

Program EVALPLOT
(Version 2017-1)

by

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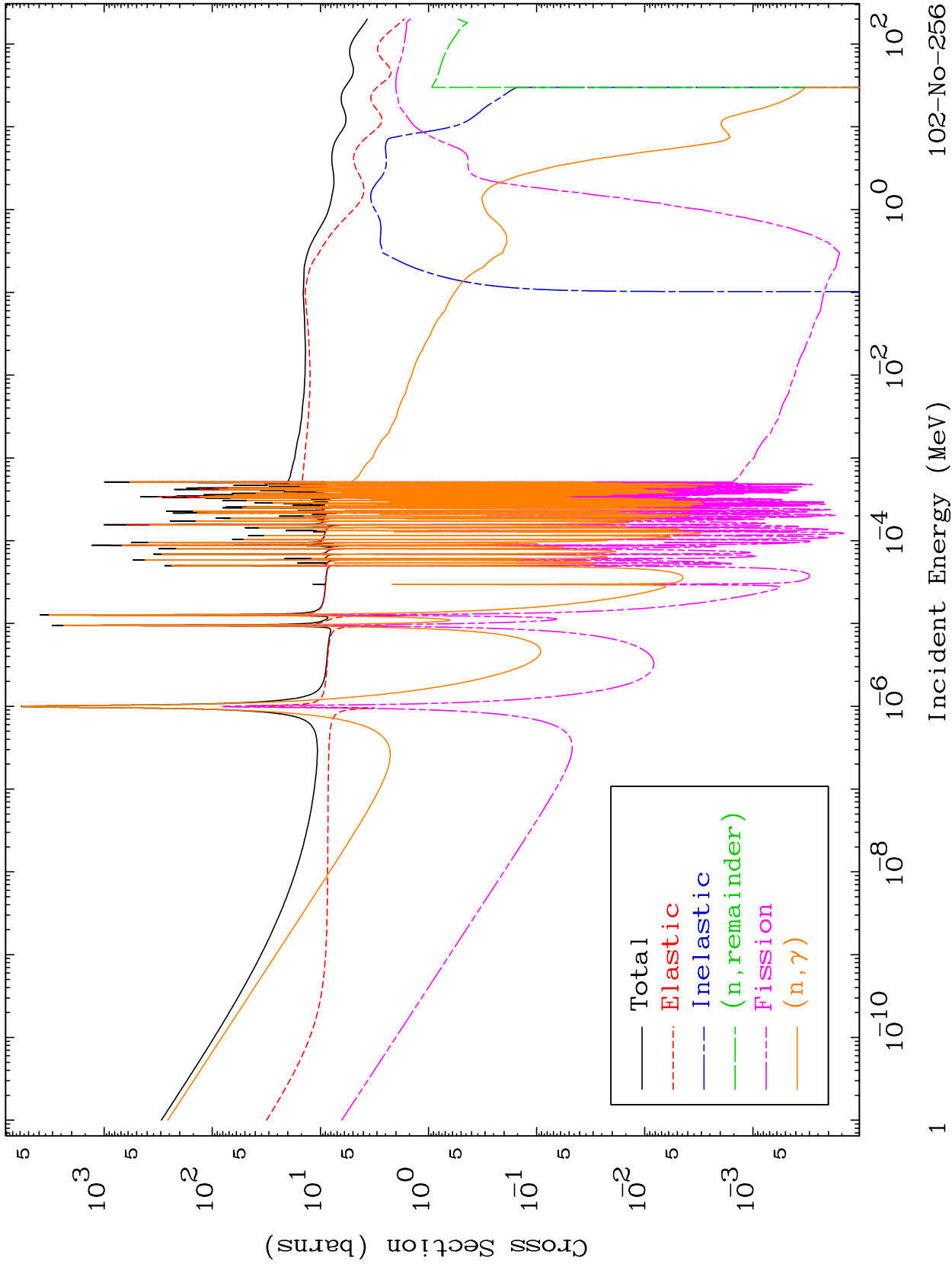
Press Mouse Button to Start

MAT 9974

Major

293 Kelvin Cross Sections

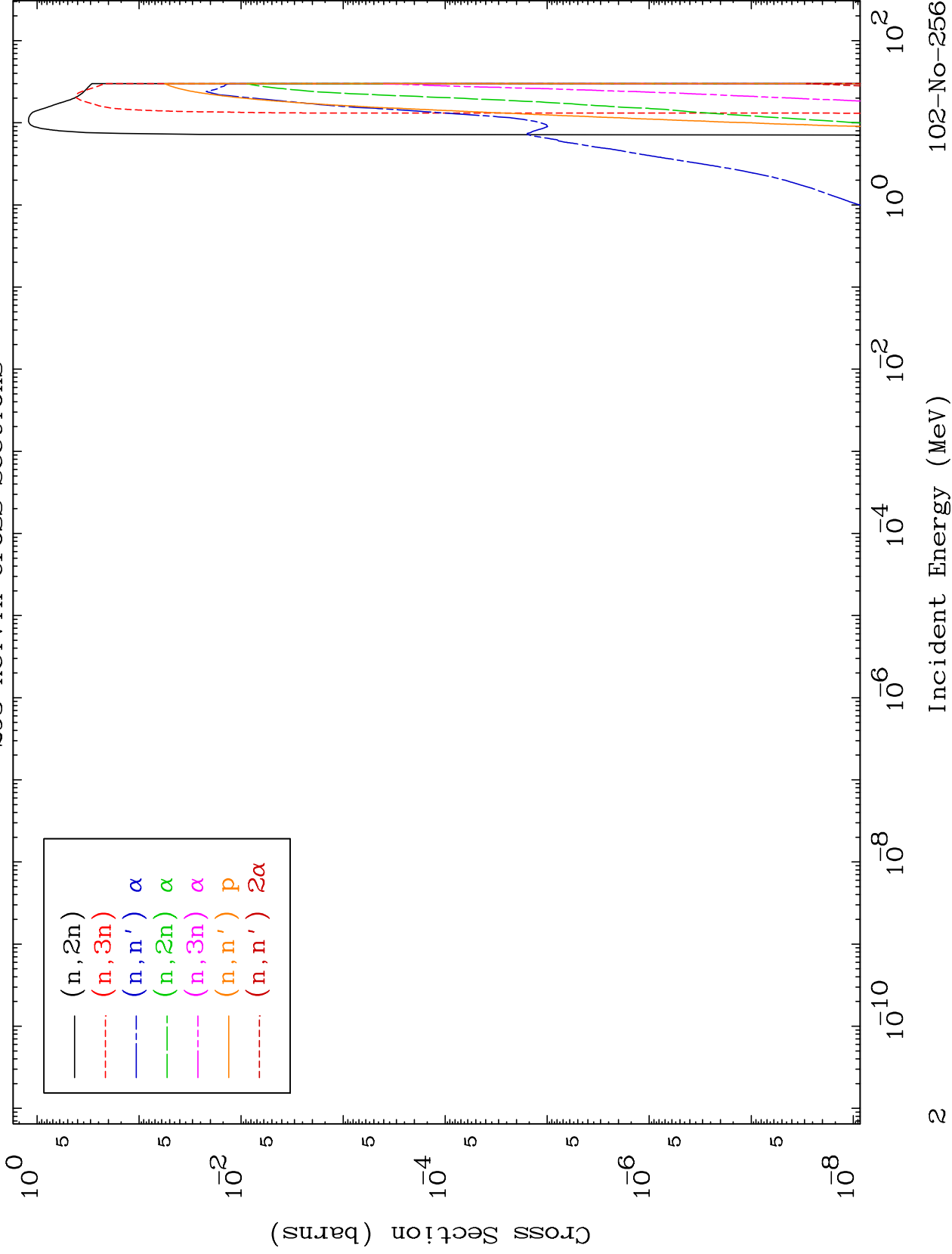
102-No-256

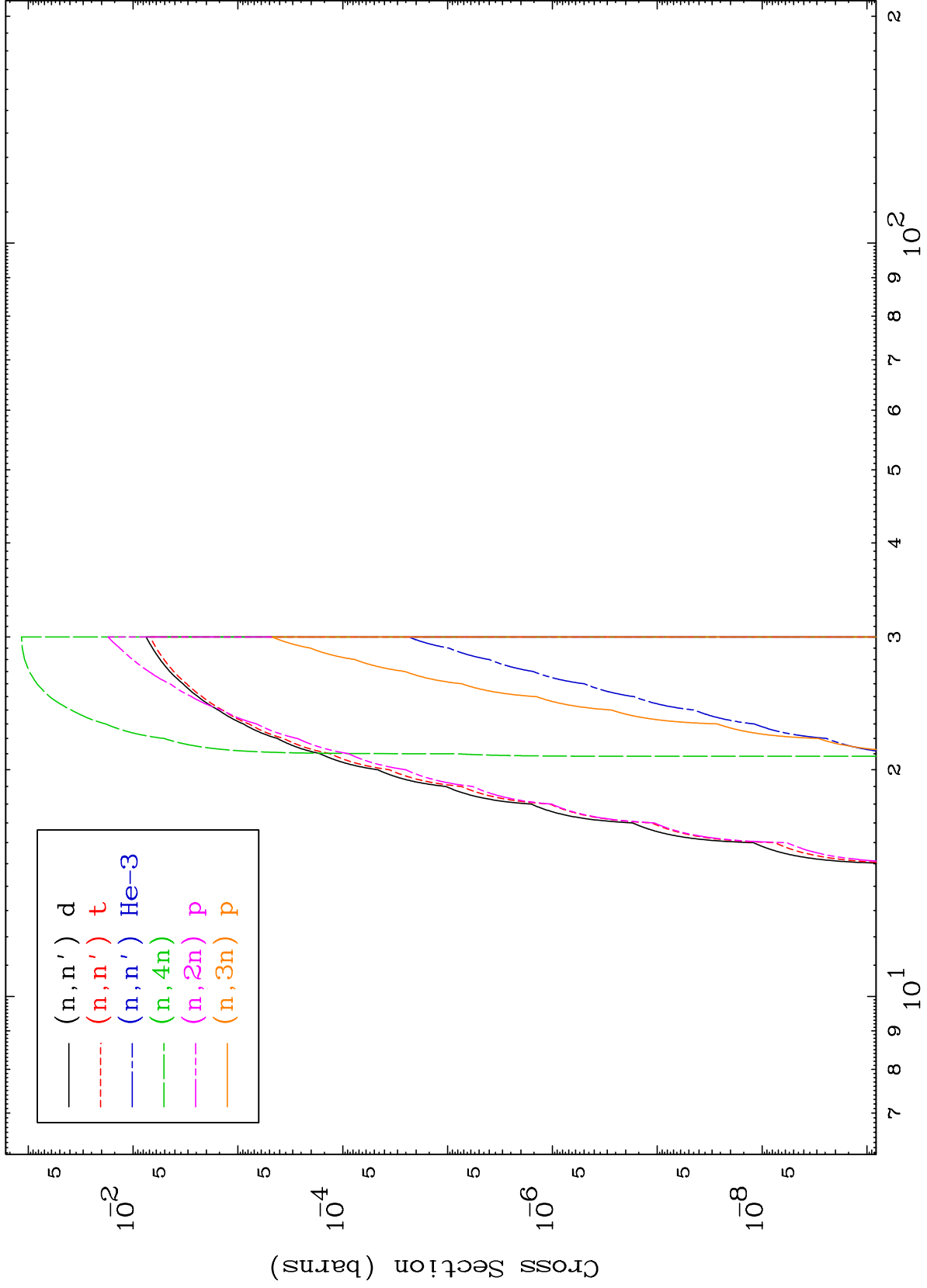


MAT 9974

Neutron Production
293 Kelvin Cross Sections

102-No-256

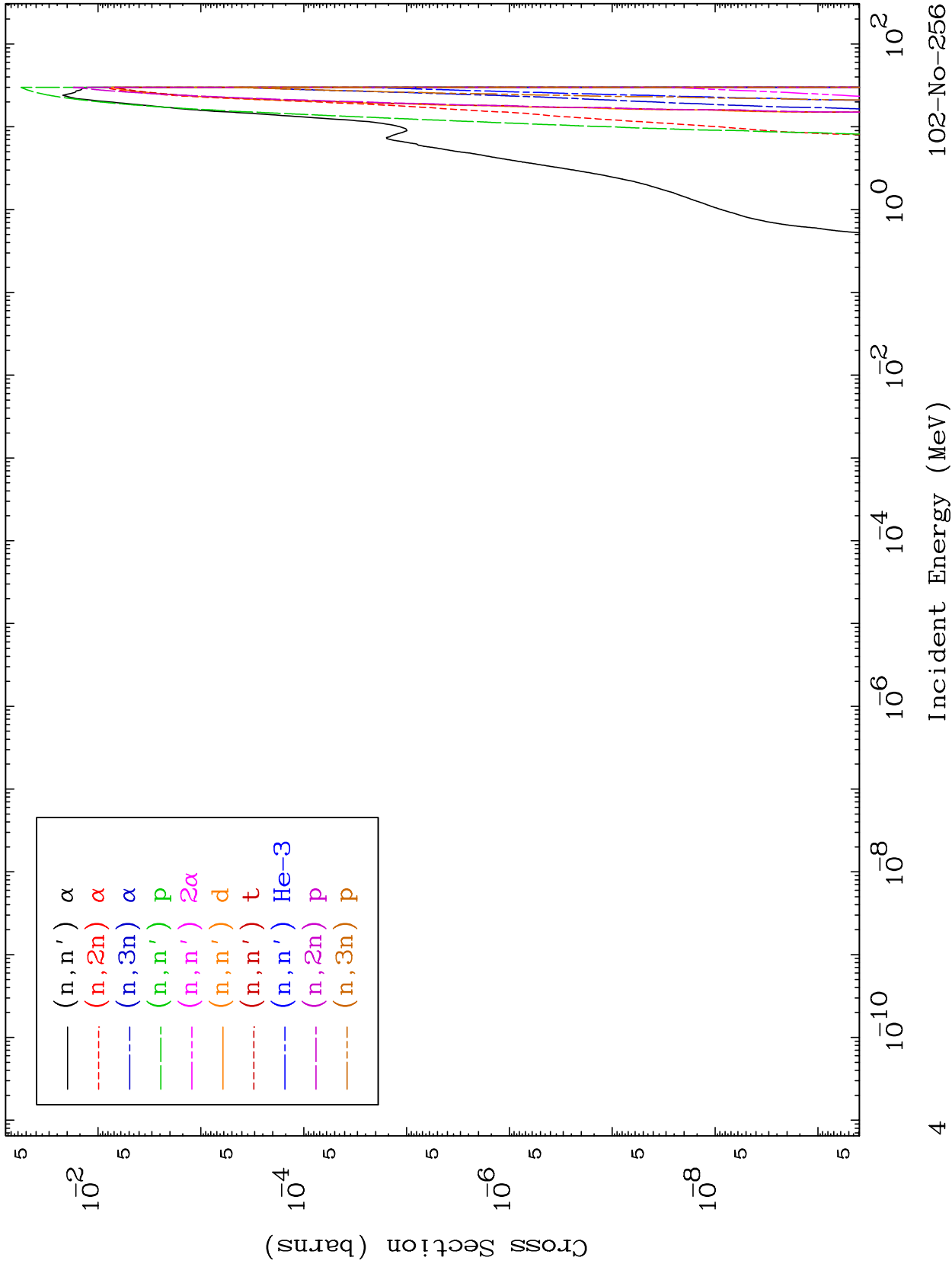




MAT 9974

Charged Particle
293 Kelvin Cross Sections

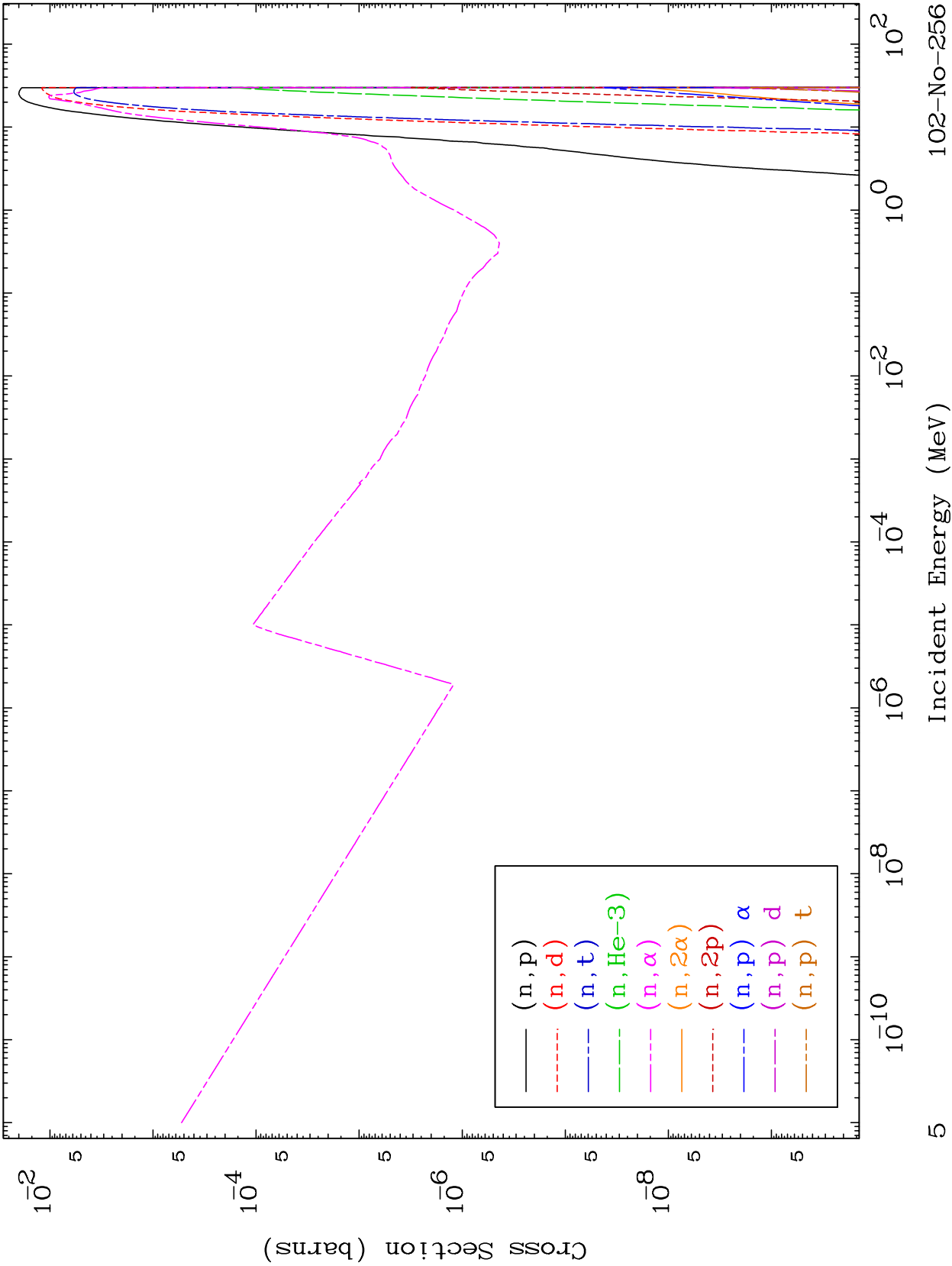
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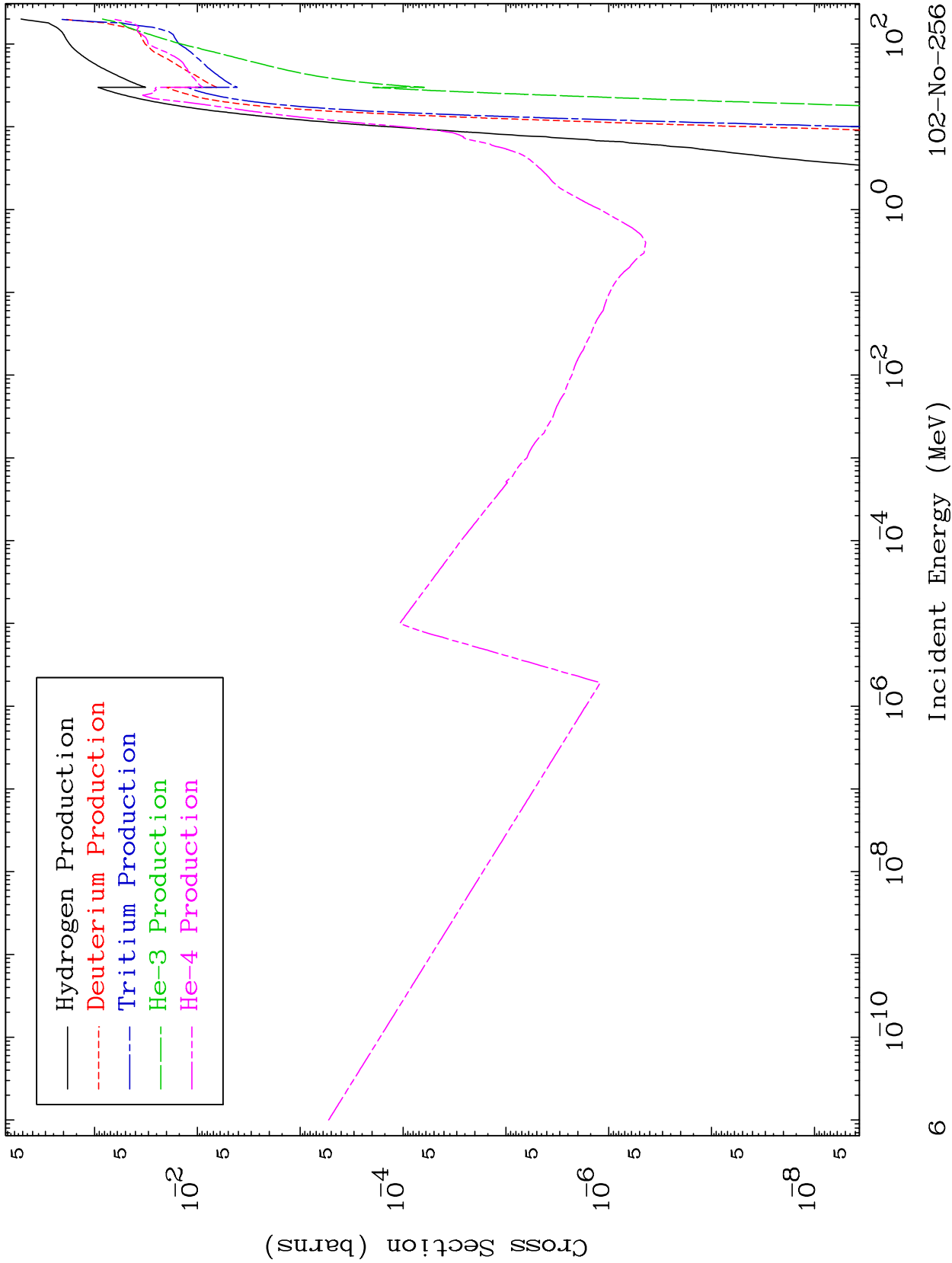


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Charged Particle
293 Kelvin Cross Sections

102-No-256

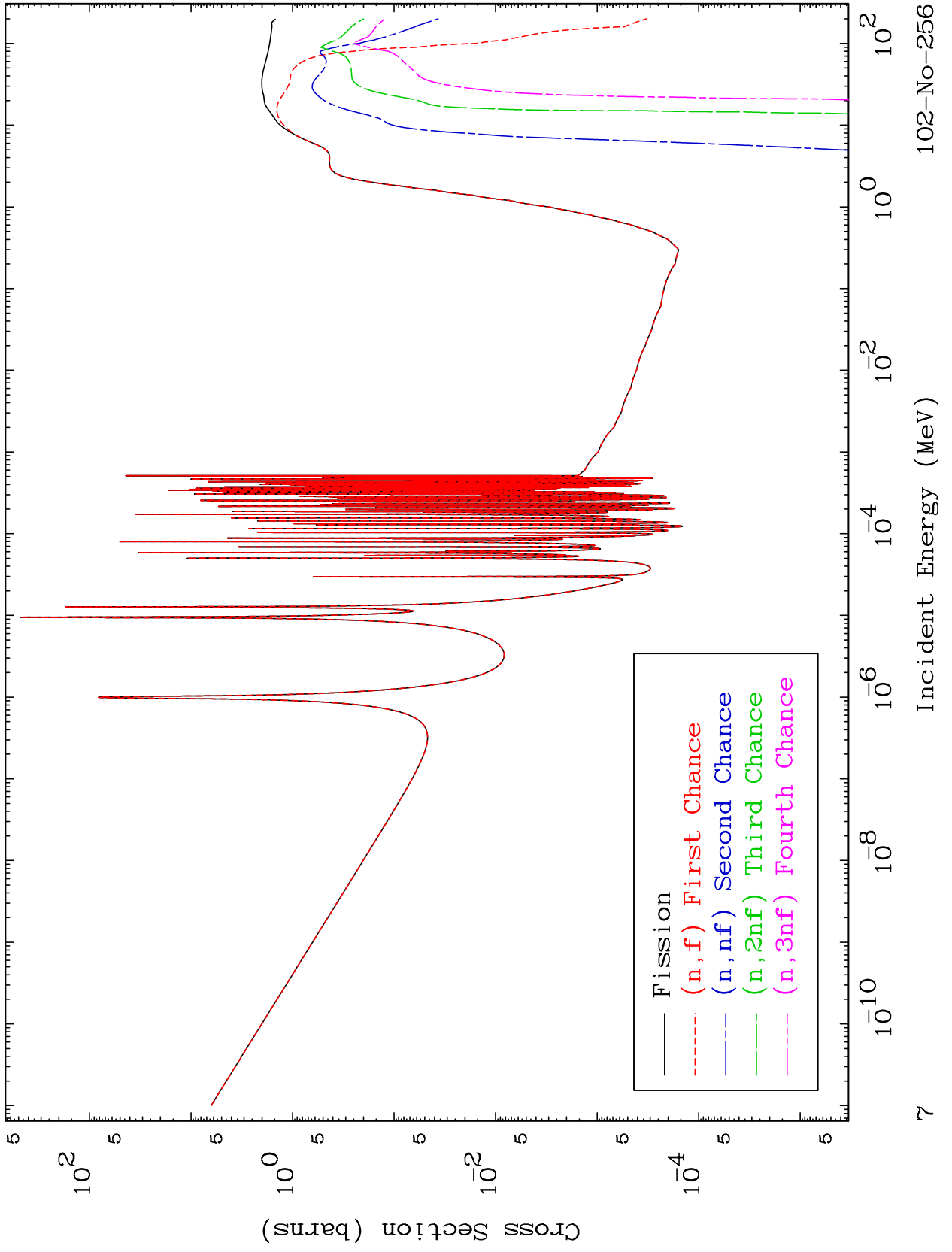




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Fission
293 Kelvin Cross Sections

102-No-256

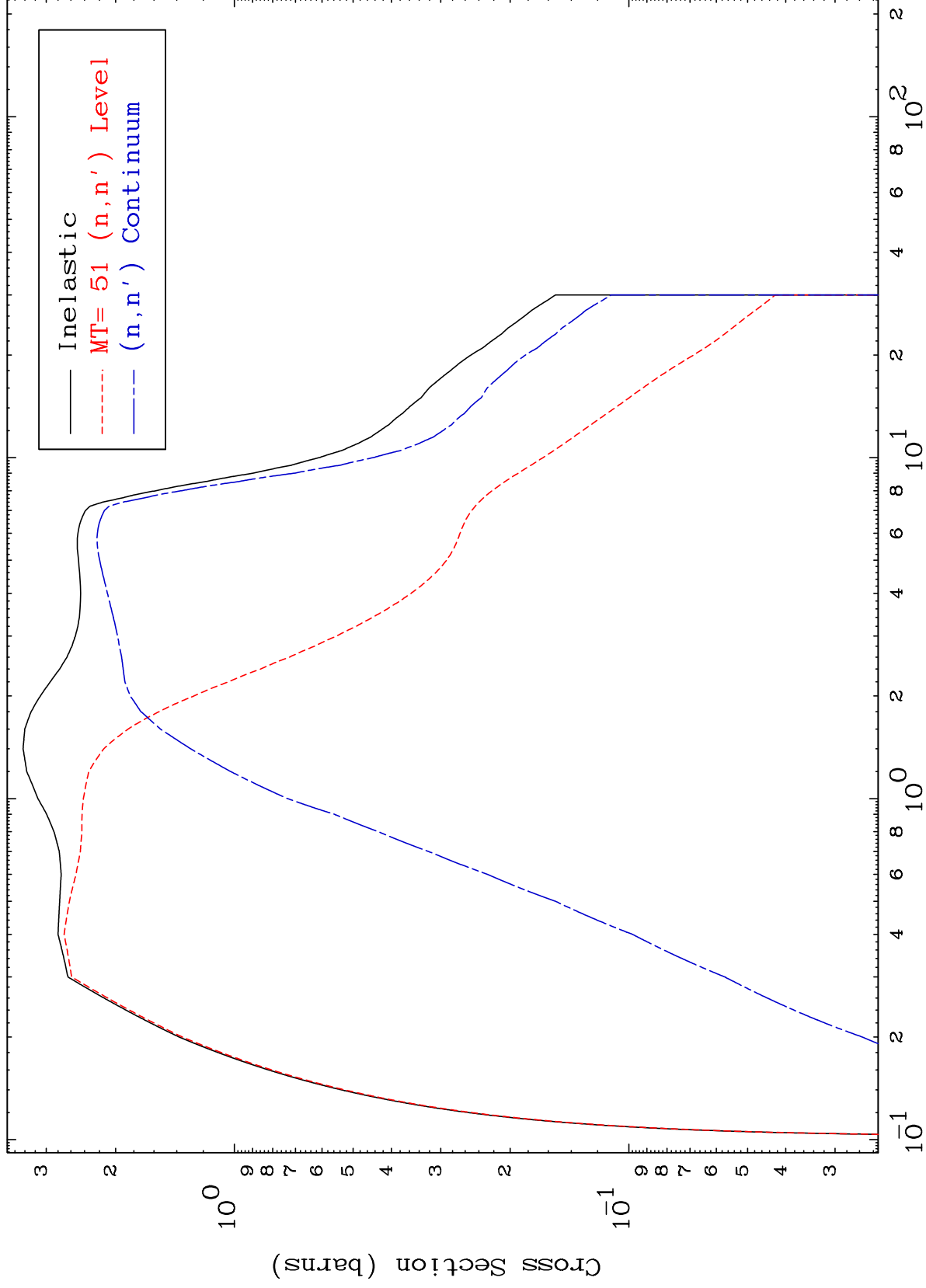


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(n,n') Level

102-No-256

293 Kelvin Cross Sections



Incident Energy (MeV)

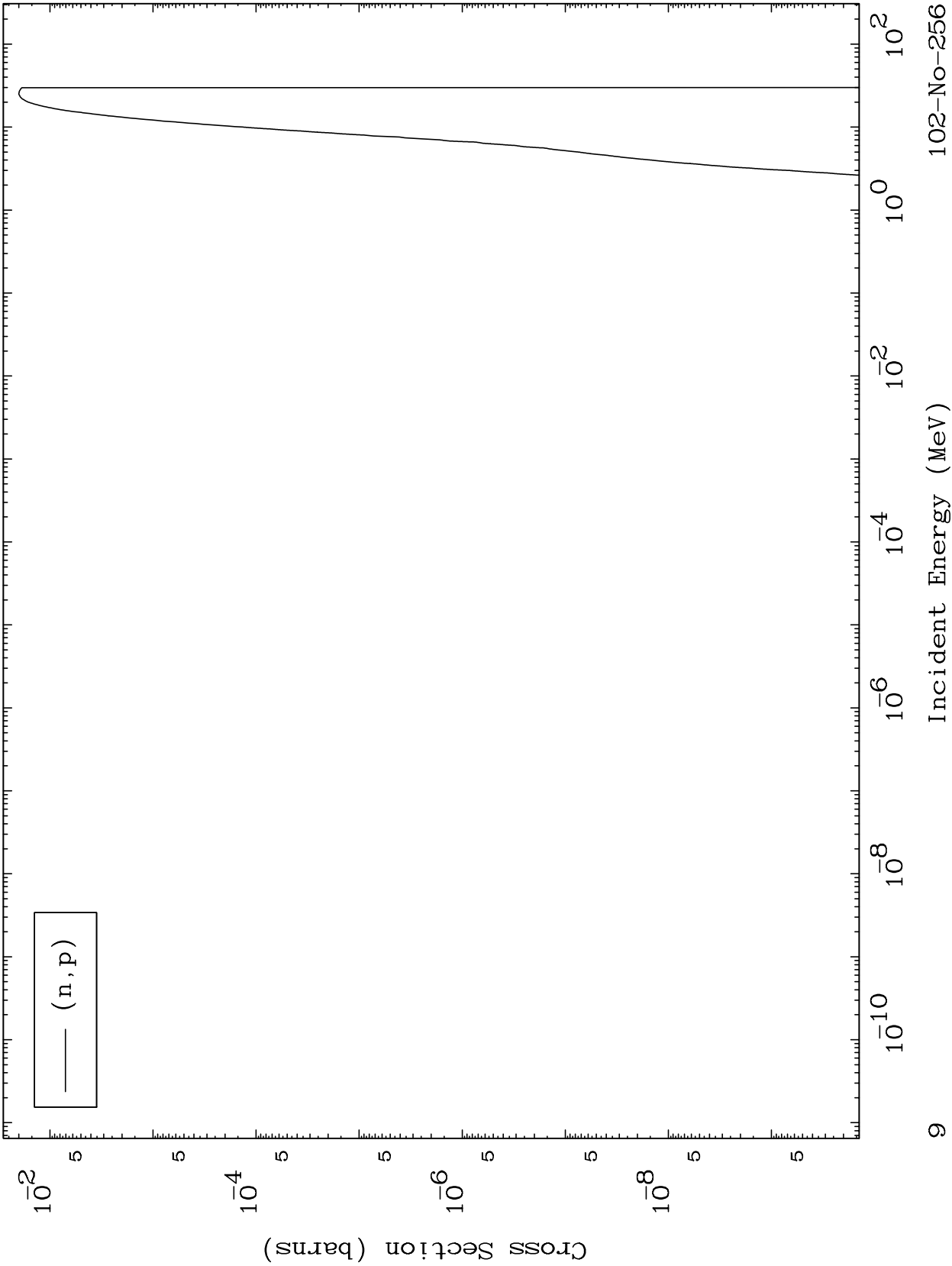
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8

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(n,p) Levels
293 Kelvin Cross Sections

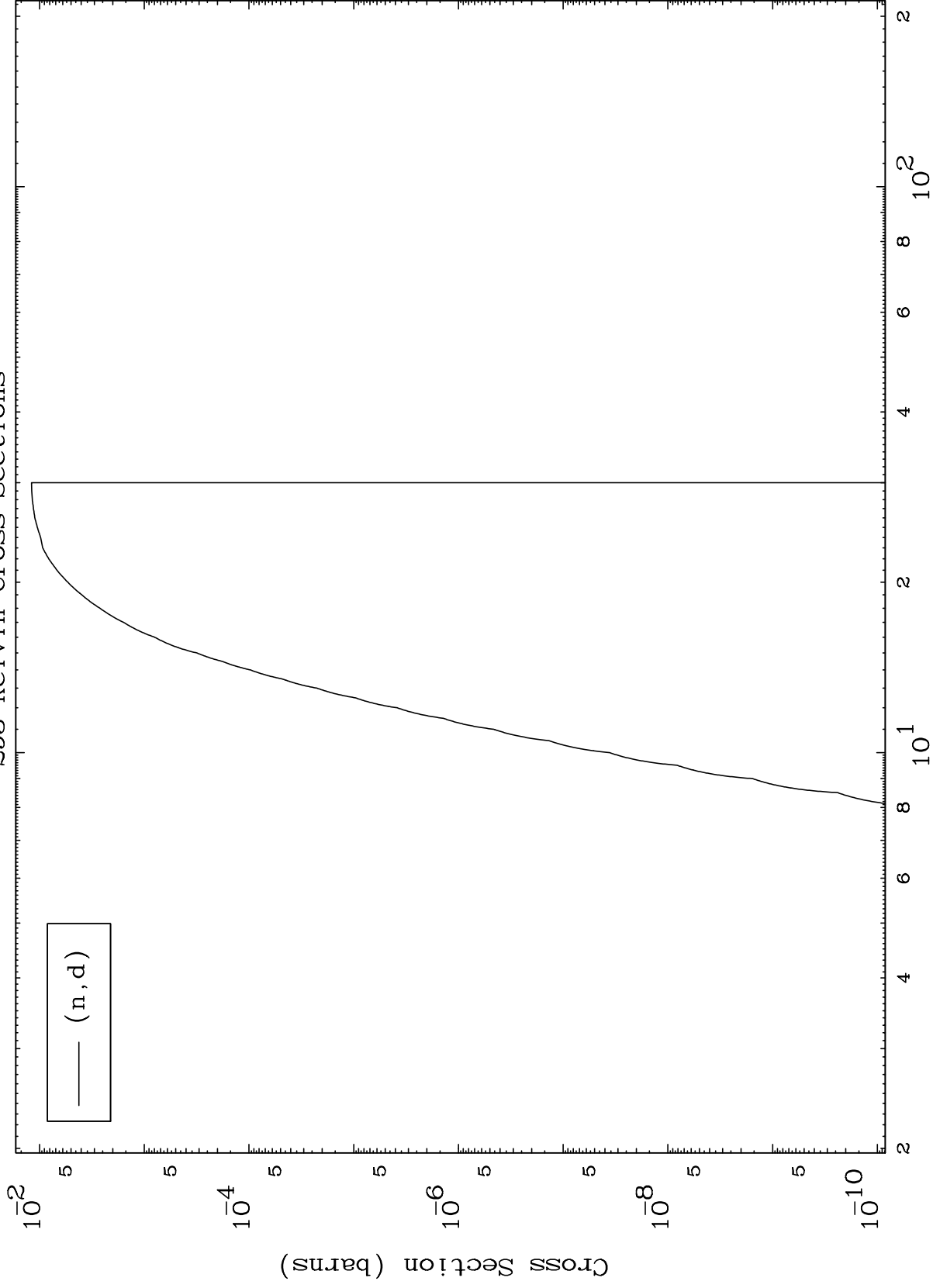
102-No-256



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(n,d) Levels
293 Kelvin Cross Sections

102-No-256



10

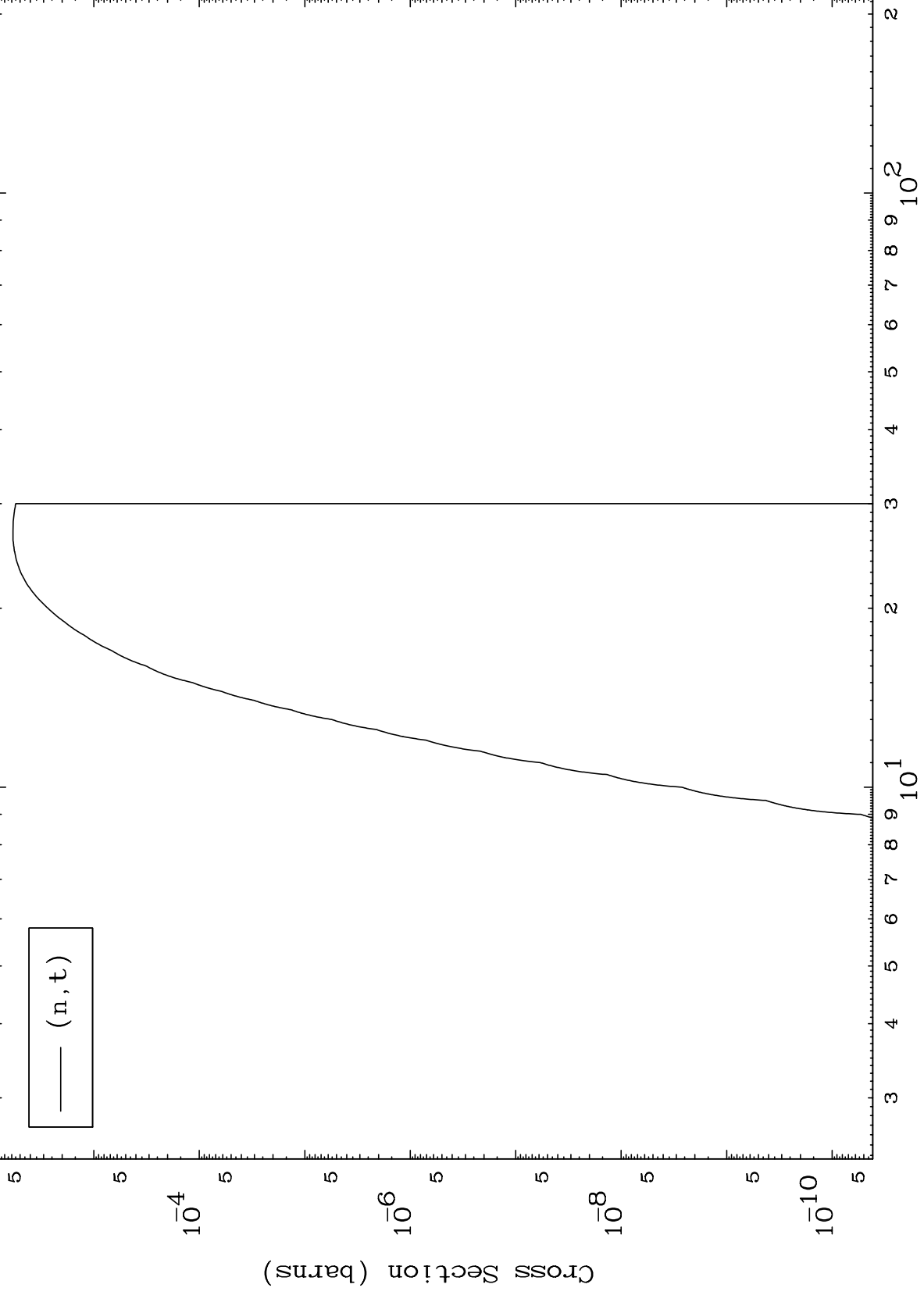
Incident Energy (MeV)

102-No-256

MAT 9974

(n,t) Levels
293 Kelvin Cross Sections

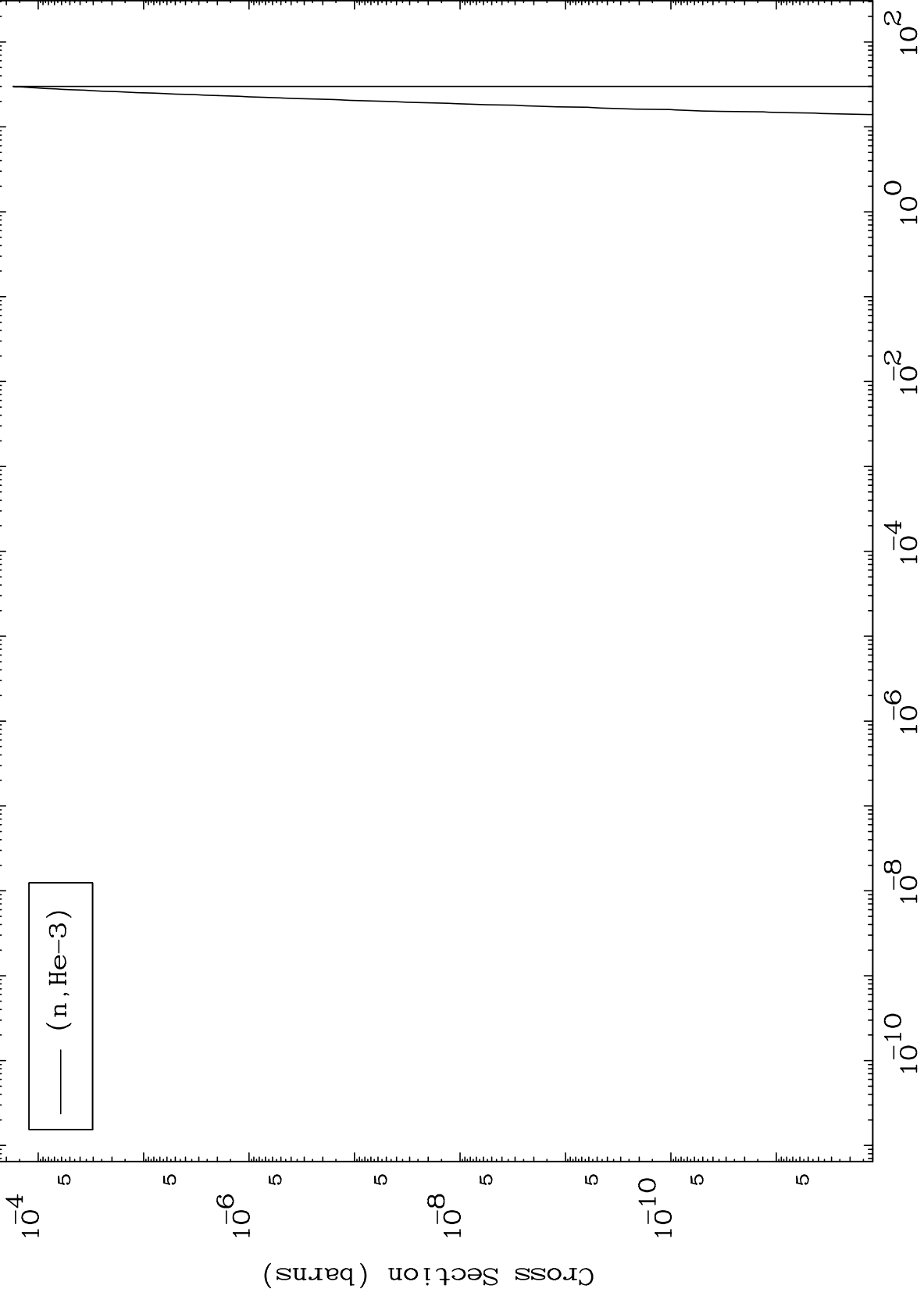
102-No-256



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(n,He3) Levels
293 Kelvin Cross Sections

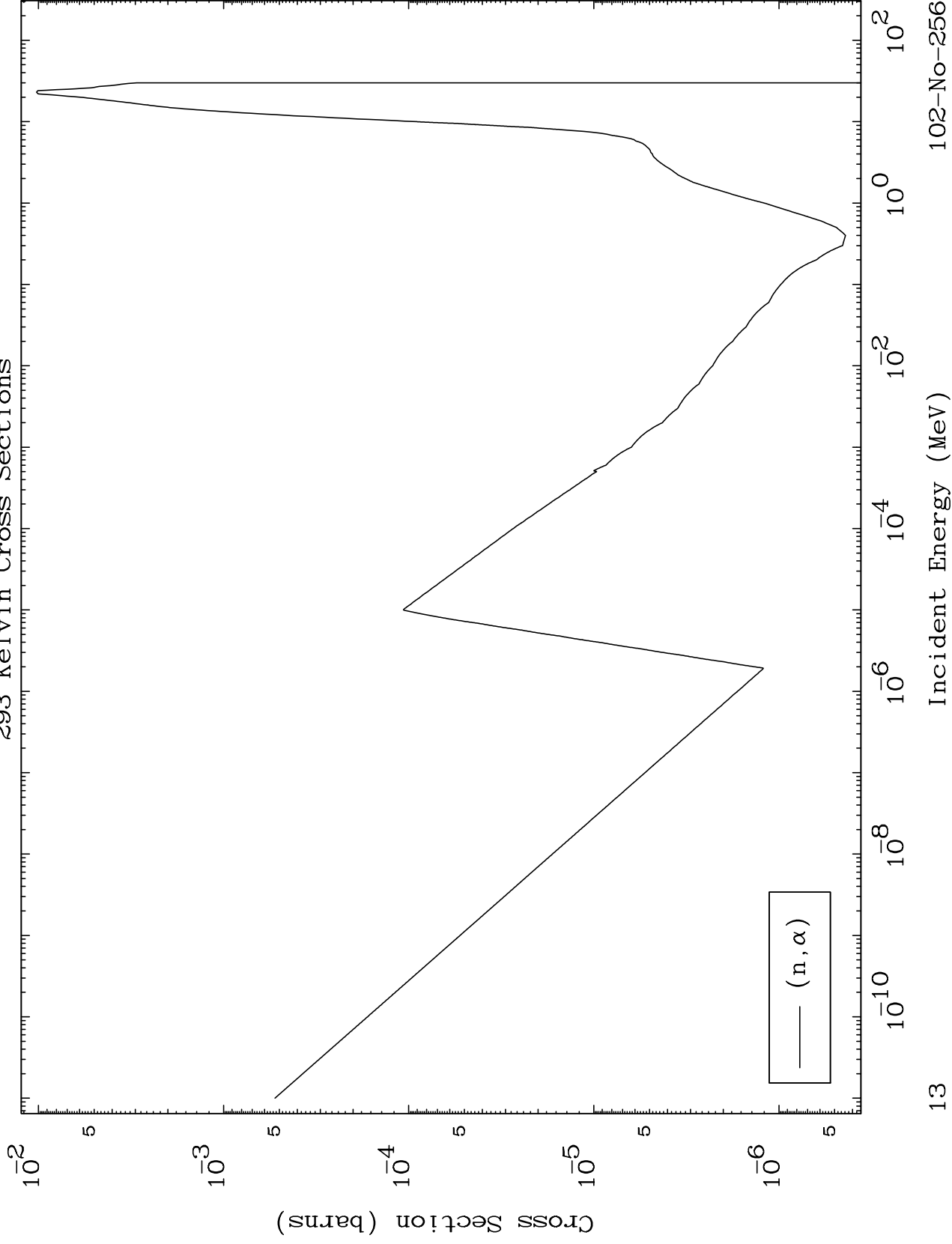
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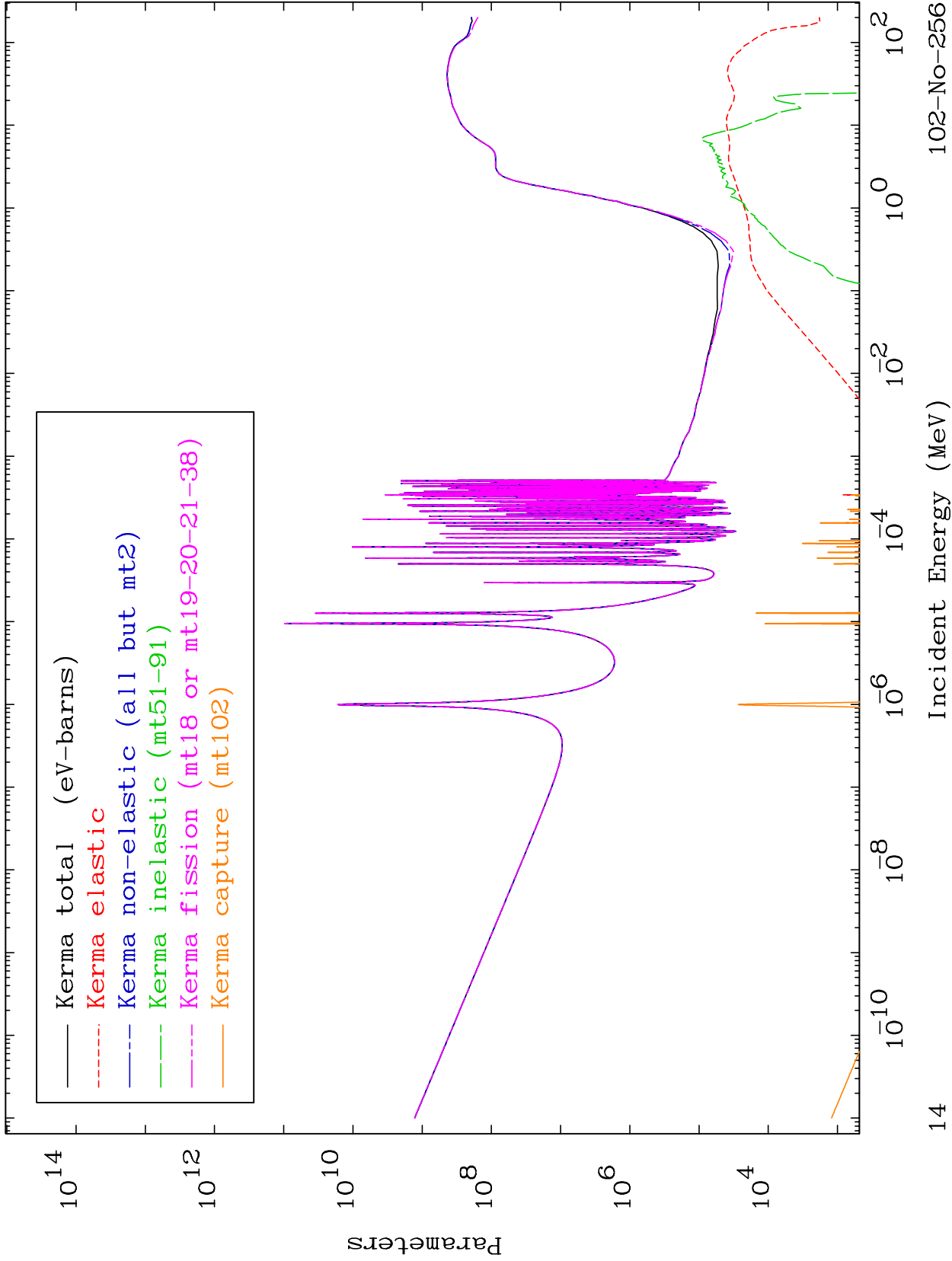


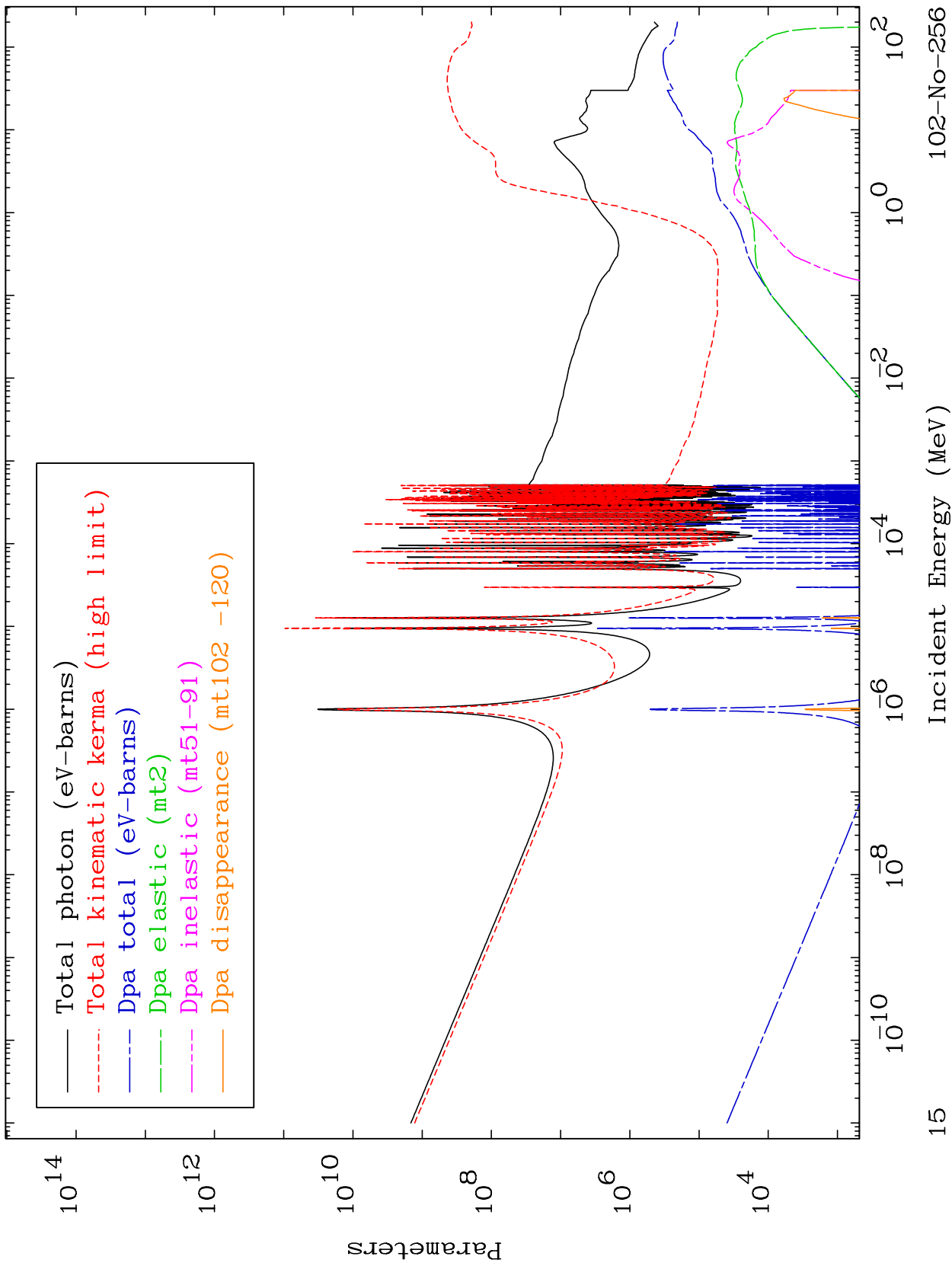
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(n, α) Levels
293 Kelvin Cross Sections

102-No-256



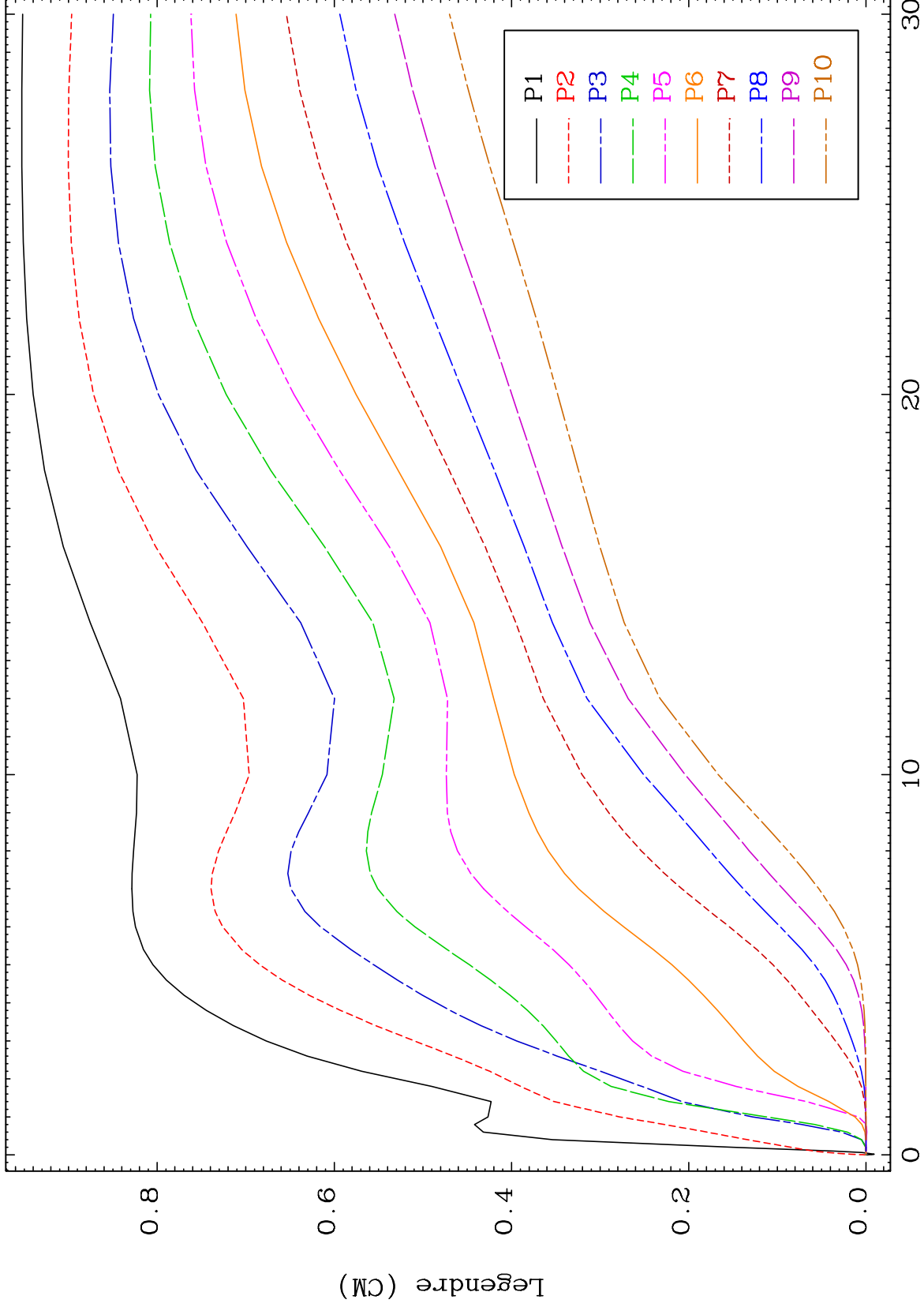




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Elastic Legendre Coefficients

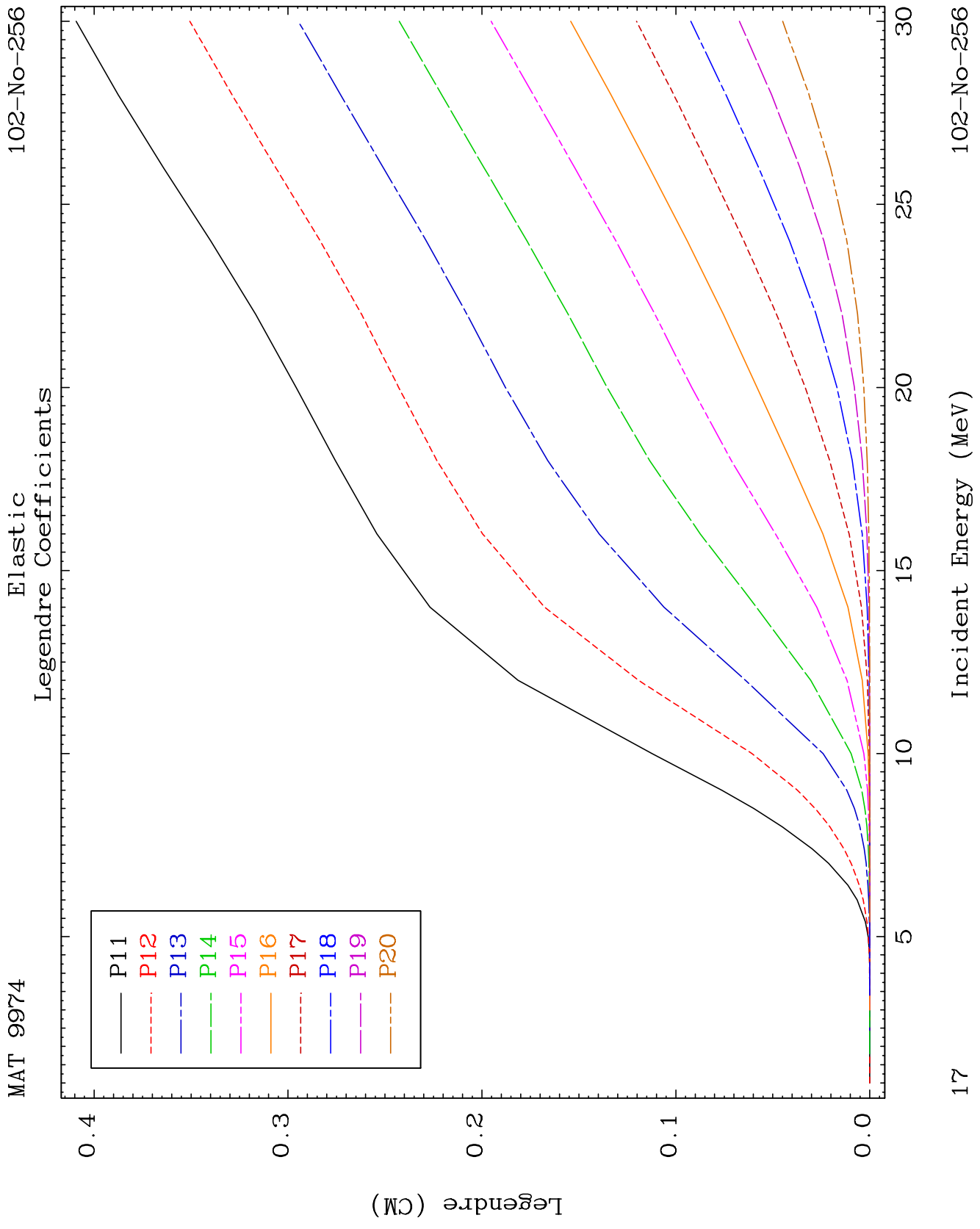
102-No-256



16

Incident Energy (MeV)

102-No-256



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Elastic Legendre Coefficients

102-No-256

17

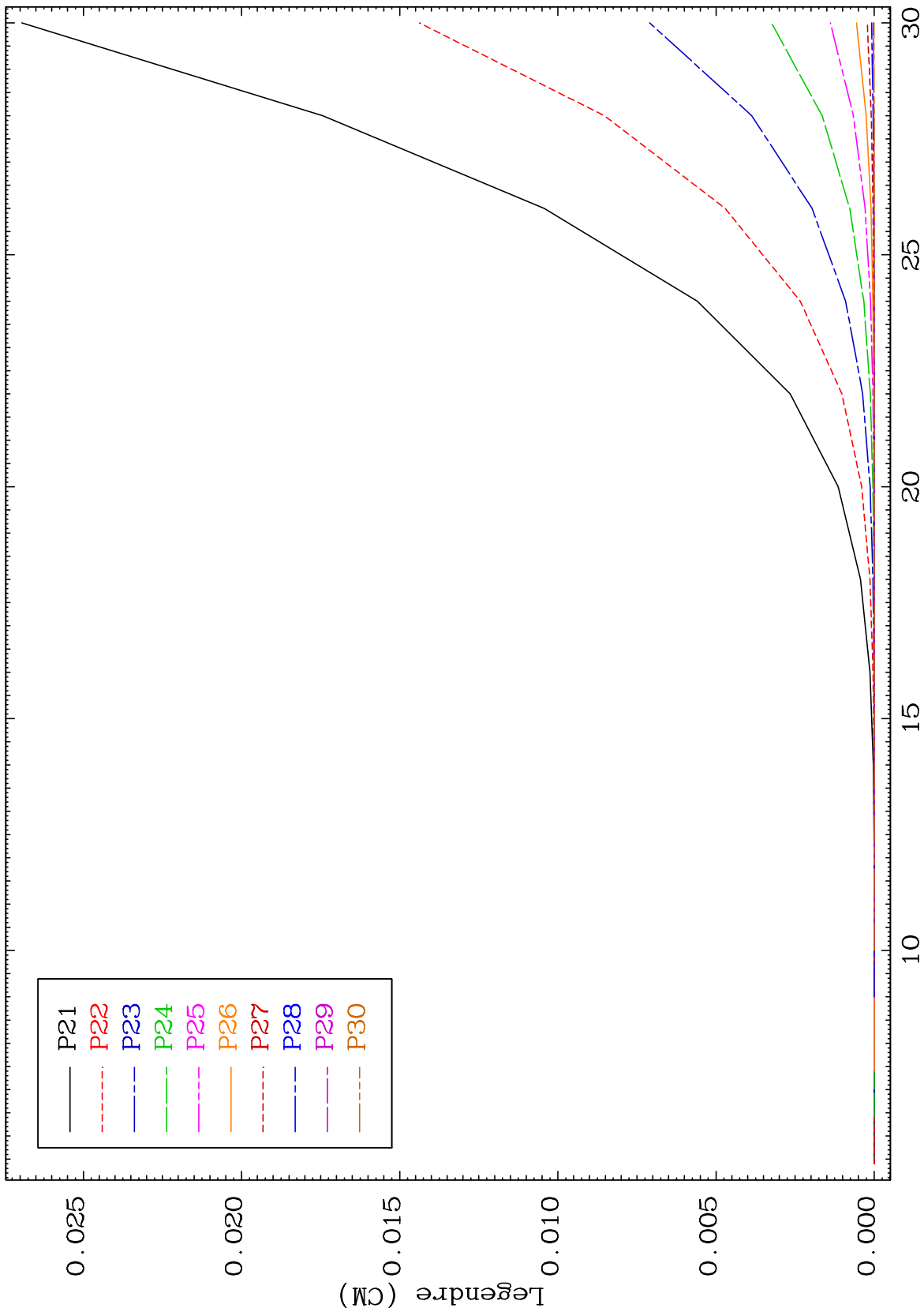
Incident Energy (MeV)

102-No-256

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Elastic Legendre Coefficients

102-No-256



18

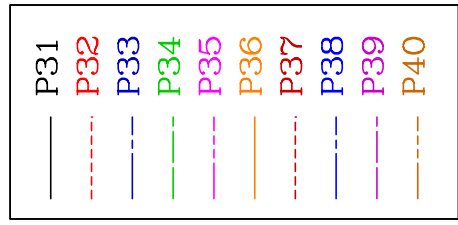
Incident Energy (MeV)

102-No-256

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Elastic
Legendre Coefficients

102-No-256



$\times 10^{-6}$

3

Legendre (CM)

2

1

0

15

20

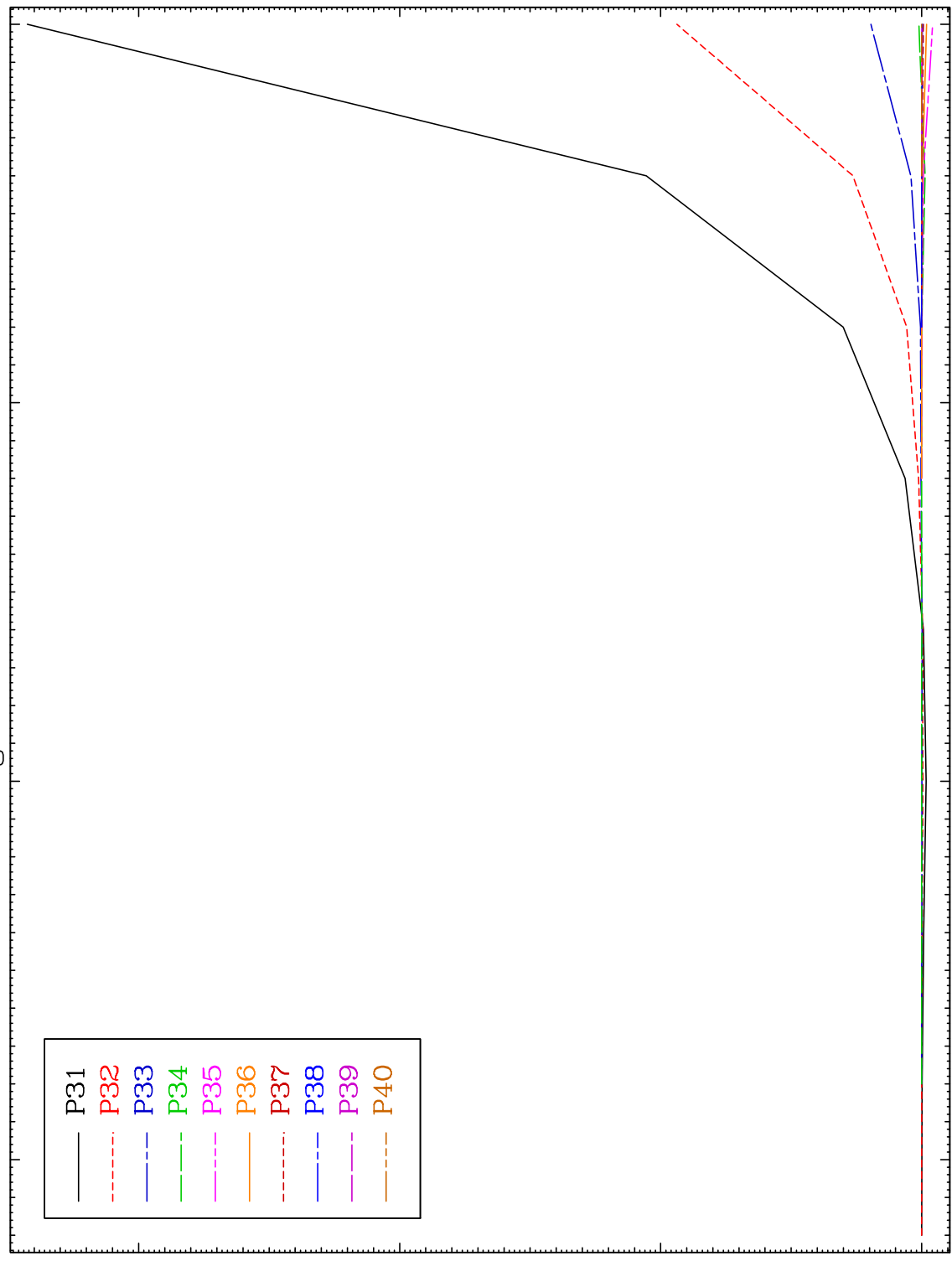
25

30

19

Incident Energy (MeV)

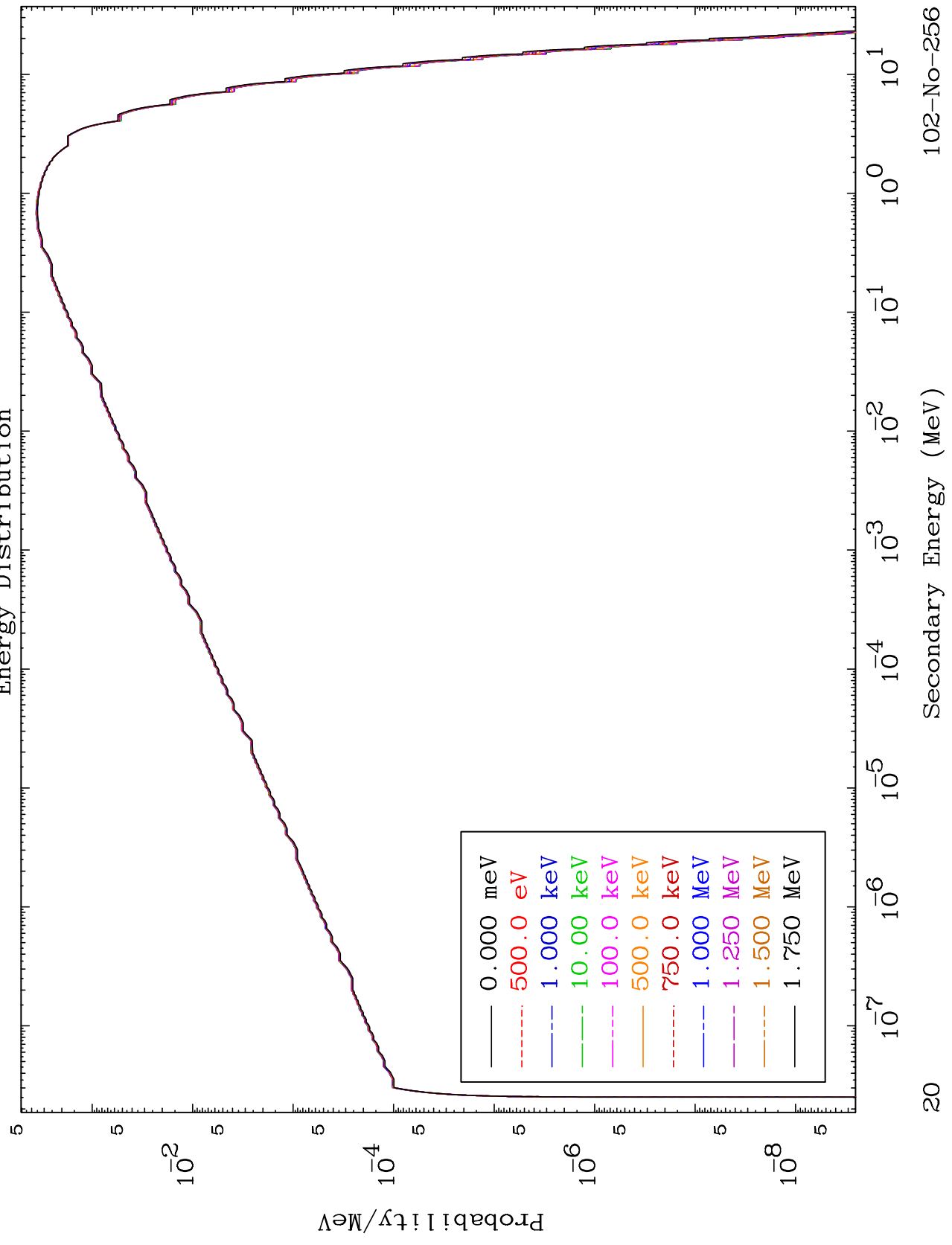
102-No-256



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Fission Energy Distribution

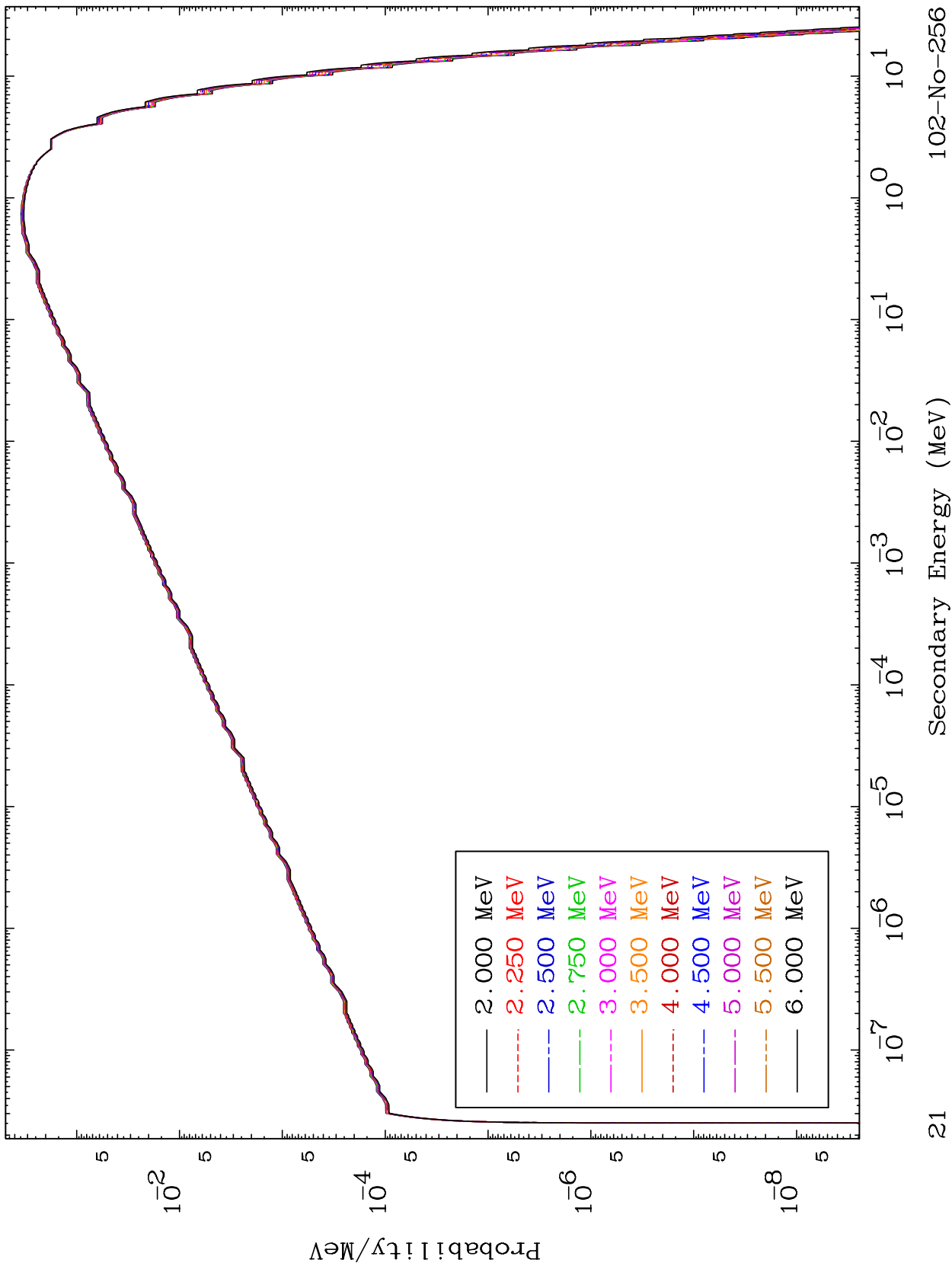
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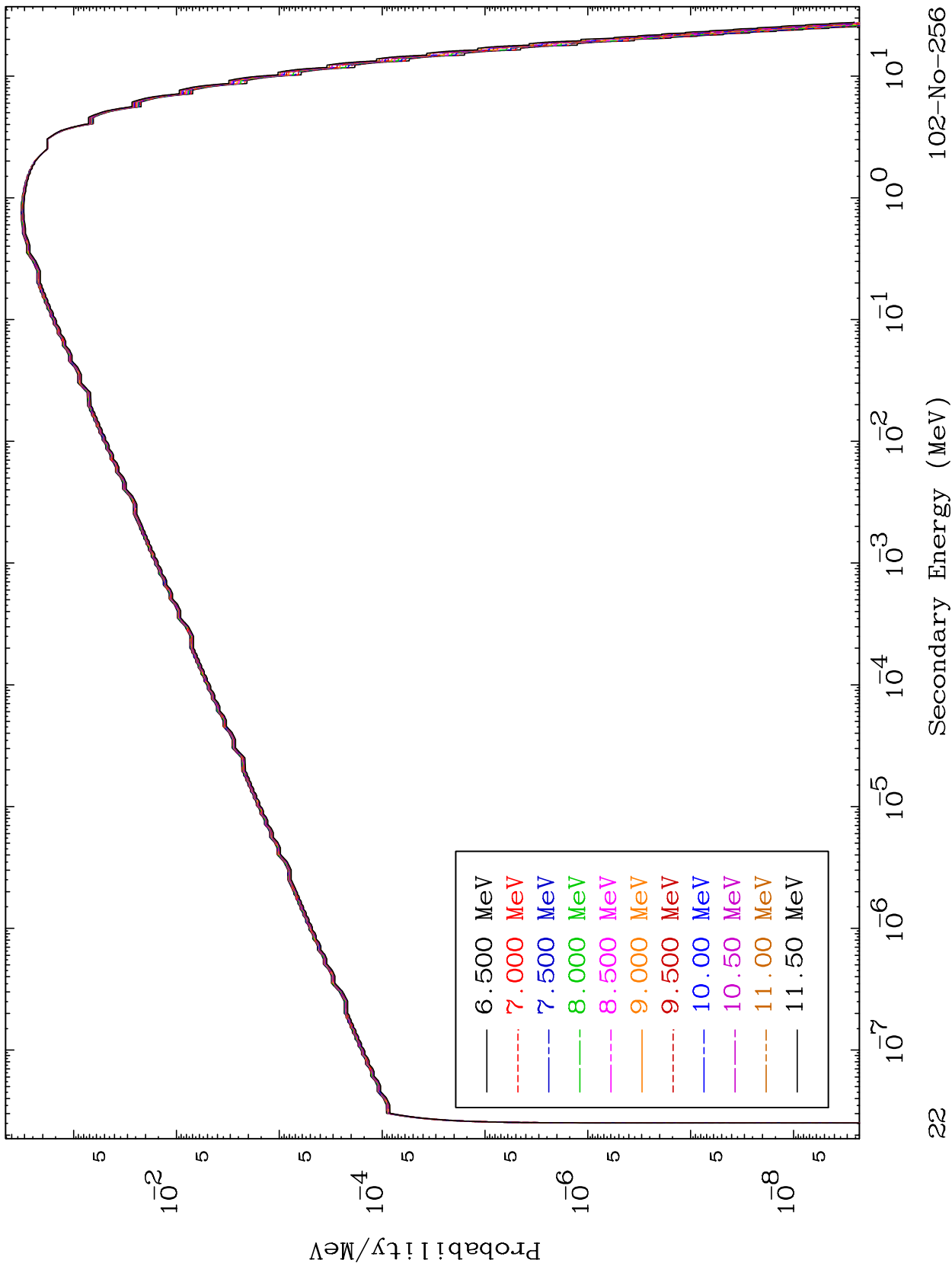
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Fission Energy Distribution

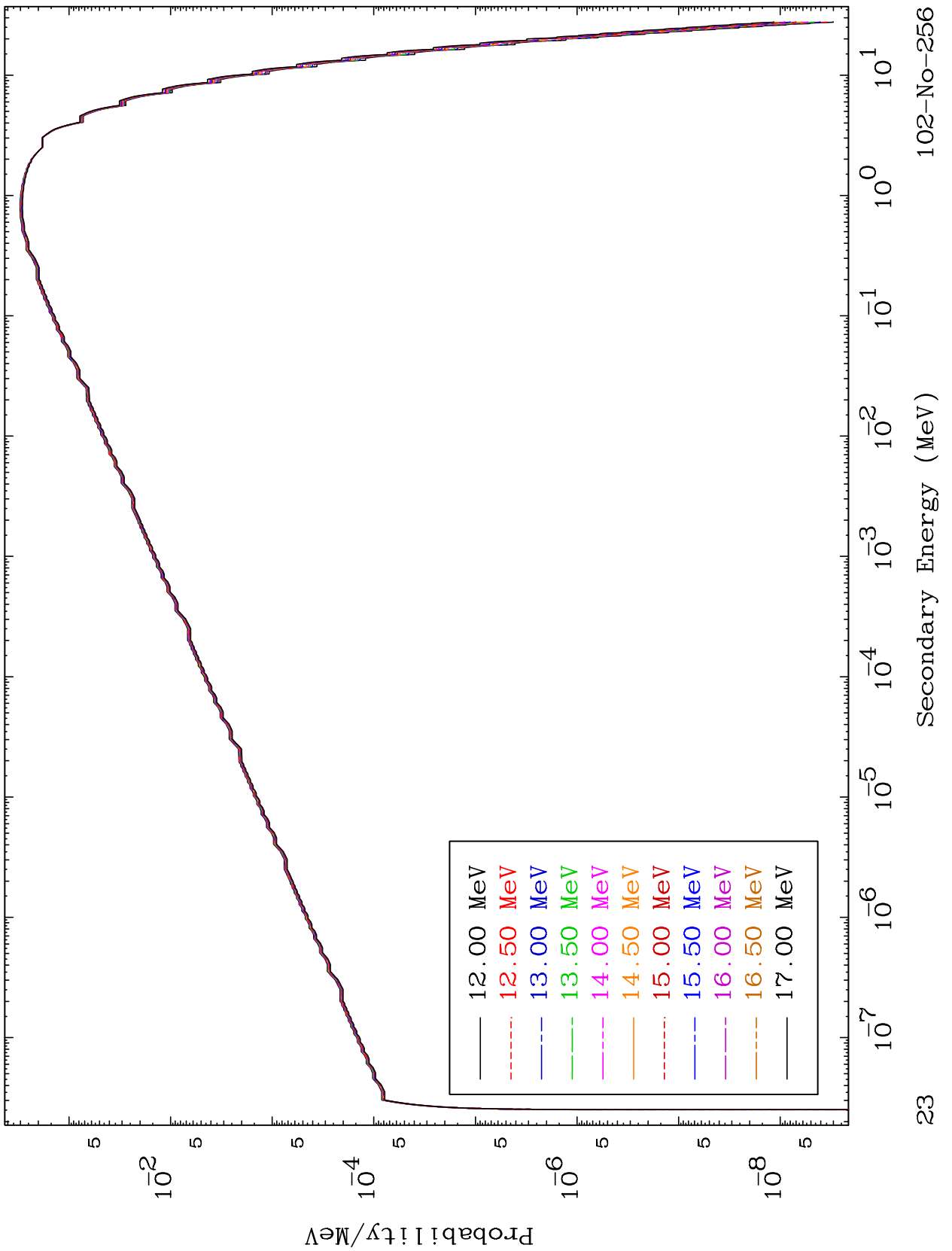
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Fission
Energy Distribution



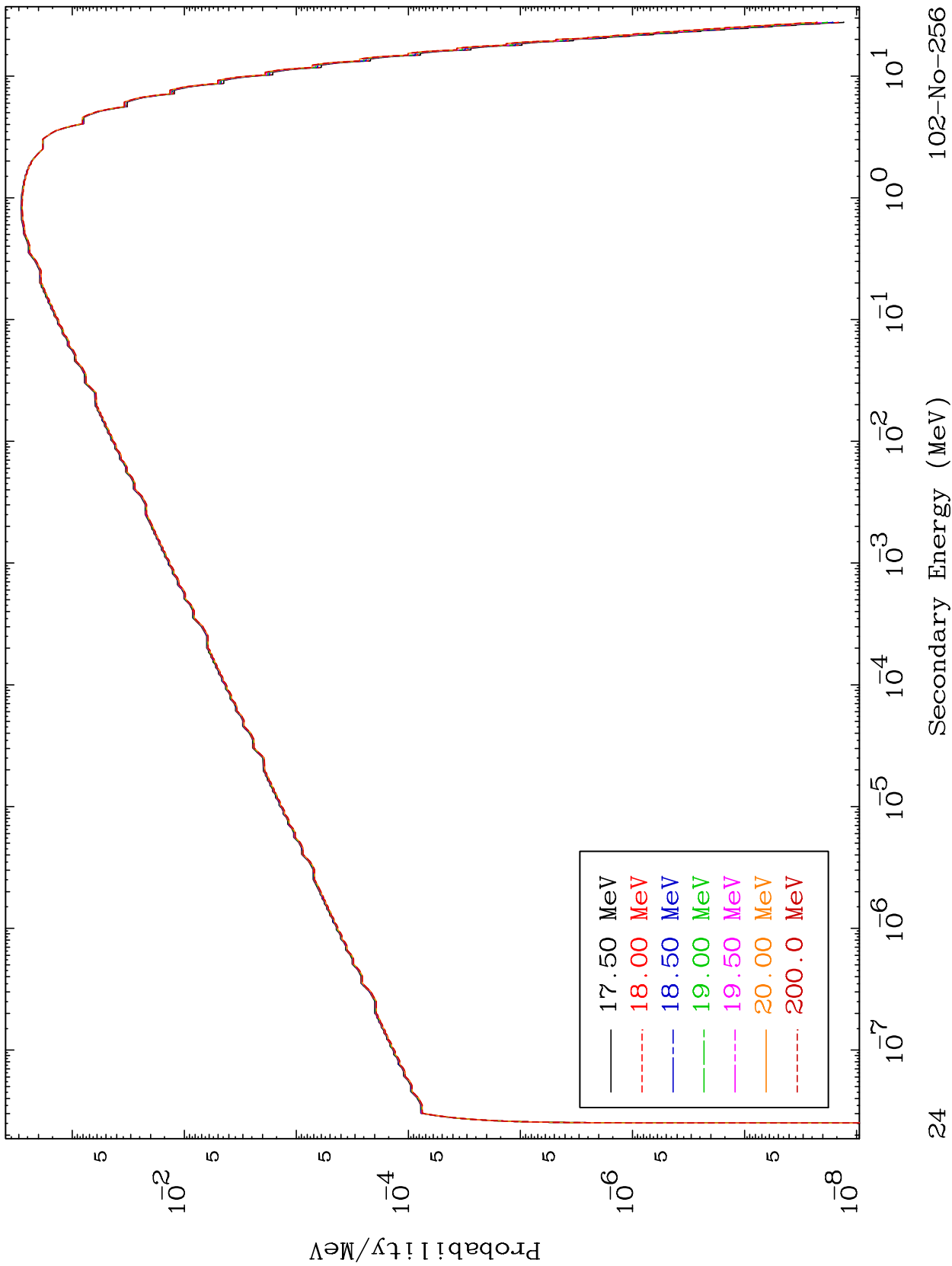
Fission
Energy Distribution



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Fission Energy Distribution

102-No-256

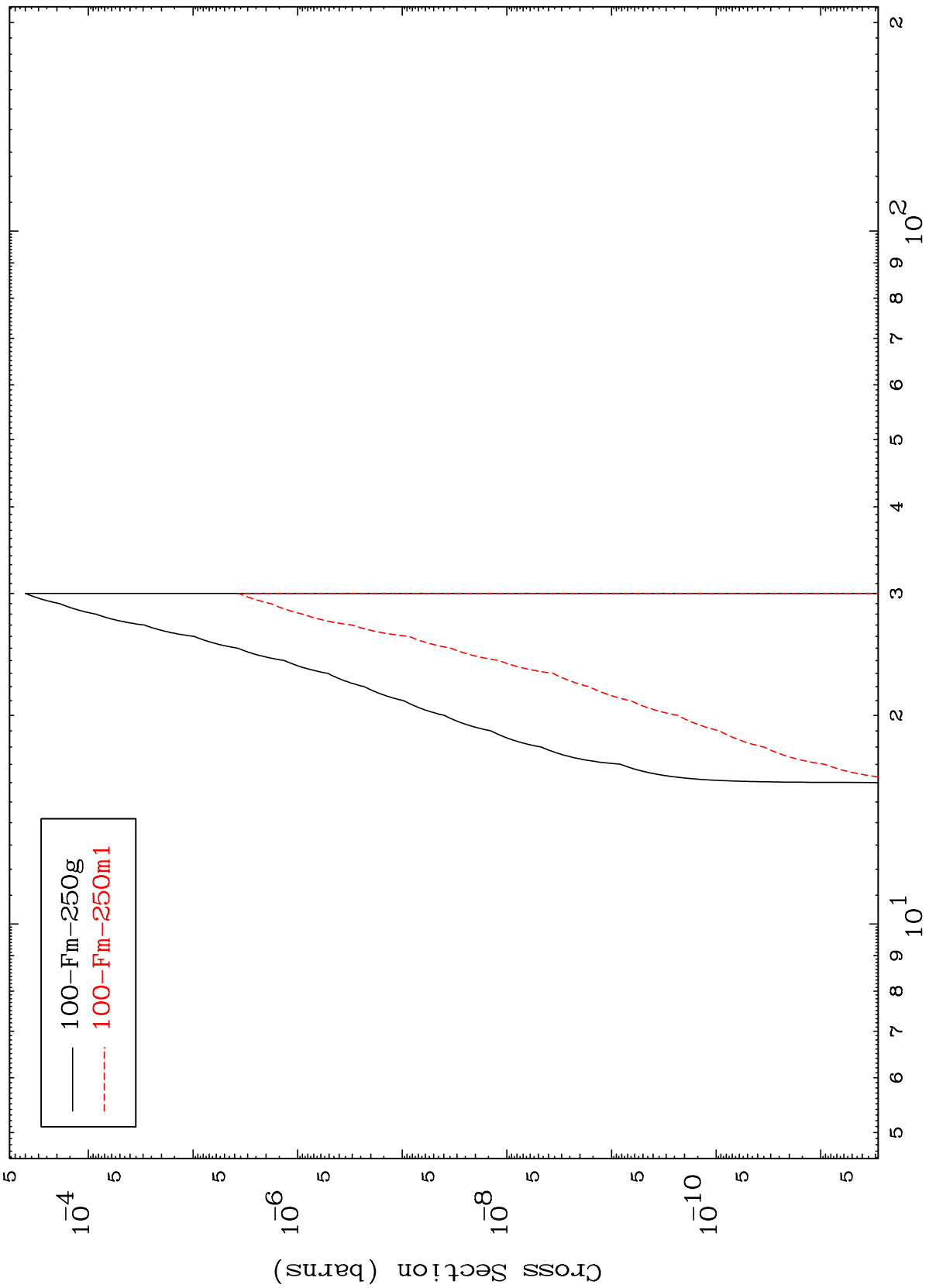


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(n,3n) α

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

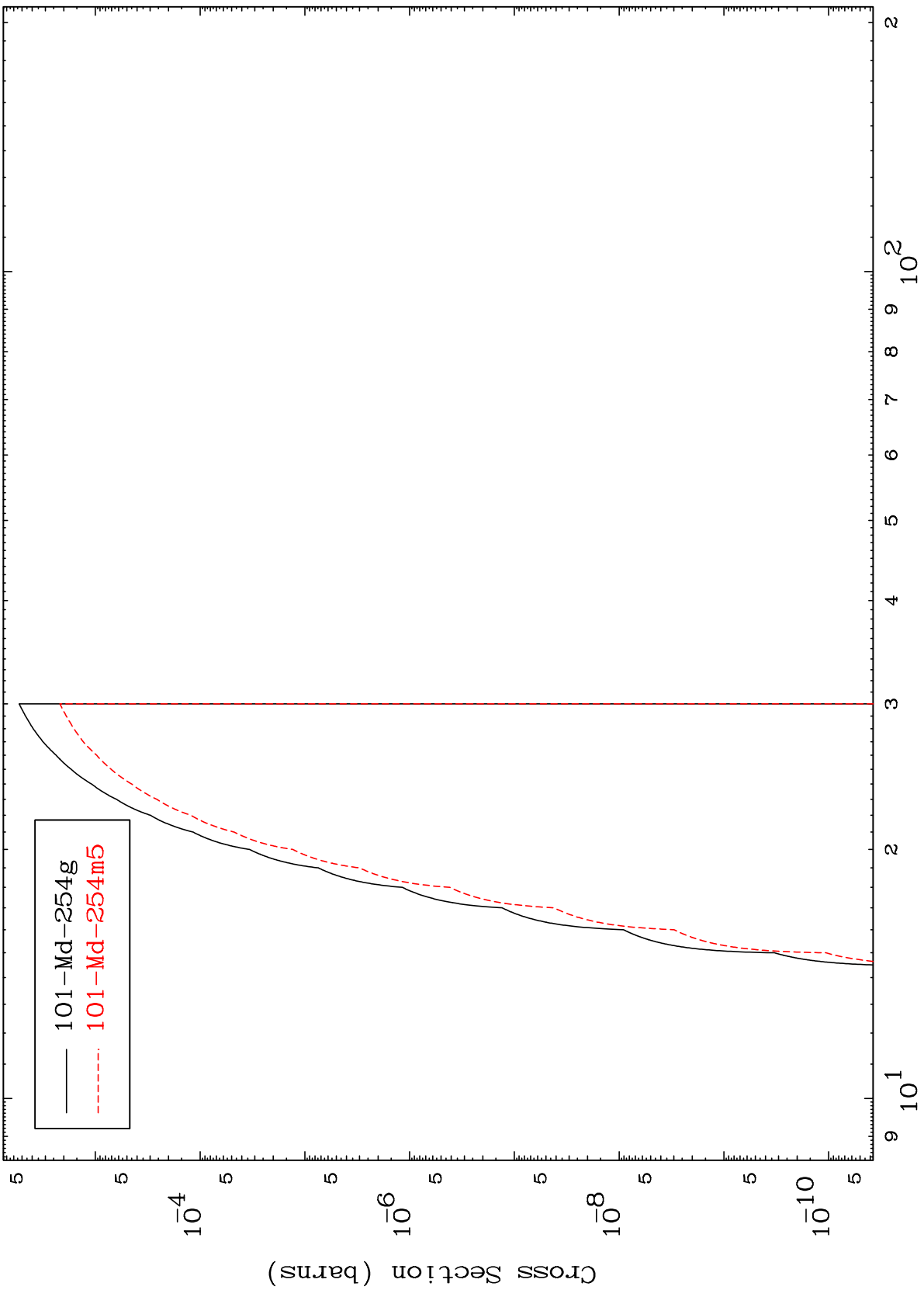
102-No-256

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(n,n') d

102-No-256

Radionuclide Production Cross Section



26

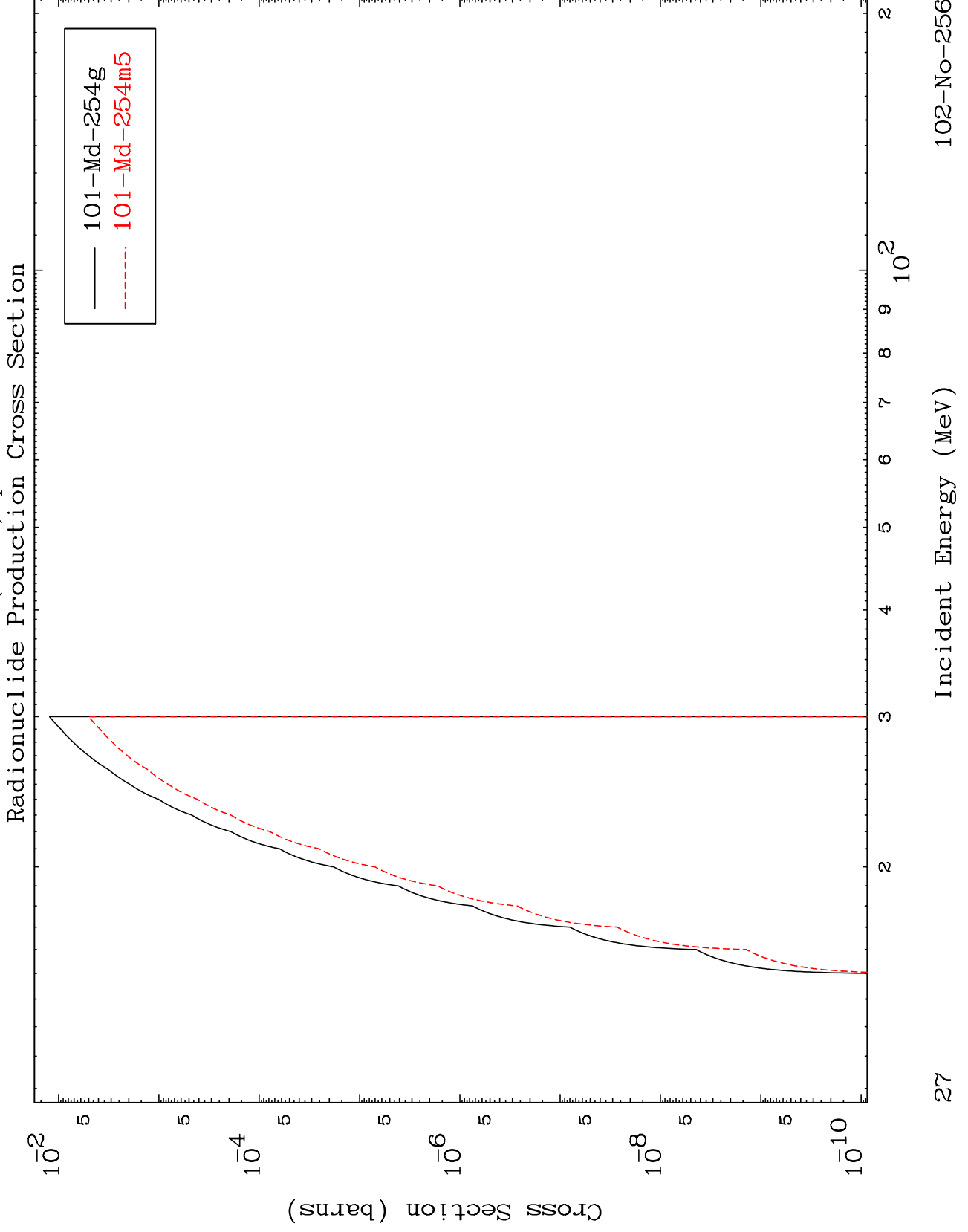
Incident Energy (MeV)

102-No-256

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(n,2n) p

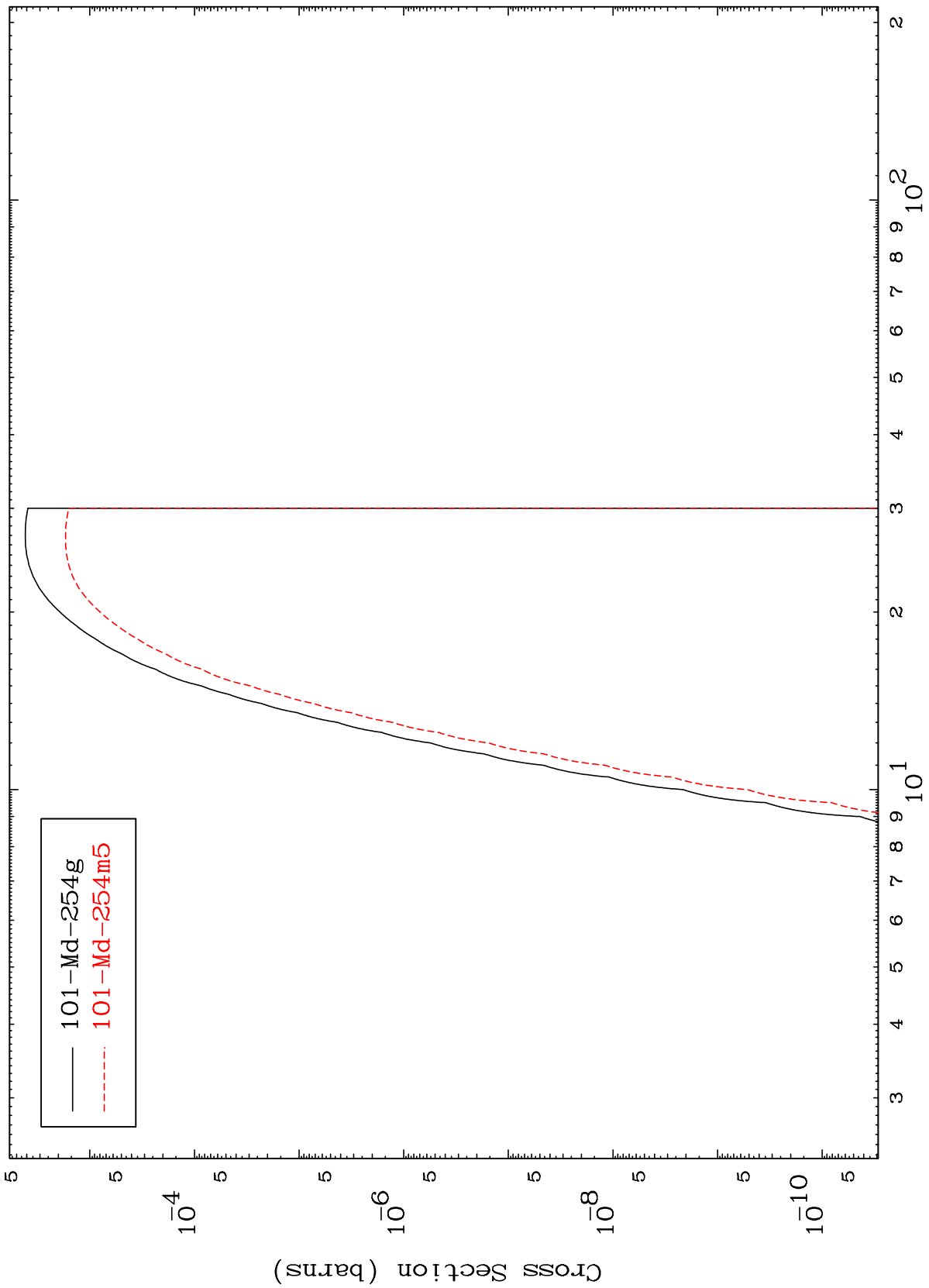
102-No-256



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102-No-256

(n, t)
Radionuclide Production Cross Section



28

Incident Energy (MeV)

102-No-256