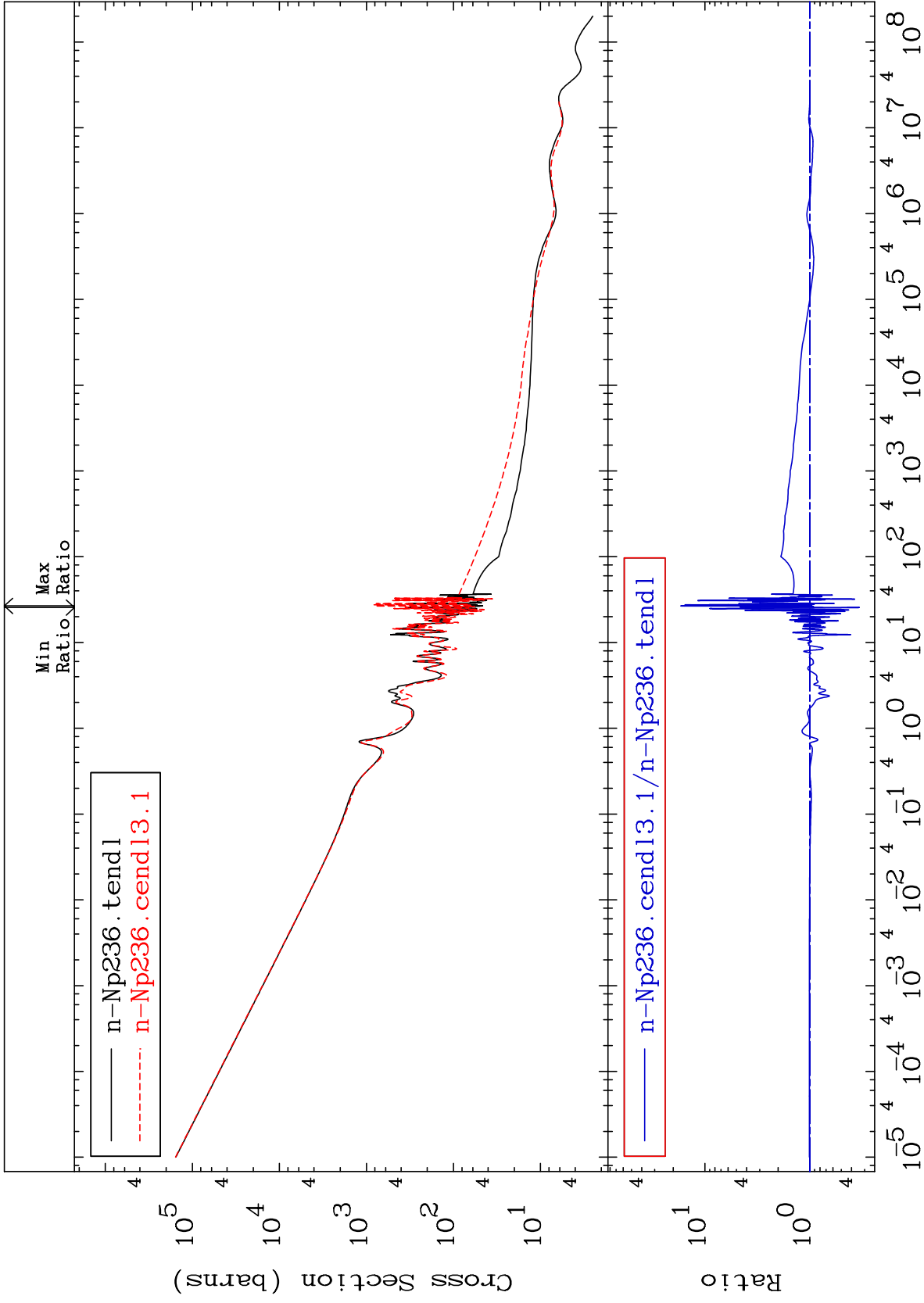


MAT 9343

Total
Cross Section

93-Np-236
-66.38 To 1610. %



Incident Energy (eV)

93-Np-236

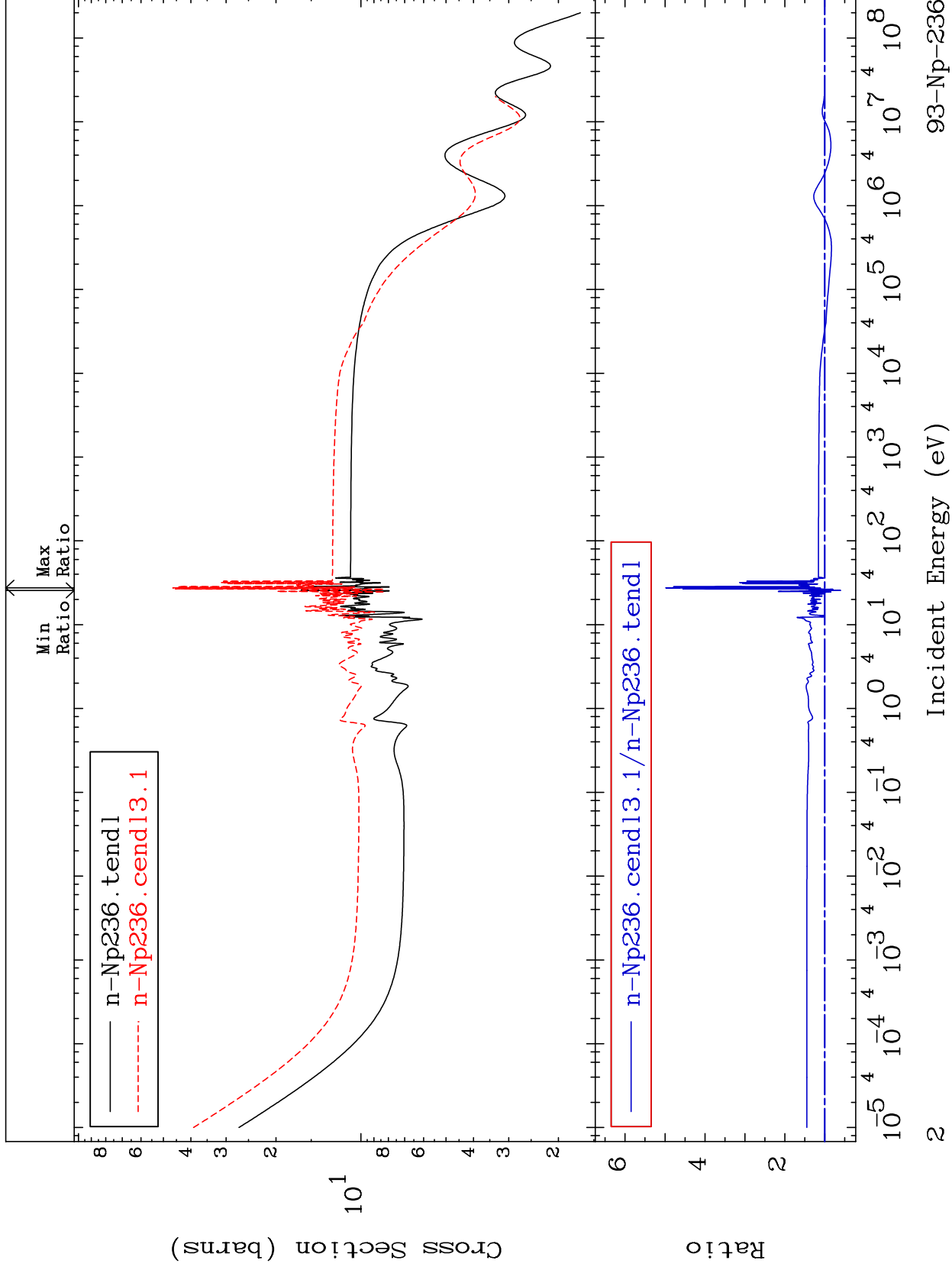
MAT 9343

Elastic

93-Np-236

Cross Section

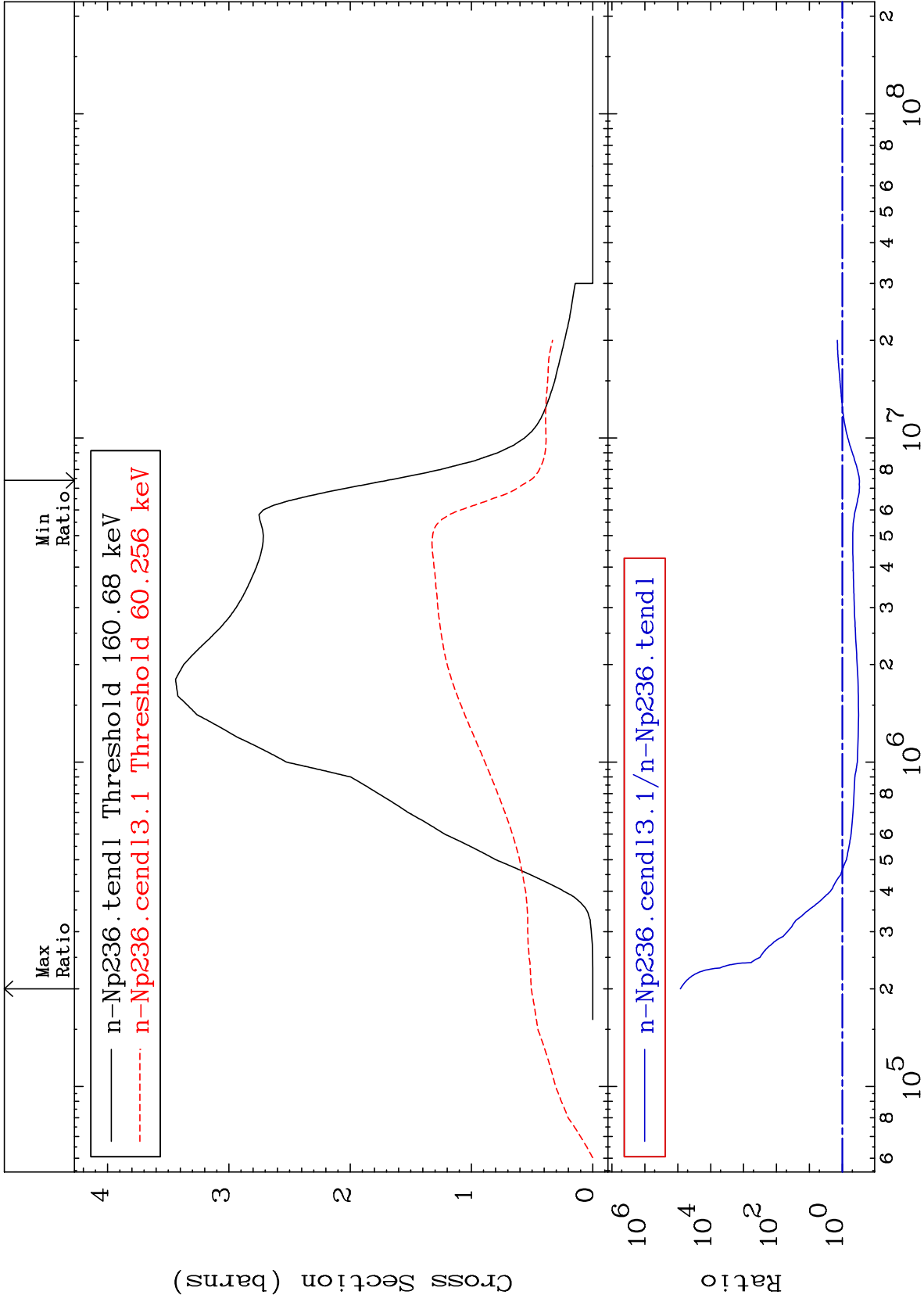
-40.17 To 397.6 %



MAT 9343

Inelastic
Cross Section

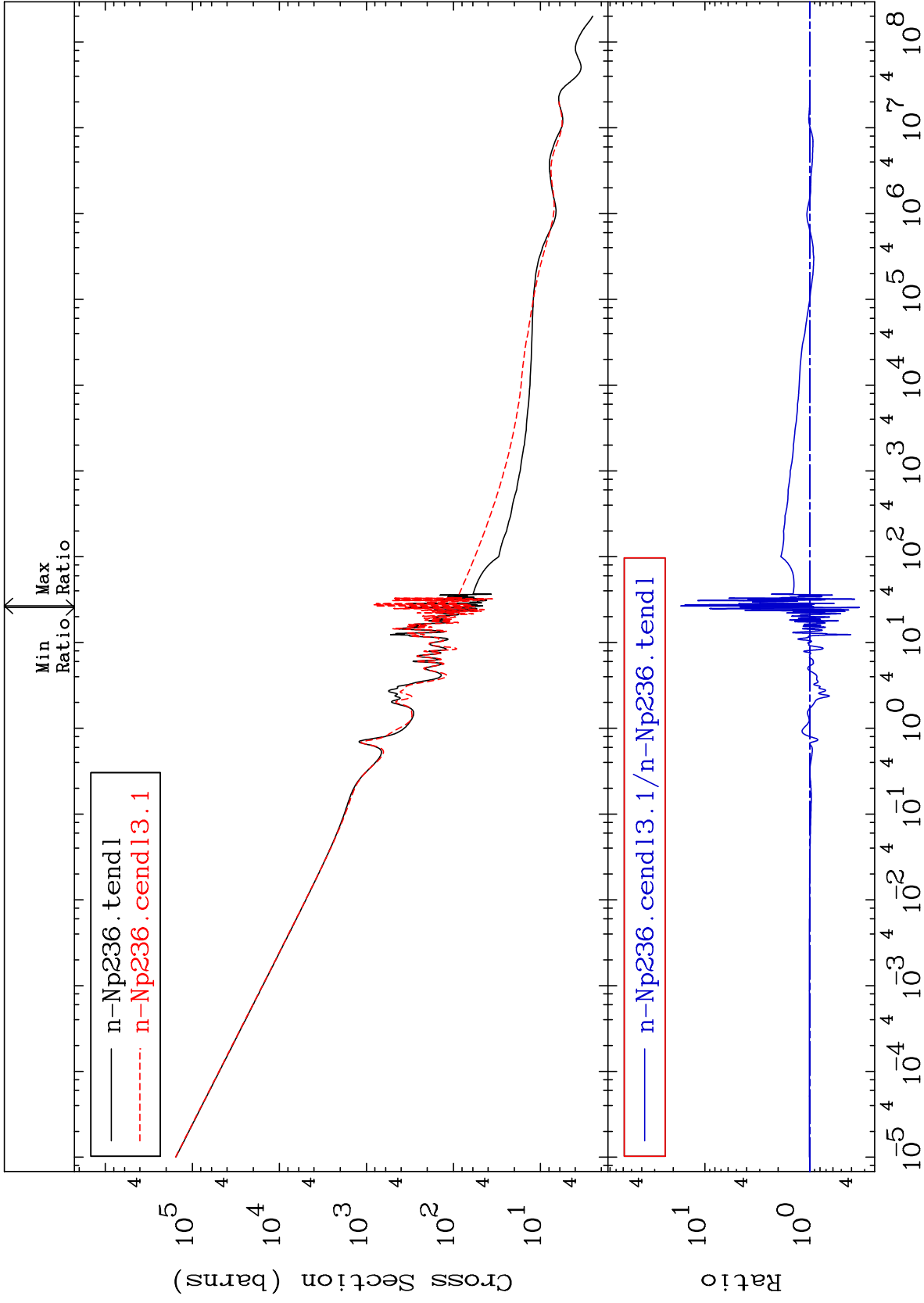
93-Np-236
-69.59 To 9999. %



MAT 9343

Total
Cross Section

93-Np-236
-66.38 To 1610. %



Incident Energy (eV)

93-Np-236

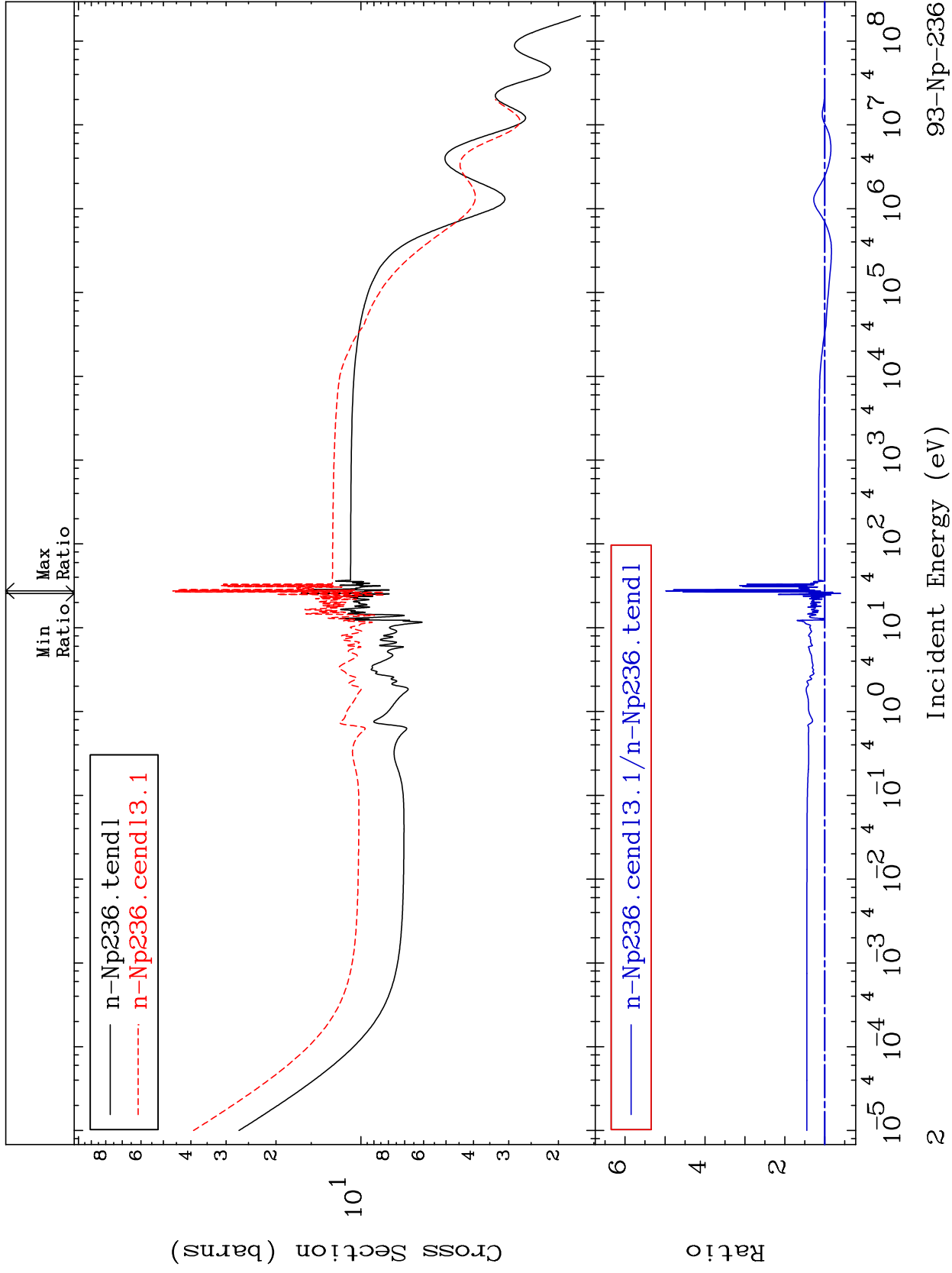
MAT 9343

Elastic

93-Np-236

Cross Section

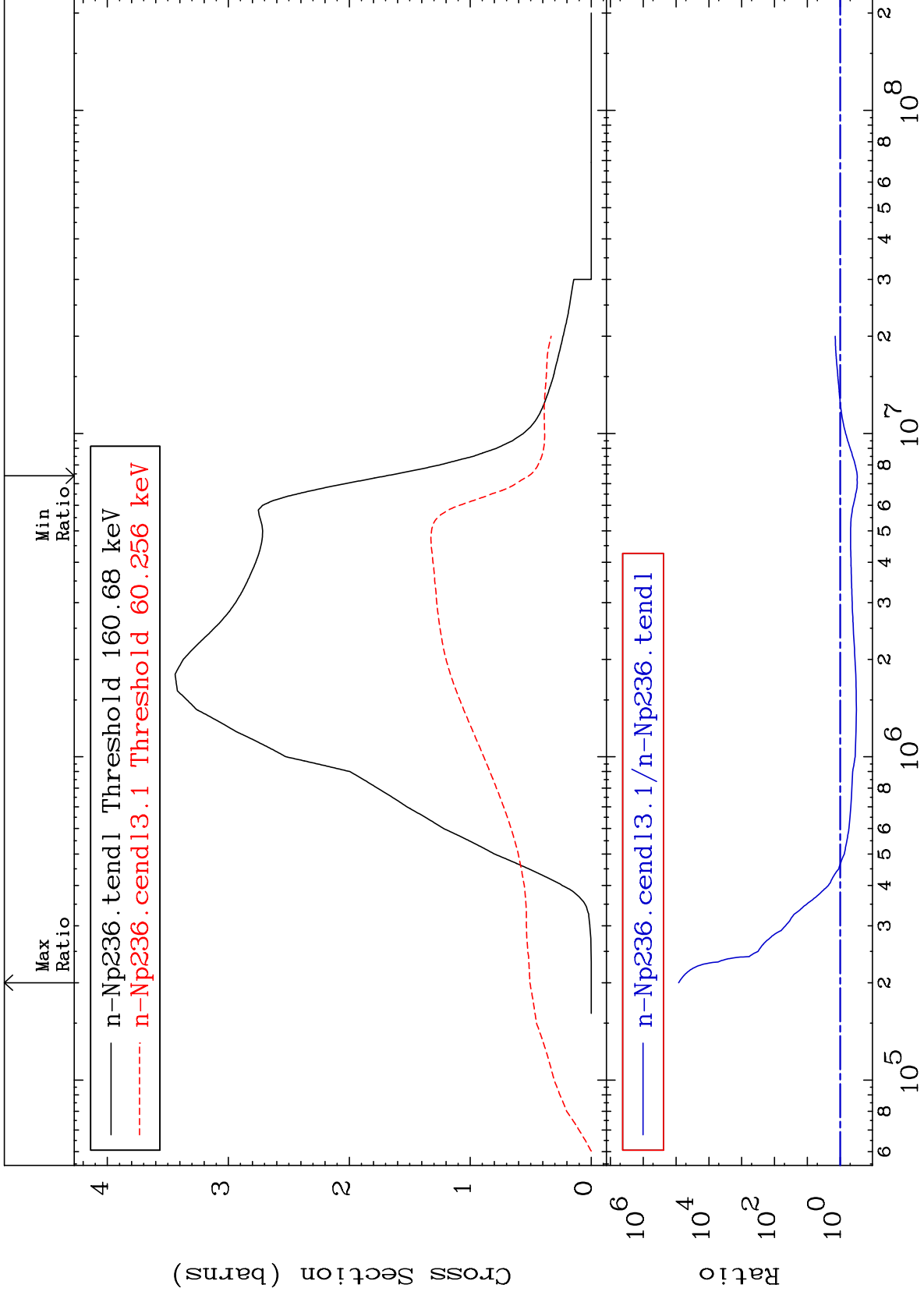
-40.17 To 397.6 %



MAT 9343

Inelastic
Cross Section

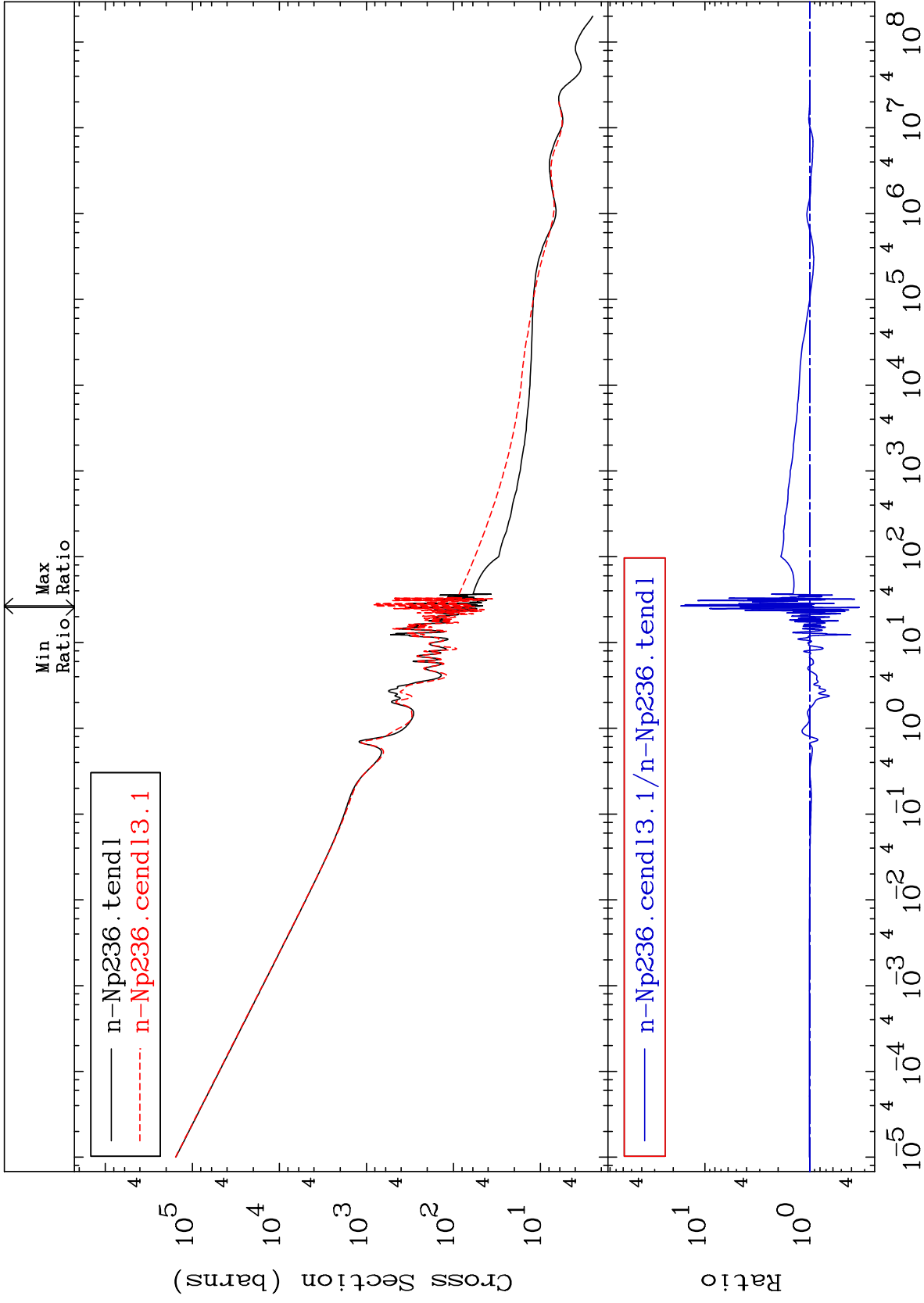
93-Np-236
-69.59 To 9999. %



MAT 9343

Total
Cross Section

93-Np-236
-66.38 To 1610. %



Incident Energy (eV)

93-Np-236

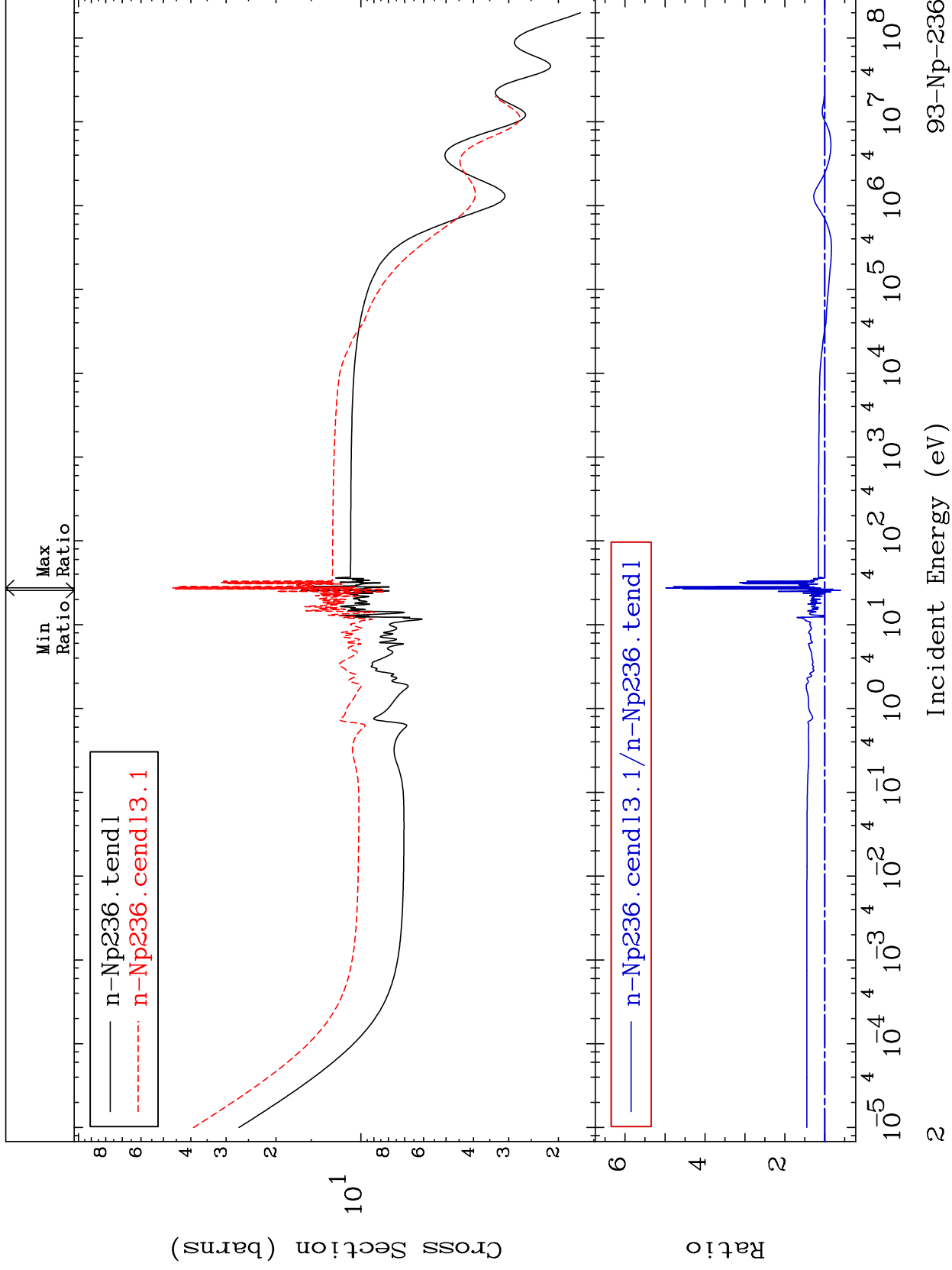
MAT 9343

Elastic

93-Np-236

Cross Section

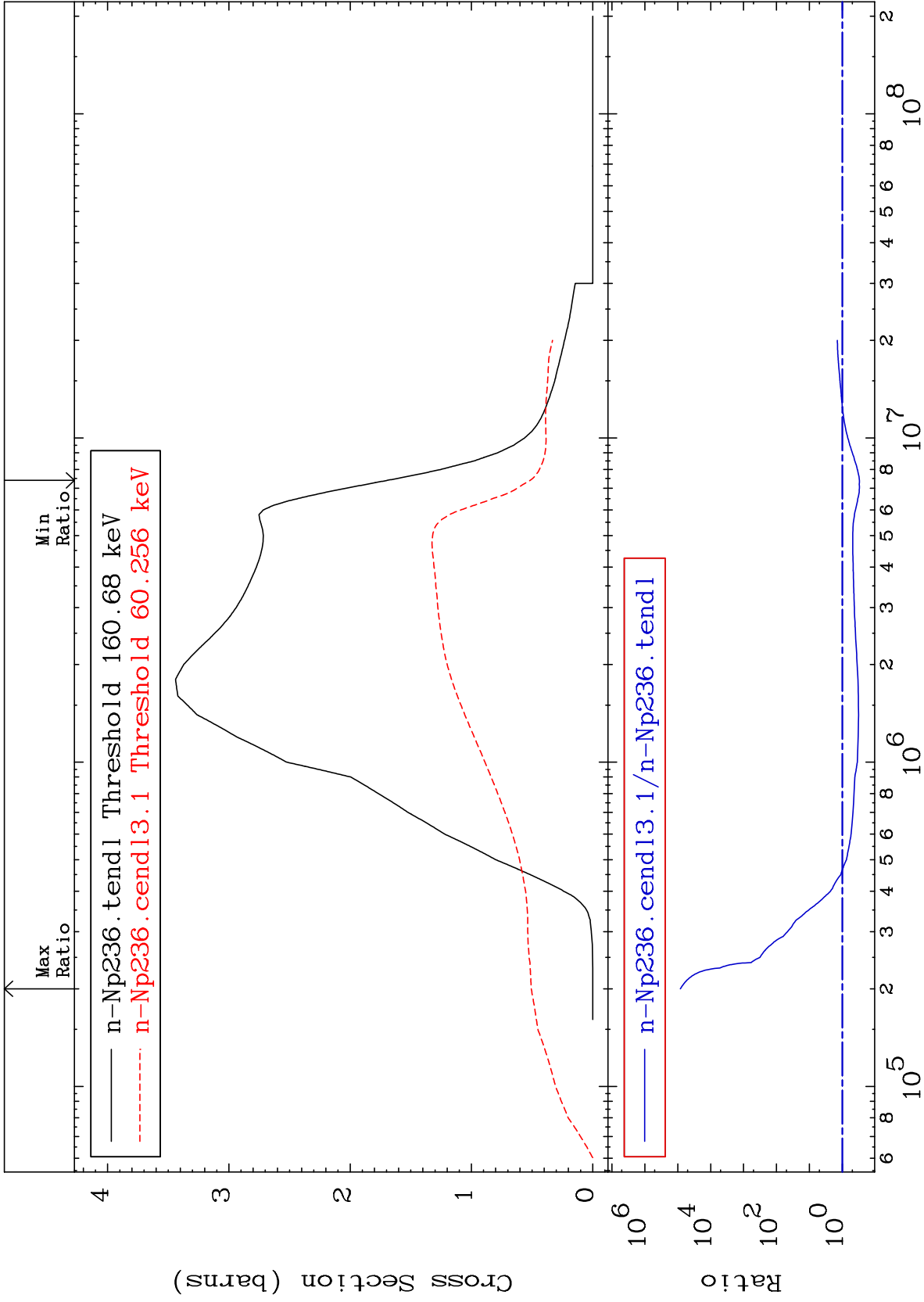
-40.17 To 397.6 %



MAT 9343

Inelastic
Cross Section

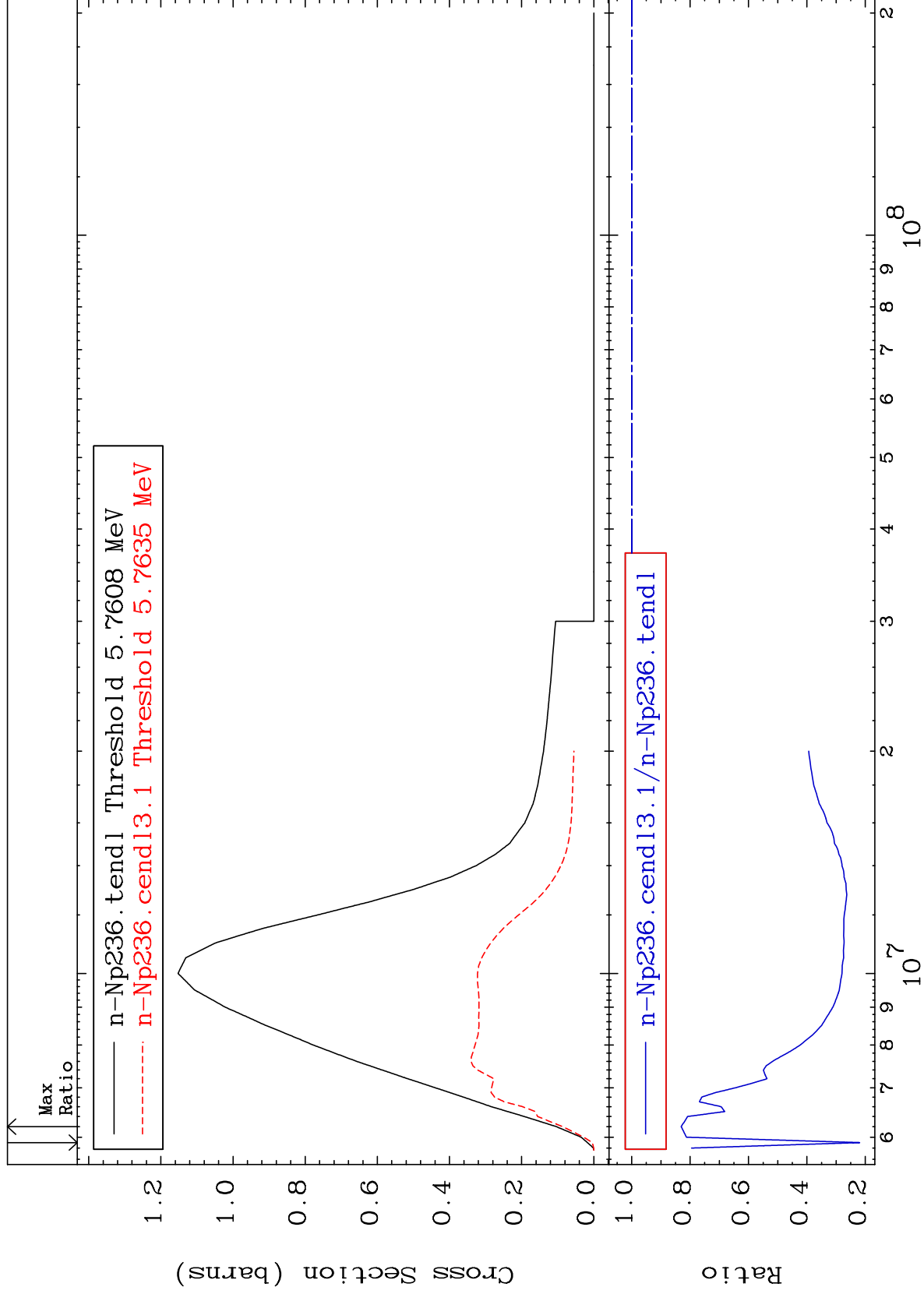
93-Np-236
-69.59 To 9999. %



MAT 9343

(n,2n)
Cross Section

93-Np-236
-78.05 To -16.94%



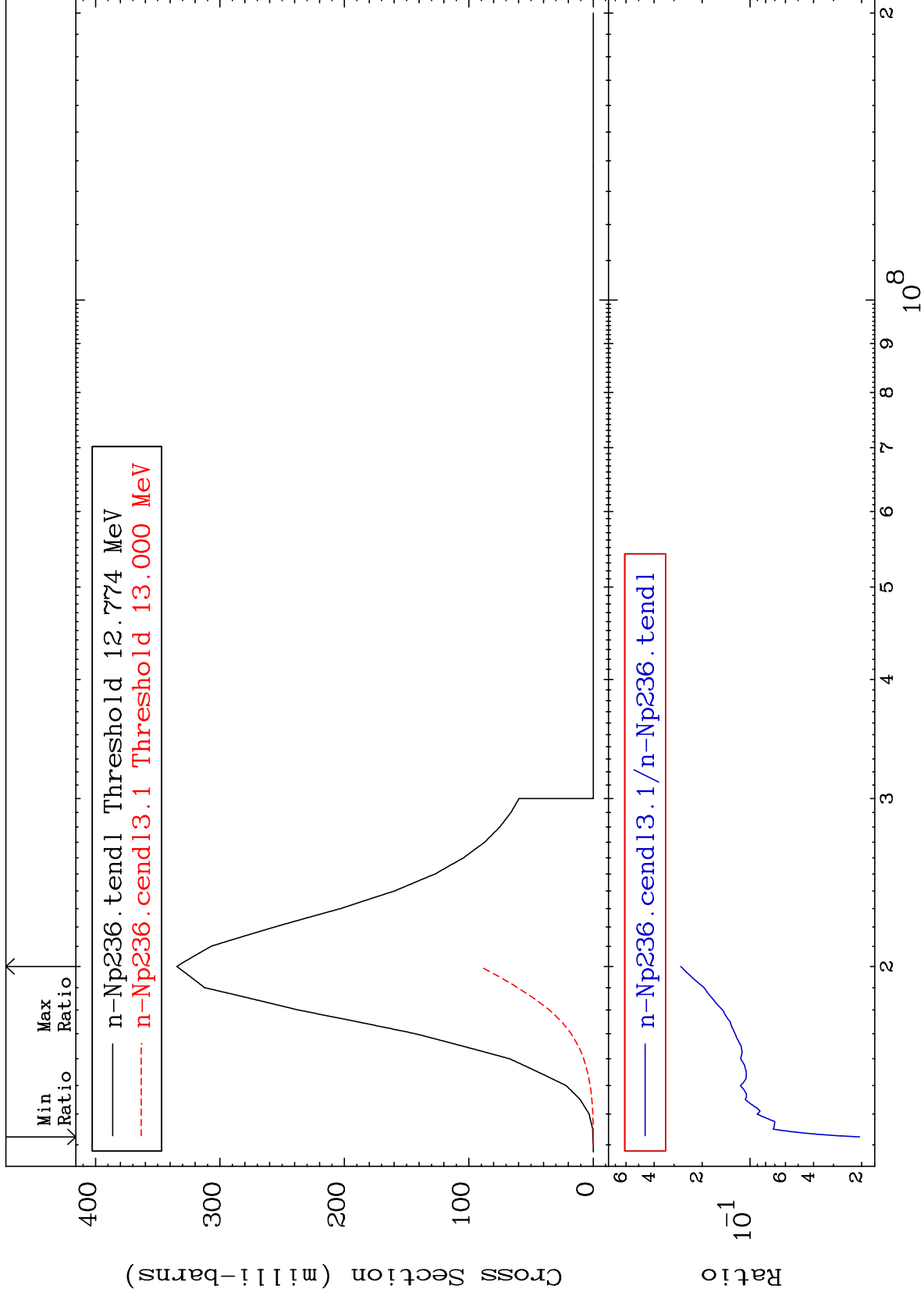
MAT 9343

(n,3n)

93-Np-236

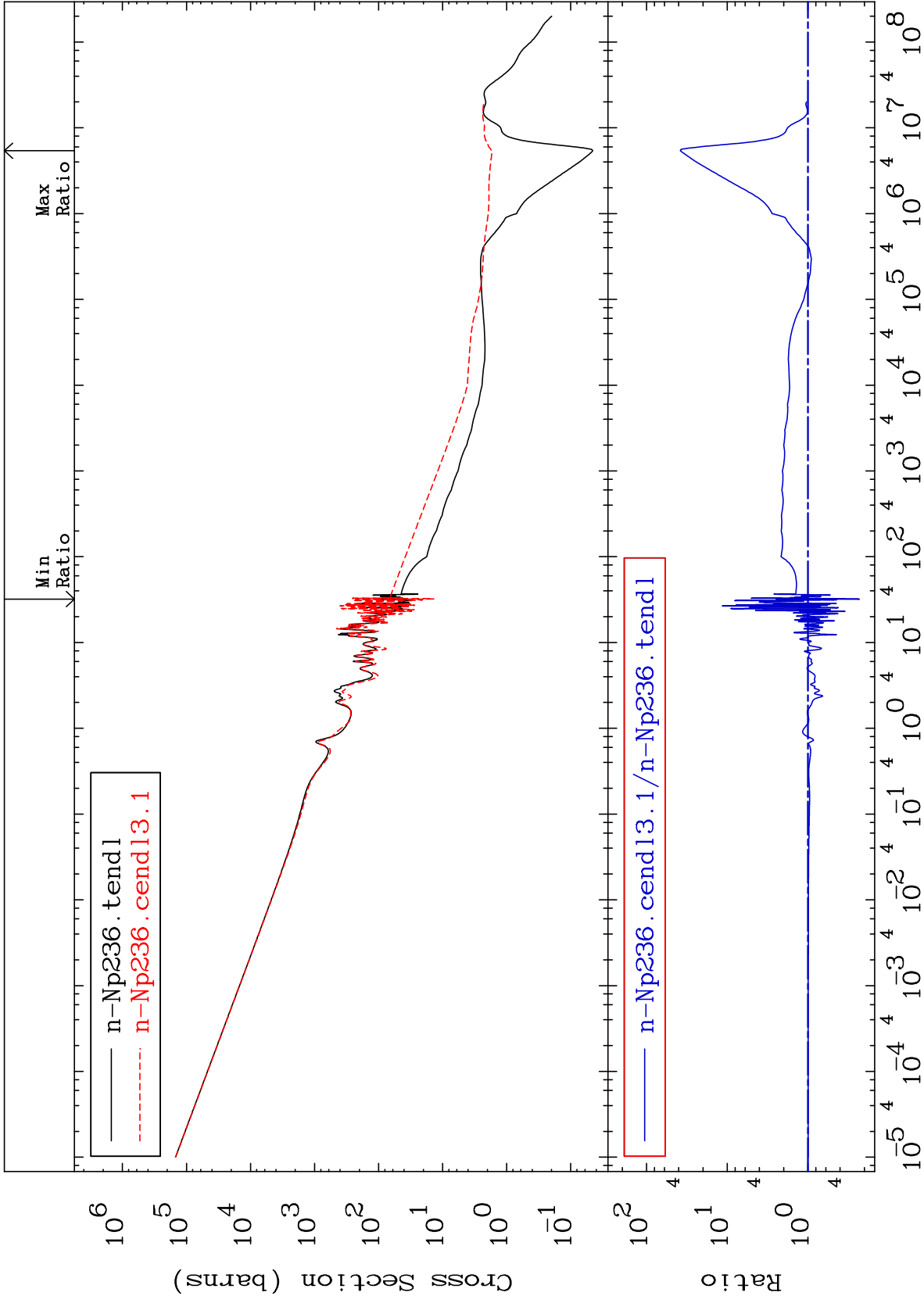
Cross Section

-97.94 To -72.77%



MAT 9343

Fission Cross Section 93-Np-236
-76.93 To 3718. %



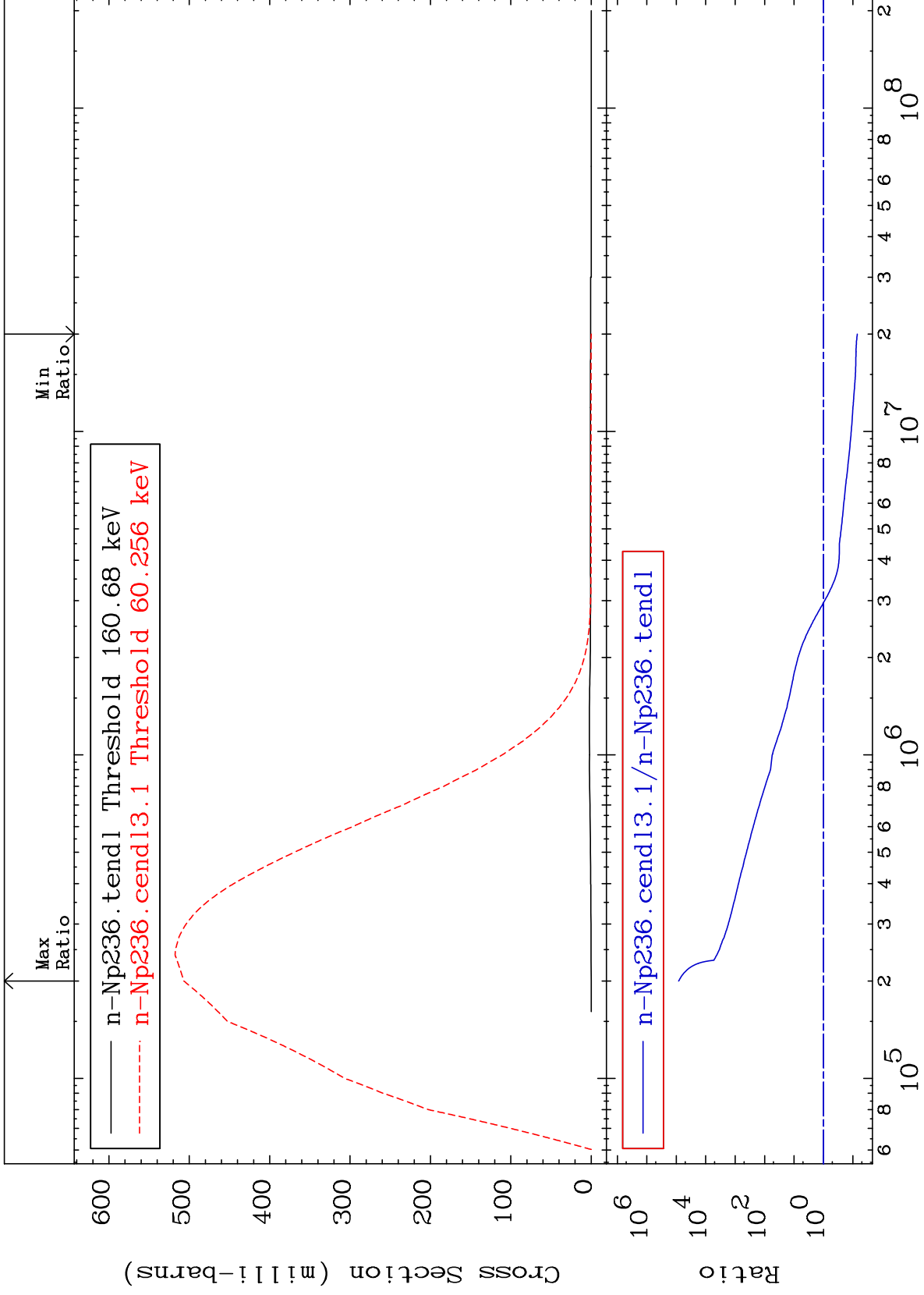
Incident Energy (eV)

93-Np-236

MAT 9343

MT= 51 (n,n') Level
Cross Section

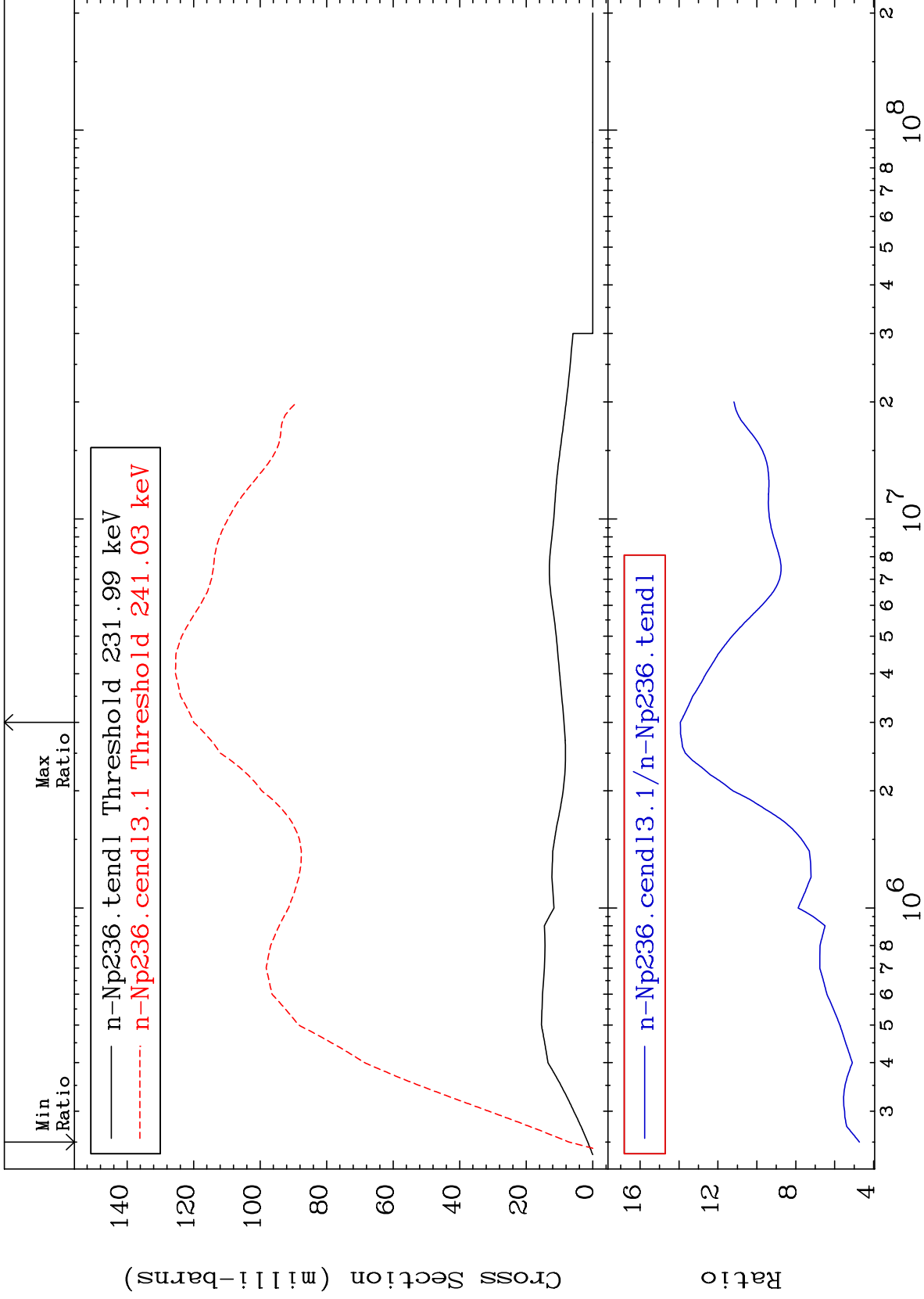
93-Np-236
-92.88 To 9999. %



MAT 9343

MT= 52 (n,n') Level
Cross Section

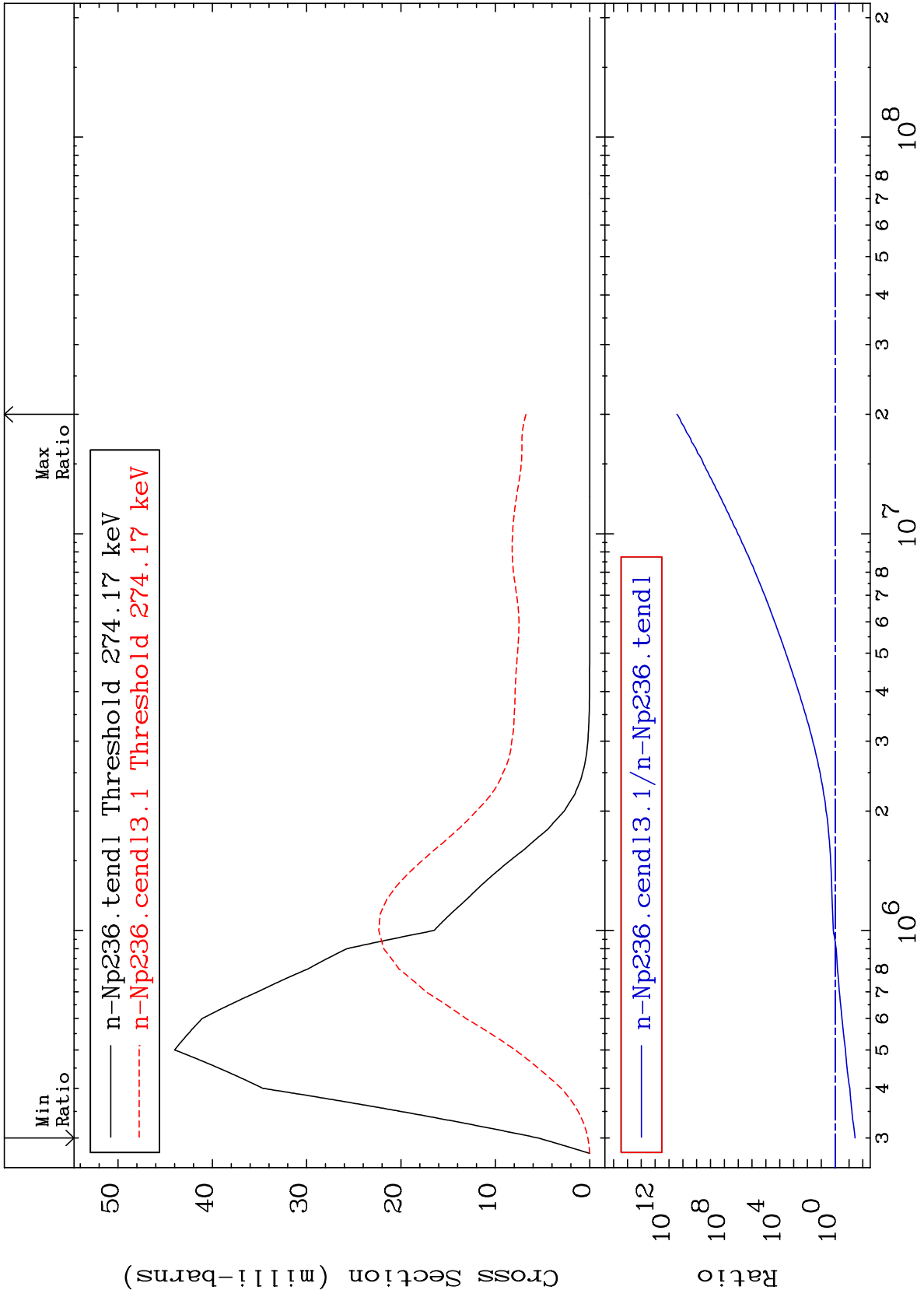
93-Np-236
372.7 To 1293. %



MAT 9343

MT= 53 (n,n') Level
Cross Section

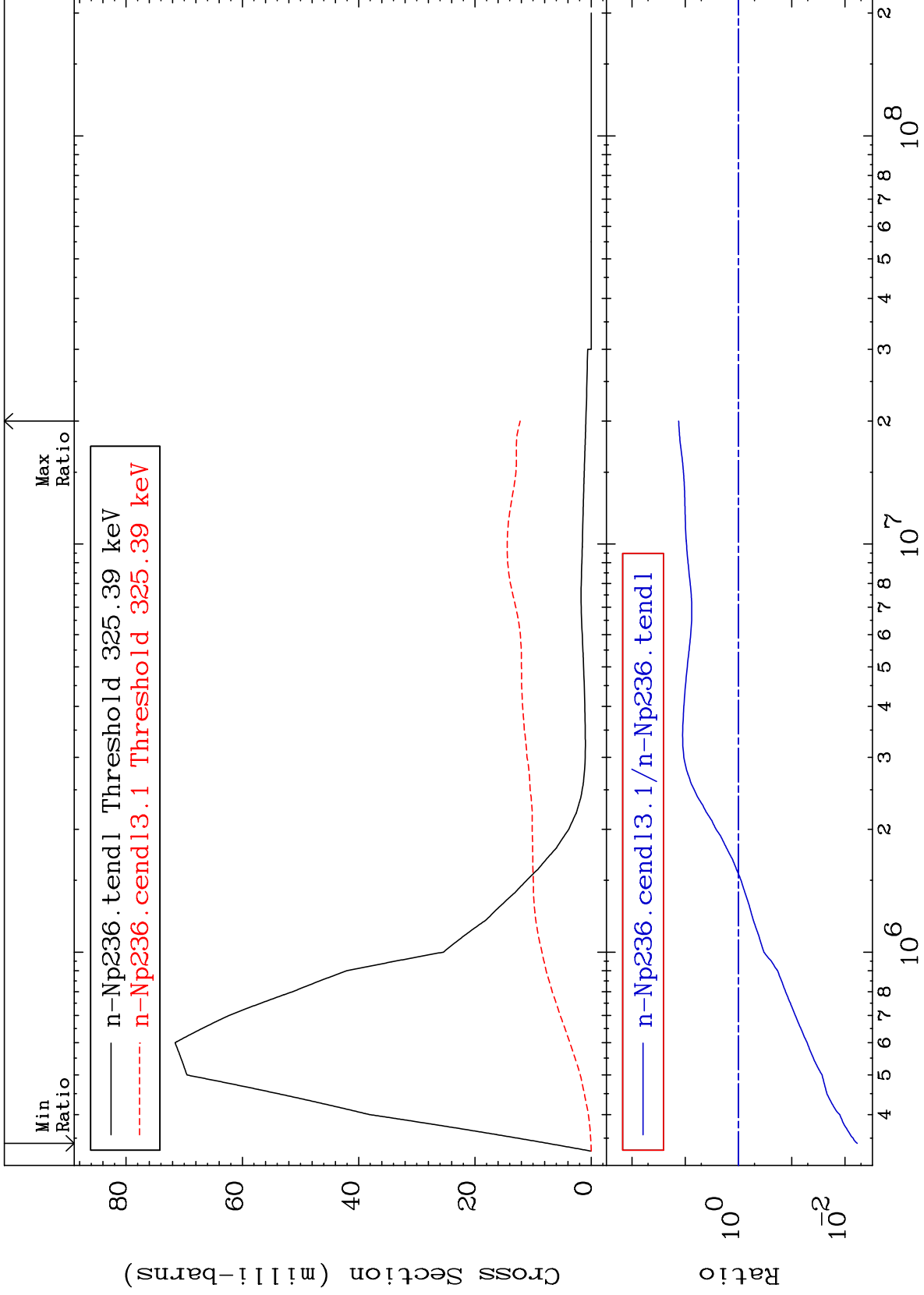
93-Np-236
-96.39 To 9999. %



MAT 9343

MT= 54 (n,n') Level
Cross Section

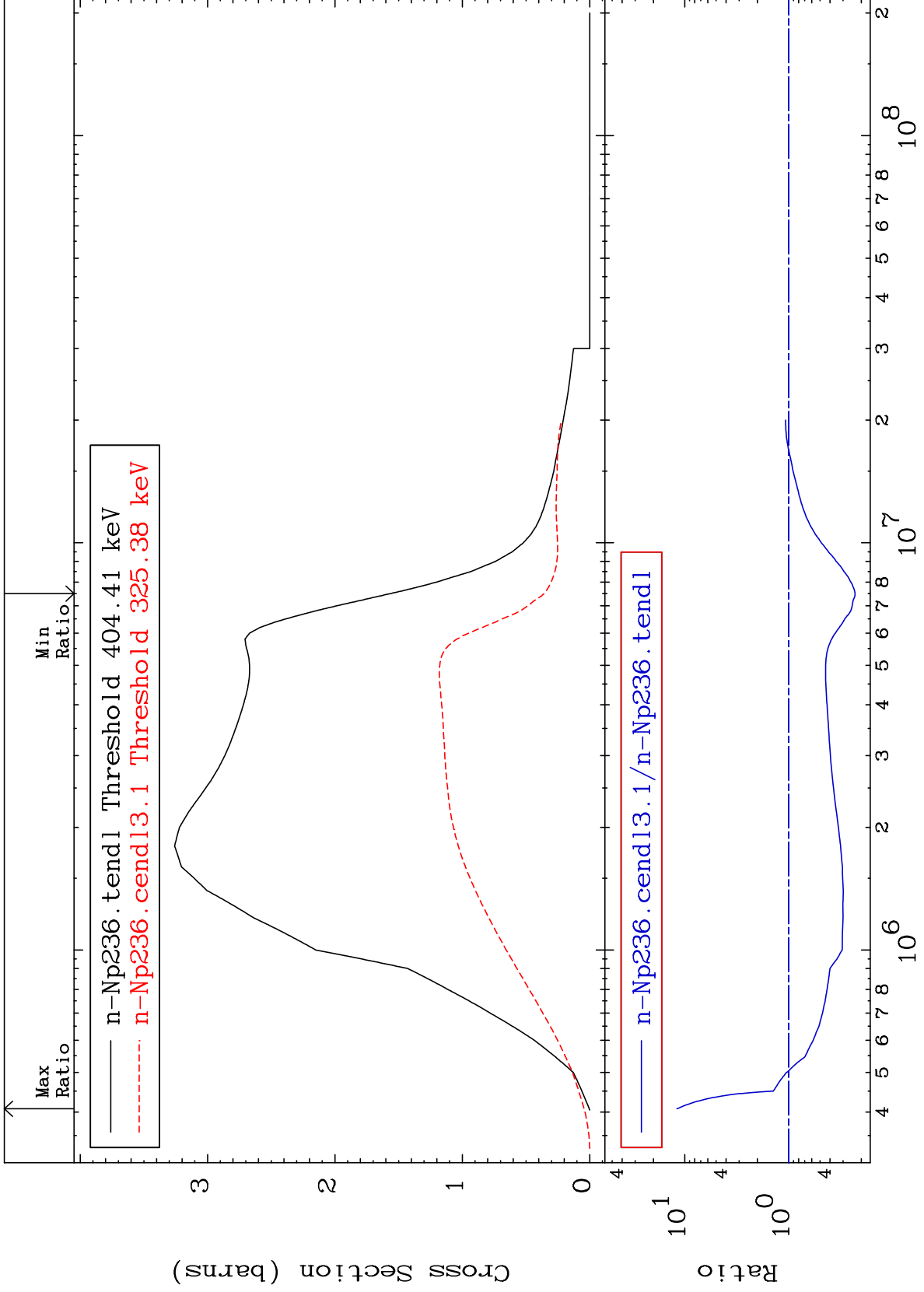
93-Np-236
-99.41 To 1235. %



MAT 9343

(n, n') Continuum
Cross Section

93-Np-236
-77.03 To 1088. %



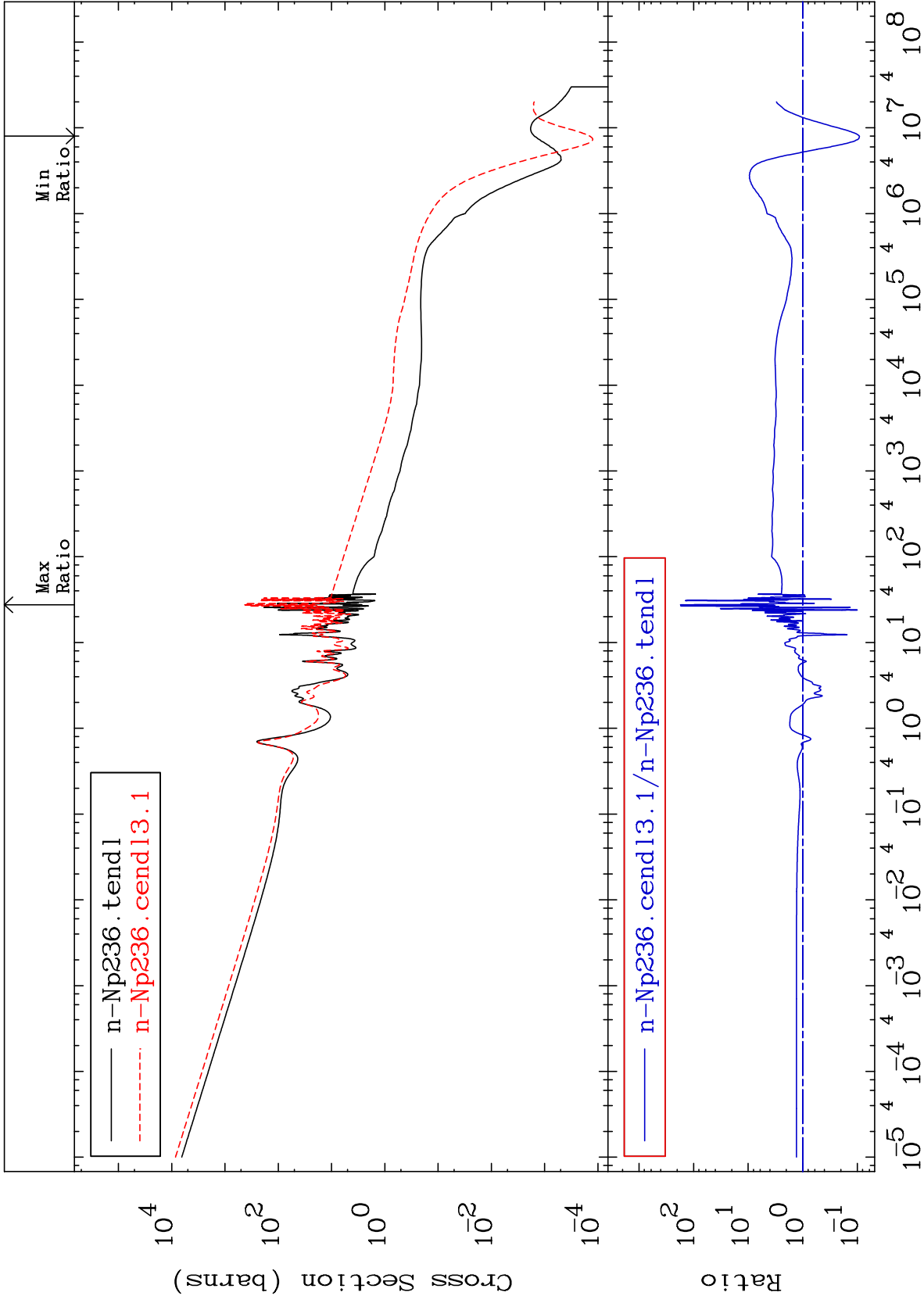
11

93-Np-236

MAT 9343

(n, γ)
Cross Section

93-Np-236
-90.85 To 9999. %



12

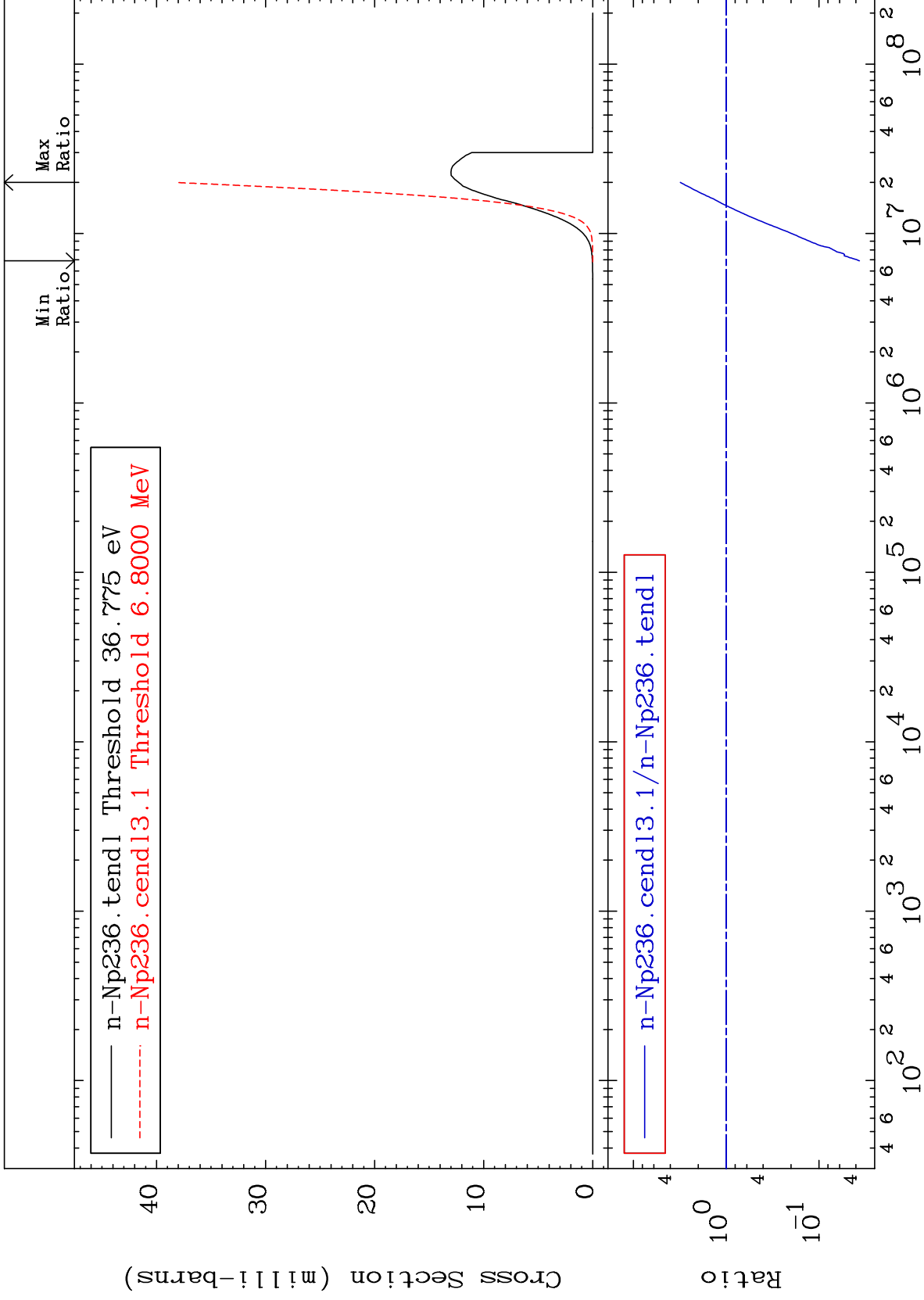
Incident Energy (eV)

93-Np-236

MAT 9343

(n,p)
Cross Section

93-Np-236
-96.33 To 211.5 %



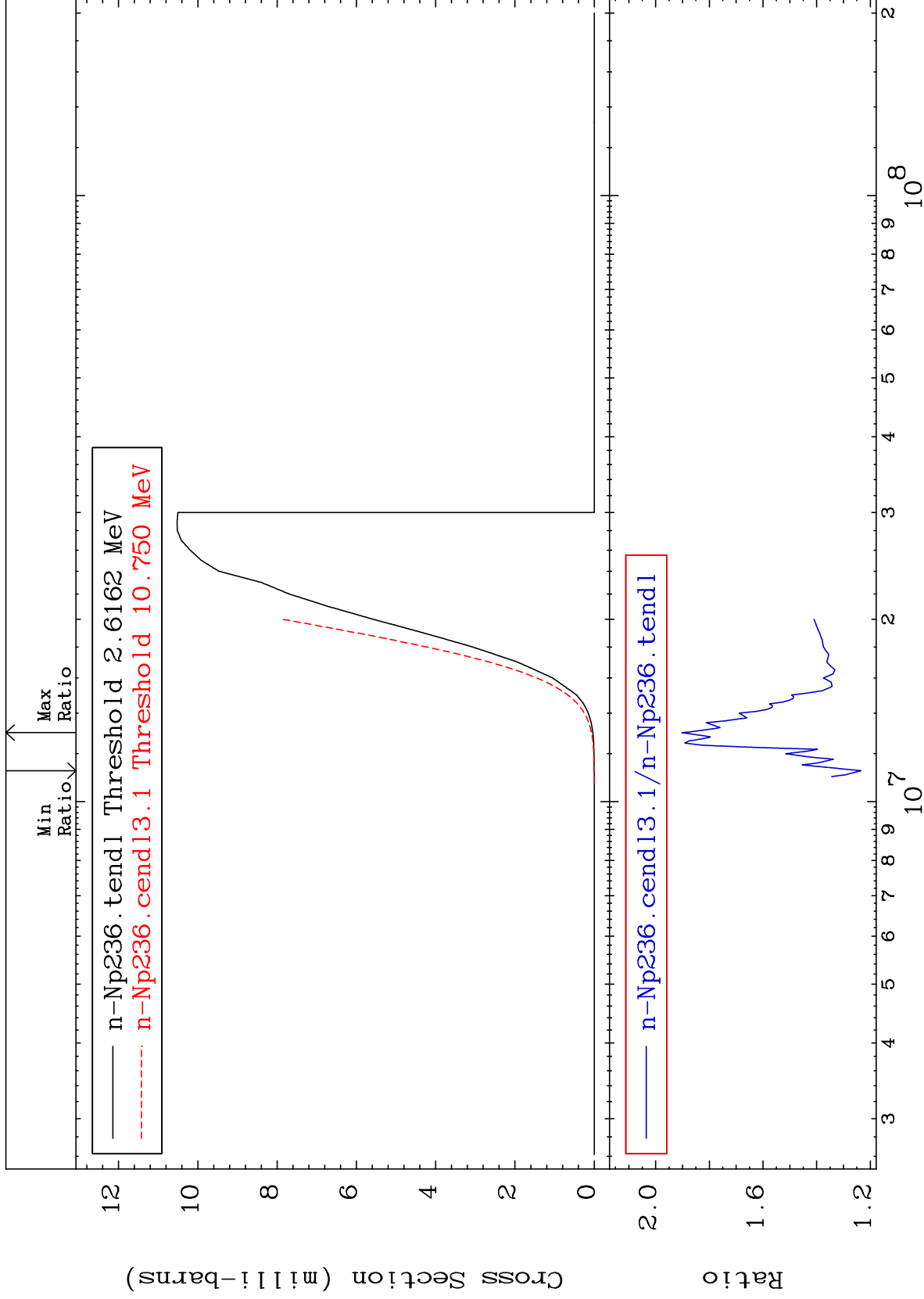
MAT 9343

(n, d)

93-Np-236

Cross Section

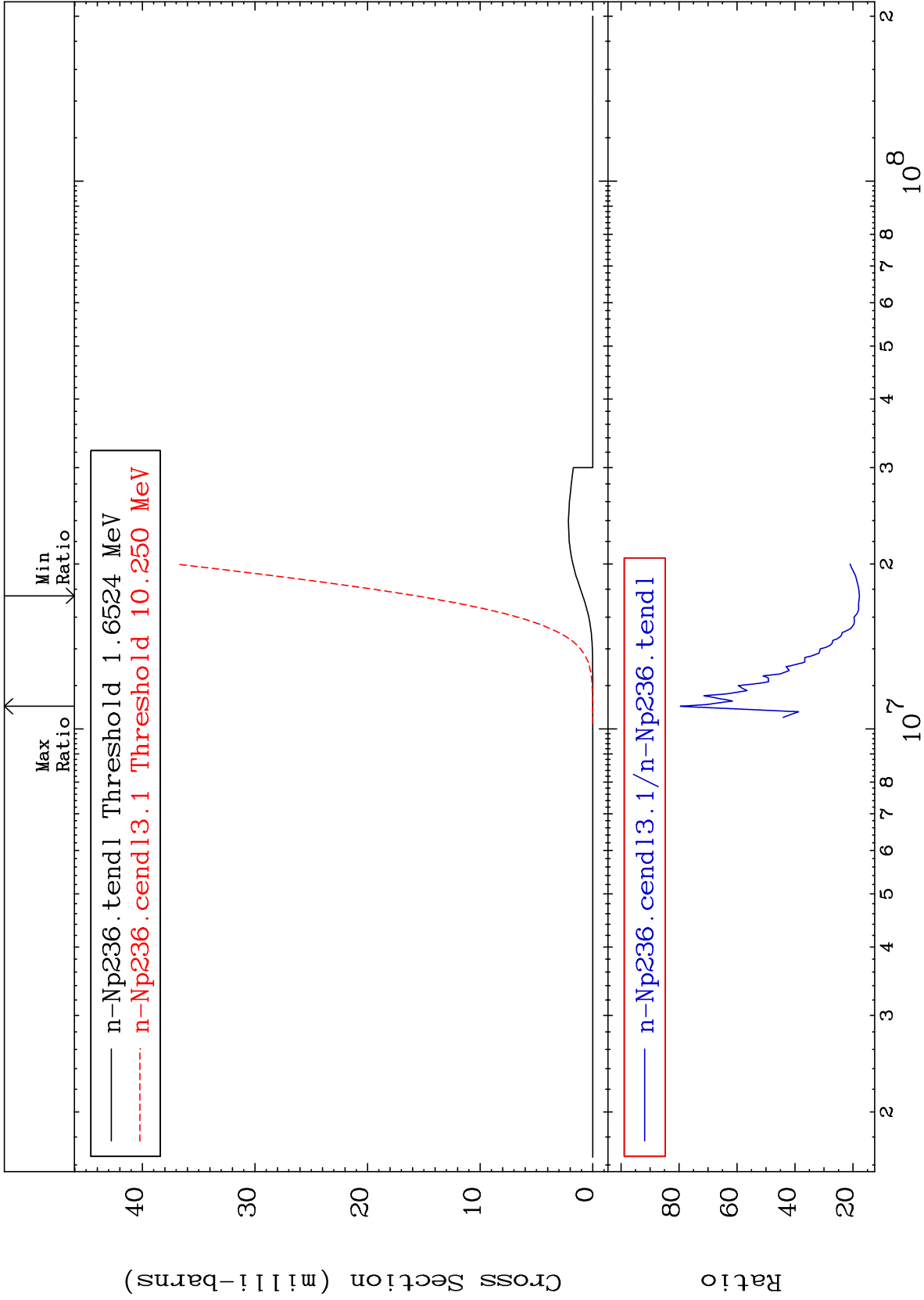
23.46 To 90.22 %



14

Incident Energy (eV)

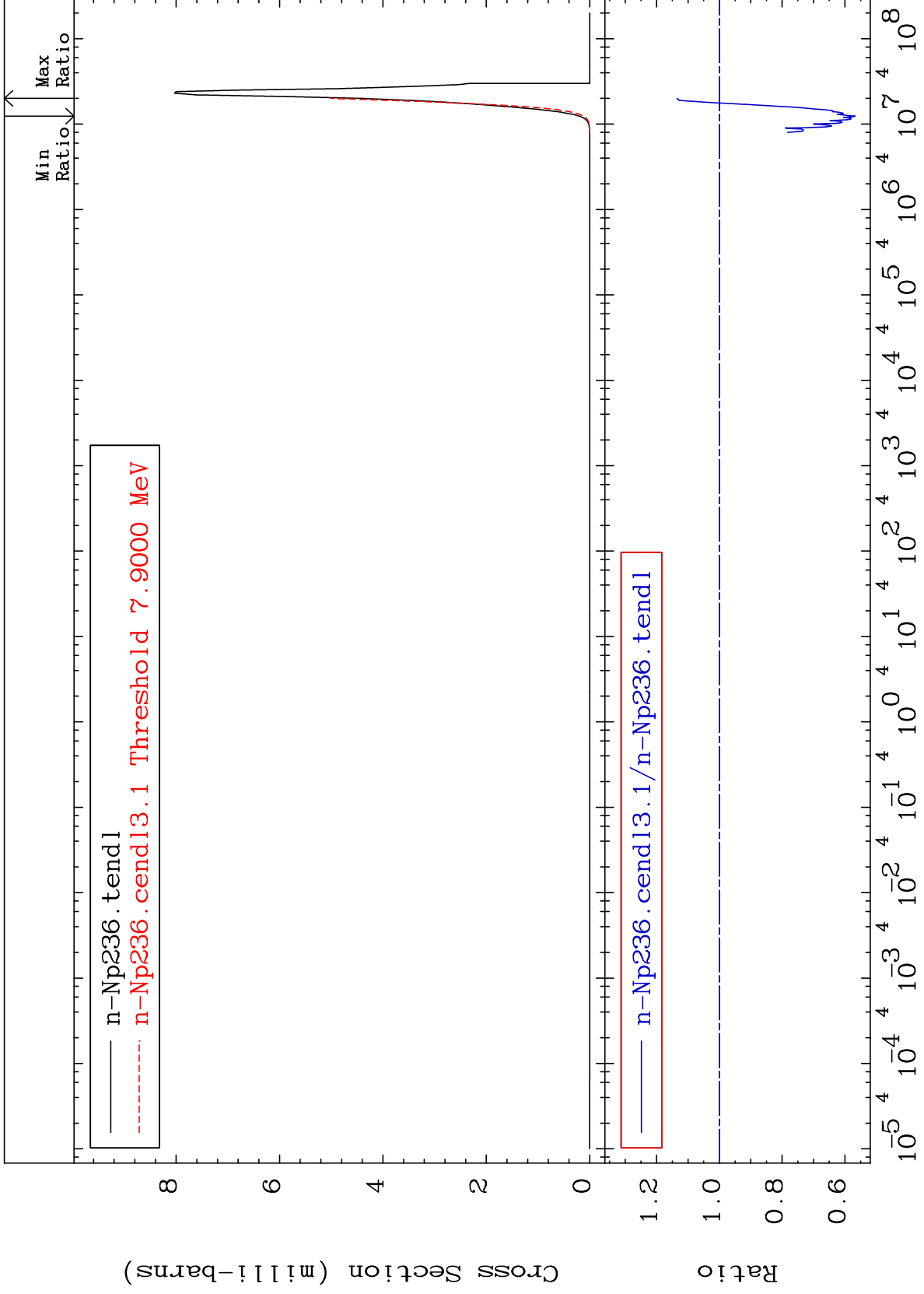
93-Np-236



MAT 9343

(n, α)
Cross Section

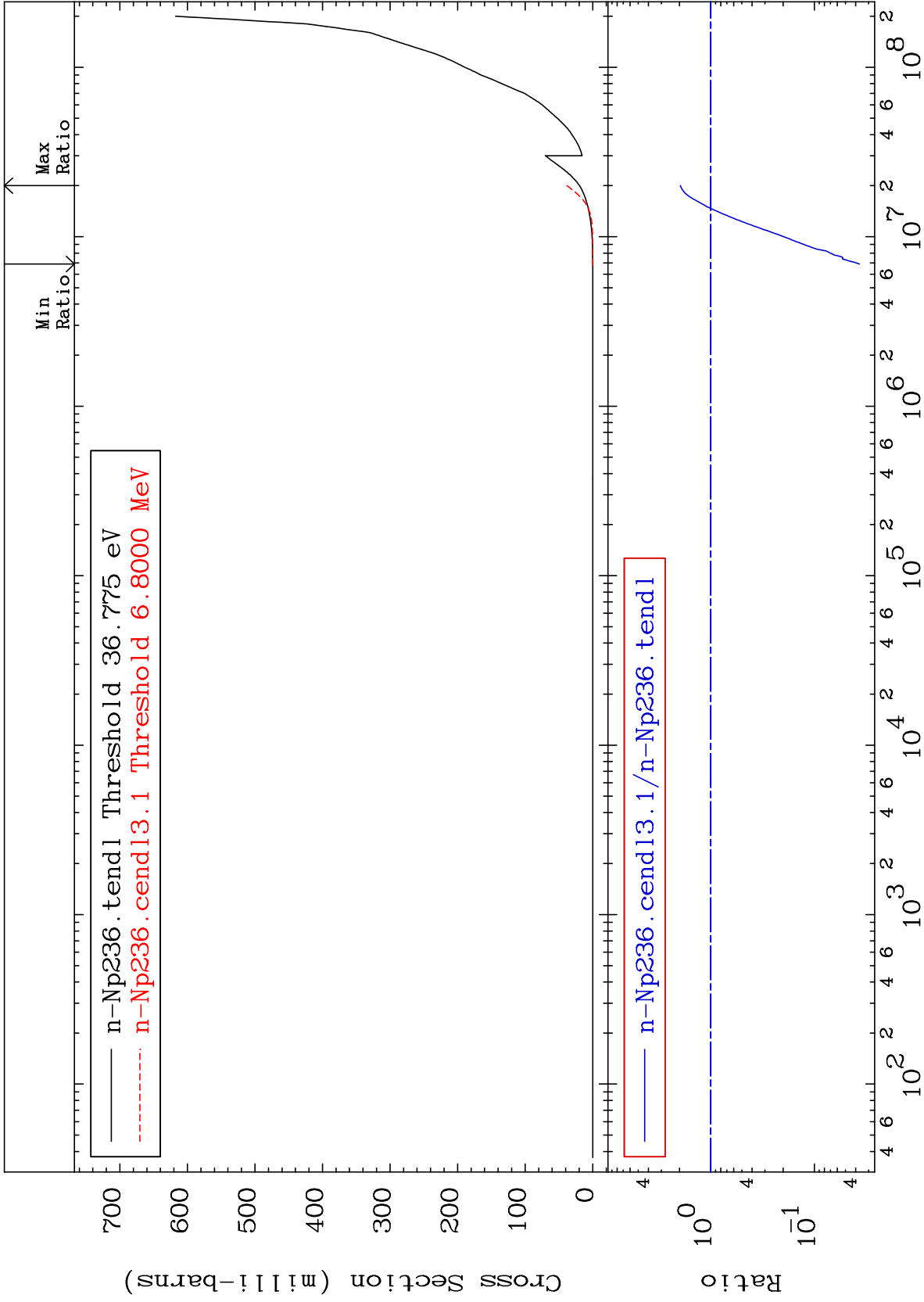
93-Np-236
-43.24 To 13.53 %



16

Incident Energy (eV)

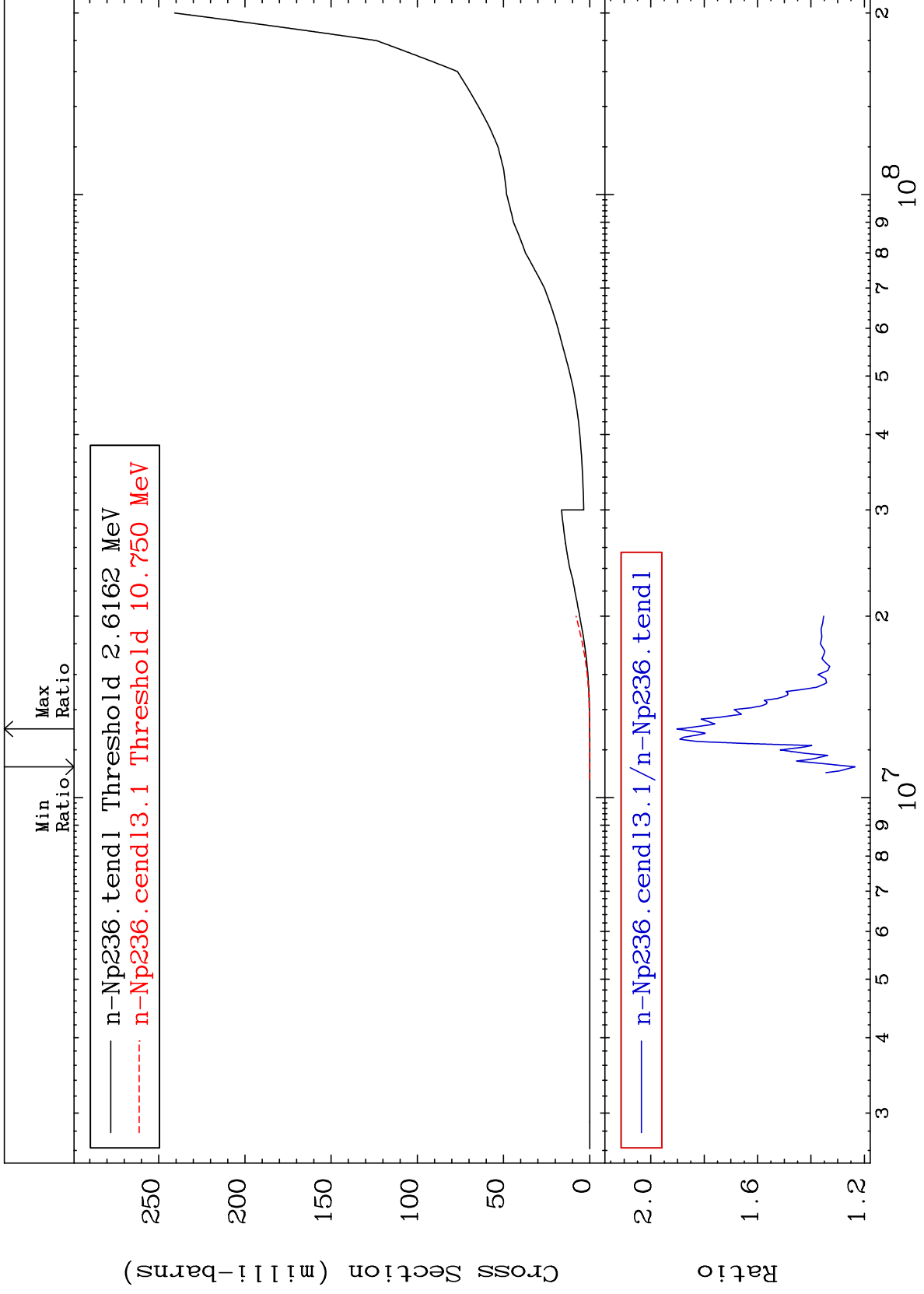
93-Np-236



MAT 9343

Deuterium Production
Cross Section

93-Np-236
23.46 To 90.22 %



18

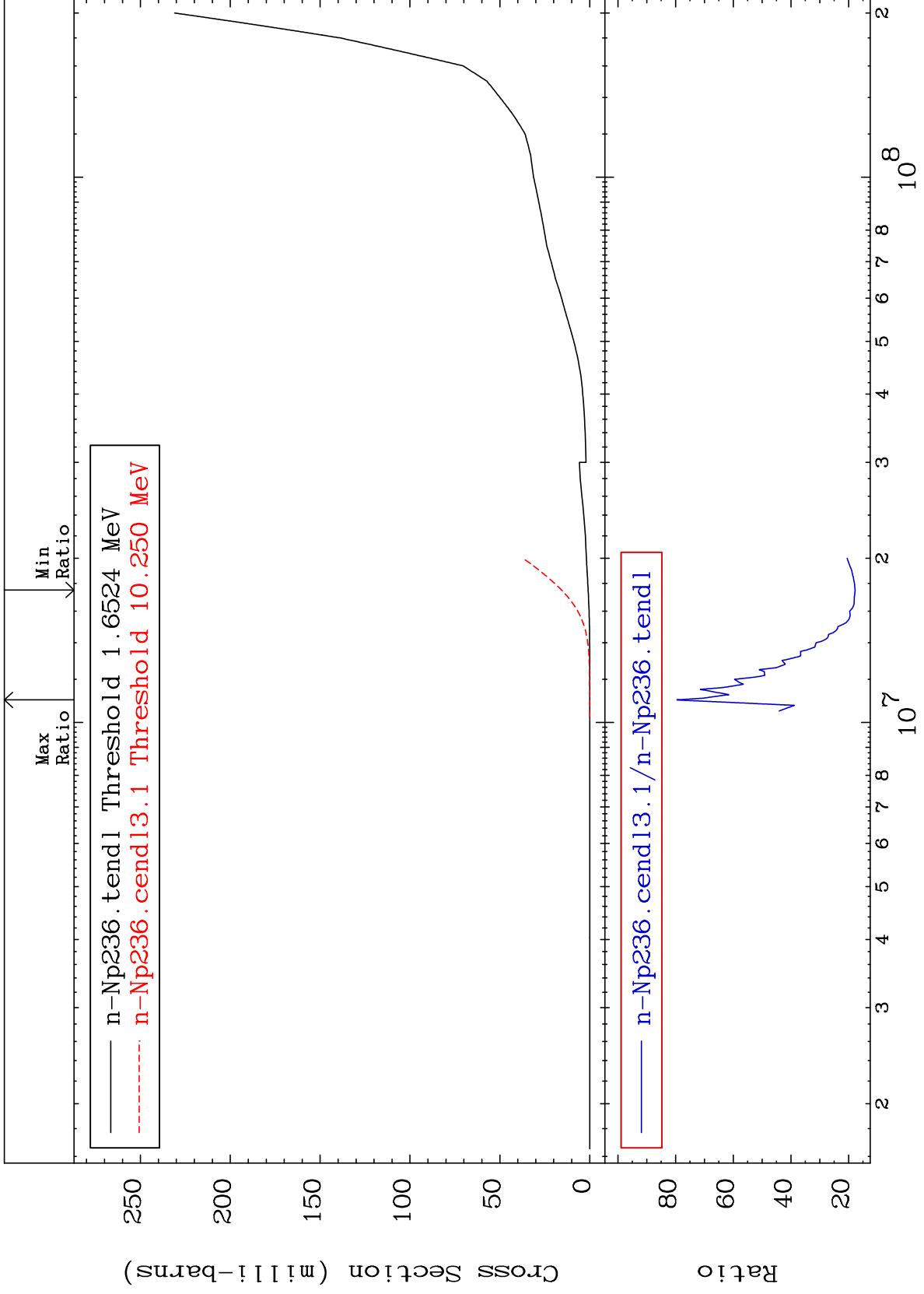
Incident Energy (eV)

93-Np-236

MAT 9343

Tritium Production
Cross Section

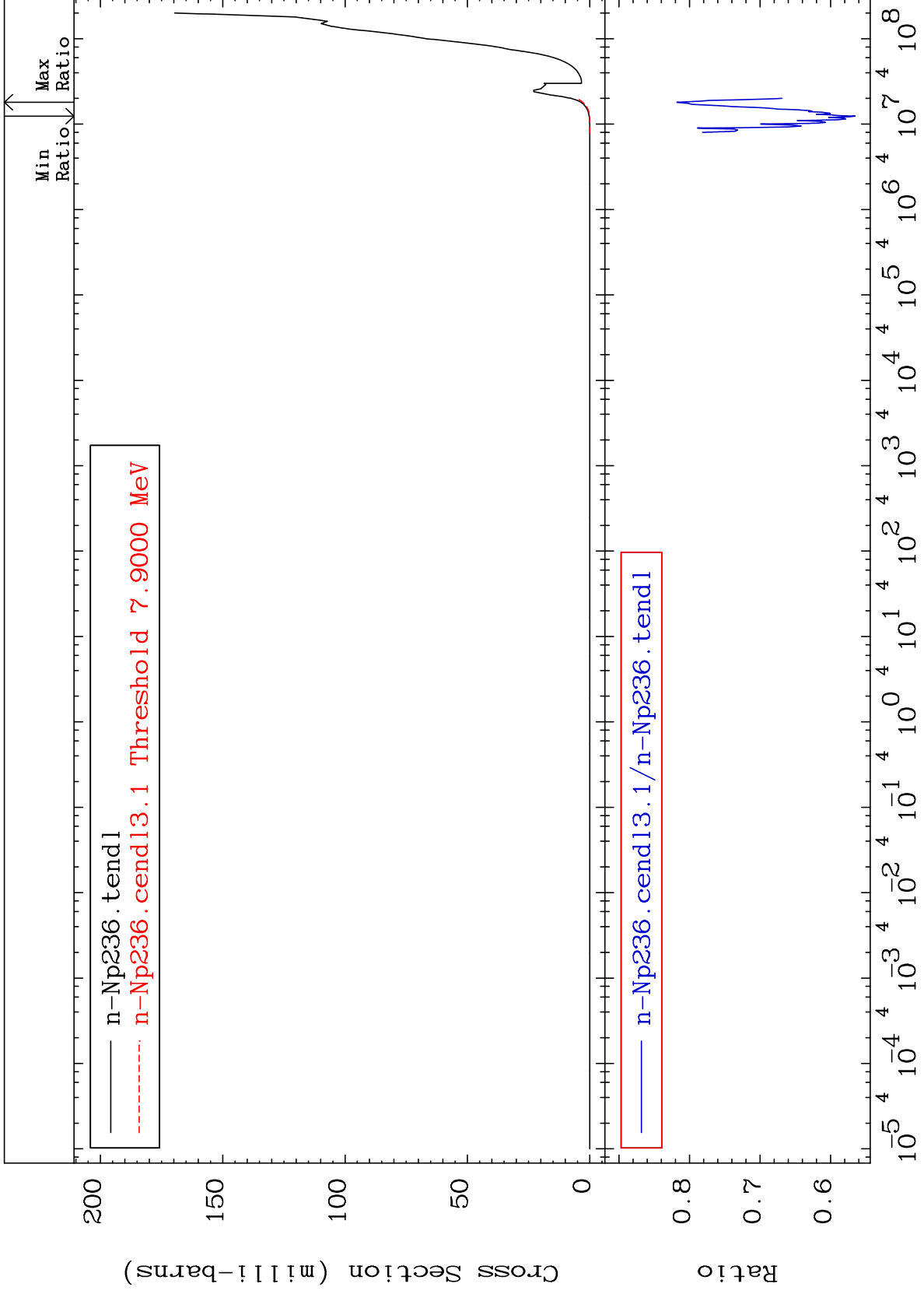
93-Np-236
1676. To 7853. %



MAT 9343

He-4 Production
Cross Section

93-Np-236
-43.46 To -18.22%



20

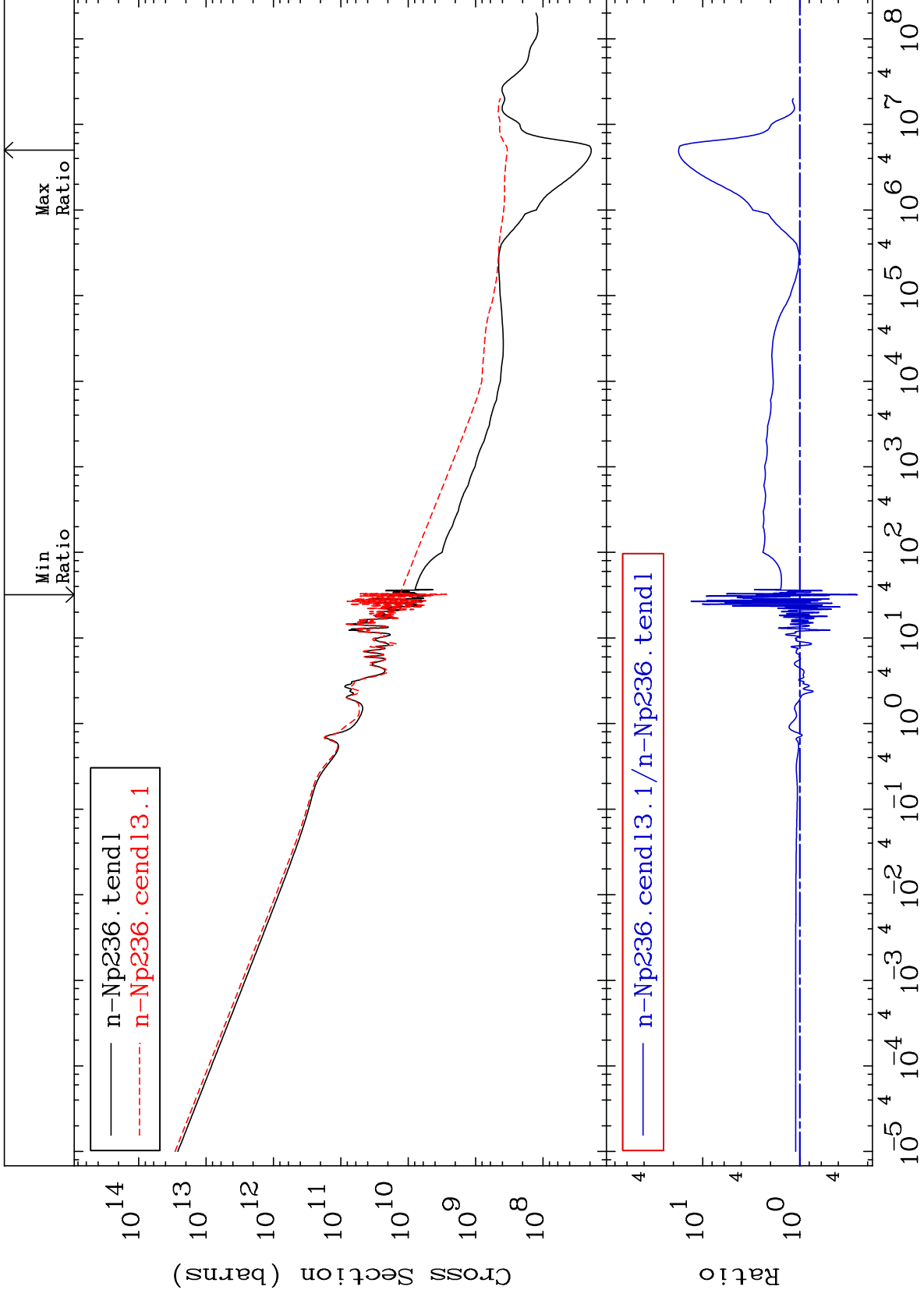
Incident Energy (eV)

93-Np-236

MAT 9343

Kerma total (eV-barns)
Cross Section

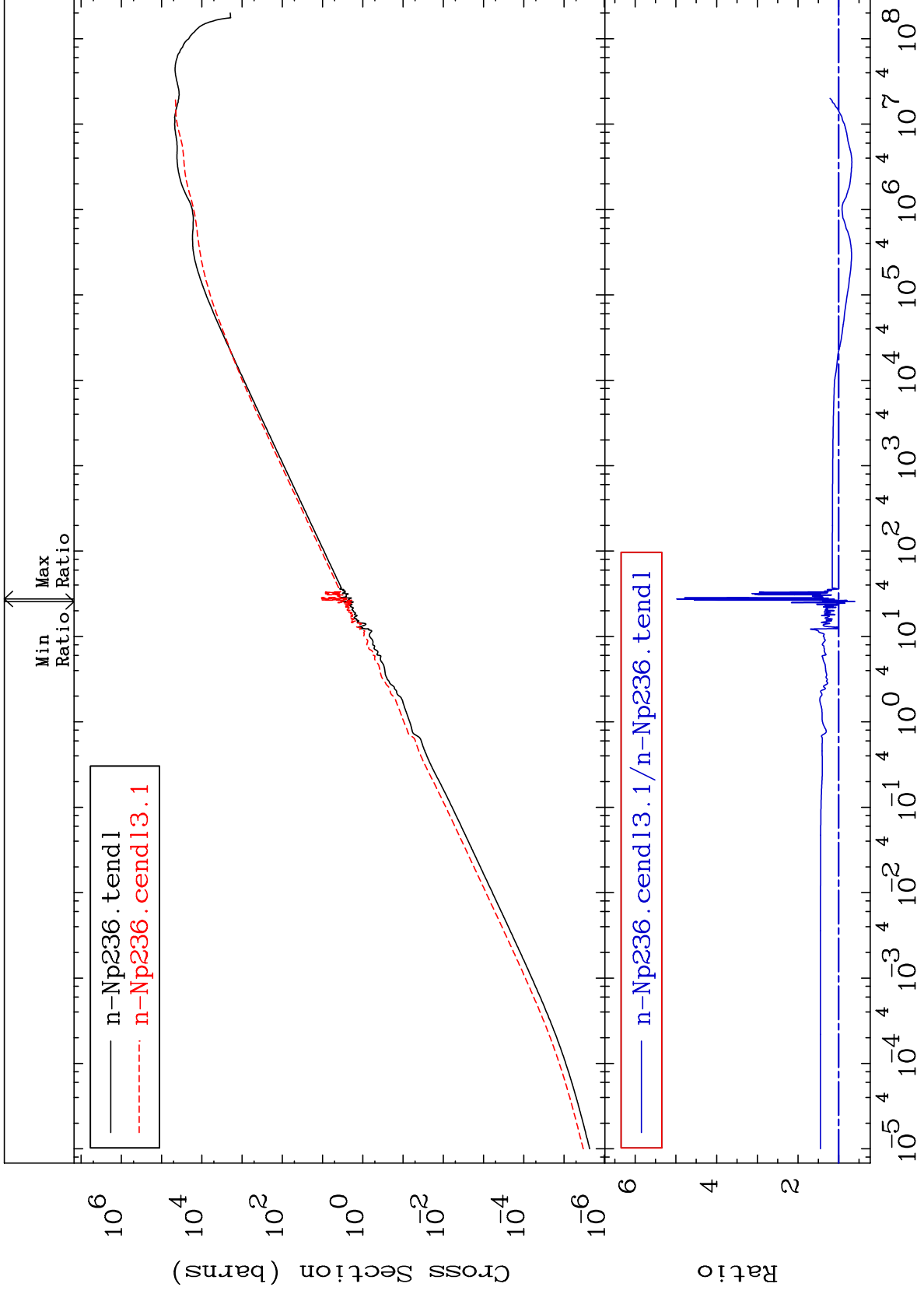
93-Np-236
-74.42 To 1666. %



MAT 9343

Kerma elastic
Cross Section

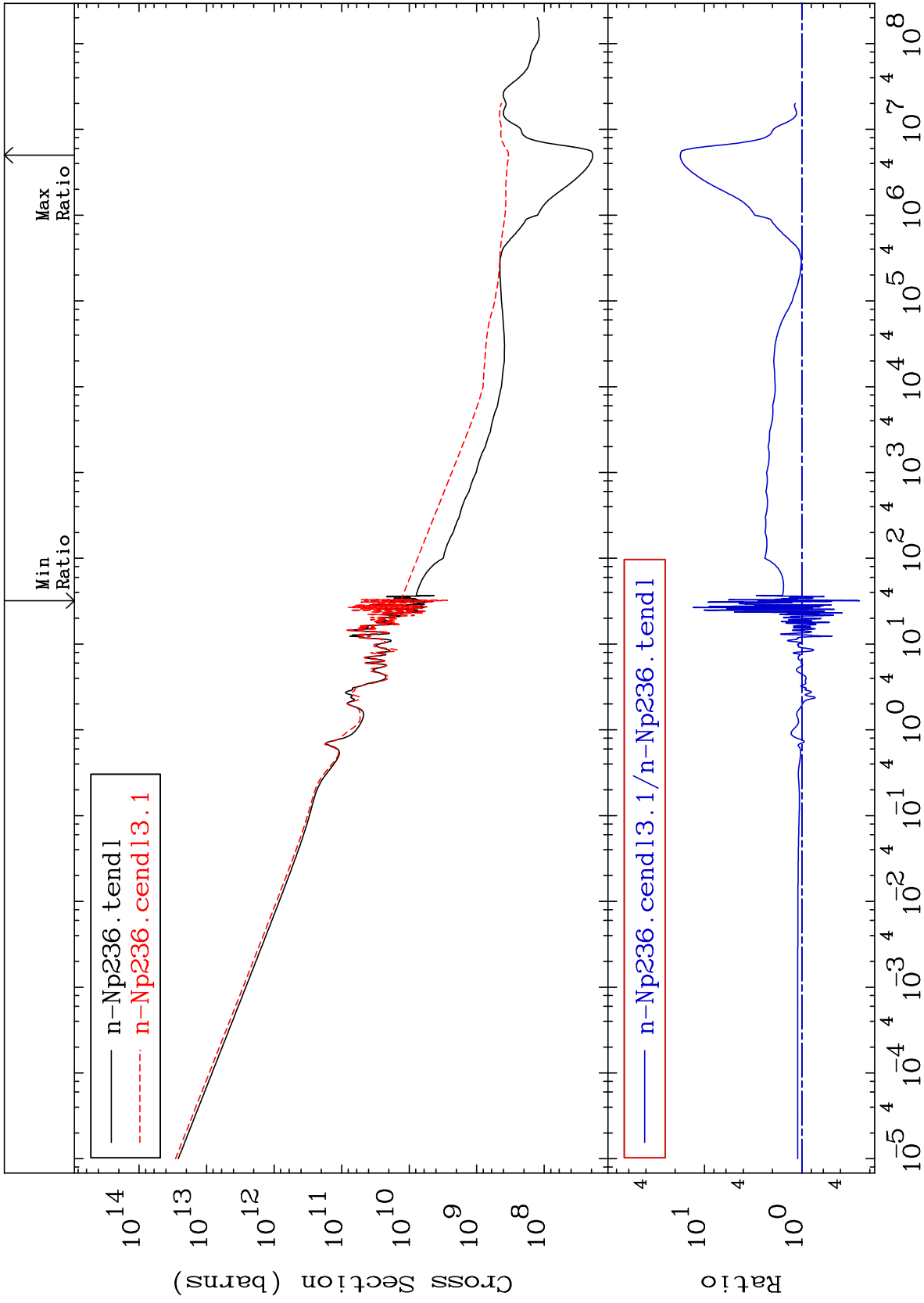
93-Np-236
-40.19 To 397.5 %

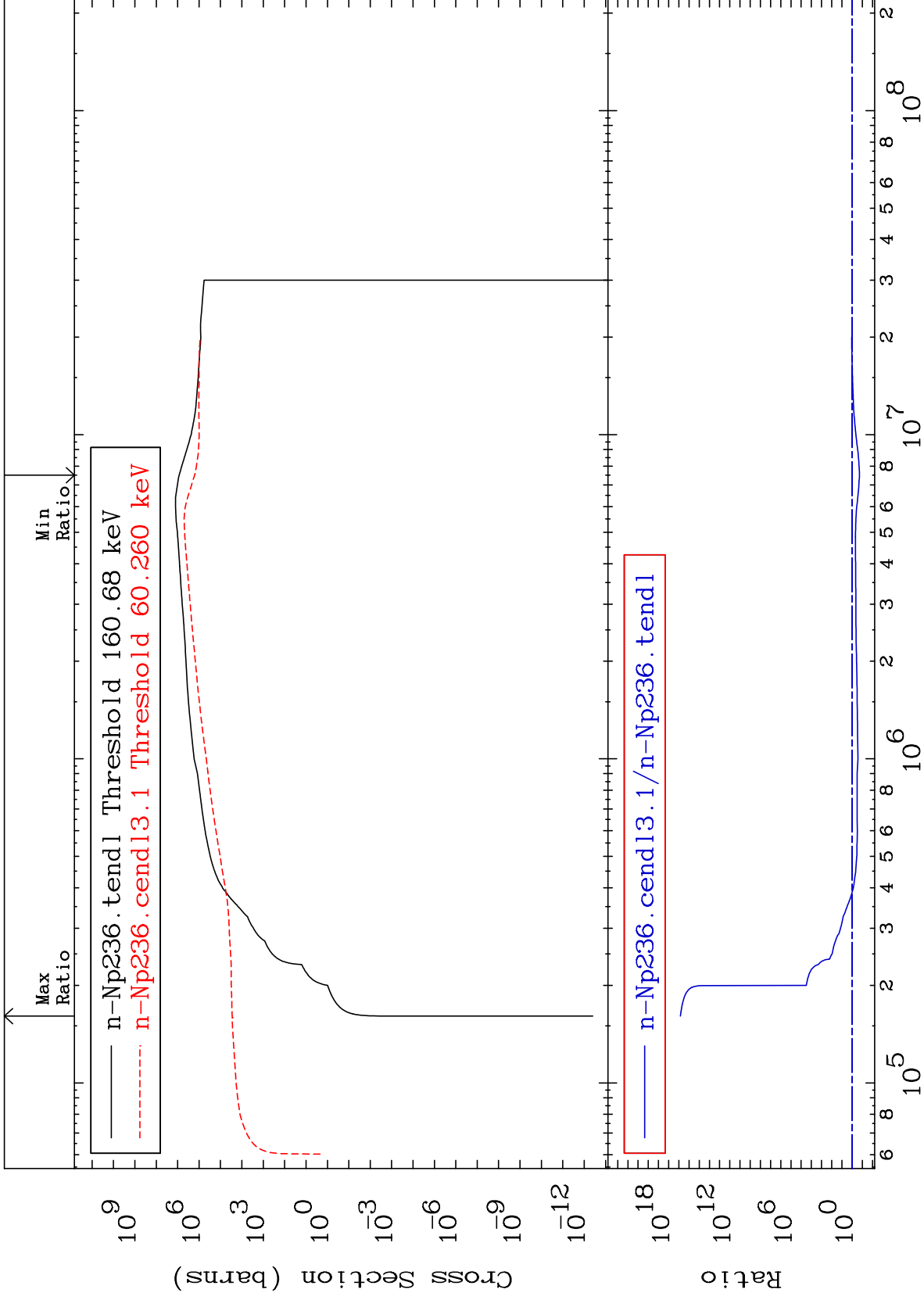


MAT 9343

Kerma non-elastic (all but mt2)
Cross Section

93-Np-236
-74.42 To 1669. %

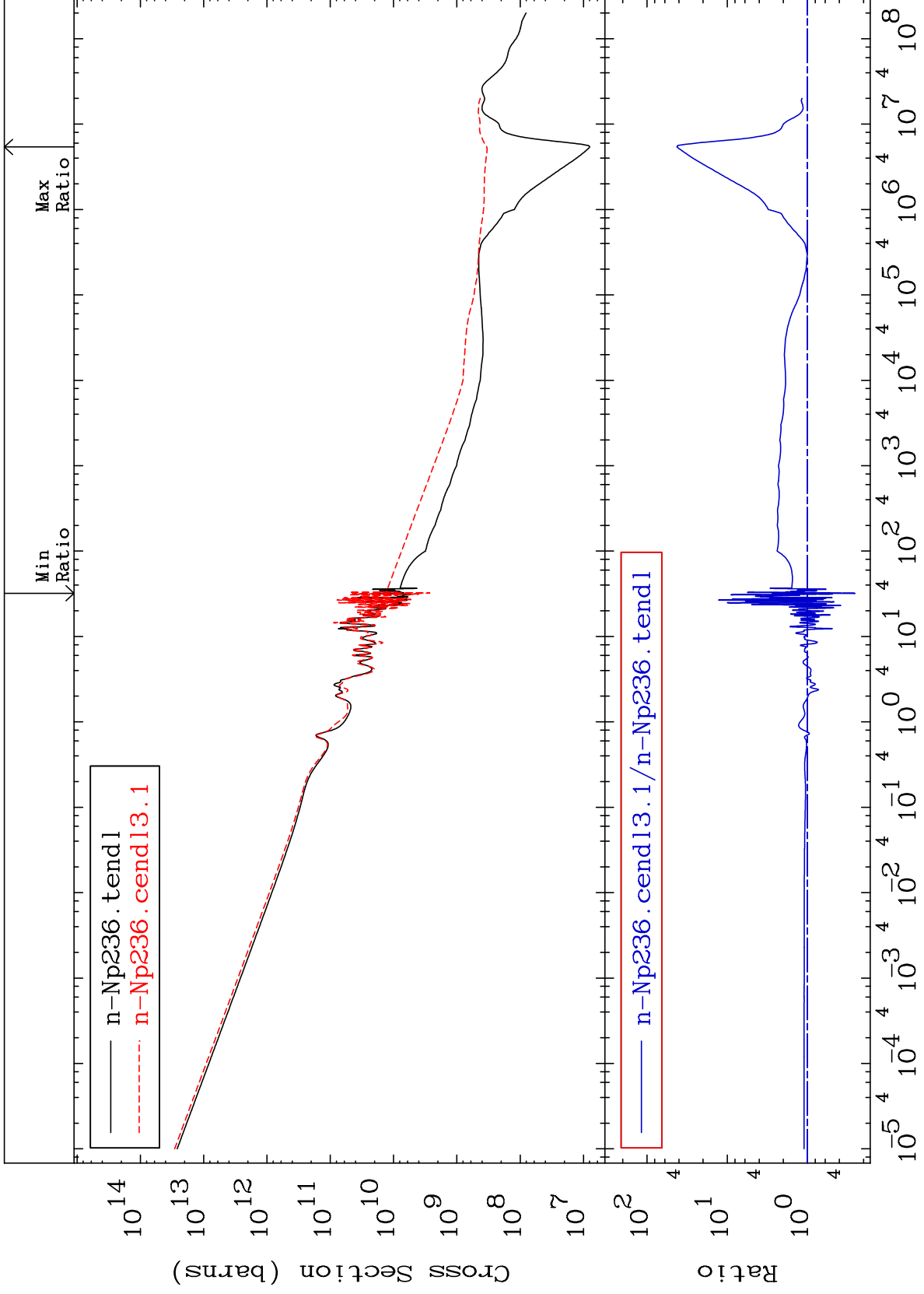




MAT 9343

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

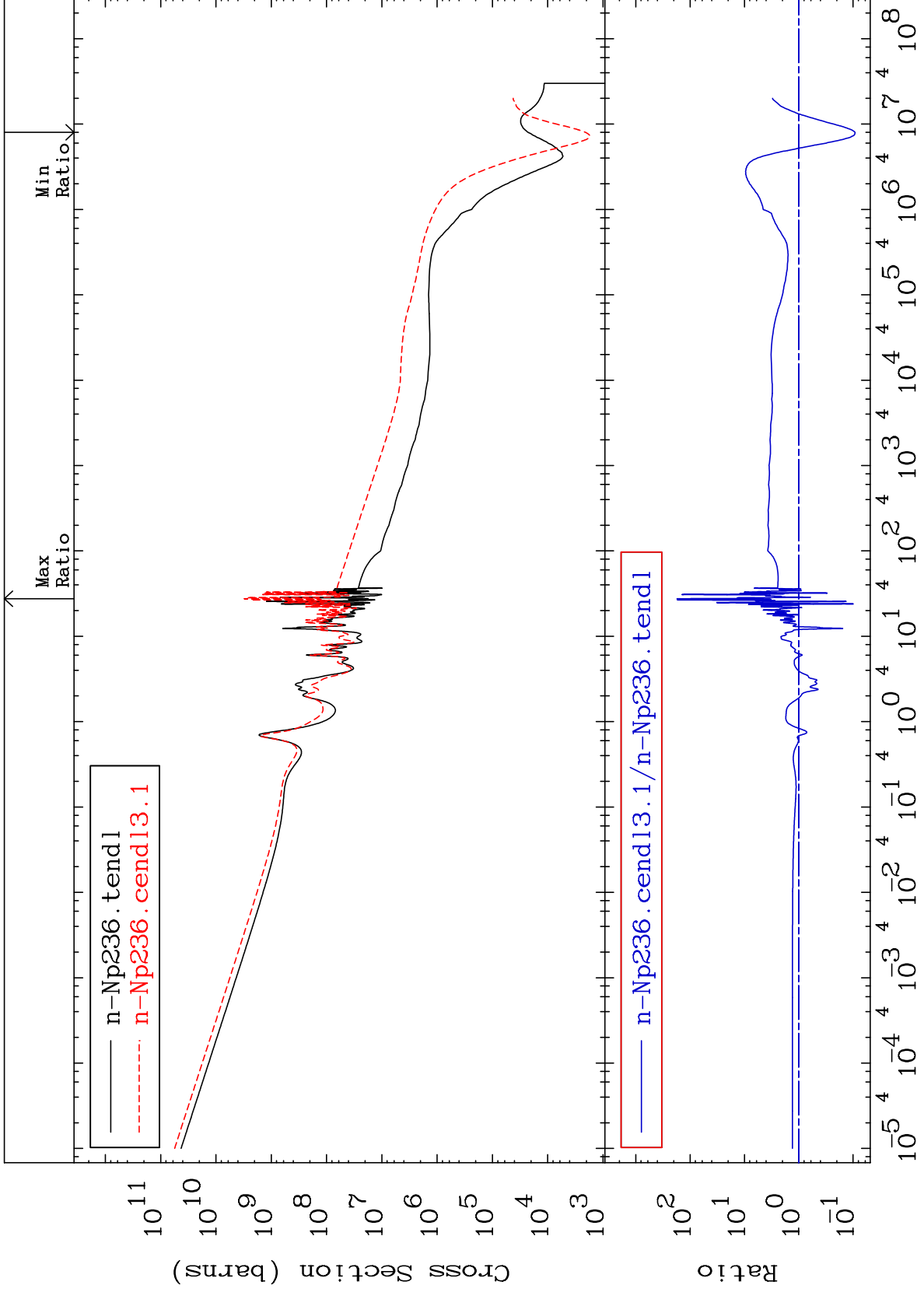
93-Np-236
-74.49 To 4131. %



MAT 9343

Kerma capture (mt102)
Cross Section

93-Np-236
-90.81 To 9999. %



26

Incident Energy (eV)

93-Np-236

MAT 9343

Total photon (eV-barns)
Cross Section

93-Np-236
-90.81 To 9999. %

