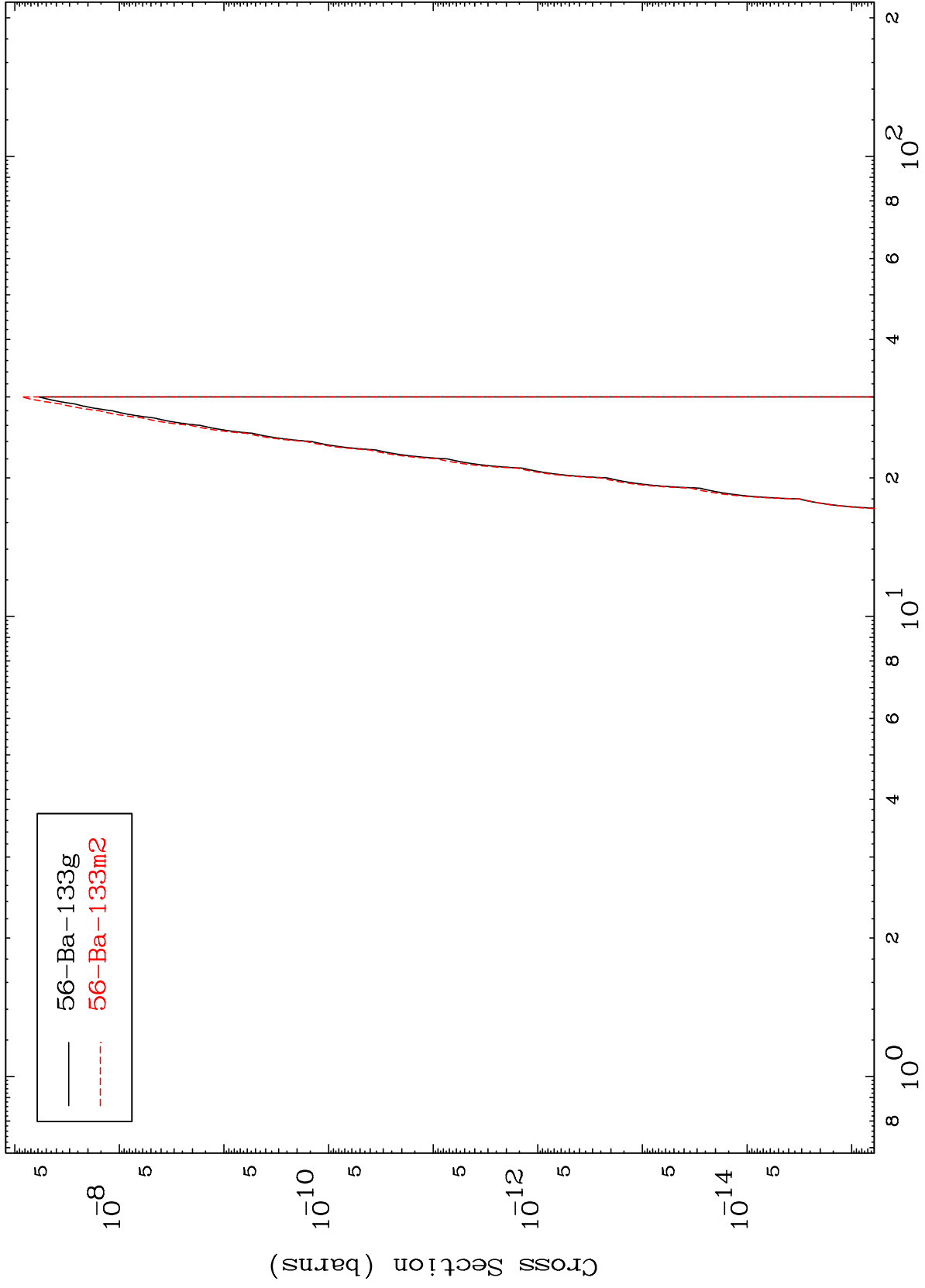
60-Nd-141

MAT 6023

(n,n') 2α

60-Nd-141

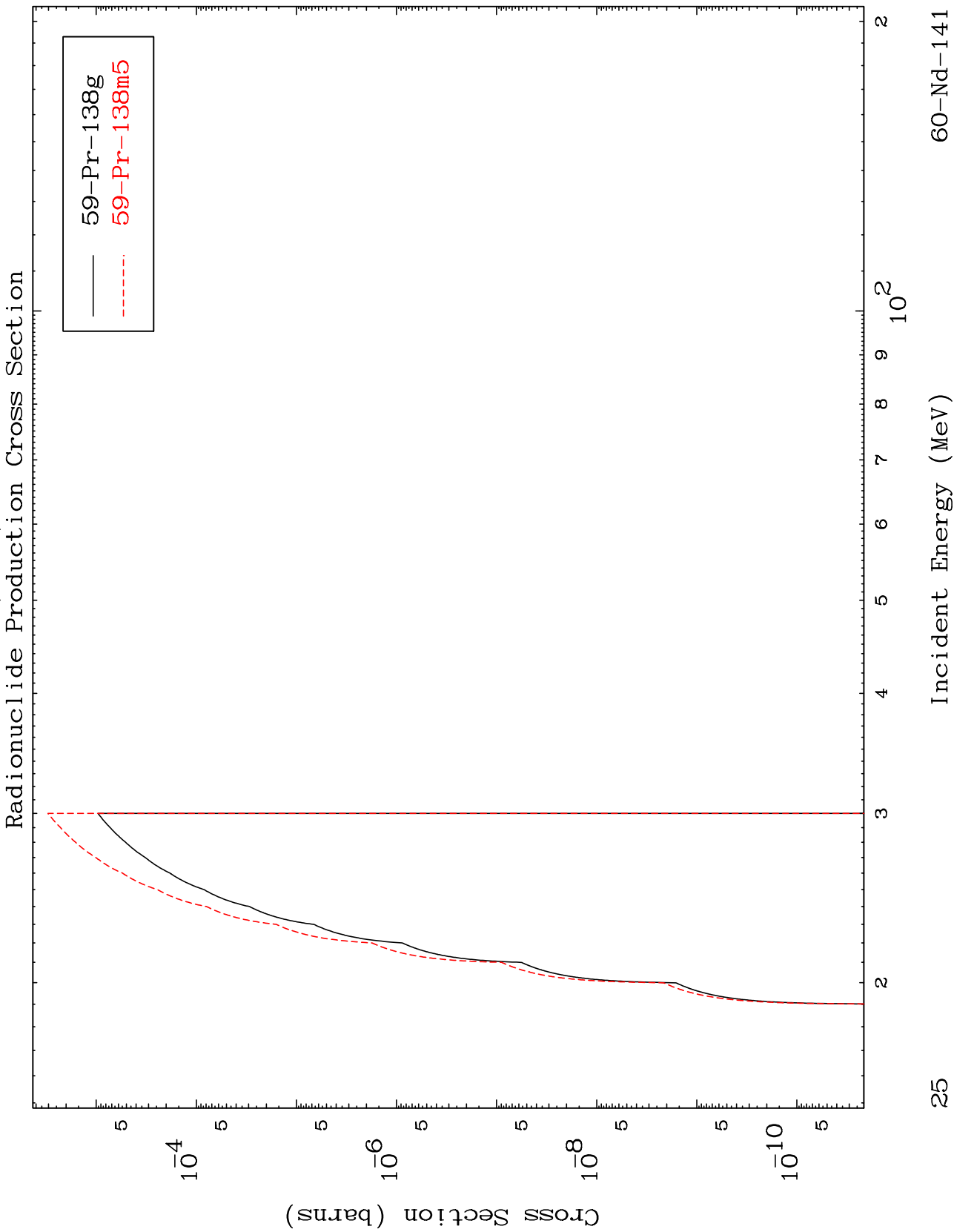
Radionuclide Production Cross Section



24

Incident Energy (MeV)

60-Nd-141

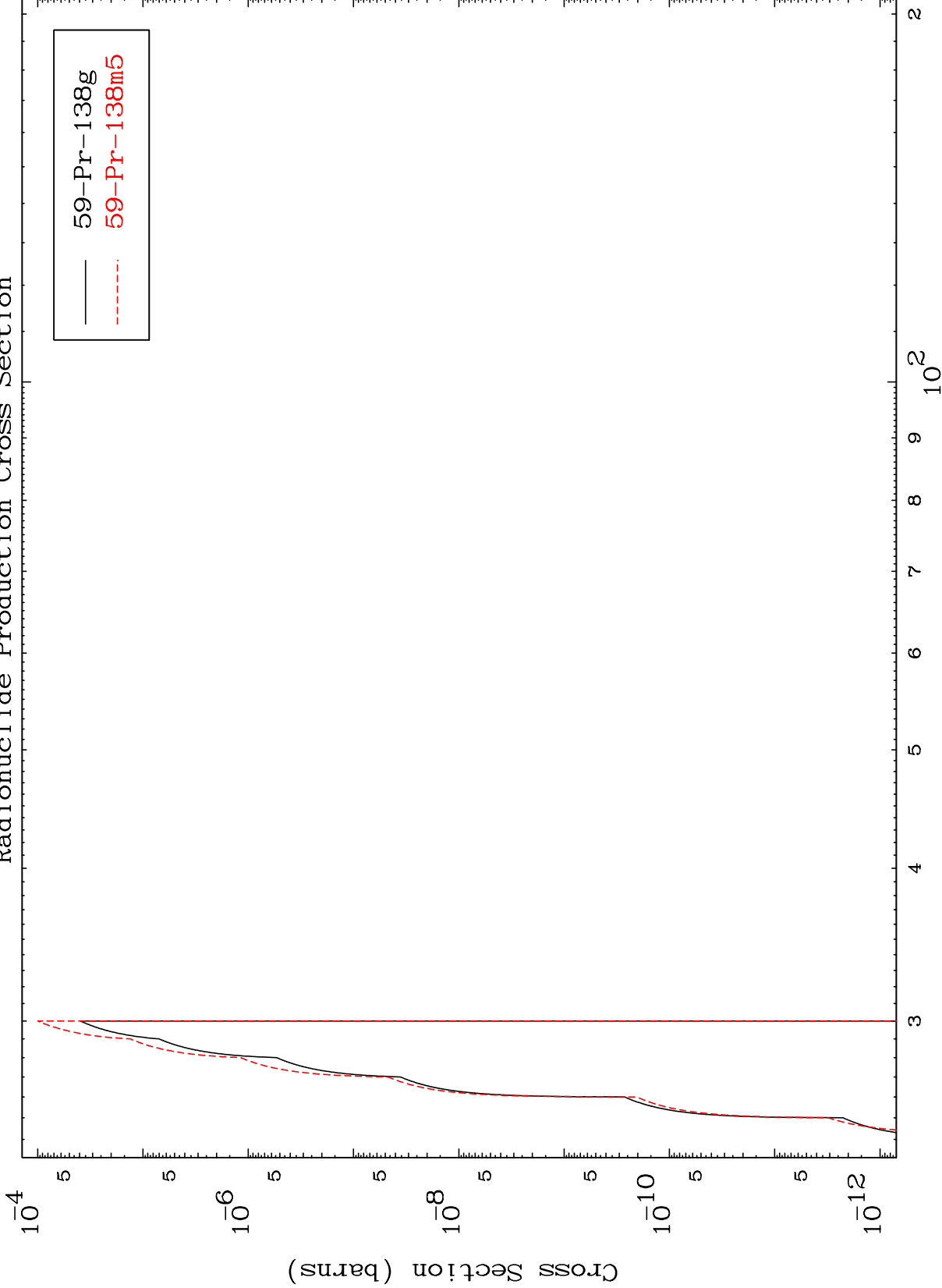


MAT 6023

(n,3n) p

60-Nd-141

Radionuclide Production Cross Section



26

Incident Energy (MeV)

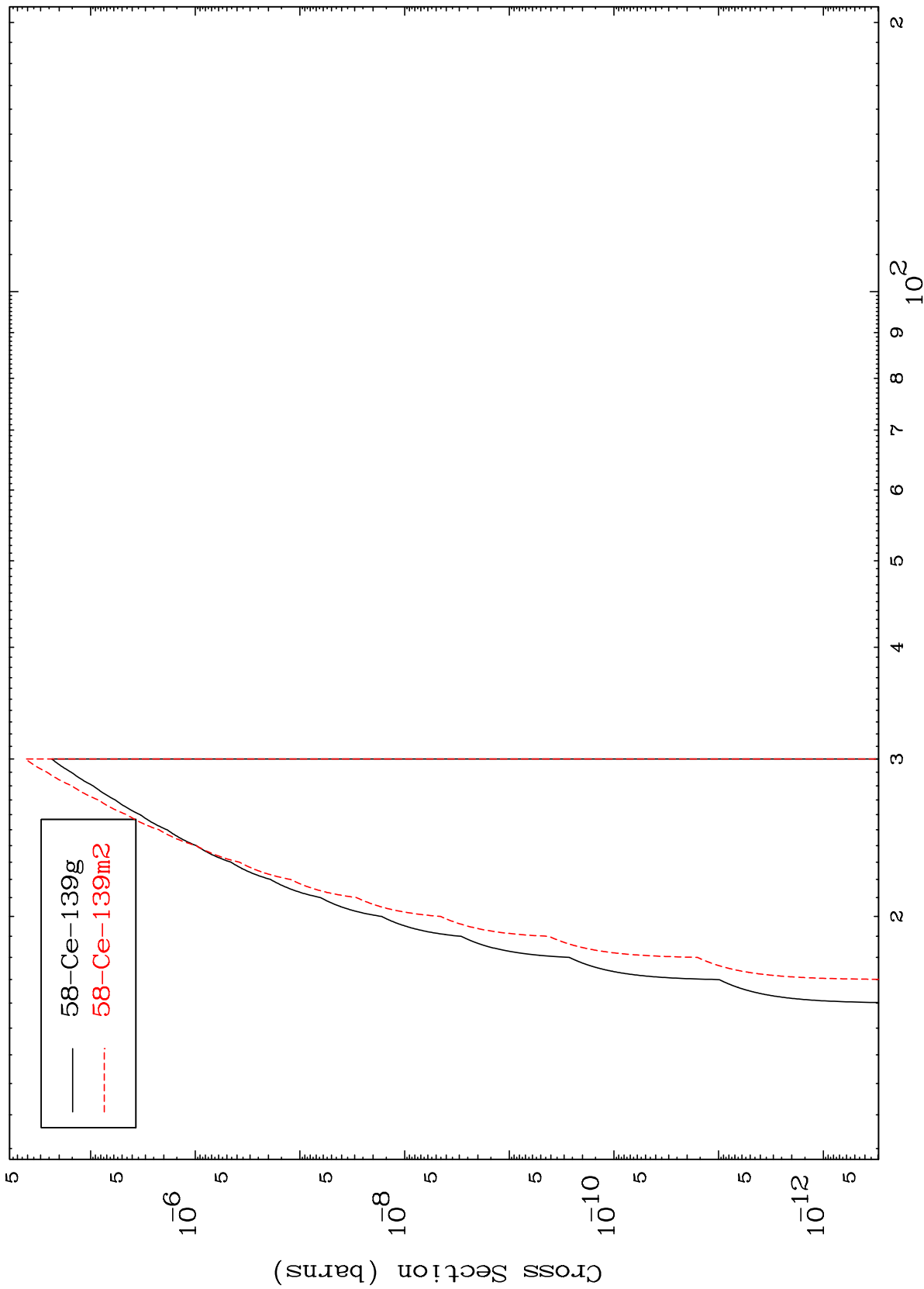
60-Nd-141

MAT 6023

(n,2n) p

60-Nd-141

Radionuclide Production Cross Section



Incident Energy (MeV)

60-Nd-141

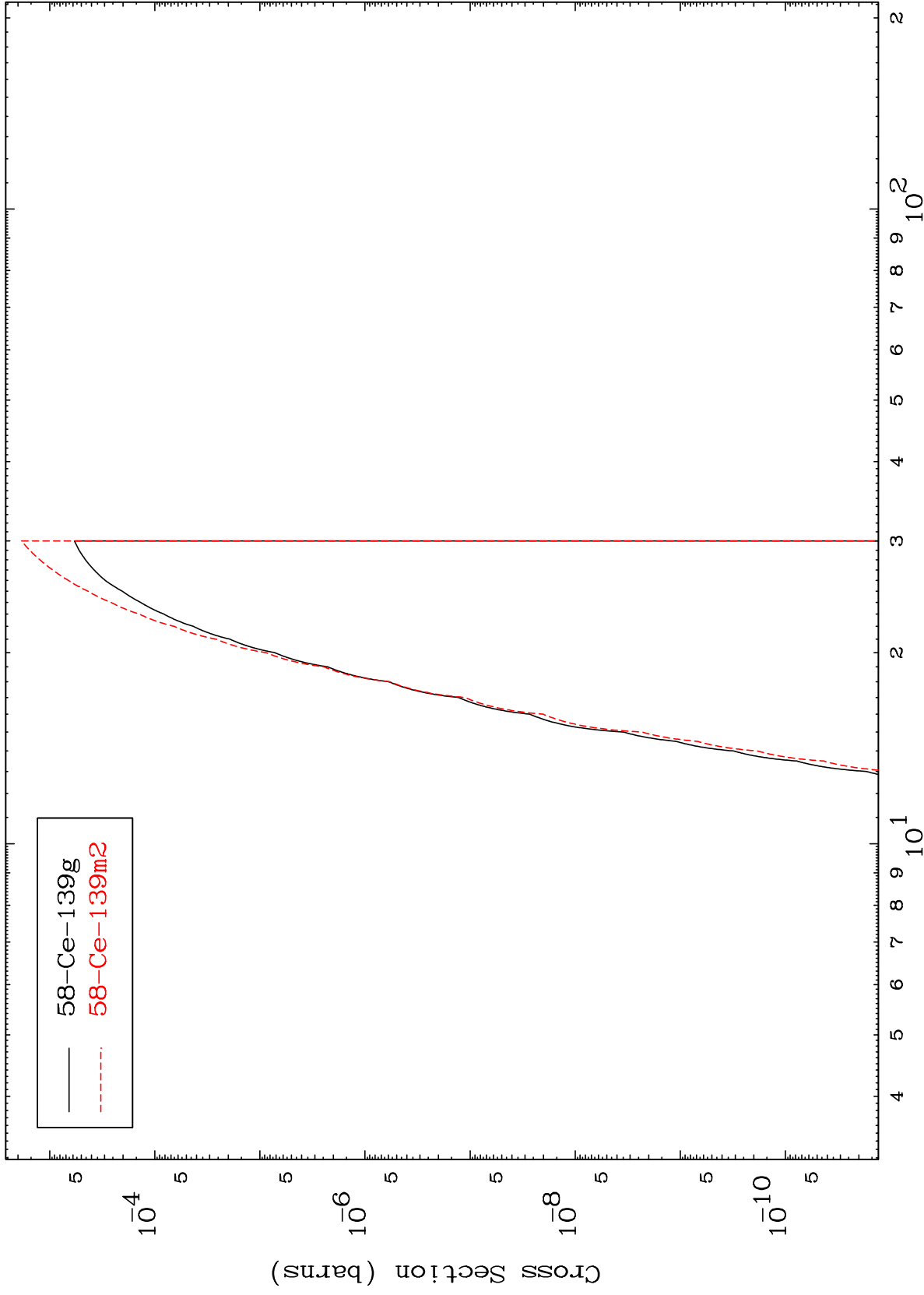
27

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(n,He-3)

60-Nd-141

Radionuclide Production Cross Section



28

Incident Energy (MeV)

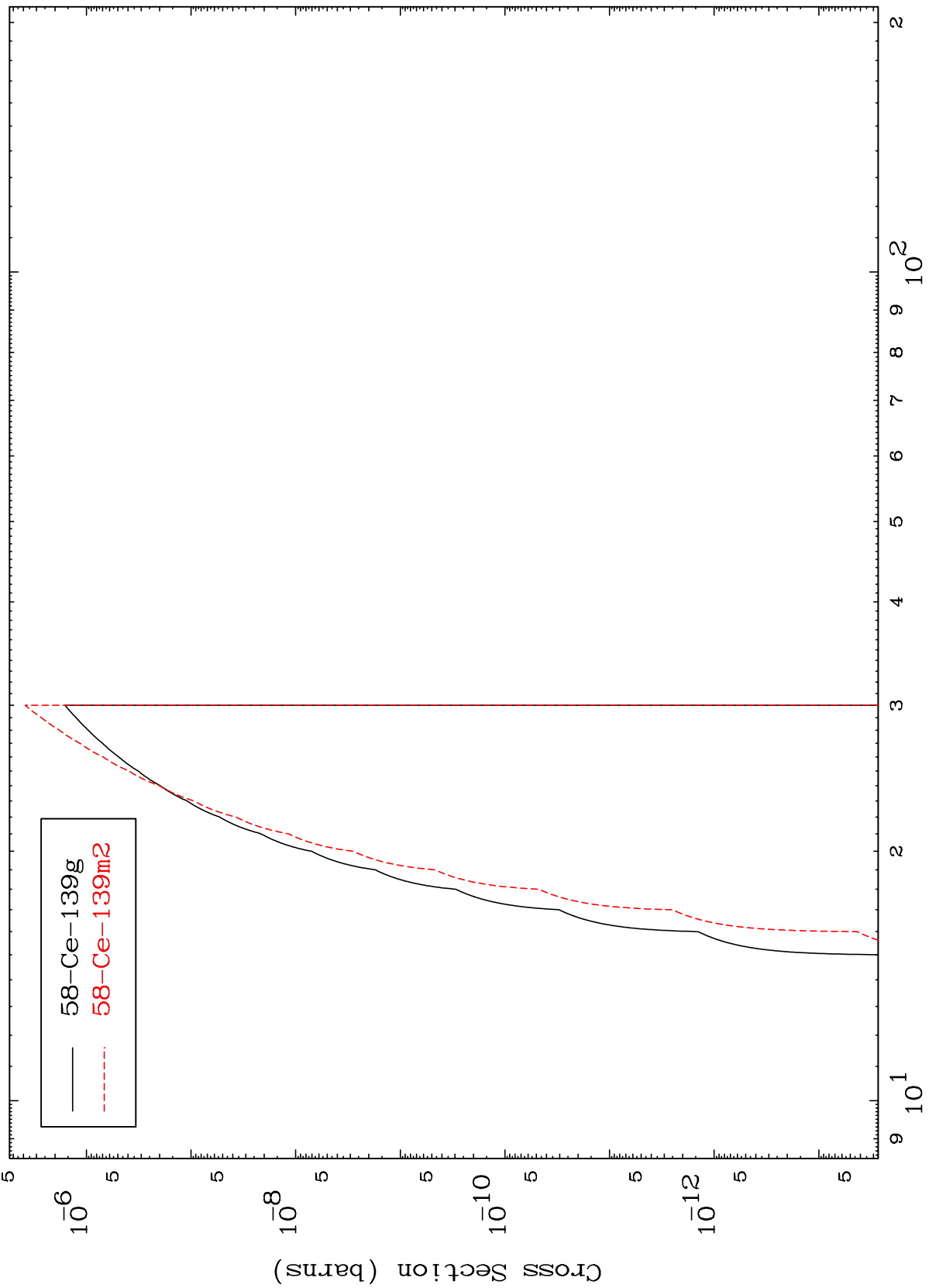
60-Nd-141

MAT 6023

(n,p) d

60-Nd-141

Radionuclide Production Cross Section



58-Ce-139g
58-Ce-139m2

29

Incident Energy (MeV)

60-Nd-141