

Program EVALPLOT
(Version 2017-1)

by

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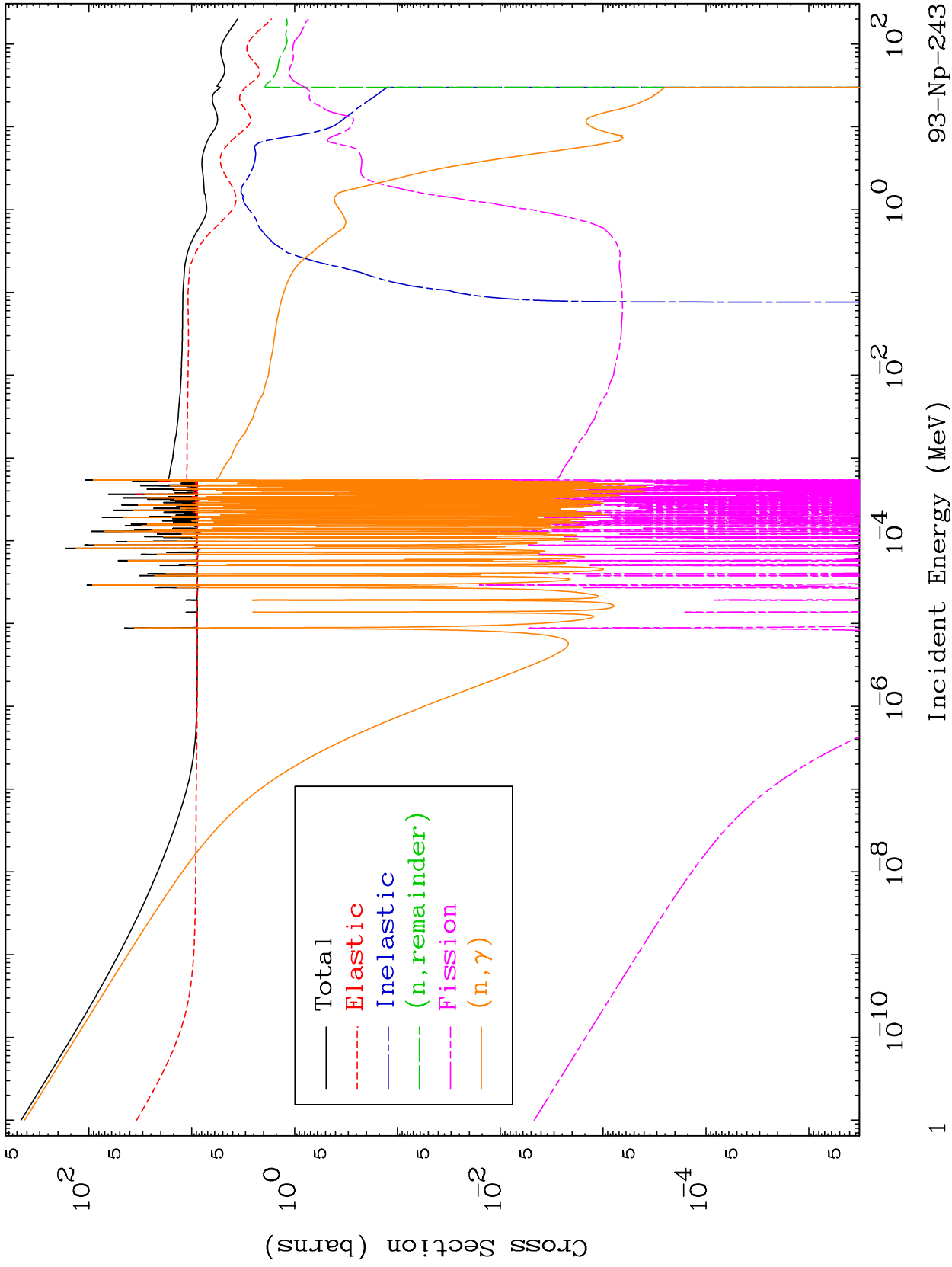
Press Mouse Button to Start

MAT 9364

Major

293 Kelvin Cross Sections

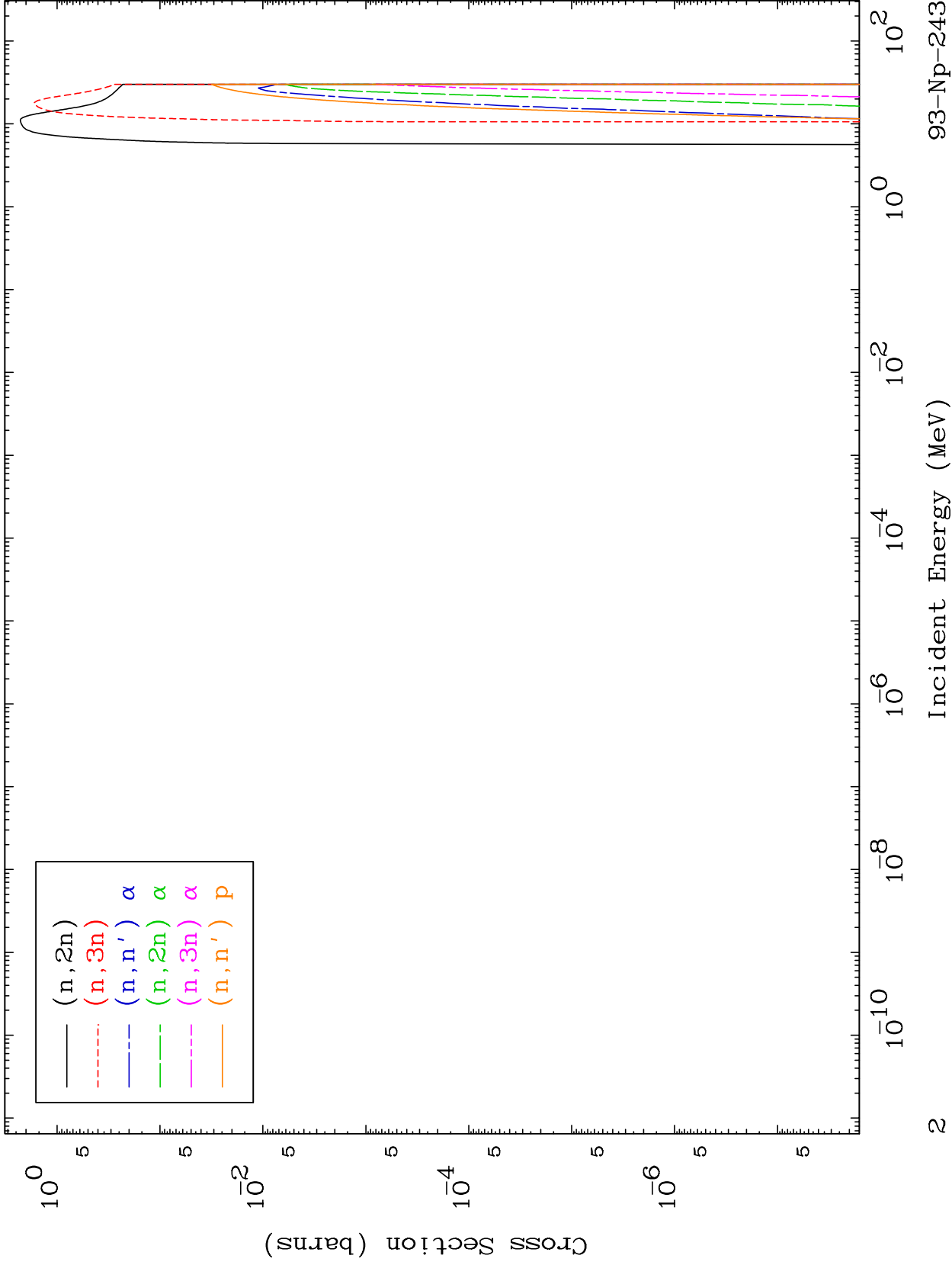
93-Np-243

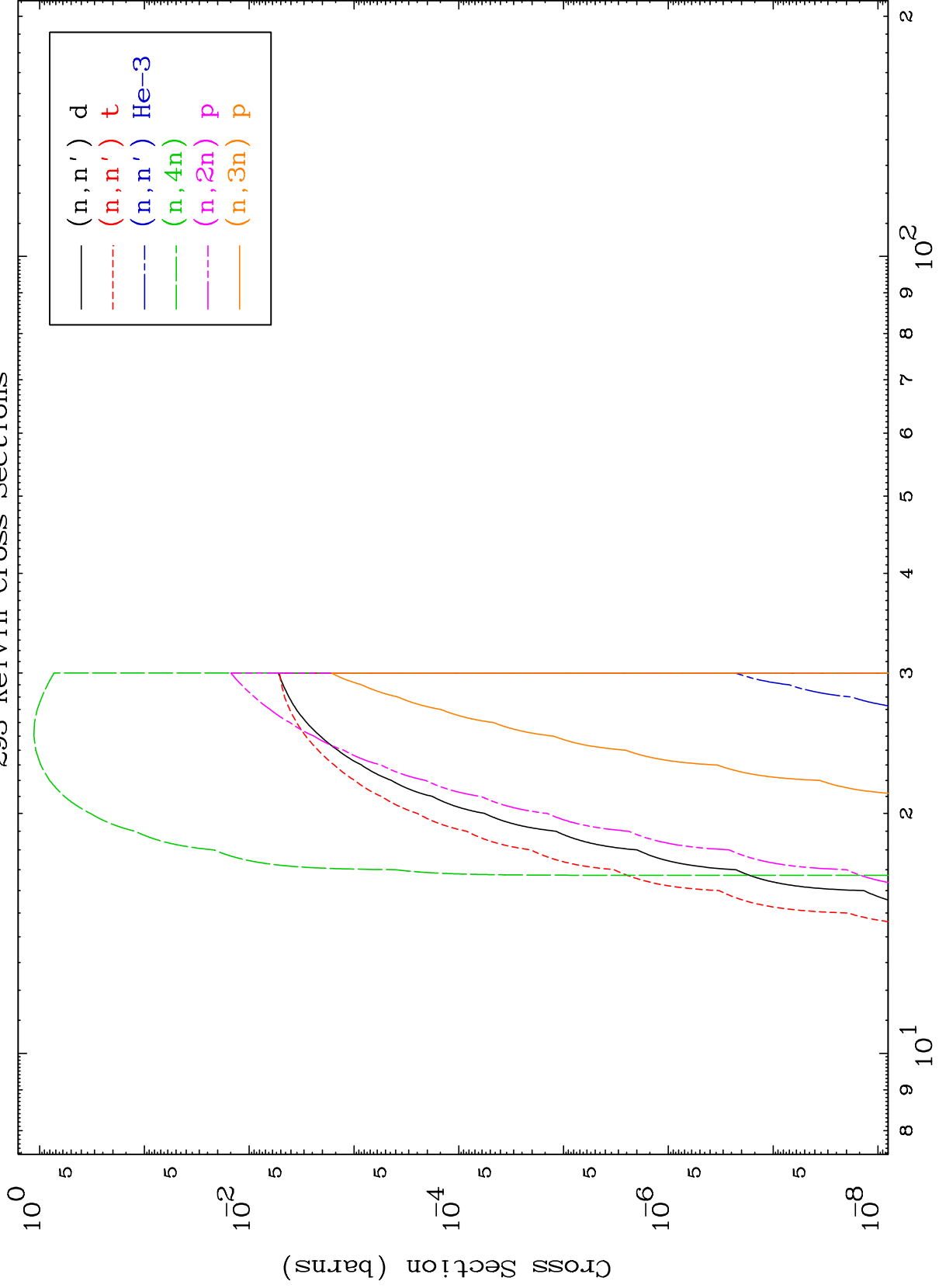


MAT 9364

Neutron Production
293 Kelvin Cross Sections

93-Np-243

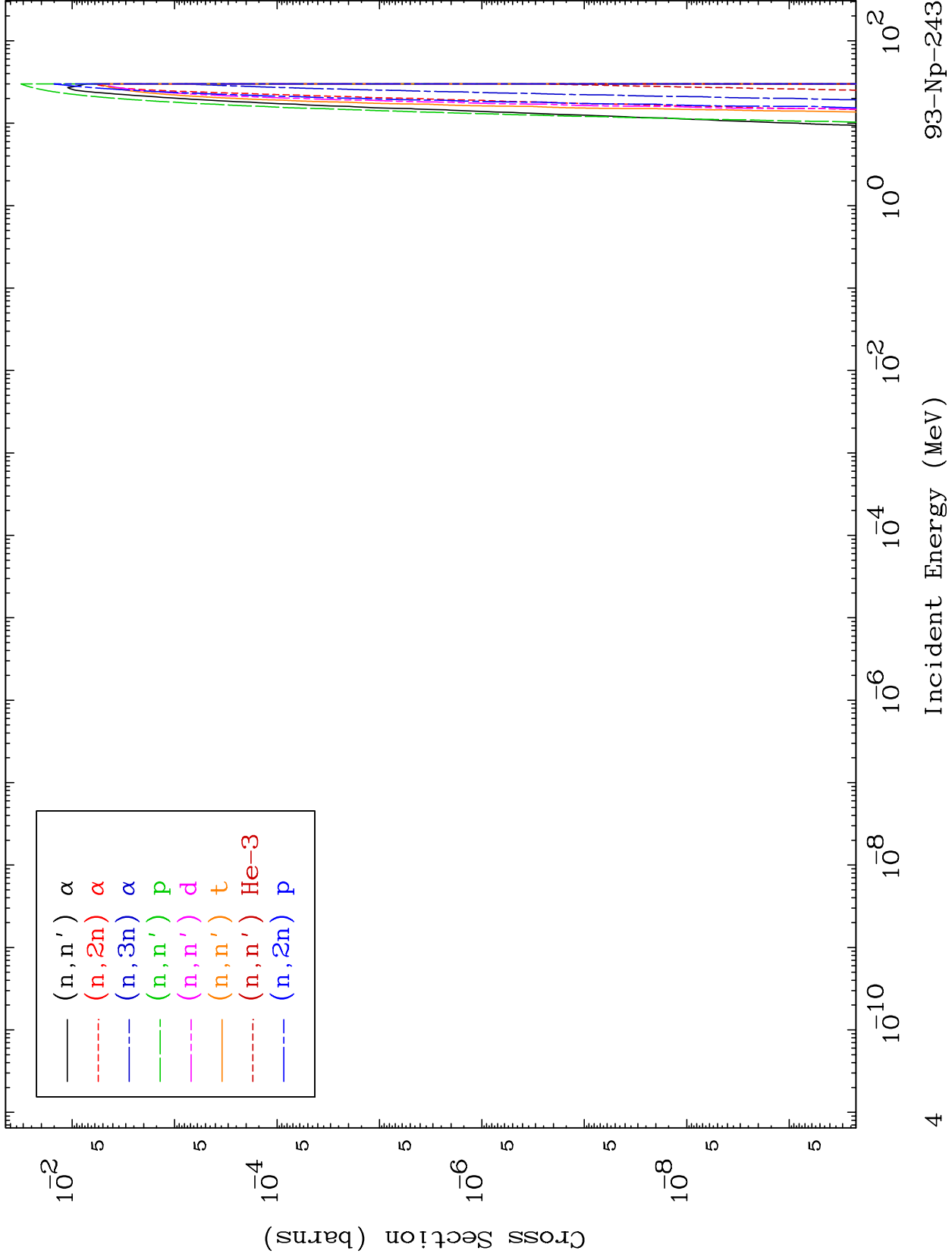




MAT 9364

Charged Particle
293 Kelvin Cross Sections

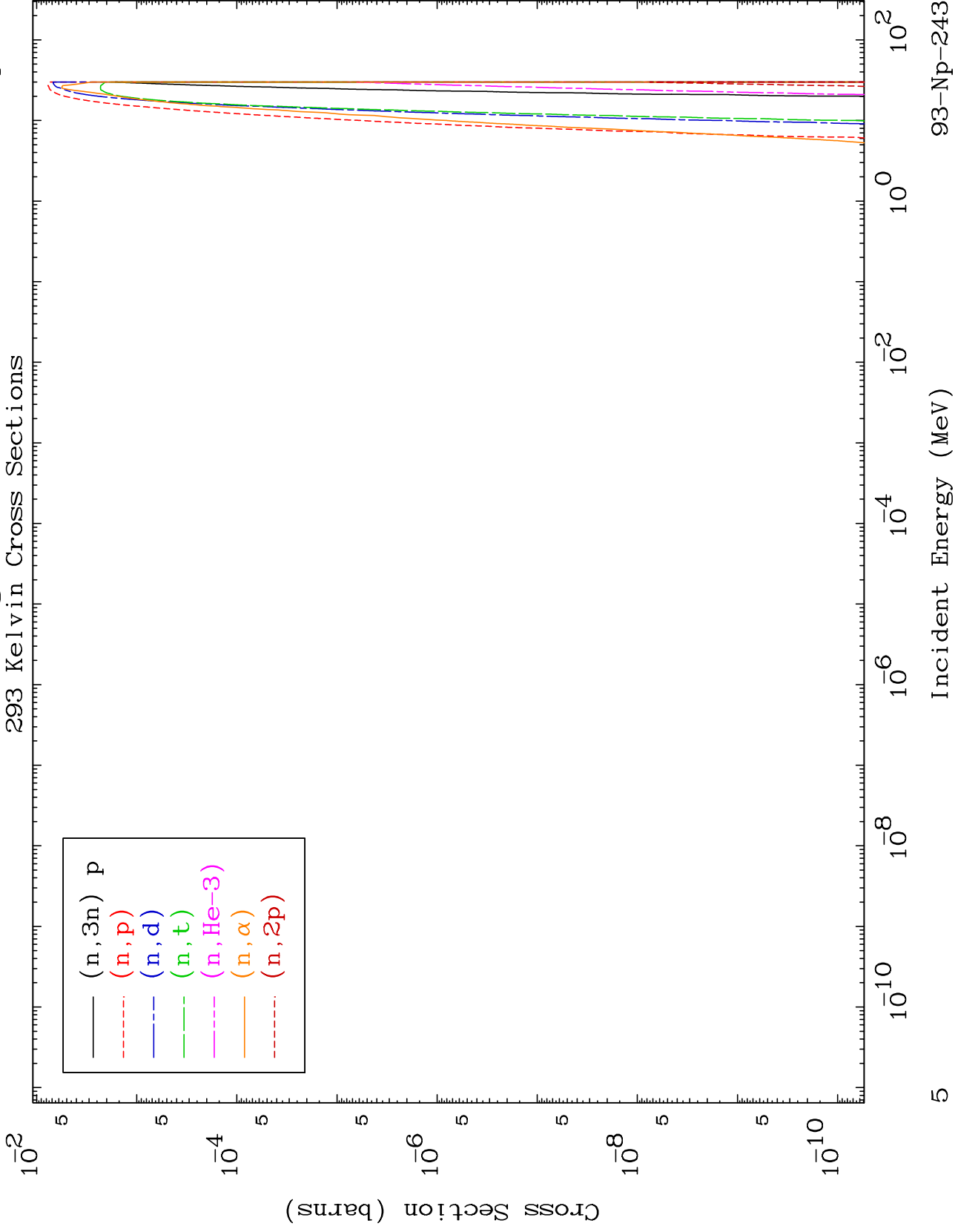
93-Np-243



MAT 9364

Charged Particle
293 Kelvin Cross Sections

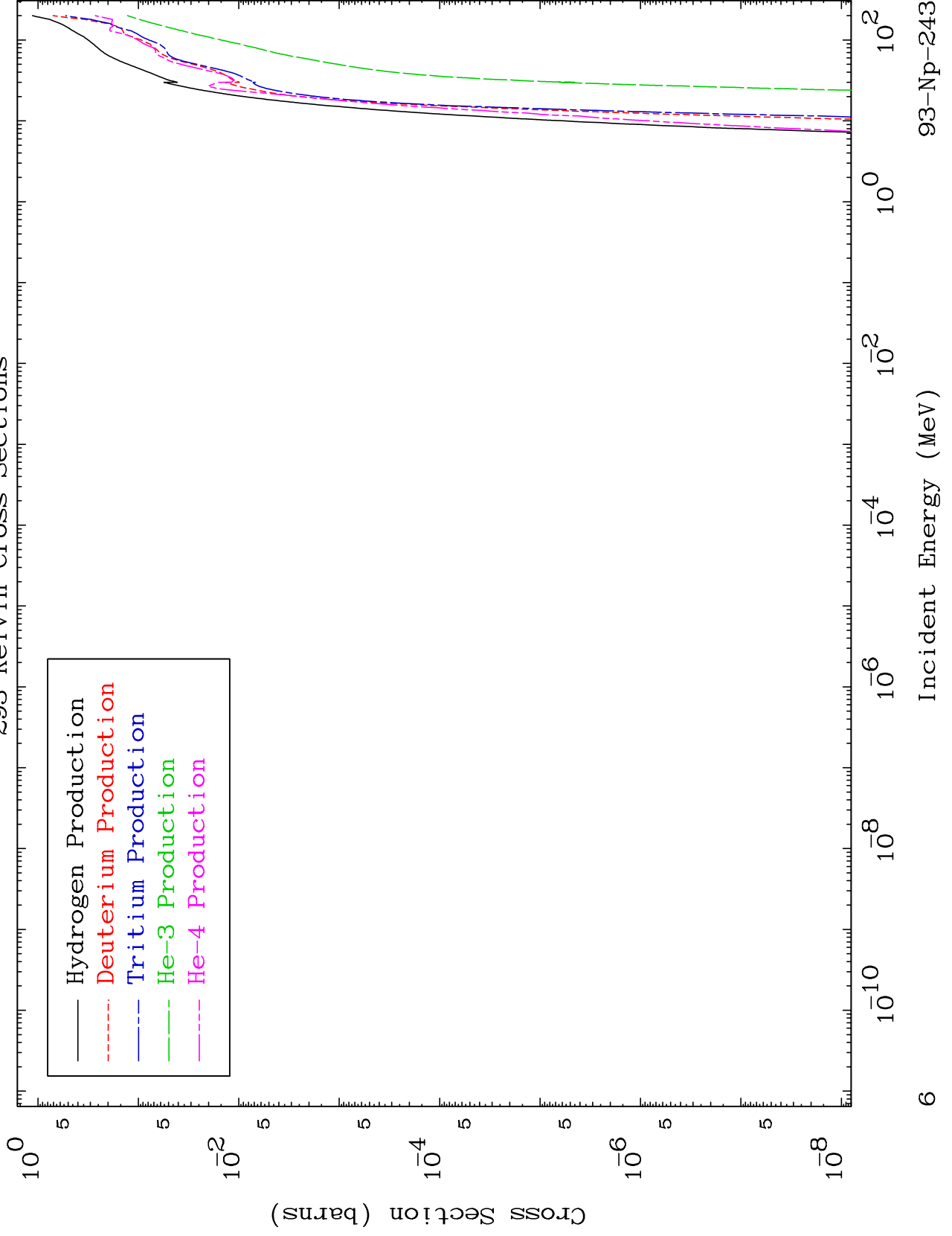
93-Np-243



MAT 9364

Particle Production
293 Kelvin Cross Sections

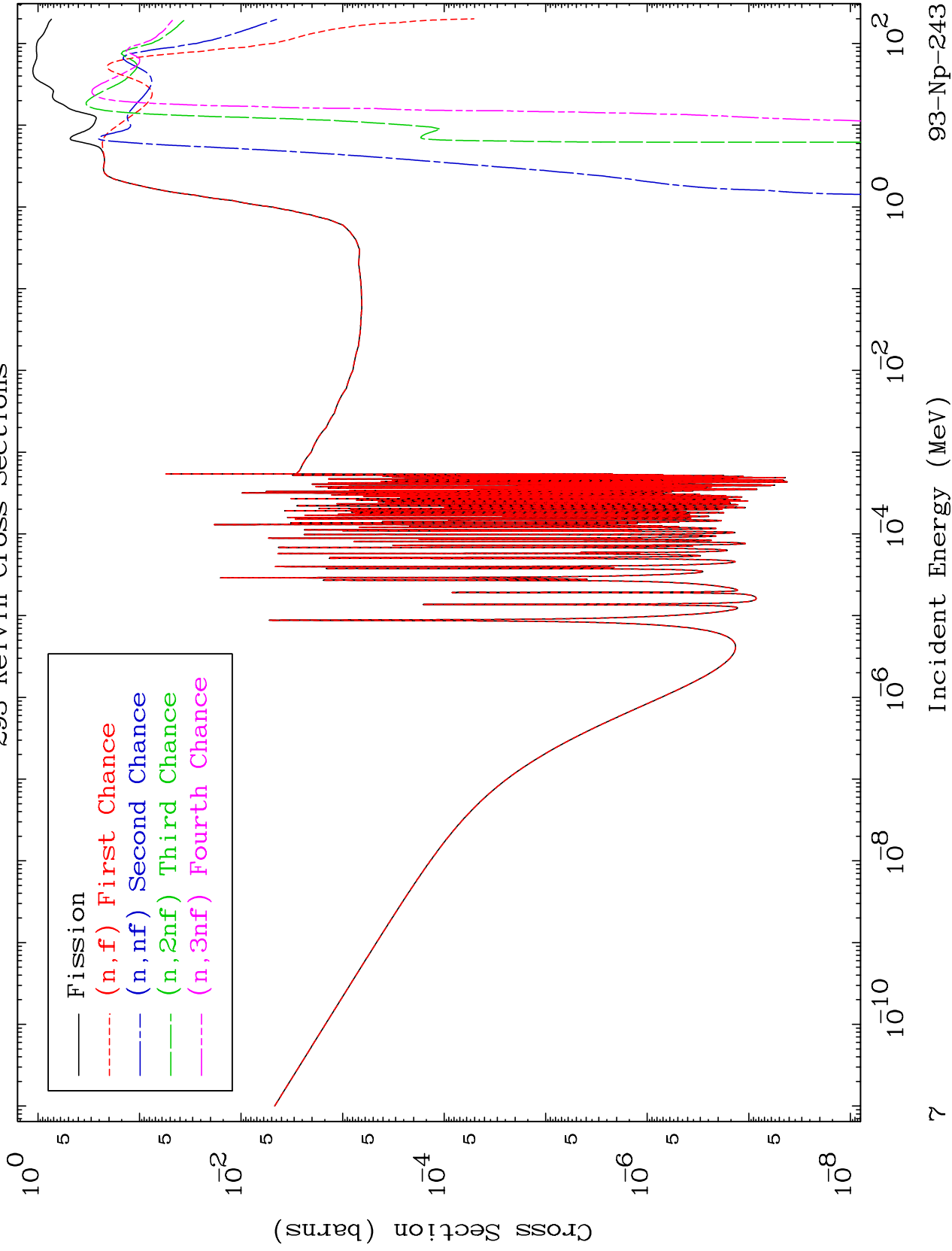
93-Np-243



MAT 9364

Fission
293 Kelvin Cross Sections

93-Np-243

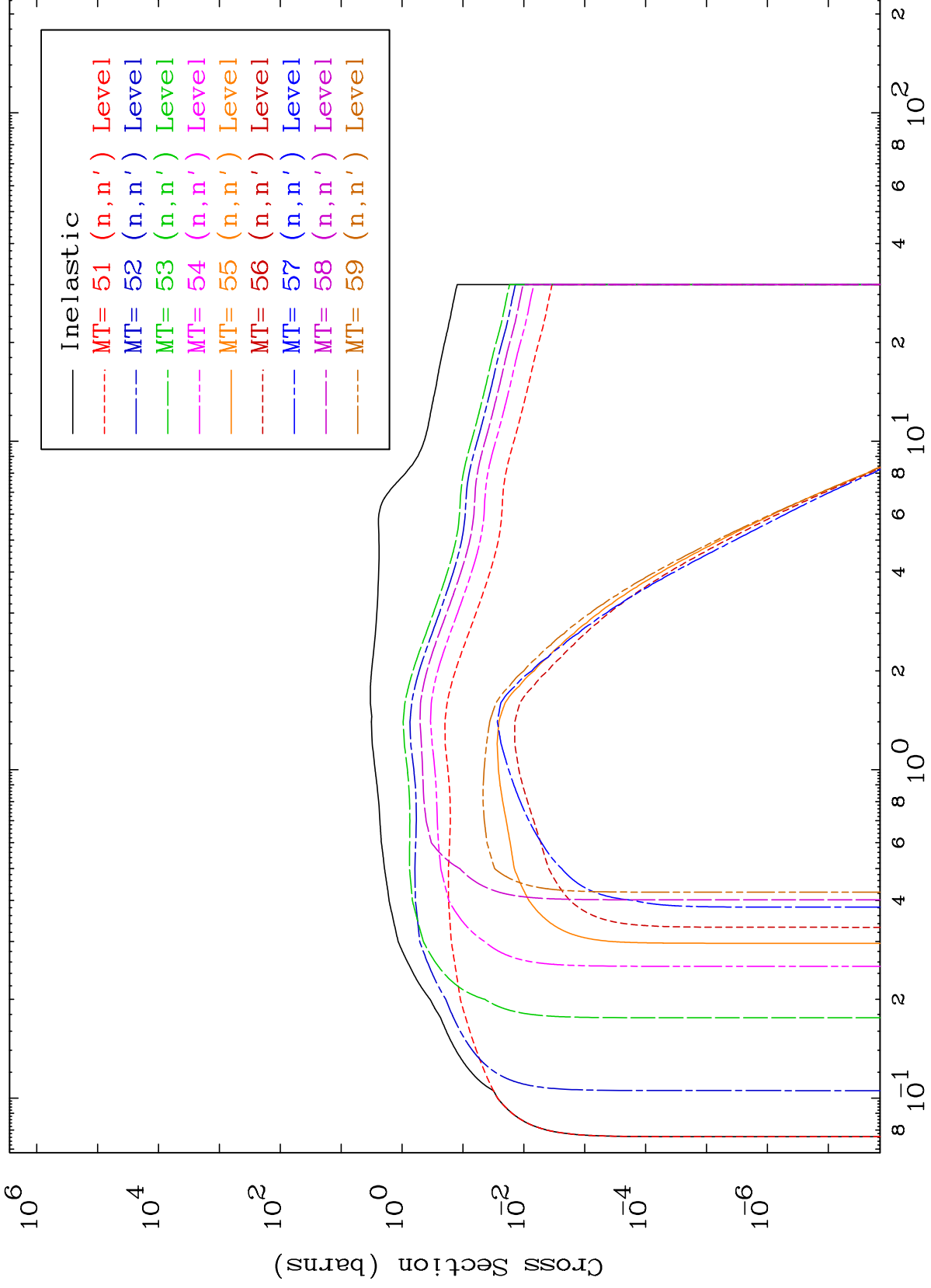


MAT 9364

(n,n') Level

93-Np-243

293 Kelvin Cross Sections



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Incident Energy (MeV)

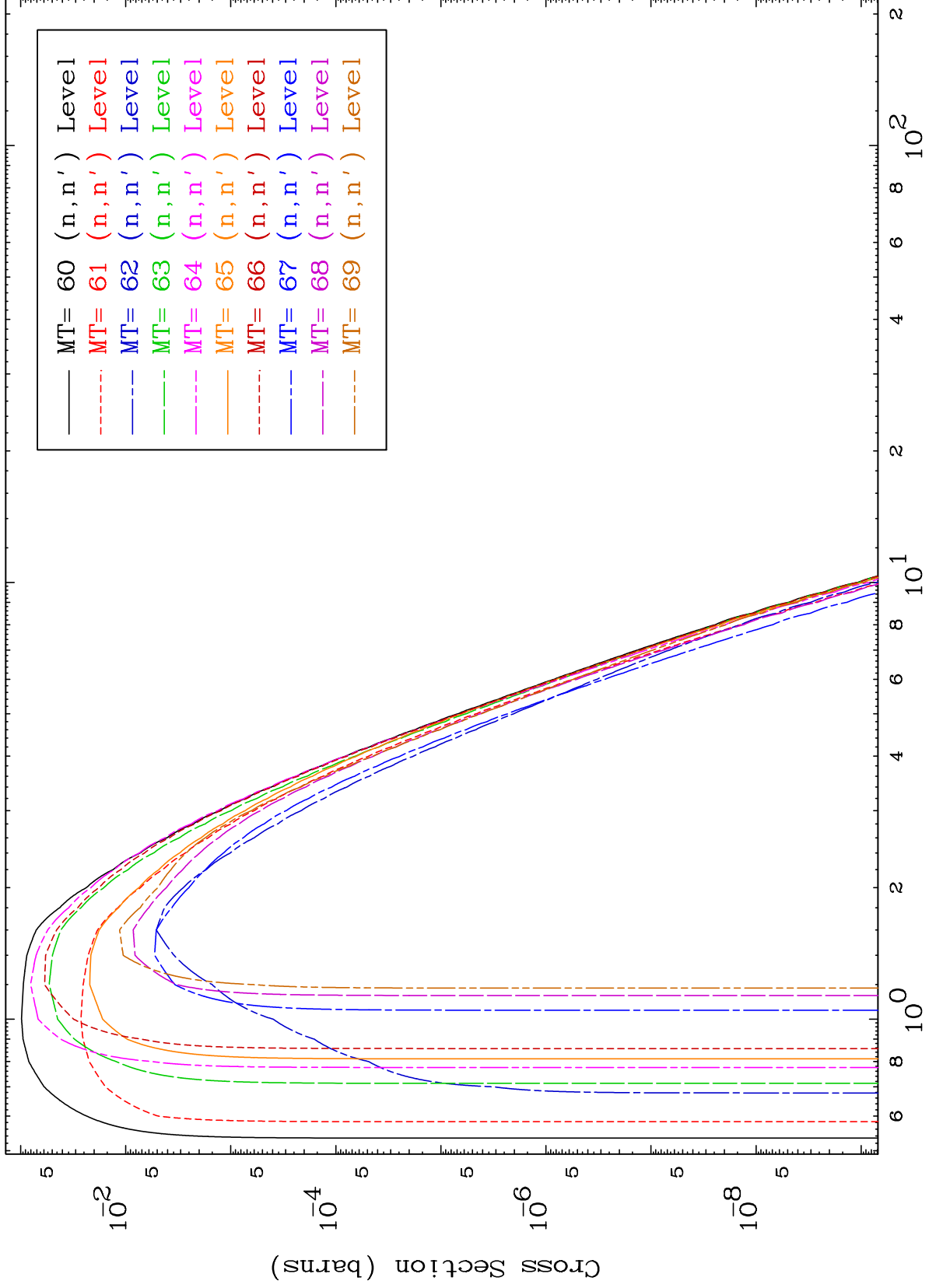
93-Np-243

MAT 9364

(n,n') Level

93-Np-243

293 Kelvin Cross Sections



9

Incident Energy (MeV)

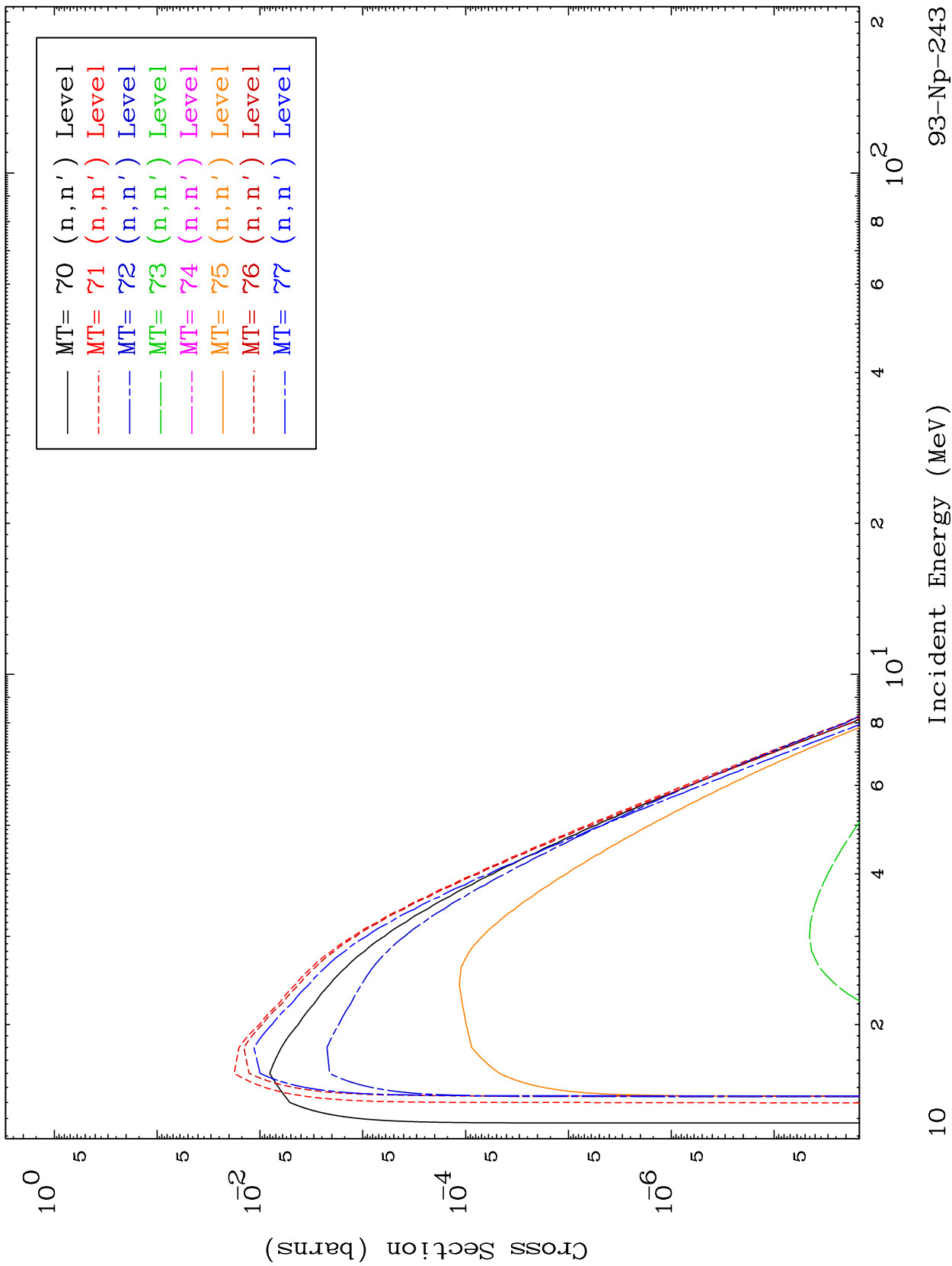
93-Np-243

MAT 9364

(n,n') Level

93-Np-243

293 Kelvin Cross Sections



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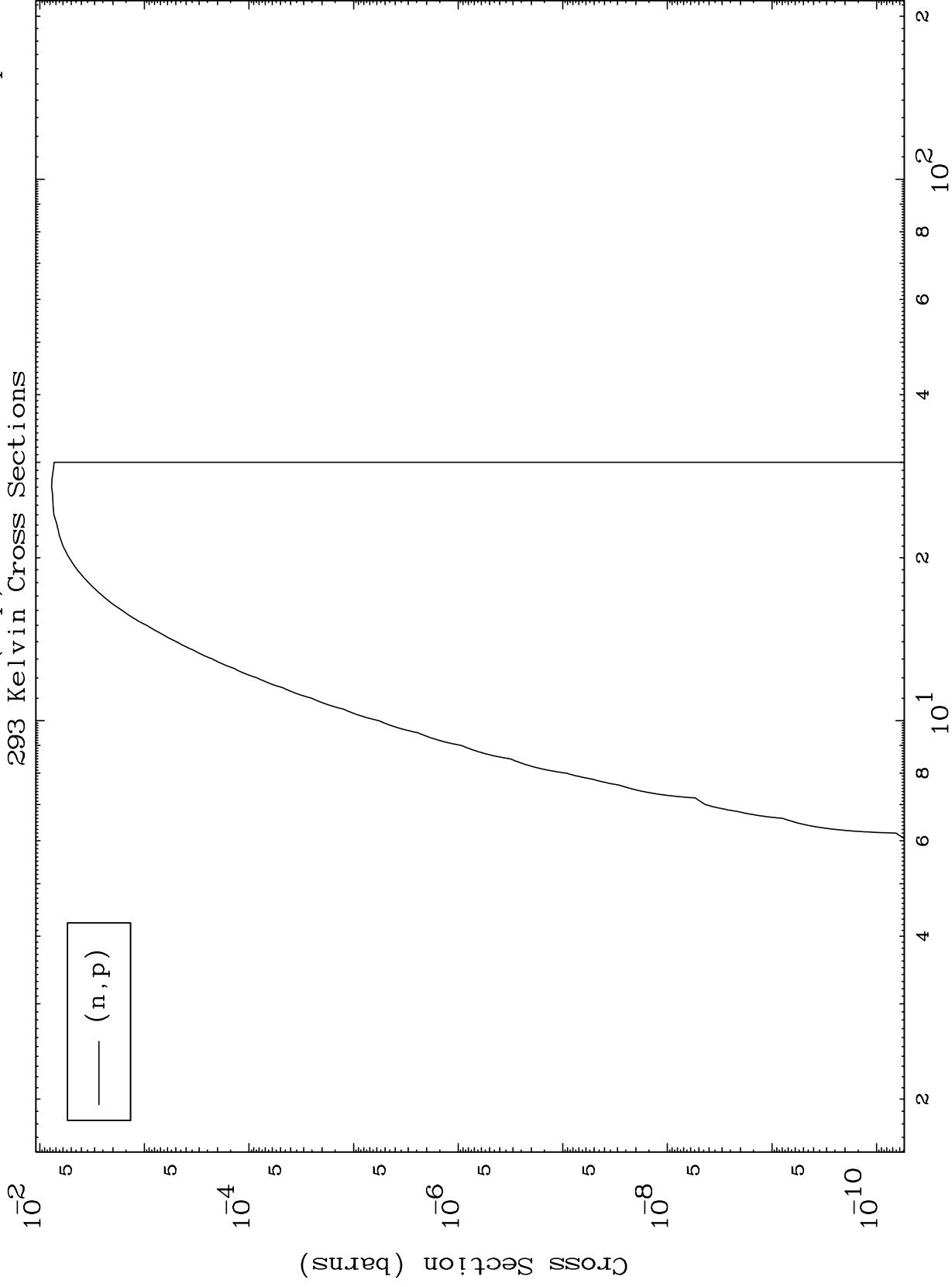
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,p) Levels
293 Kelvin Cross Sections

93-Np-243



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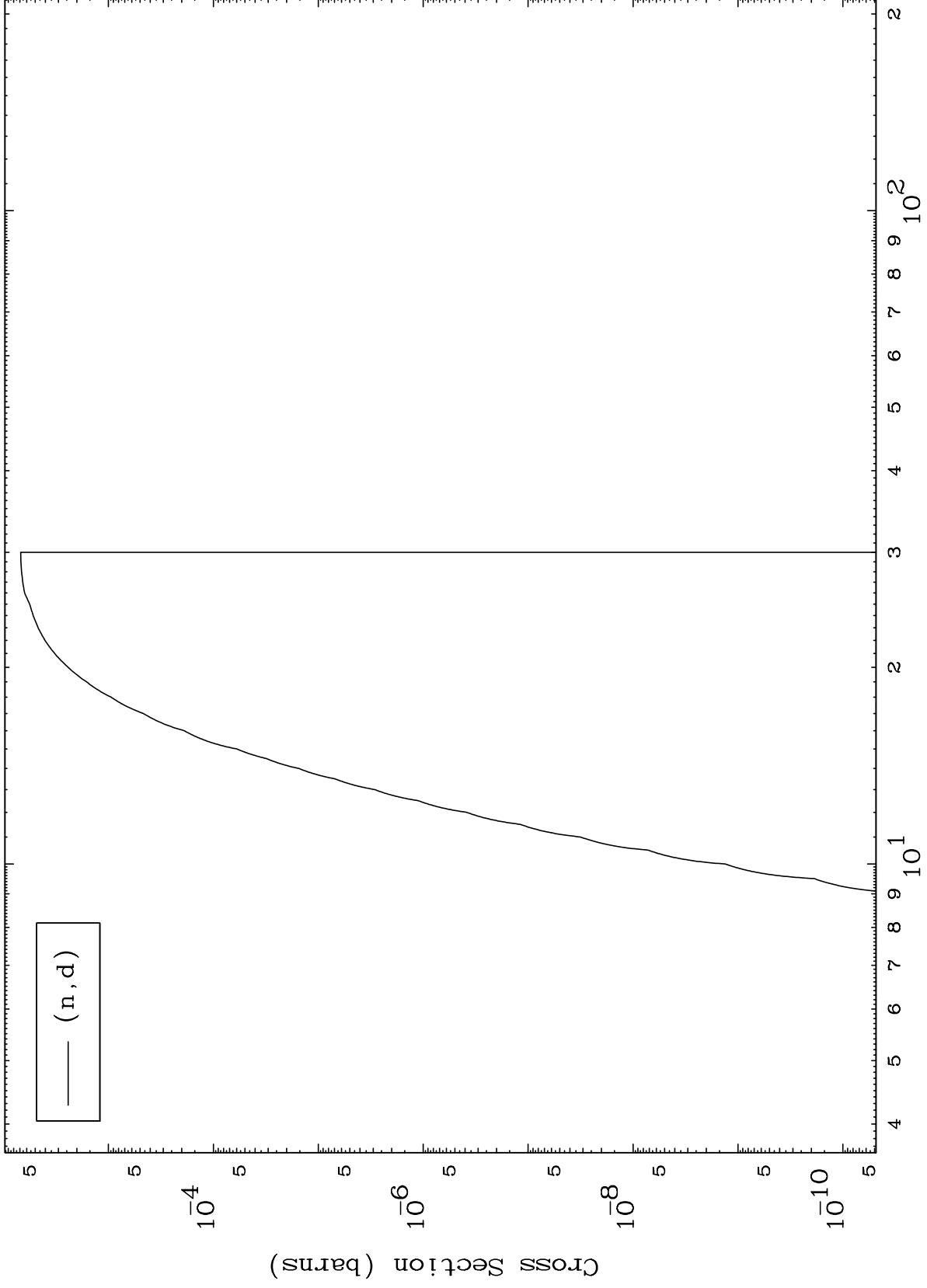
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,d) Levels
293 Kelvin Cross Sections

93-Np-243



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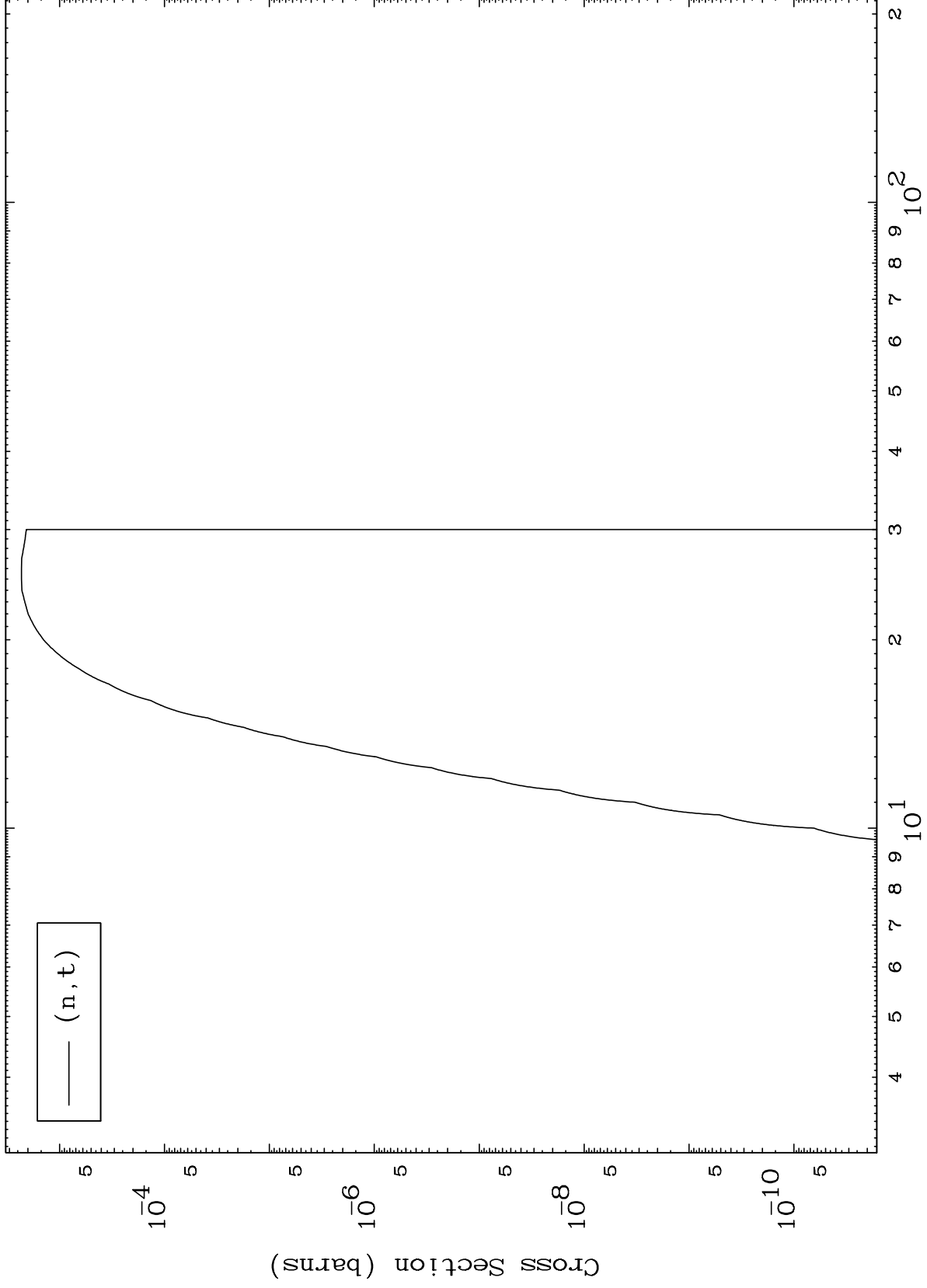
Incident Energy (MeV)

93-Np-243

MAT 9364

(n,t) Levels
293 Kelvin Cross Sections

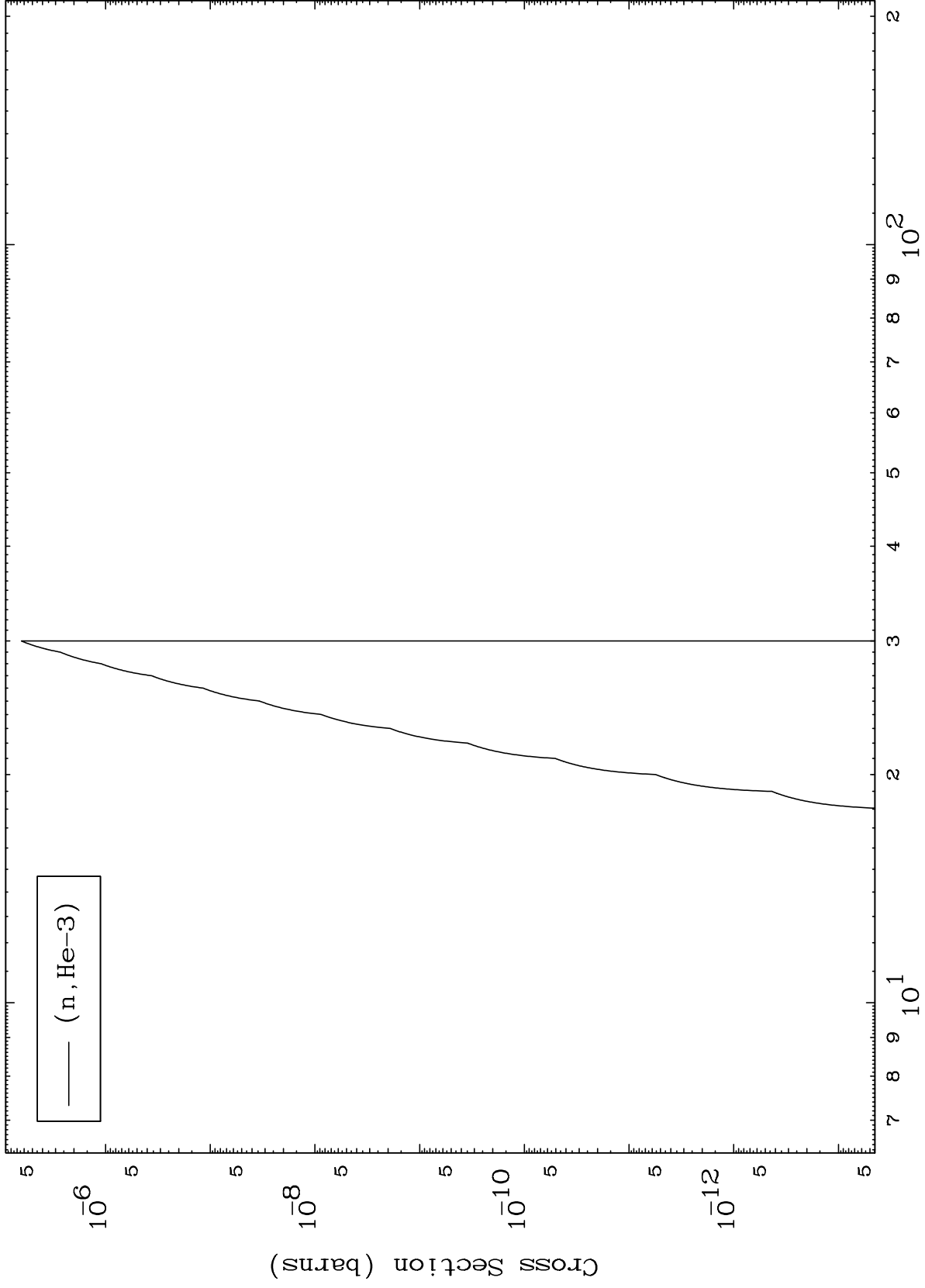
93-Np-243



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(n,He3) Levels
293 Kelvin Cross Sections

93-Np-243



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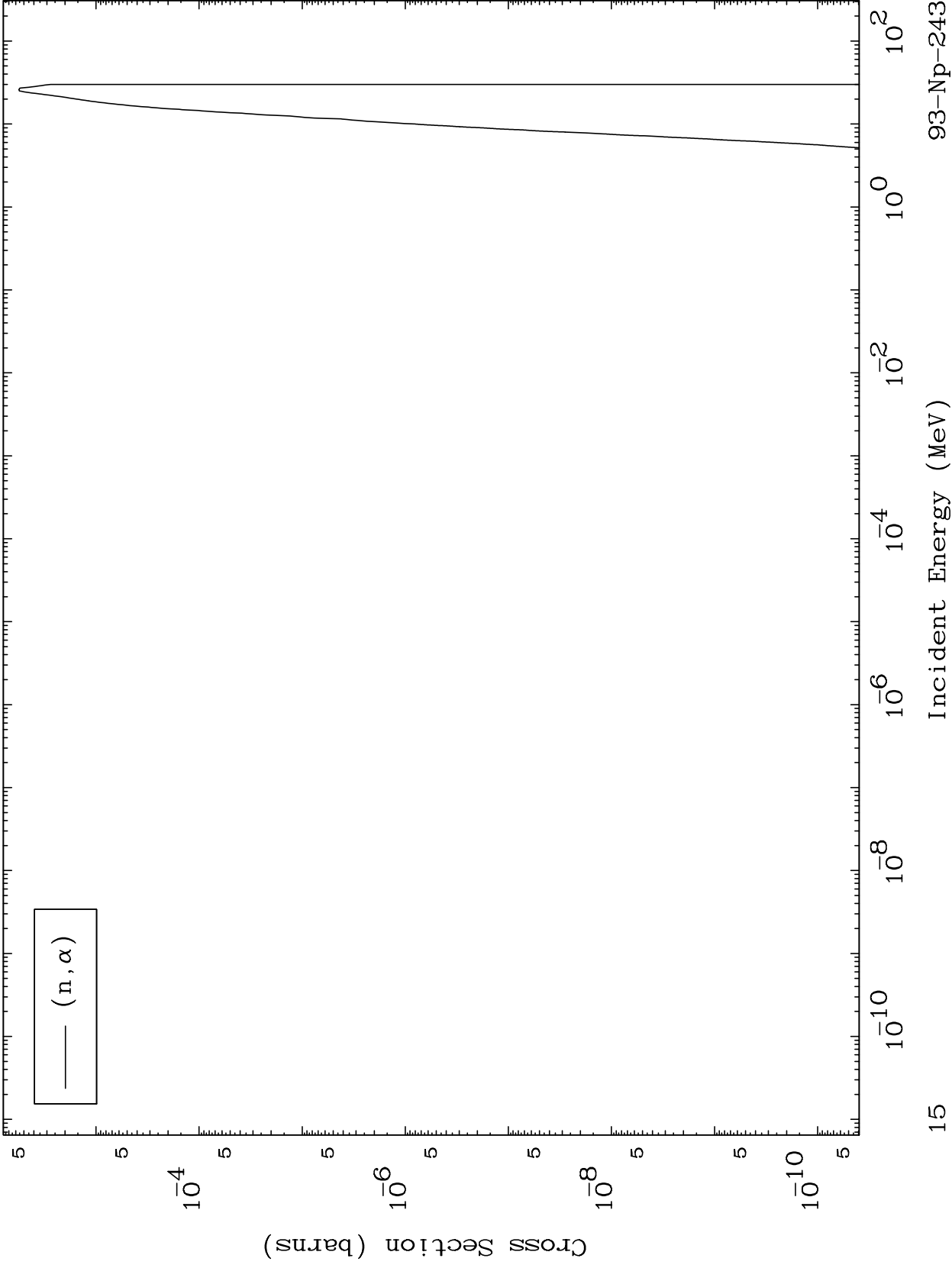
Incident Energy (MeV)

93-Np-243

MAT 9364

(n, α) Levels
293 Kelvin Cross Sections

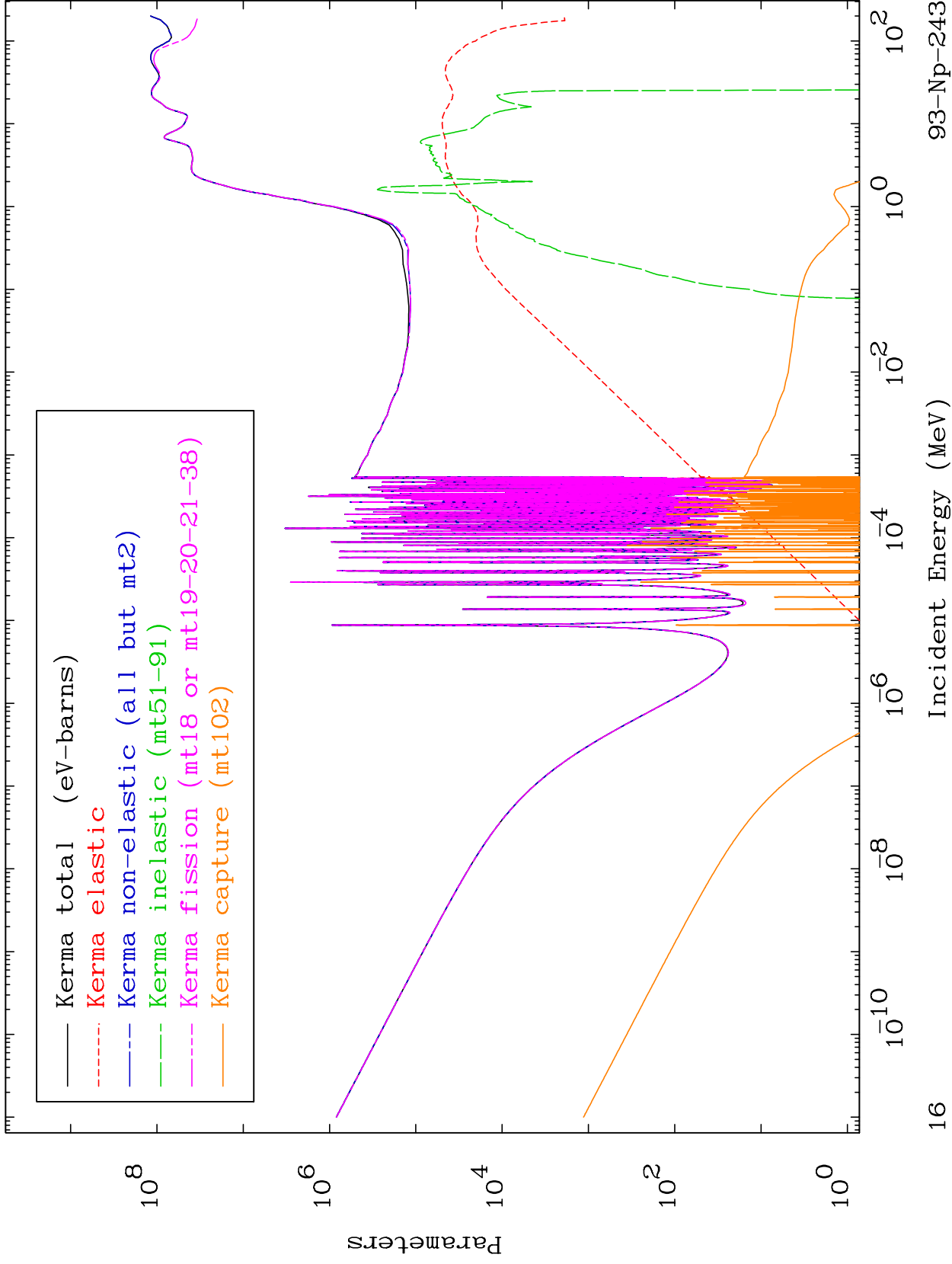
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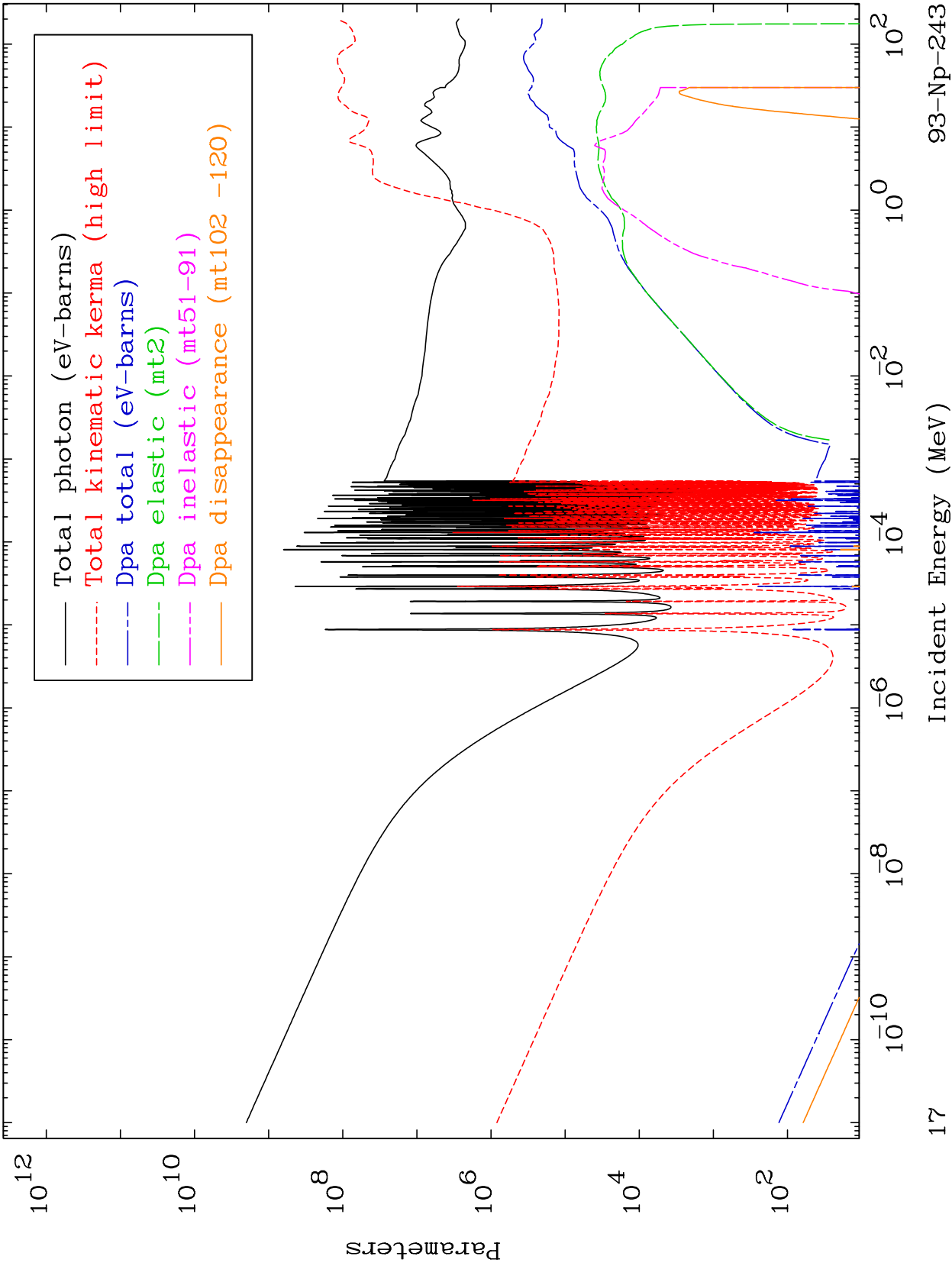


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Incident Energy (MeV)

93-Np-243

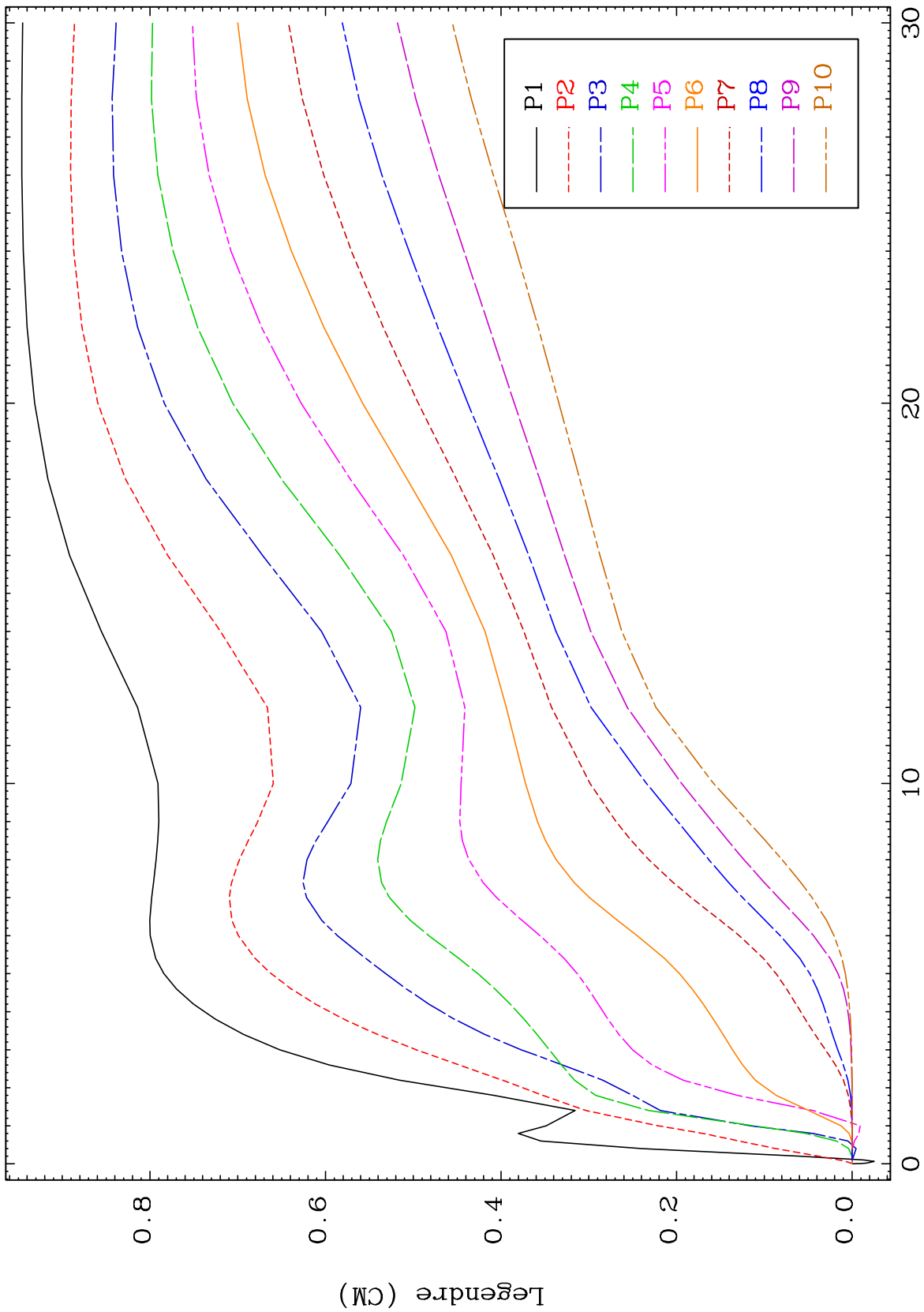




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93-Np-243

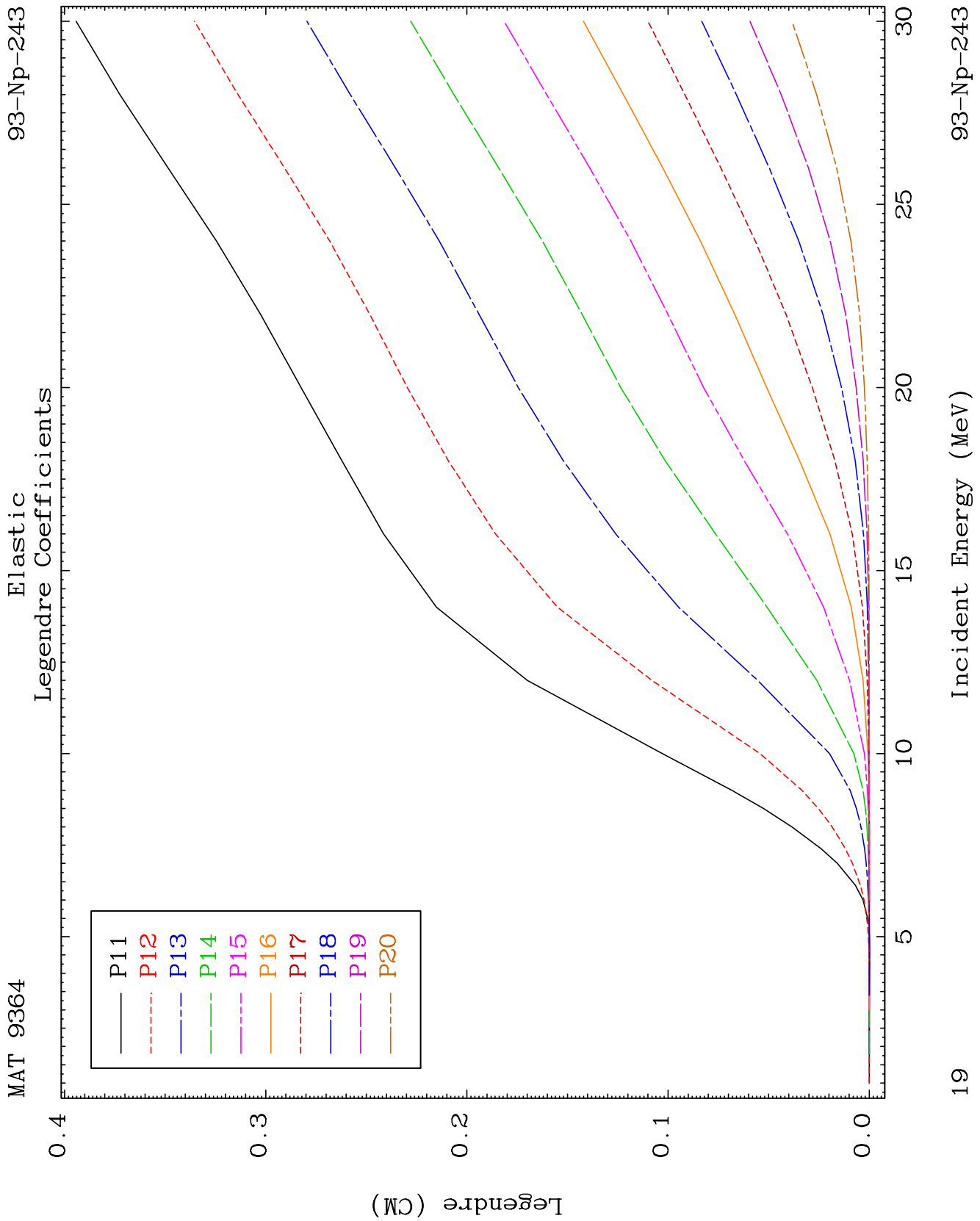
Elastic Legendre Coefficients



93-Np-243

Incident Energy (MeV)

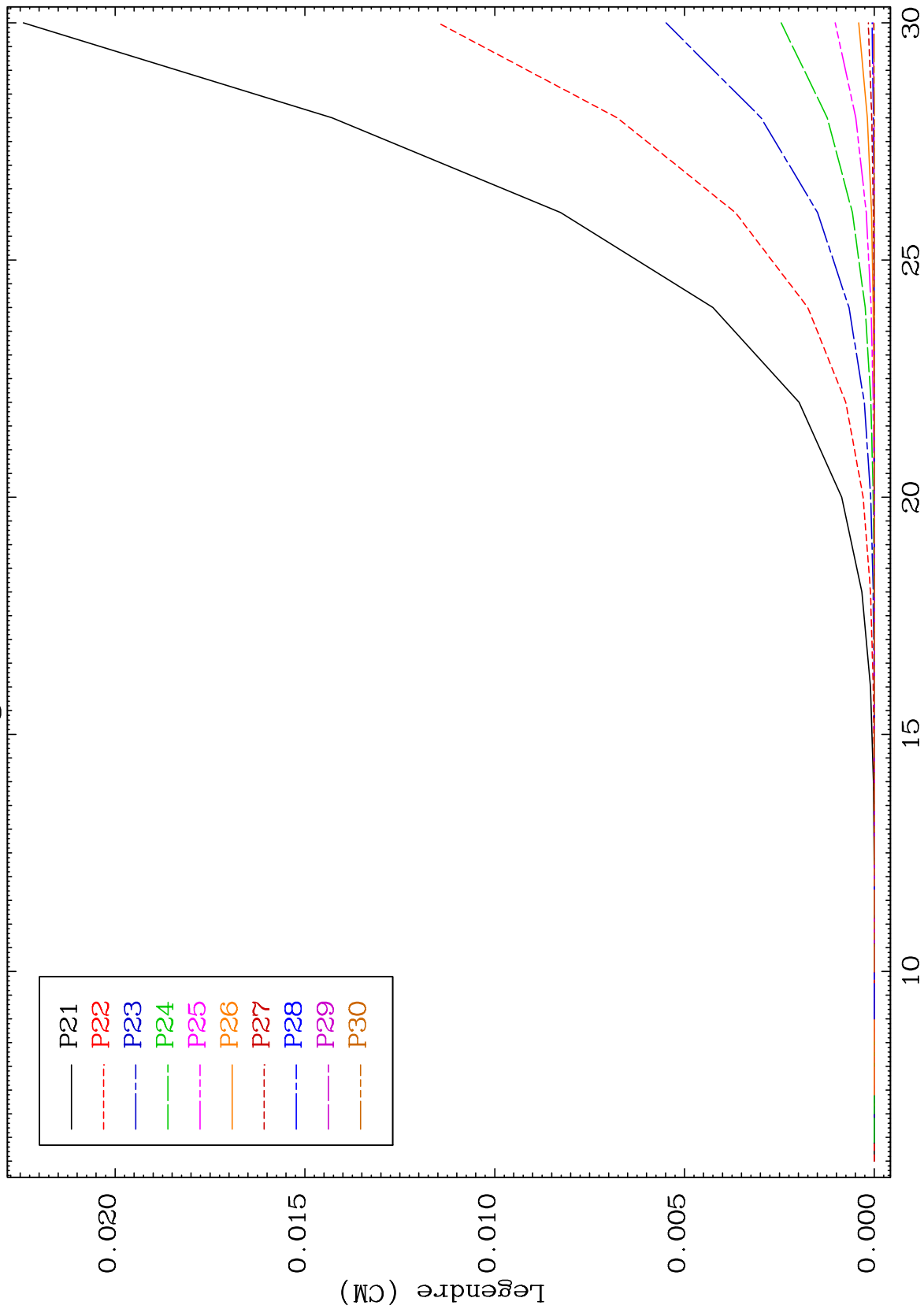
18



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Elastic Legendre Coefficients

93-Np-243



93-Np-243

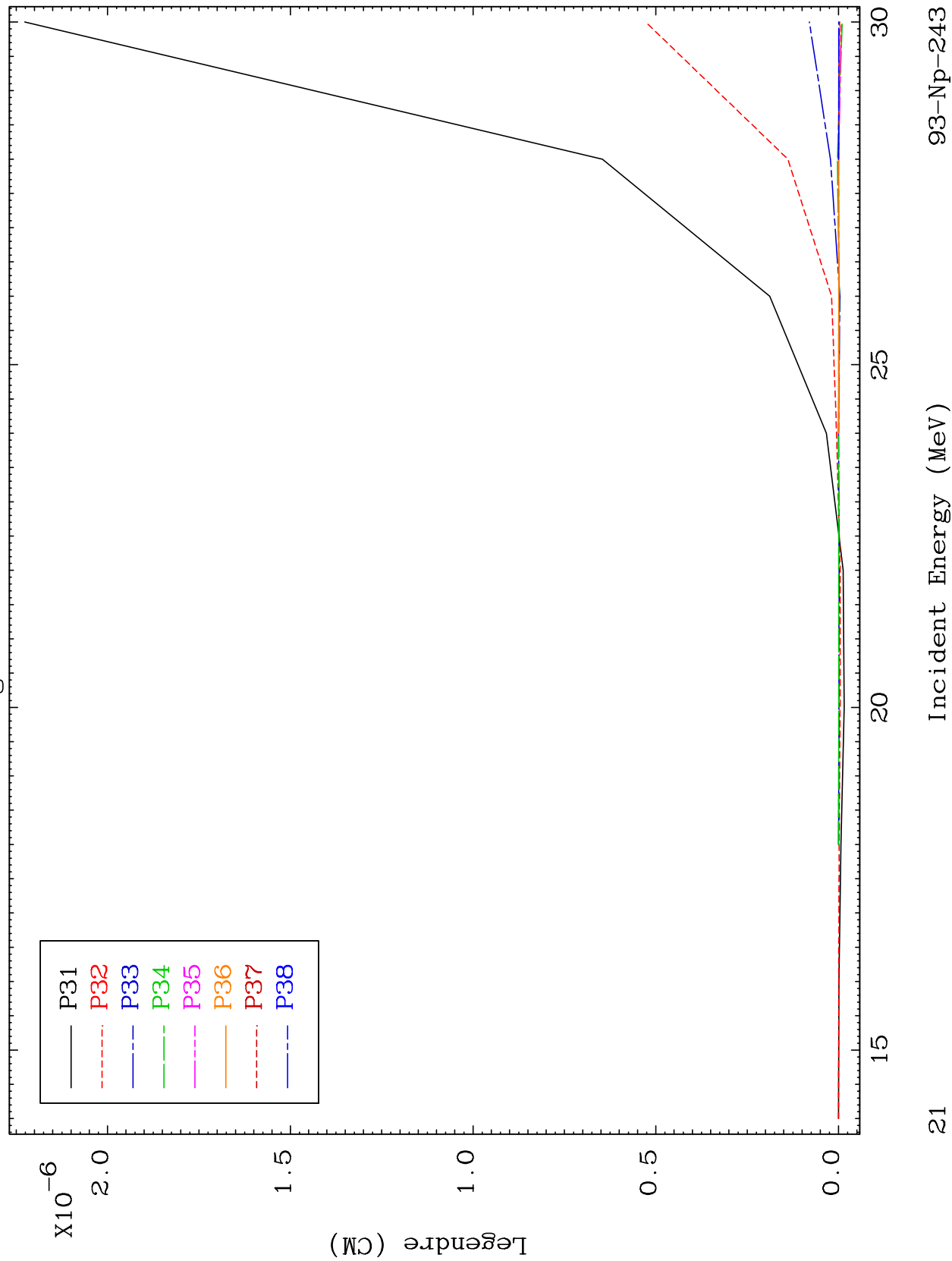
Incident Energy (MeV)

20

MAT 9364

Elastic Legendre Coefficients

93-Np-243



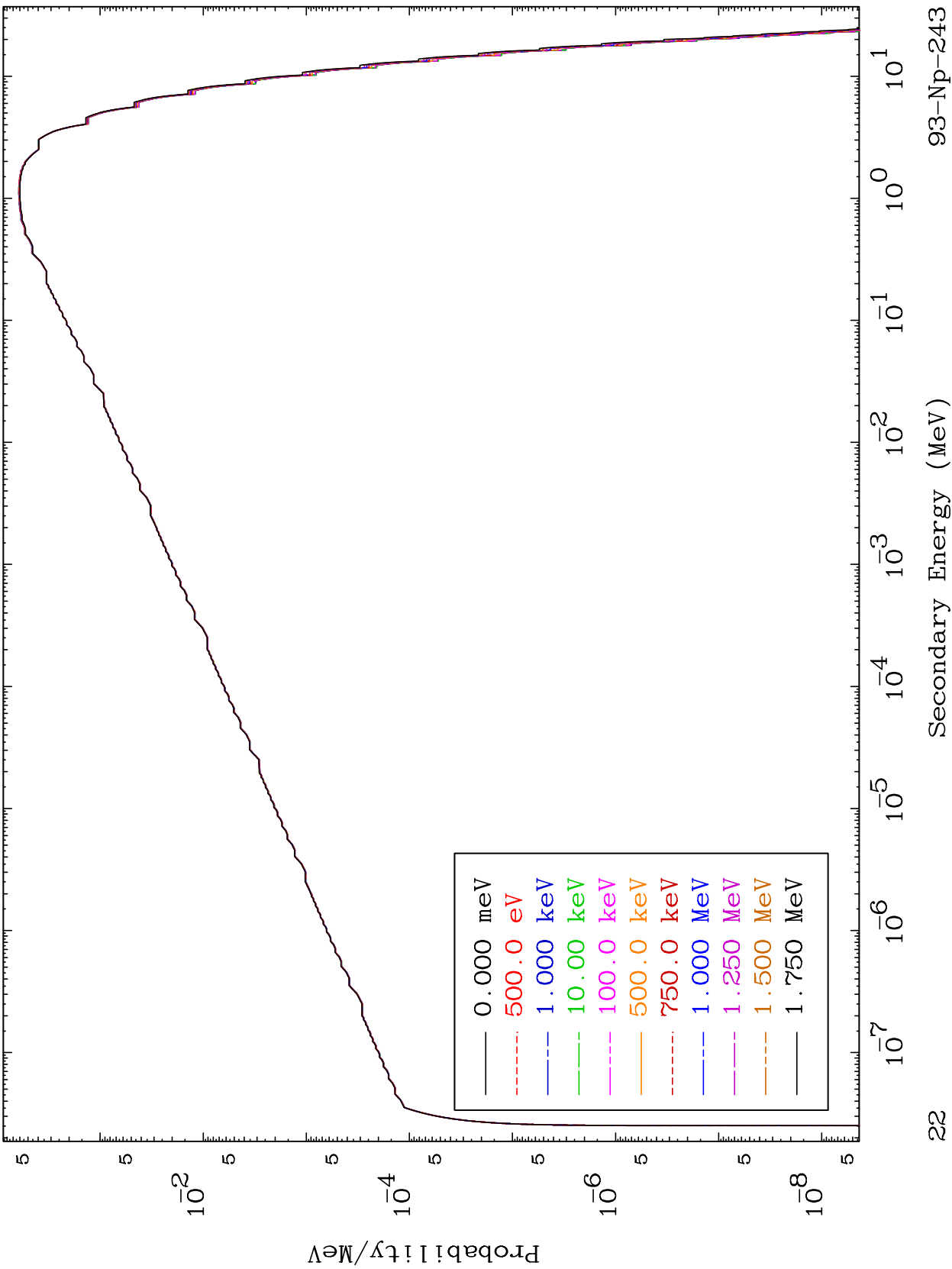
21

93-Np-243

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Fission Energy Distribution

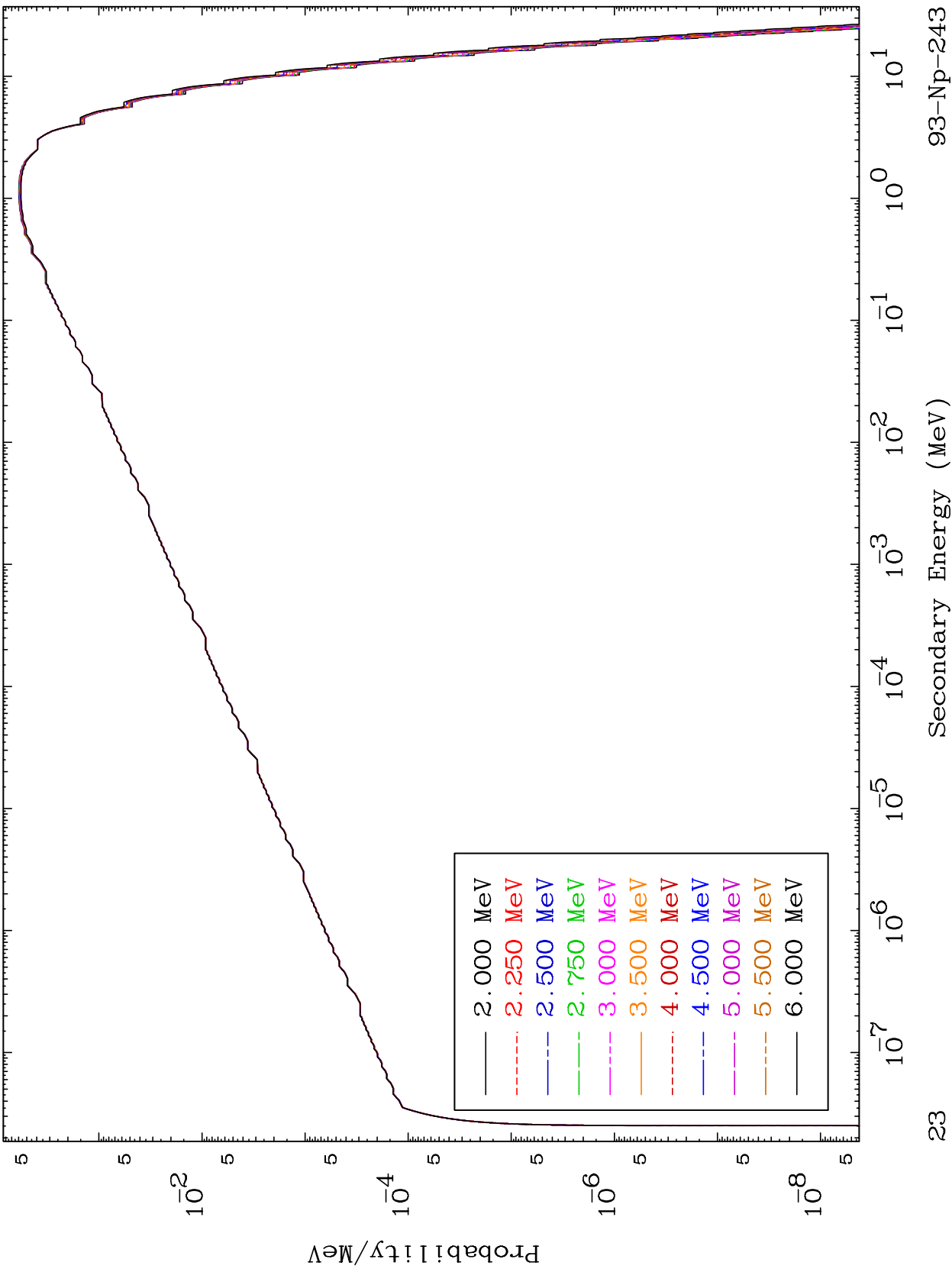
93-Np-243



MAT 9364

Fission Energy Distribution

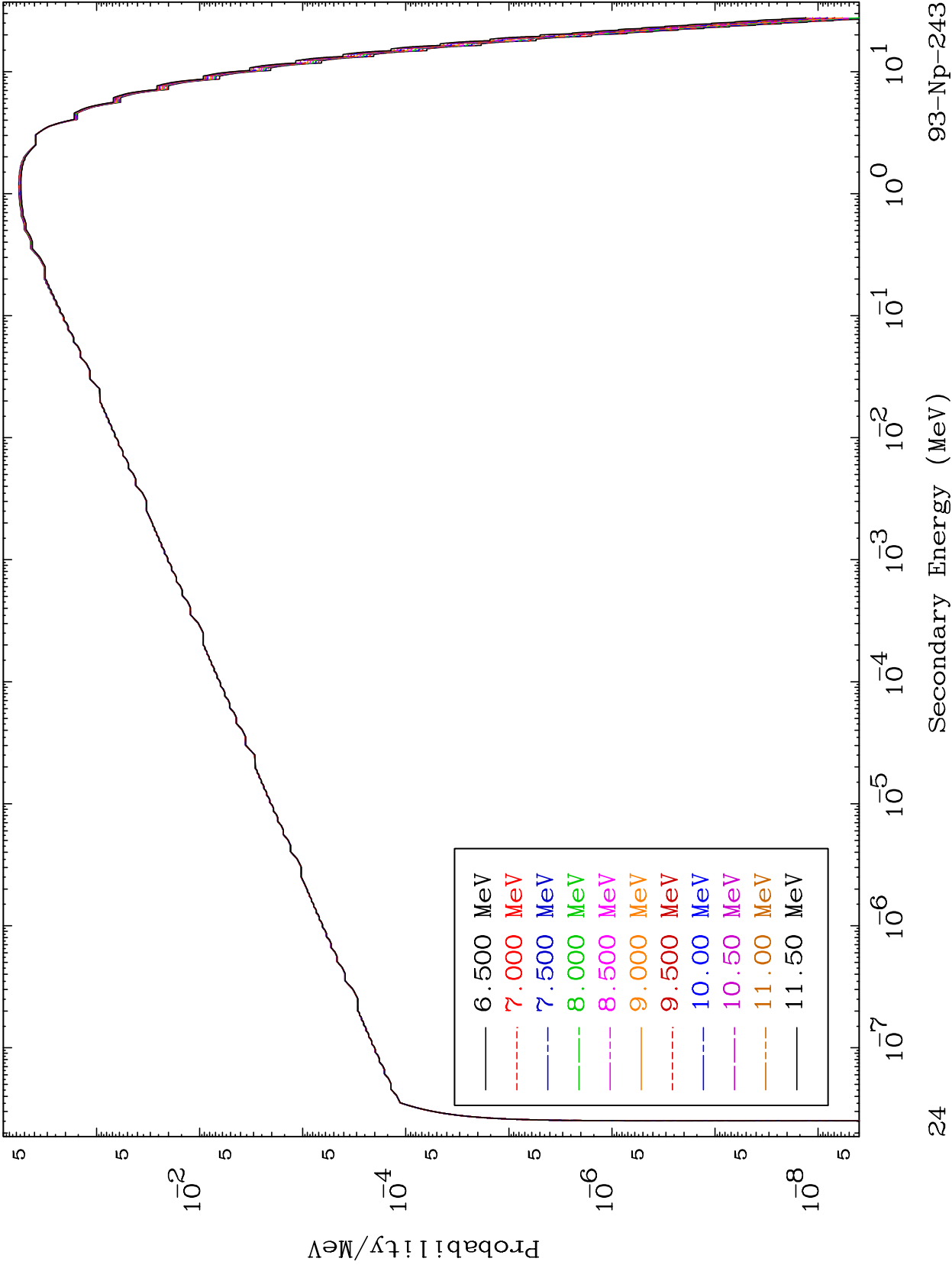
93-Np-243



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Fission
Energy Distribution

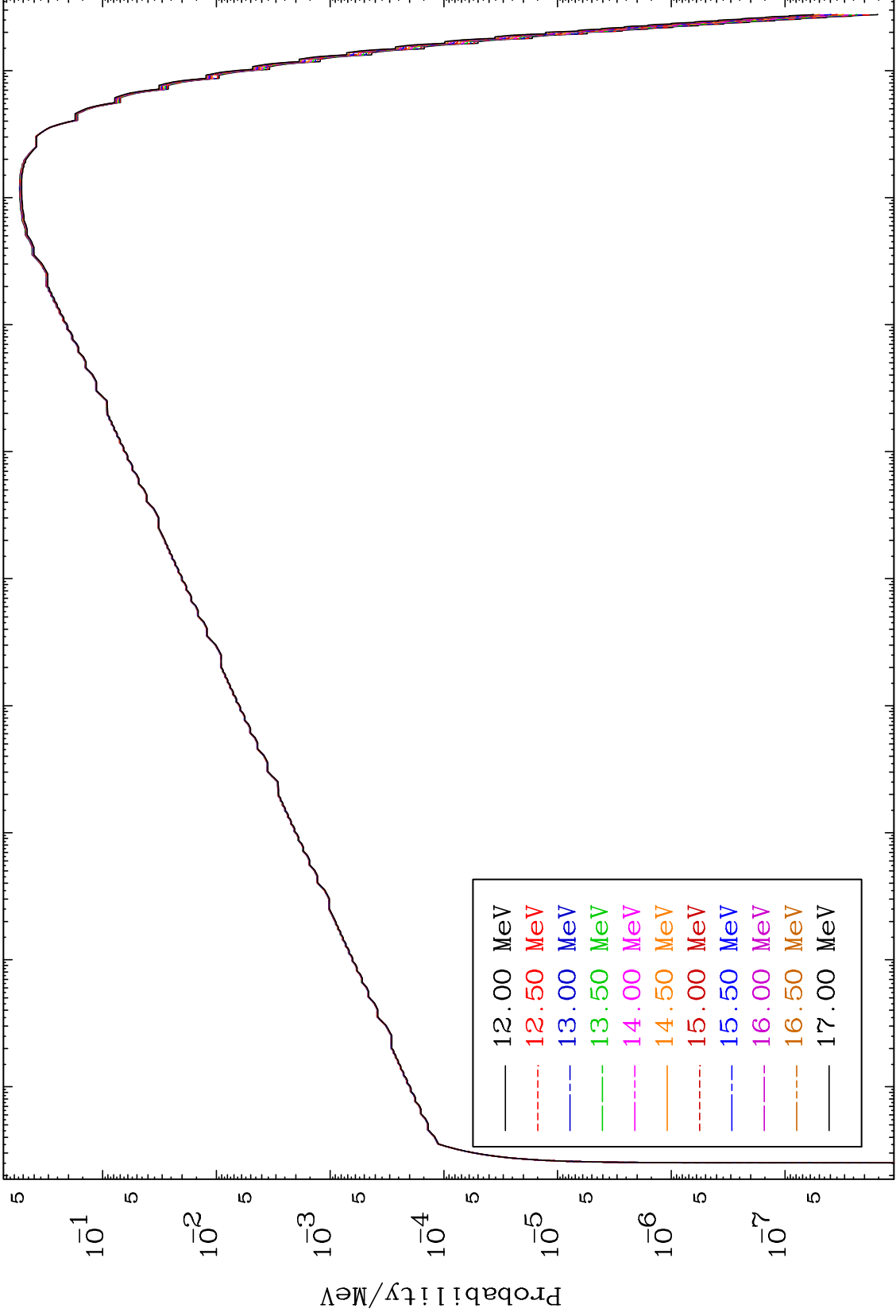
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MAT 9364

Fission Energy Distribution

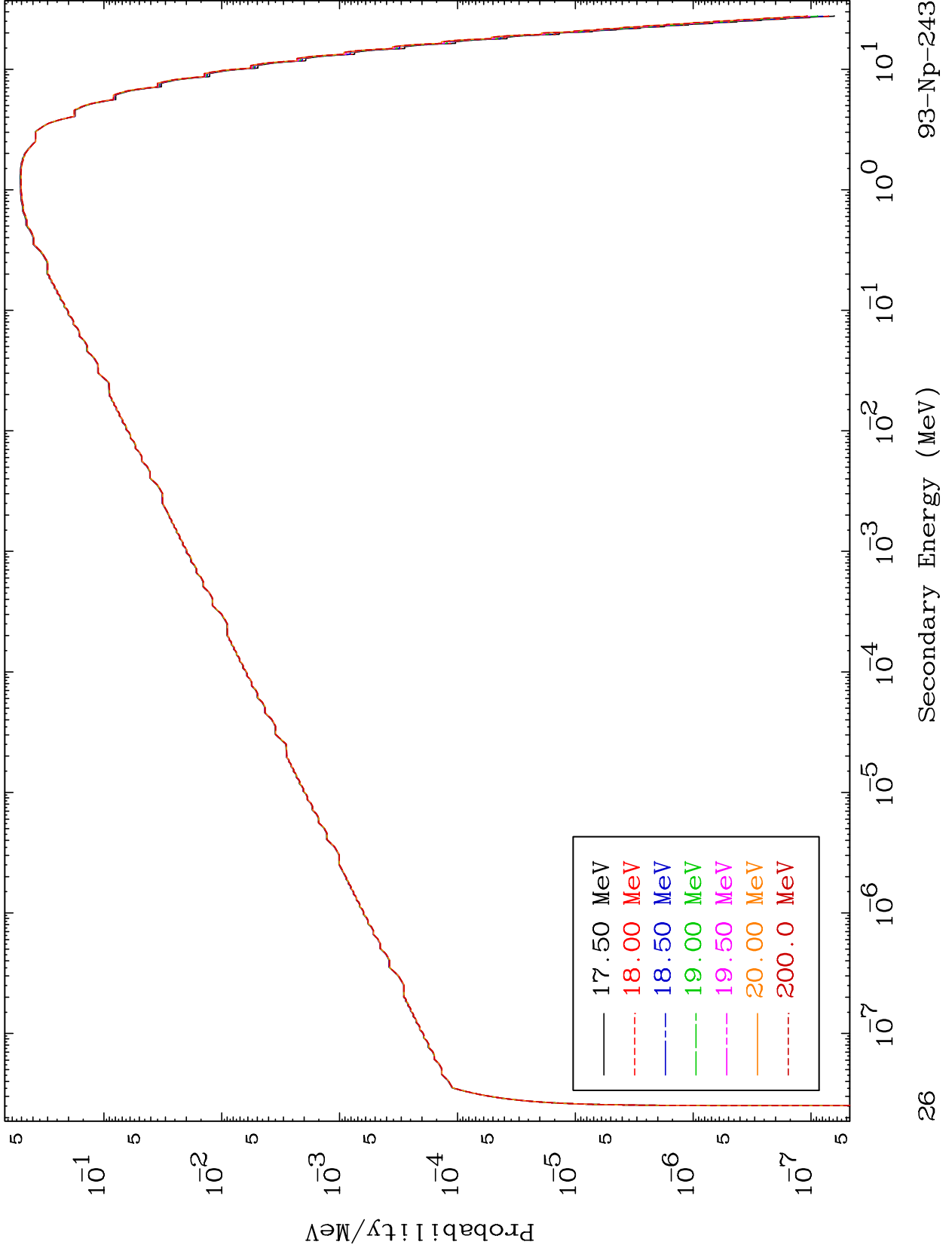
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MAT 9364

Fission Energy Distribution

93-Np-243

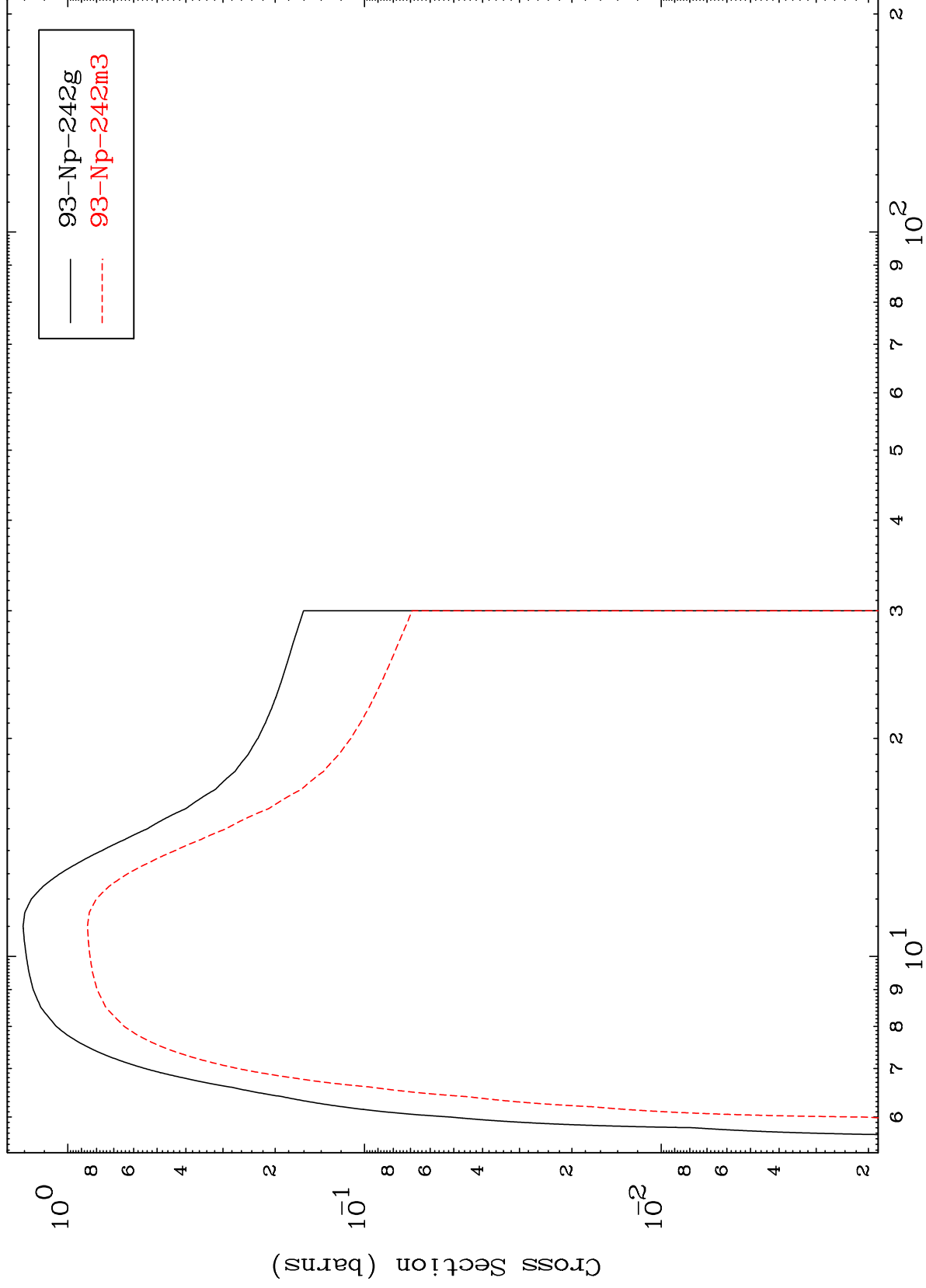


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(n,2n)

93-Np-243

Radionuclide Production Cross Section



27

Incident Energy (MeV)

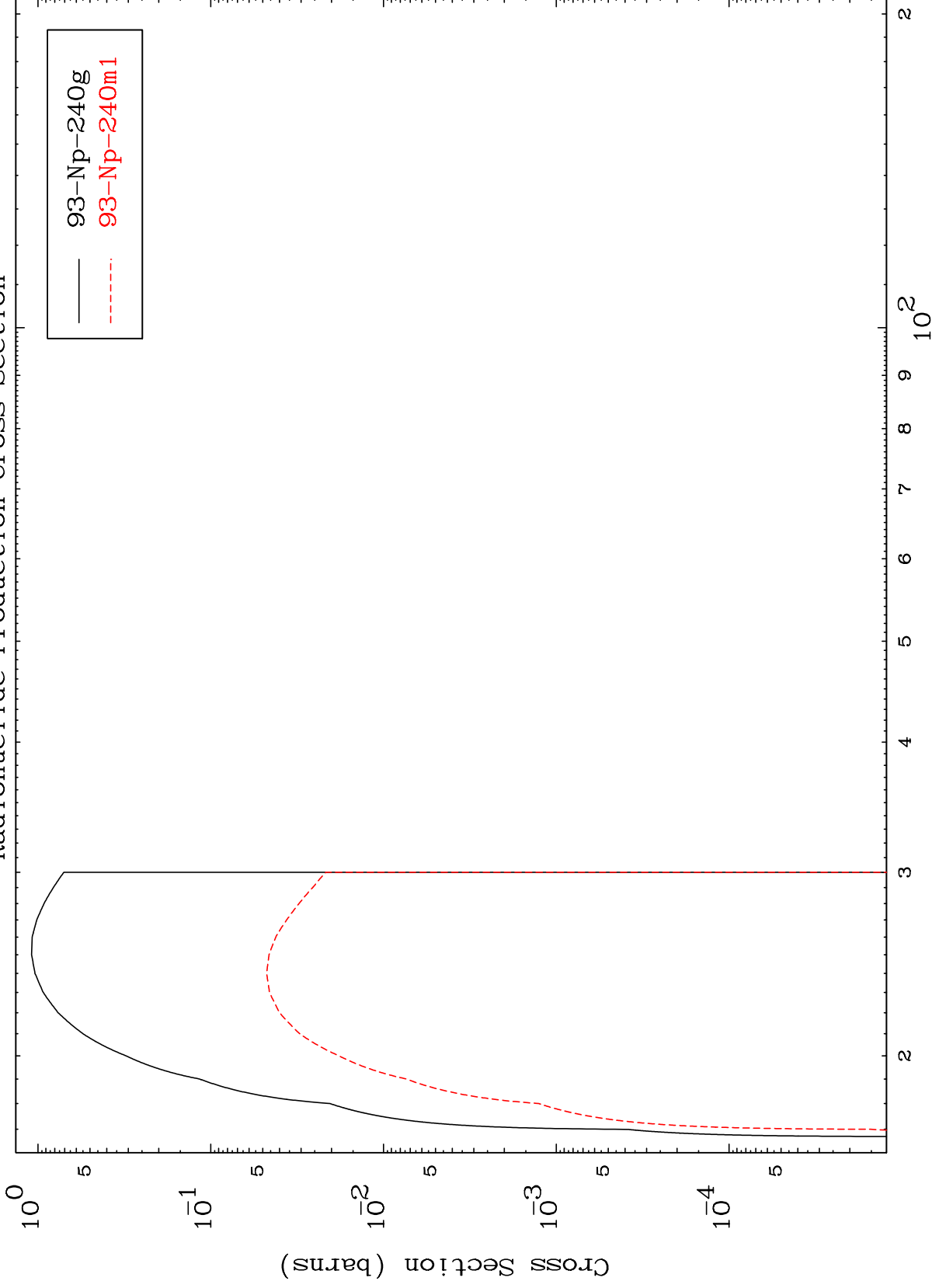
93-Np-243

MAT 9364

(n,4n)

93-Np-243

Radionuclide Production Cross Section



28

Incident Energy (MeV)

93-Np-243