

Program EVALPLOT
(Version 2017-1)

by

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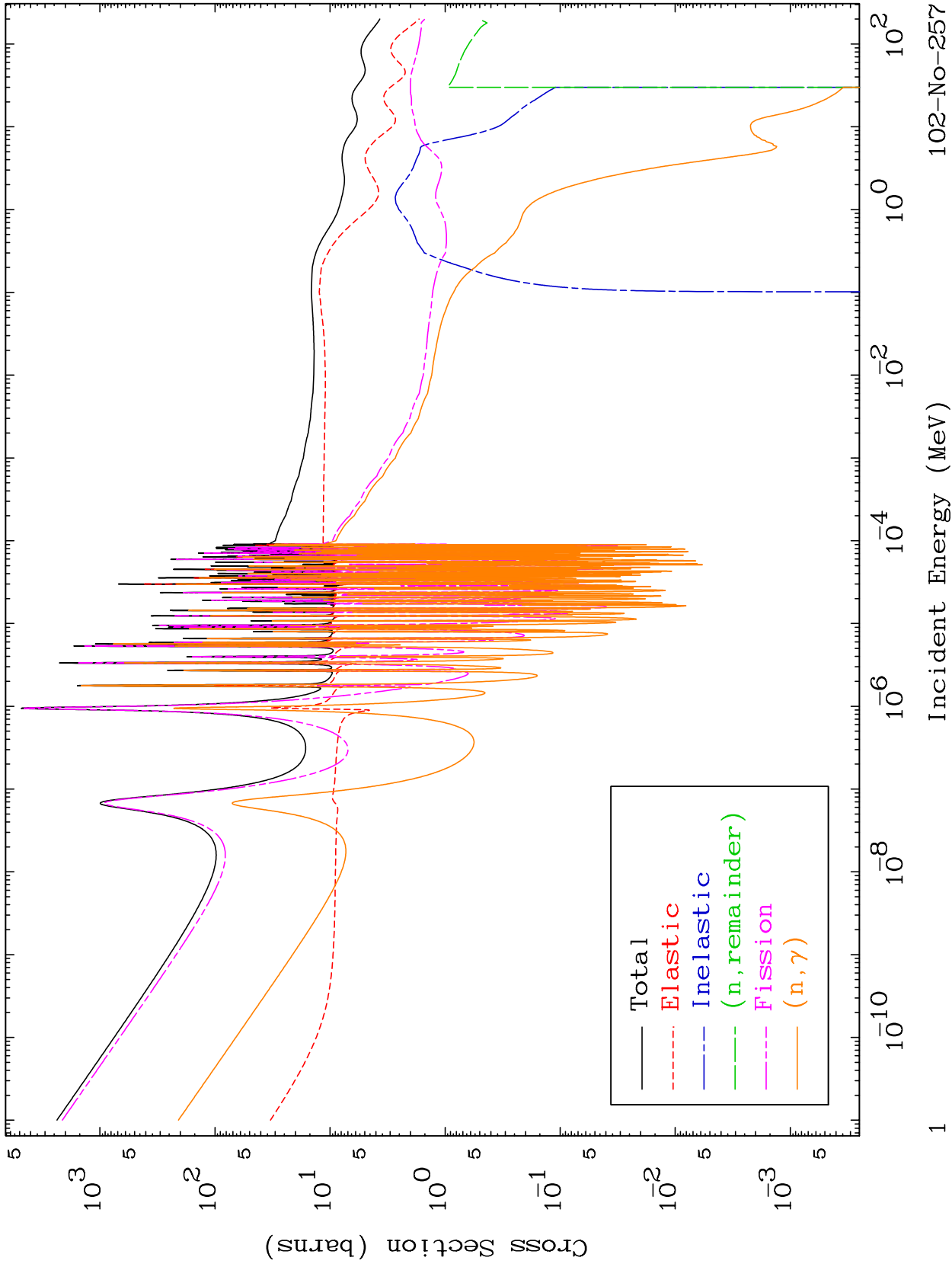
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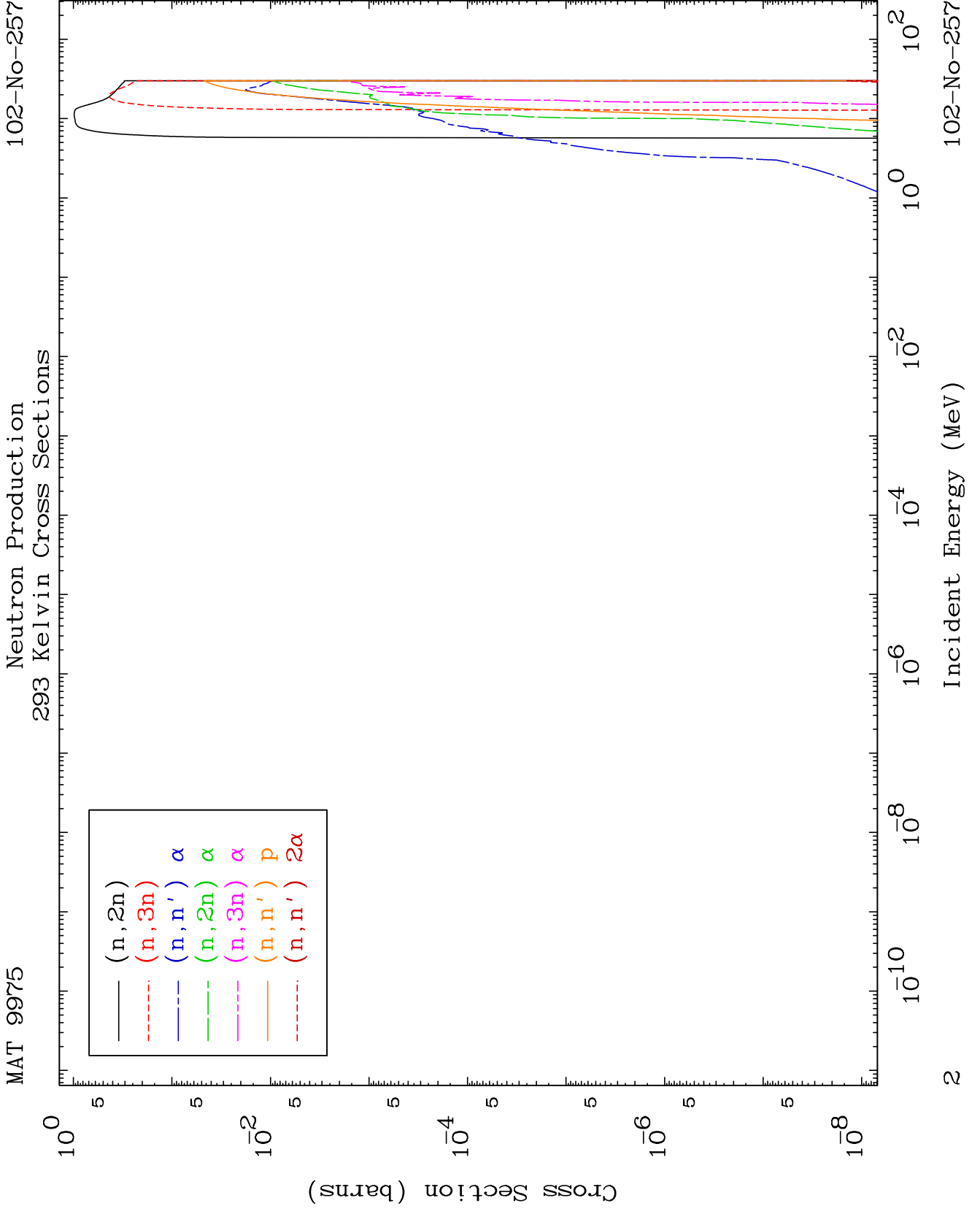
MAT 9975

Major

293 Kelvin Cross Sections

102-No-257

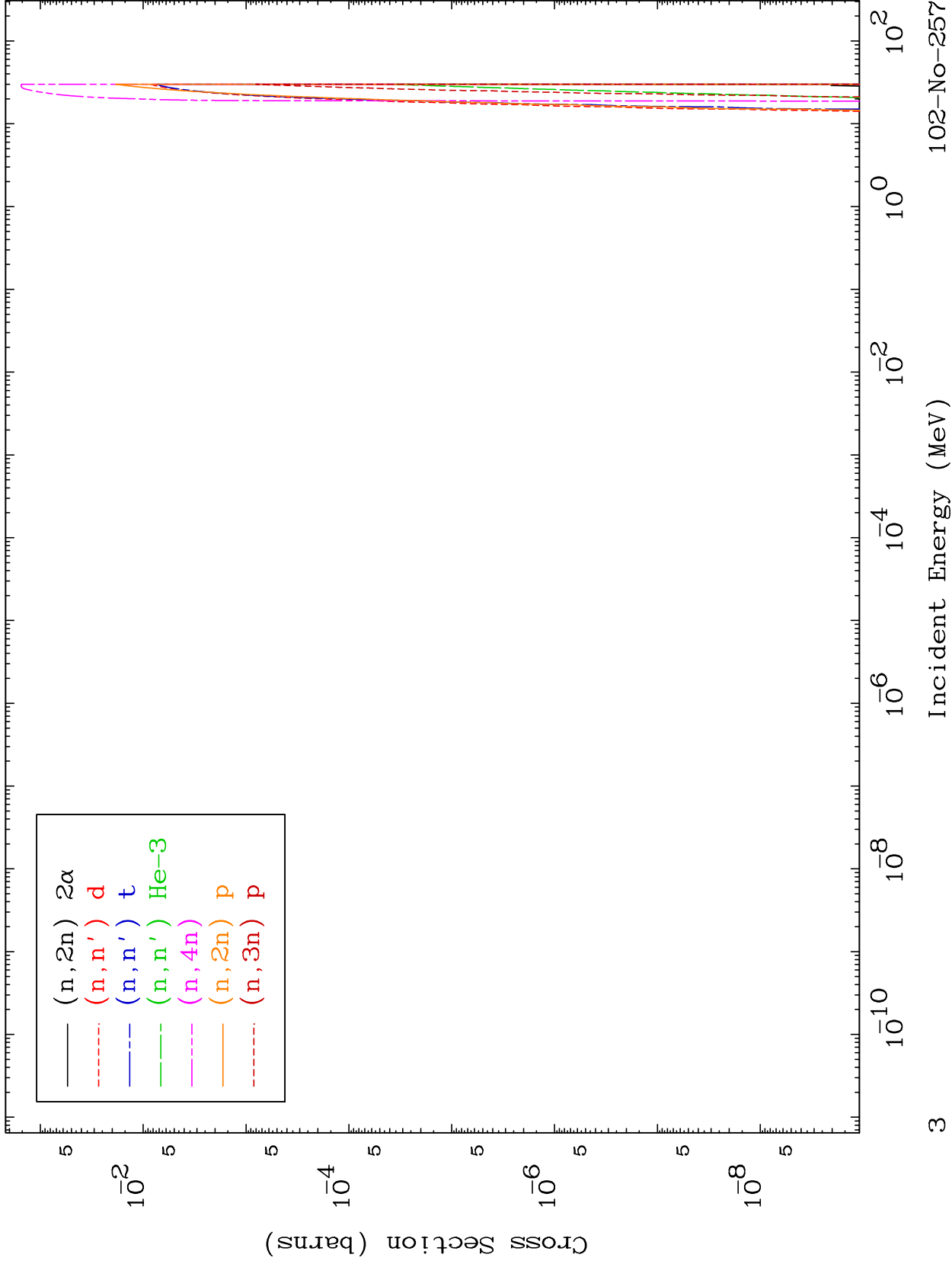




MAT 9975

Neutron Production
293 Kelvin Cross Sections

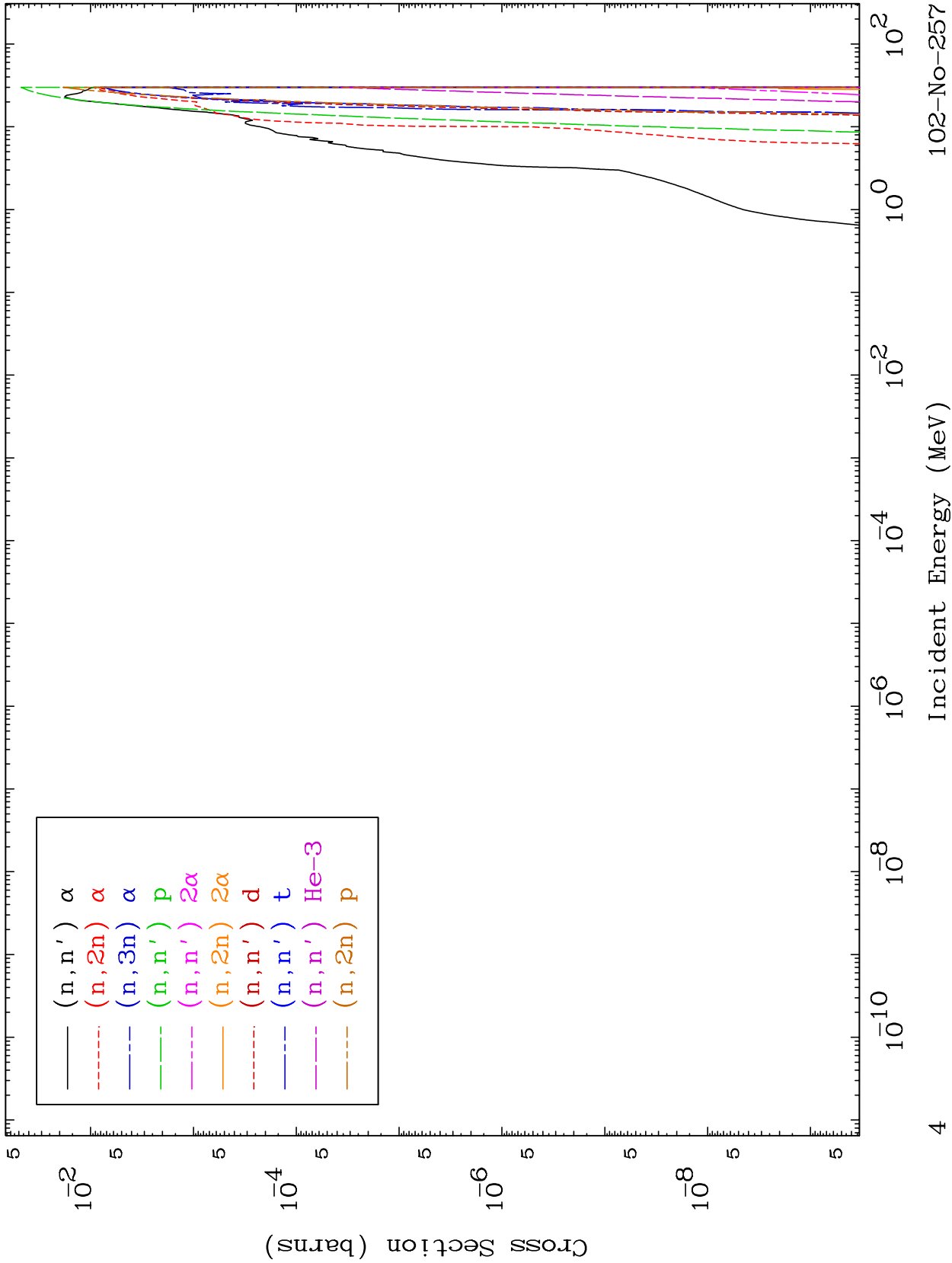
102-No-257



MAT 9975

Charged Particle
293 Kelvin Cross Sections

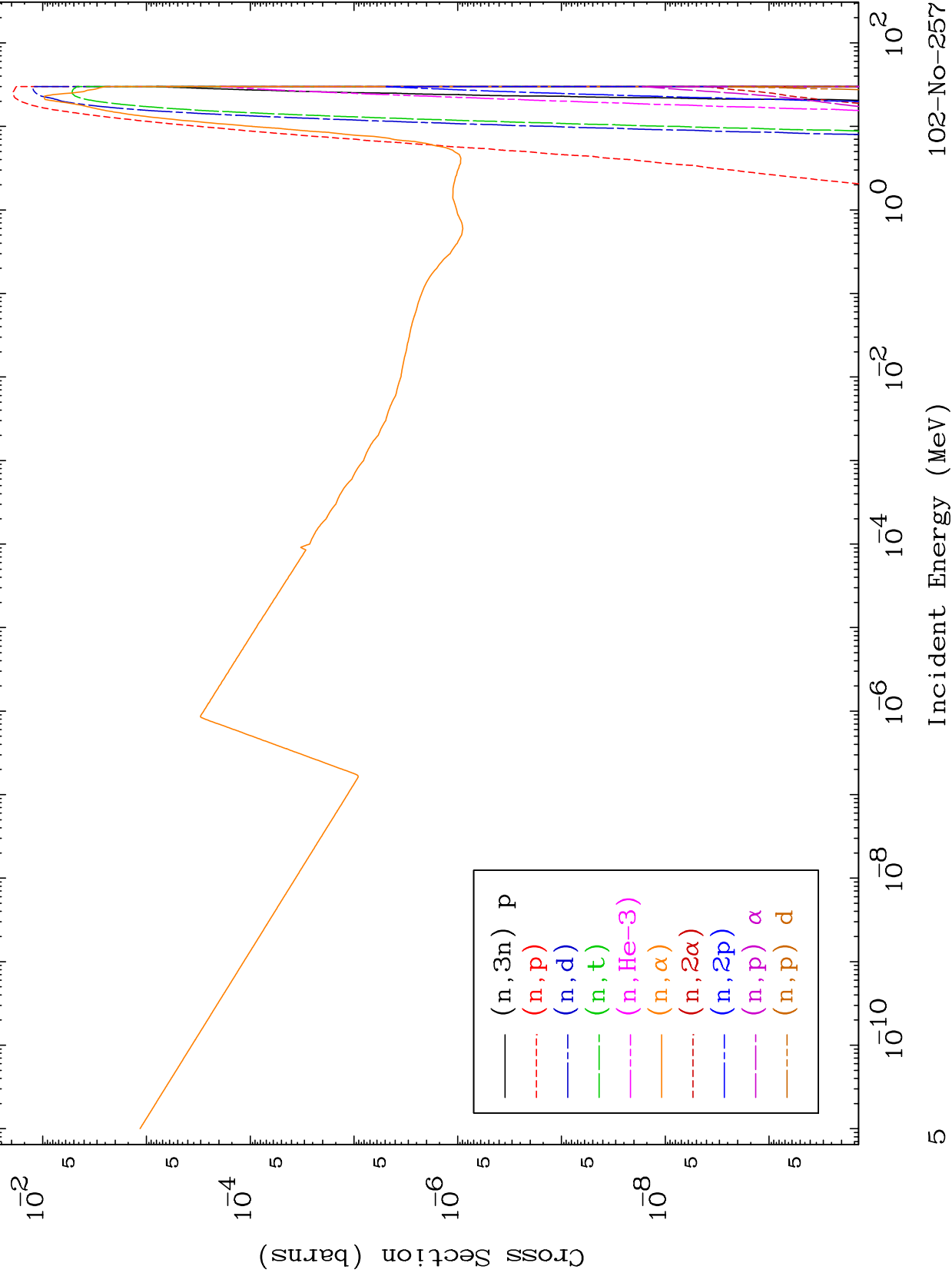
102-No-257



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Charged Particle
293 Kelvin Cross Sections

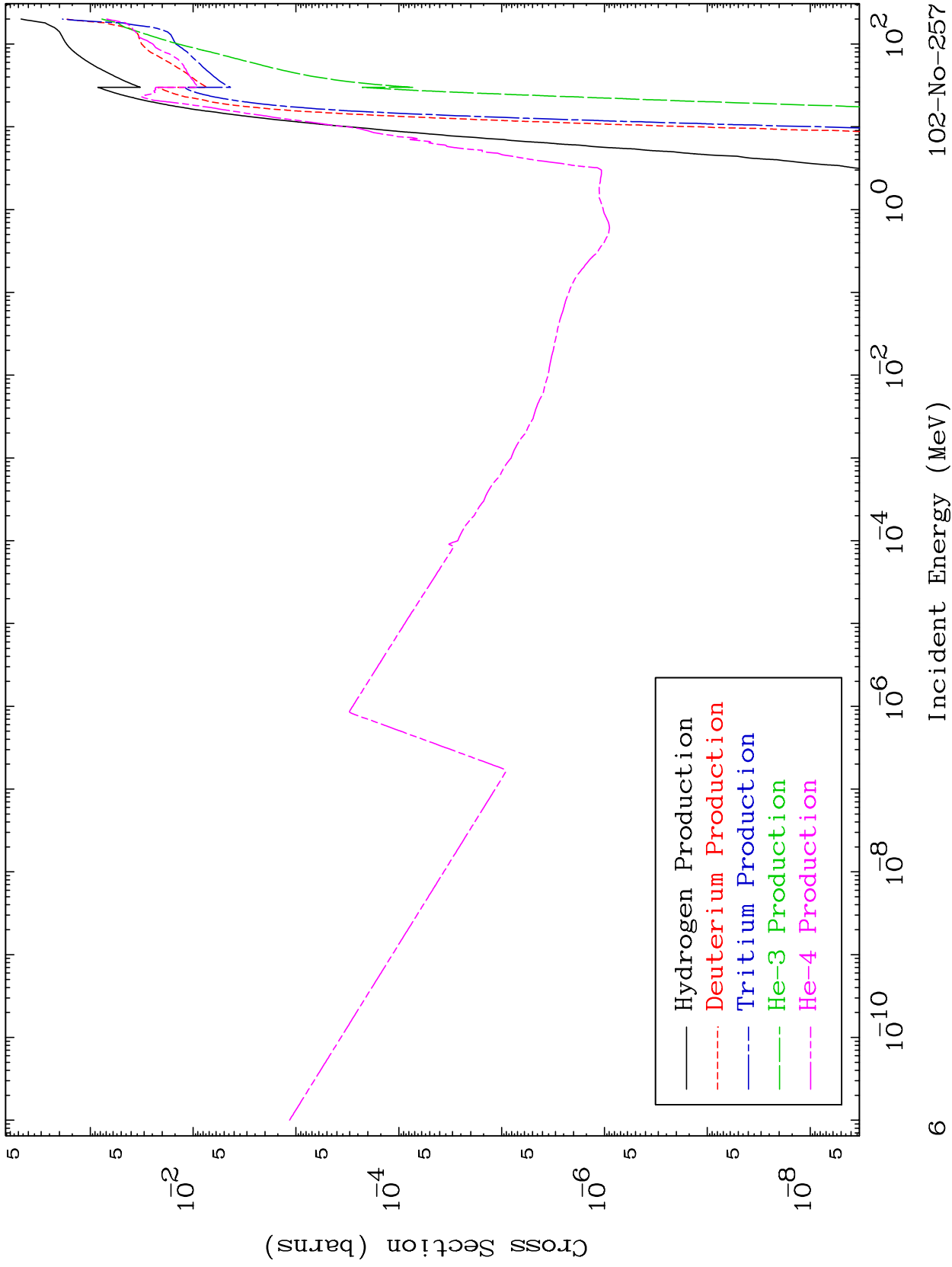
102-No-257



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Particle Production
293 Kelvin Cross Sections

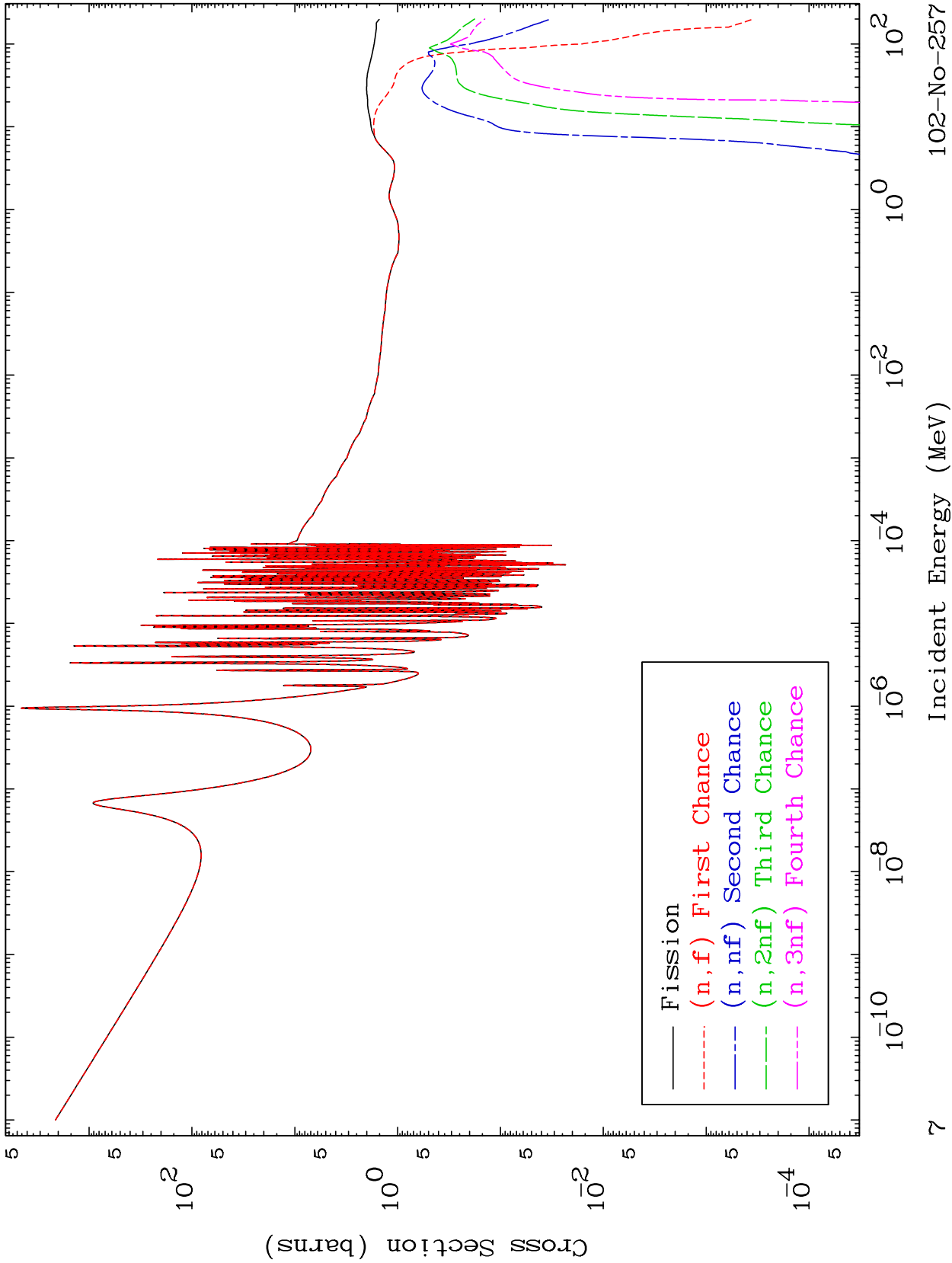
102-No-257



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Fission
293 Kelvin Cross Sections

102-No-257

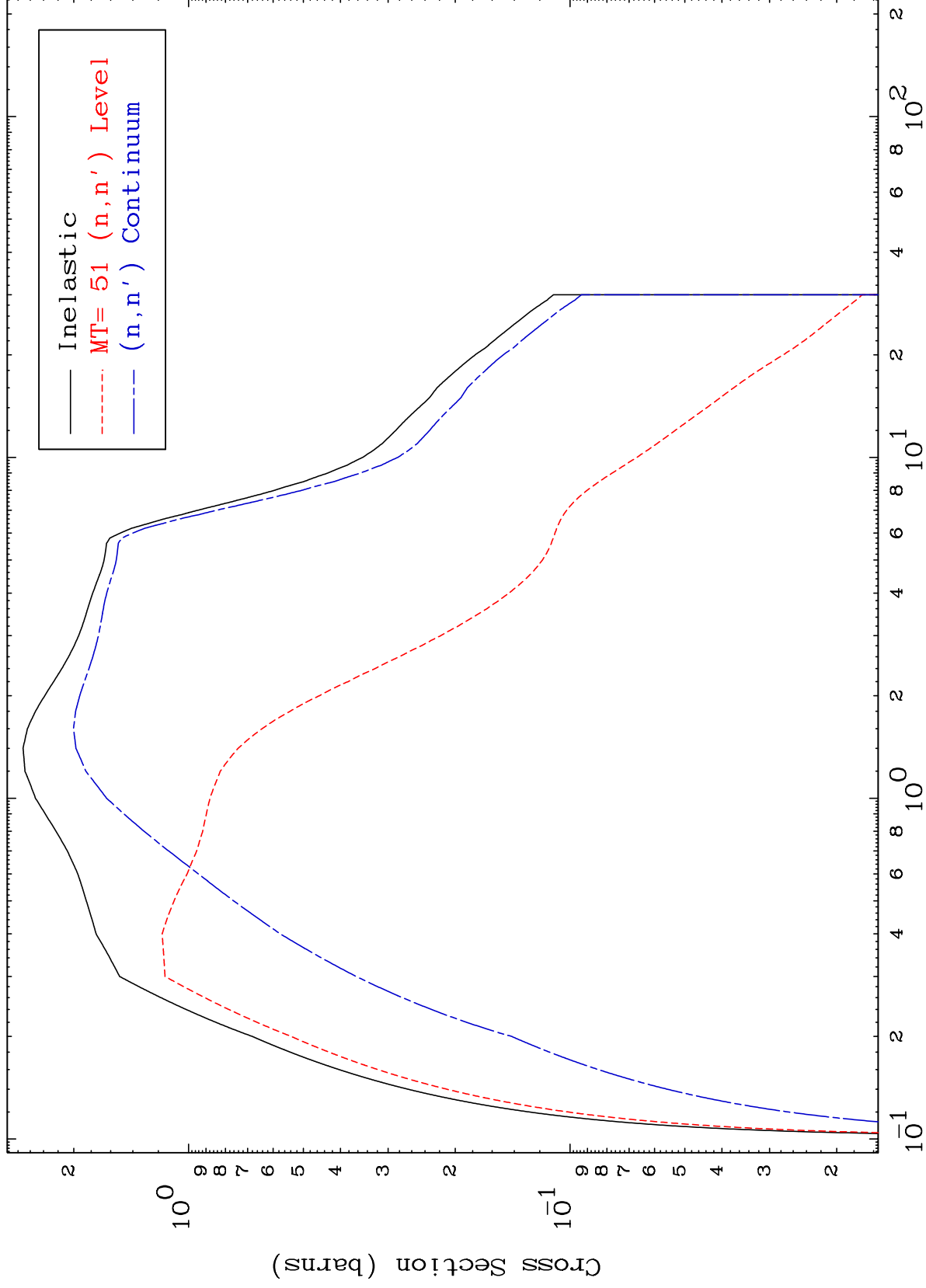


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(n,n') Level

102-No-257

293 Kelvin Cross Sections



Incident Energy (MeV)

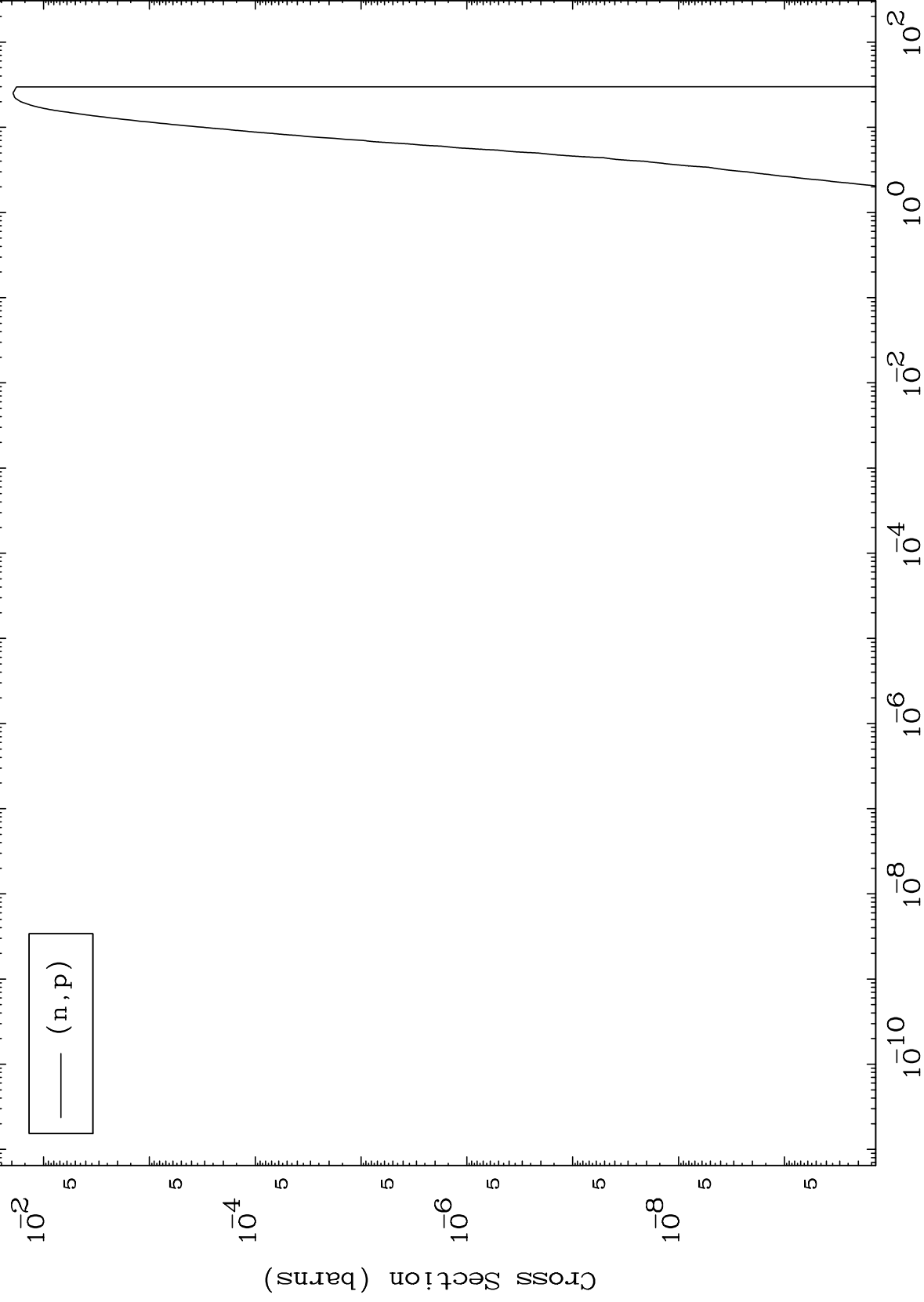
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(n,p) Levels
293 Kelvin Cross Sections

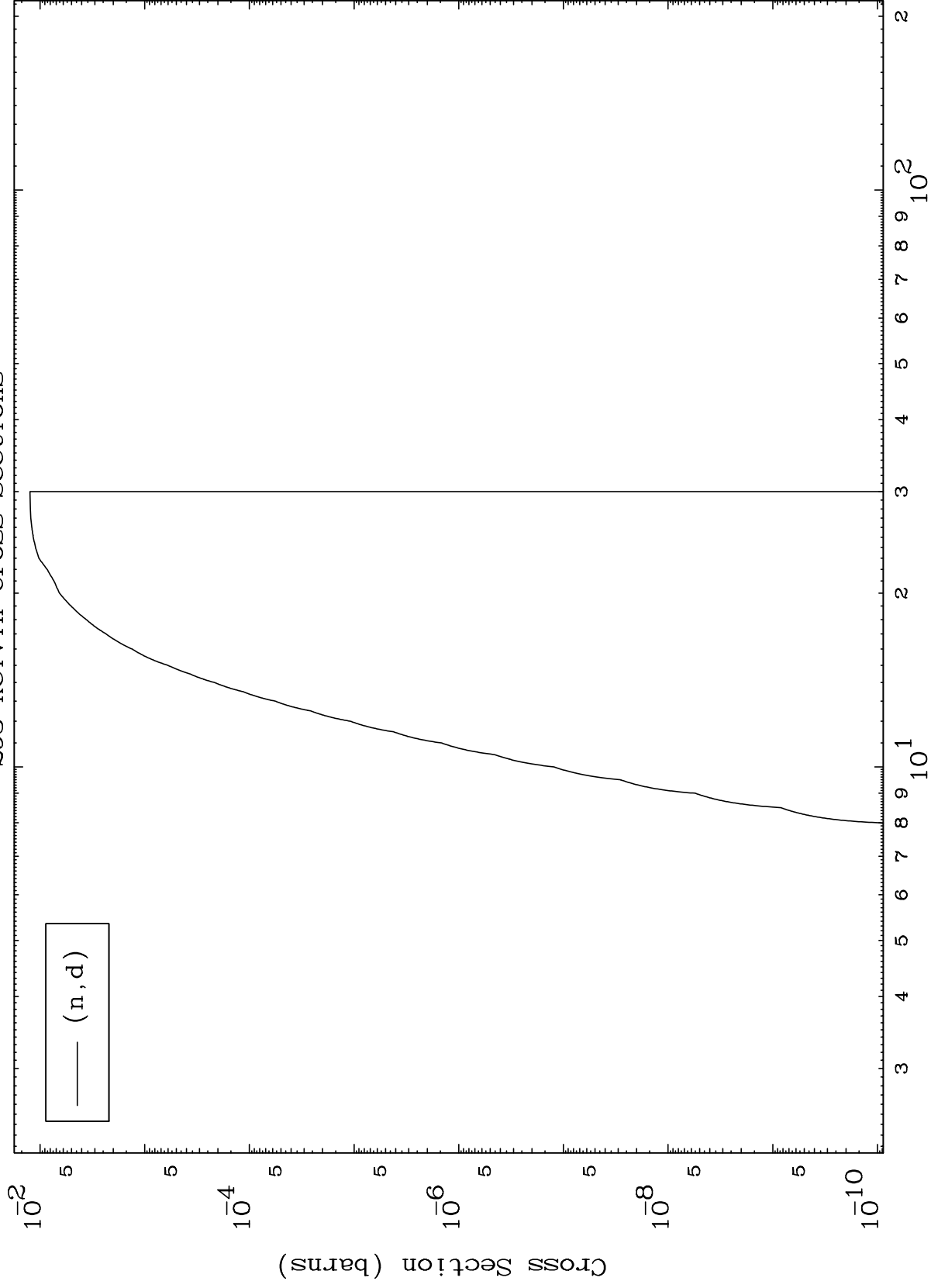
102-No-257



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(n,d) Levels
293 Kelvin Cross Sections

102-No-257



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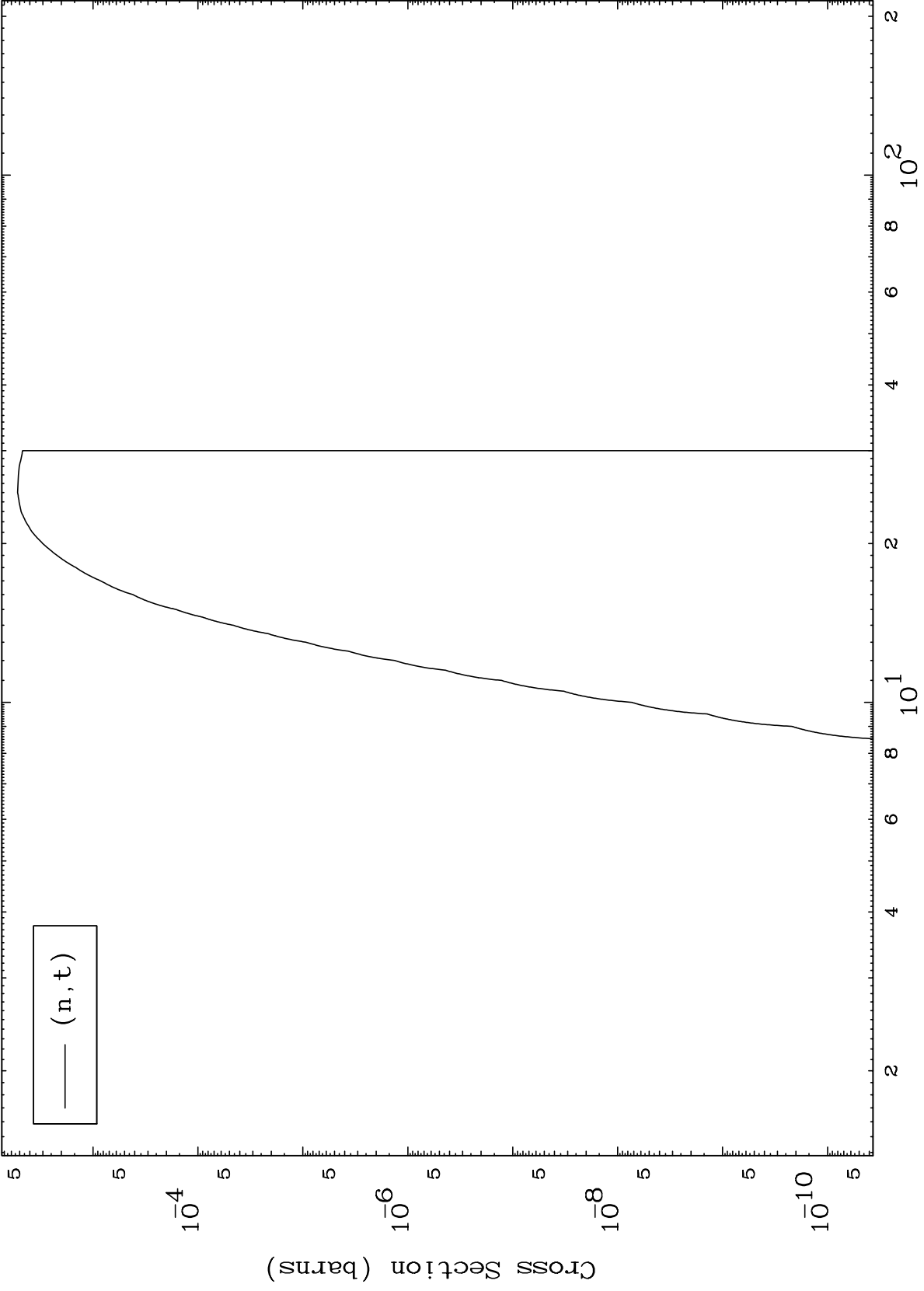
Incident Energy (MeV)

102-No-257

MAT 9975

(n,t) Levels
293 Kelvin Cross Sections

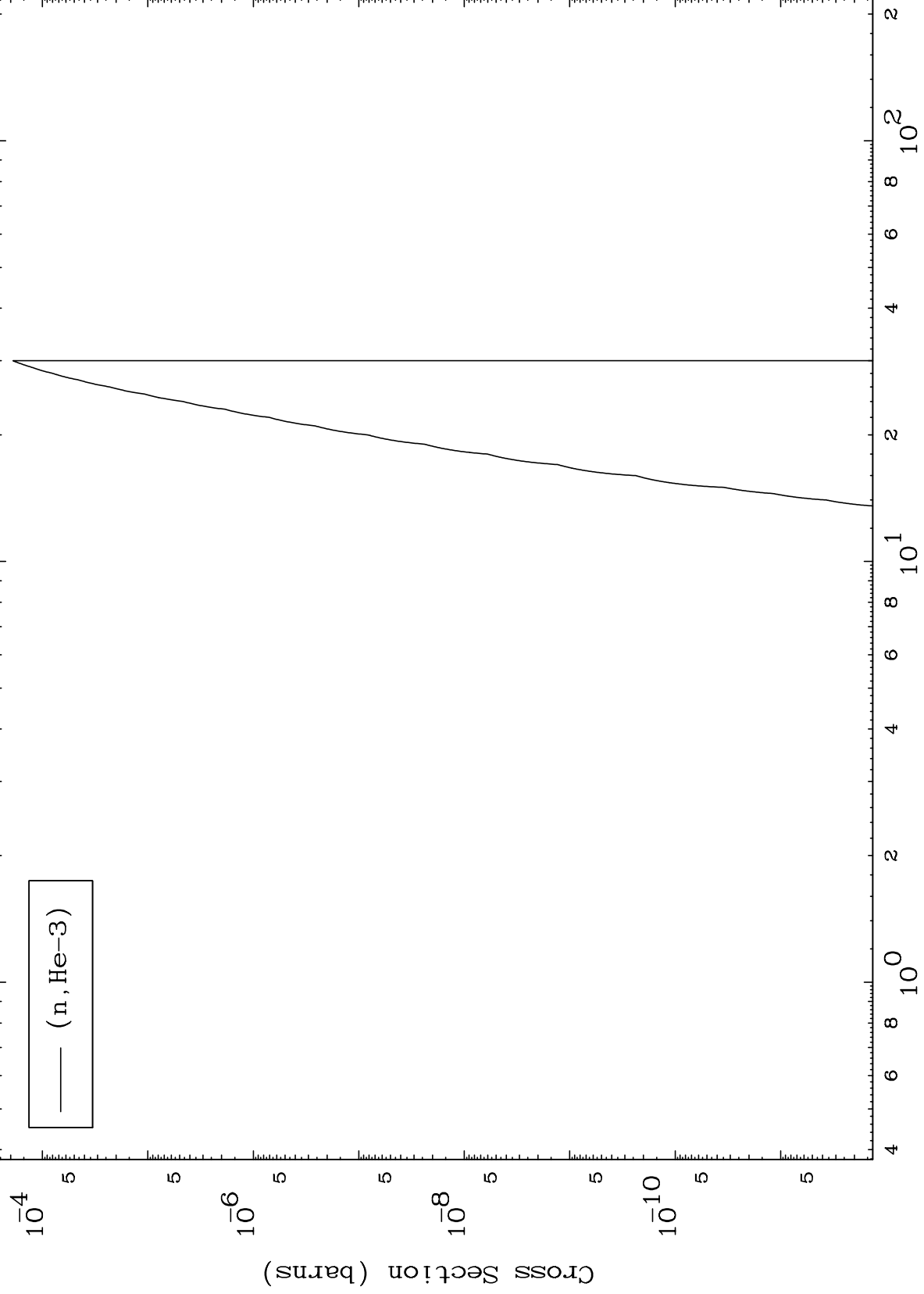
102-No-257



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(n,He3) Levels
293 Kelvin Cross Sections

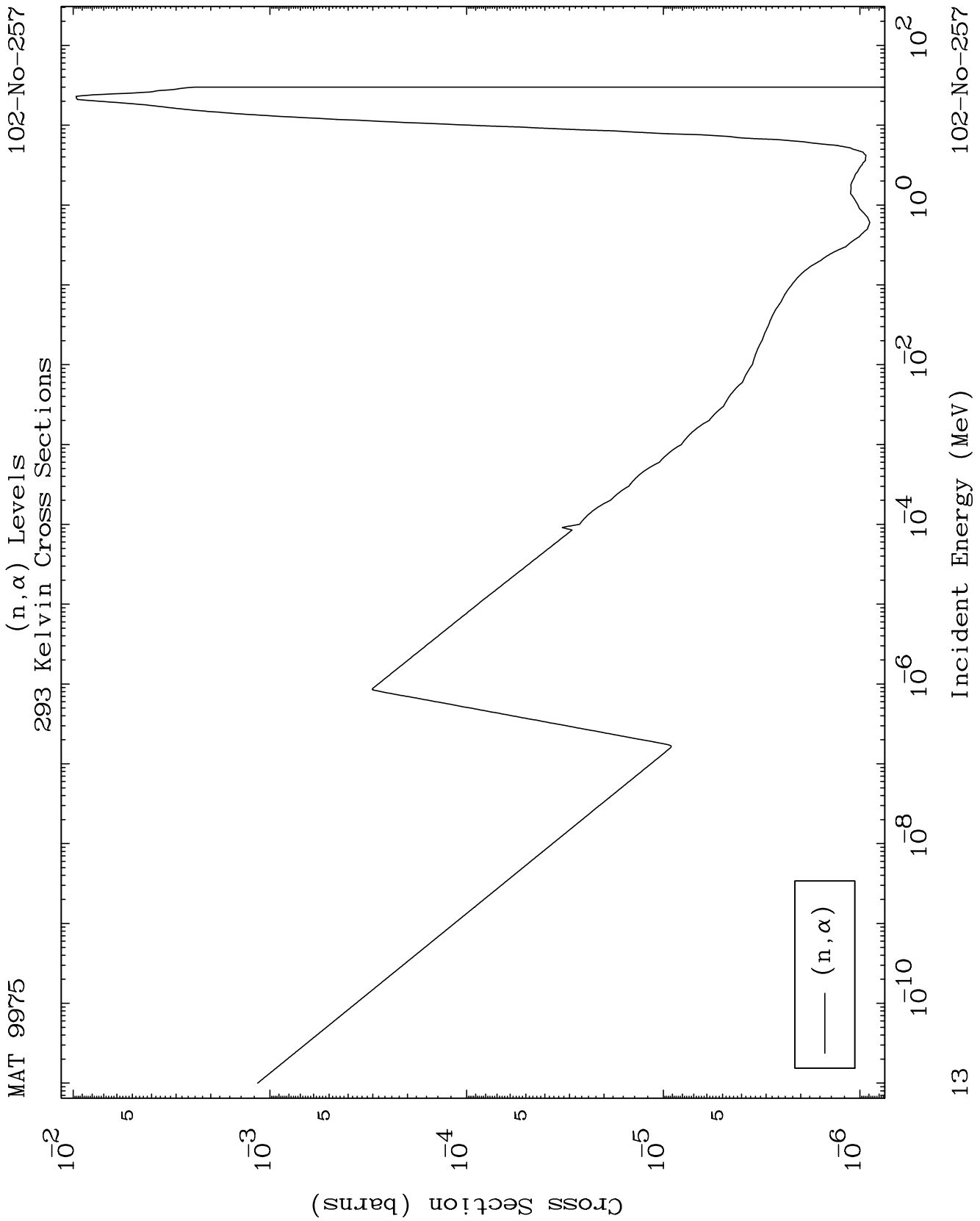
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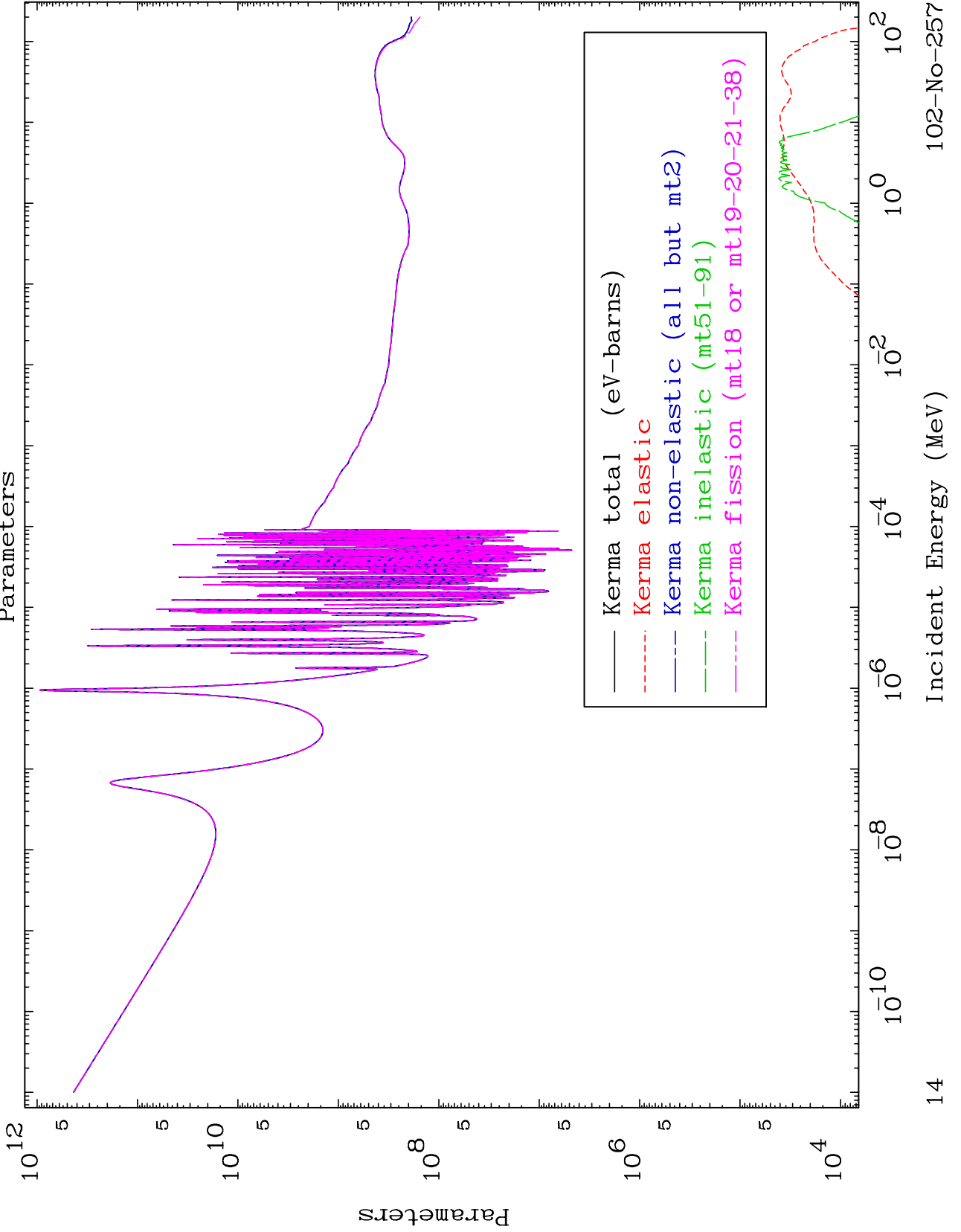


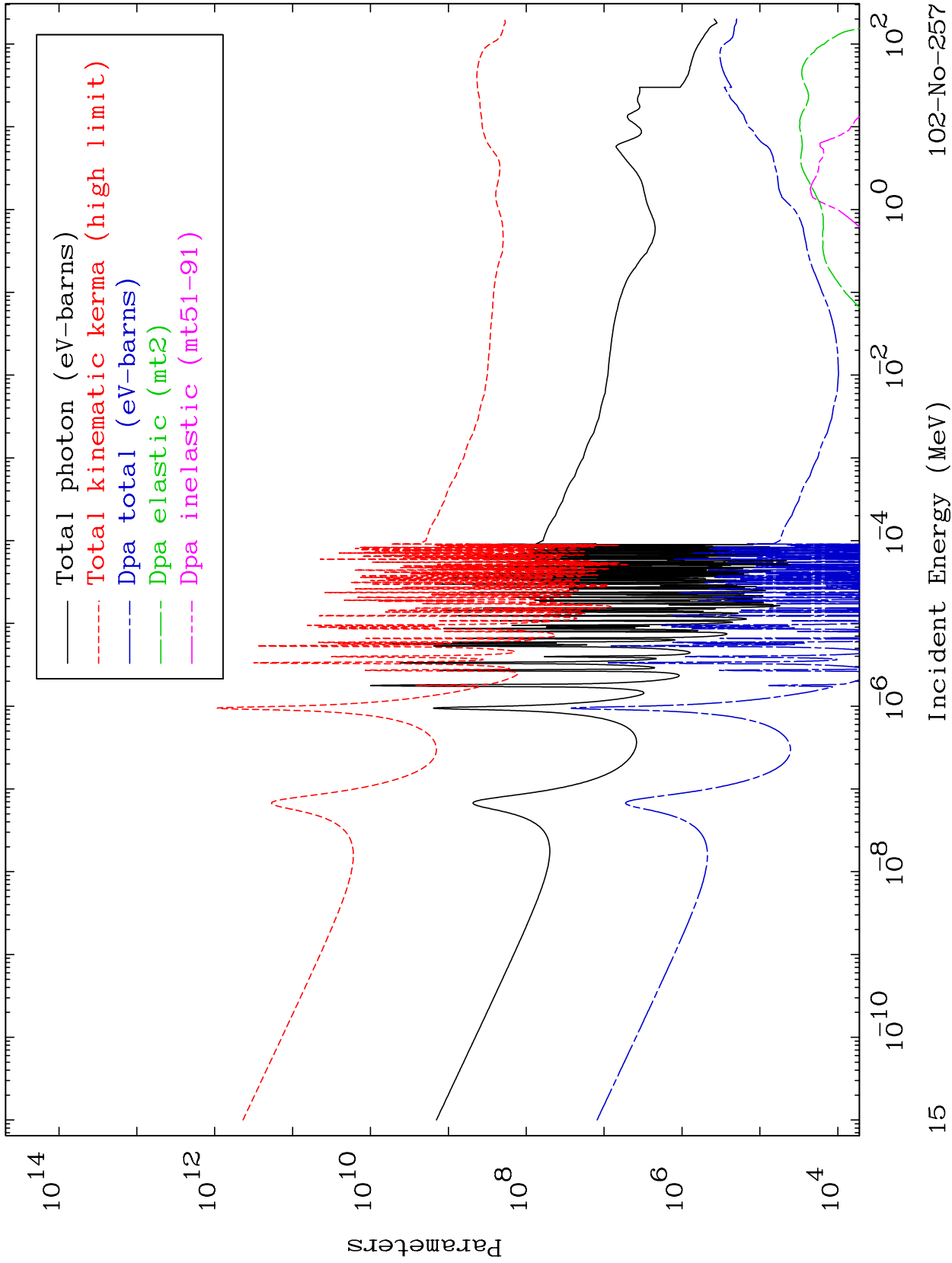
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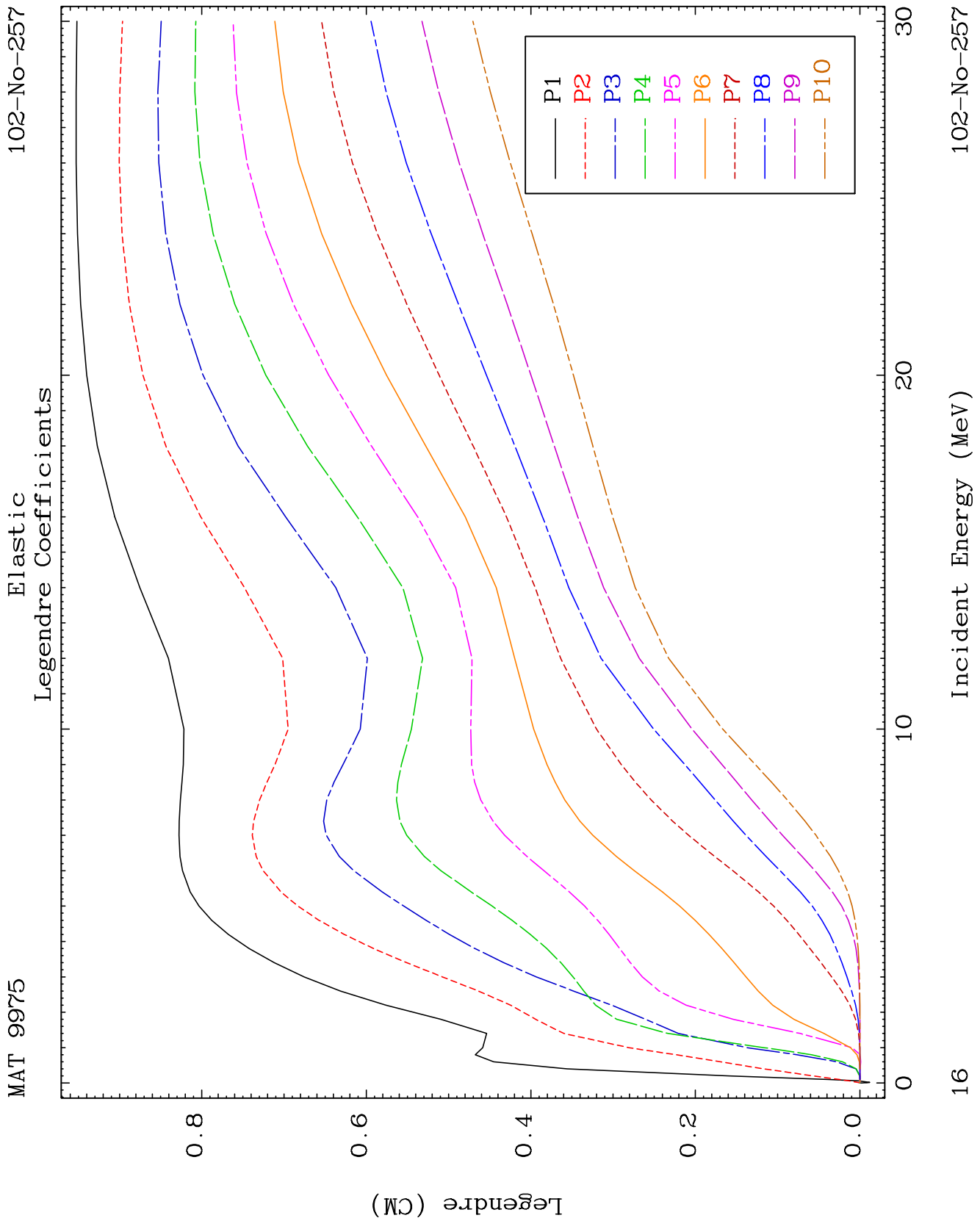
Incident Energy (MeV)

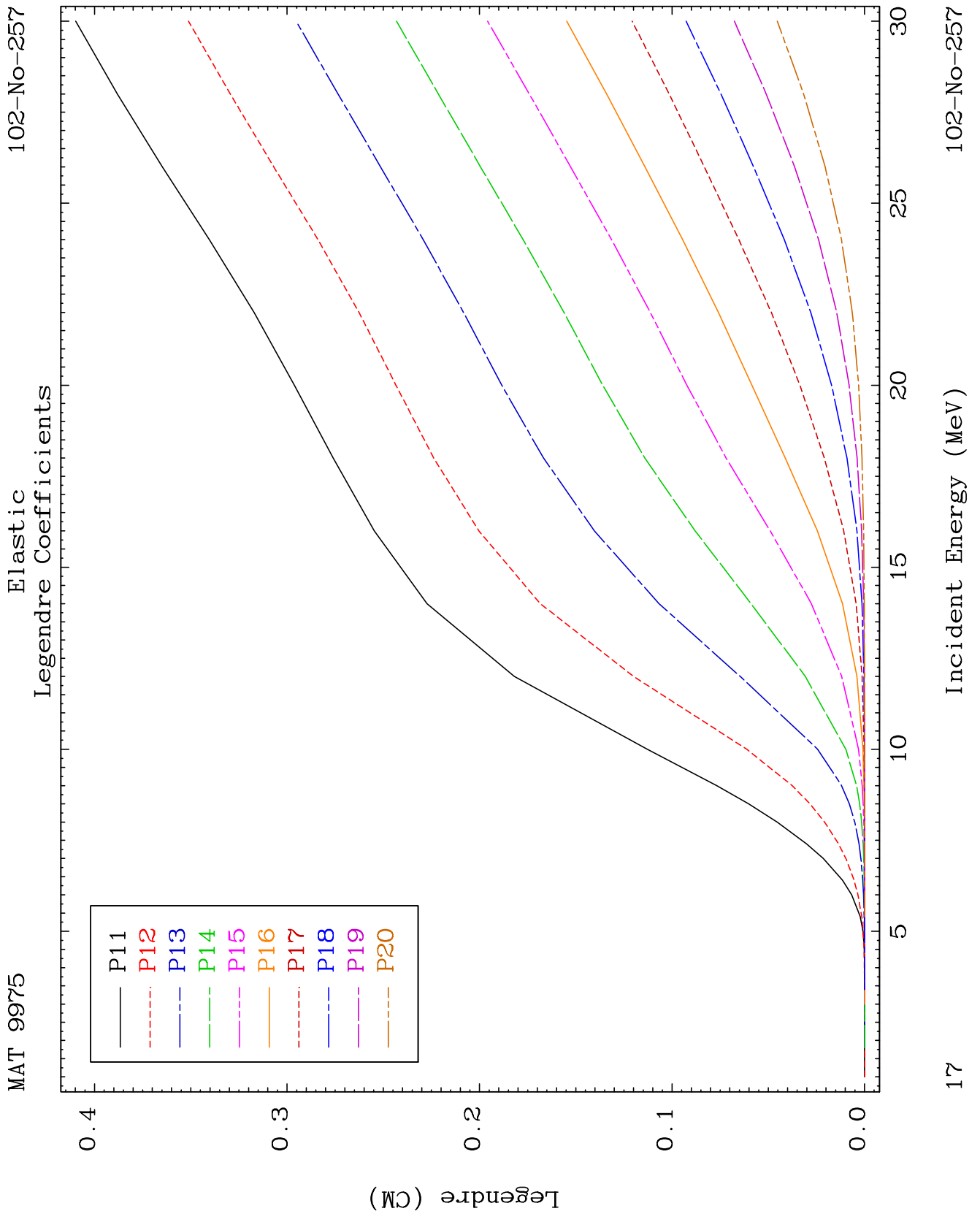
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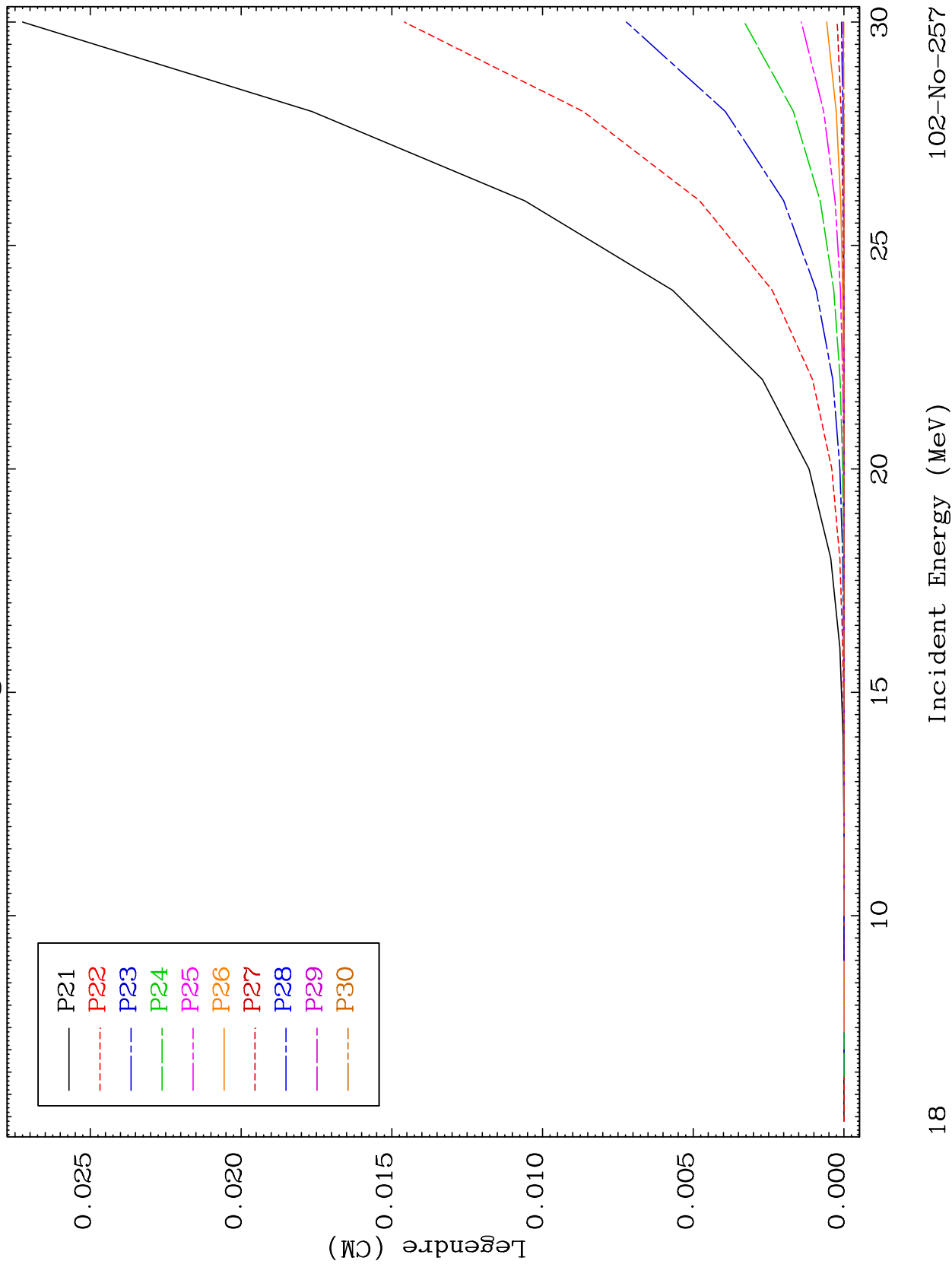




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Elastic Legendre Coefficients

102-No-257



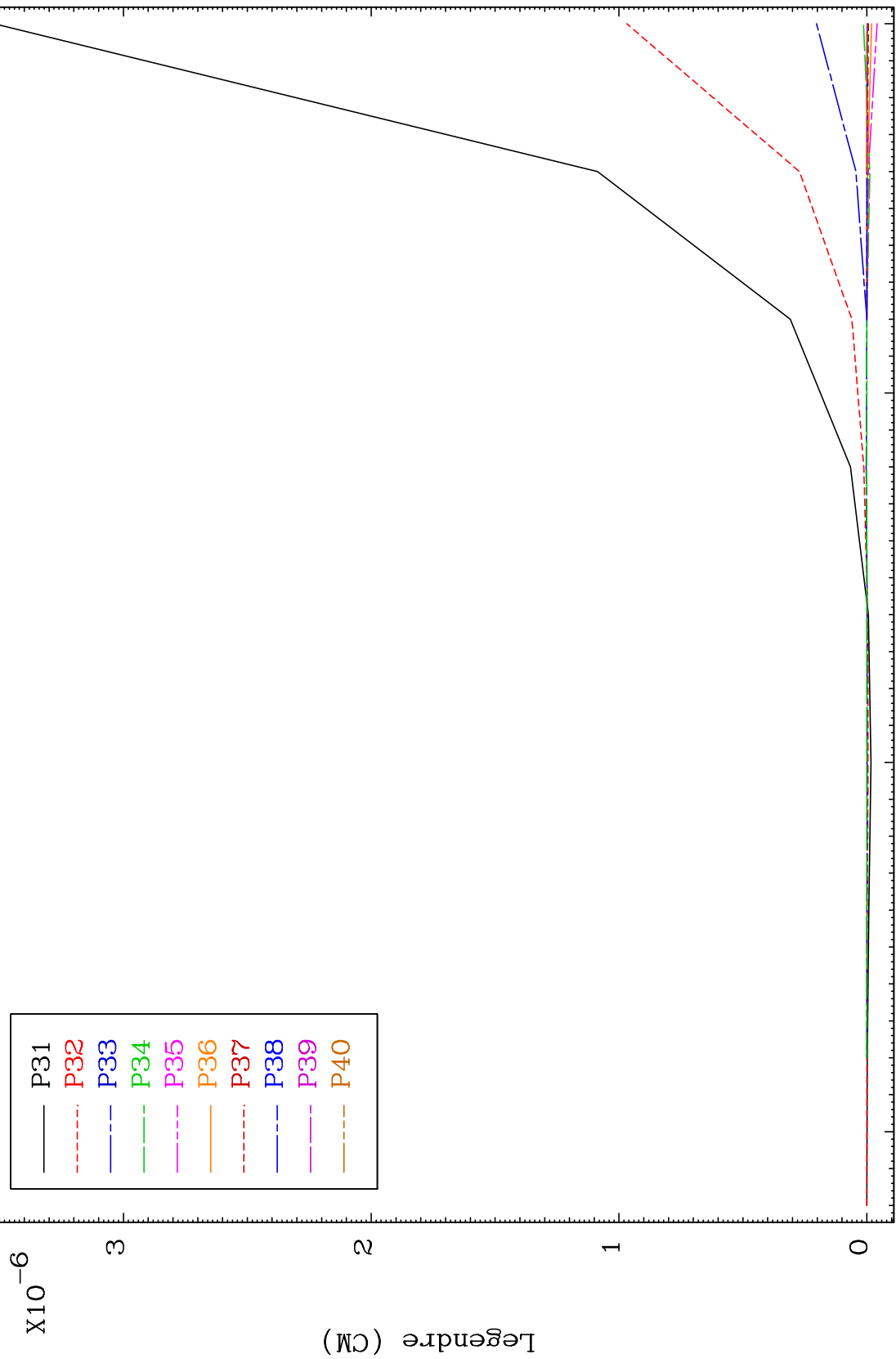
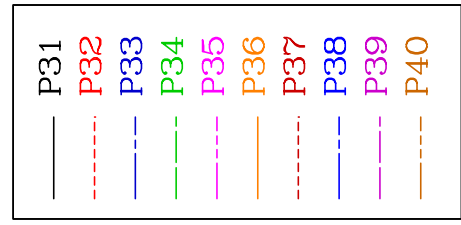
18

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Elastic Legendre Coefficients

102-No-257



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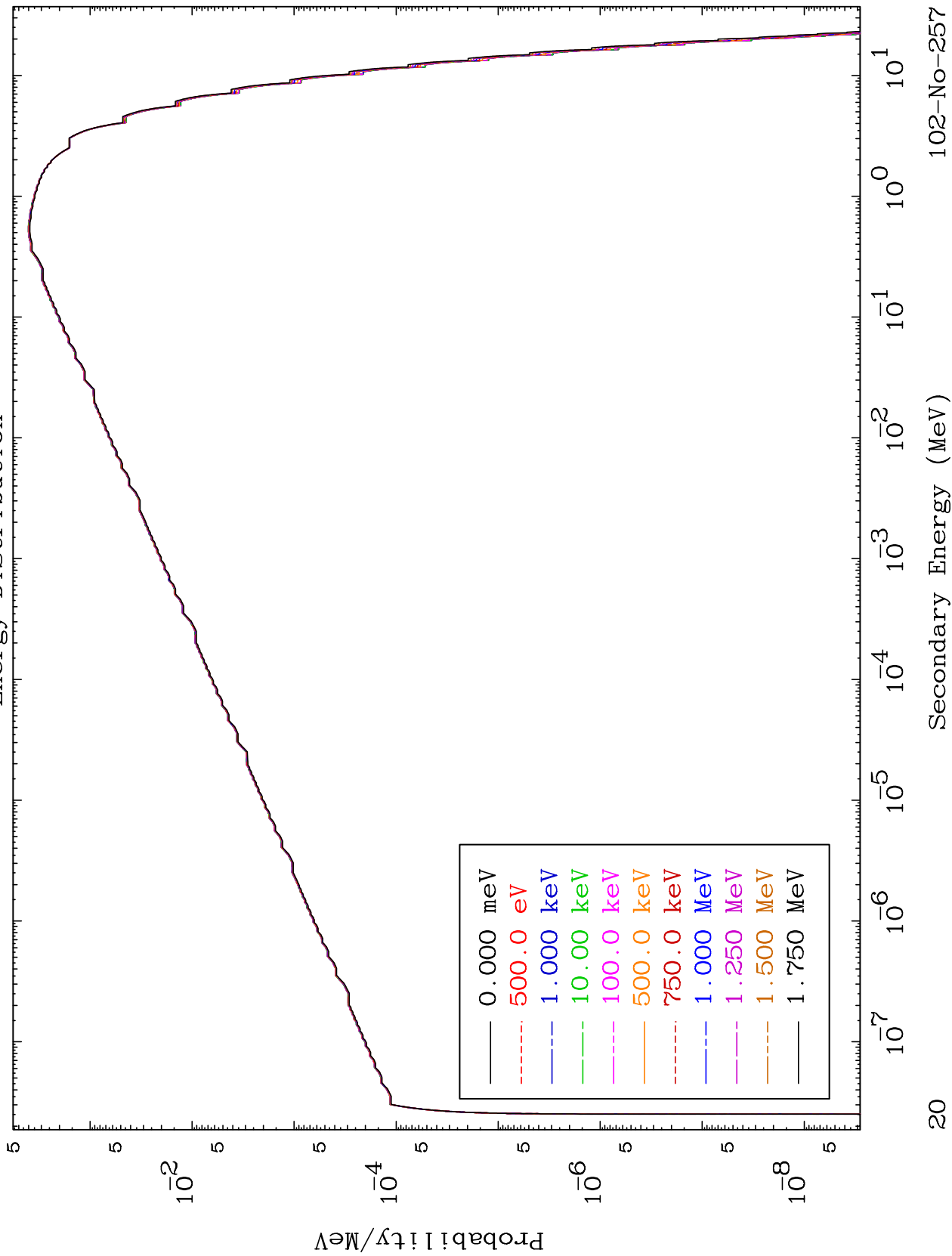
Incident Energy (MeV)

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Fission Energy Distribution

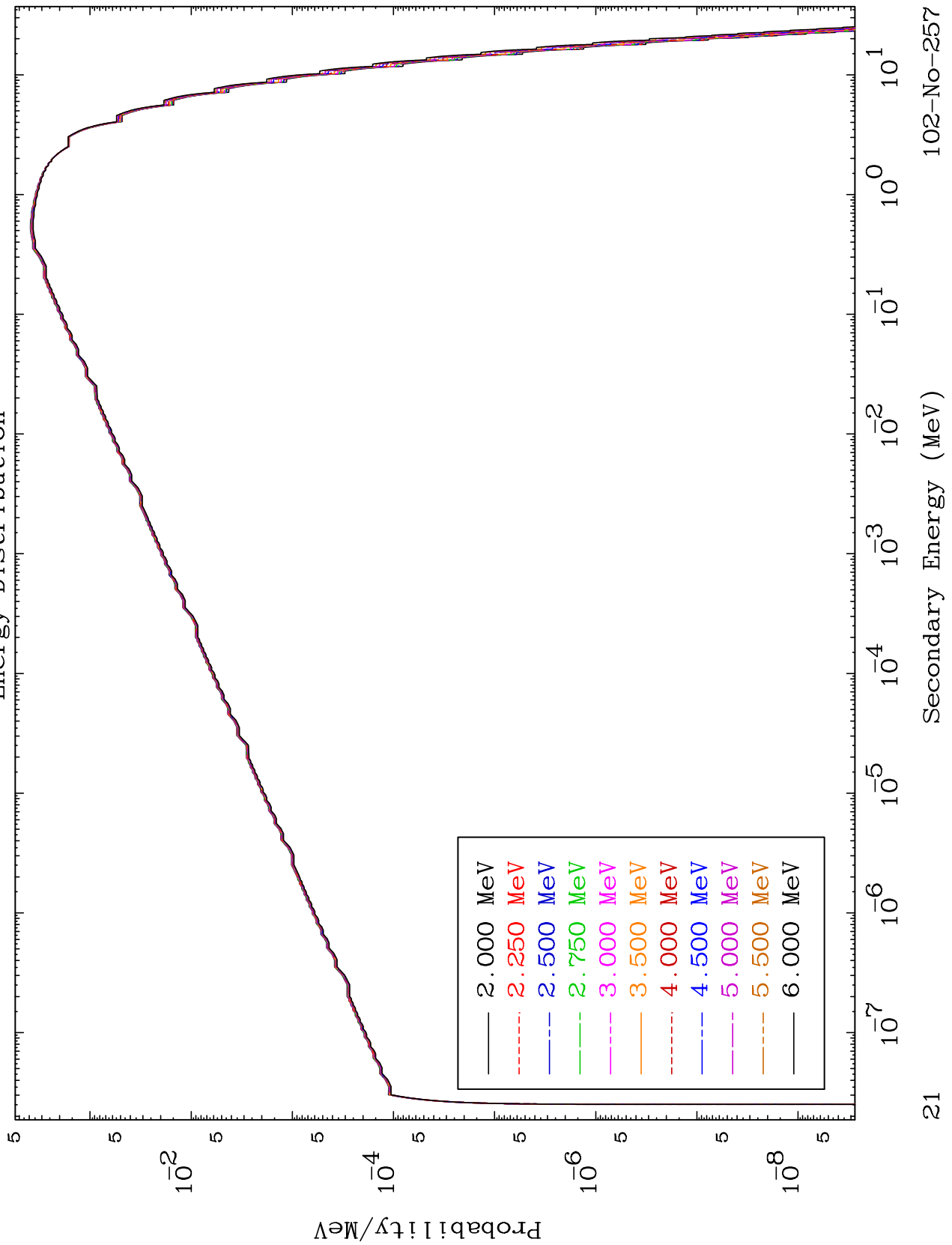
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Fission Energy Distribution

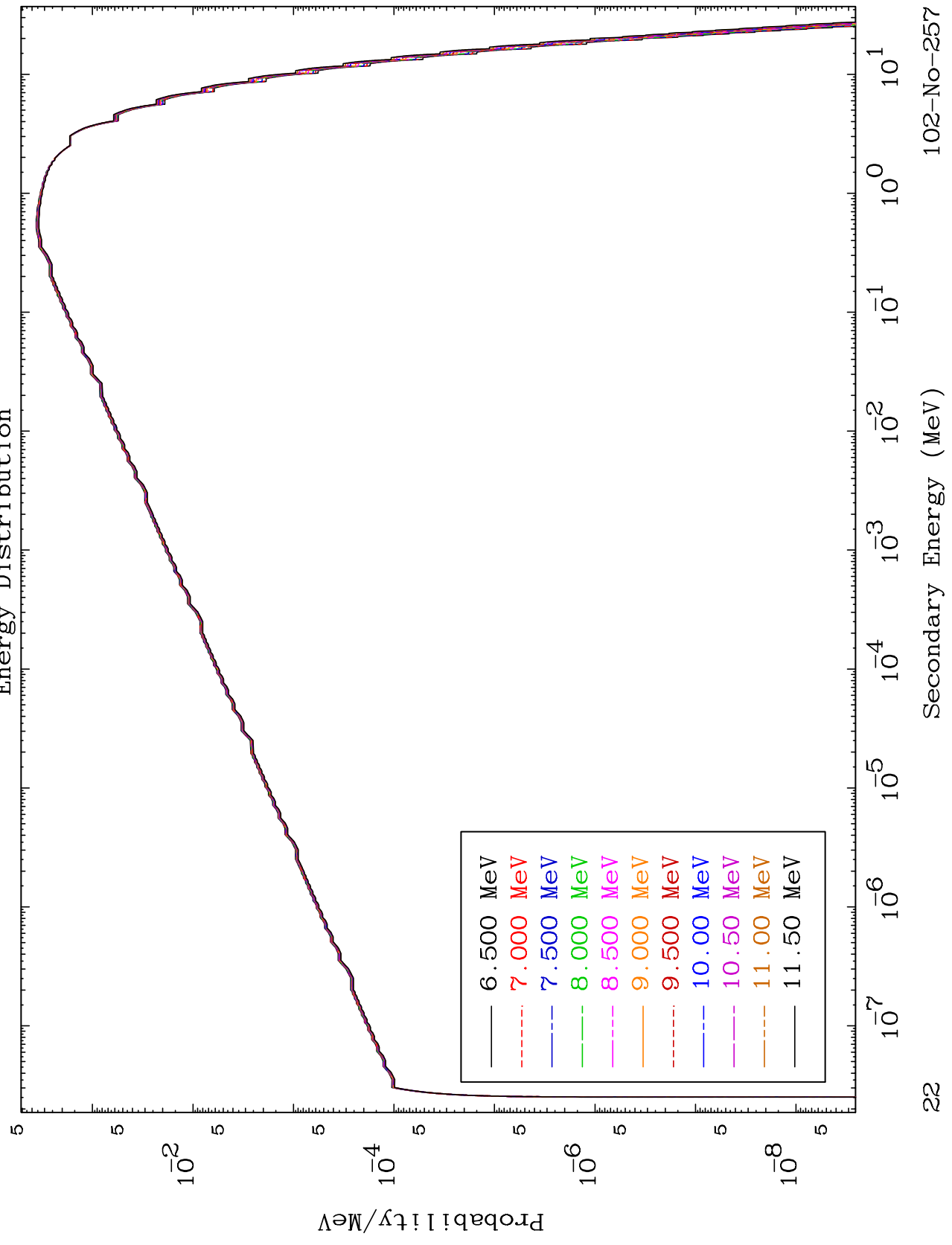
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Fission Energy Distribution

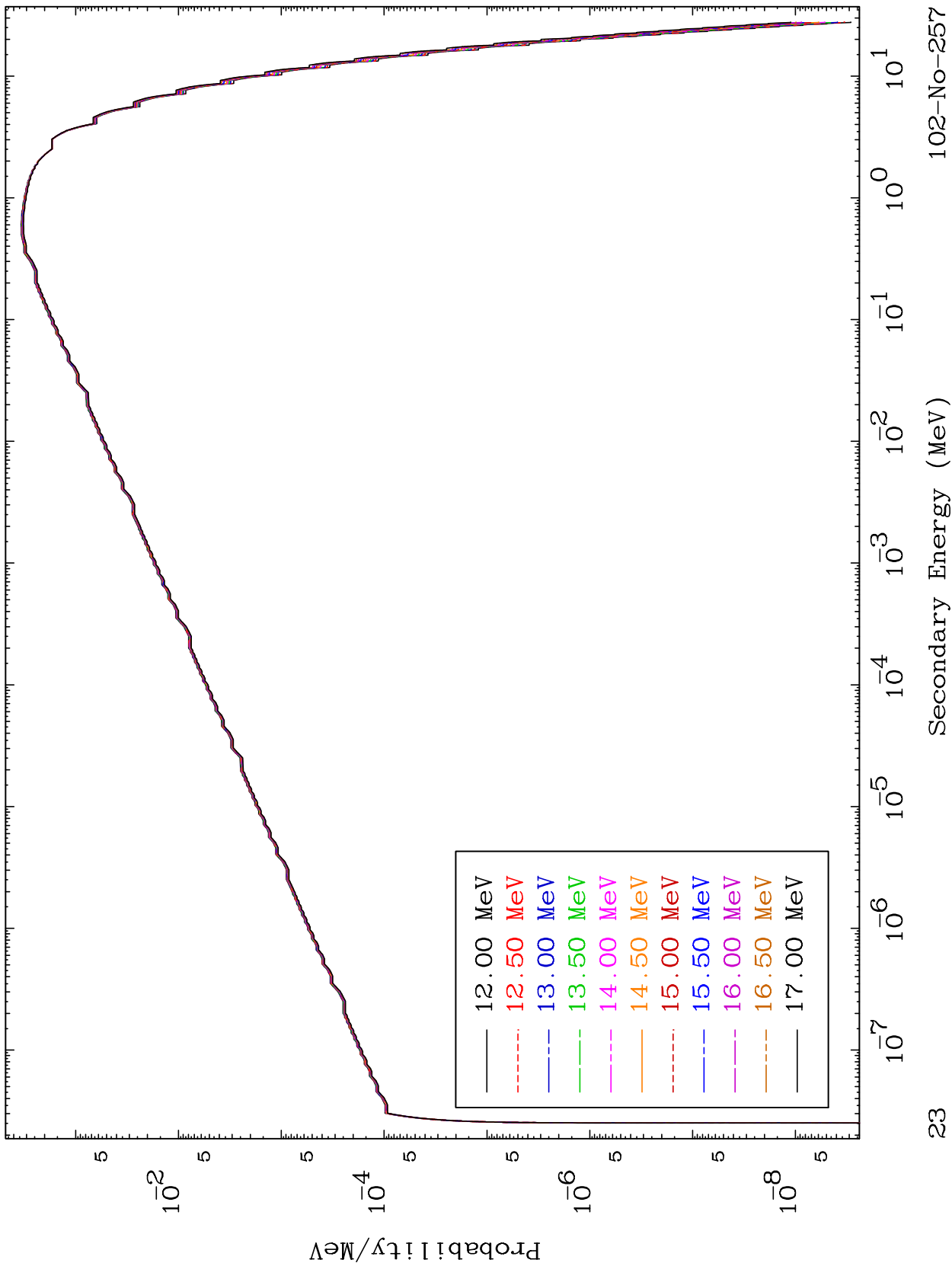
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Fission Energy Distribution

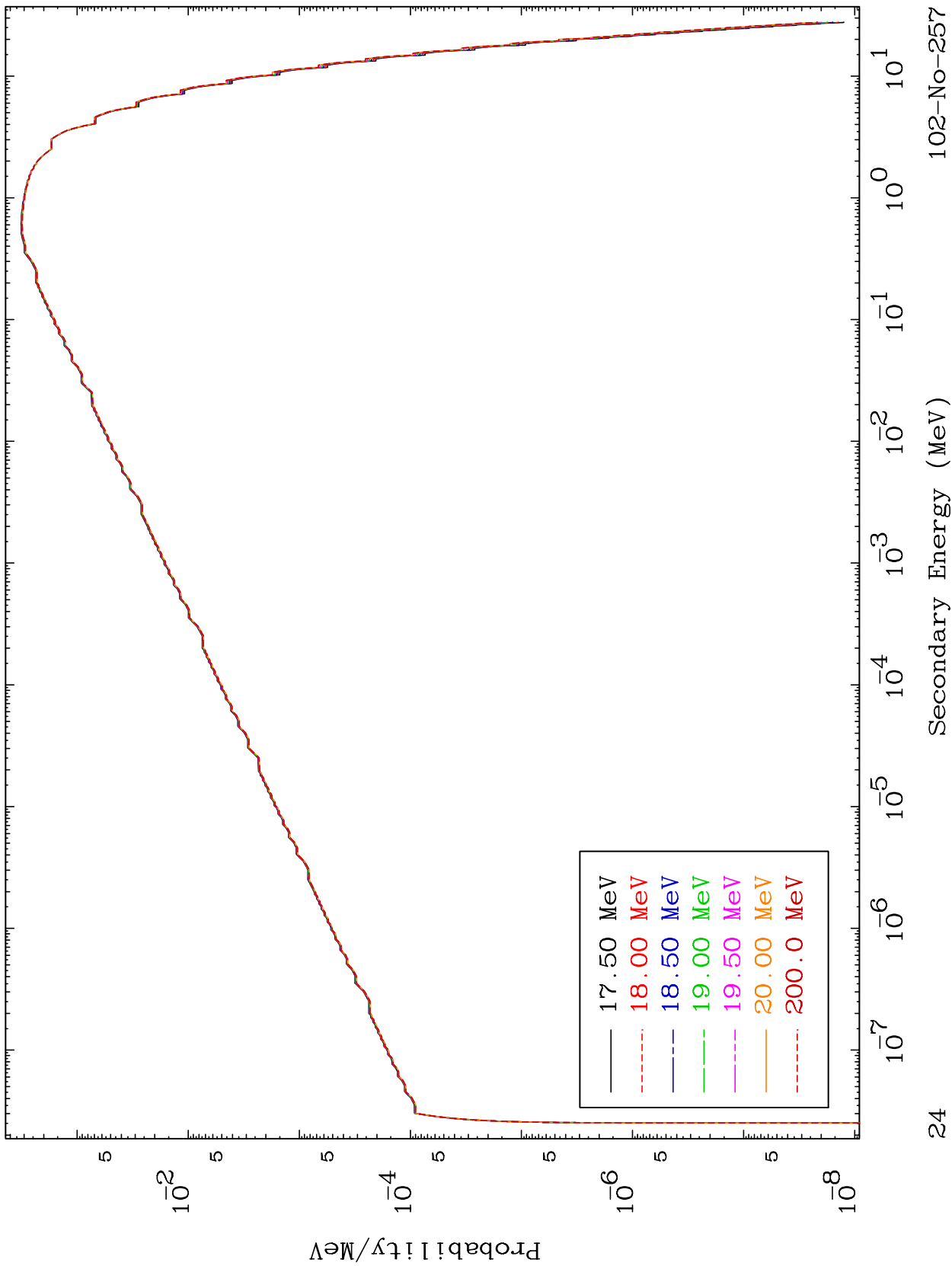
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Fission Energy Distribution

102-No-257

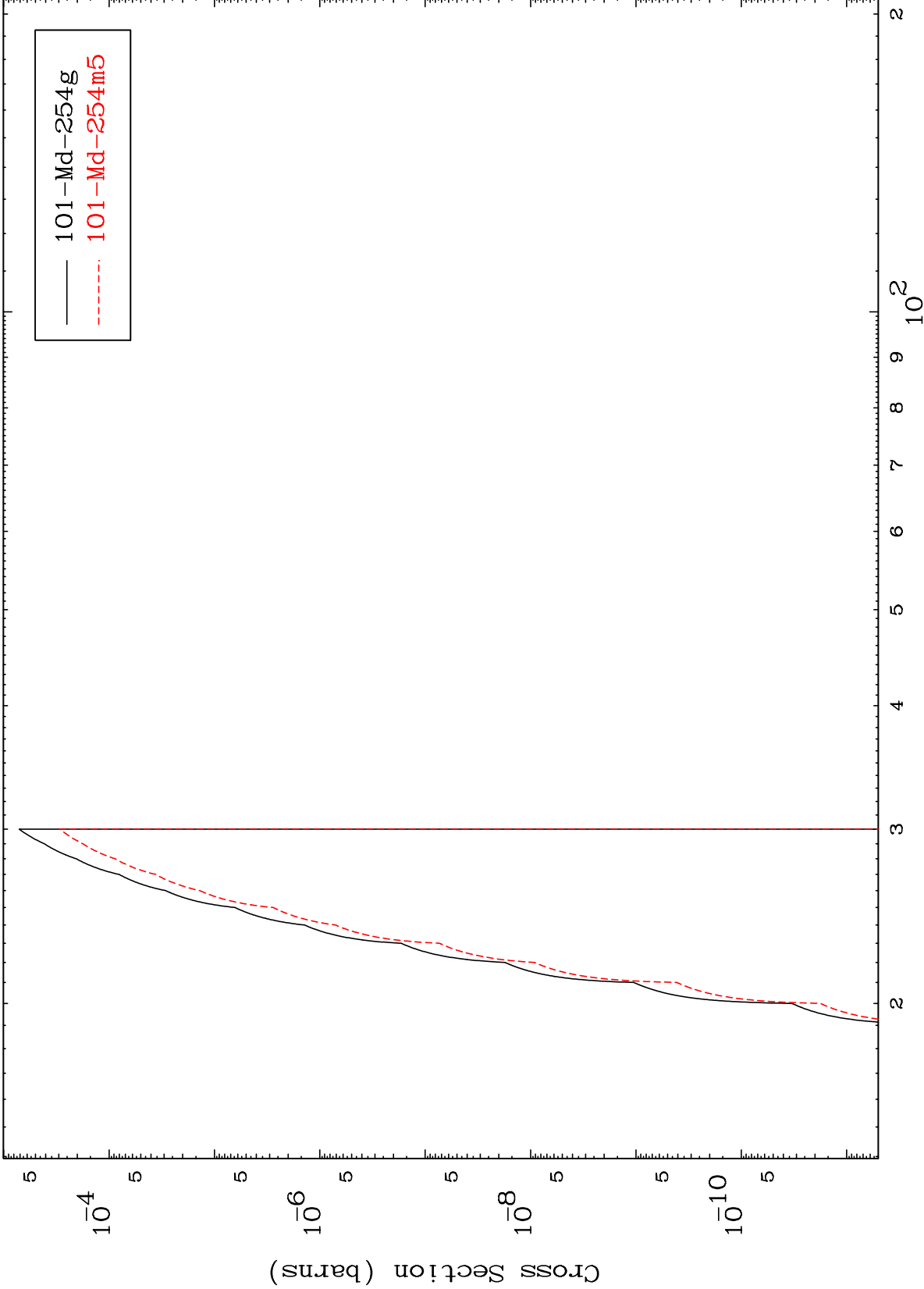


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(n,2n) d

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

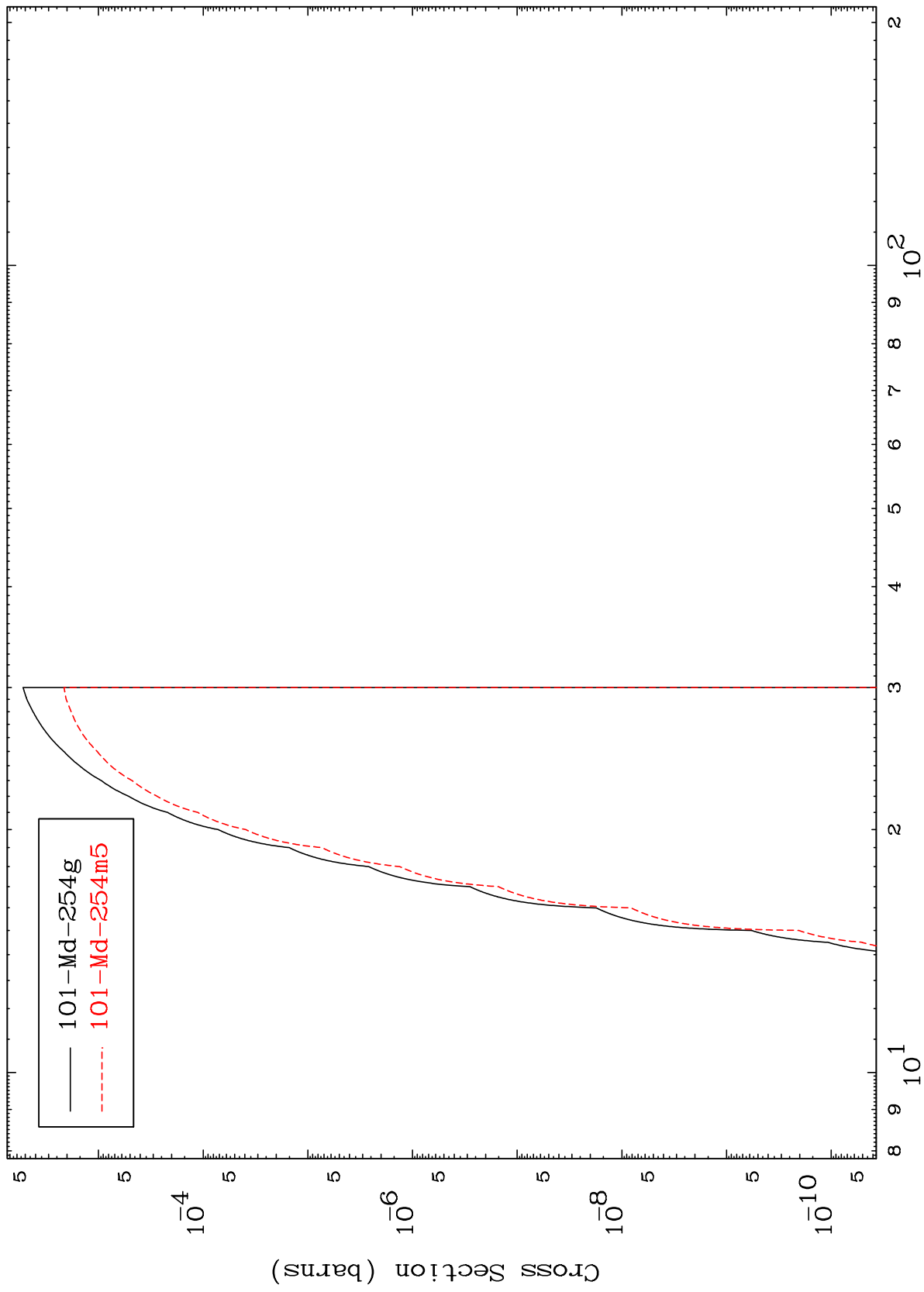
102-No-257

MAT 9975

(n,n') t

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

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Radionuclide Production Cross Section

