

Program EVALPLOT  
(Version 2017-1)

by

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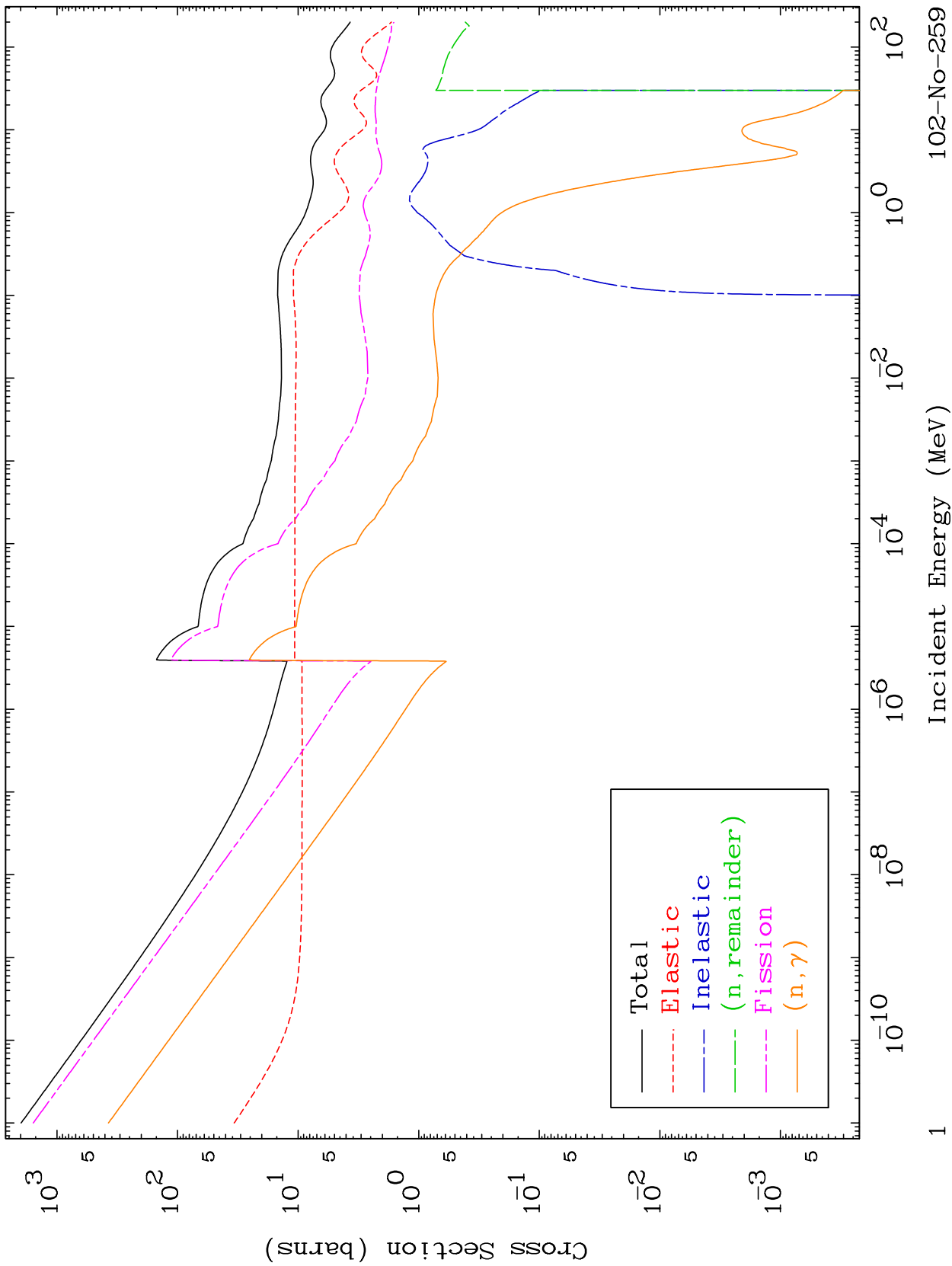
Press Mouse Button to Start

MAT 9977

Major

293 Kelvin Cross Sections

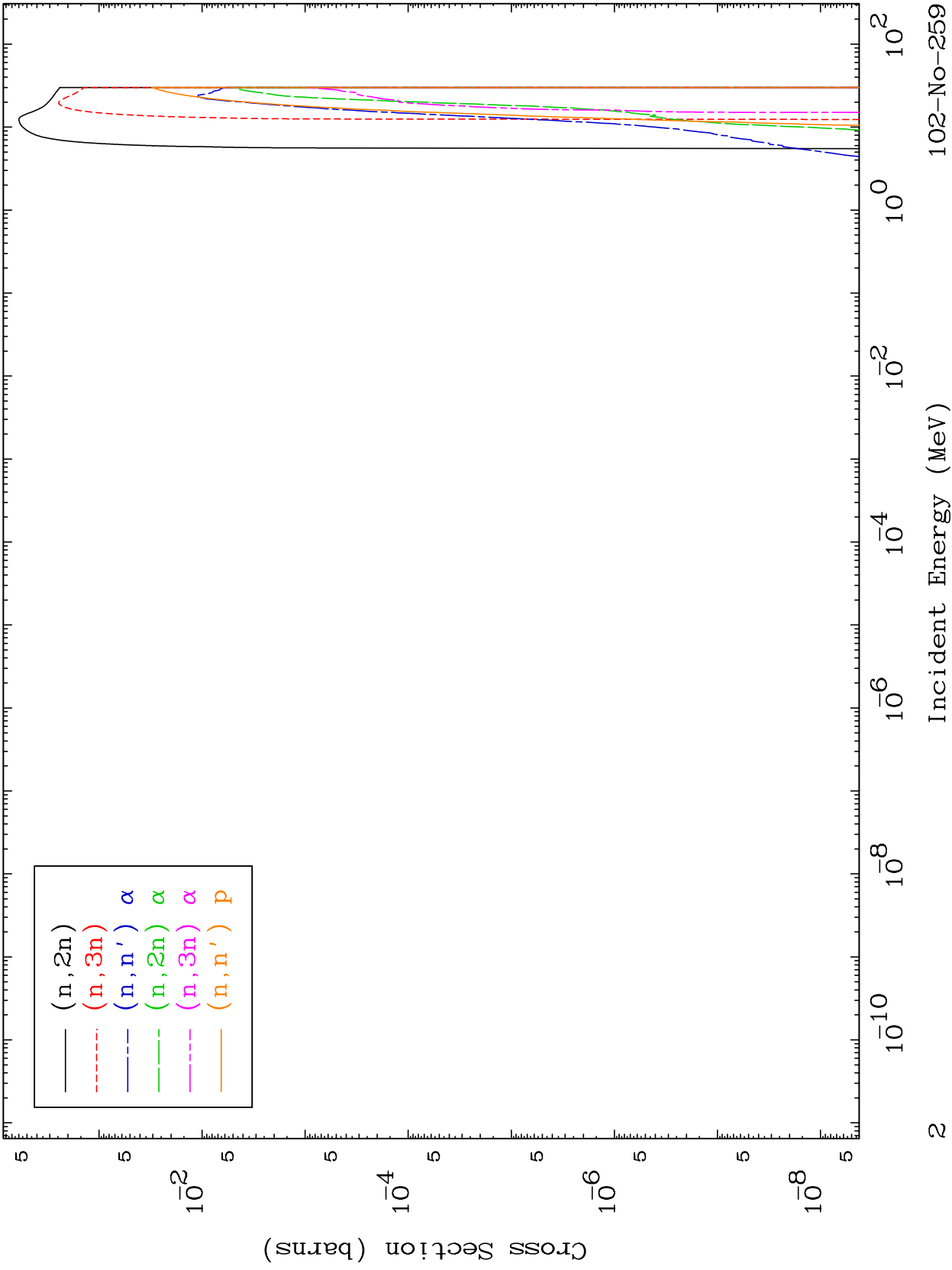
102-No-259

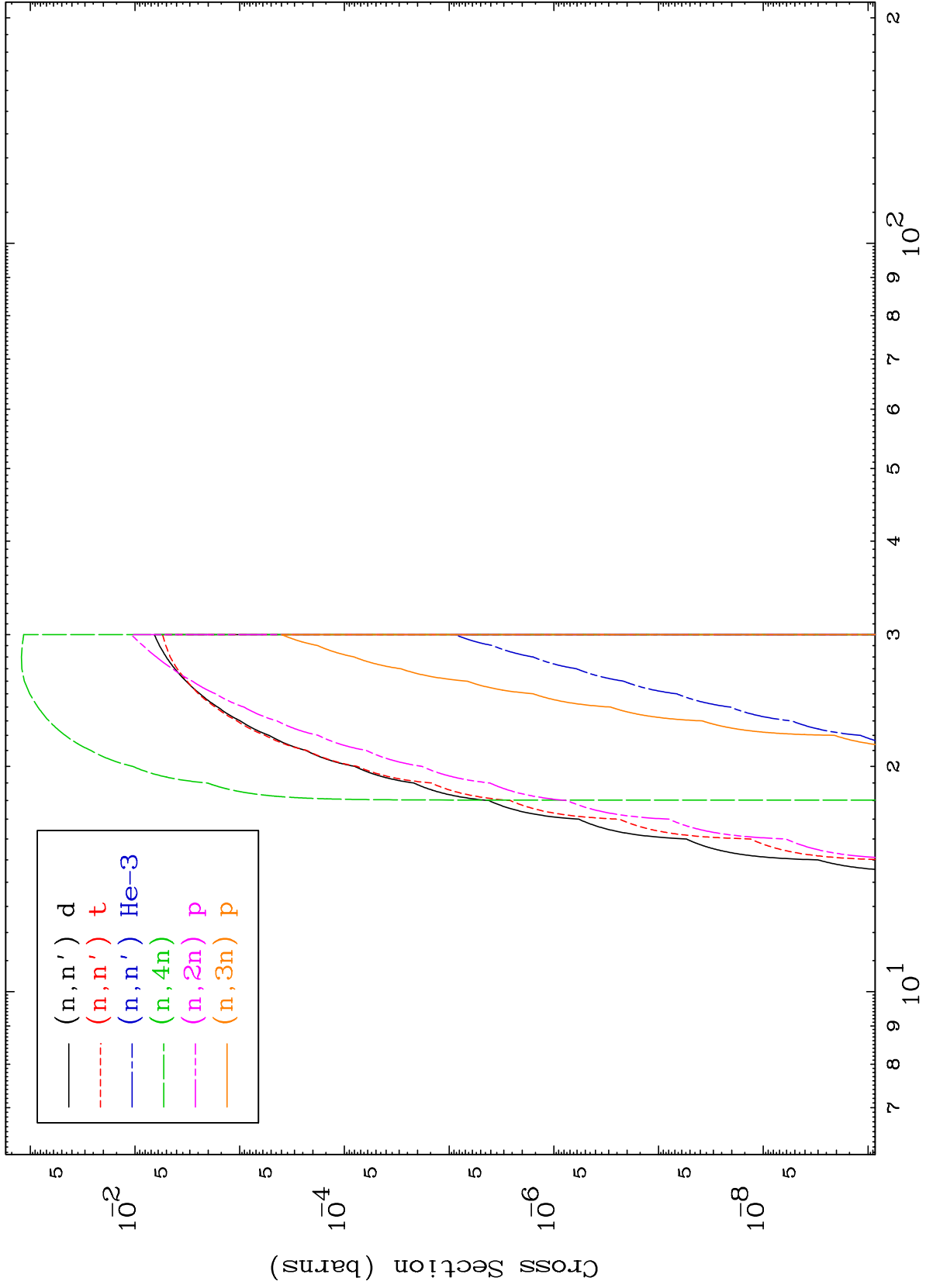


MAT 9977

Neutron Production  
293 Kelvin Cross Sections

102-No-259

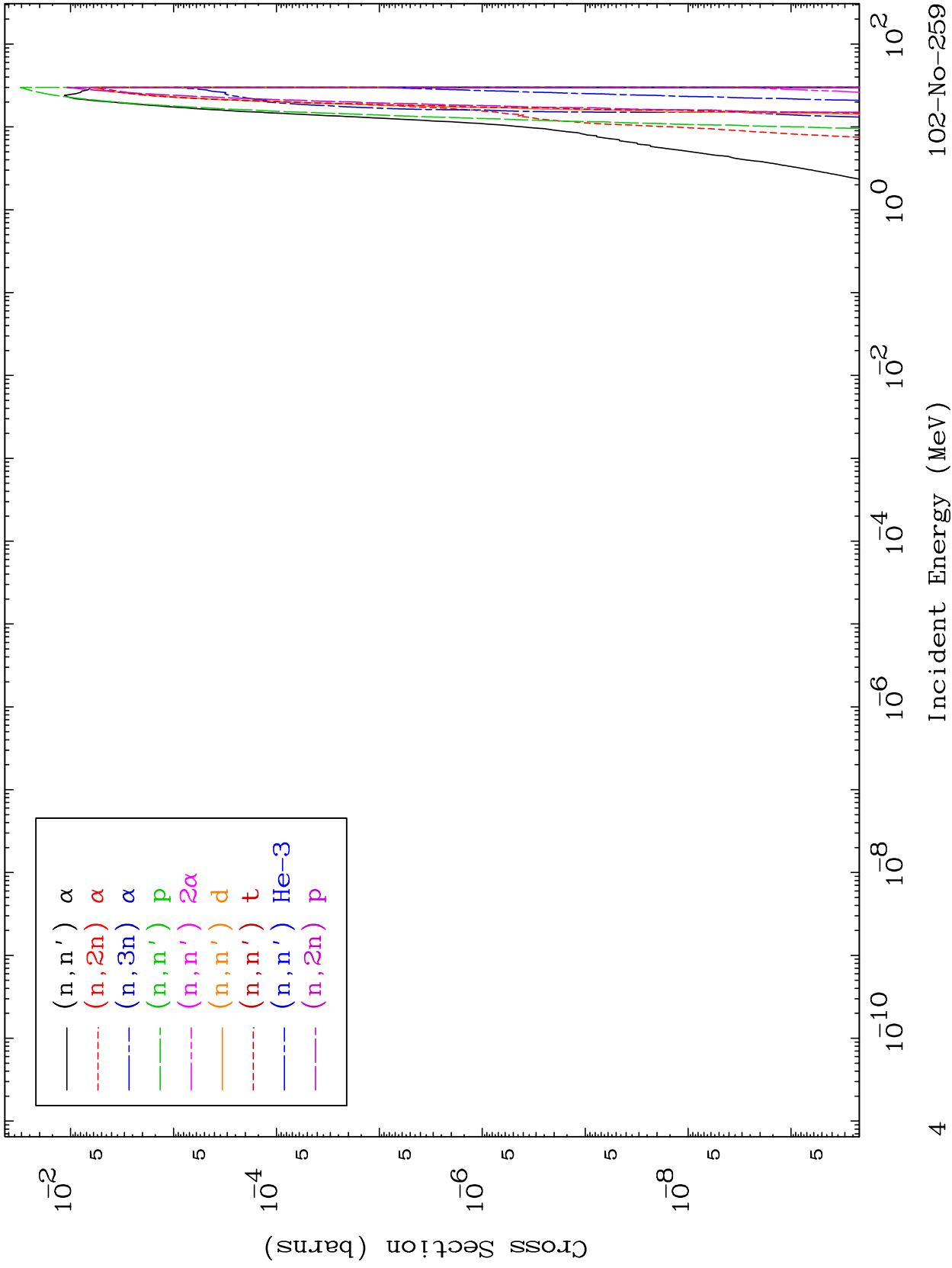




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Charged Particle  
293 Kelvin Cross Sections

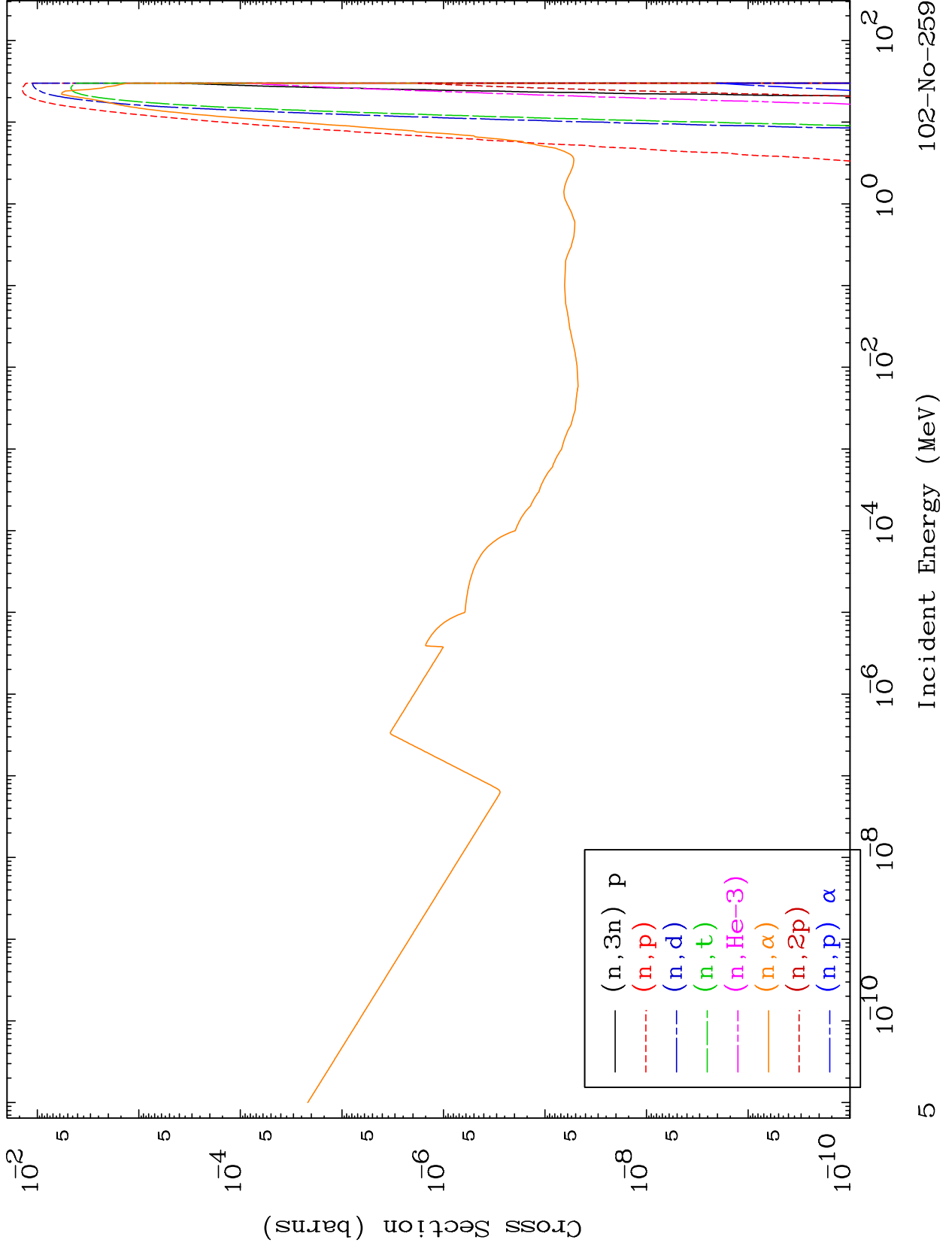
102-No-259

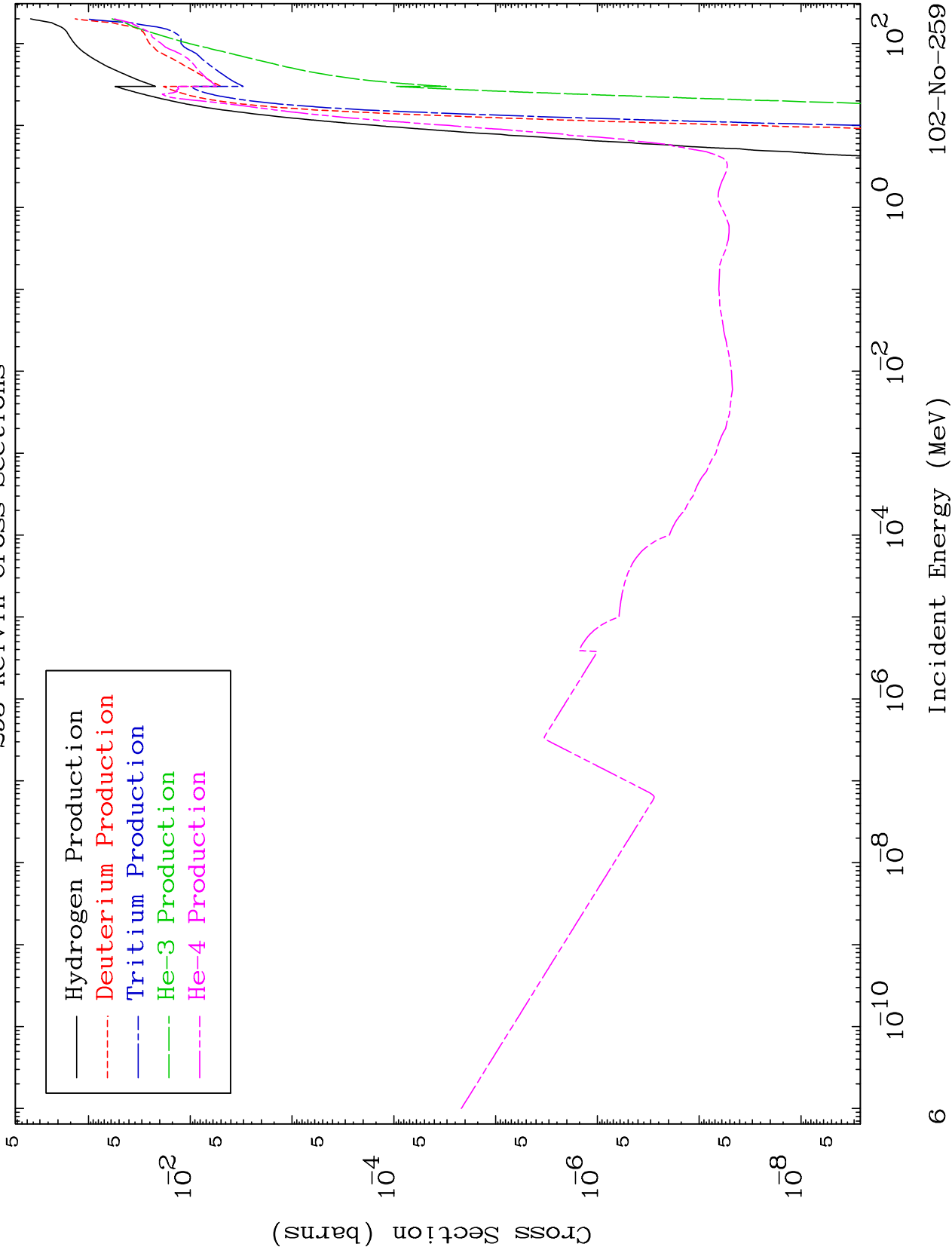


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Charged Particle  
293 Kelvin Cross Sections

102-No-259

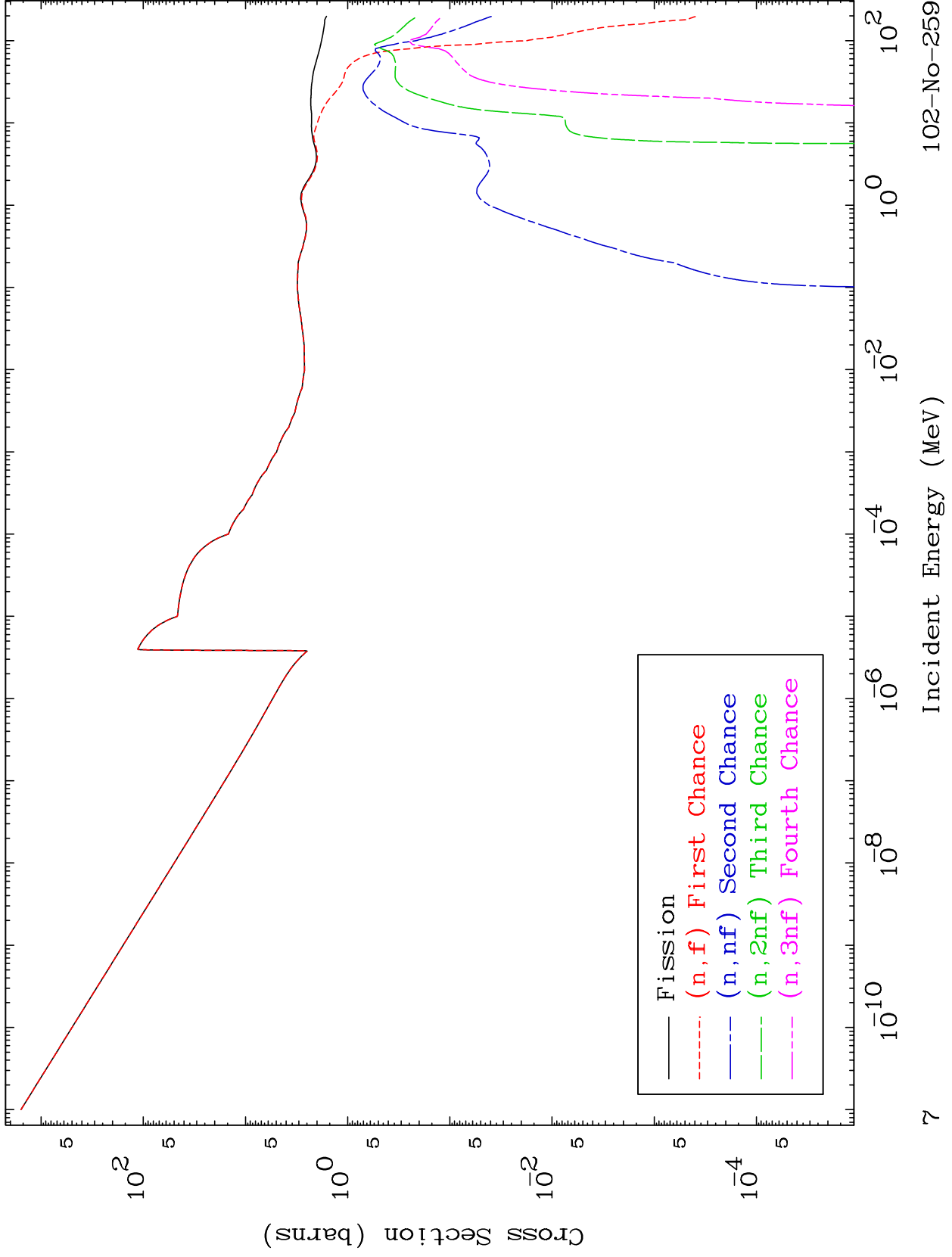




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Fission  
293 Kelvin Cross Sections

102-No-259



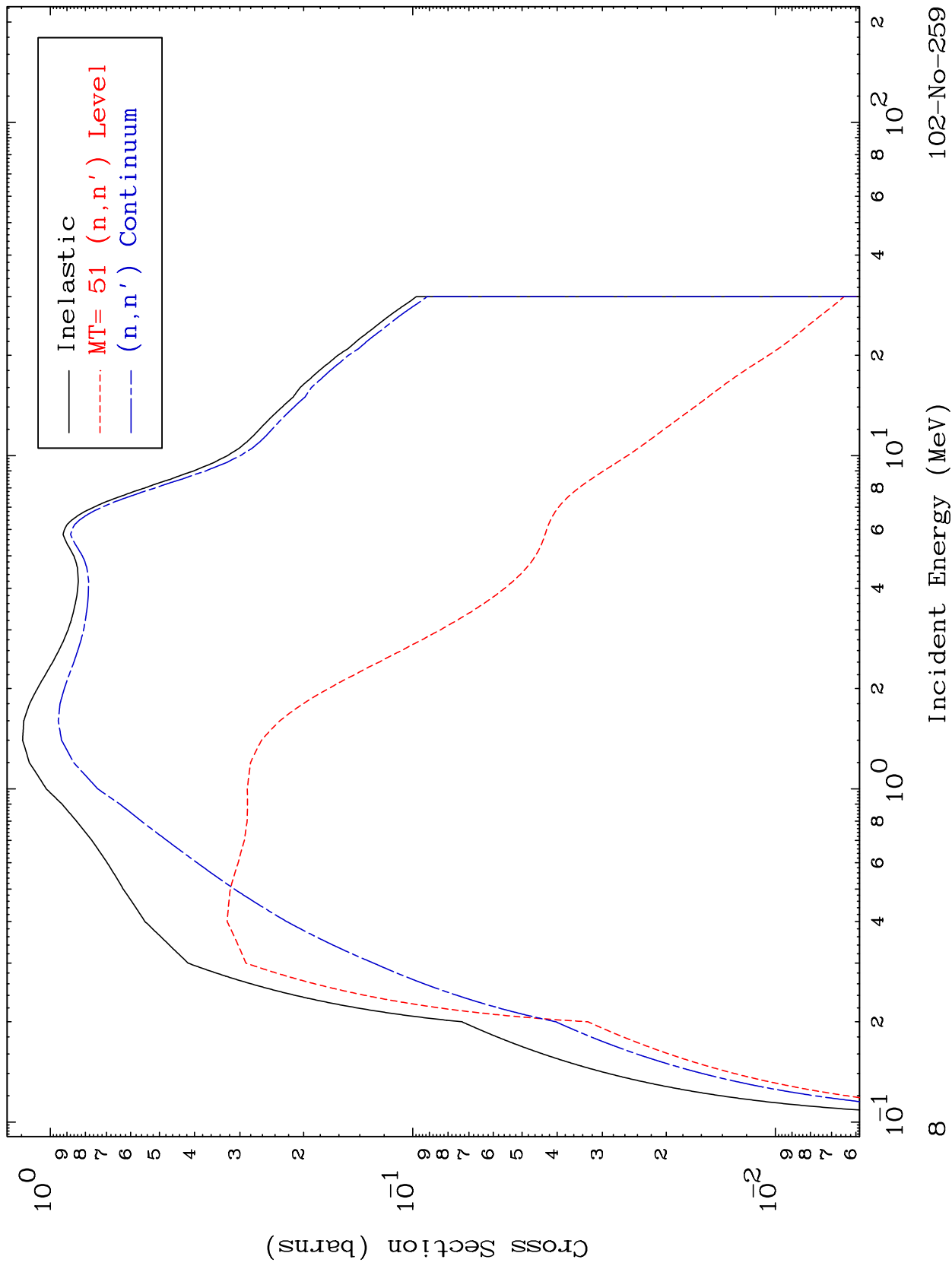


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(n,n') Level

102-No-259

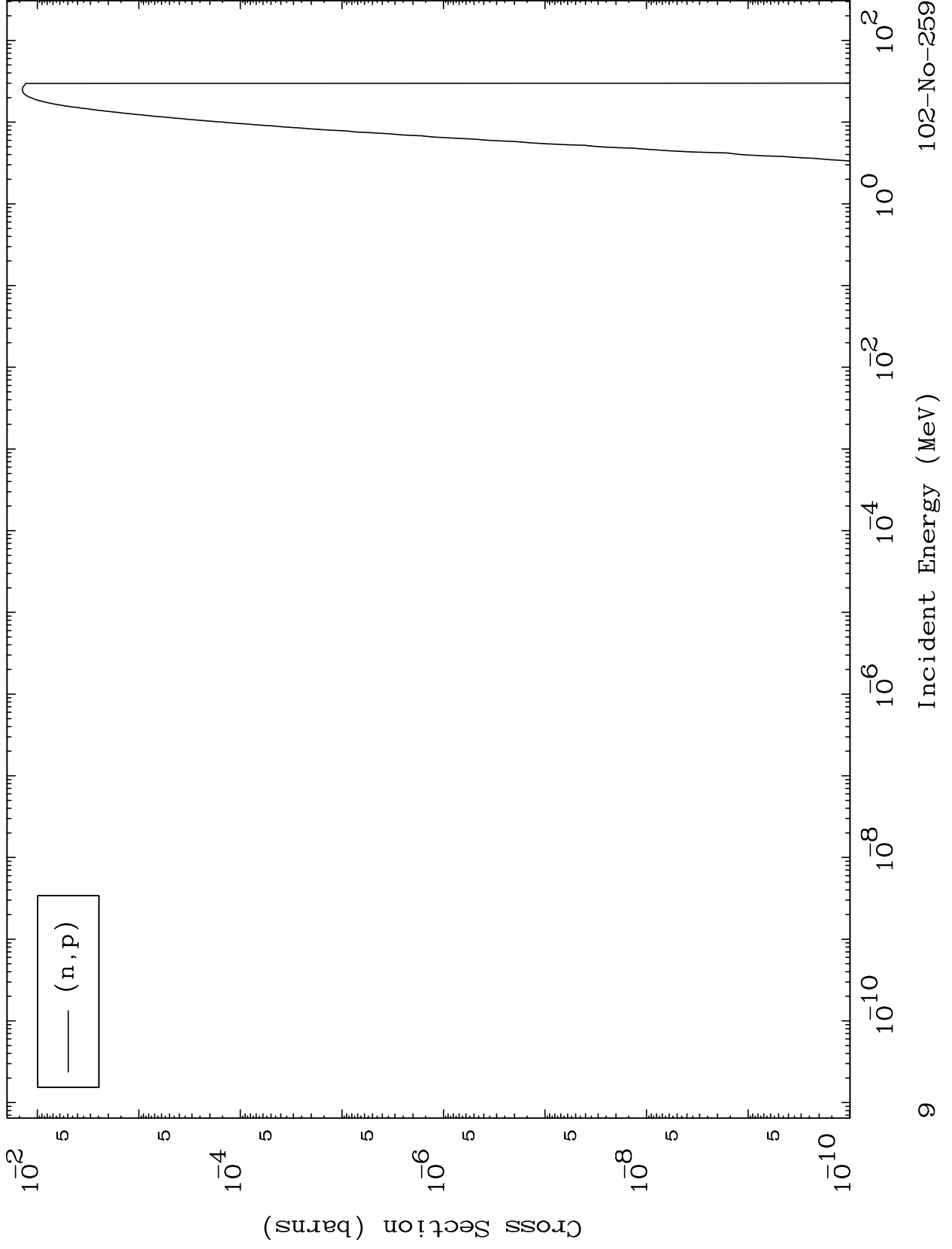
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

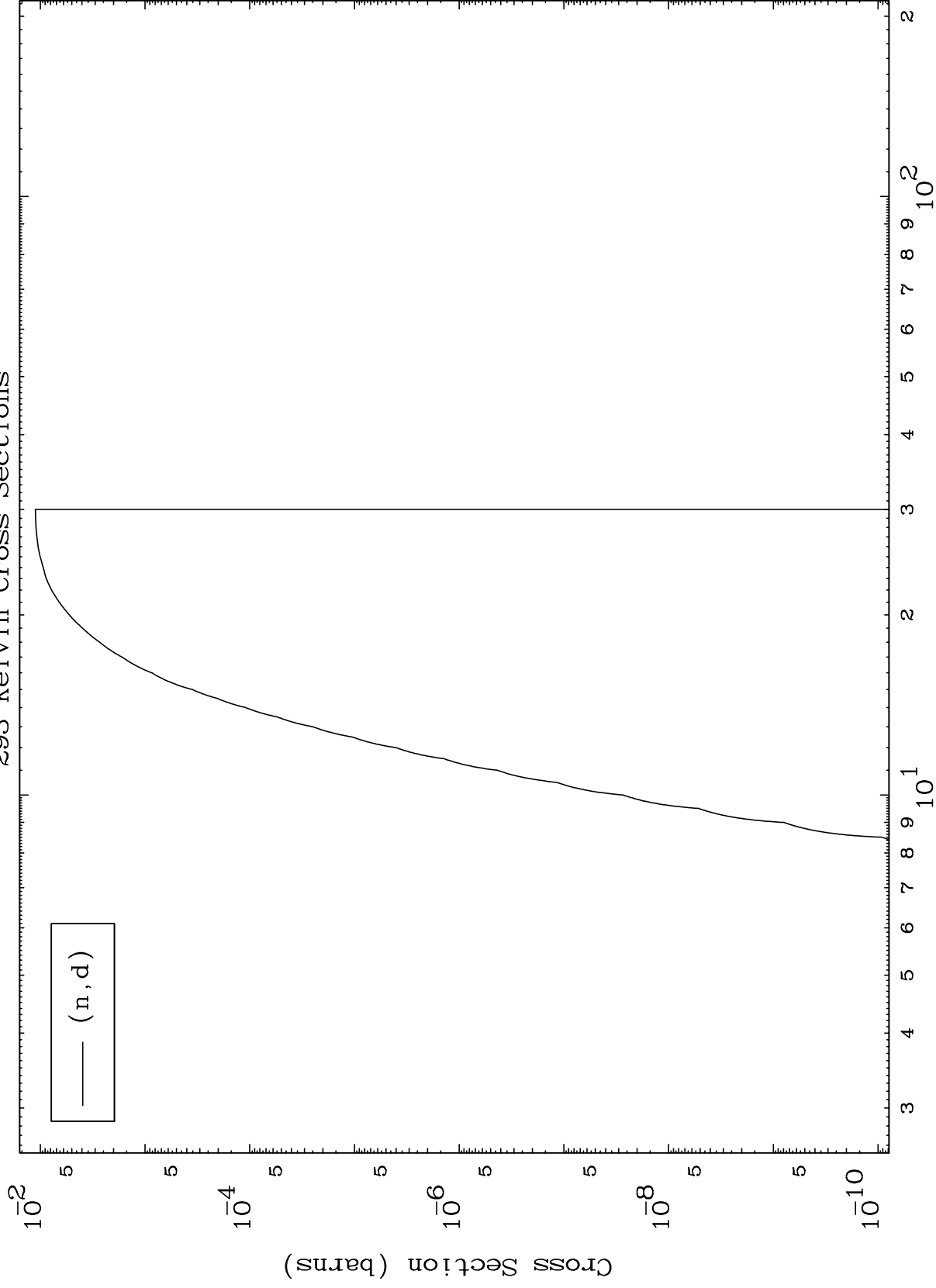
102-No-259



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(n,d) Levels  
293 Kelvin Cross Sections

102-No-259



10

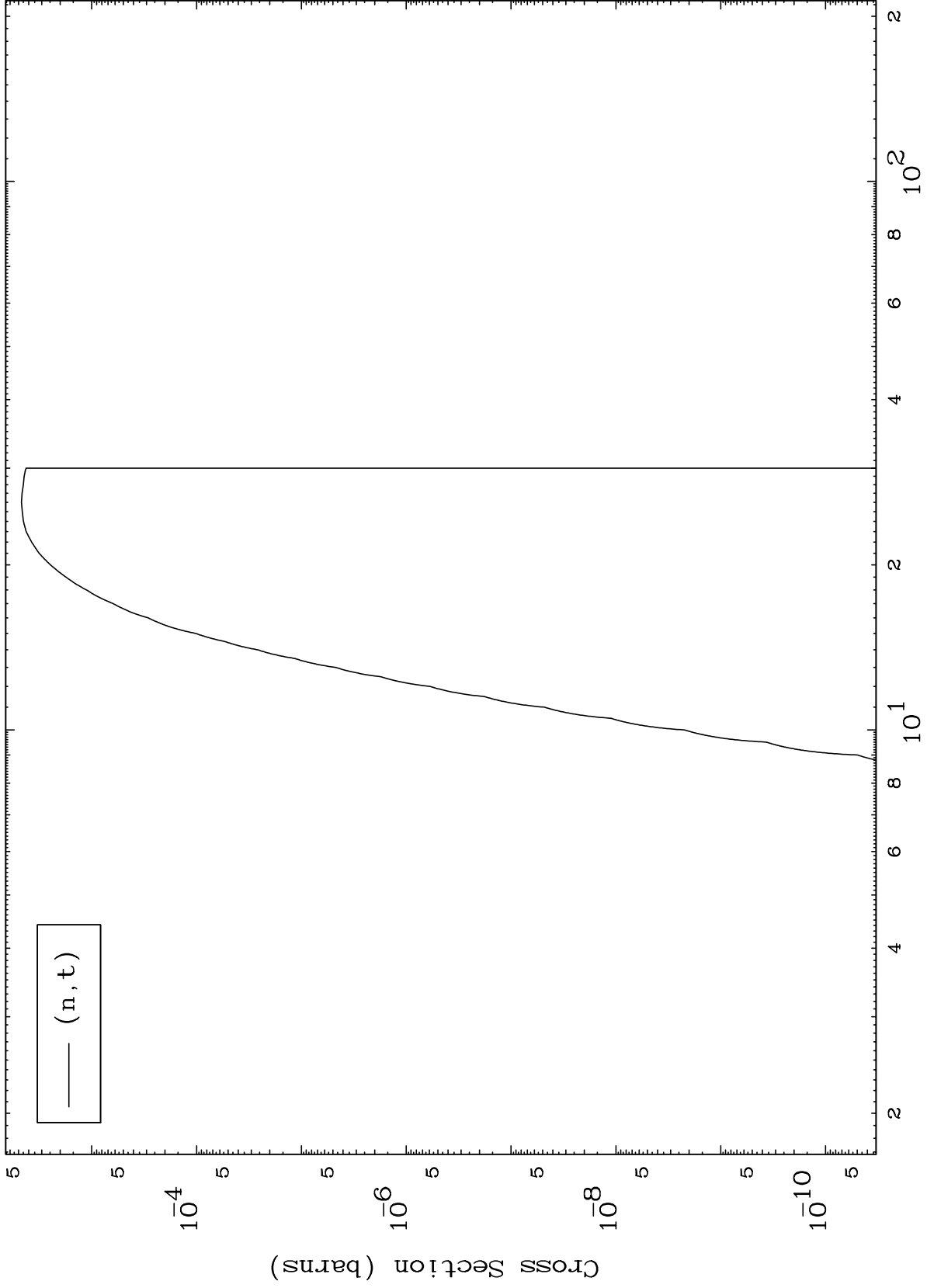
Incident Energy (MeV)

102-No-259

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(n,t) Levels  
293 Kelvin Cross Sections

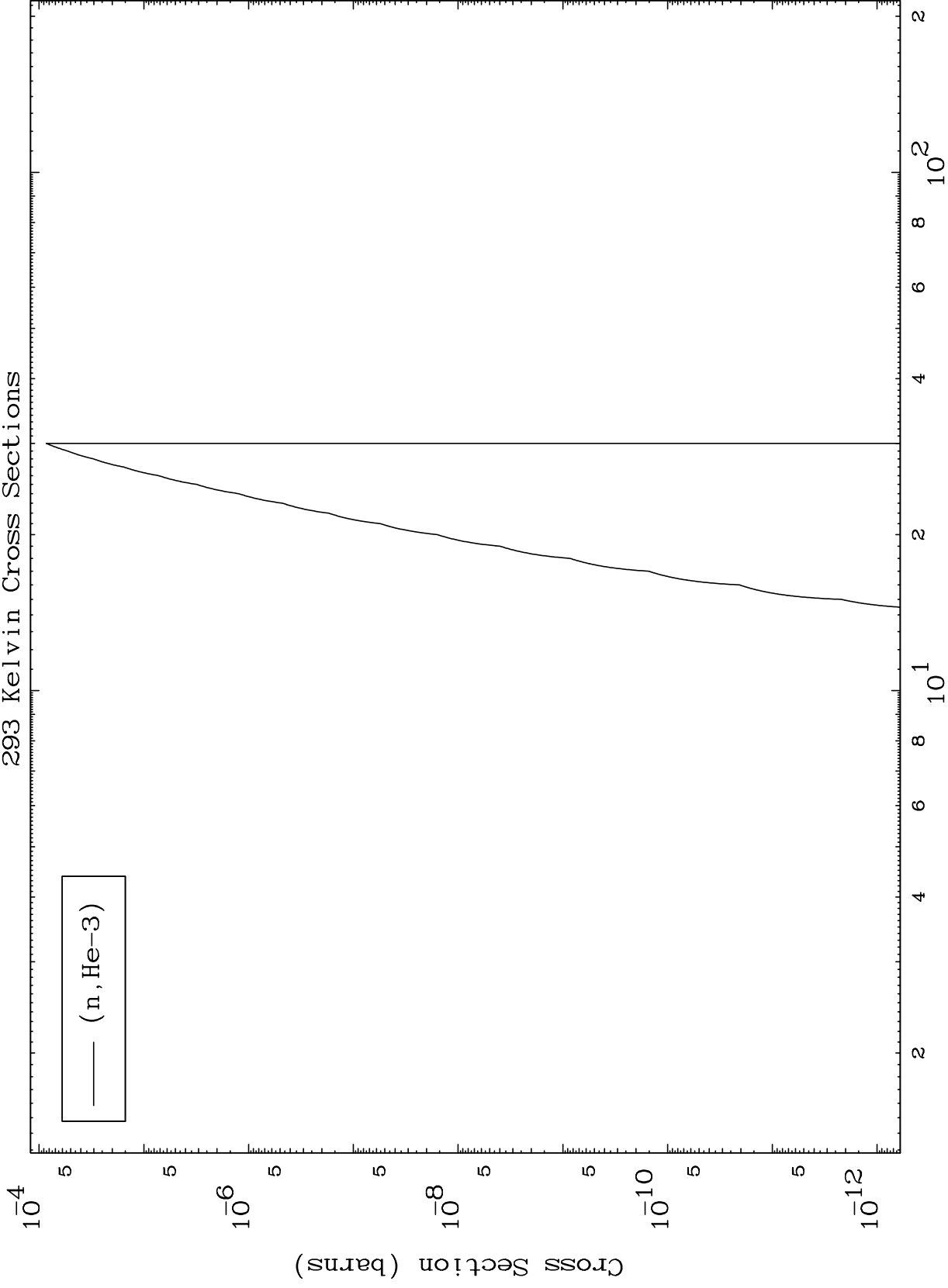
102-No-259



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(n,He3) Levels  
293 Kelvin Cross Sections

102-No-259



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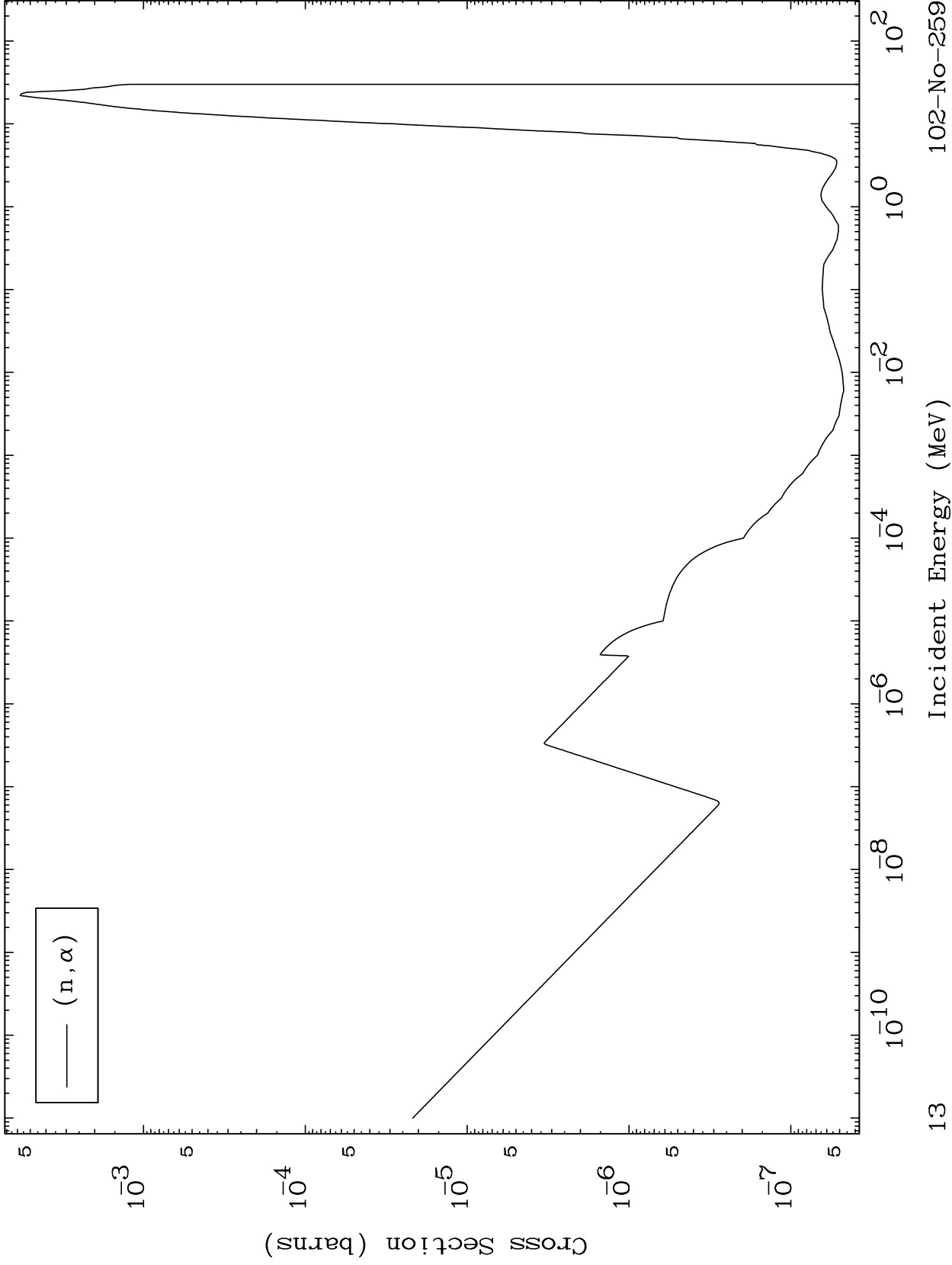
Incident Energy (MeV)

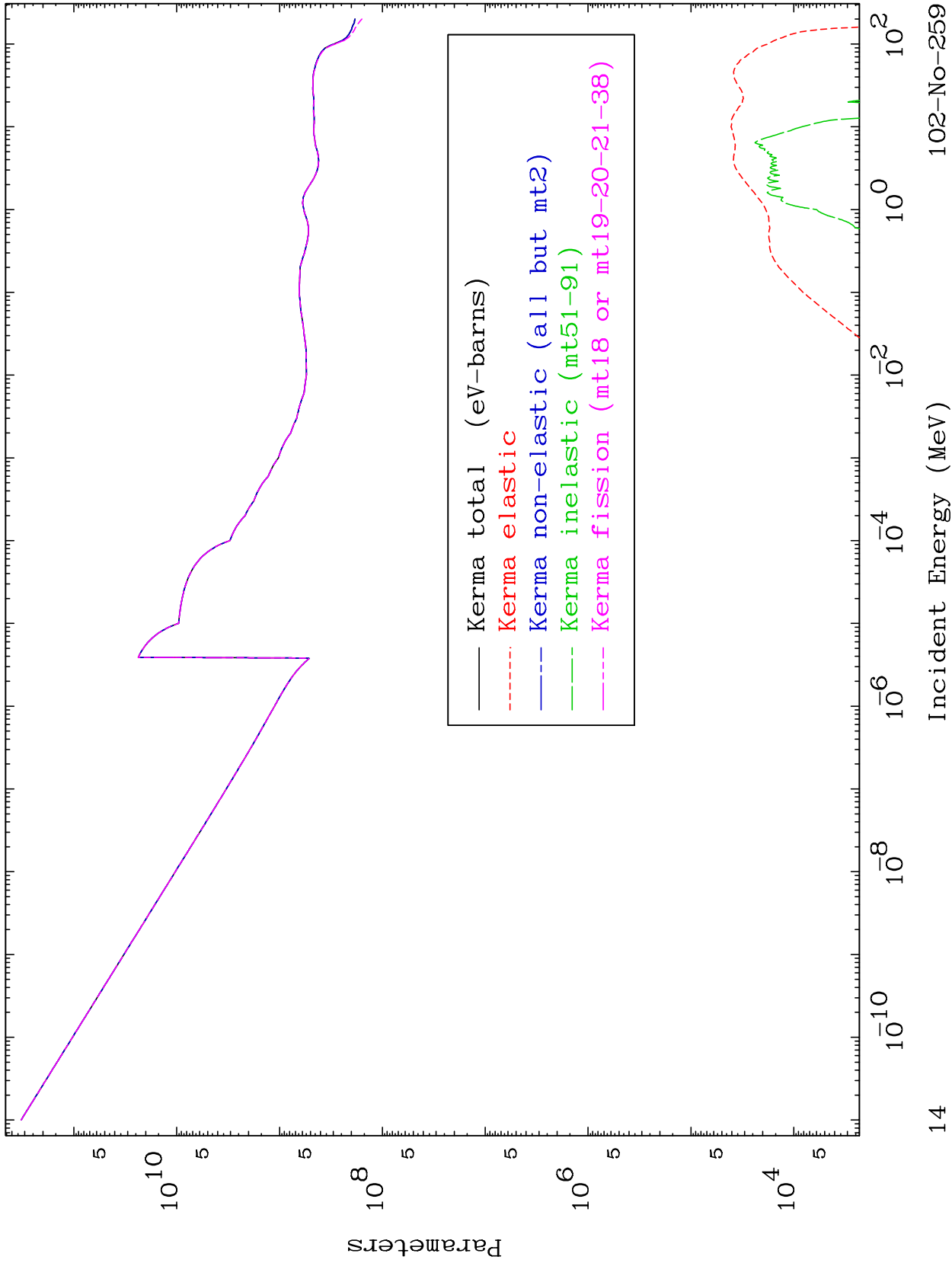
102-No-259

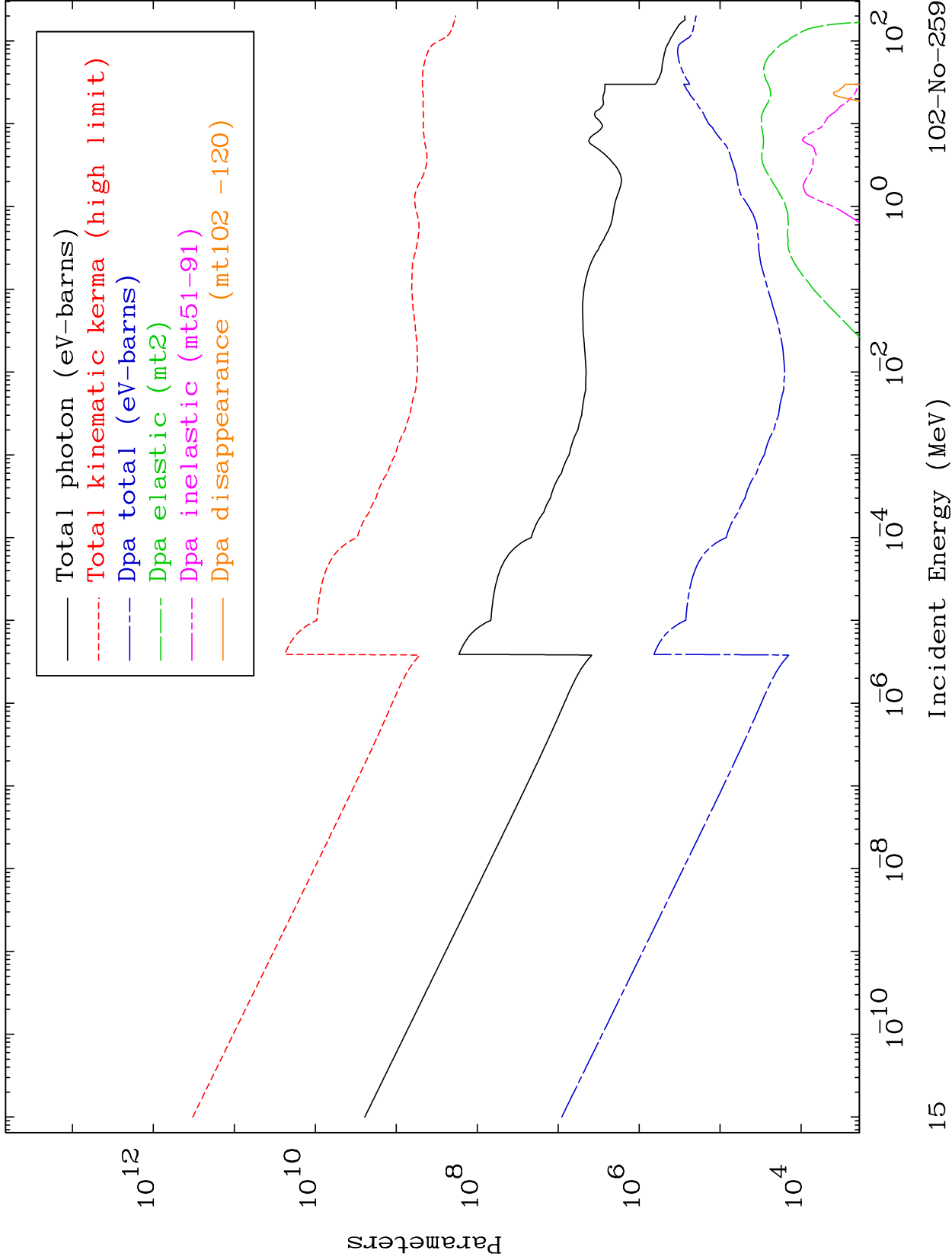
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

102-No-259





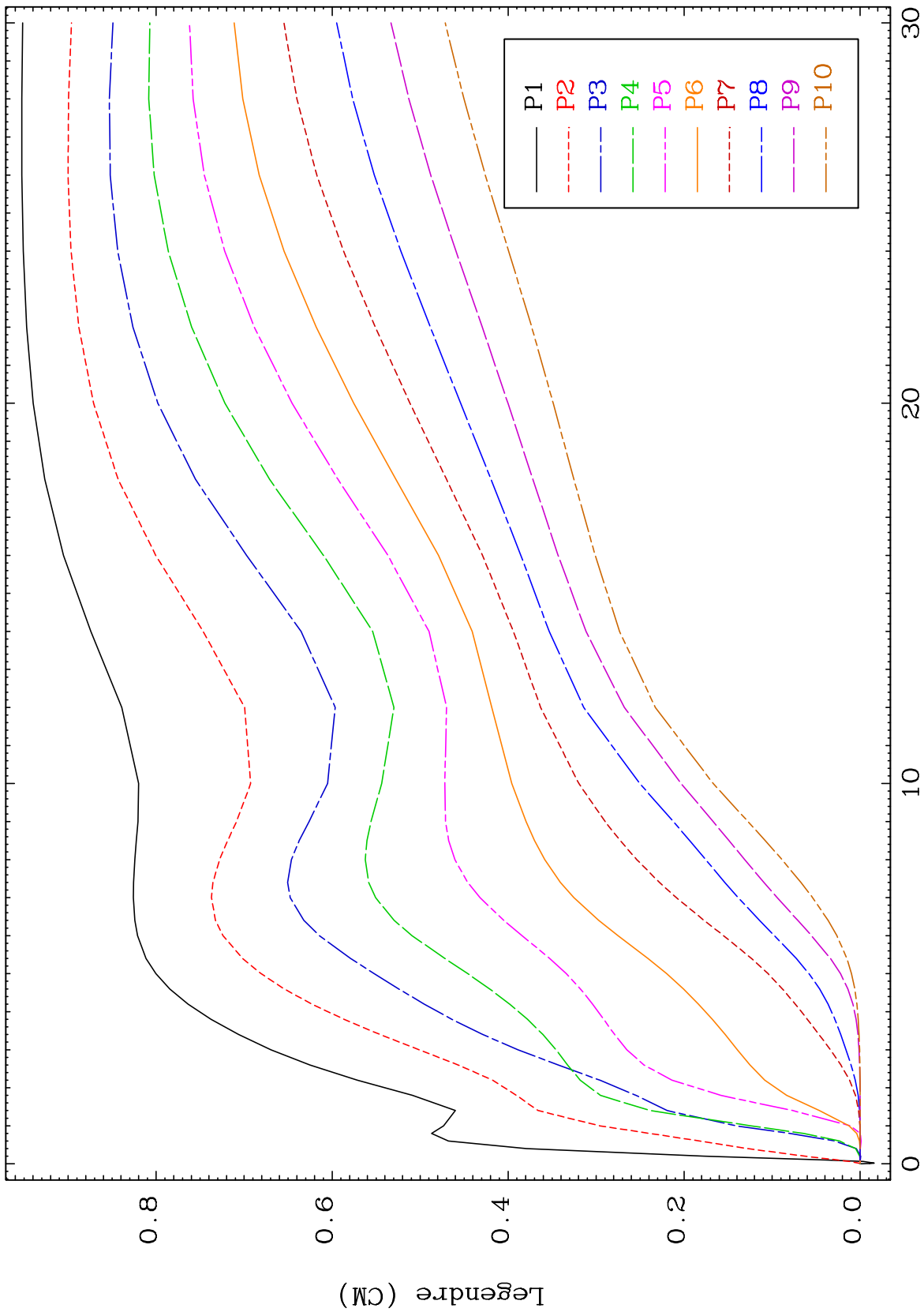




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Elastic Legendre Coefficients

102-No-259



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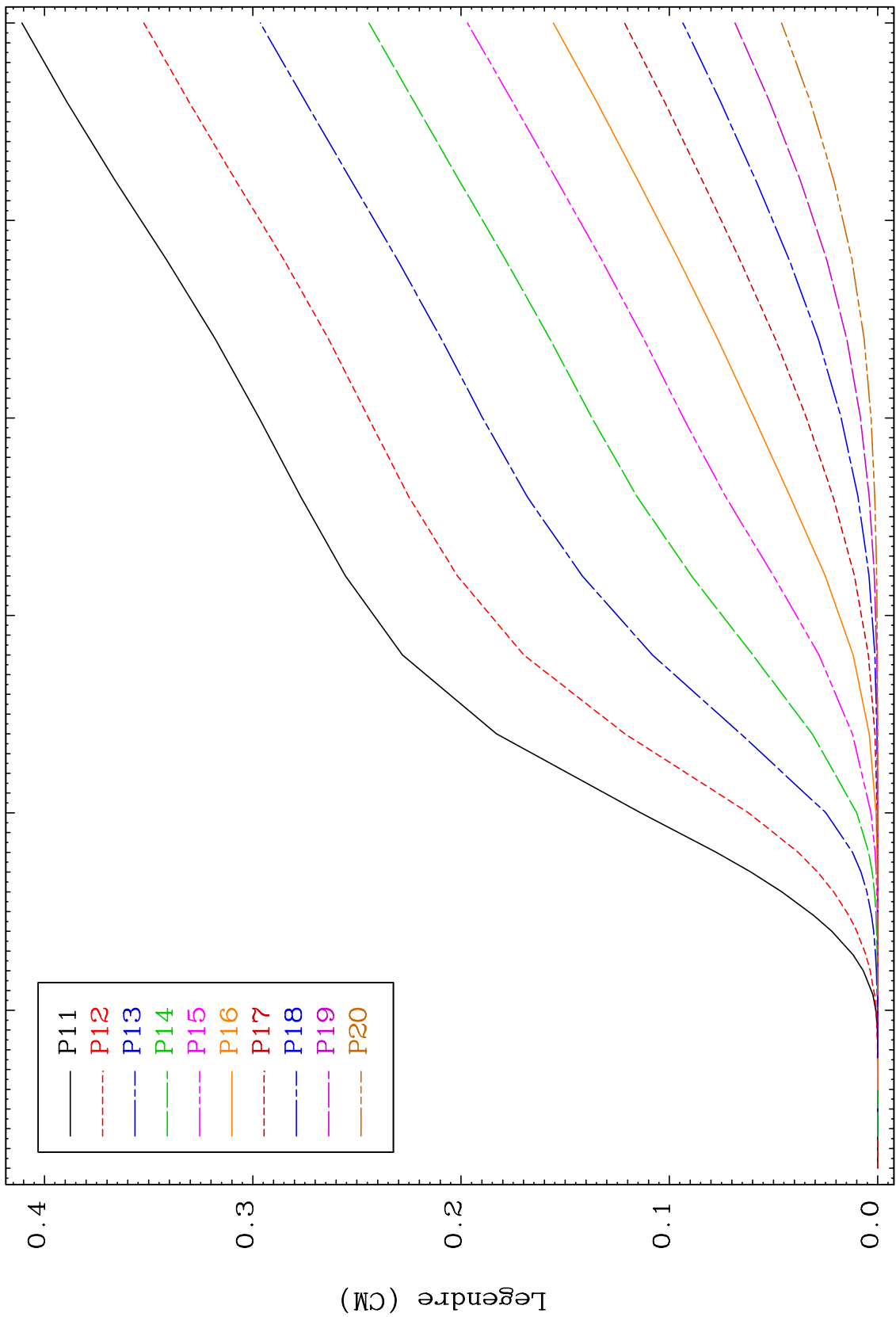
Incident Energy (MeV)

102-No-259

MAT 9977

Elastic Legendre Coefficients

102-No-259



17

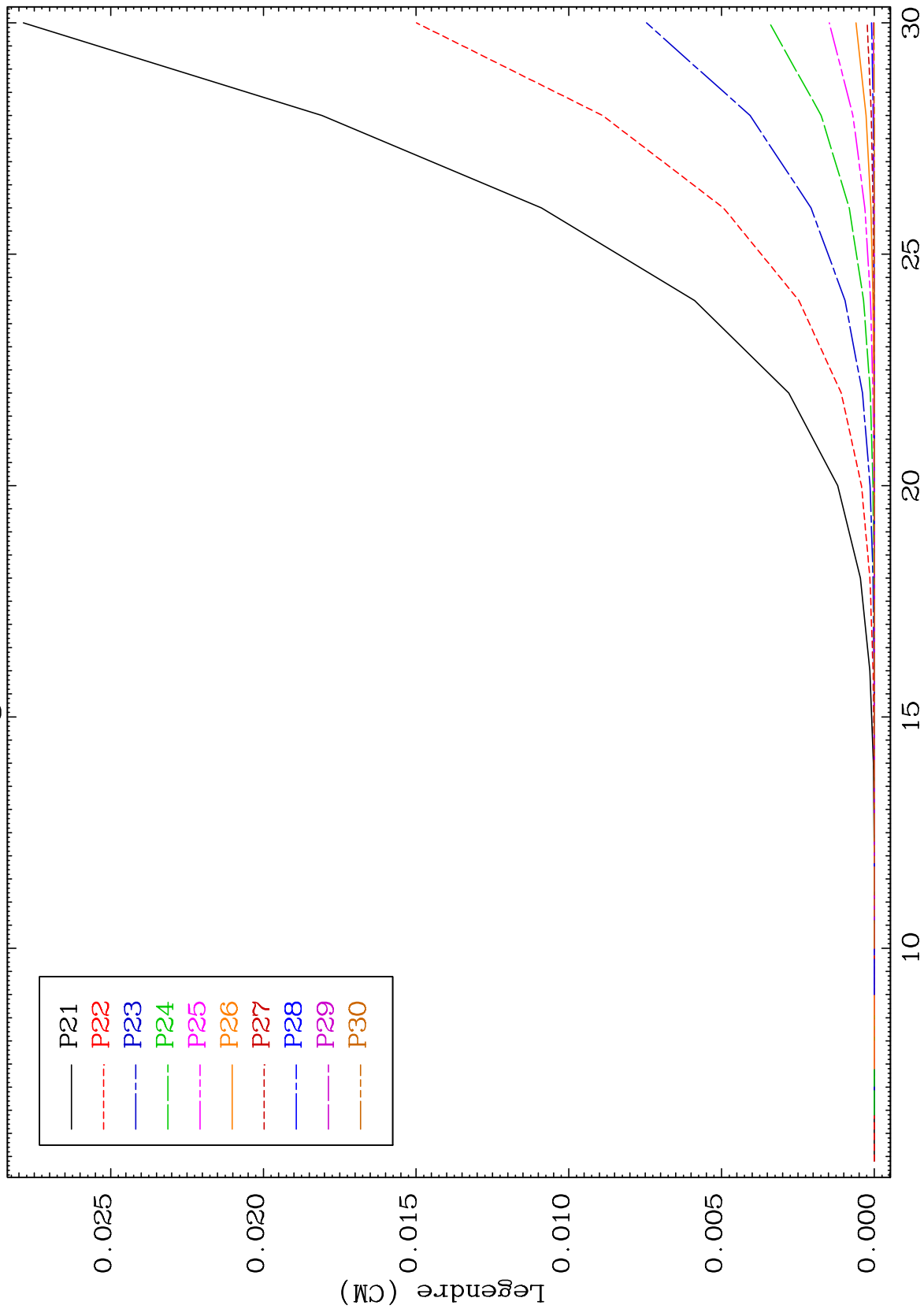
Incident Energy (MeV)

102-No-259

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Elastic Legendre Coefficients

102-No-259



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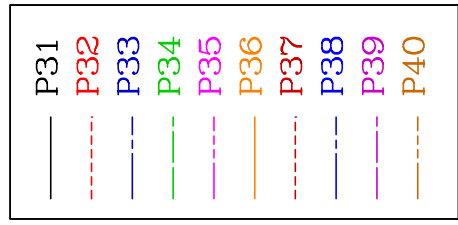
Incident Energy (MeV)

102-No-259

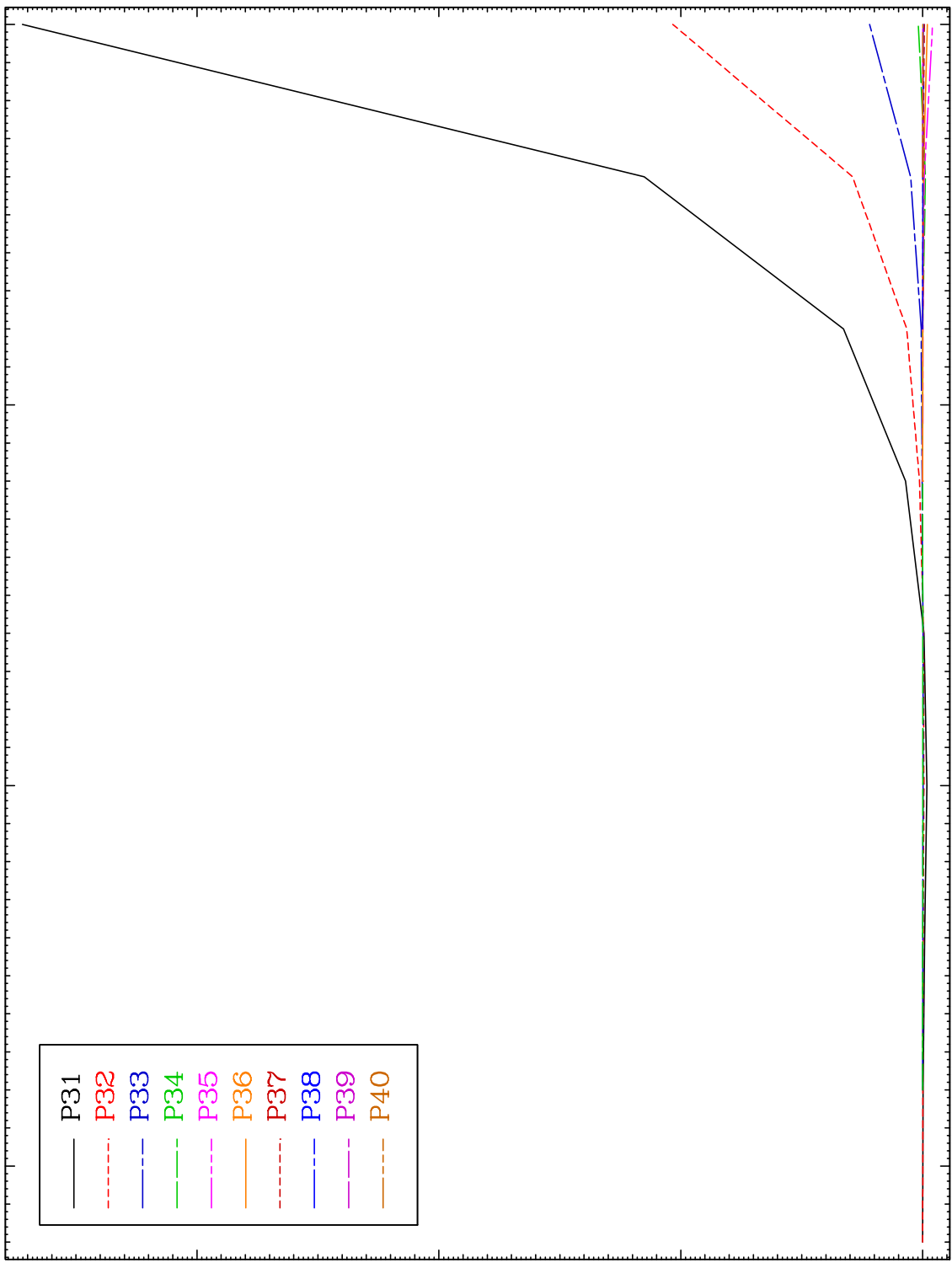
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Elastic  
Legendre Coefficients

102-No-259



Legendre (CM)  
 $\times 10^{-6}$



19

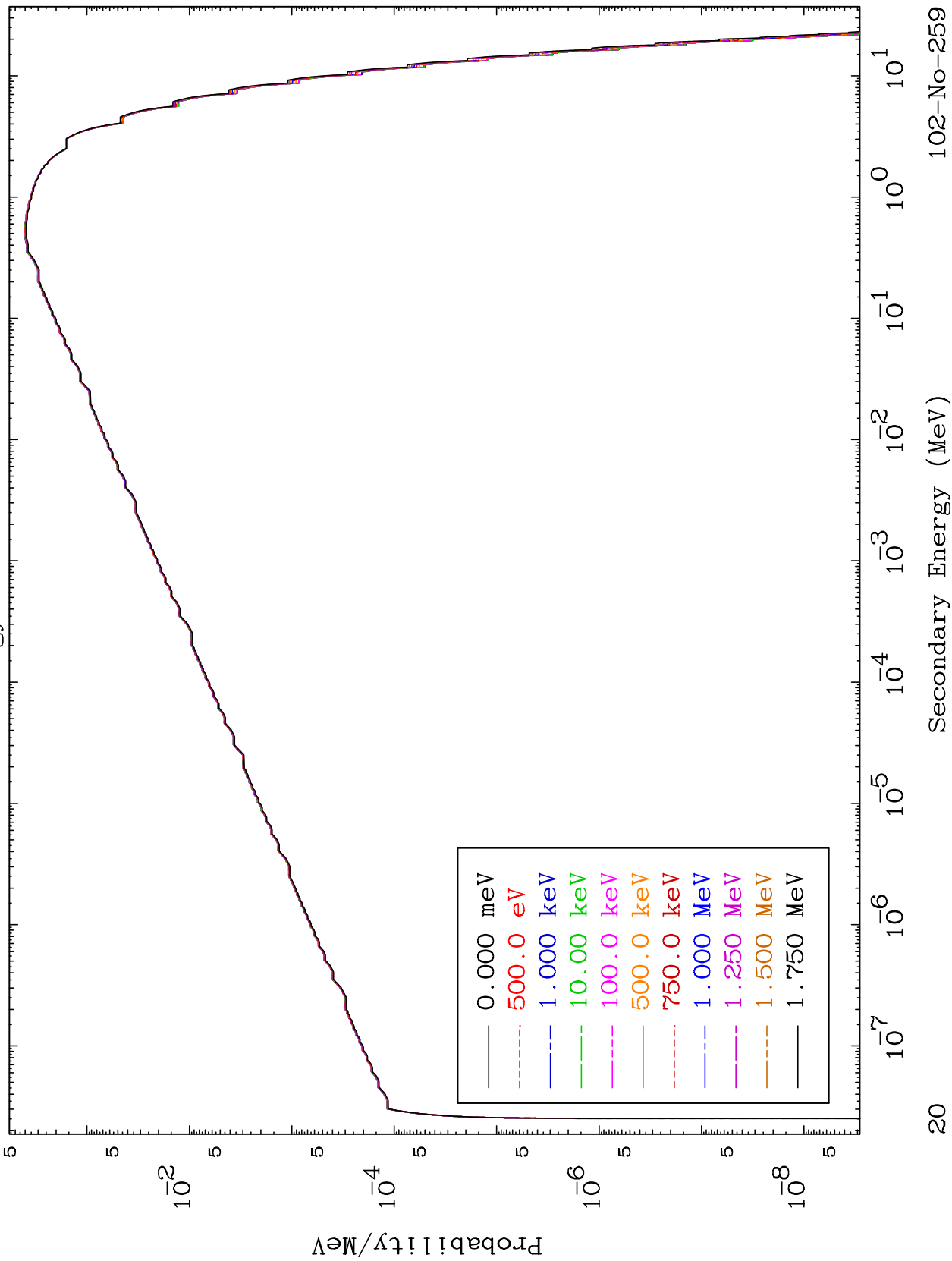
Incident Energy (MeV)

102-No-259

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### Fission Energy Distribution

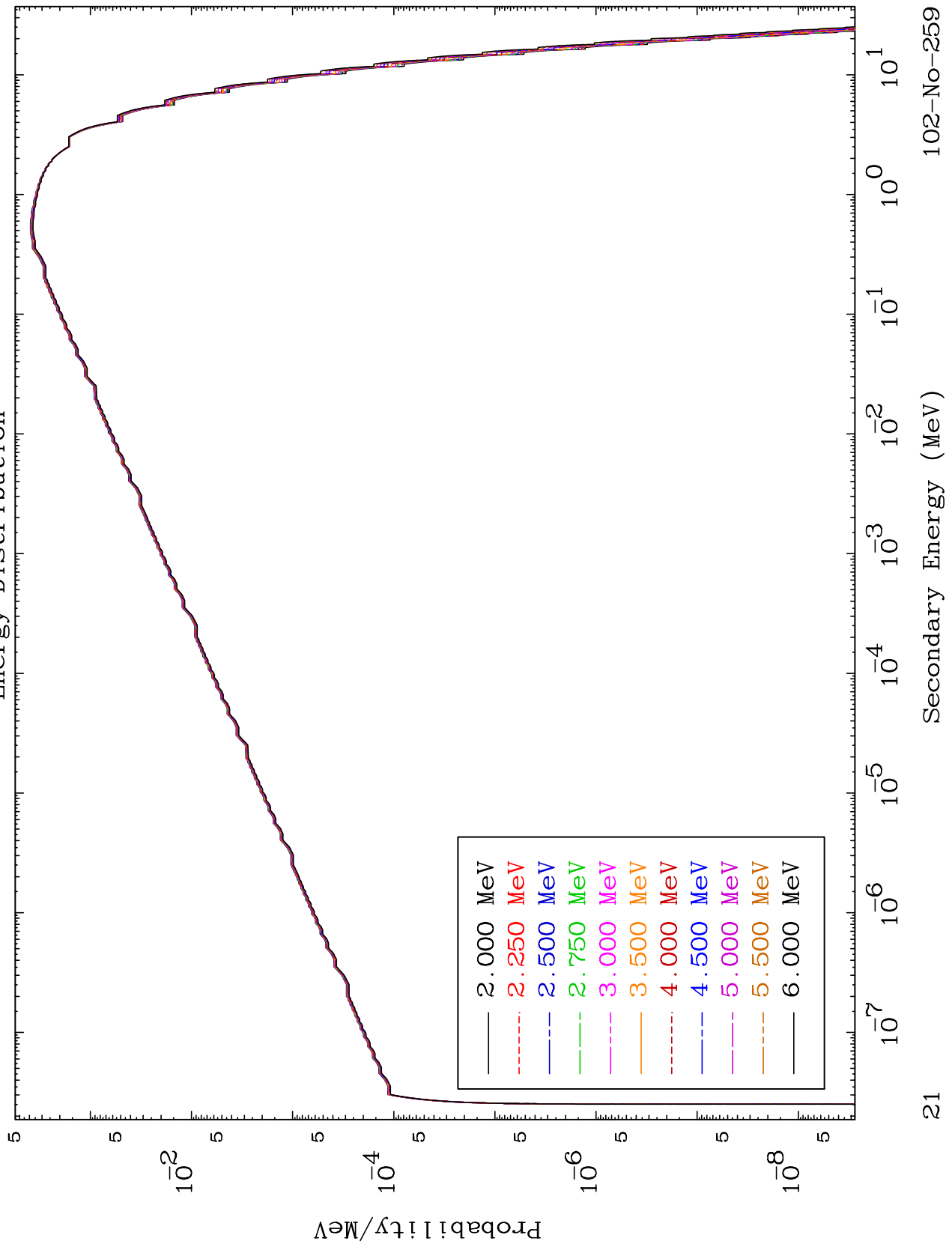
102-No-259



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### Fission Energy Distribution

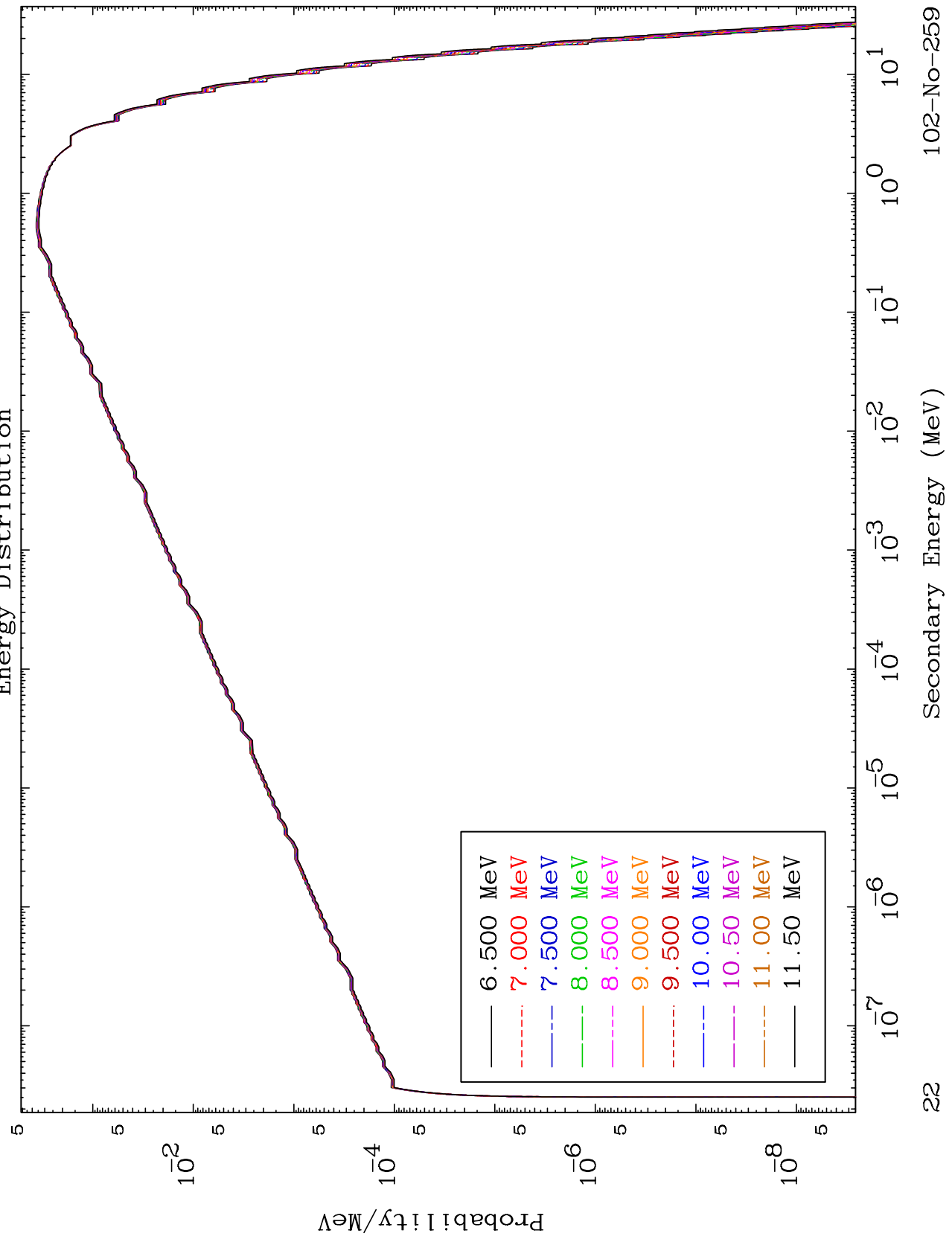
102-No-259



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### Fission Energy Distribution

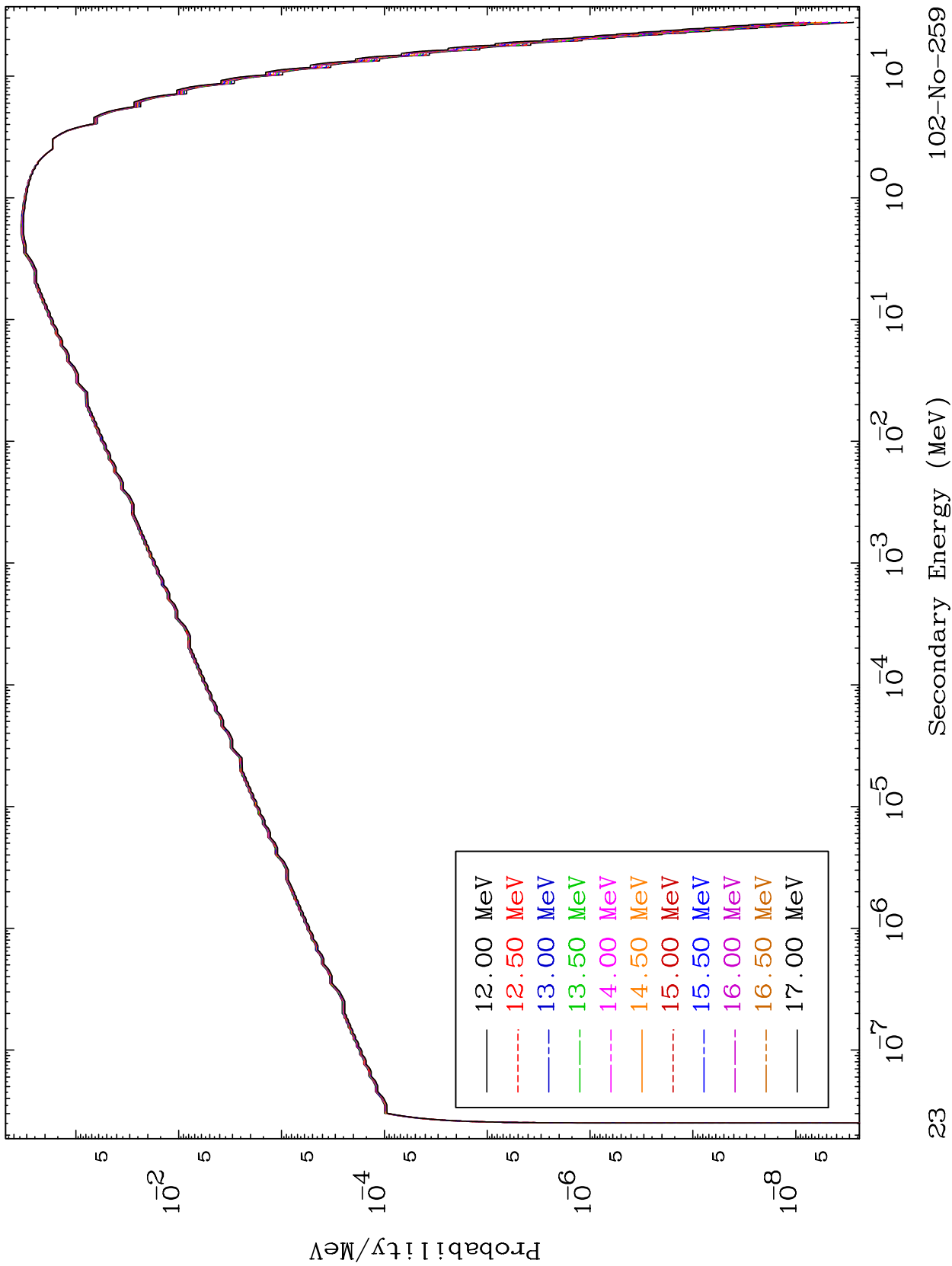
102-No-259



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### Fission Energy Distribution

102-No-259

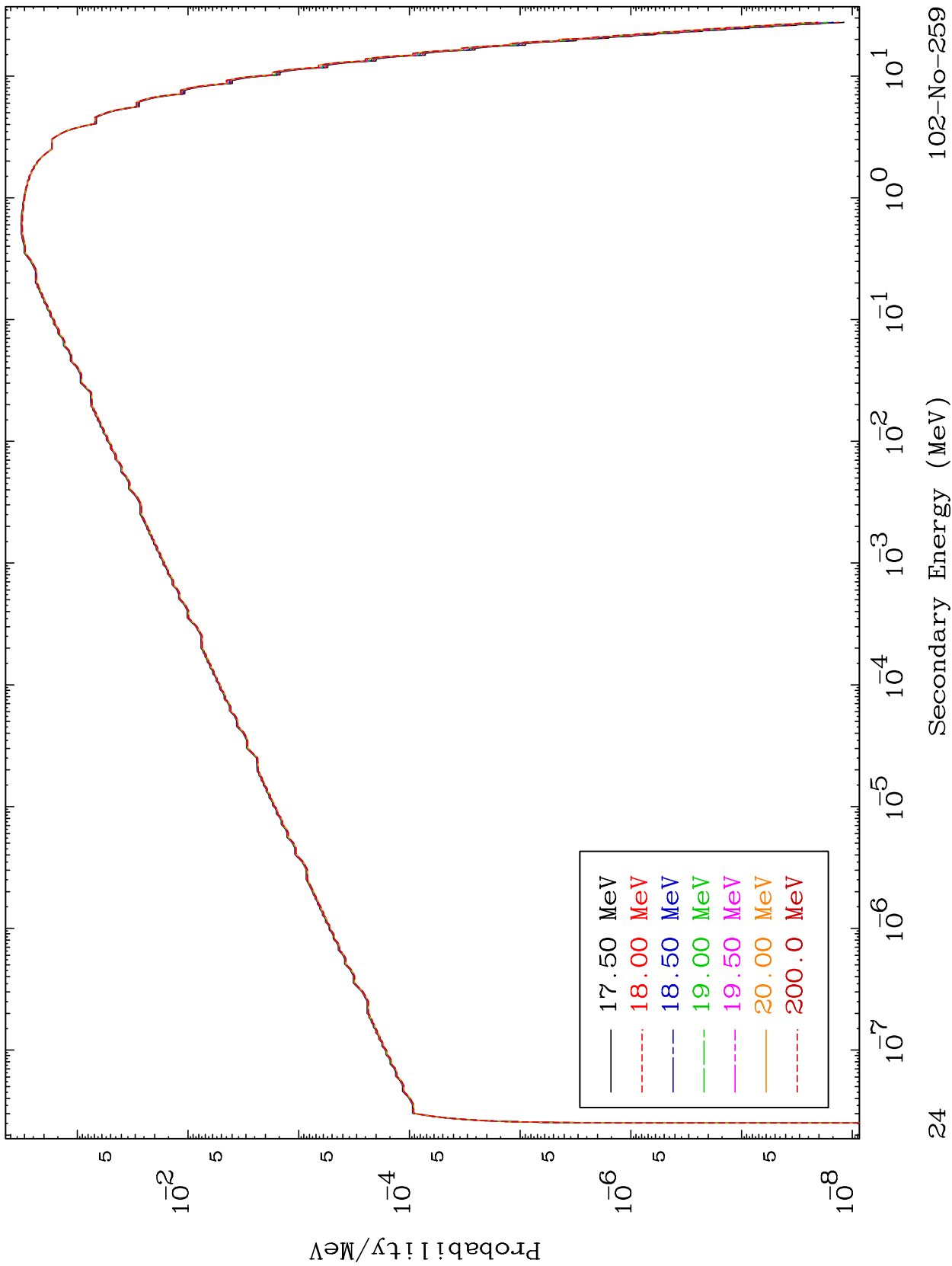




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### Fission Energy Distribution

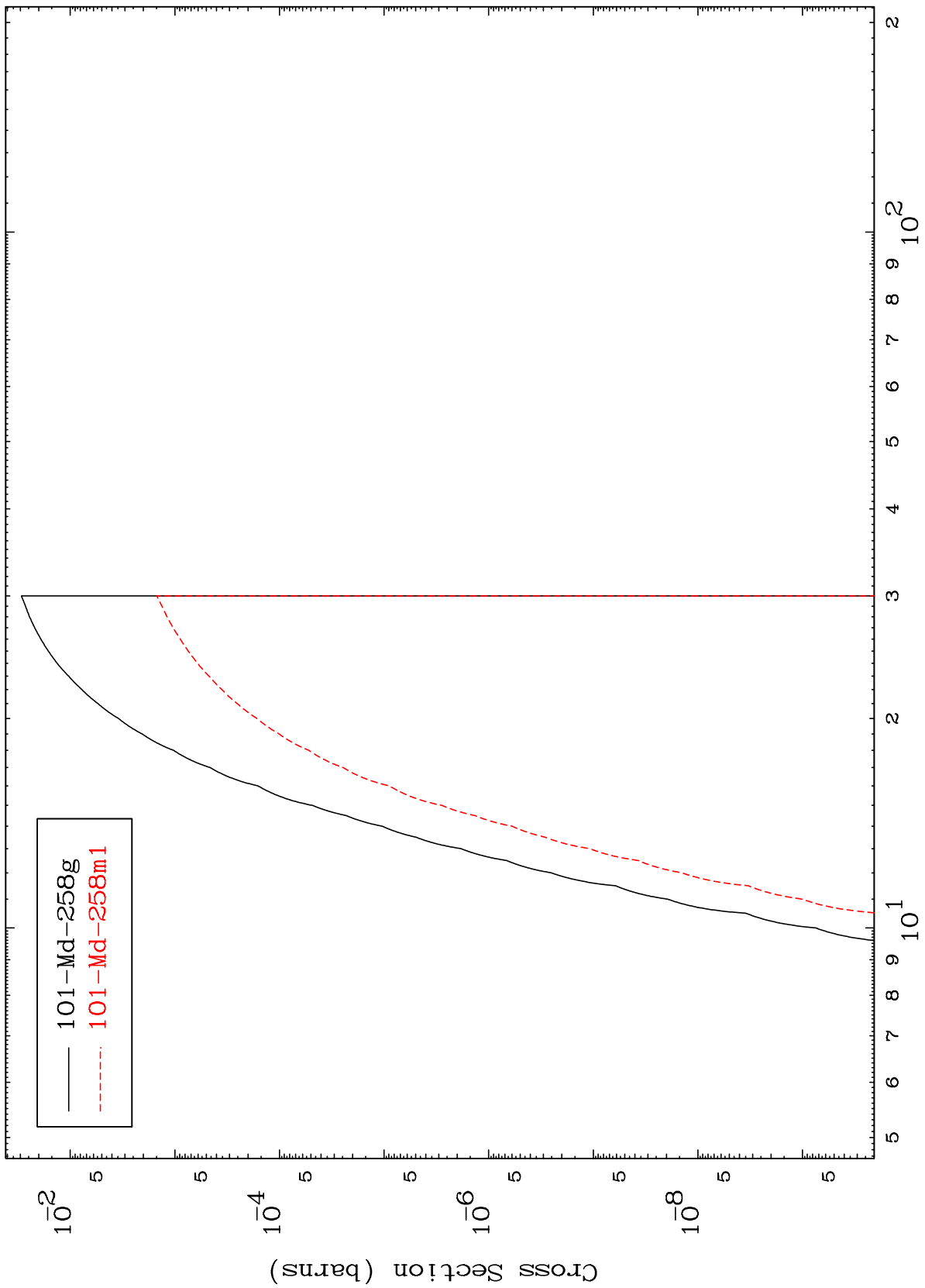
102-No-259



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102-No-259

(n,n') p  
Radionuclide Production Cross Section



— 101-Md-258g  
- - - 101-Md-258m1

25

Incident Energy (MeV)

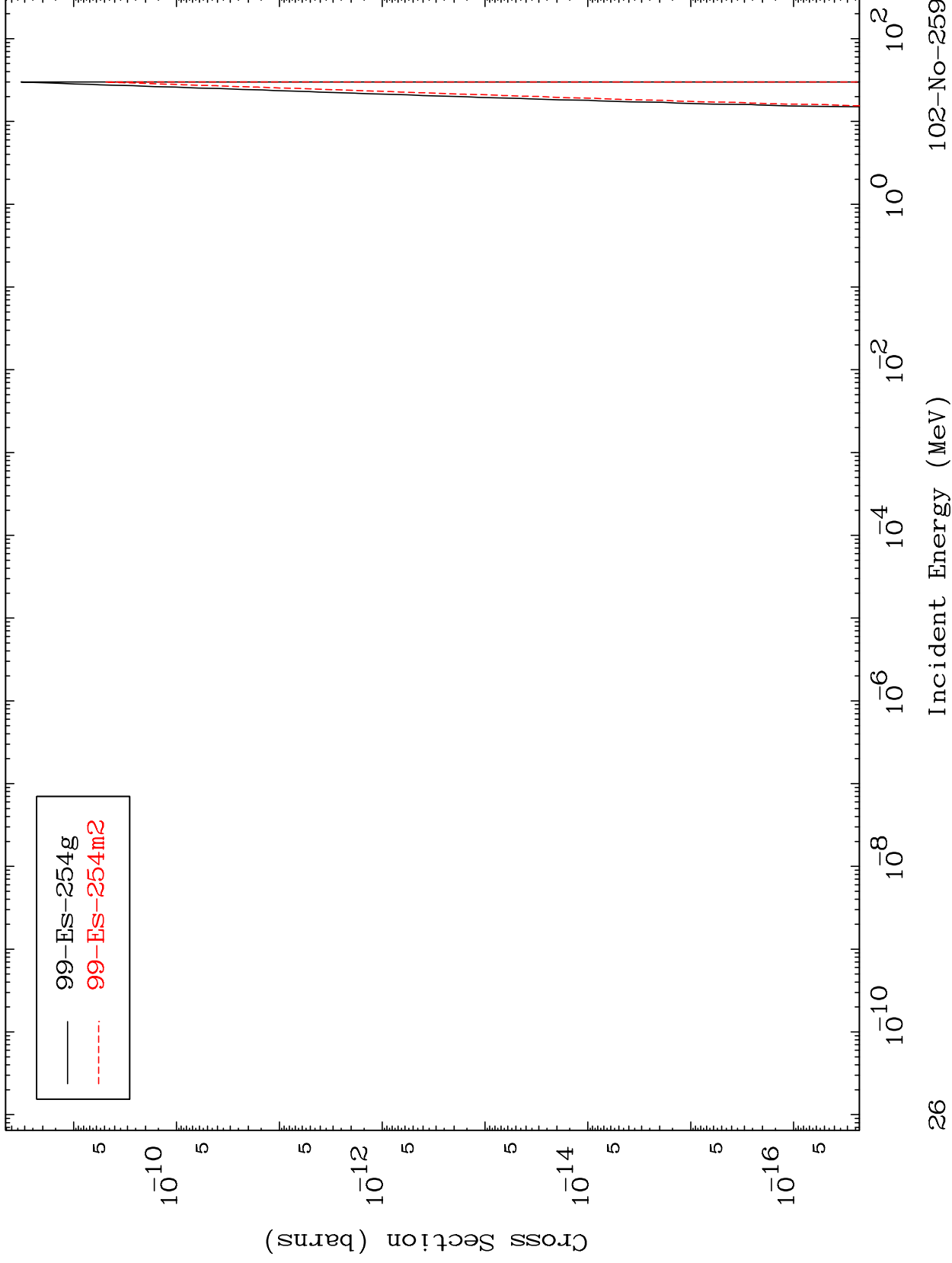
102-No-259

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(n,n') p  $\alpha$

102-No-259

Radionuclide Production Cross Section



26

Incident Energy (MeV)

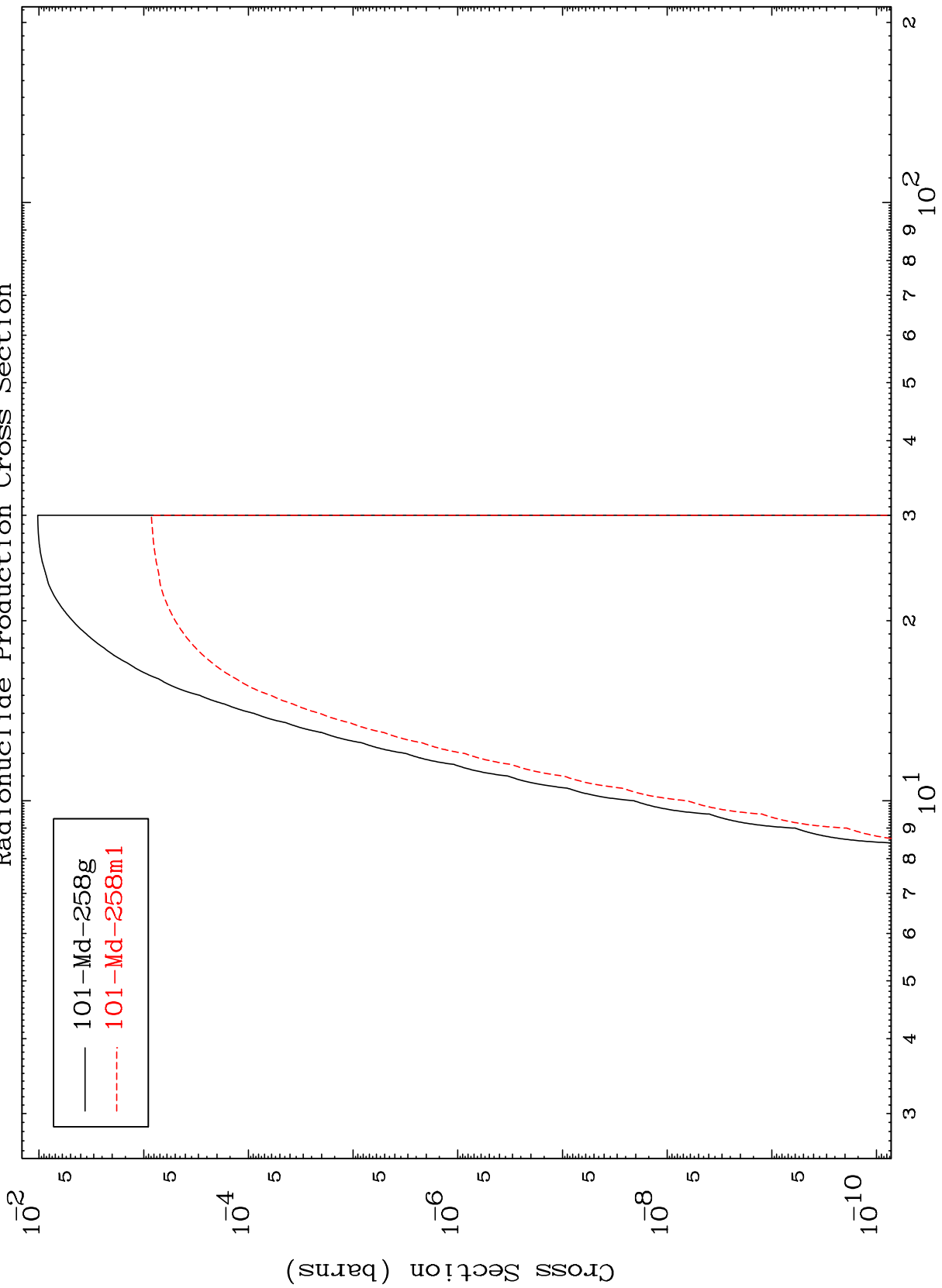
102-No-259

MAT 9977

(n,d)

102-No-259

Radionuclide Production Cross Section



Legend:  
— 101-Md-258g  
- - - 101-Md-258m1

Incident Energy (MeV)

102-No-259

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