

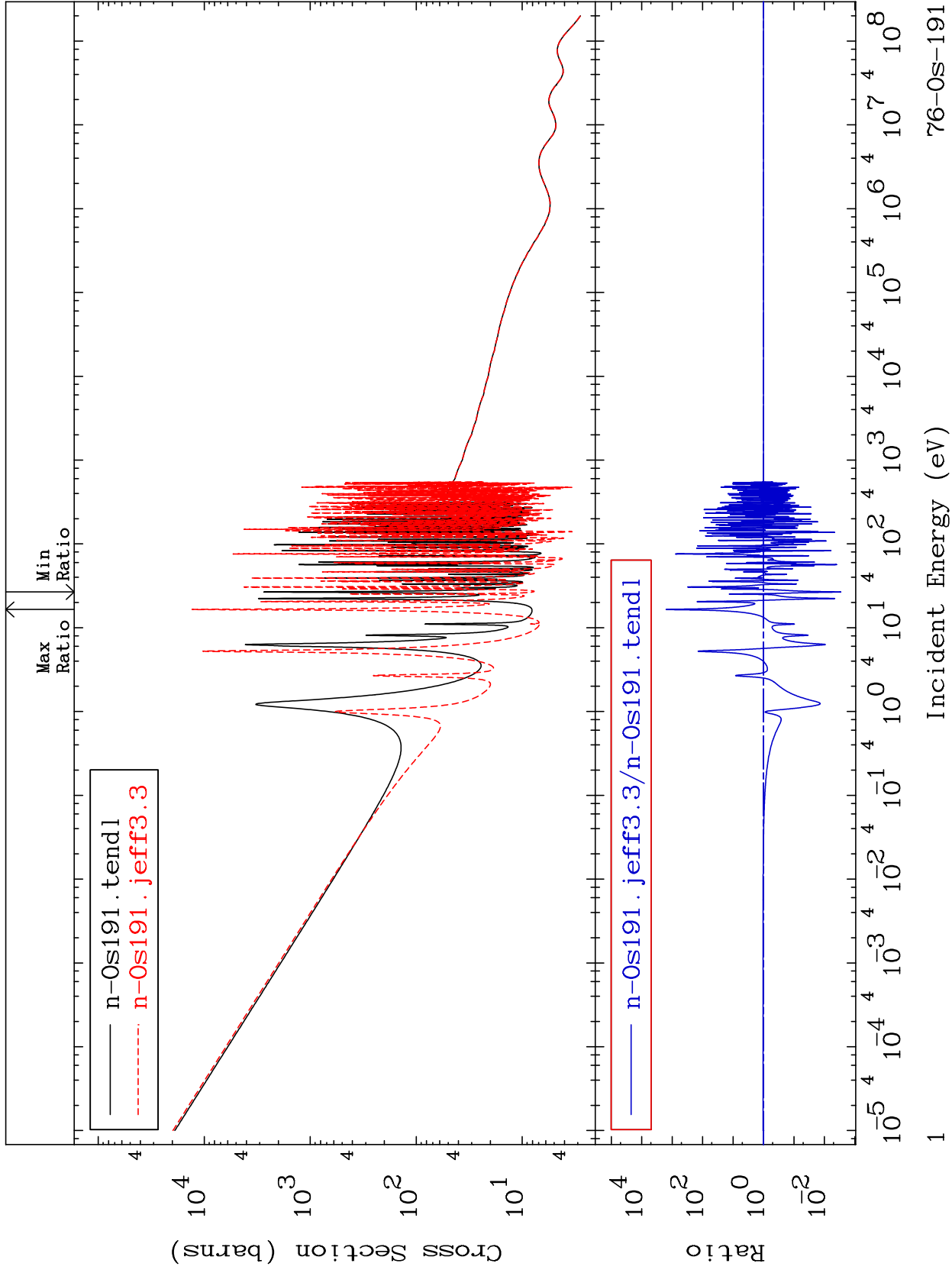
MAT 7646

Total

76-0s-191

Cross Section

-99.71 To 9999. %



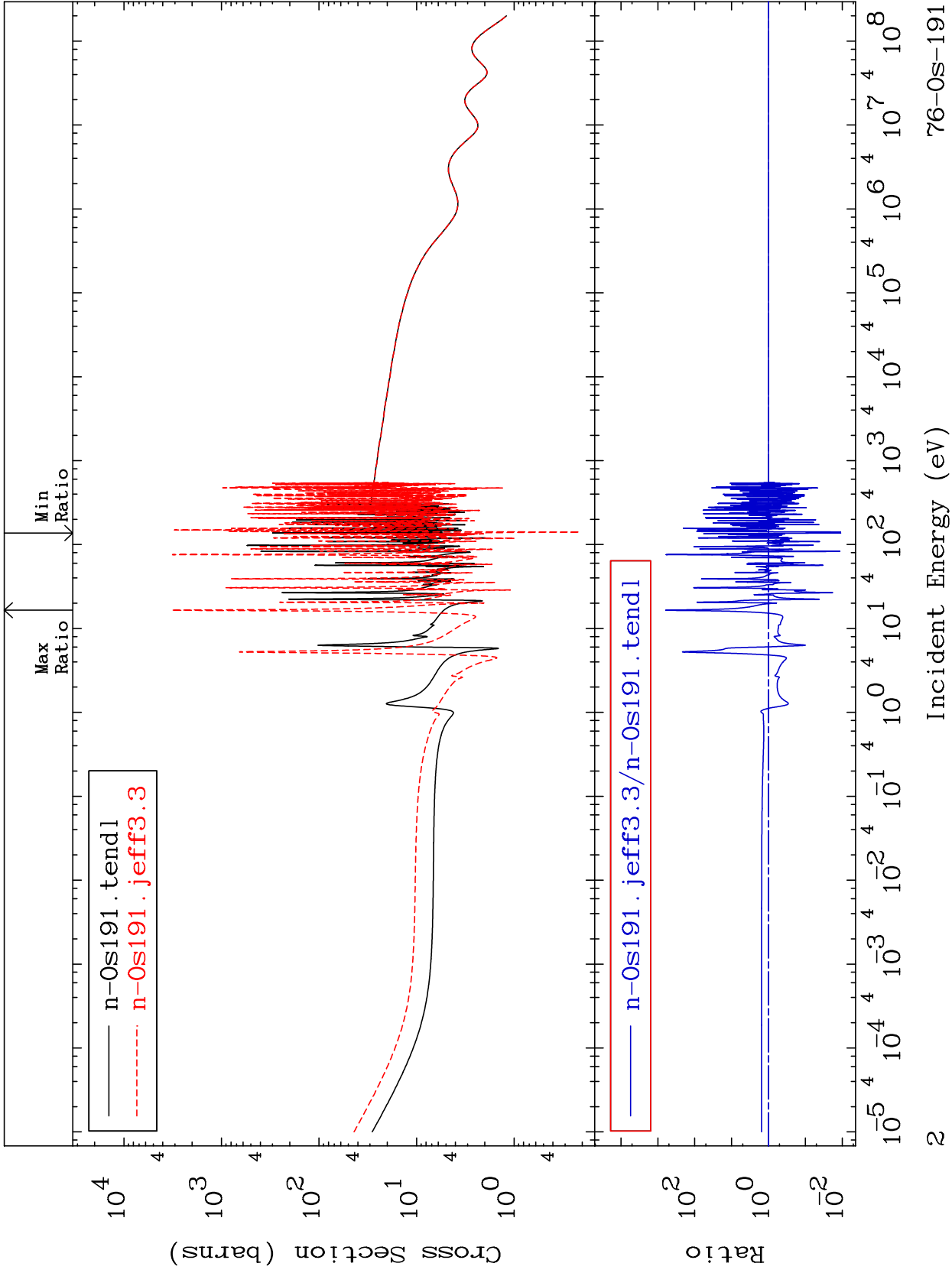
Incident Energy (eV)

76-0s-191

MAT 7646

Elastic Cross Section

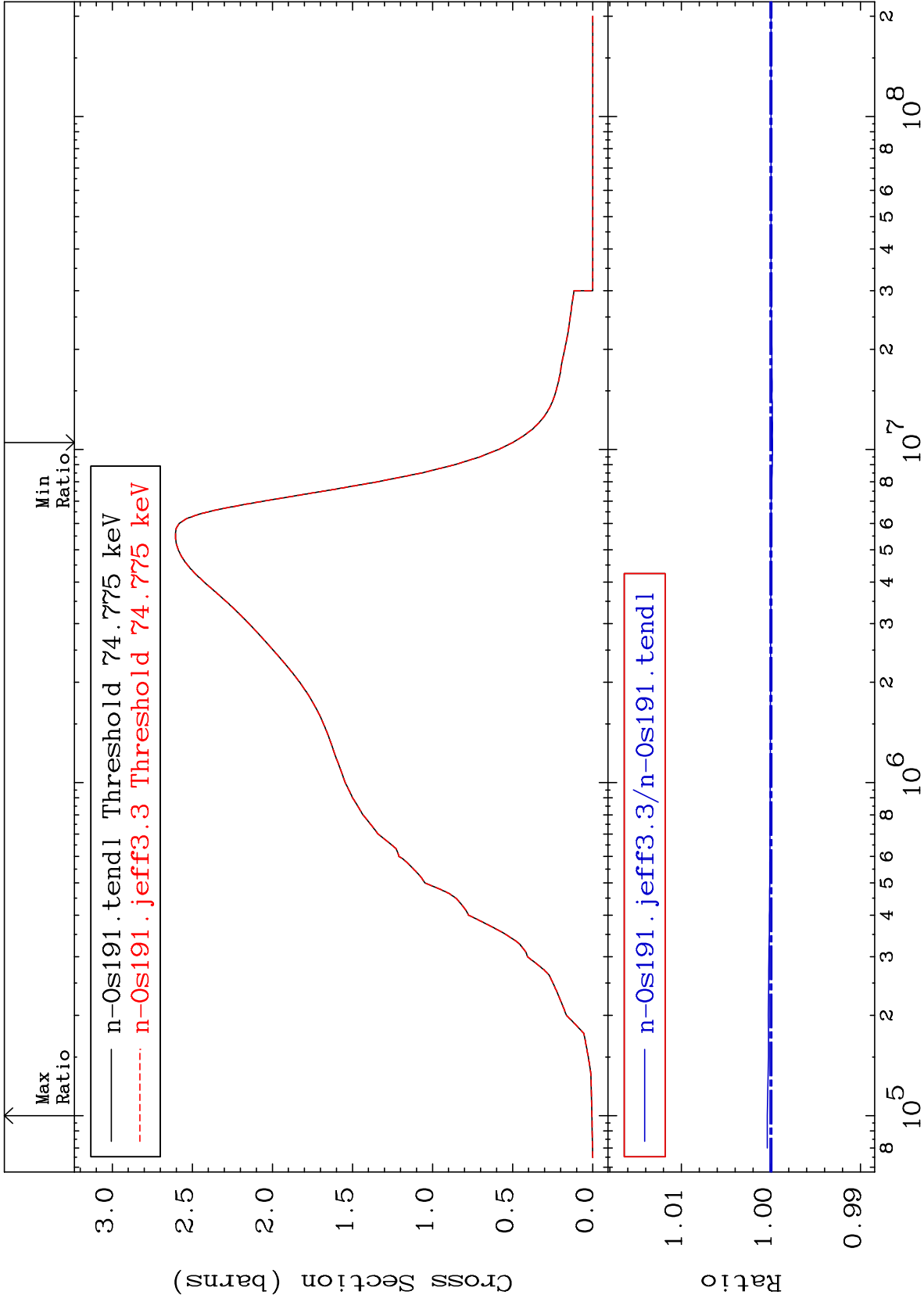
76-0s-191
-98.90 To 9999. %



MAT 7646

Inelastic
Cross Section

76-0s-191
-0.018 To 0.041 %



3

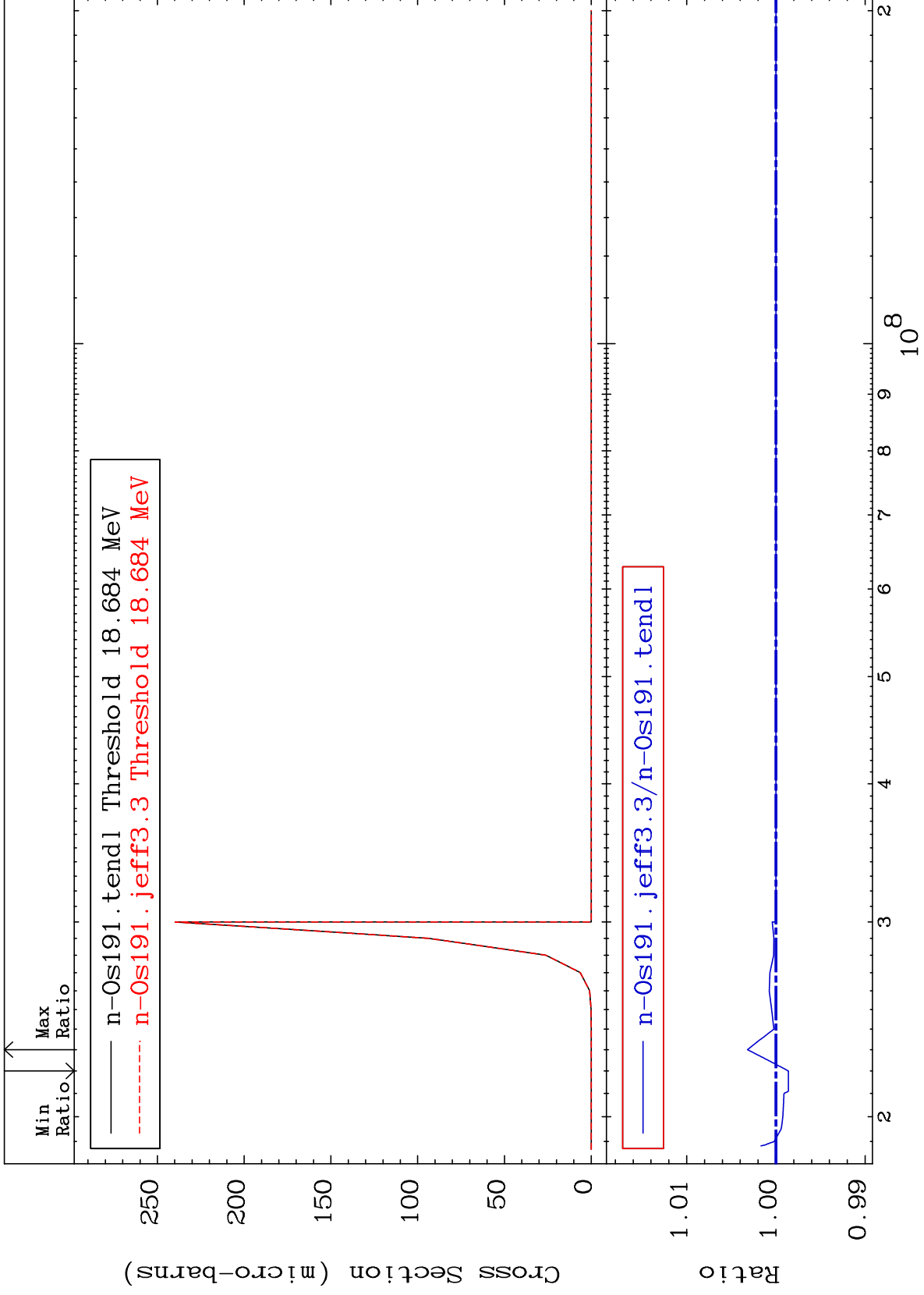
Incident Energy (eV)

76-0s-191

MAT 7646

(n,2n) d
Cross Section

76-0s-191
-0.141 To 0.320 %



4

Incident Energy (eV)

76-0s-191

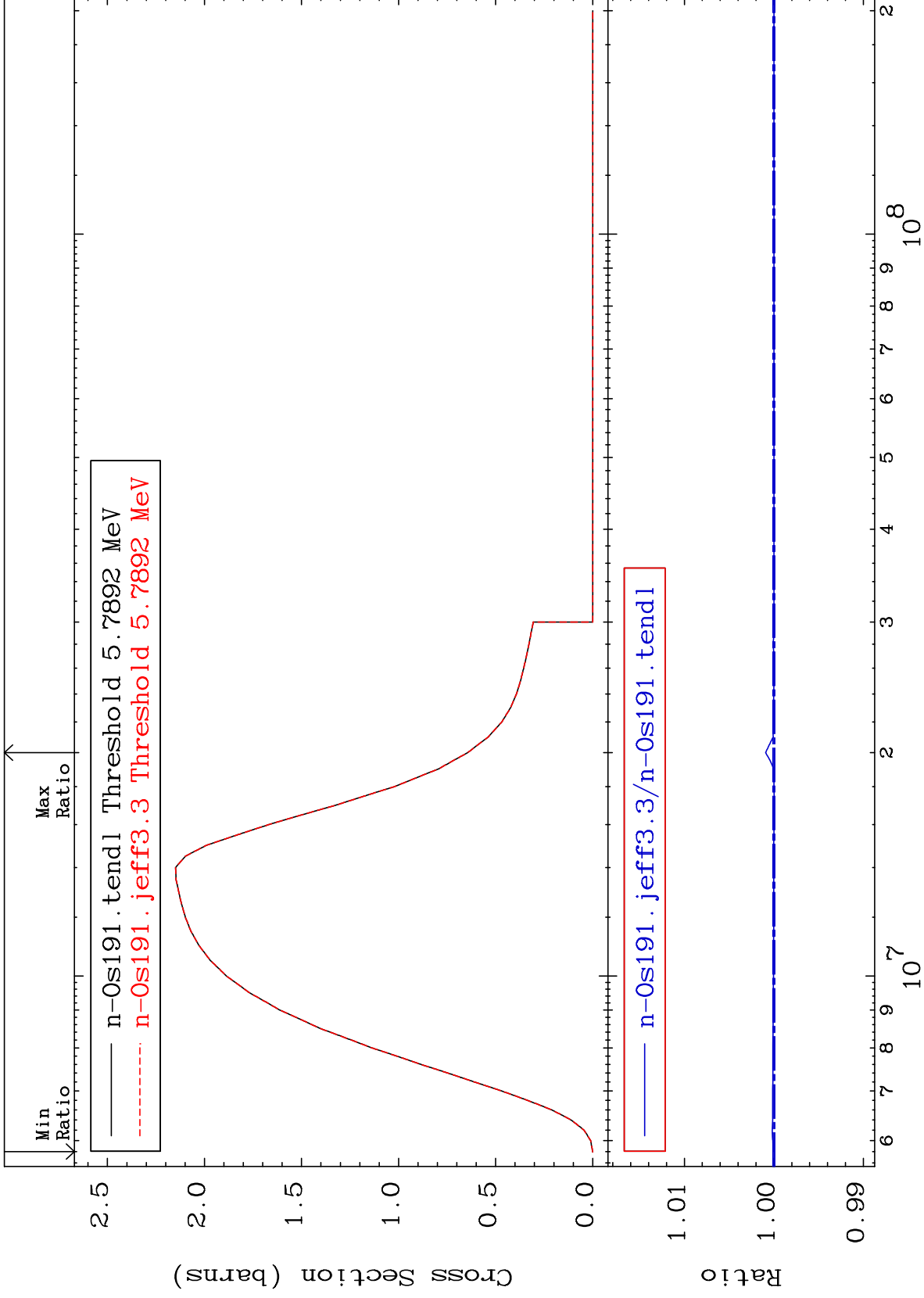
MAT 7646

(n,2n)

76-0s-191

Cross Section

-0.004 To 0.093 %



5

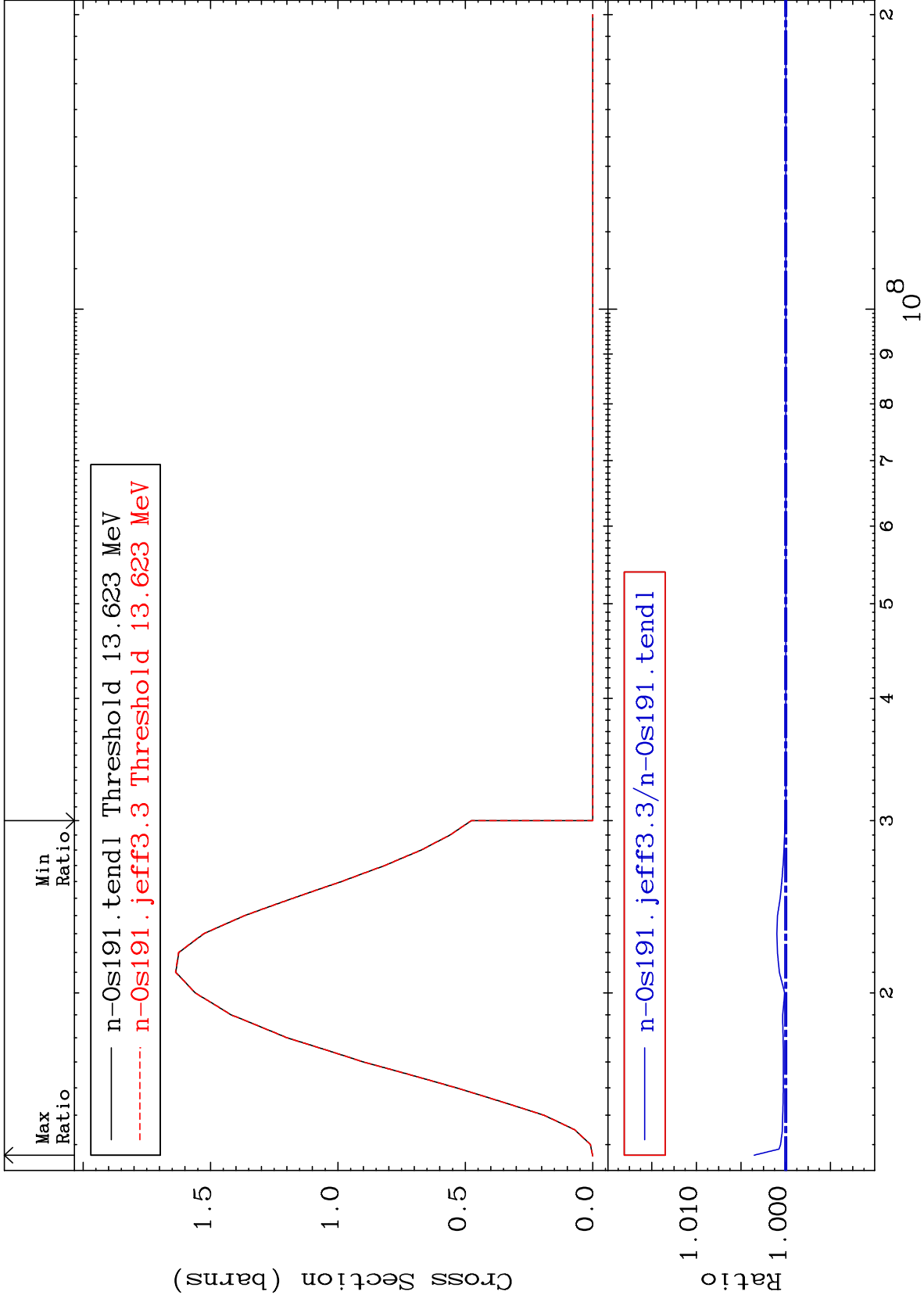
Incident Energy (eV)

76-0s-191

Cross Section

0.000

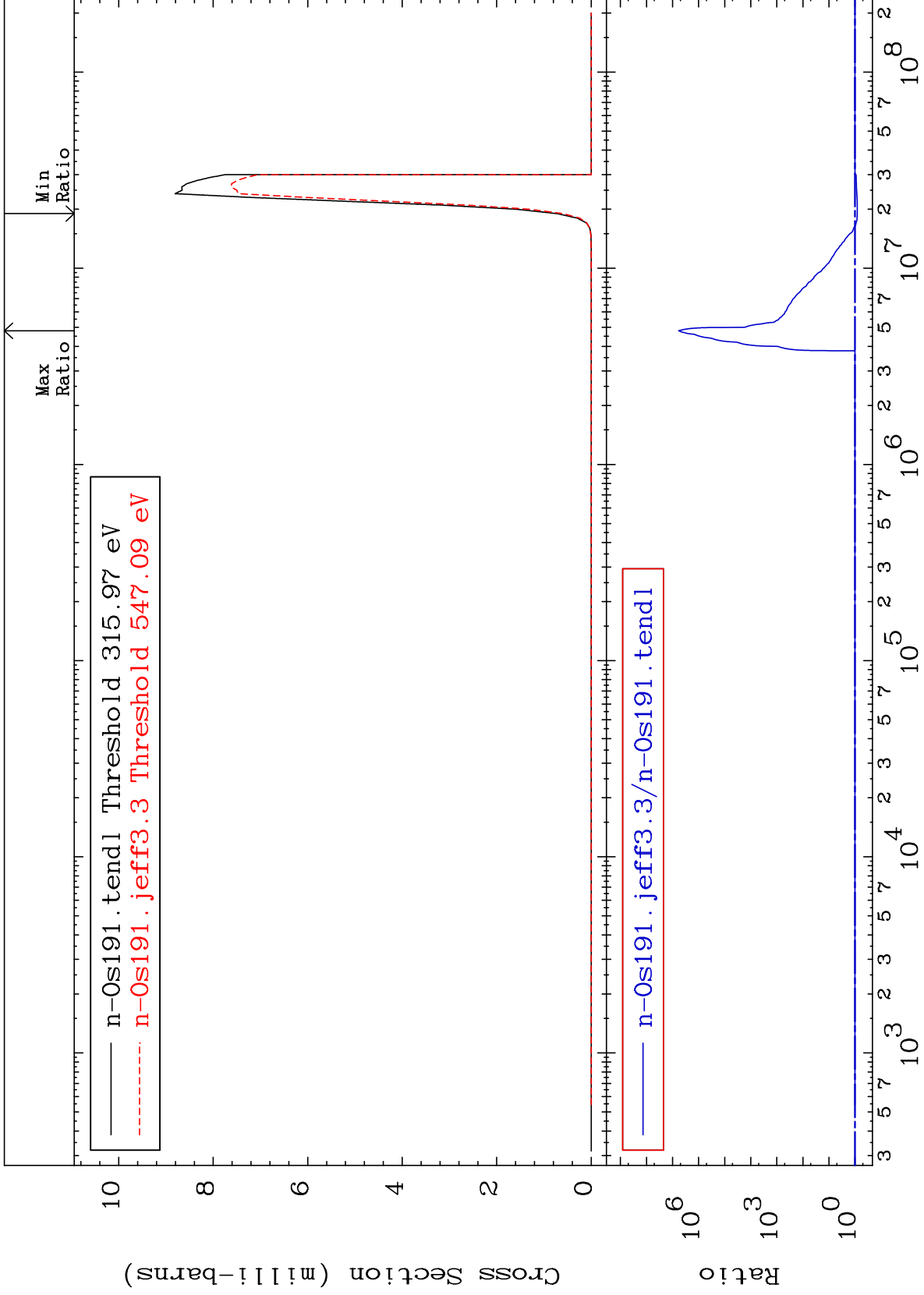
To 0.355 %



MAT 7646

(n, n') α
Cross Section

76-0s-191
-17.96 To 9999. %



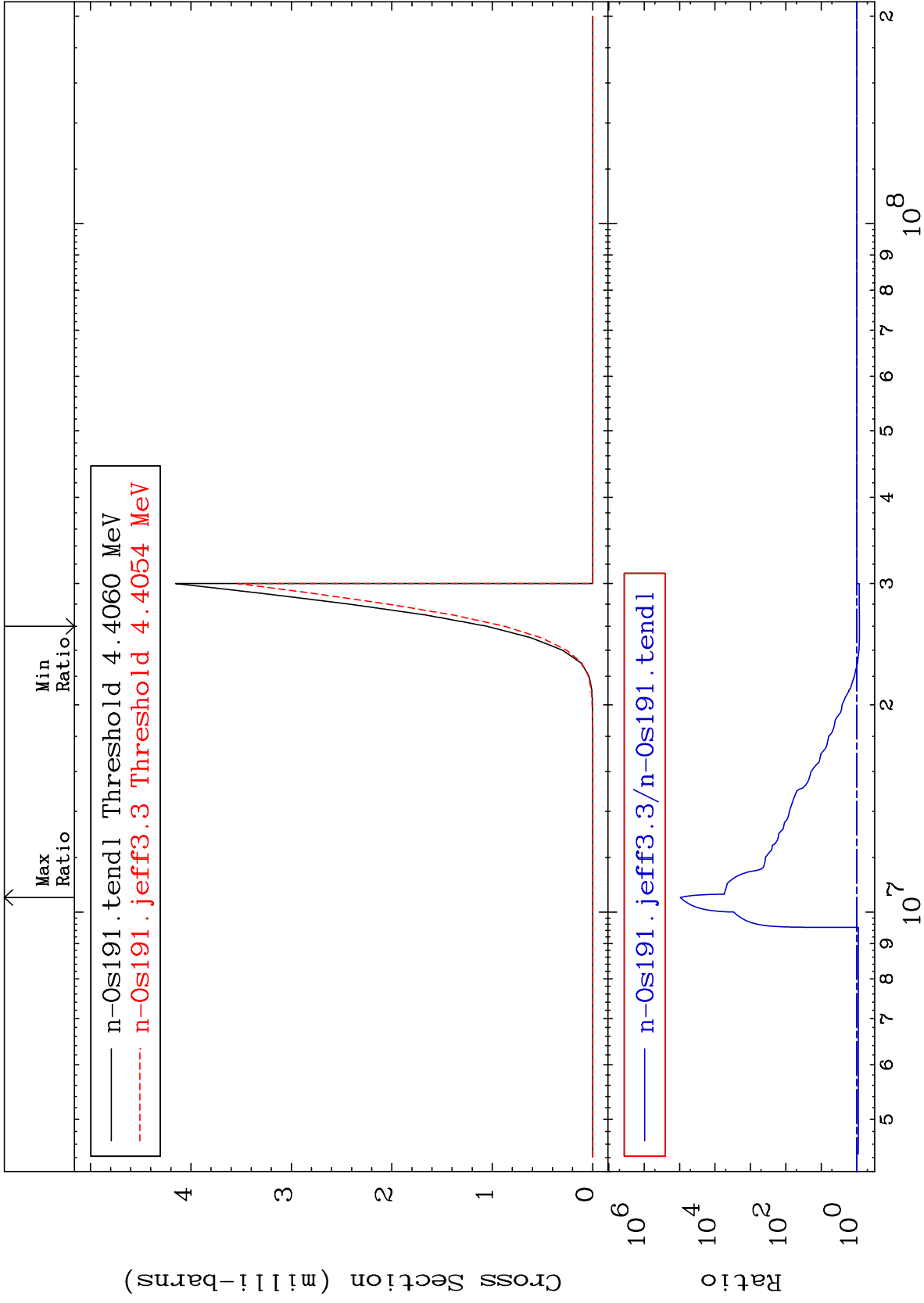
MAT 7646

(n,2n) α

76-0s-191

Cross Section

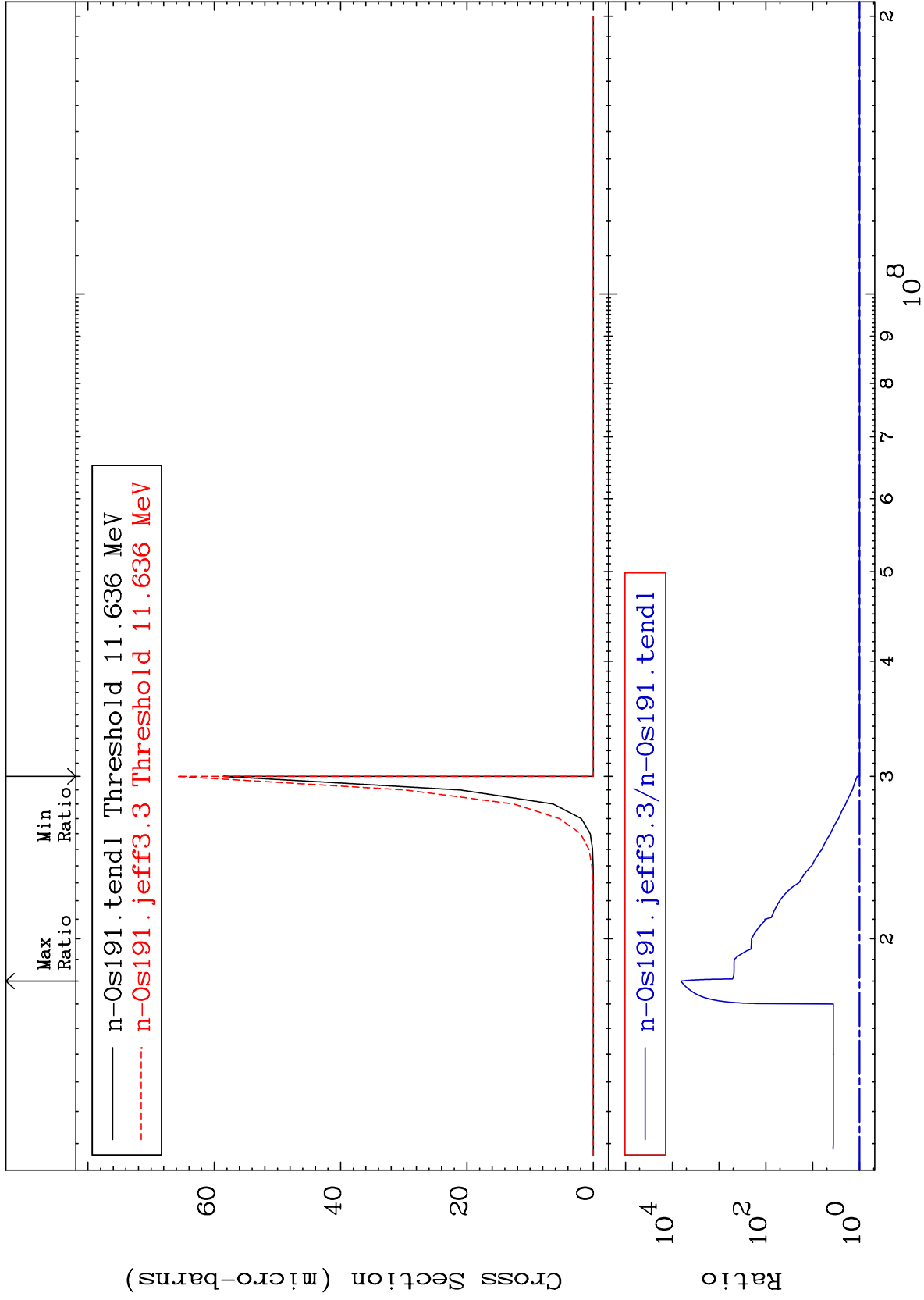
-16.66 To 9999. %



MAT 7646

(n,3n) α
Cross Section

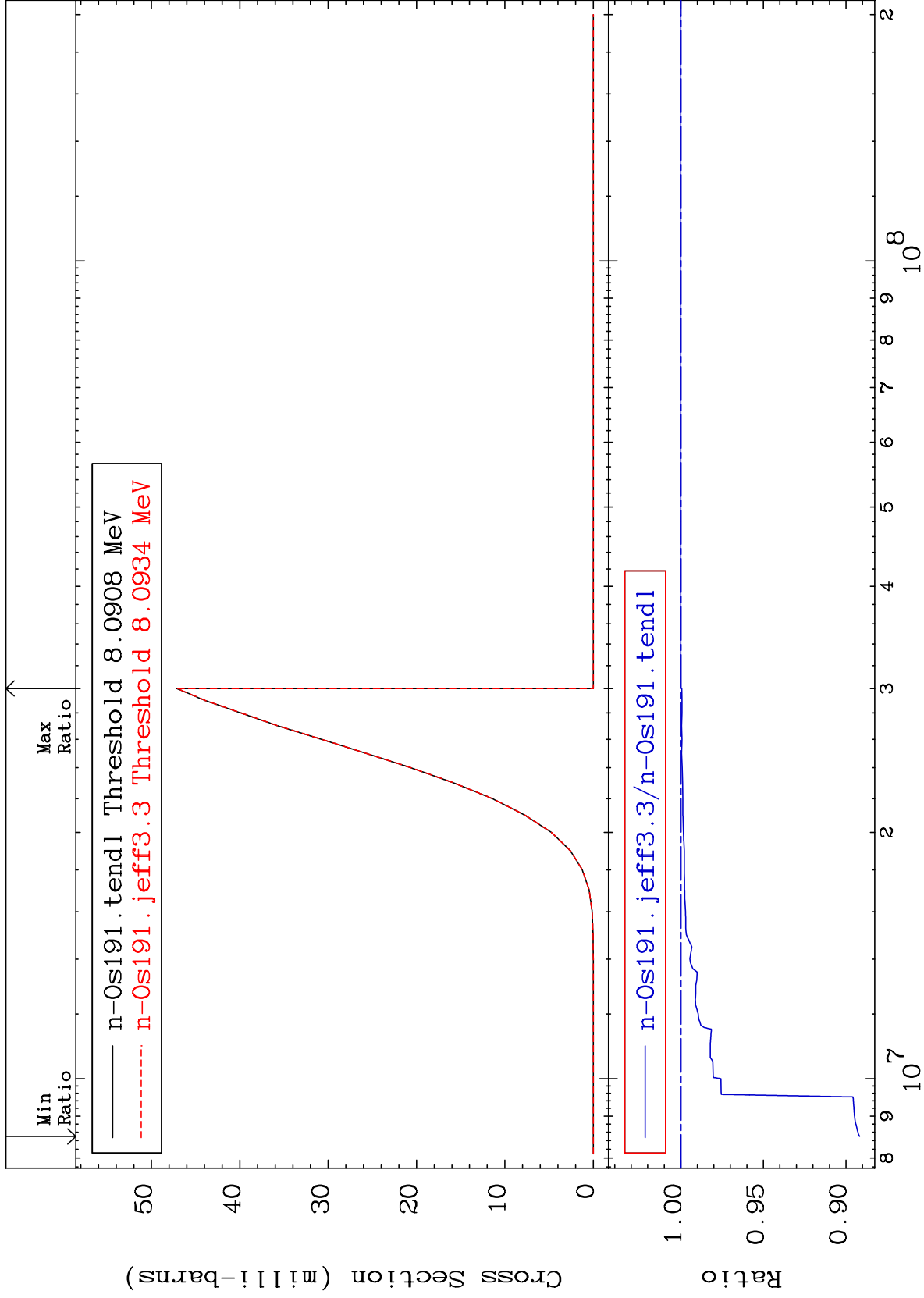
76-0s-191
To 9999. %
0.000



MAT 7646

(n,n') p
Cross Section

76-0s-191
-10.82 To 0.000 %



76-0s-191

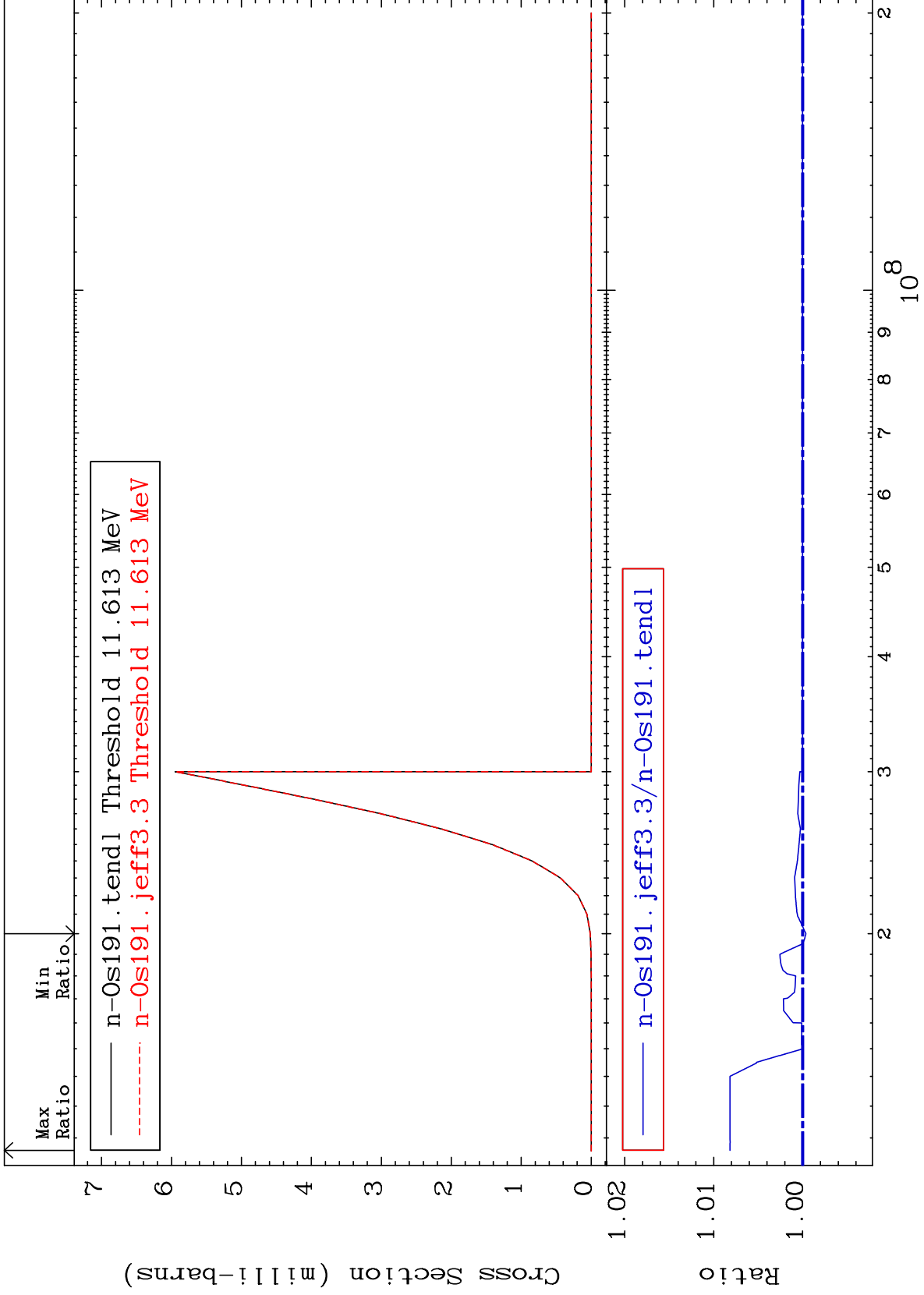
Incident Energy (eV)

10

MAT 7646

(n,n') d
Cross Section

76-0s-191
-0.036 To 0.816 %



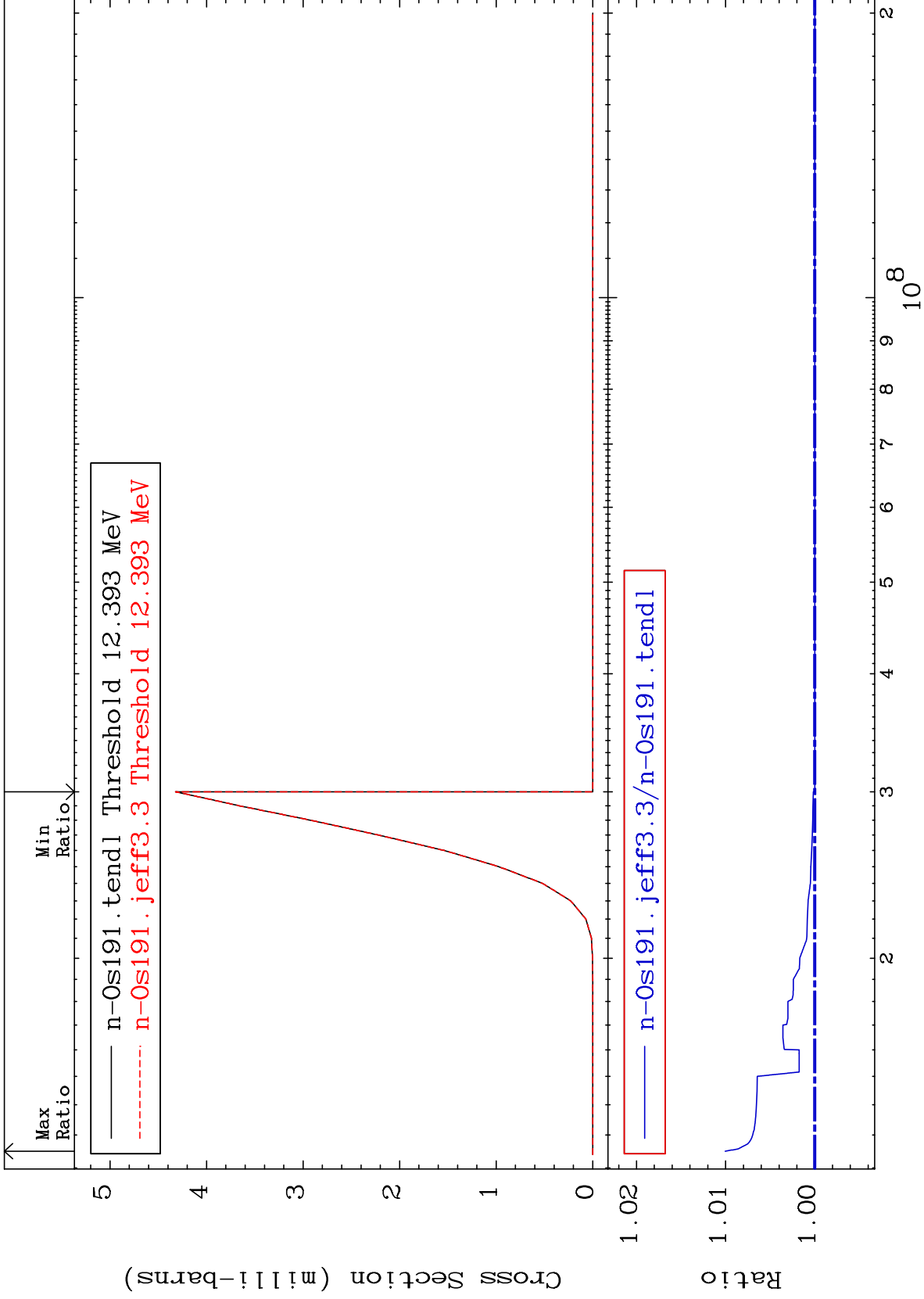
MAT 7646

(n,n') t

76-0s-191

Cross Section

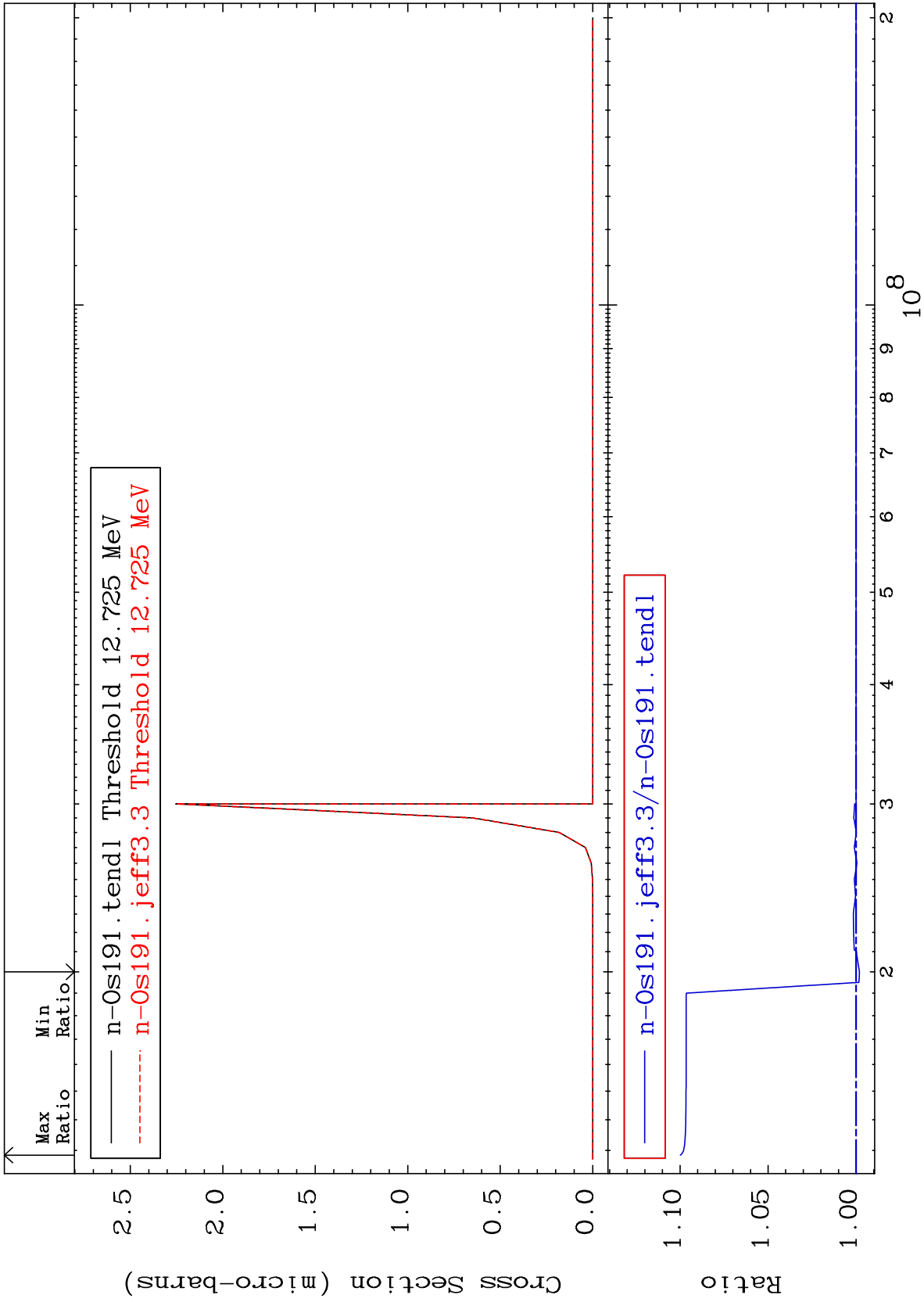
0.000 To 1.006 %



12

Incident Energy (eV)

76-0s-191



MAT 7646

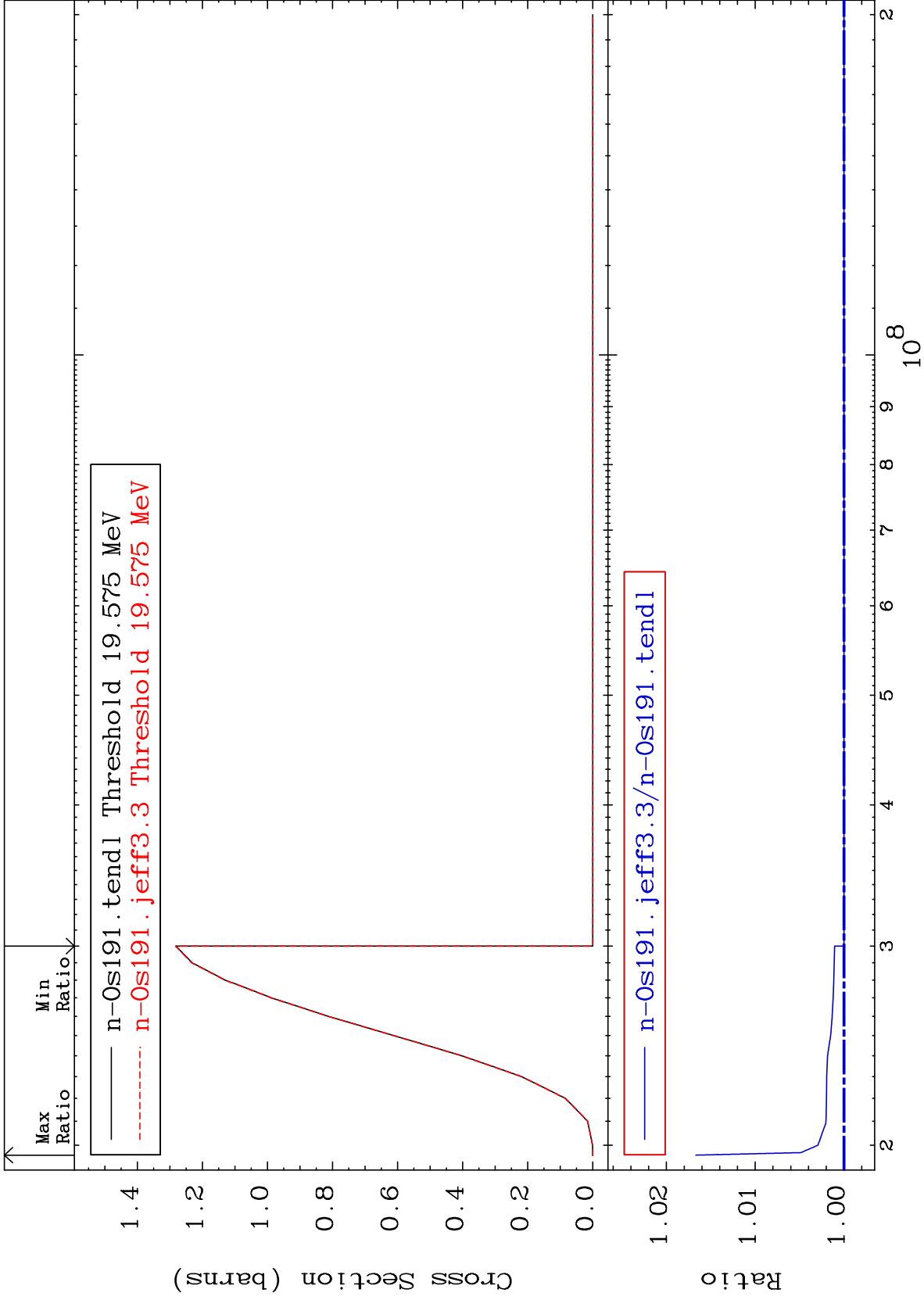
(n,4n)

76-0s-191

Cross Section

0.000

To 1.669 %



14

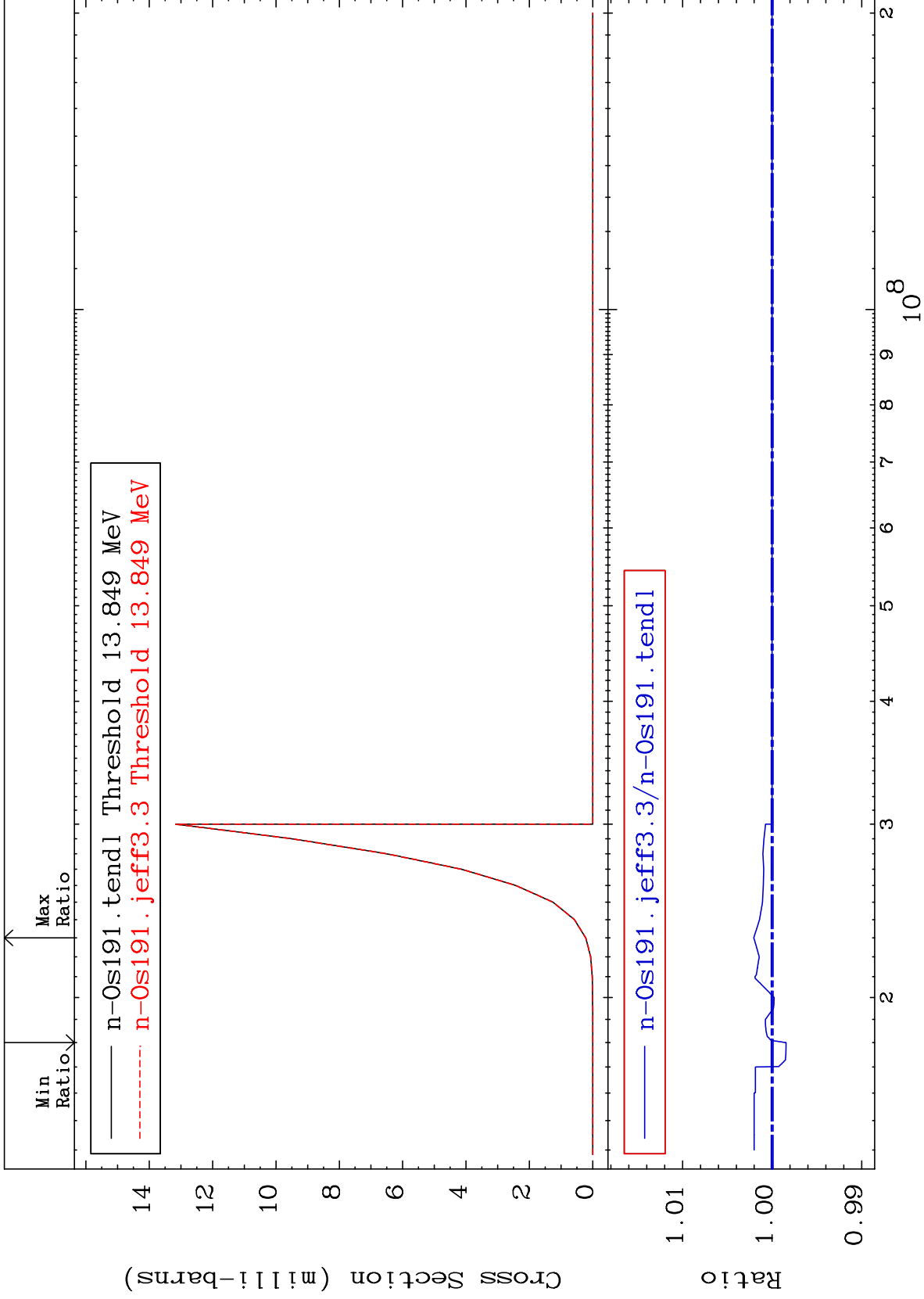
Incident Energy (eV)

76-0s-191

MAT 7646

(n,2n) p
Cross Section

76-0s-191
-0.155 To 0.204 %



15

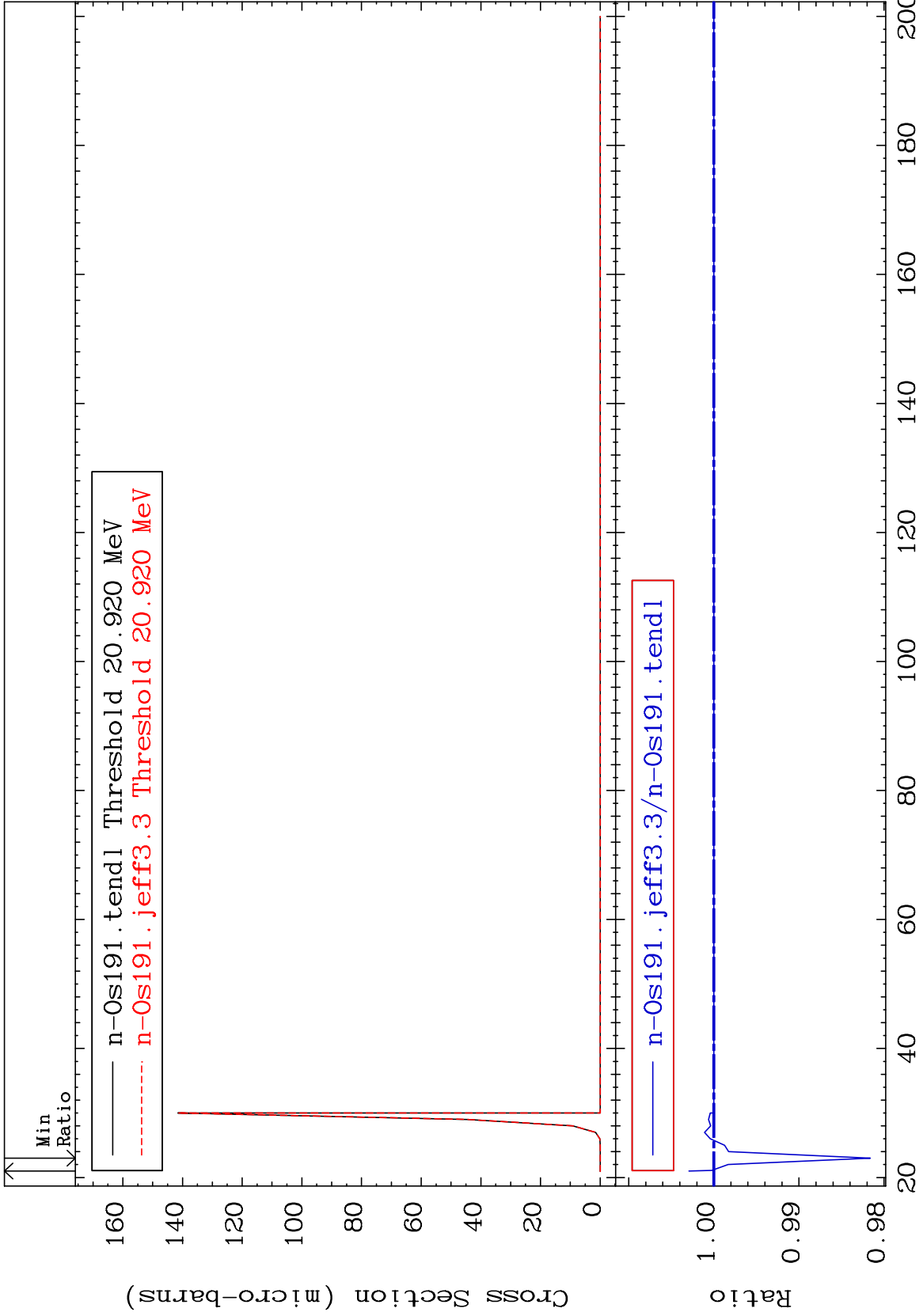
Incident Energy (eV)

76-0s-191

MAT 7646

(n,3n) p
Cross Section

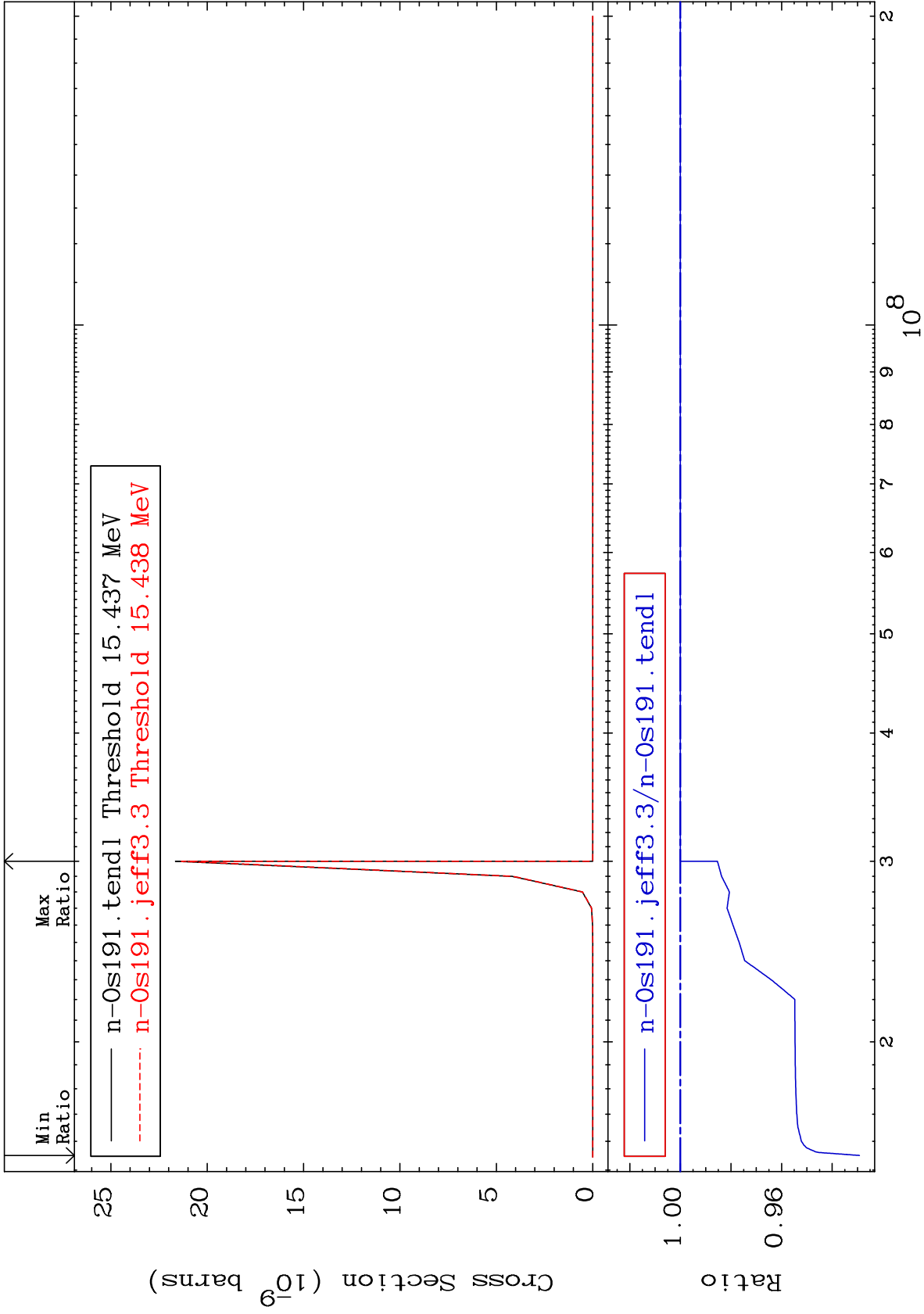
76-0s-191
-1.838 To 0.293 %



16

Incident Energy (MeV)

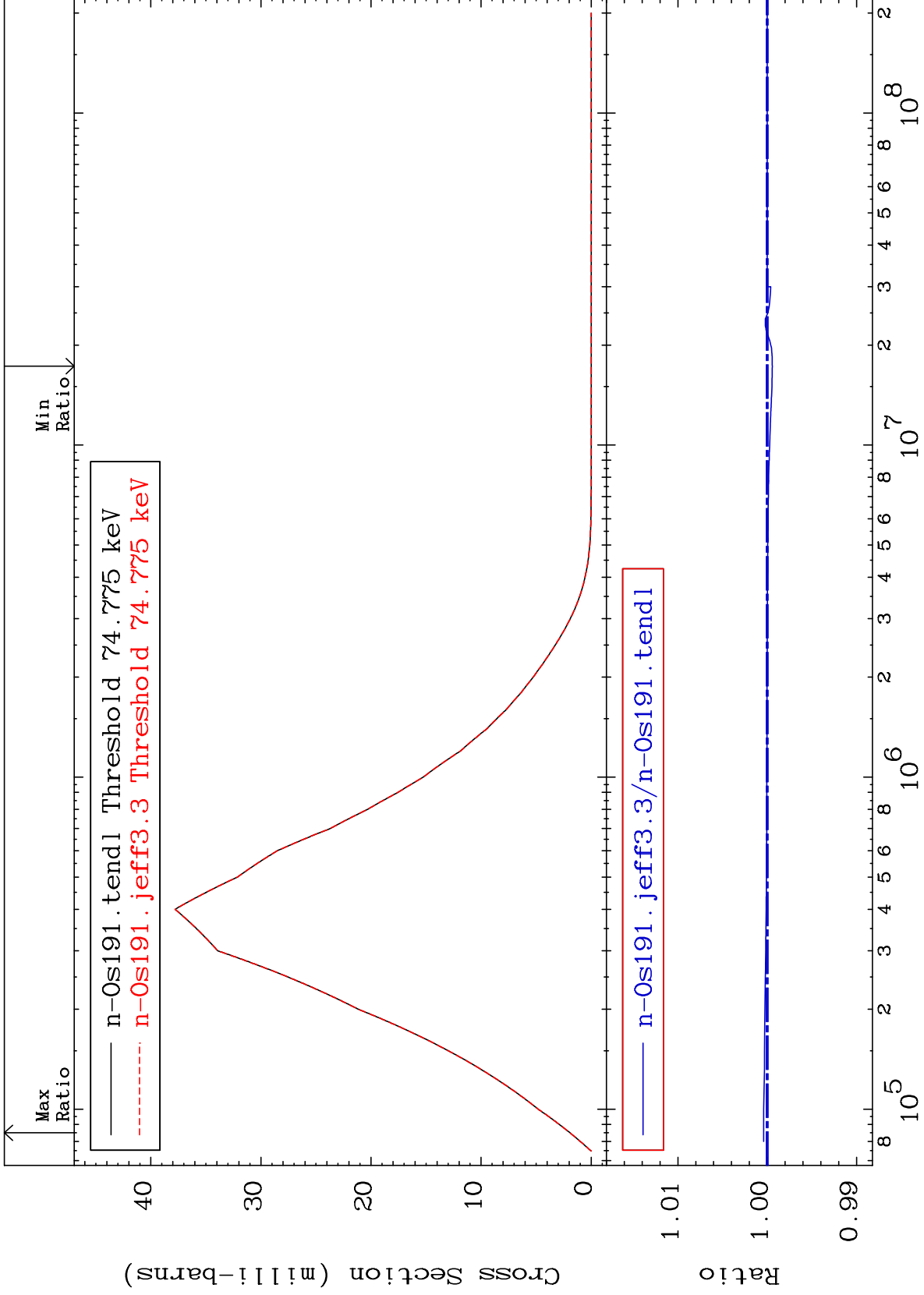
76-0s-191



MAT 7646

MT= 51 (n,n') Level
Cross Section

76-0s-191
-0.057 To 0.041 %



18

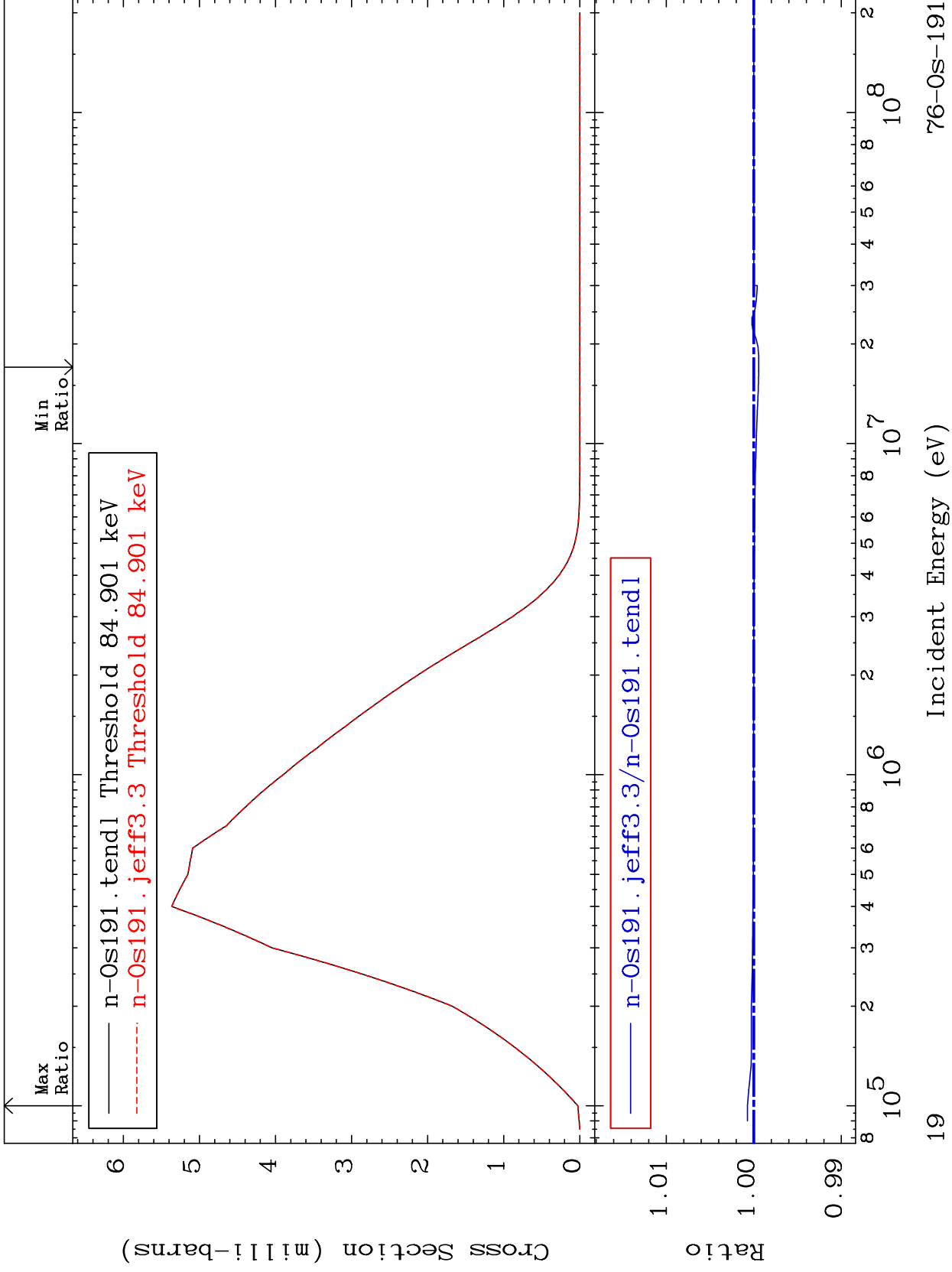
Incident Energy (eV)

76-0s-191

MAT 7646

MT= 52 (n,n') Level
Cross Section

76-0s-191
-0.057 To 0.073 %



19

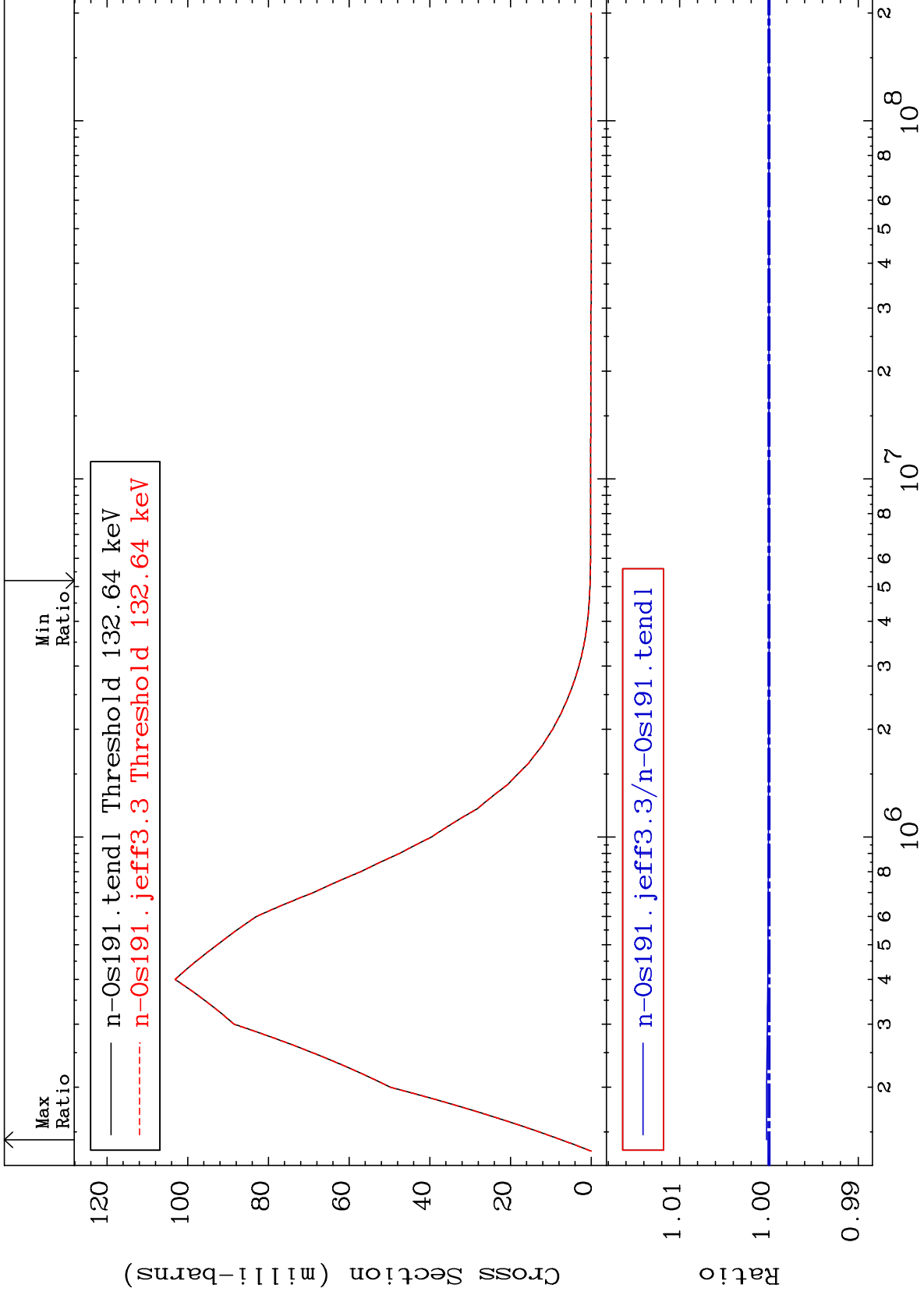
Incident Energy (eV)

76-0s-191

MAT 7646

MT= 53 (n,n') Level
Cross Section

76-0s-191
-0.004 To 0.026 %



20

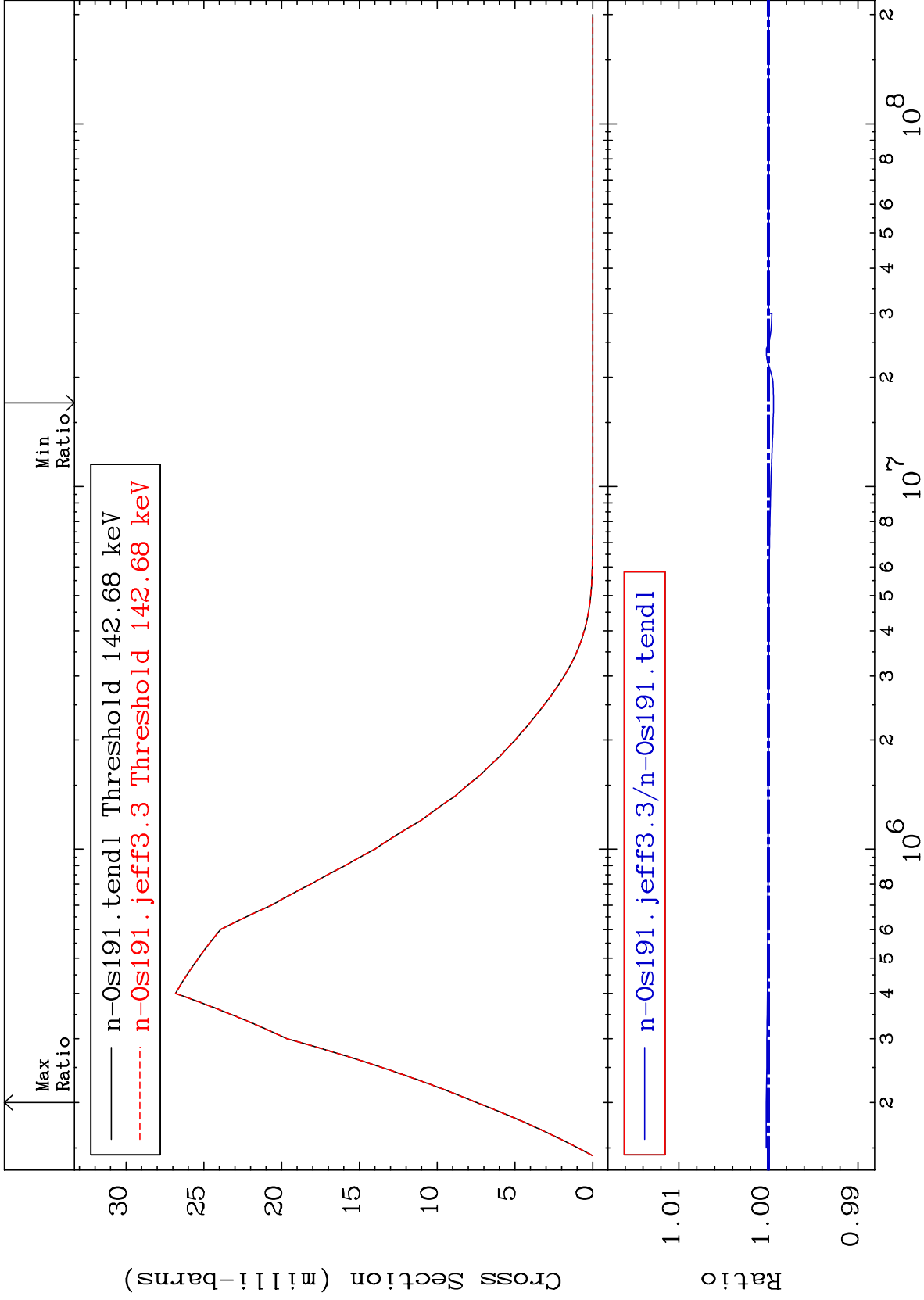
Incident Energy (eV)

76-0s-191

MAT 7646

MT= 54 (n,n') Level
Cross Section

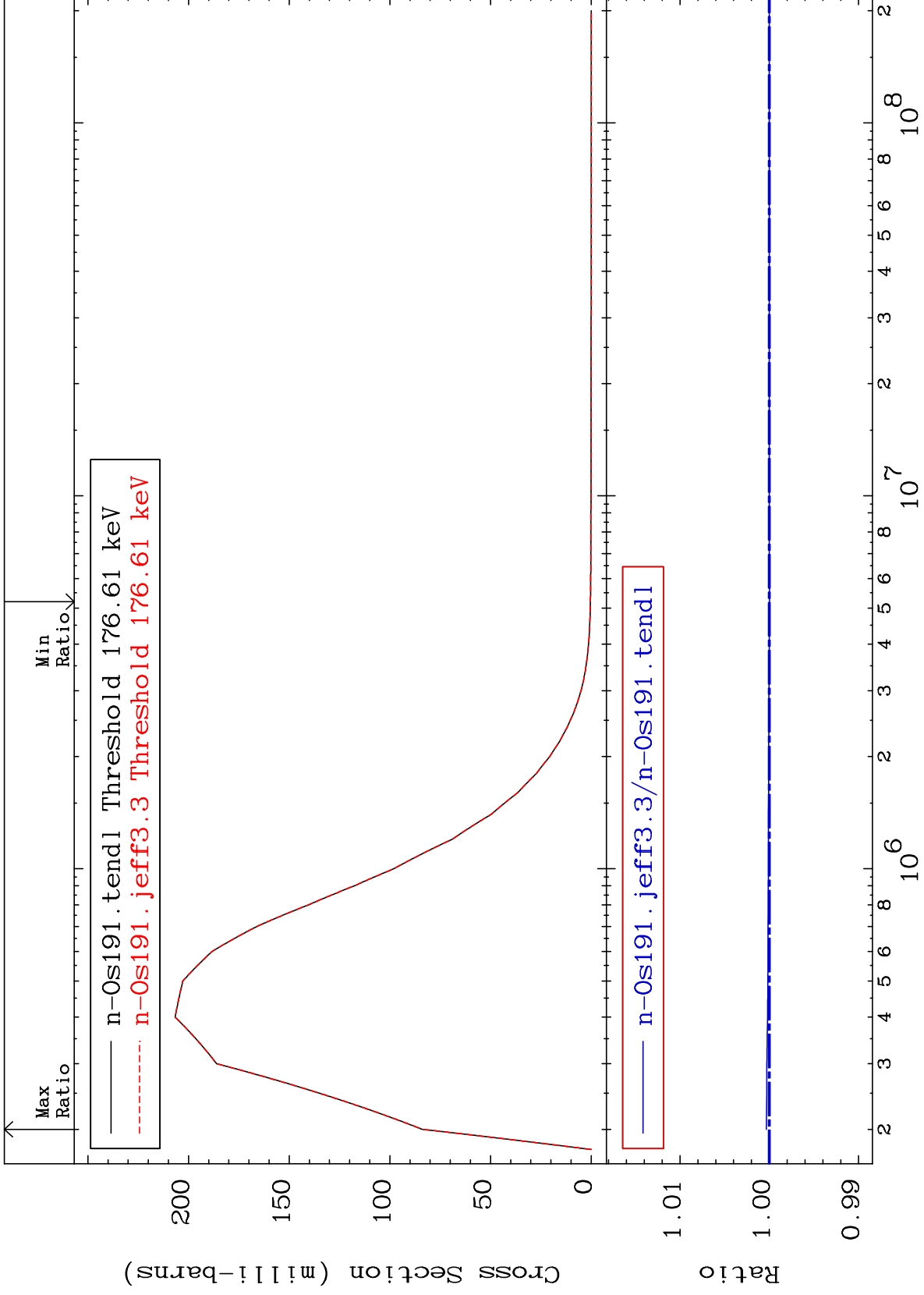
76-0s-191
-0.057 To 0.024 %



MAT 7646

MT= 55 (n,n') Level
Cross Section

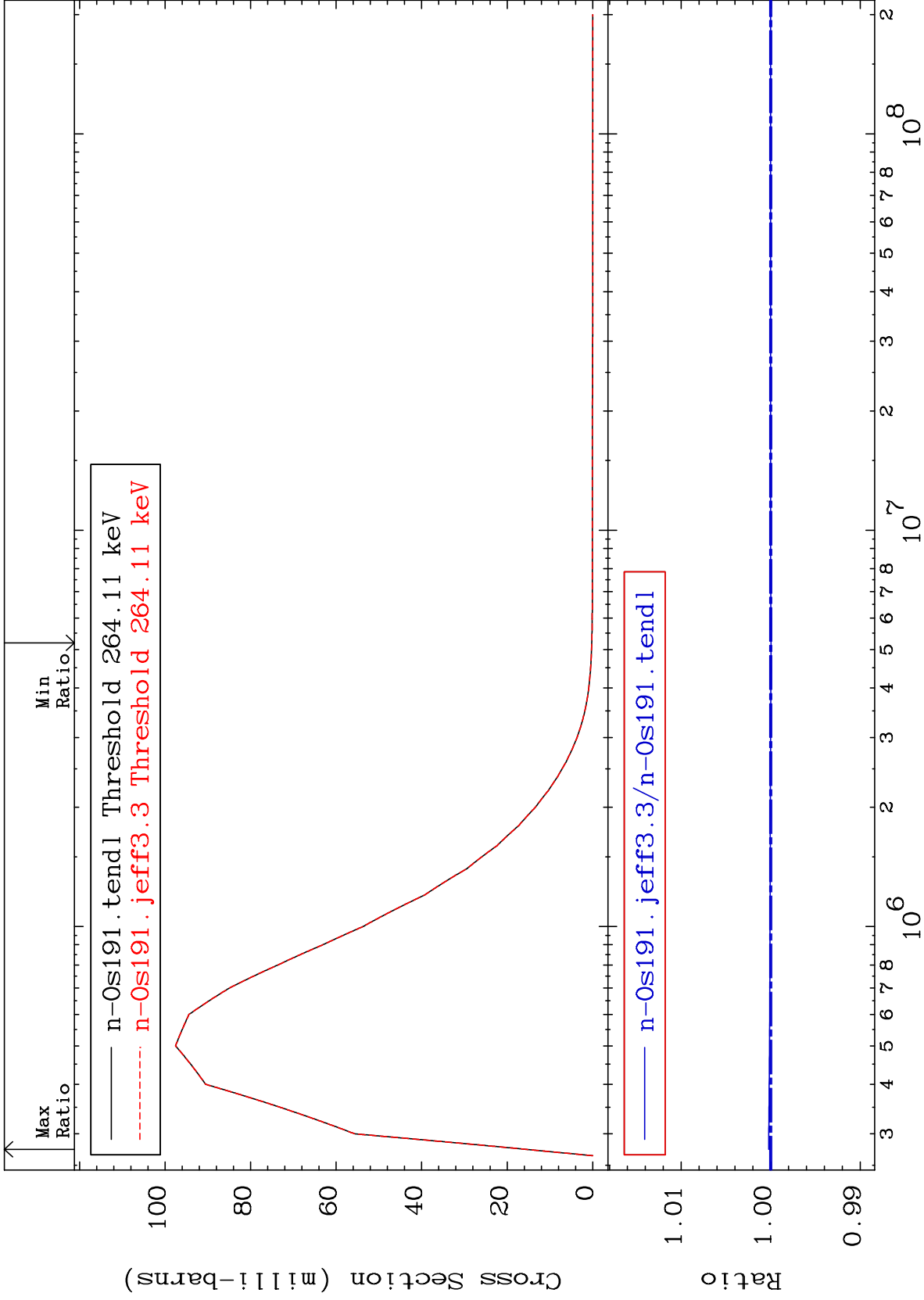
76-0s-191
-0.002 To 0.032 %



MAT 7646

MT= 56 (n,n') Level
Cross Section

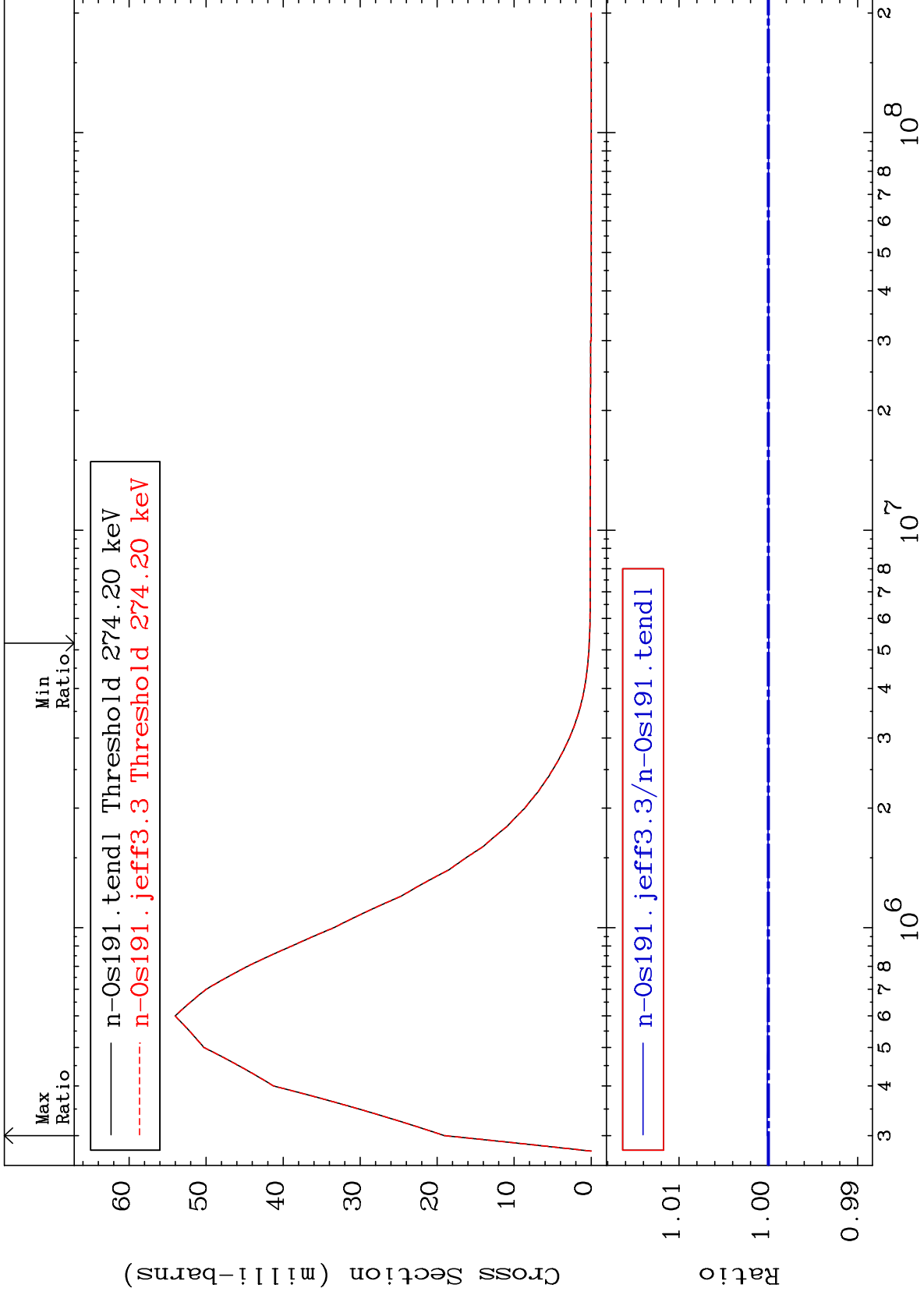
76-0s-191
-0.004 To 0.022 %



MAT 7646

MT= 57 (n,n') Level
Cross Section

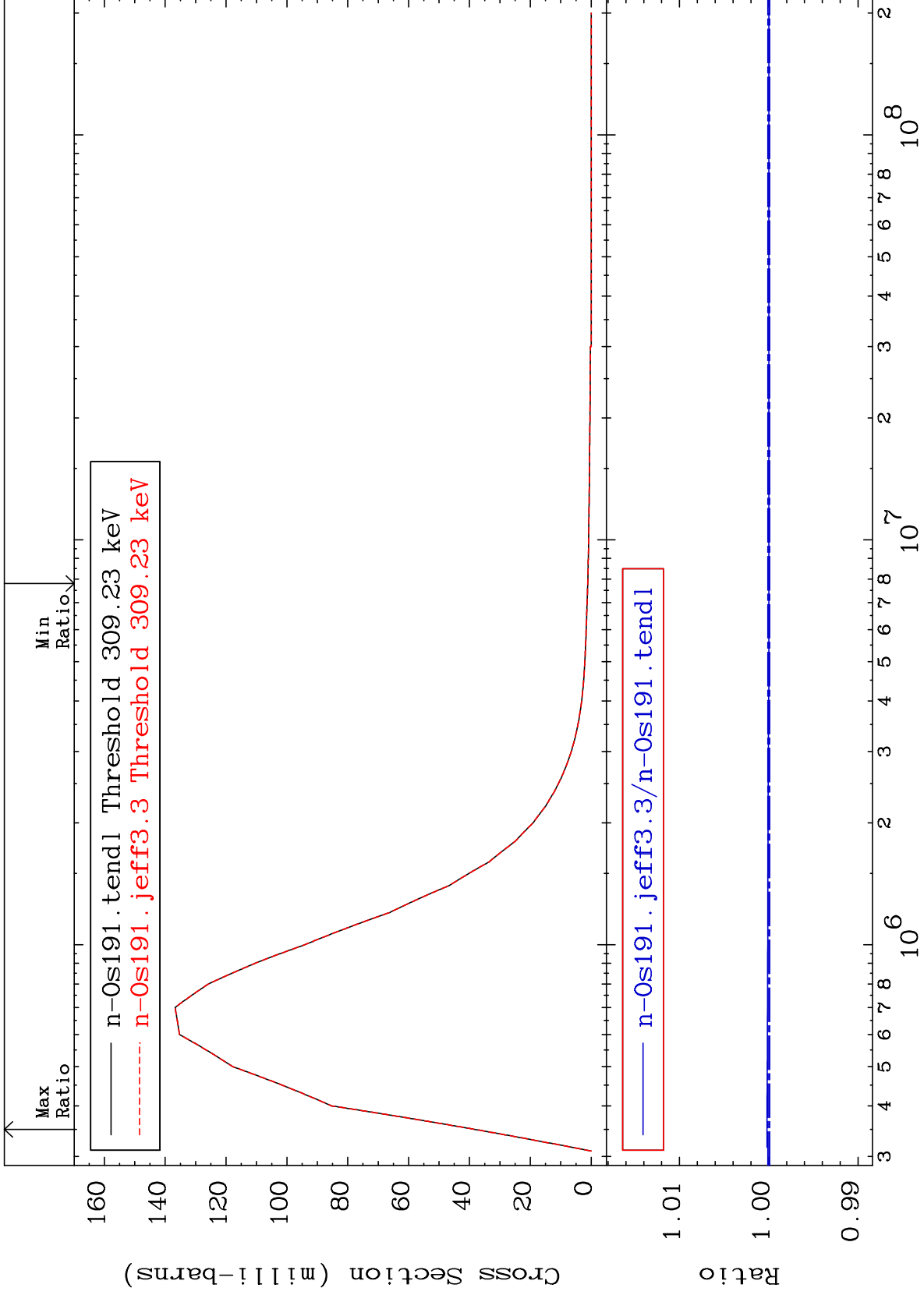
76-0s-191
-0.004 To 0.014 %



MAT 7646

MT= 58 (n,n') Level
Cross Section

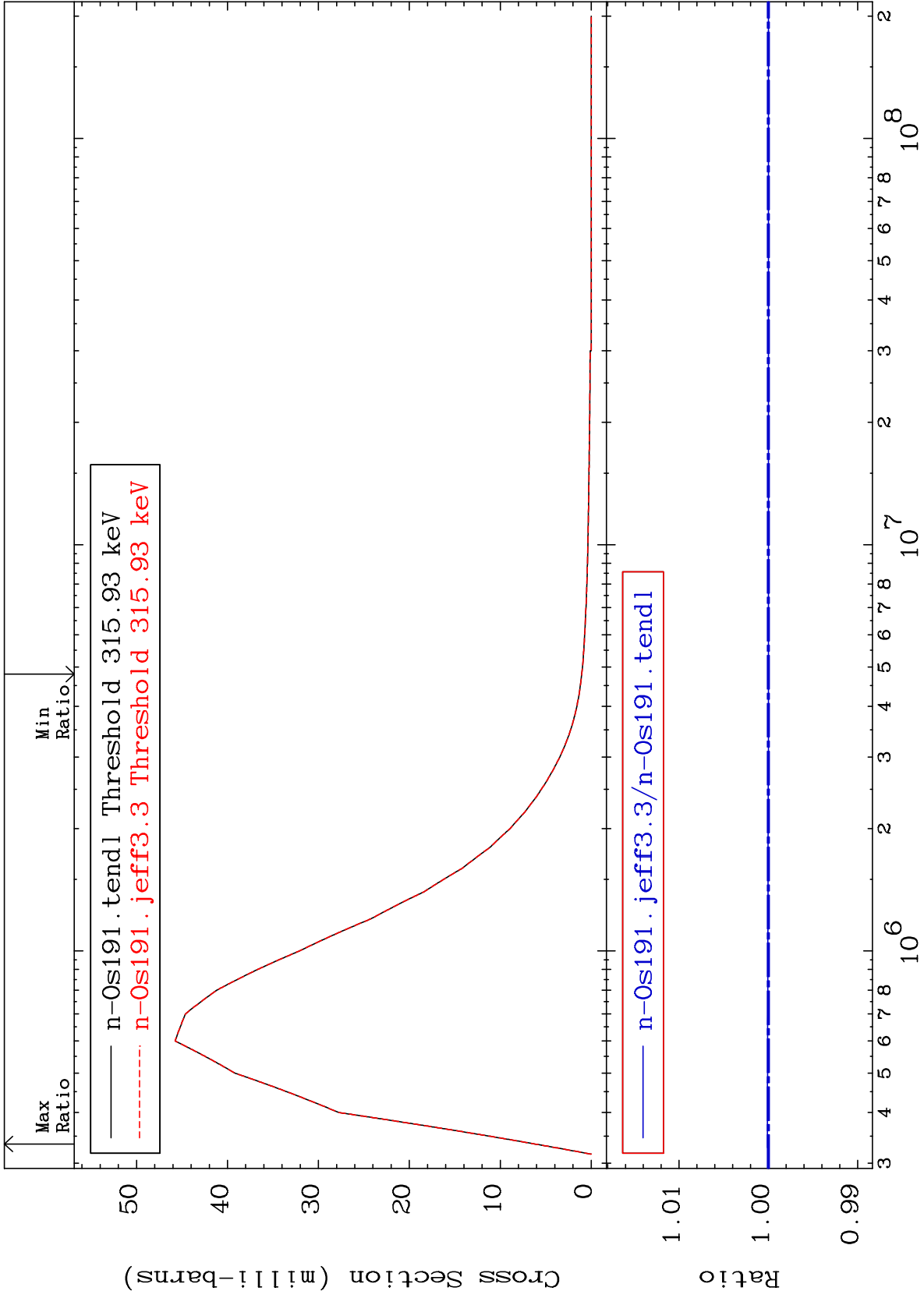
76-0s-191
0.000 To 0.020 %



MAT 7646

MT= 59 (n,n') Level
Cross Section

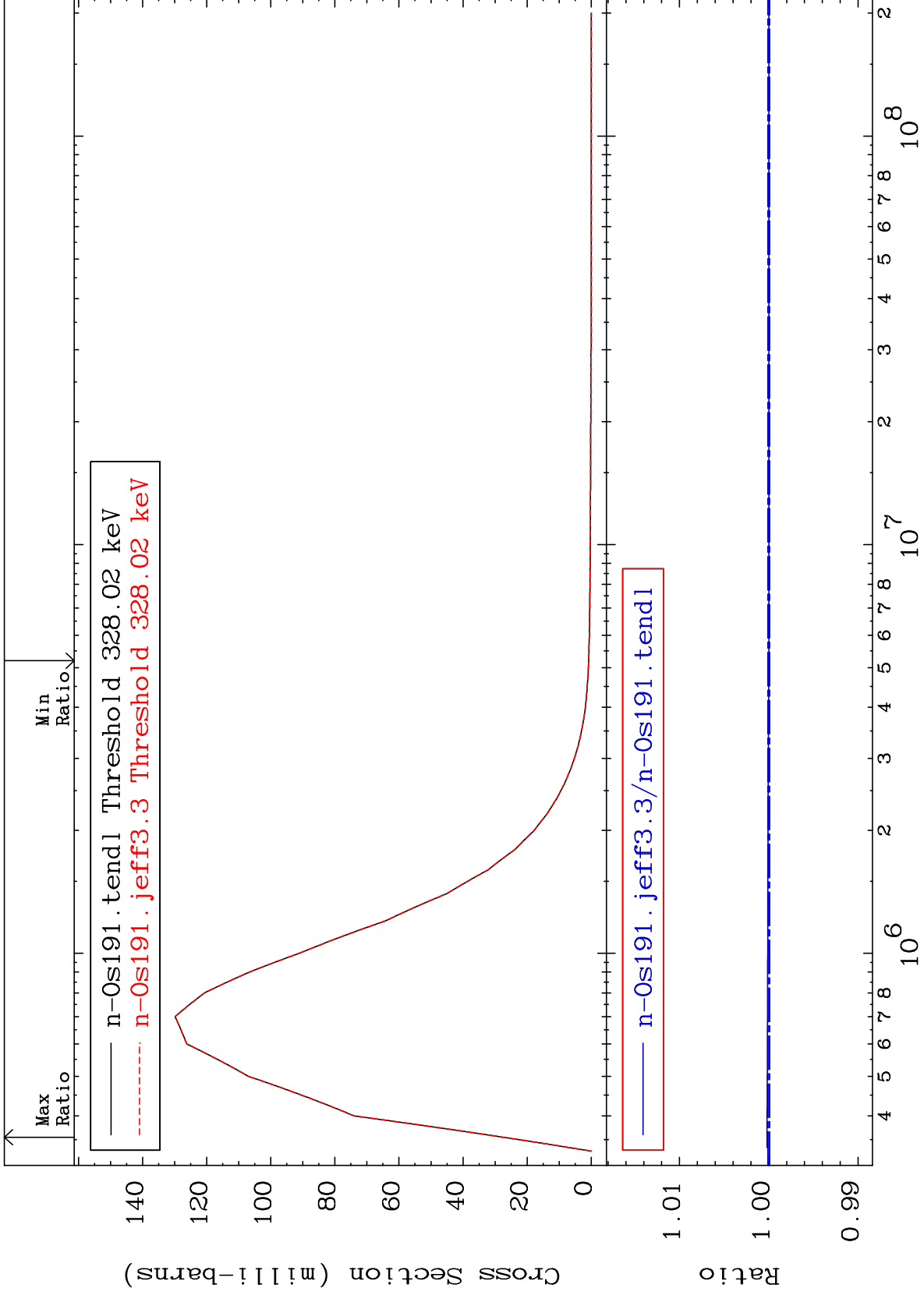
76-0s-191
-0.002 To 0.011 %



MAT 7646

MT= 60 (n,n') Level
Cross Section

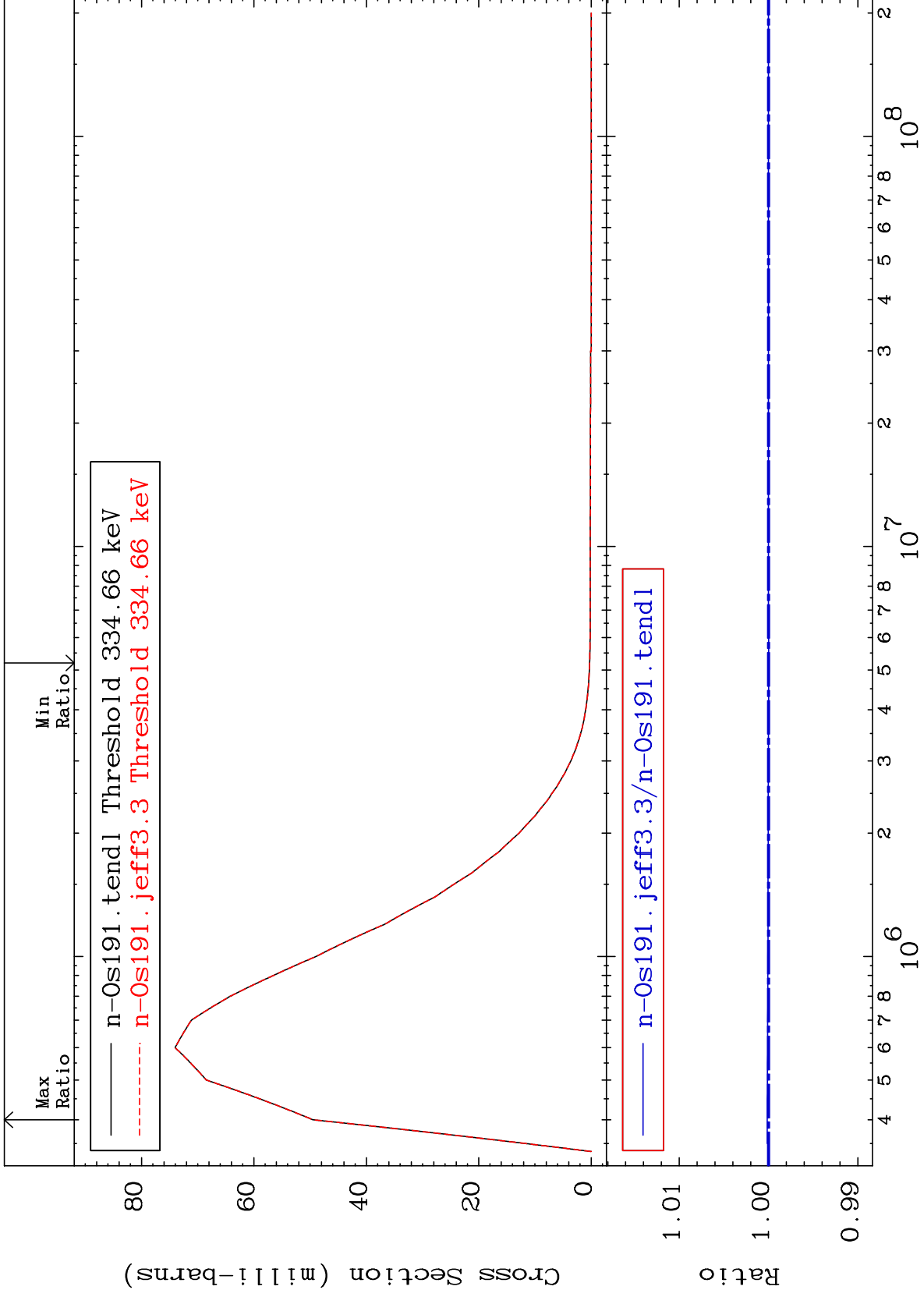
76-0s-191
0.000 To 0.020 %



MAT 7646

MT= 61 (n,n') Level
Cross Section

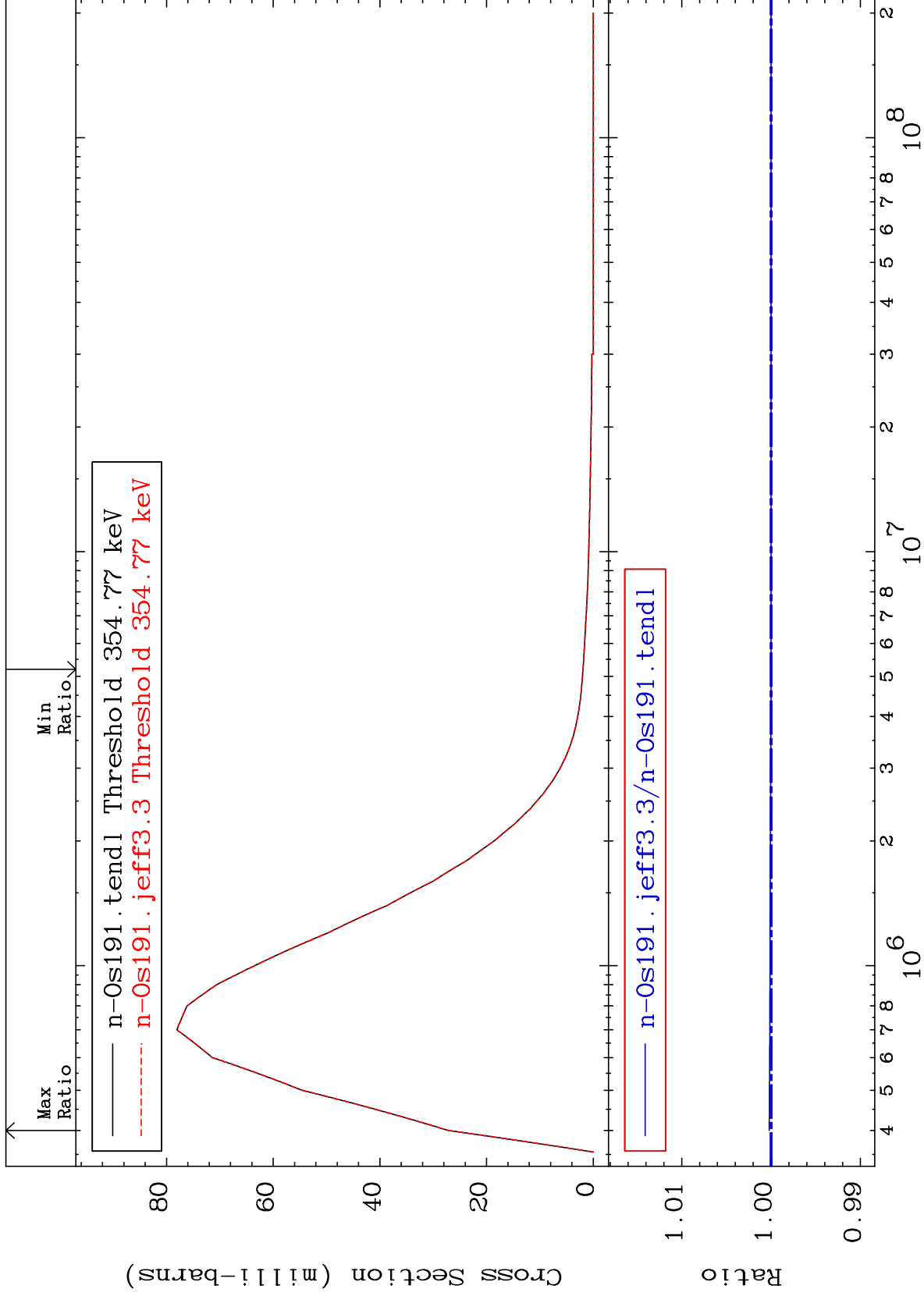
76-0s-191
-0.003 To 0.016 %



MAT 7646

MT= 62 (n,n') Level
Cross Section

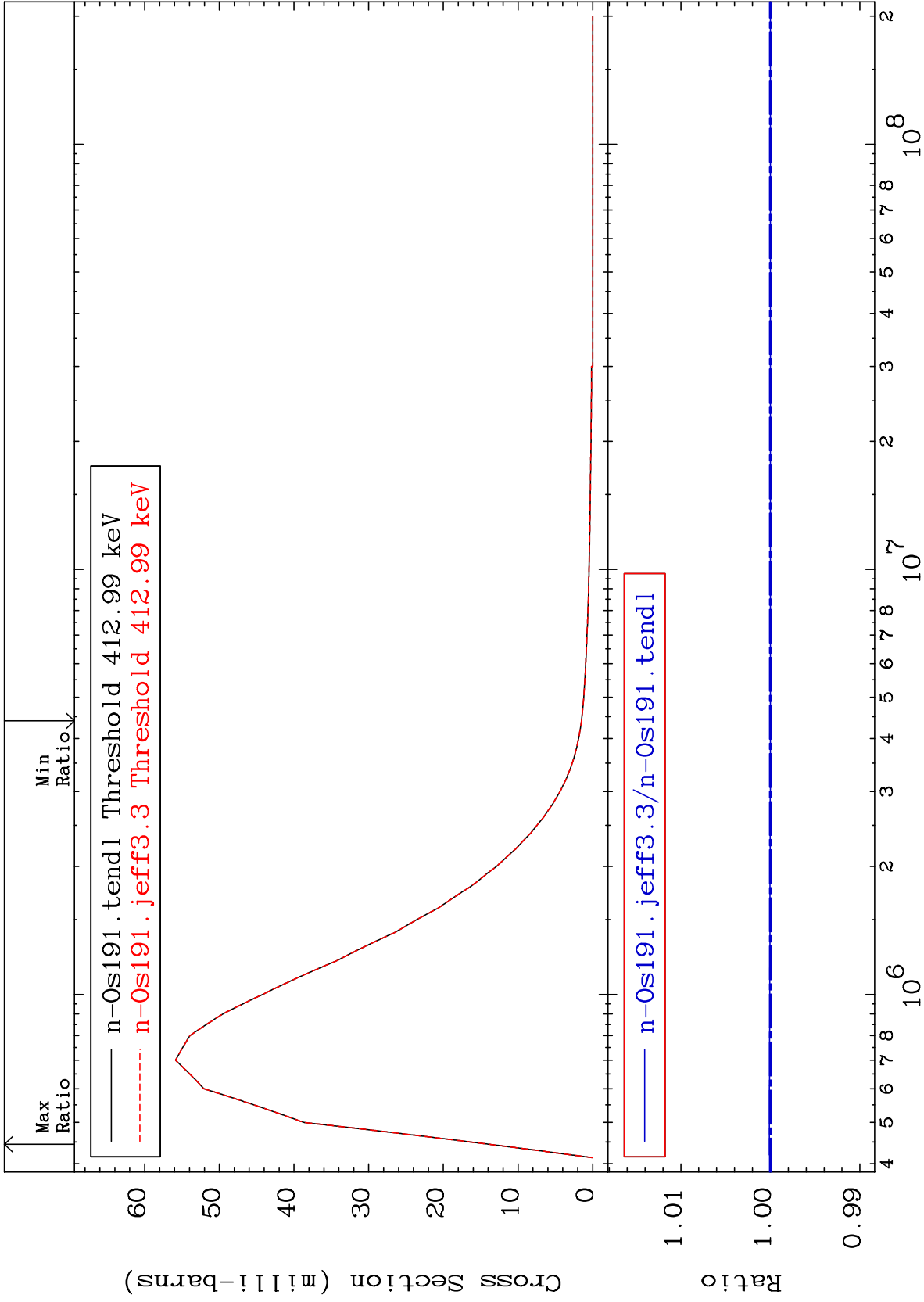
76-0s-191
0.000 To 0.021 %



MAT 7646

MT= 63 (n,n') Level
Cross Section

76-0s-191
-0.001 To 0.012 %



30

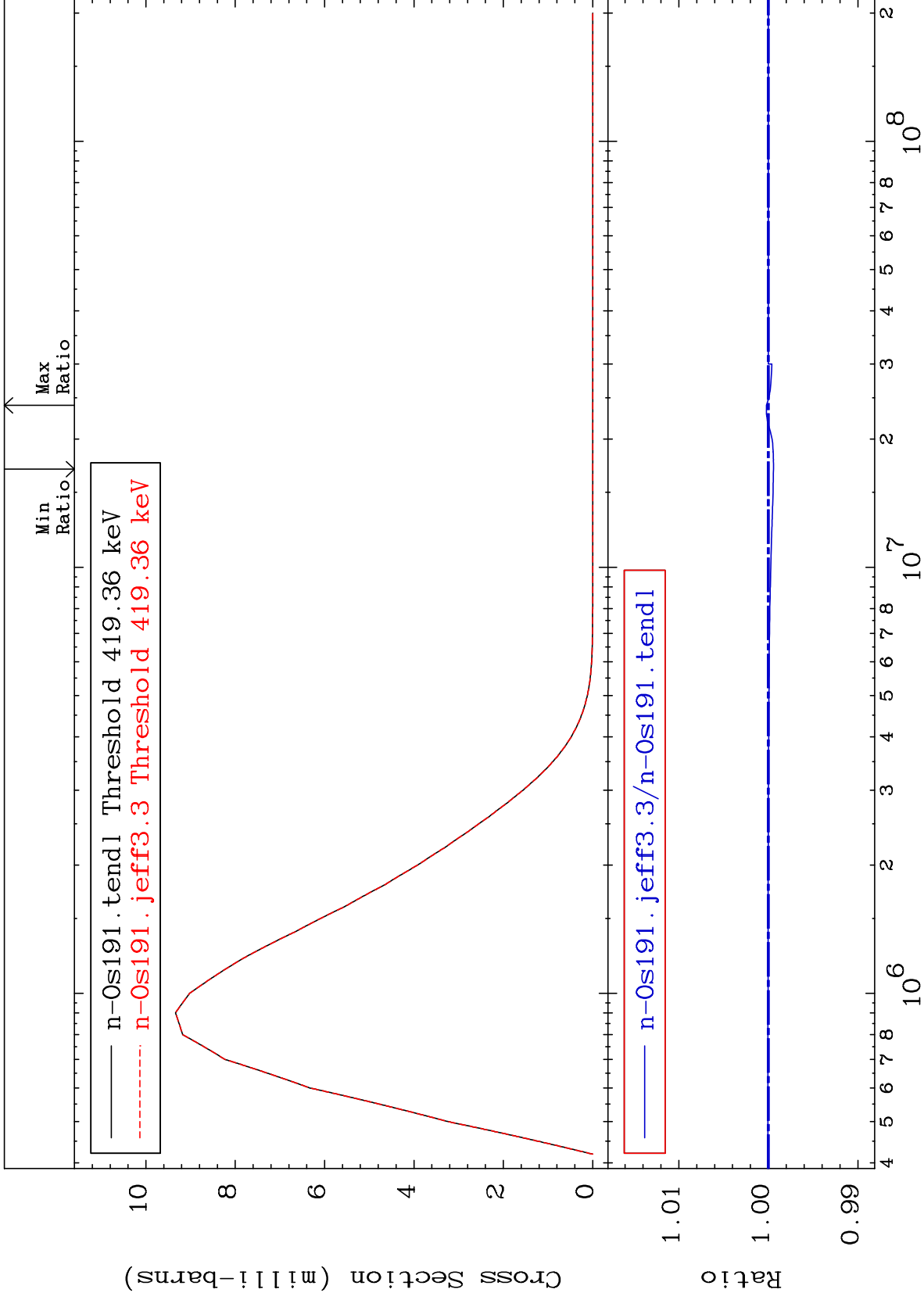
Incident Energy (eV)

76-0s-191

MAT 7646

MT= 64 (n,n') Level
Cross Section

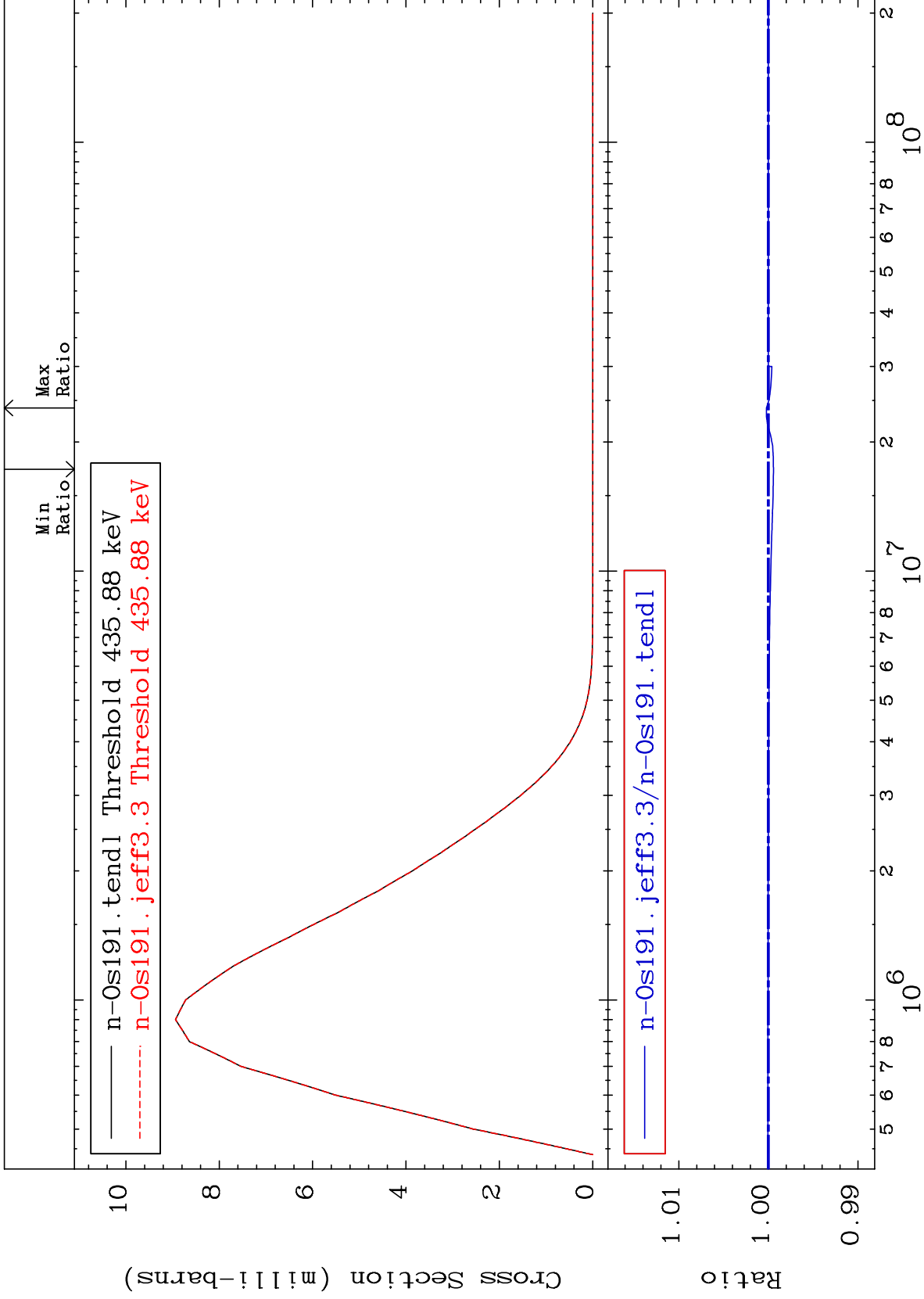
76-0s-191
-0.057 To 0.022 %



MAT 7646

MT= 65 (n,n') Level
Cross Section

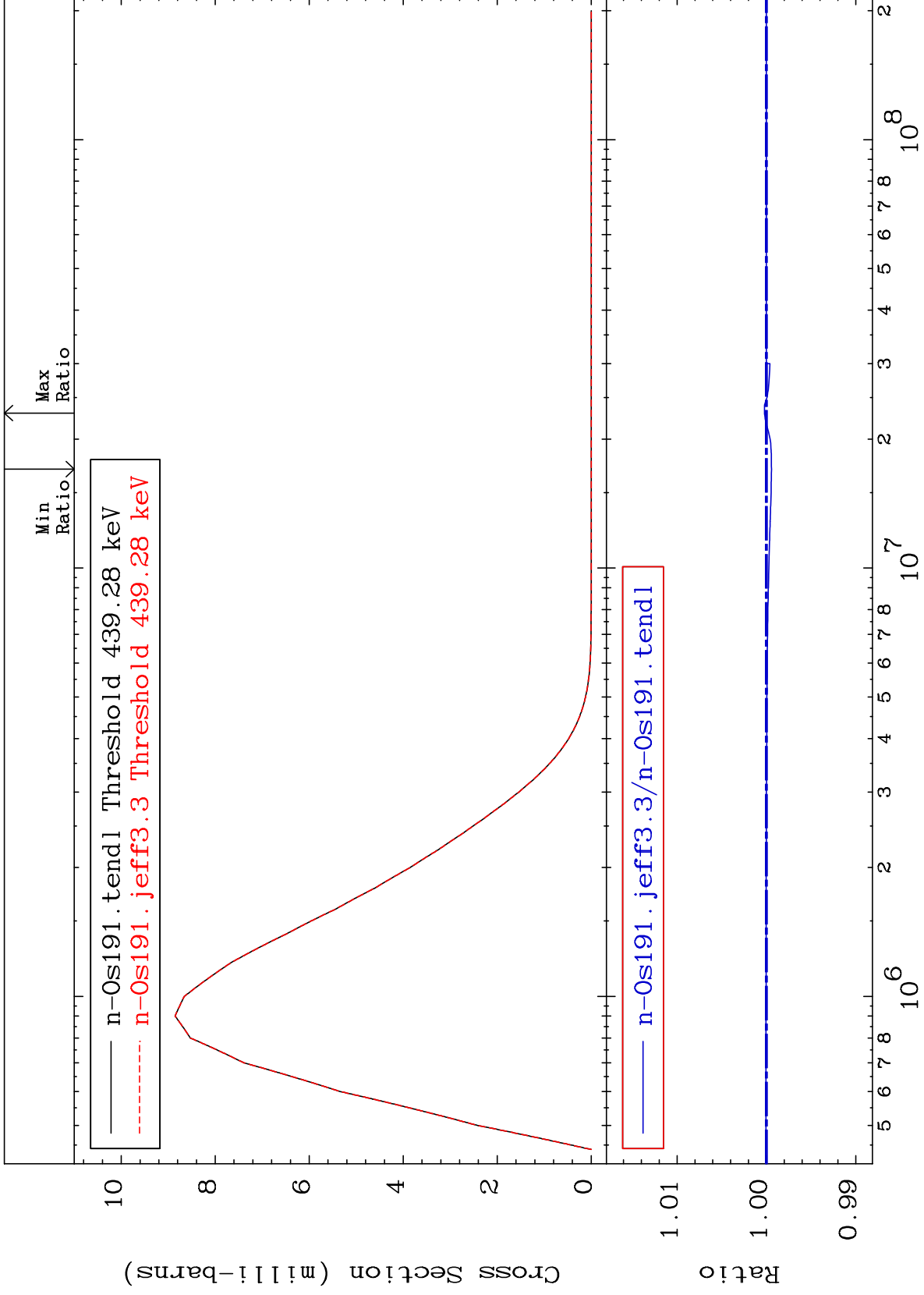
76-0s-191
-0.057 To 0.022 %



MAT 7646

MT= 66 (n,n') Level
Cross Section

76-0s-191
-0.057 To 0.022 %

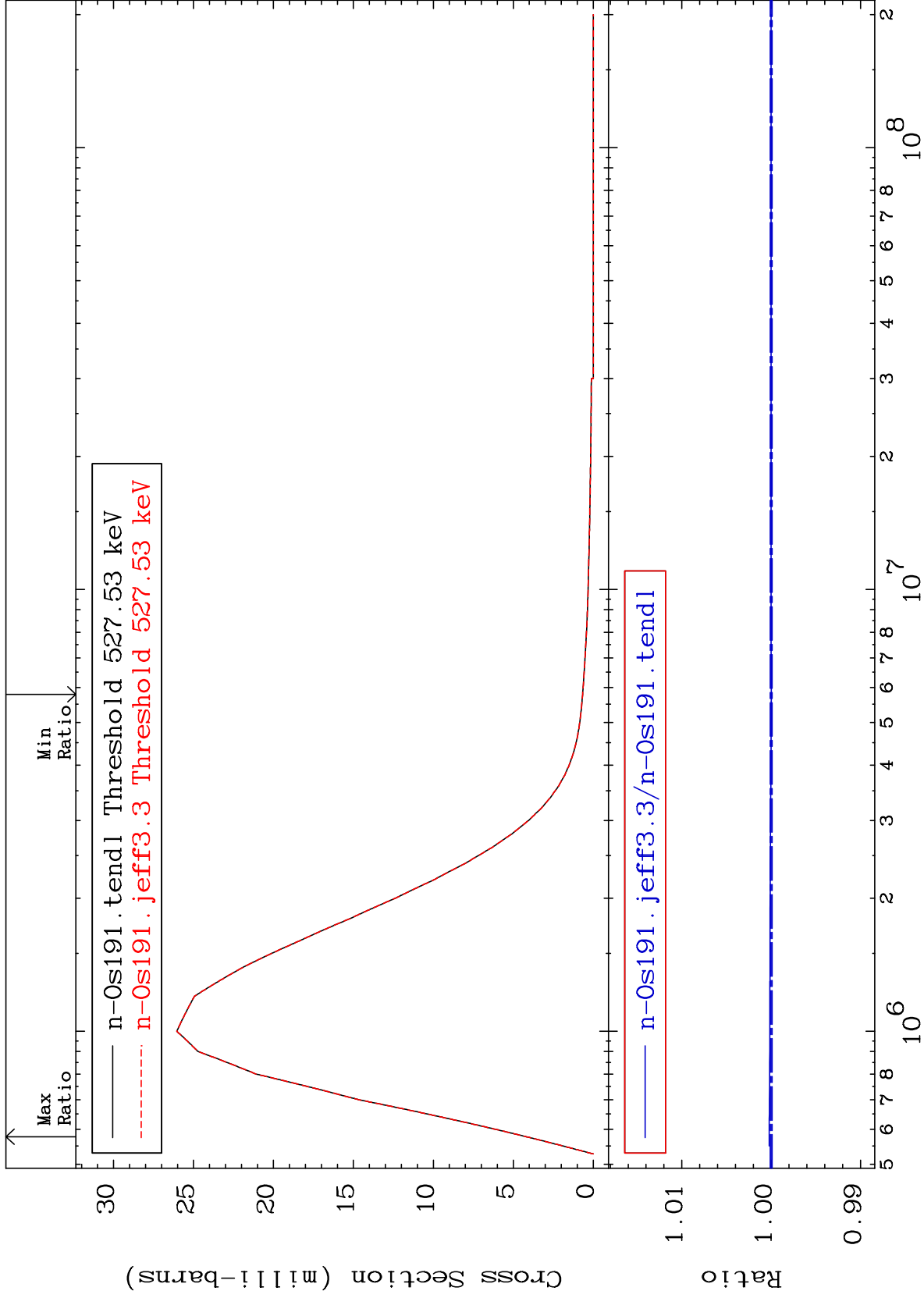


MAT 7646

MT= 74 (n,n') Level

76-0s-191

Cross Section
0.000 To 0.023 %



34

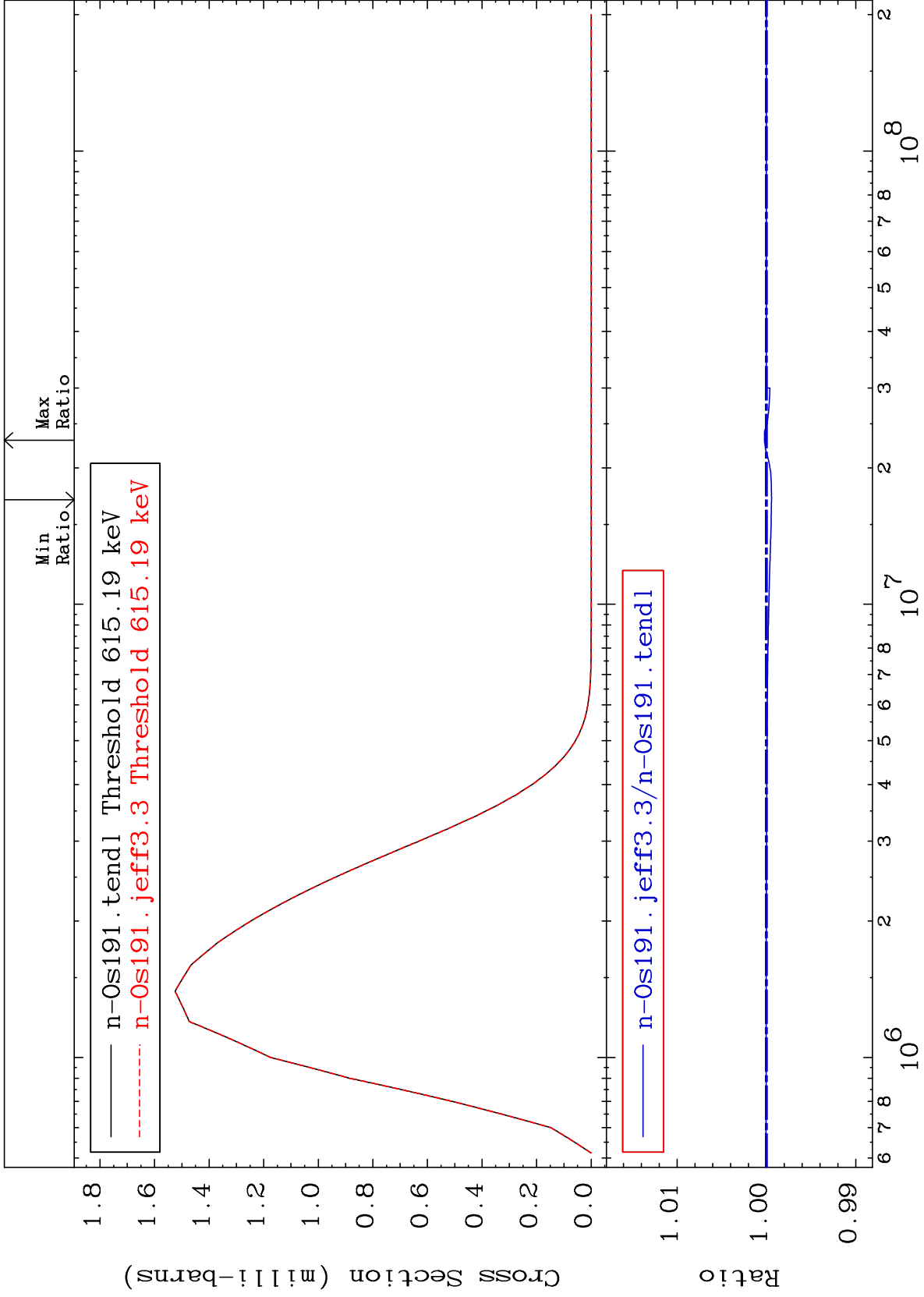
Incident Energy (eV)

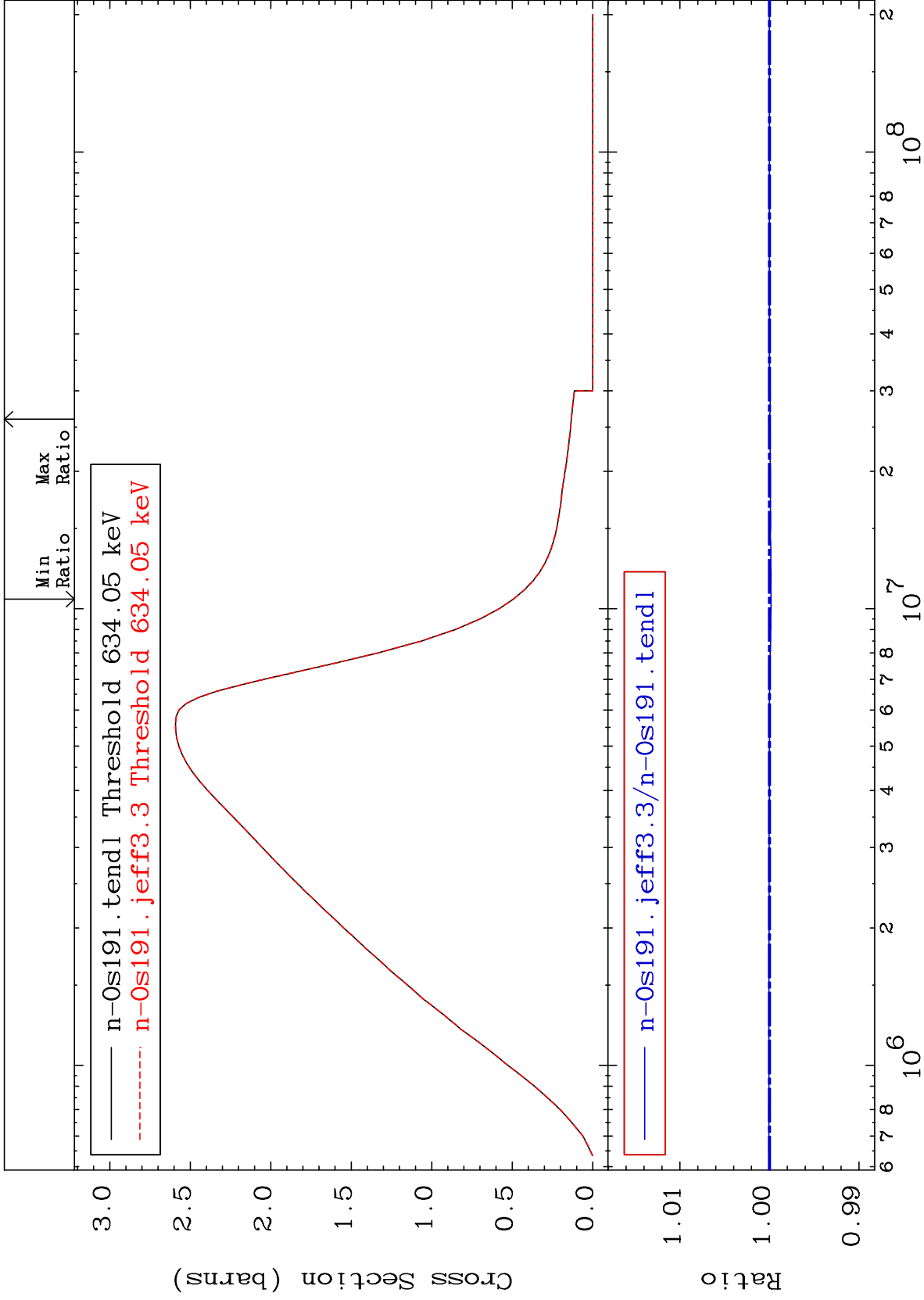
76-0s-191

MAT 7646

MT= 78 (n,n') Level
Cross Section

76-0s-191
-0.057 To 0.022 %

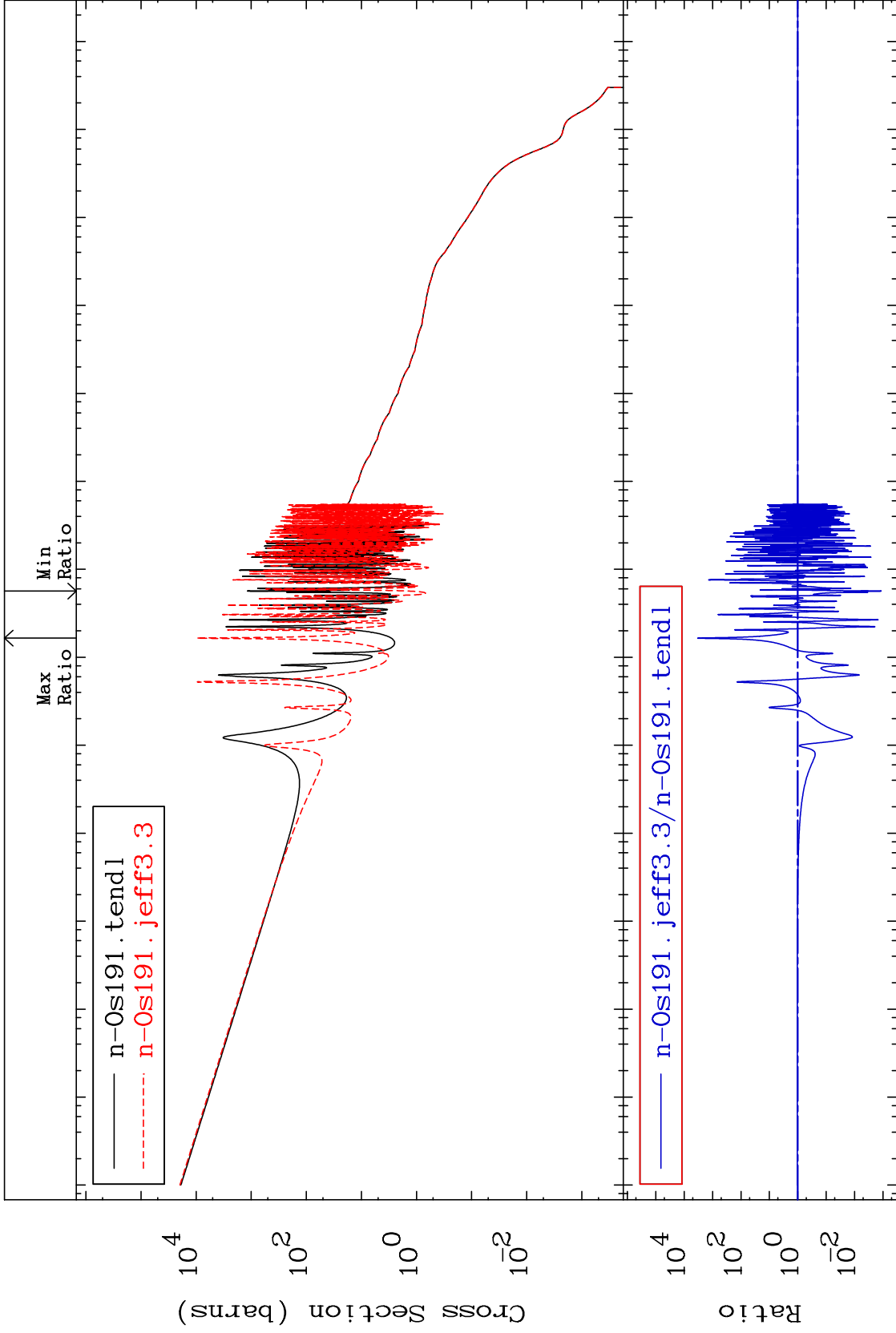




MAT 7646

(n, γ)
Cross Section

76-0s-191
-99.88 To 9999. %



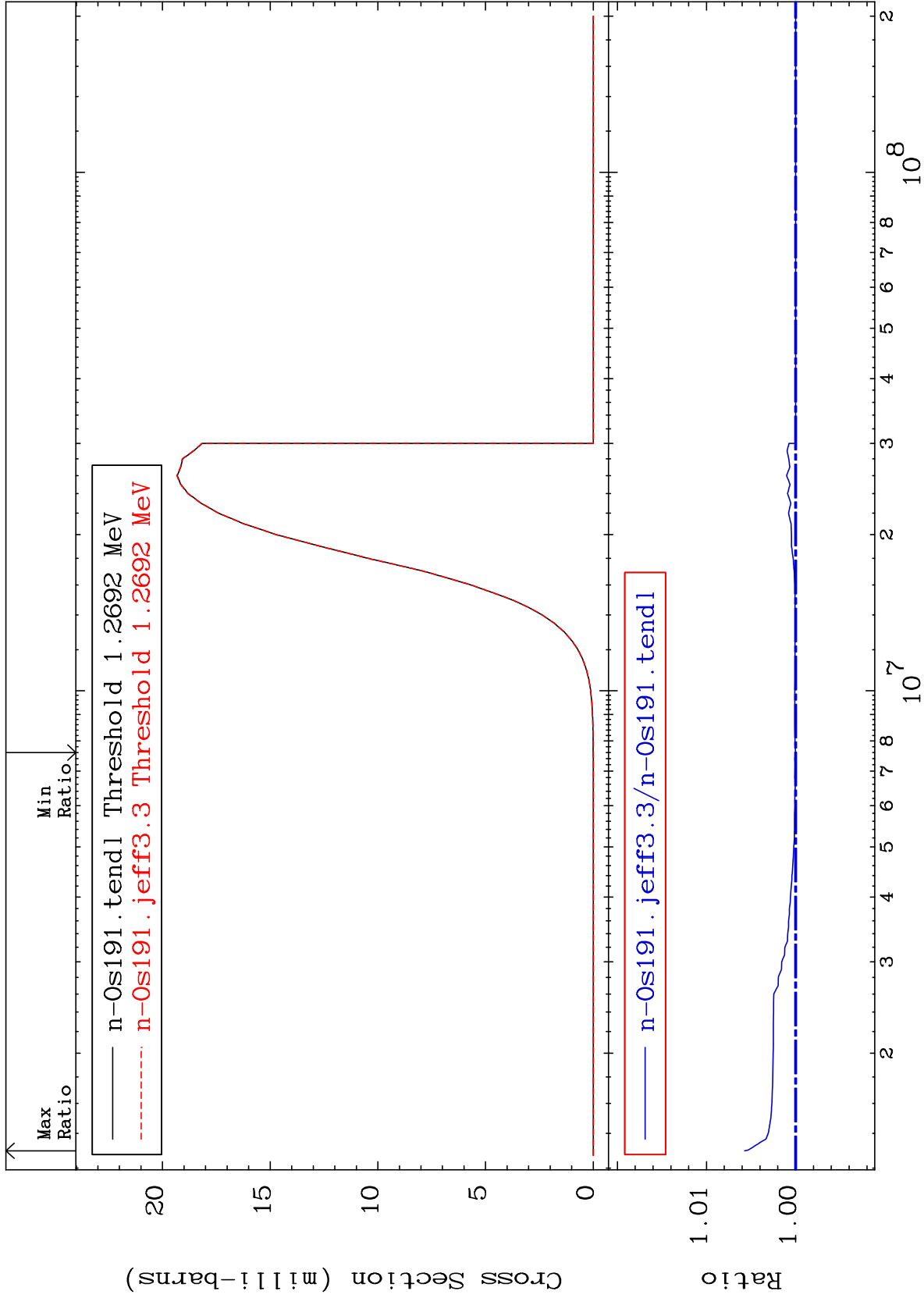
MAT 7646

(n,p)

76-0s-191

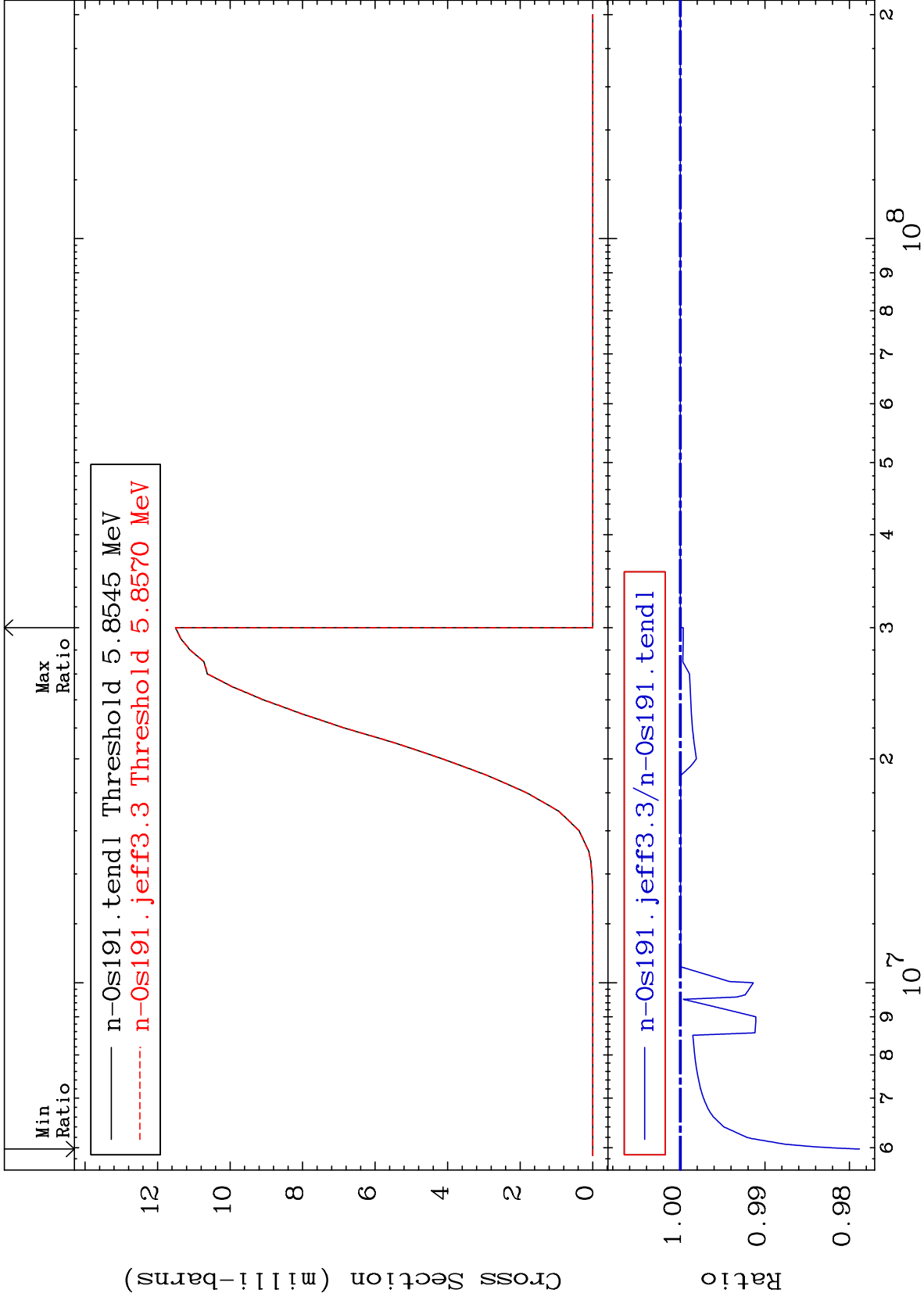
Cross Section

0.000 To 0.572 %



Cross Section

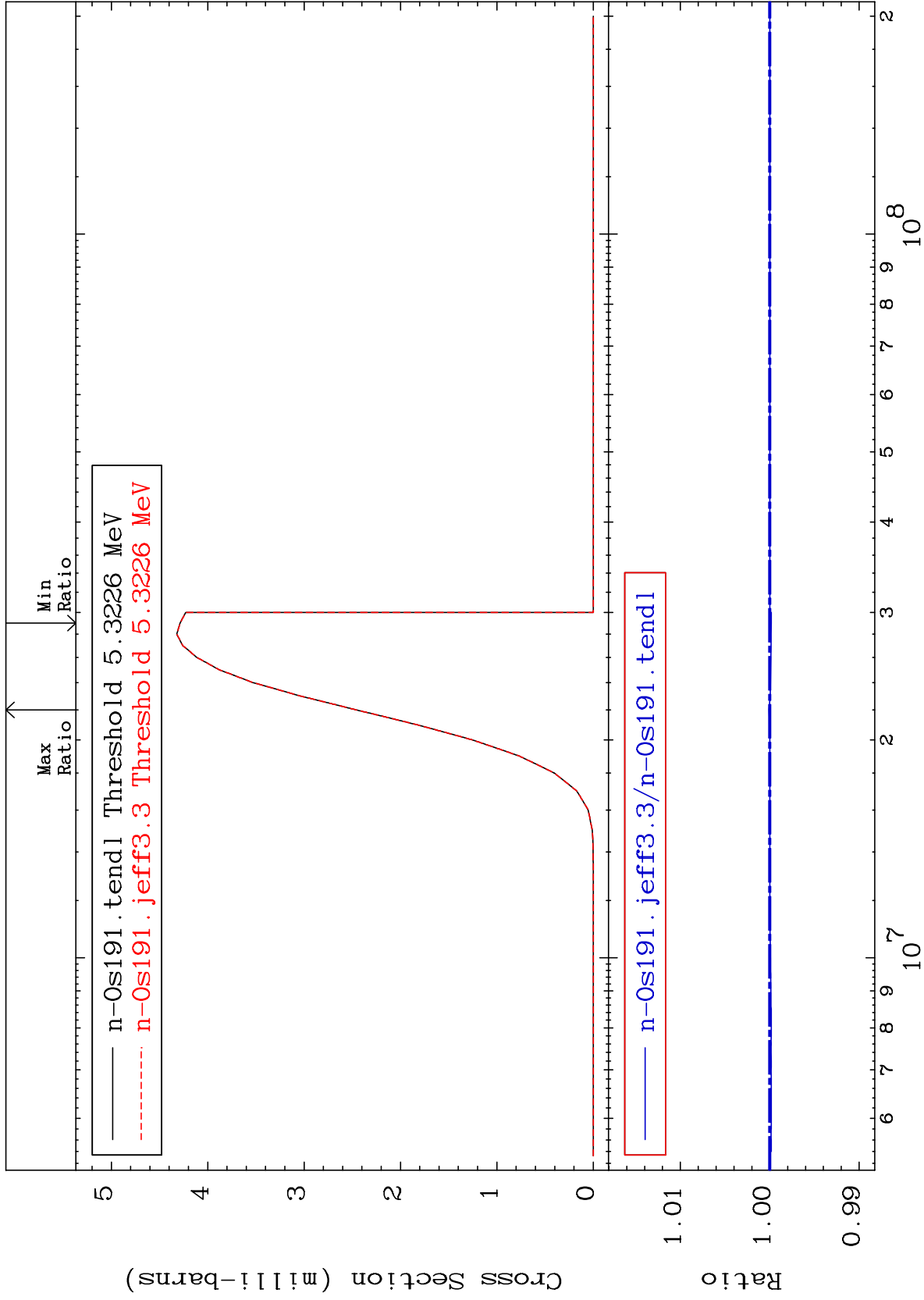
-2.118 To 0.000 %



MAT 7646

76-0s-191

(n, t)
Cross Section
-0.018 To 0.009 %



40

Incident Energy (eV)

76-0s-191

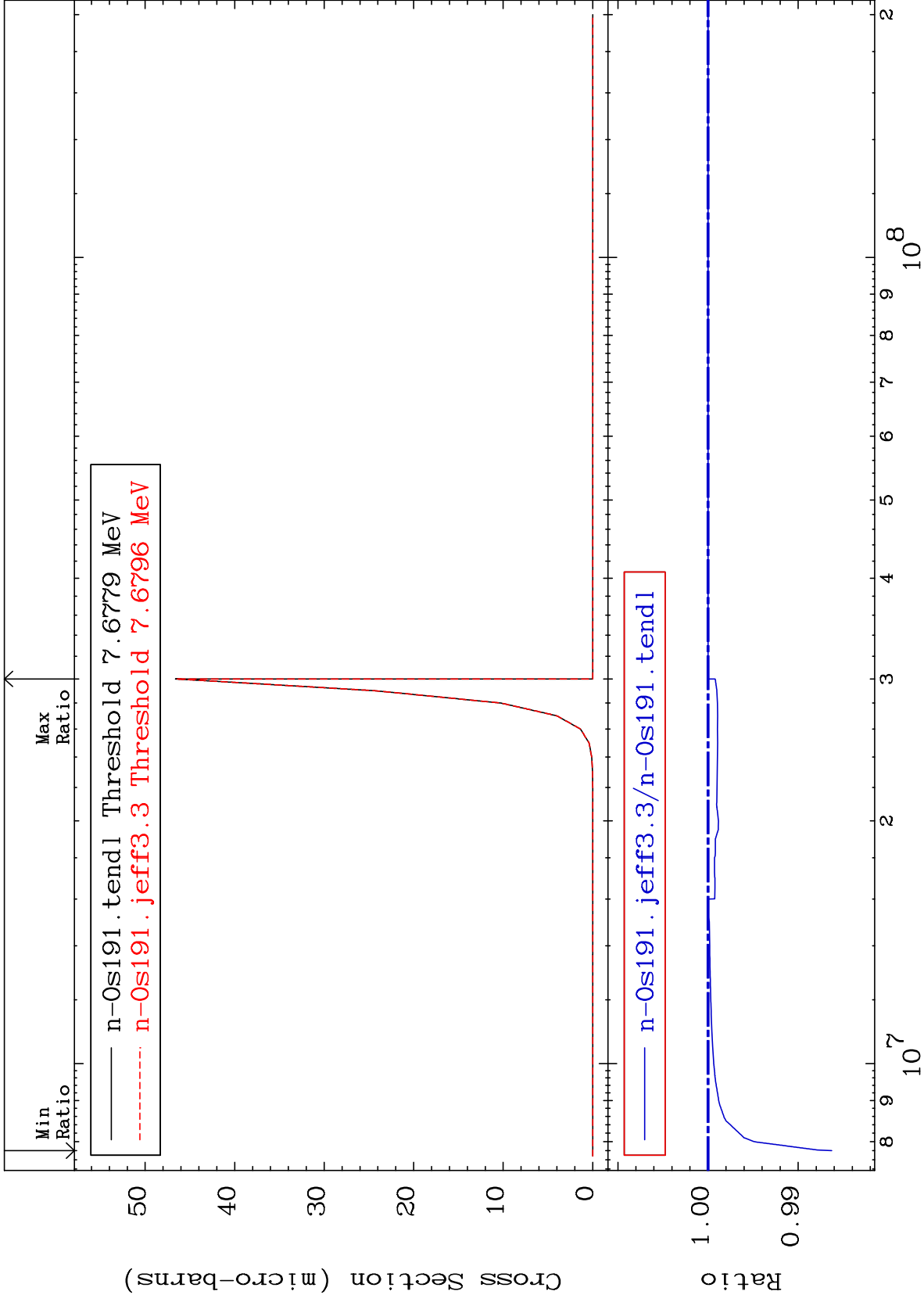
MAT 7646

(n, He-3)

76-0s-191

Cross Section

-1.373 To 0.000 %



41

Incident Energy (eV)

76-0s-191

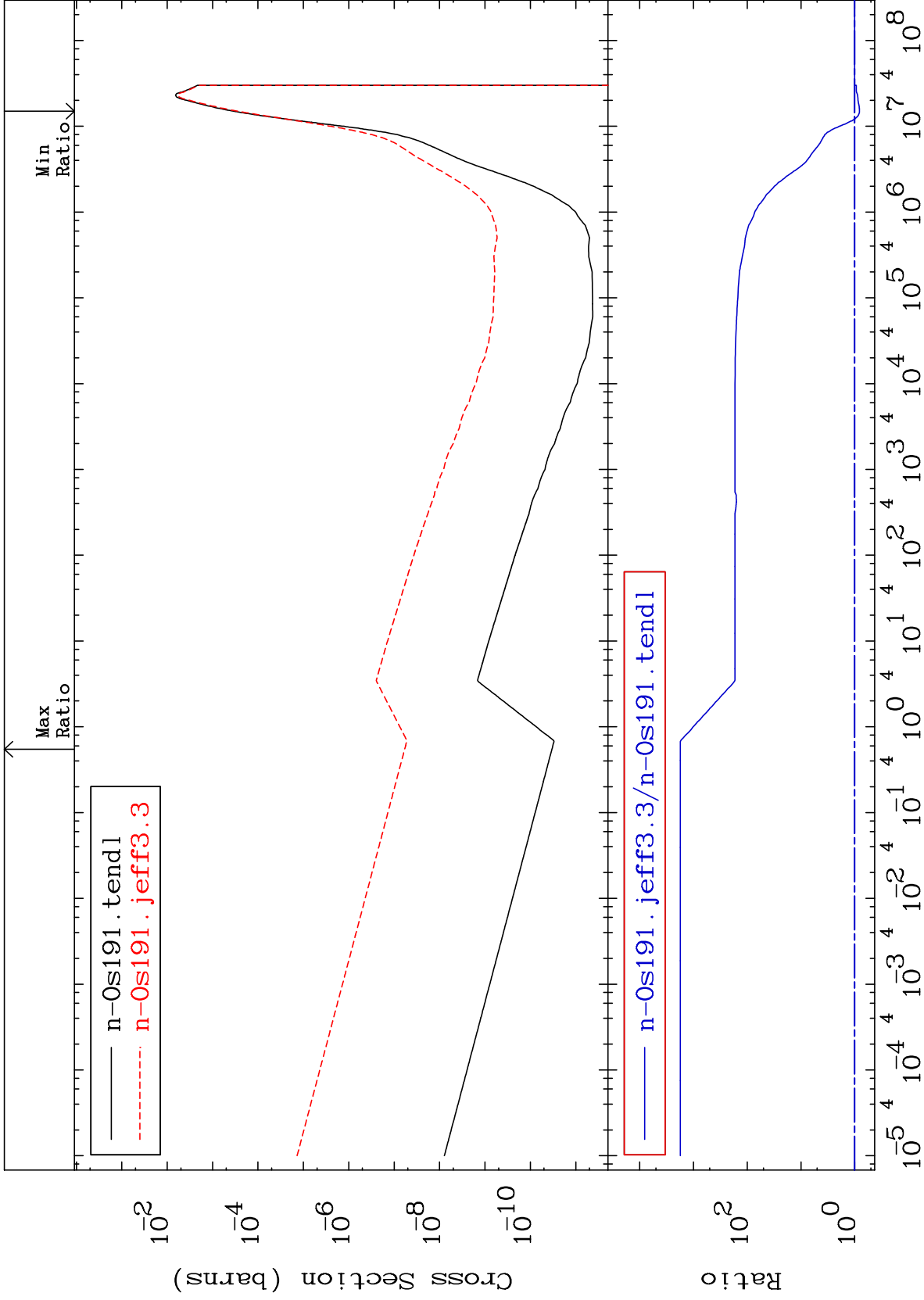
MAT 7646

(n, α)

76-0s-191

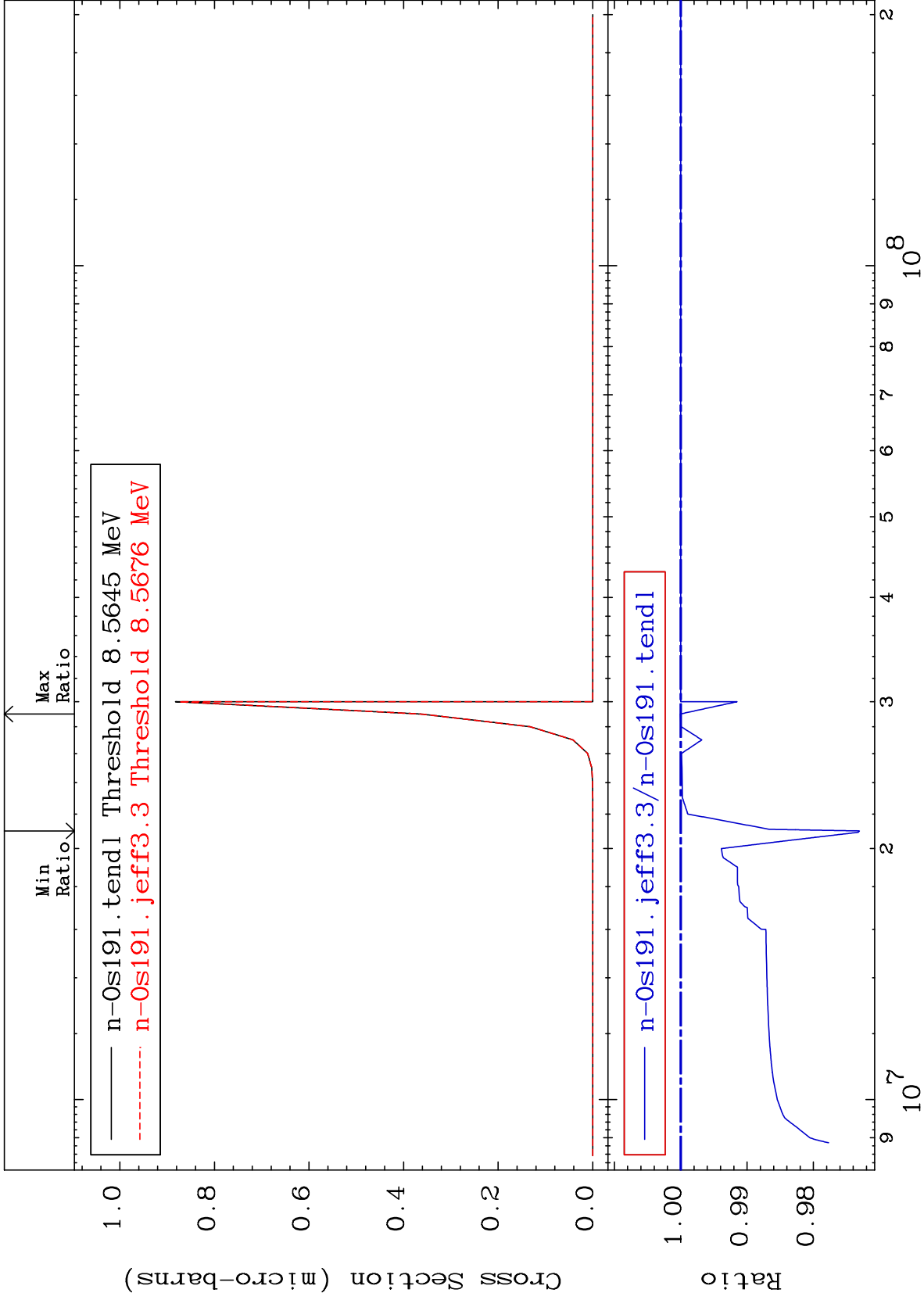
Cross Section

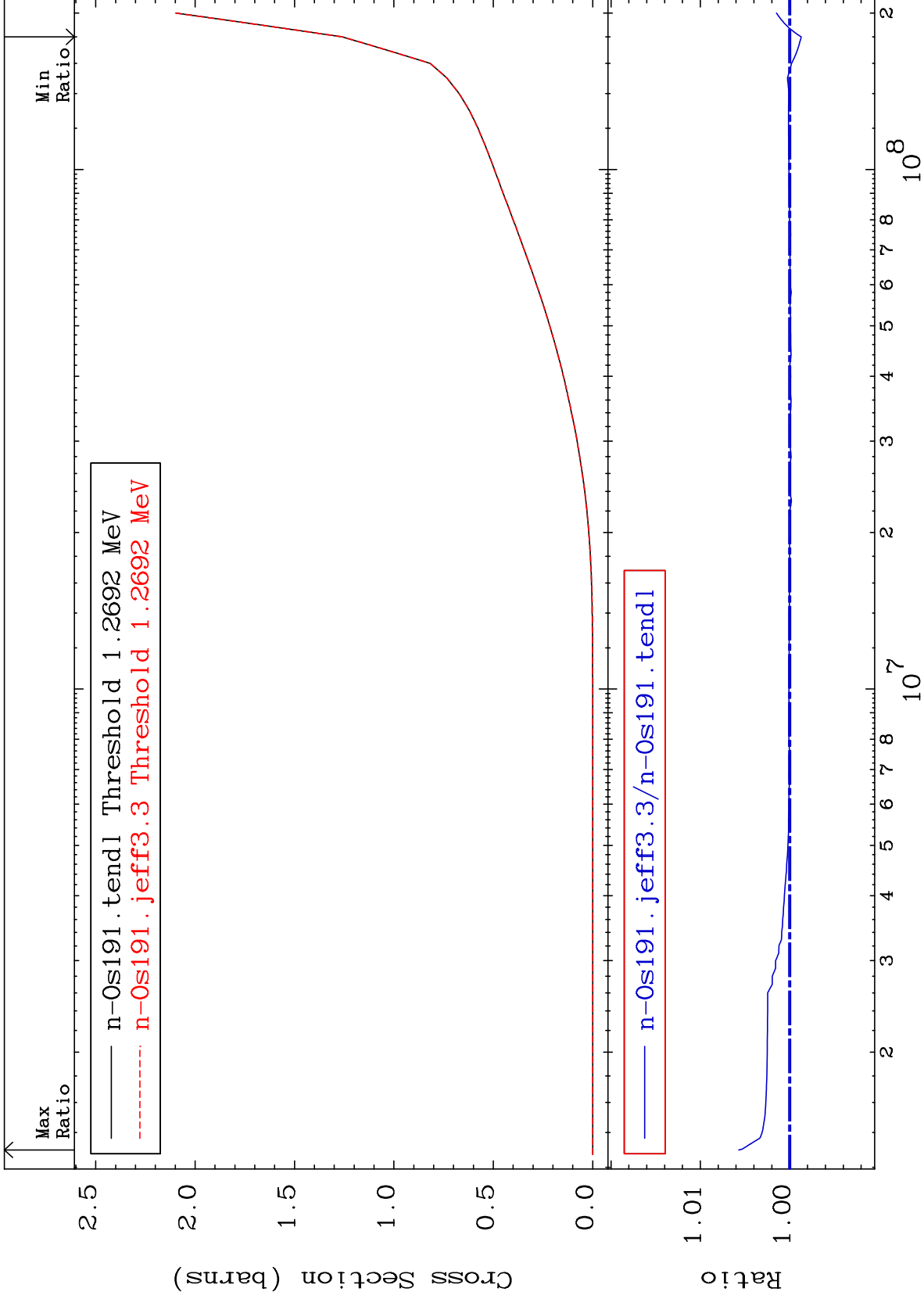
-18.31 To 9999. %



Cross Section

-2.693 To 0.006 %

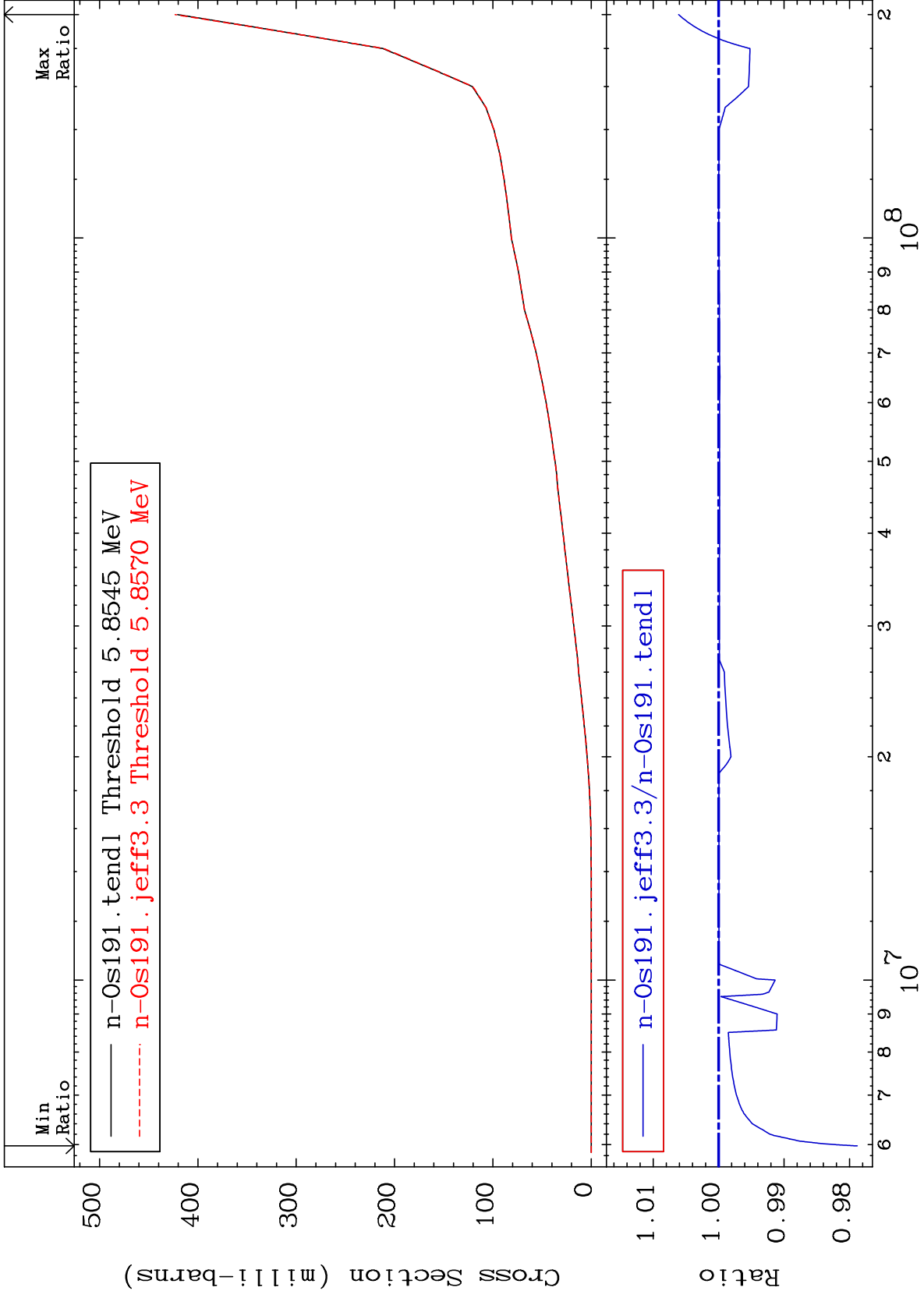




MAT 7646

Deuterium Production
Cross Section

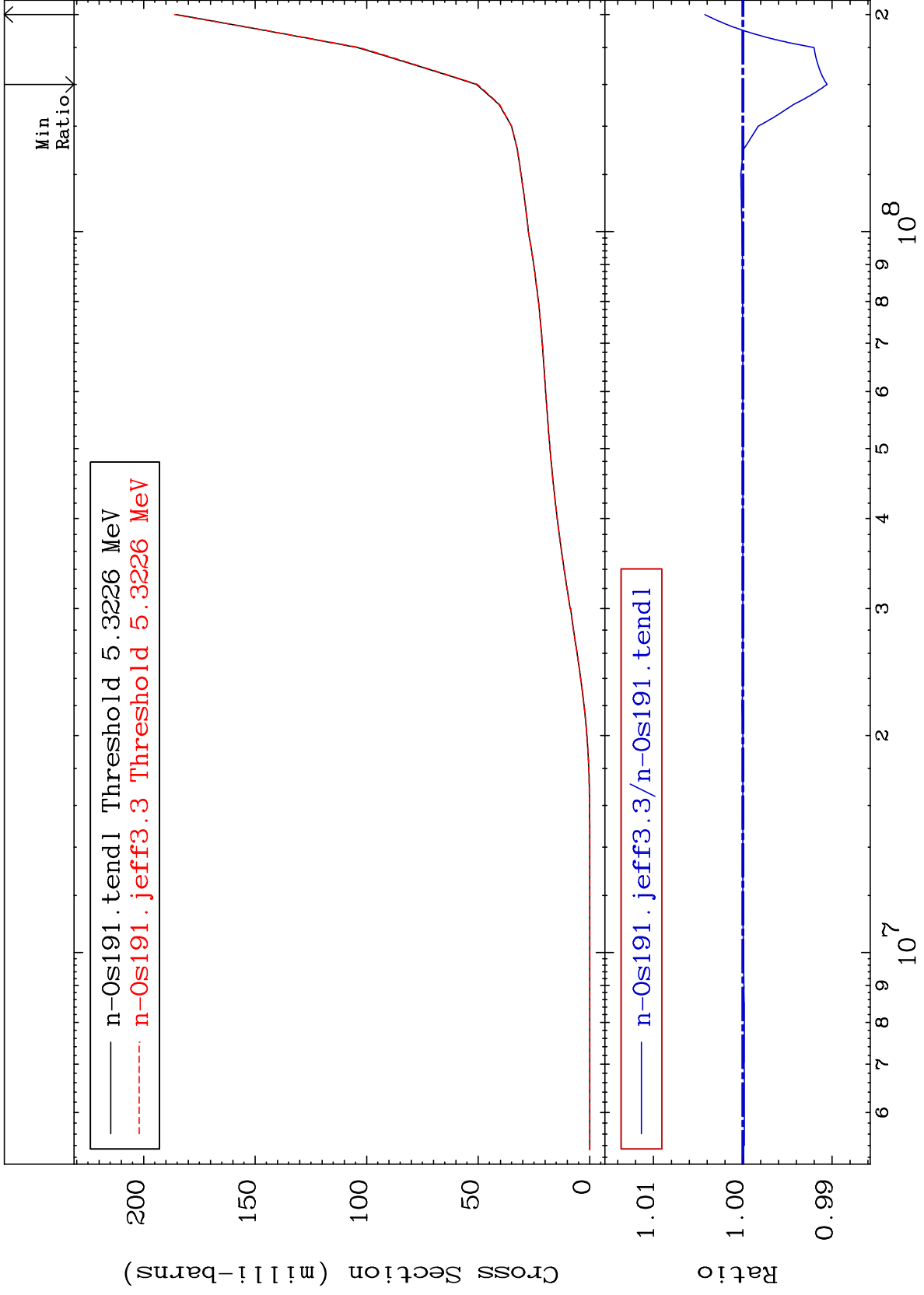
76-0s-191
-2.118 To 0.611 %



MAT 7646

Tritium Production
Cross Section

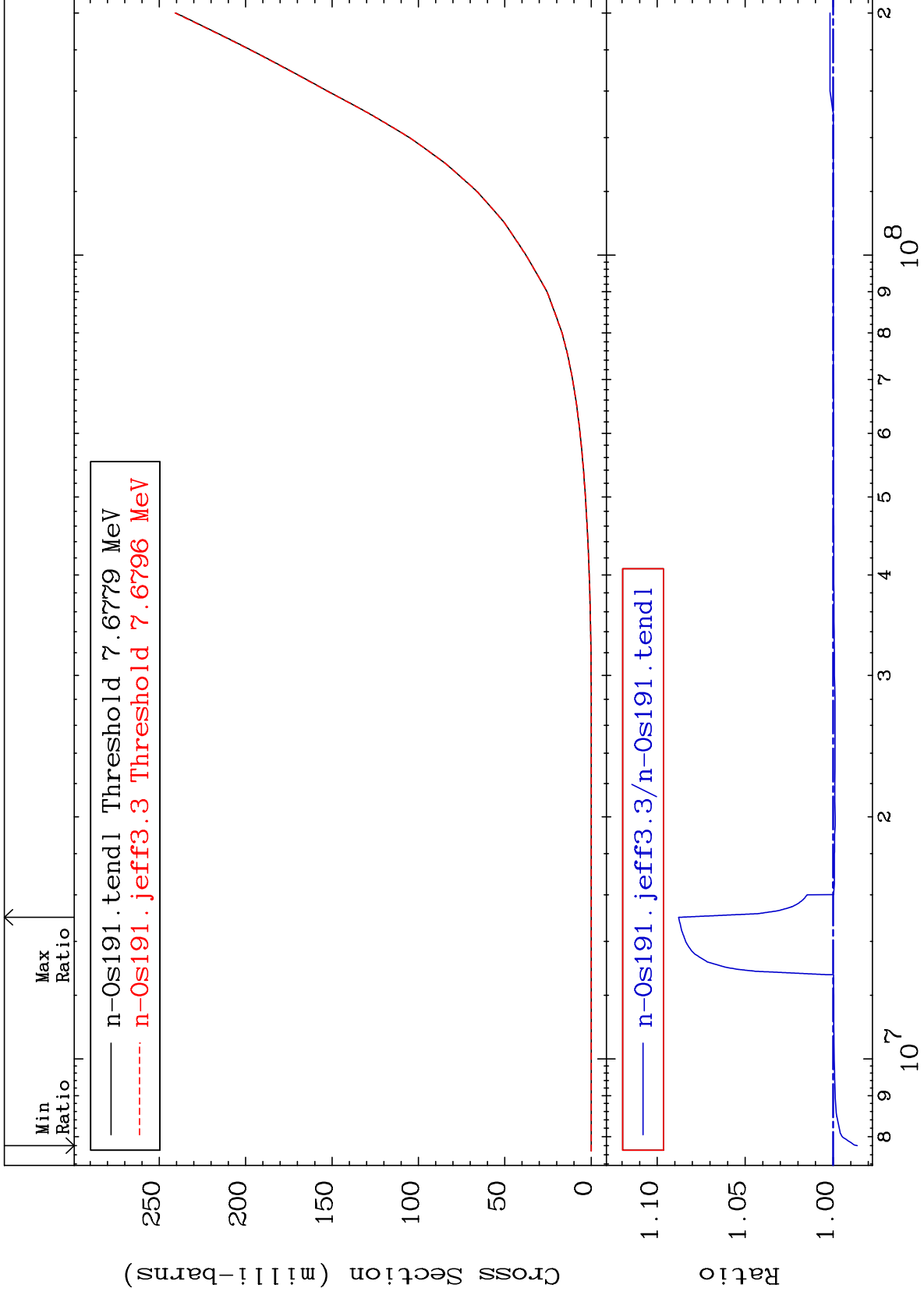
76-0s-191
-0.943 To 0.424 %



MAT 7646

He-3 Production
Cross Section

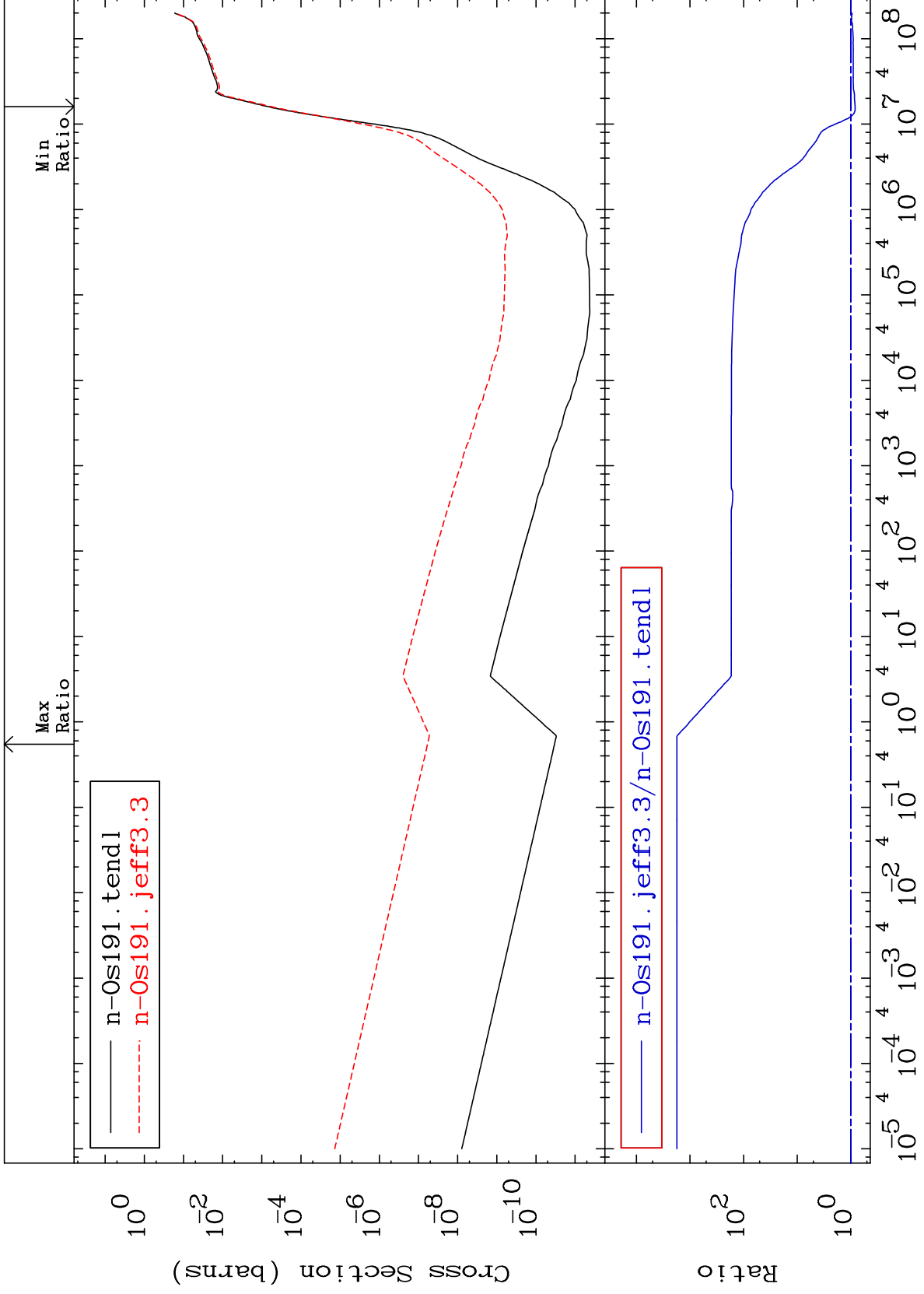
76-0s-191
-1.373 To 8.784 %



MAT 7646

He-4 Production
Cross Section

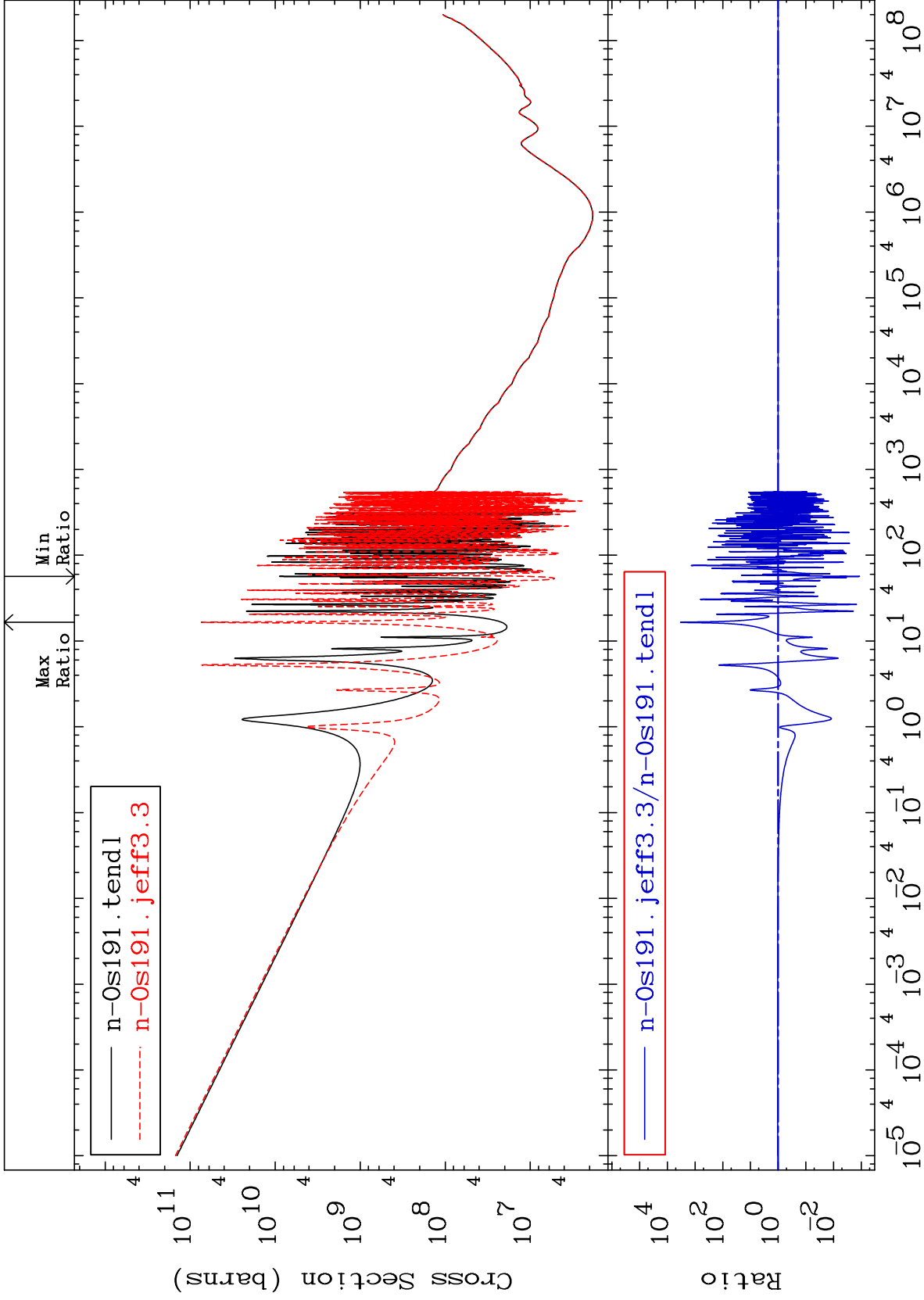
76-0s-191
-16.84 To 9999. %



48

Incident Energy (eV)

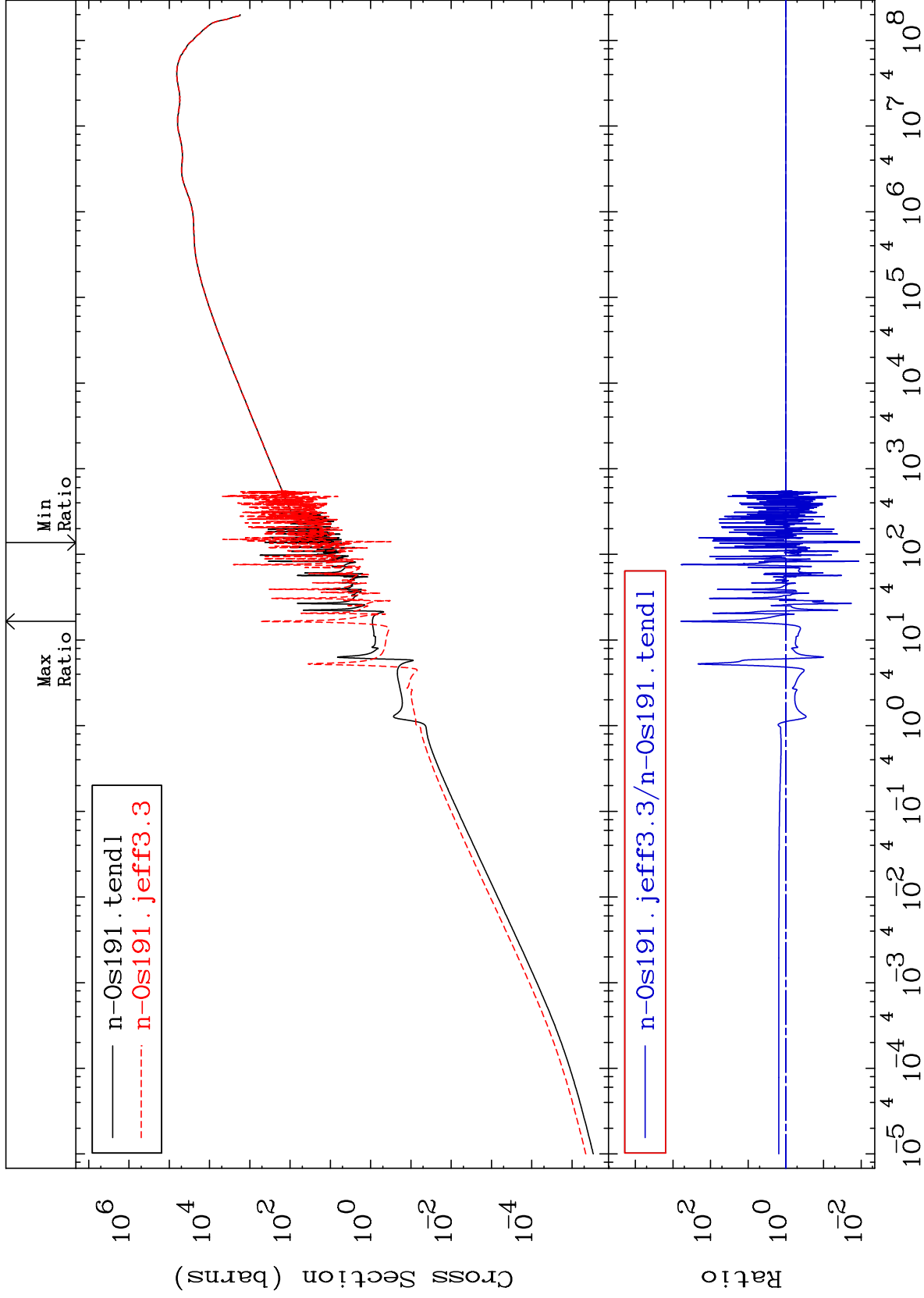
76-0s-191



MAT 7646

Kerma elastic
Cross Section

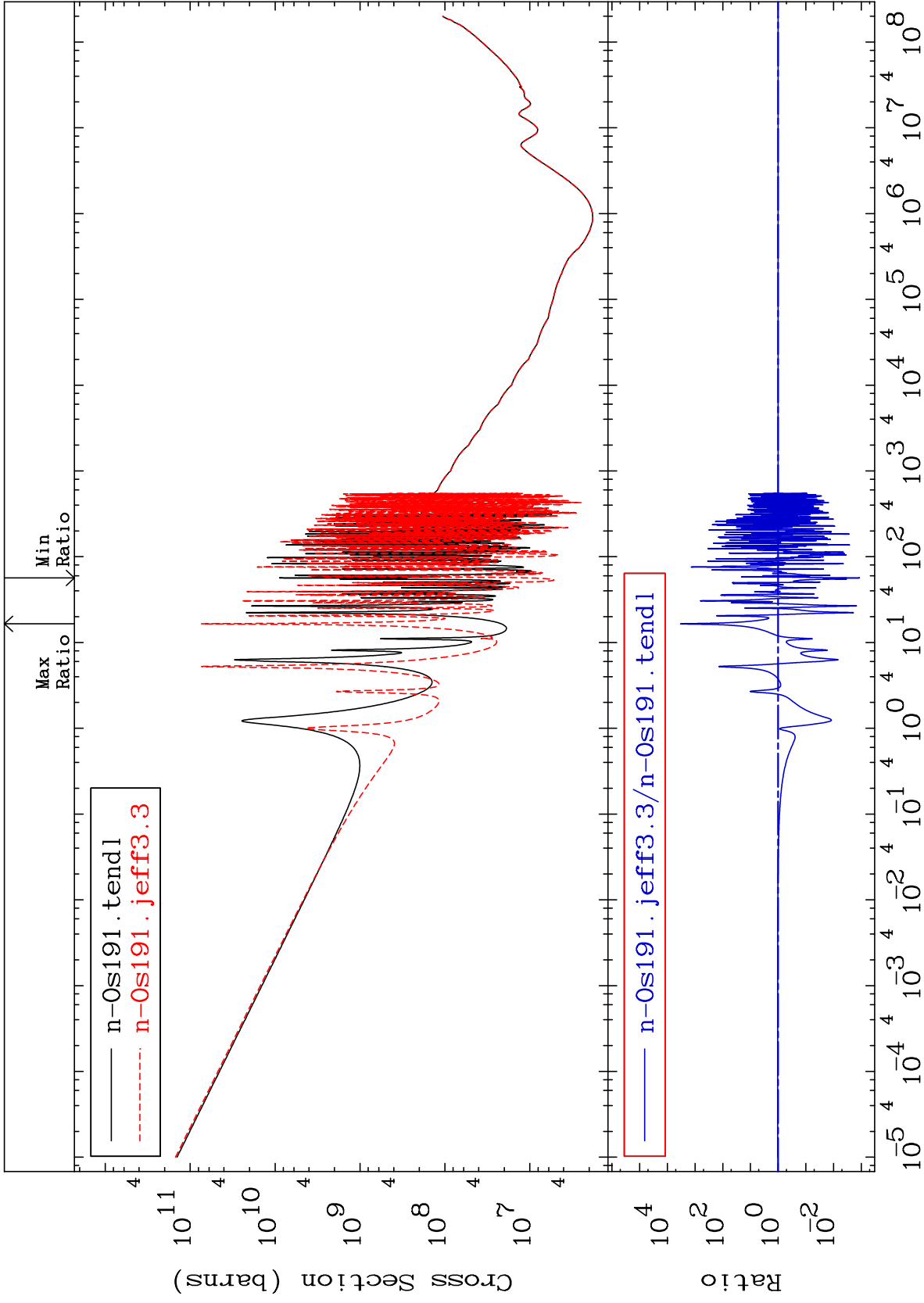
76-0s-191
-98.90 To 9999. %



50

Incident Energy (eV)

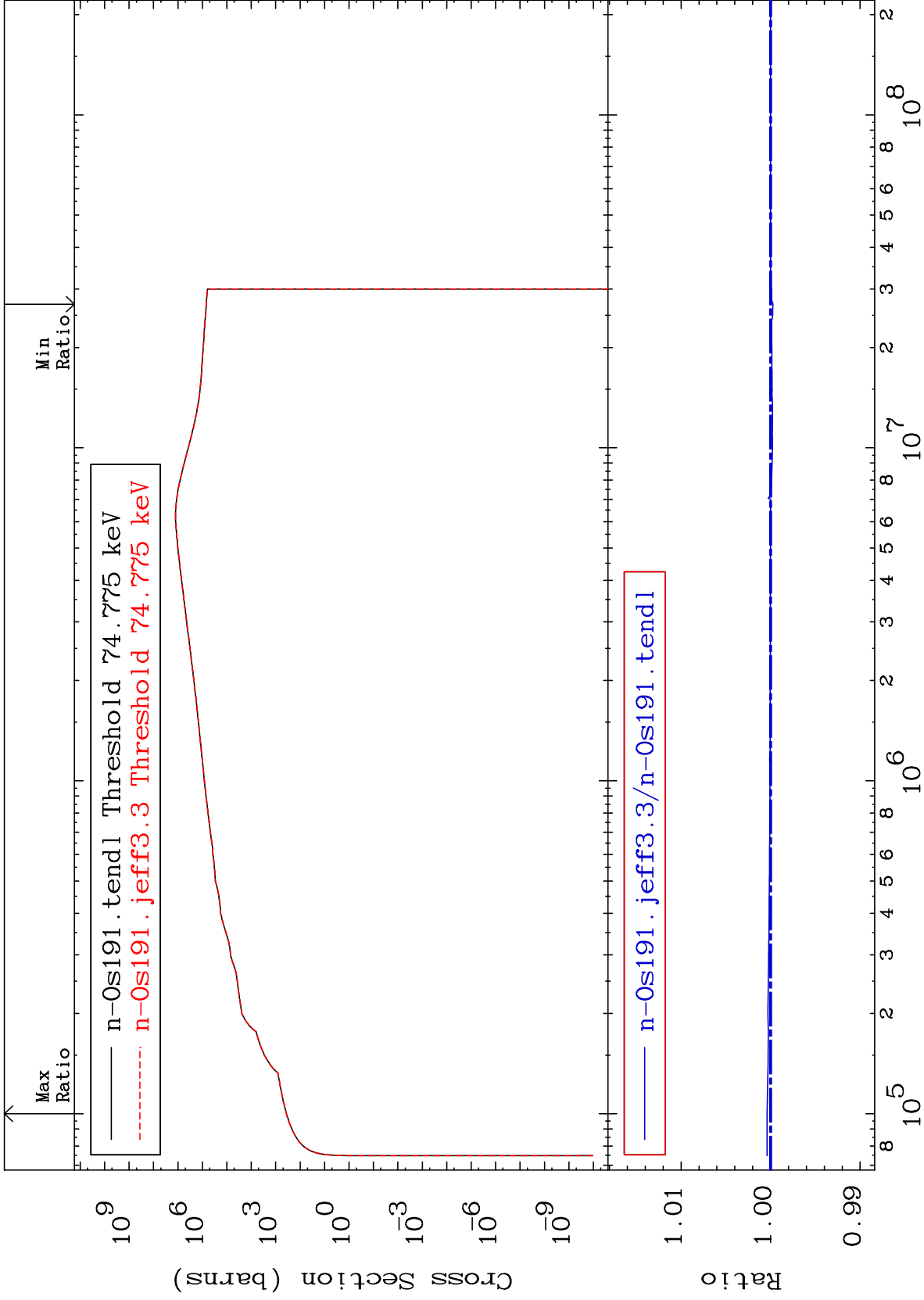
76-0s-191

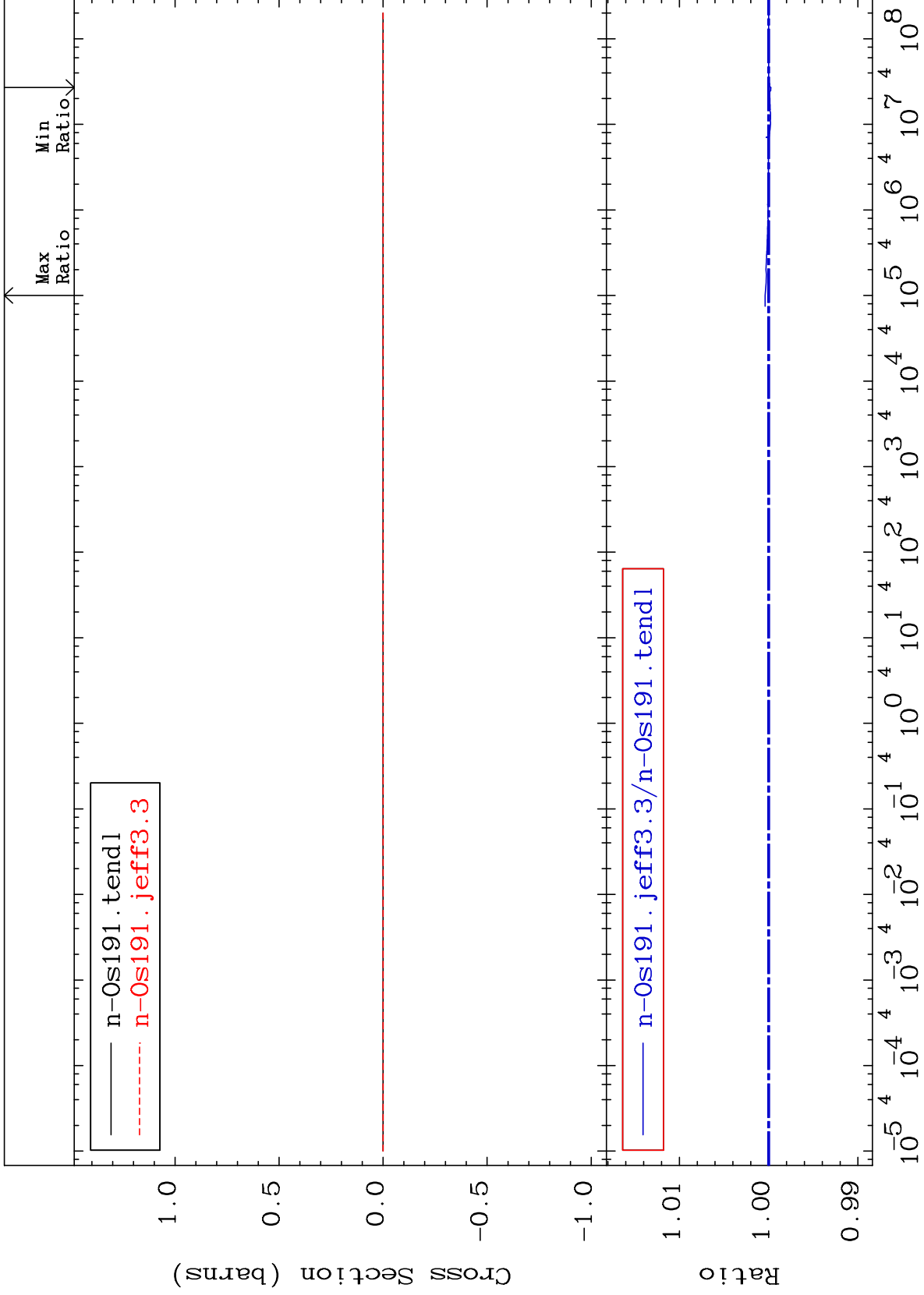


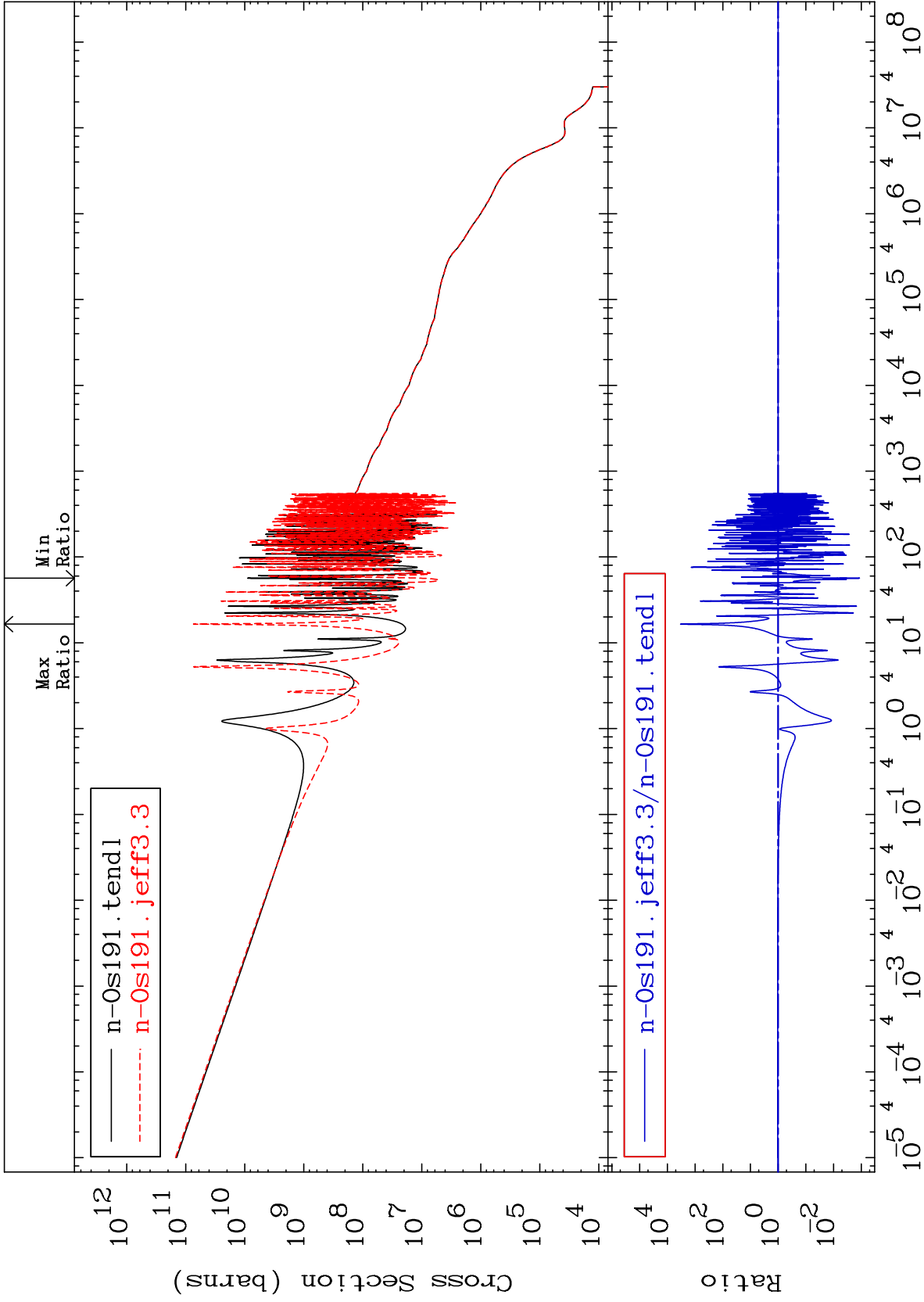
MAT 7646

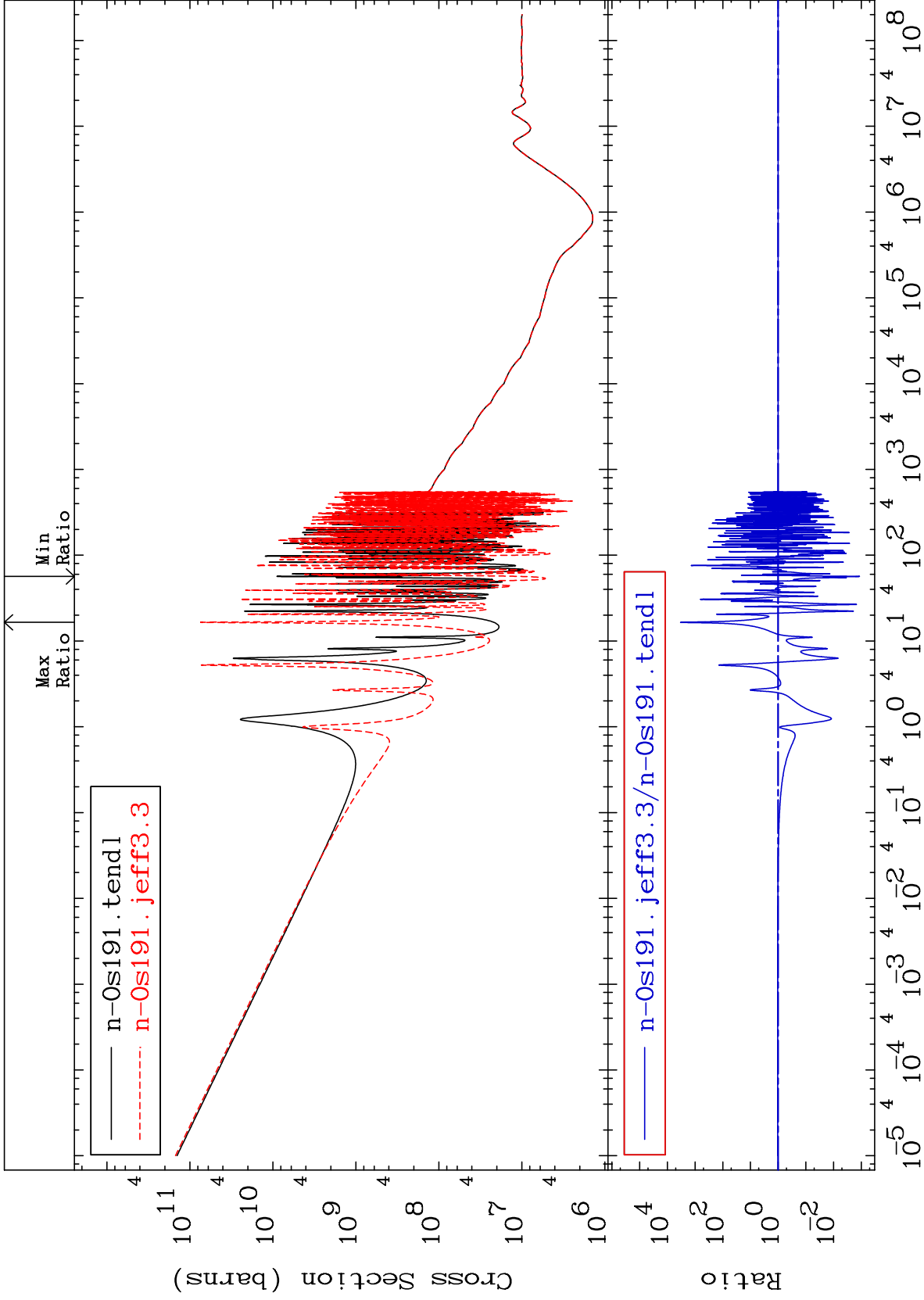
Kerma inelastic (mt51-91)
Cross Section

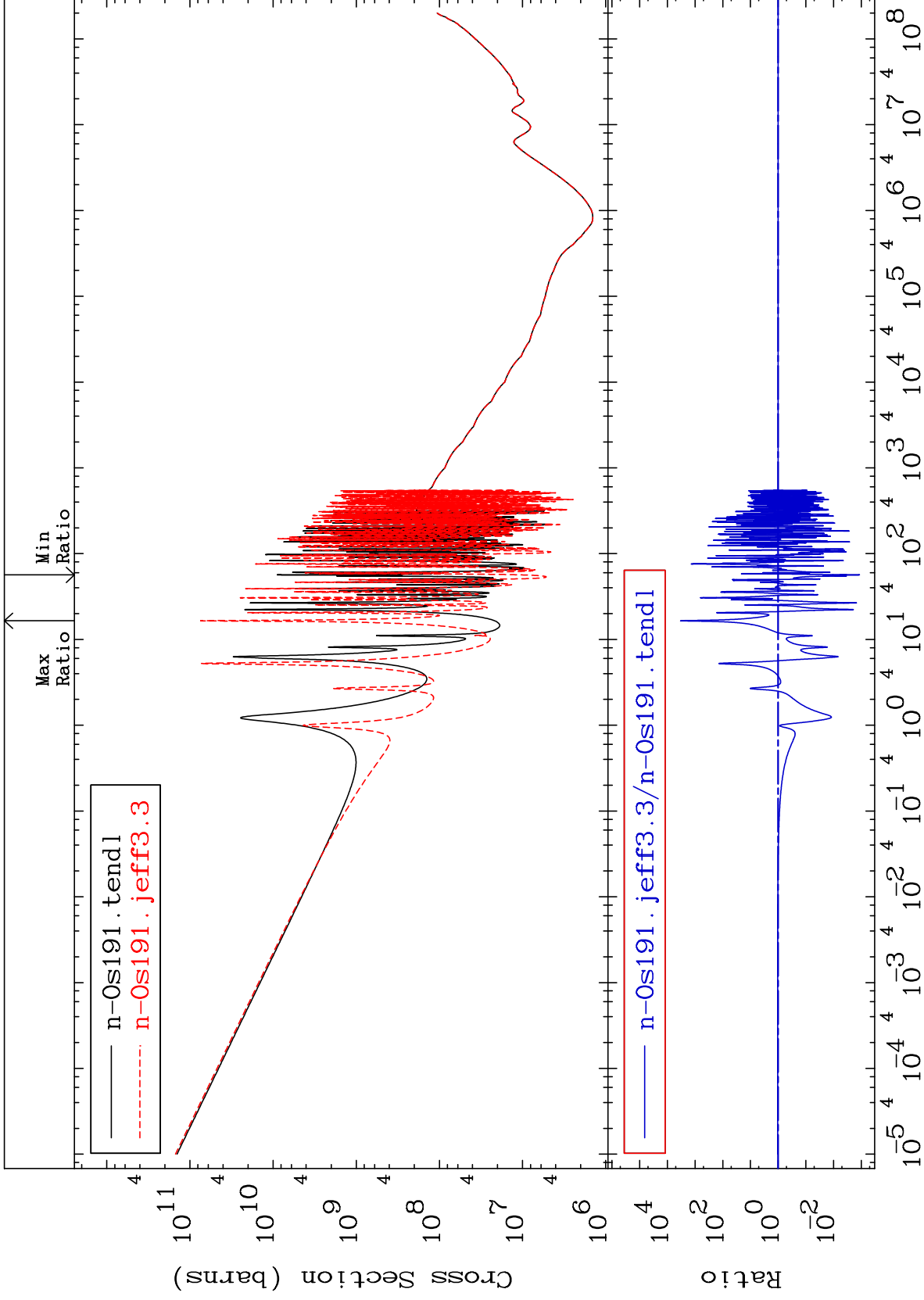
76-0s-191
-0.025 To 0.041 %

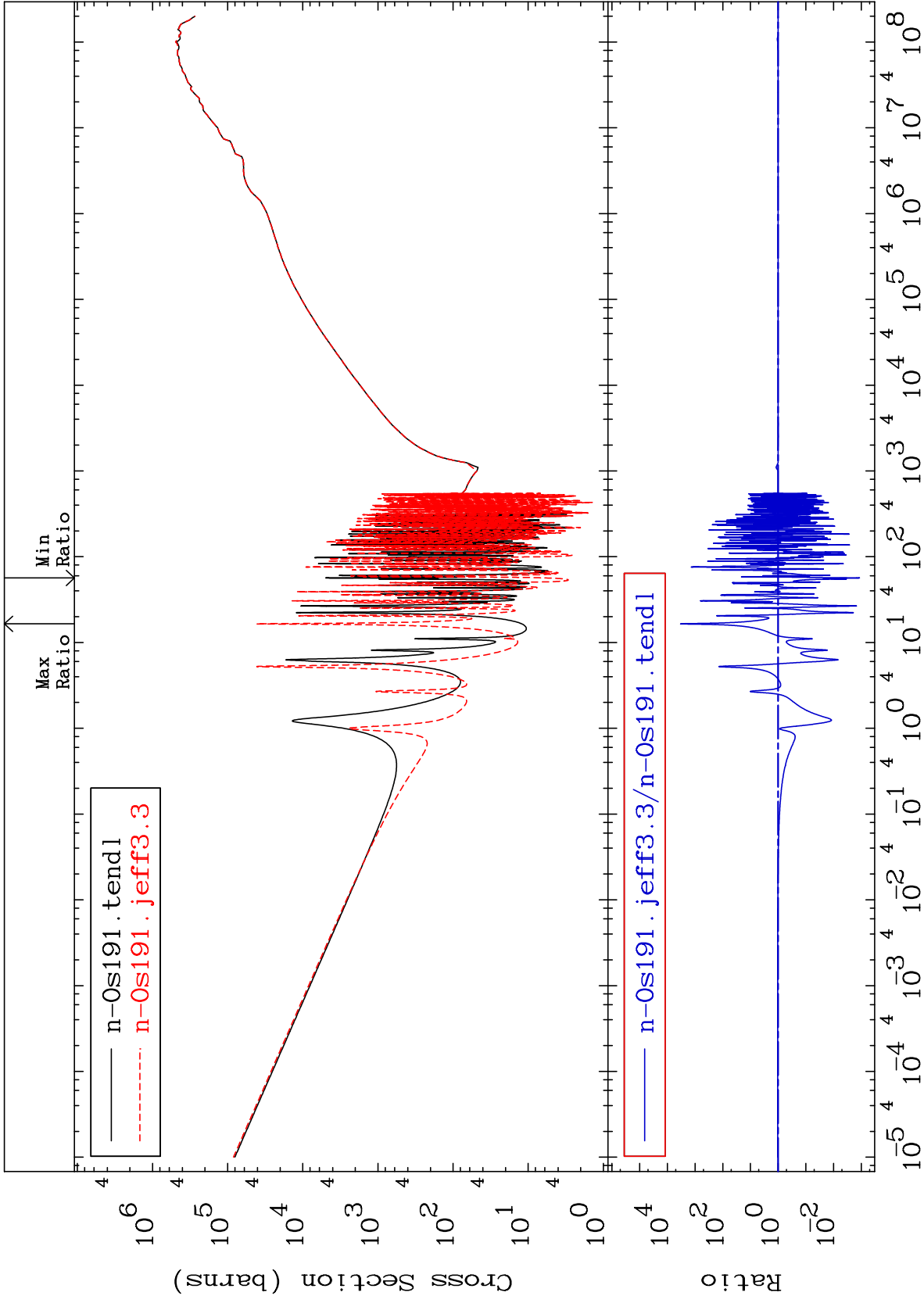


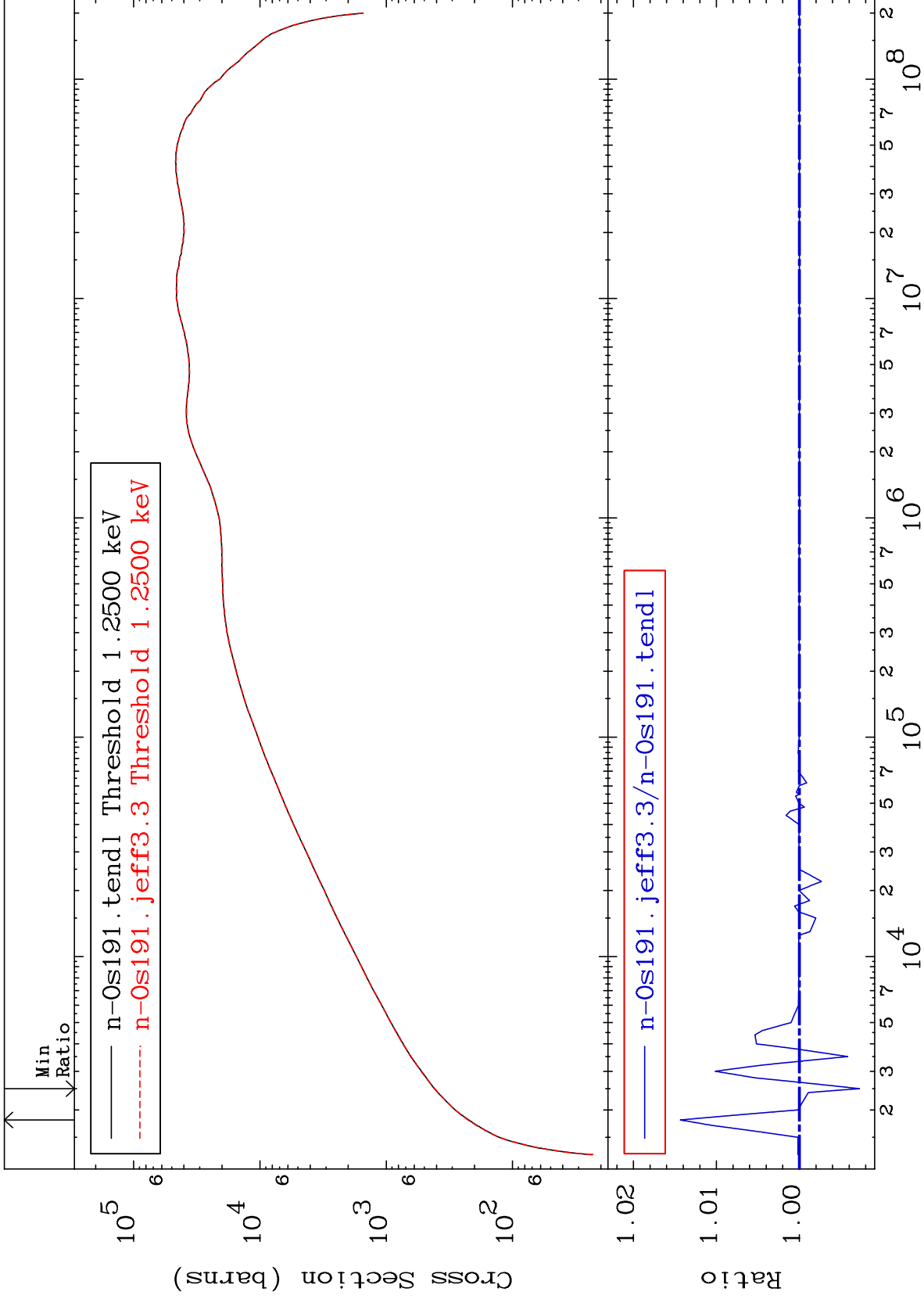








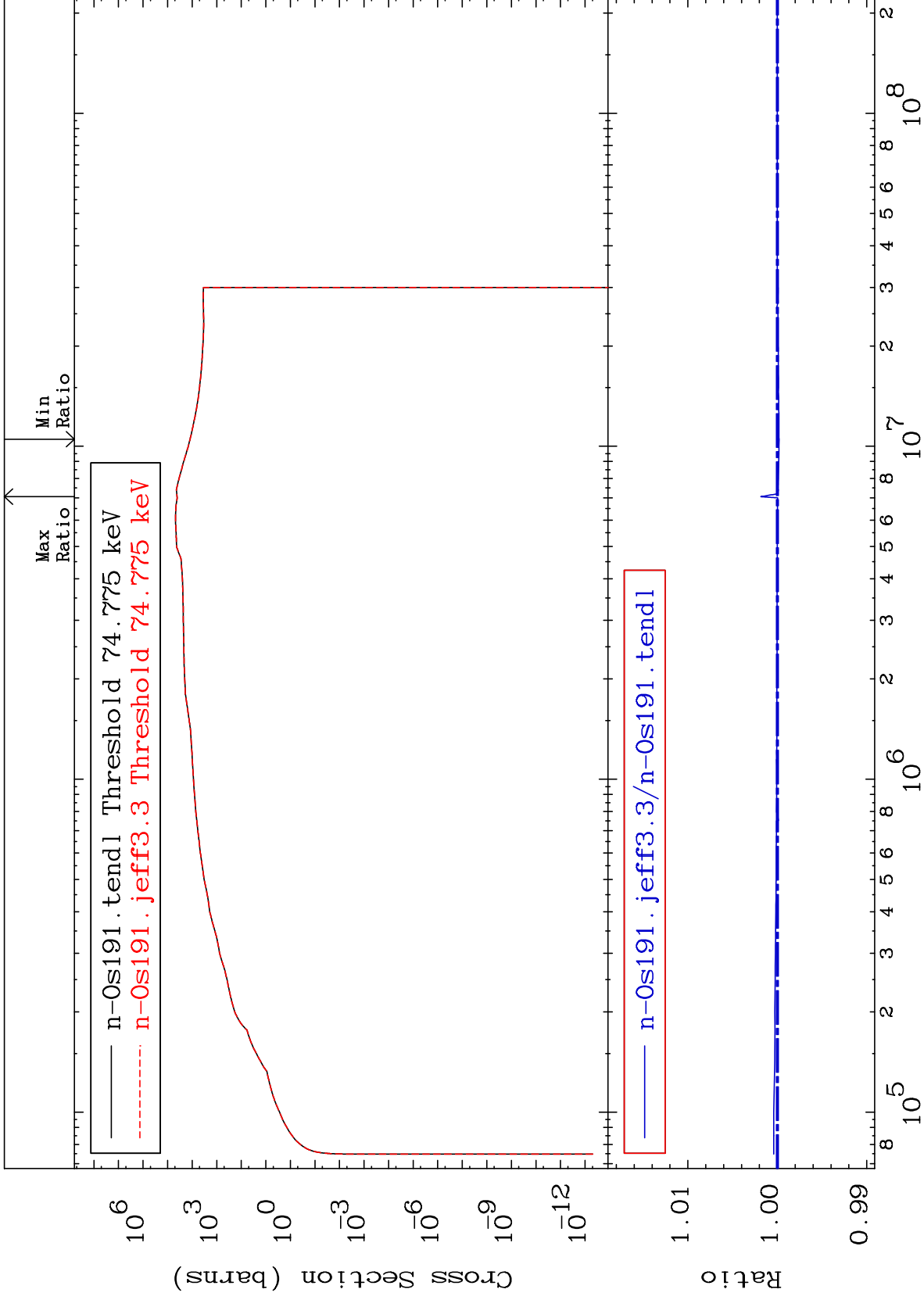




MAT 7646

Dpa inelastic (mt51-91)
Cross Section

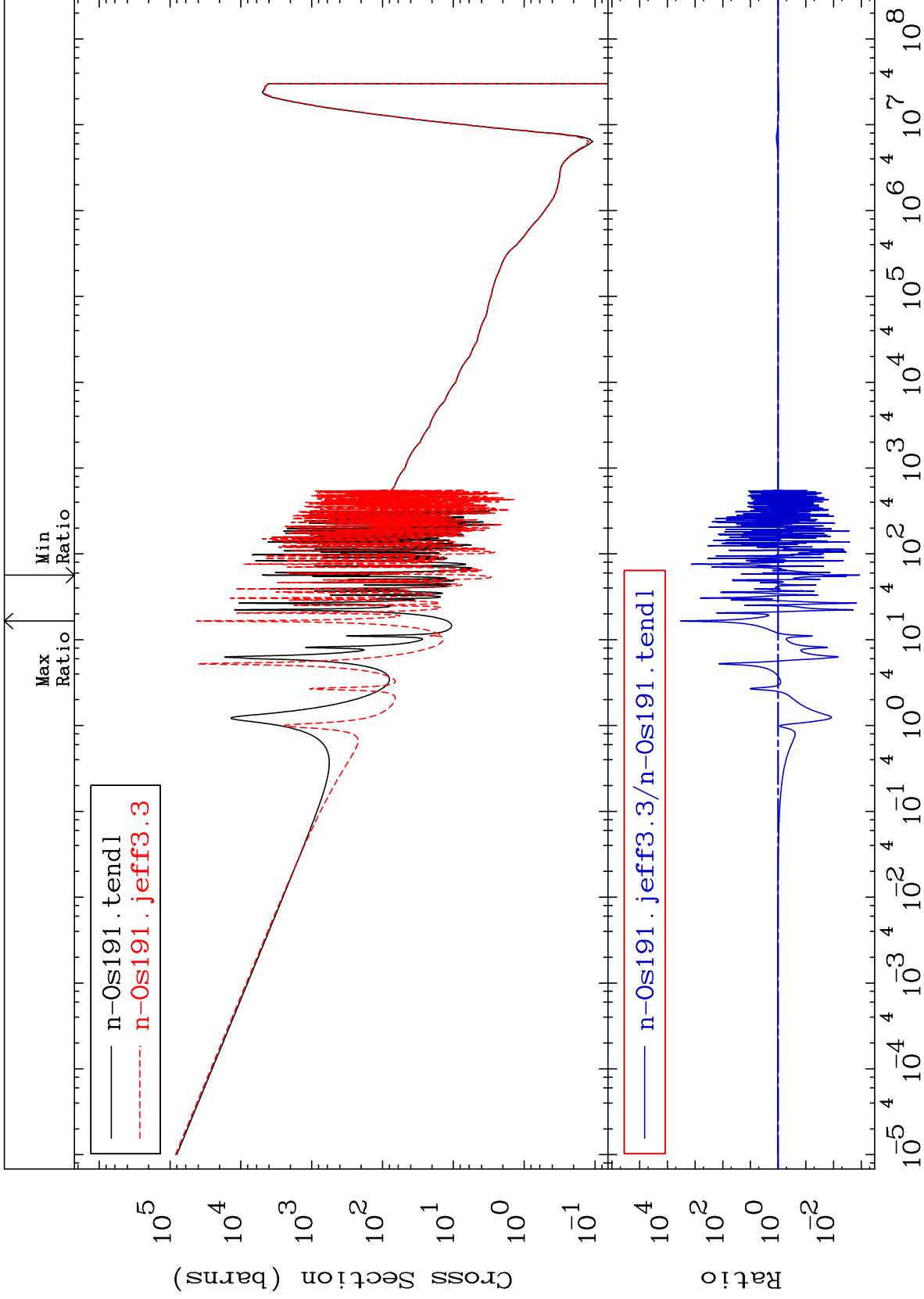
76-0s-191
-0.020 To 0.186 %



59

Incident Energy (eV)

76-0s-191

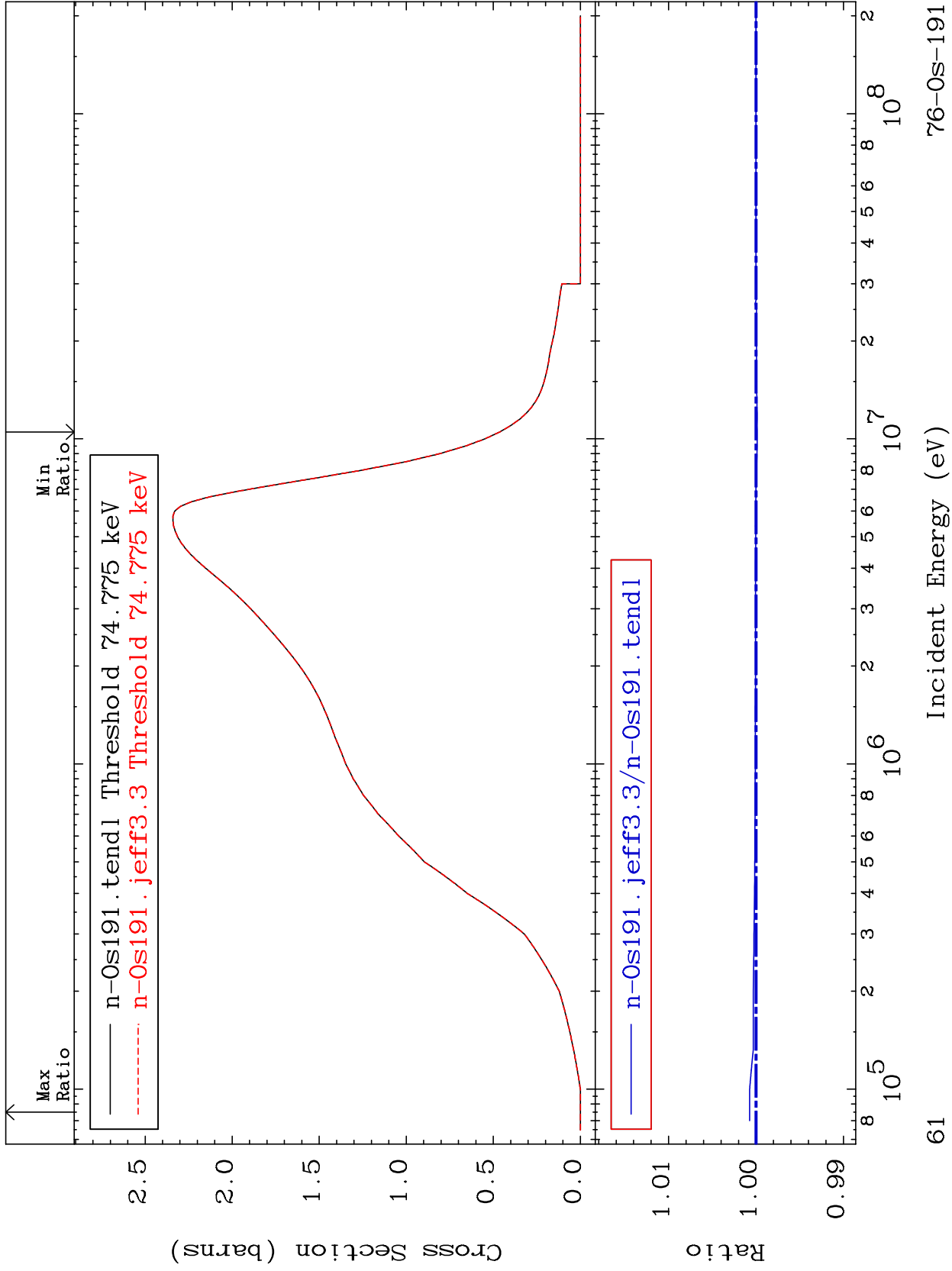


MAT 7646

Inelastic: 76-0s-191g

76-0s-191

Radionuclide Production Cross Section -0.013 To 0.073 %



61

Incident Energy (eV)

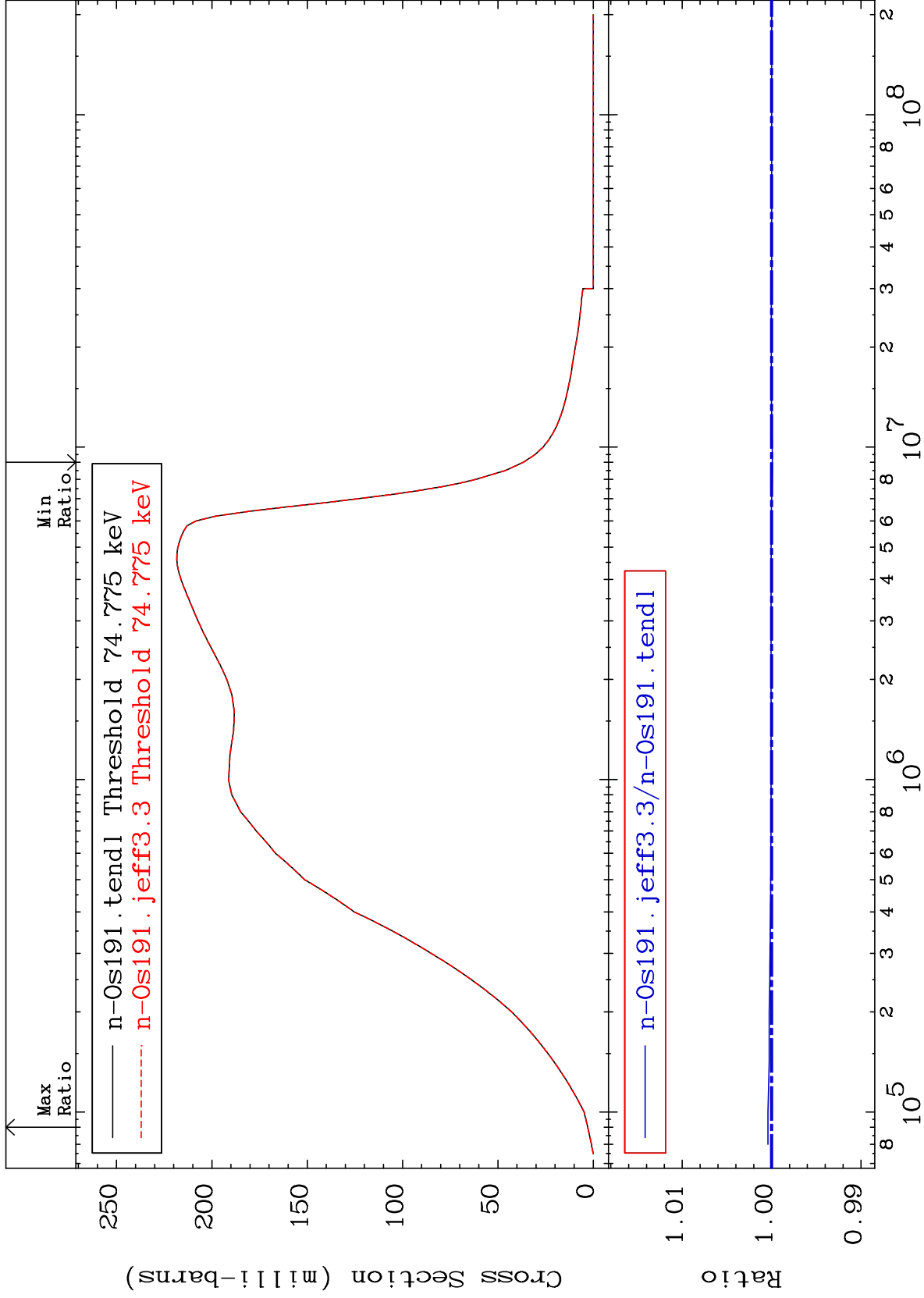
76-0s-191

MAT 7646

Inelastic: 76-Os-191m1

76-Os-191

Radionuclide Production Cross Section -0.008 To 0.041 %



62

Incident Energy (eV)

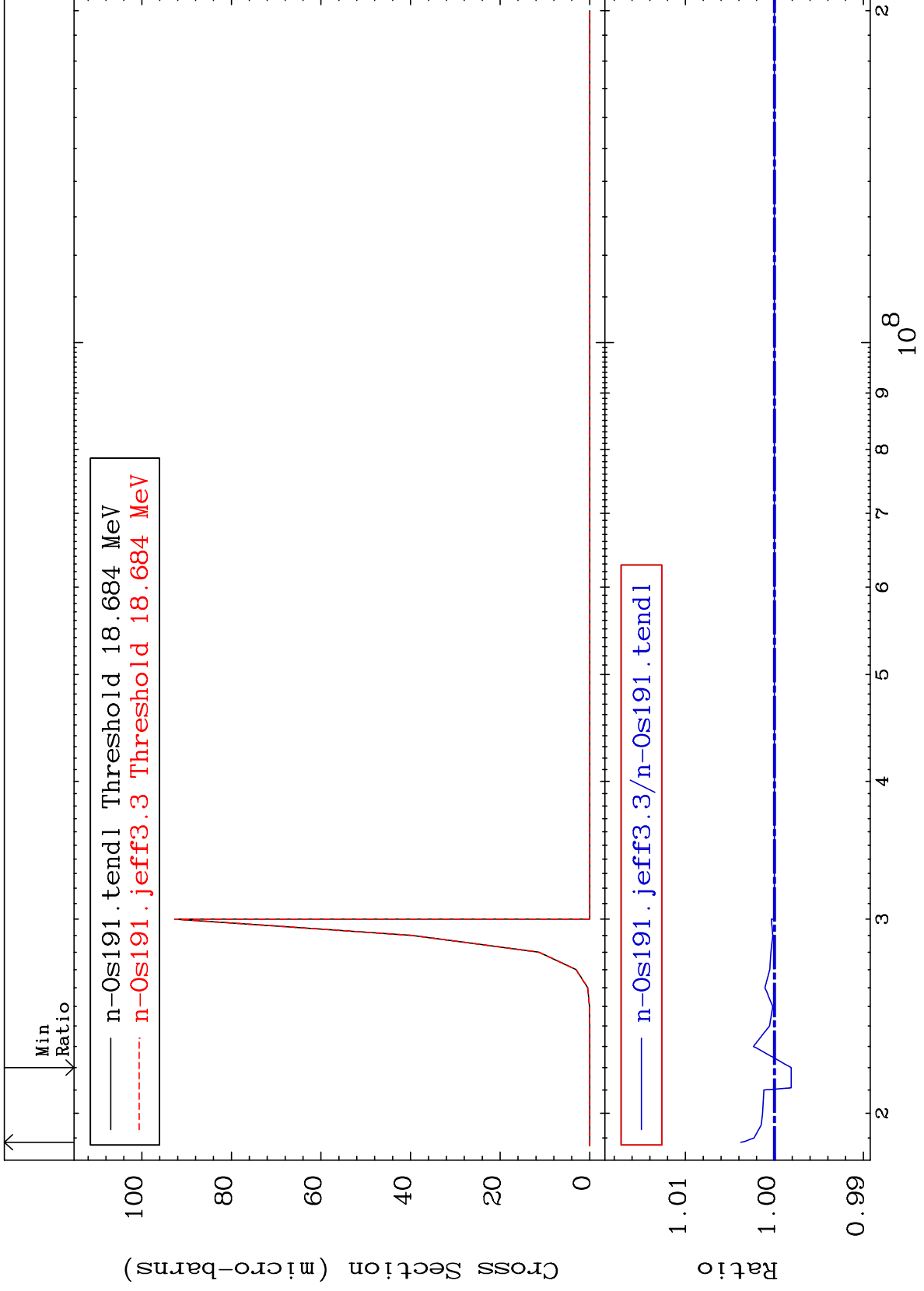
76-Os-191

MAT 7646

(n,2n) d:75-Re-188g

76-0s-191

Radionuclide Production Cross Section -0.188 To 0.378 %



63

Incident Energy (eV)

76-0s-191

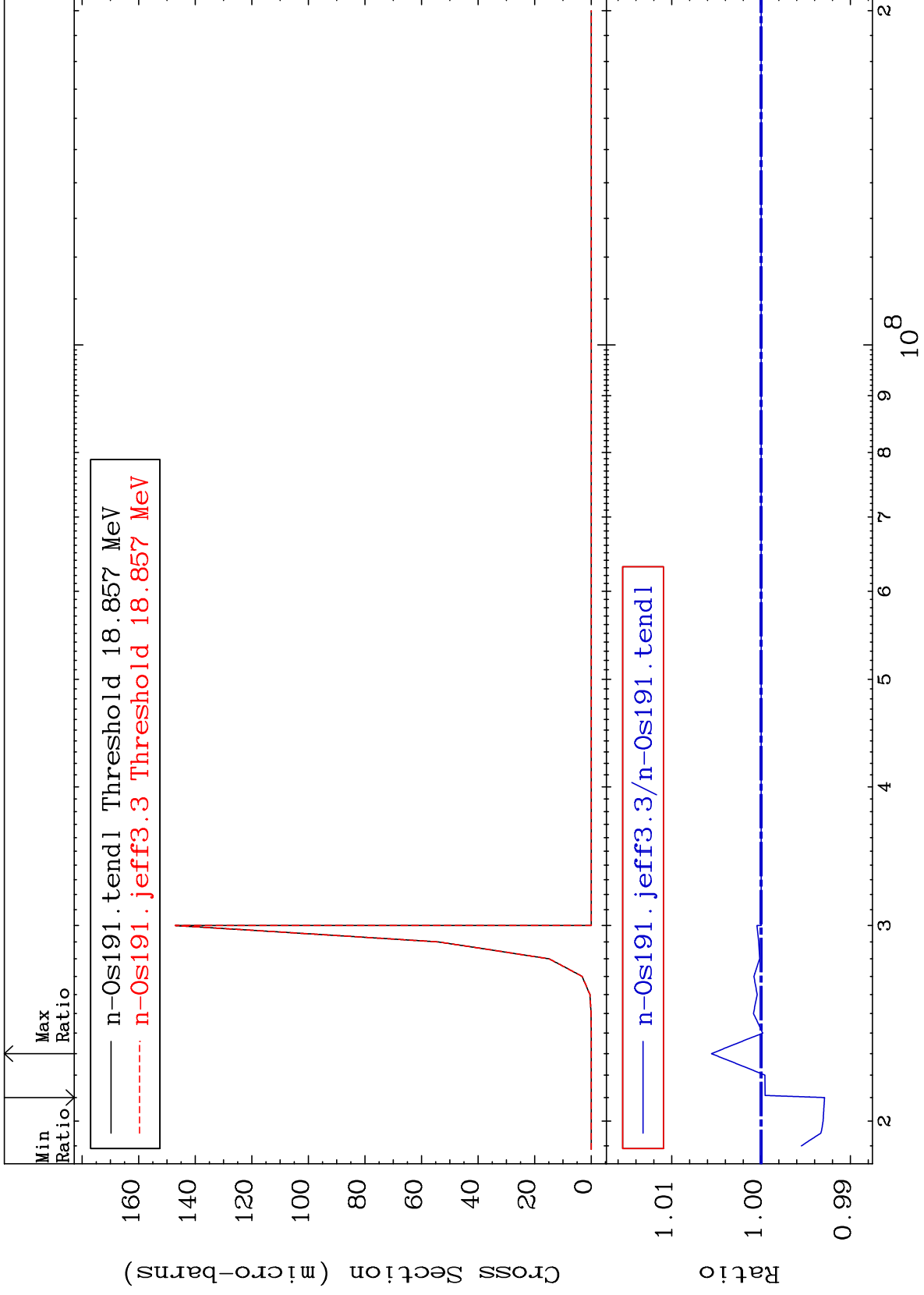
MAT 7646

(n,2n) d:75-Re-188m7

76-0s-191

Radionuclide Production Cross Section

-0.710 To 0.556 %



64

Incident Energy (eV)

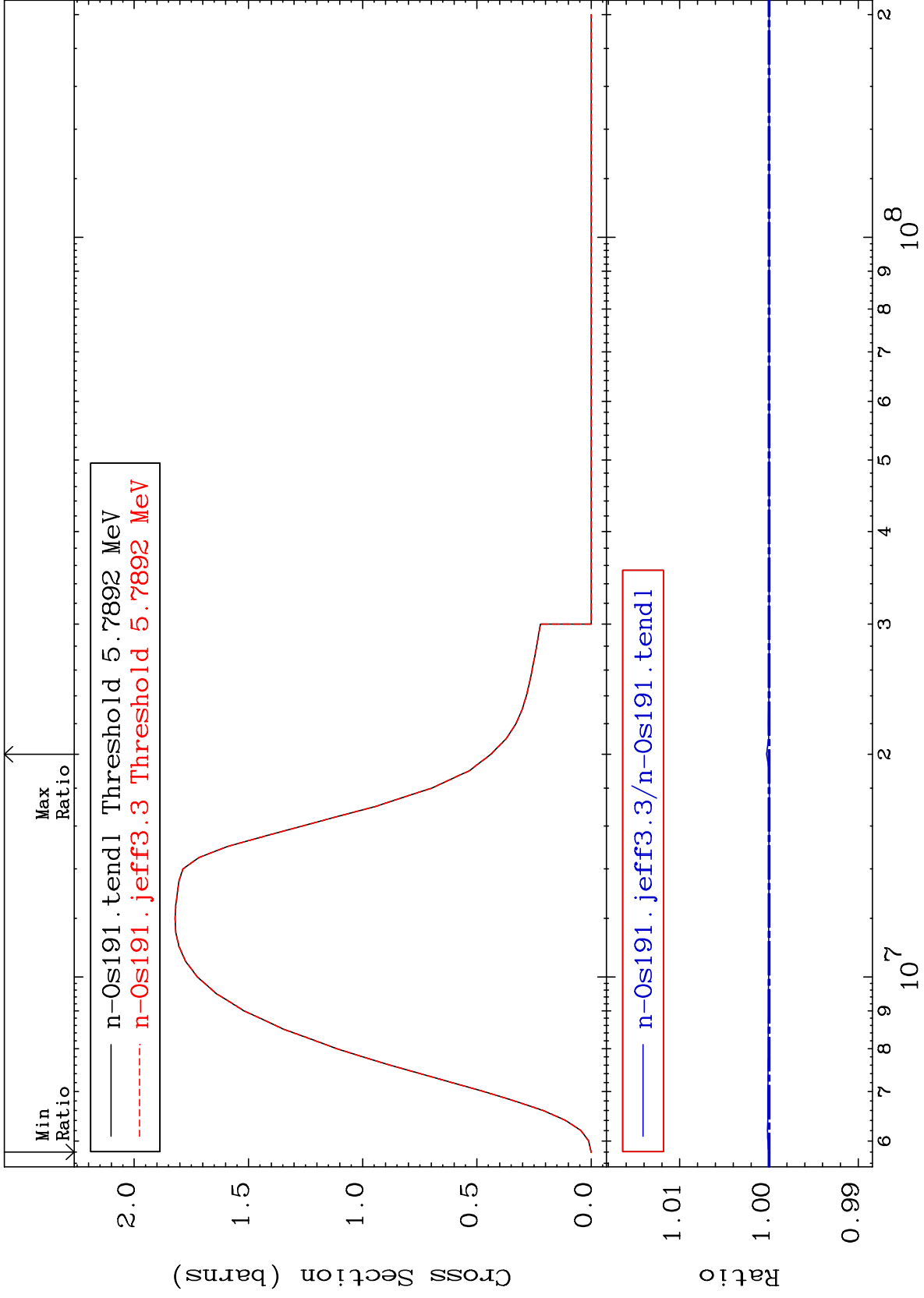
76-0s-191

MAT 7646

(n,2n):76-0s-190g

76-0s-191

Radionuclide Production Cross Section -0.004 To 0.028 %

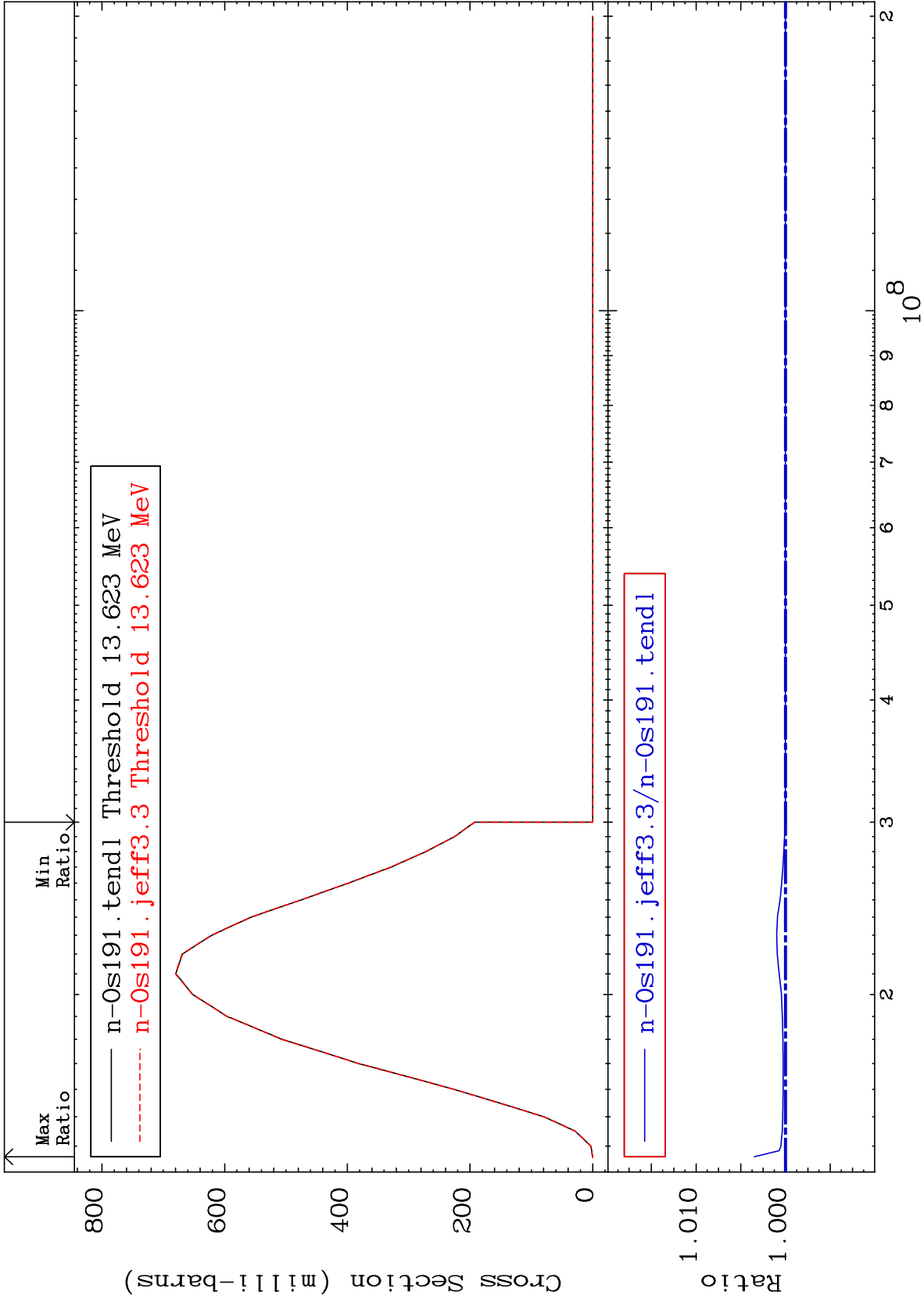


65

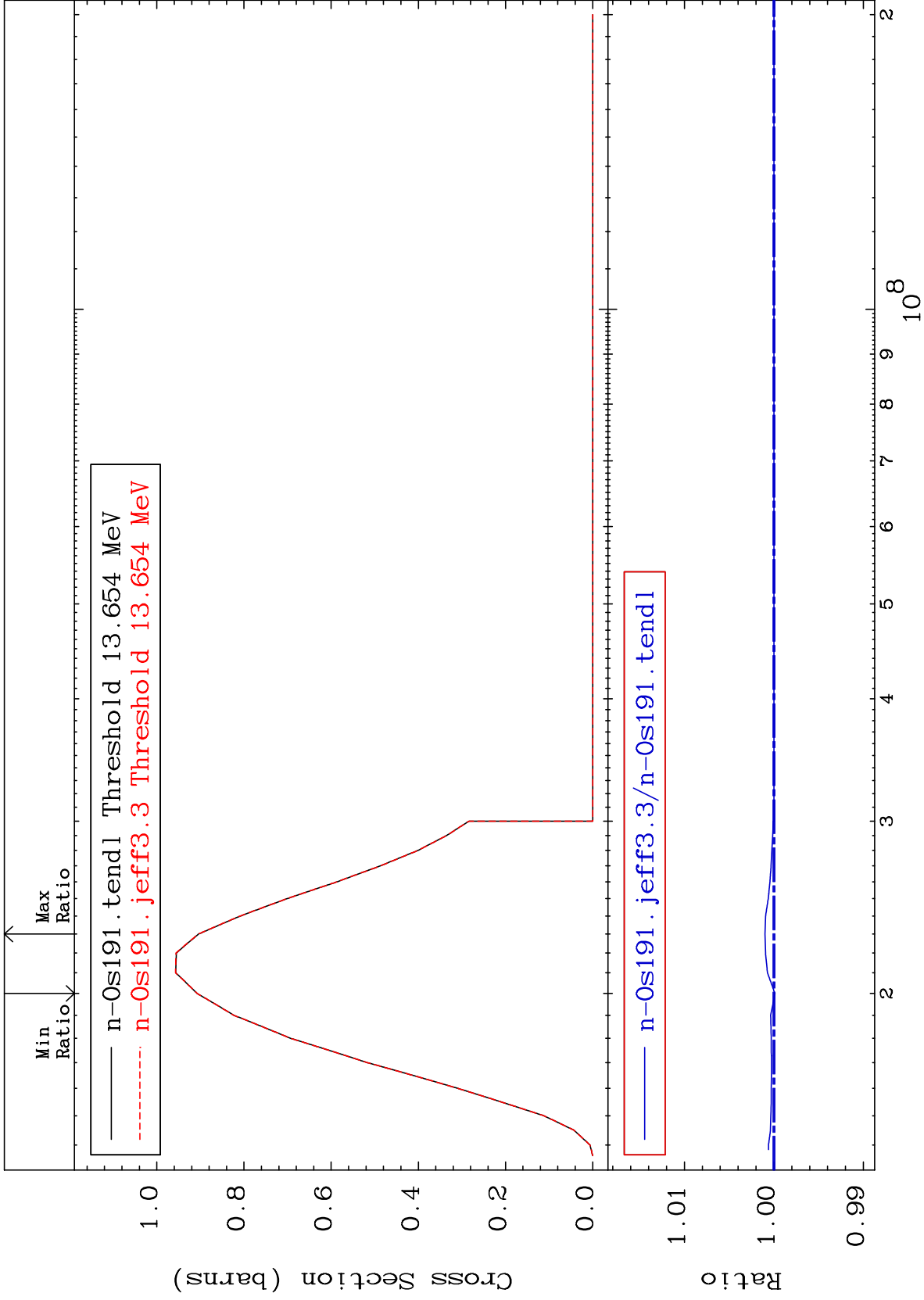
Incident Energy (eV)

76-0s-191

Radionuclide Production Cross Section 0.000 To 0.349 %



Radionuclide Production Cross Section -0.011 To 0.101 %



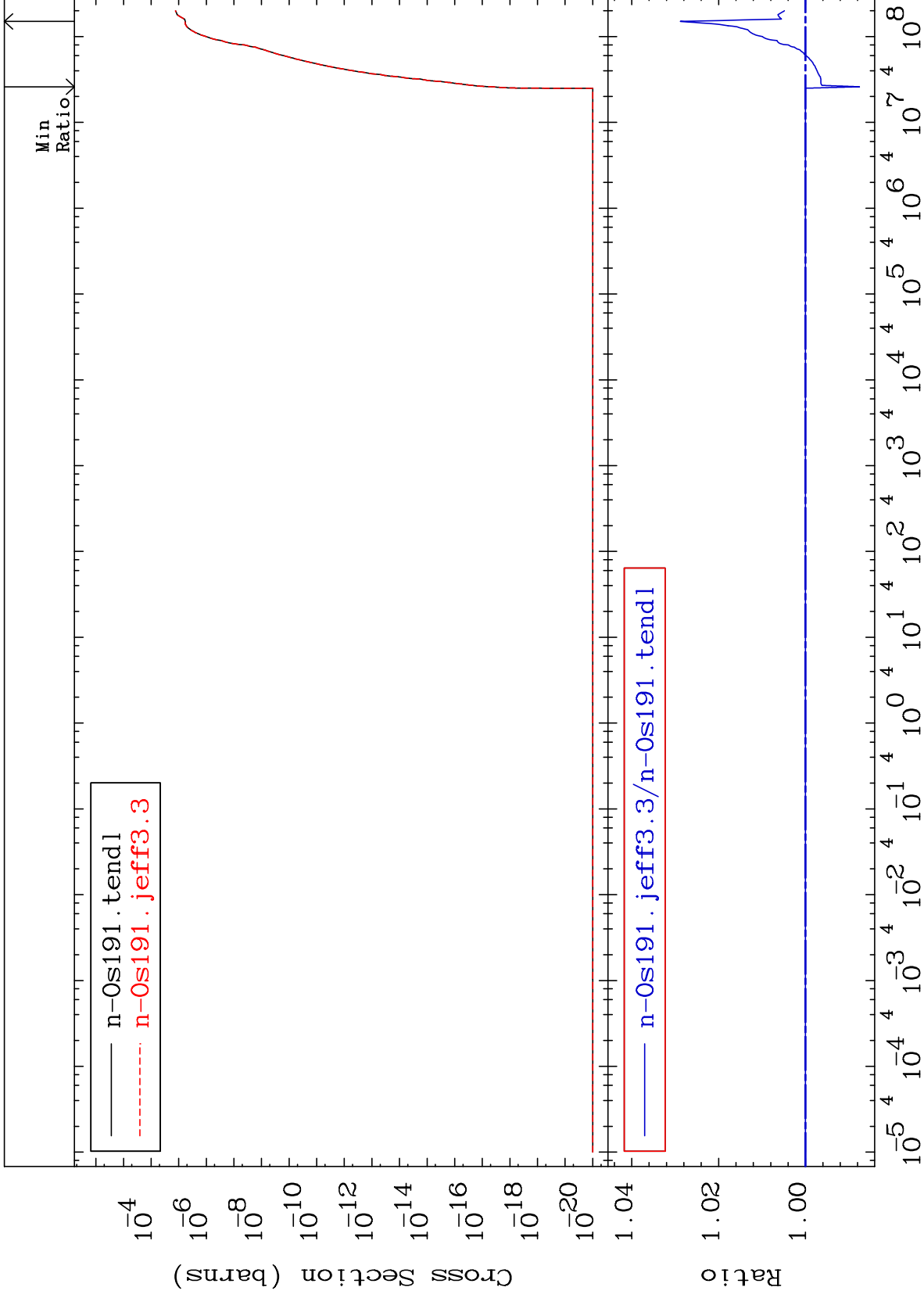
MAT 7646

Fission: Photon

76-0s-191

Radionuclide Production Cross Section

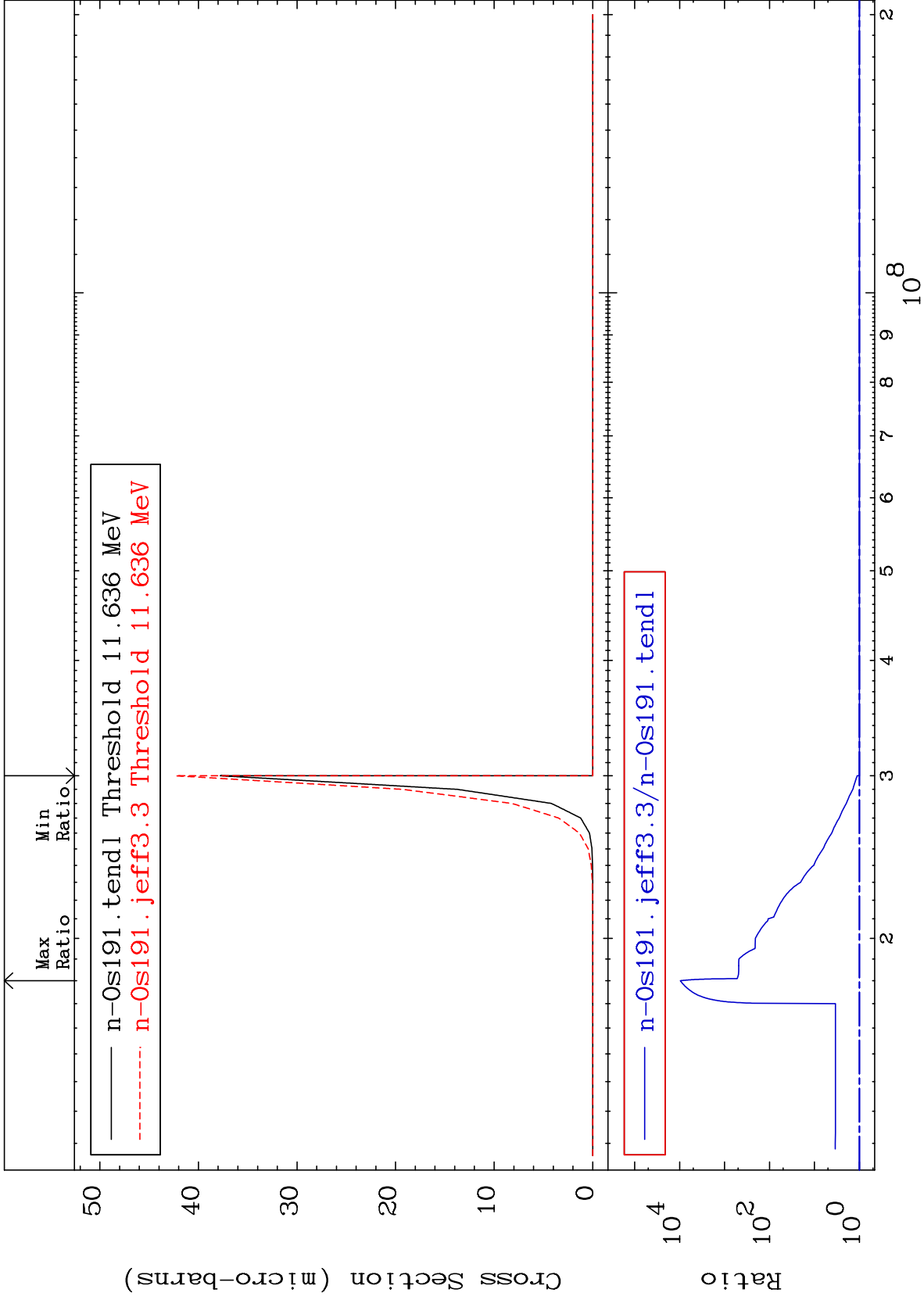
-1.242 To 2.880 %



68

Incident Energy (eV)

76-0s-191

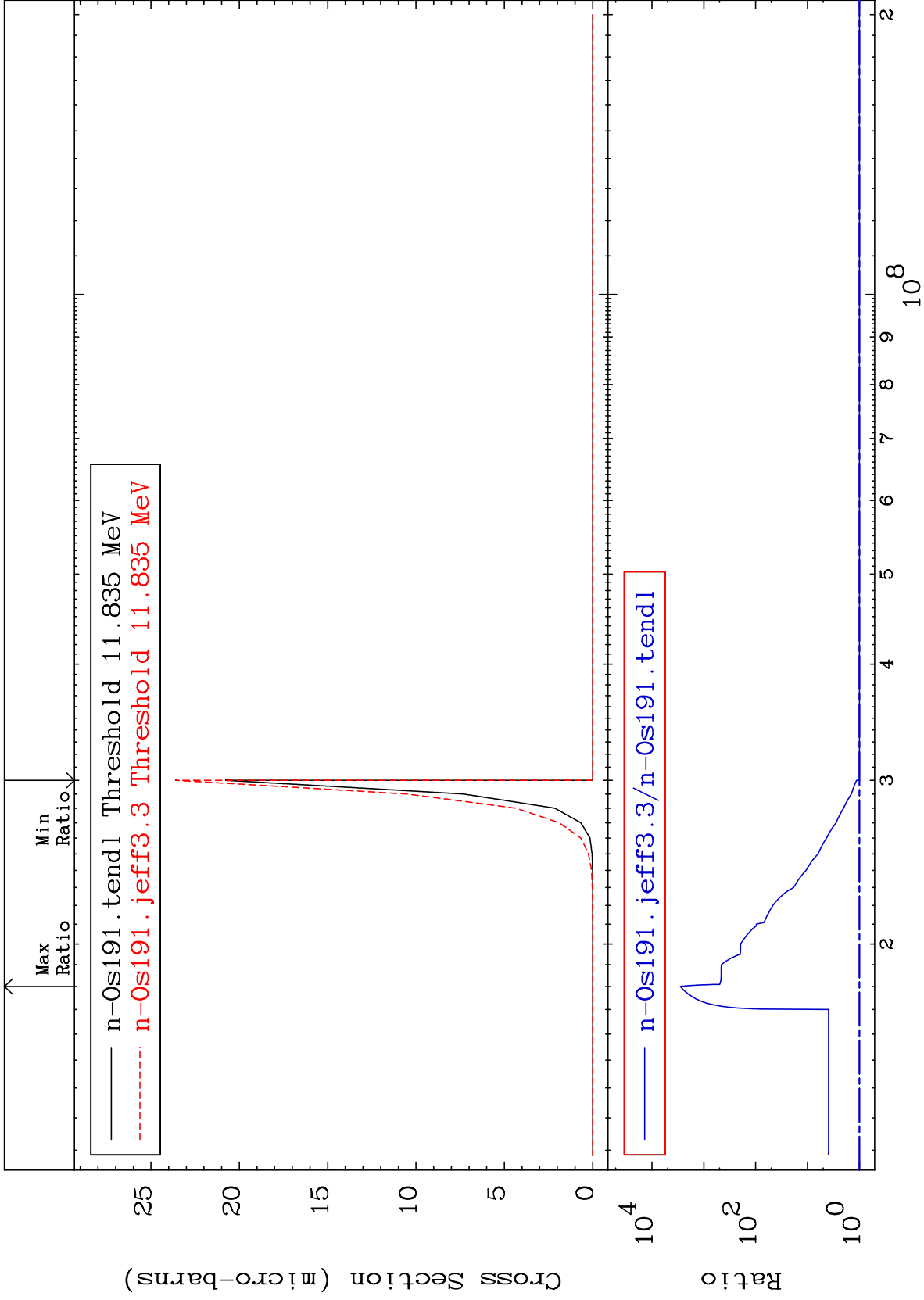


MAT 7646

(n,3n) α :74-W -185m6

76-0s-191

Radionuclide Production Cross Section 0.000 To 9999. %



70

Incident Energy (eV)

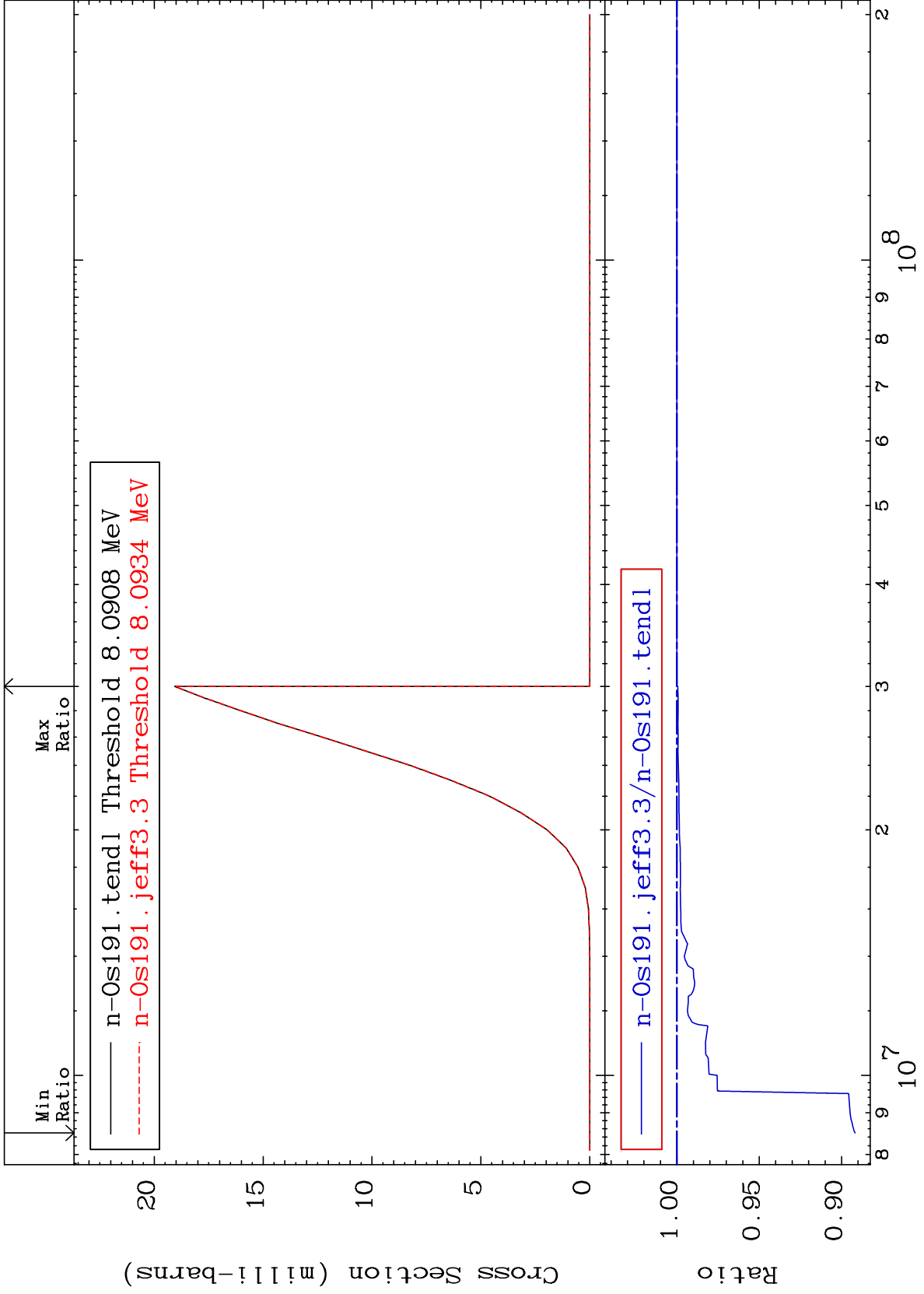
76-0s-191

MAT 7646

(n, n') p: 75-Re-190g

76-0s-191

Radionuclide Production Cross Section -10.82 To 0.000 %



71

Incident Energy (eV)

76-0s-191

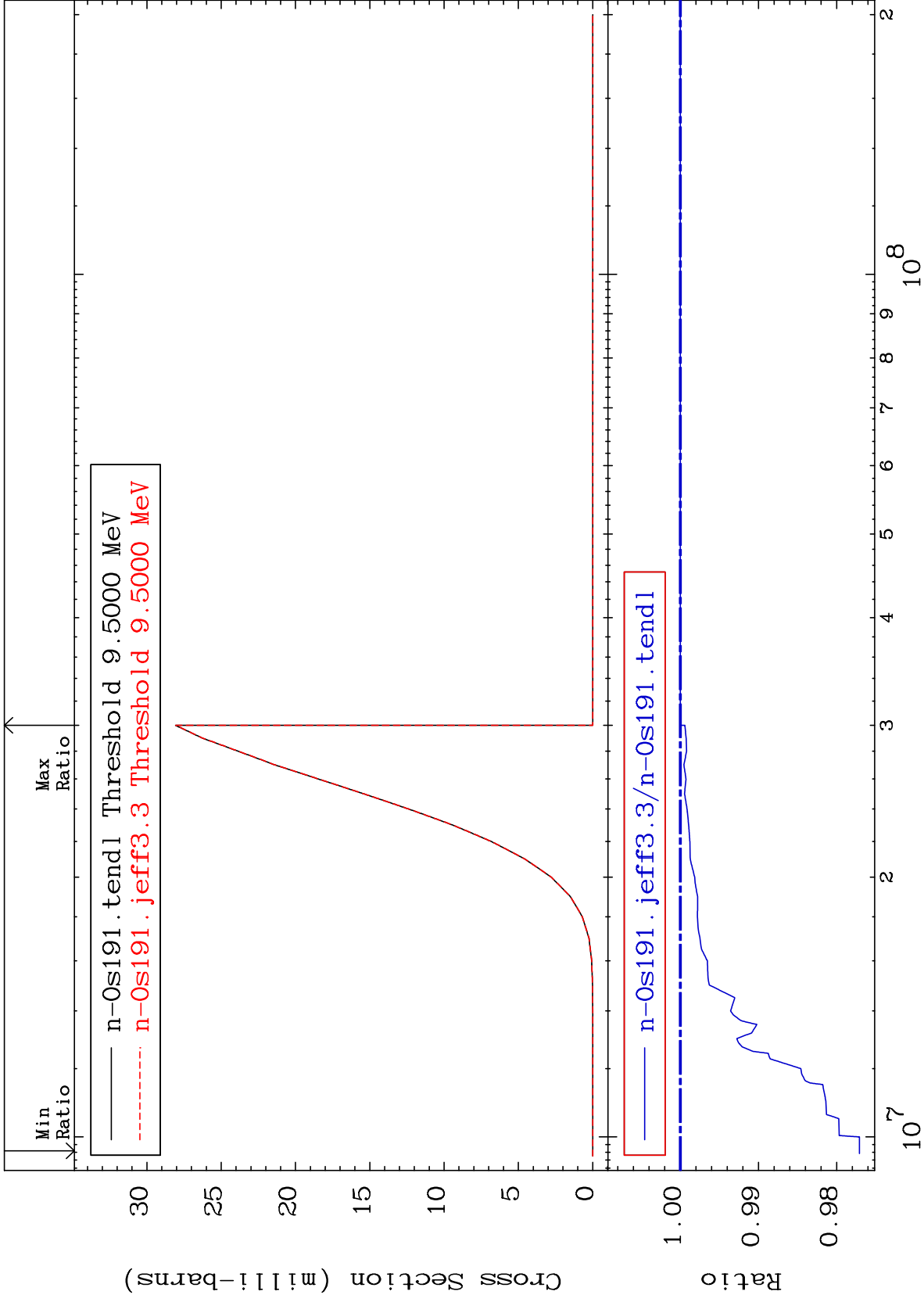
MAT 7646

(n, n') p: 75-Re-190m3

76-0s-191

Radionuclide Production Cross Section

-2.290 To 0.000 %

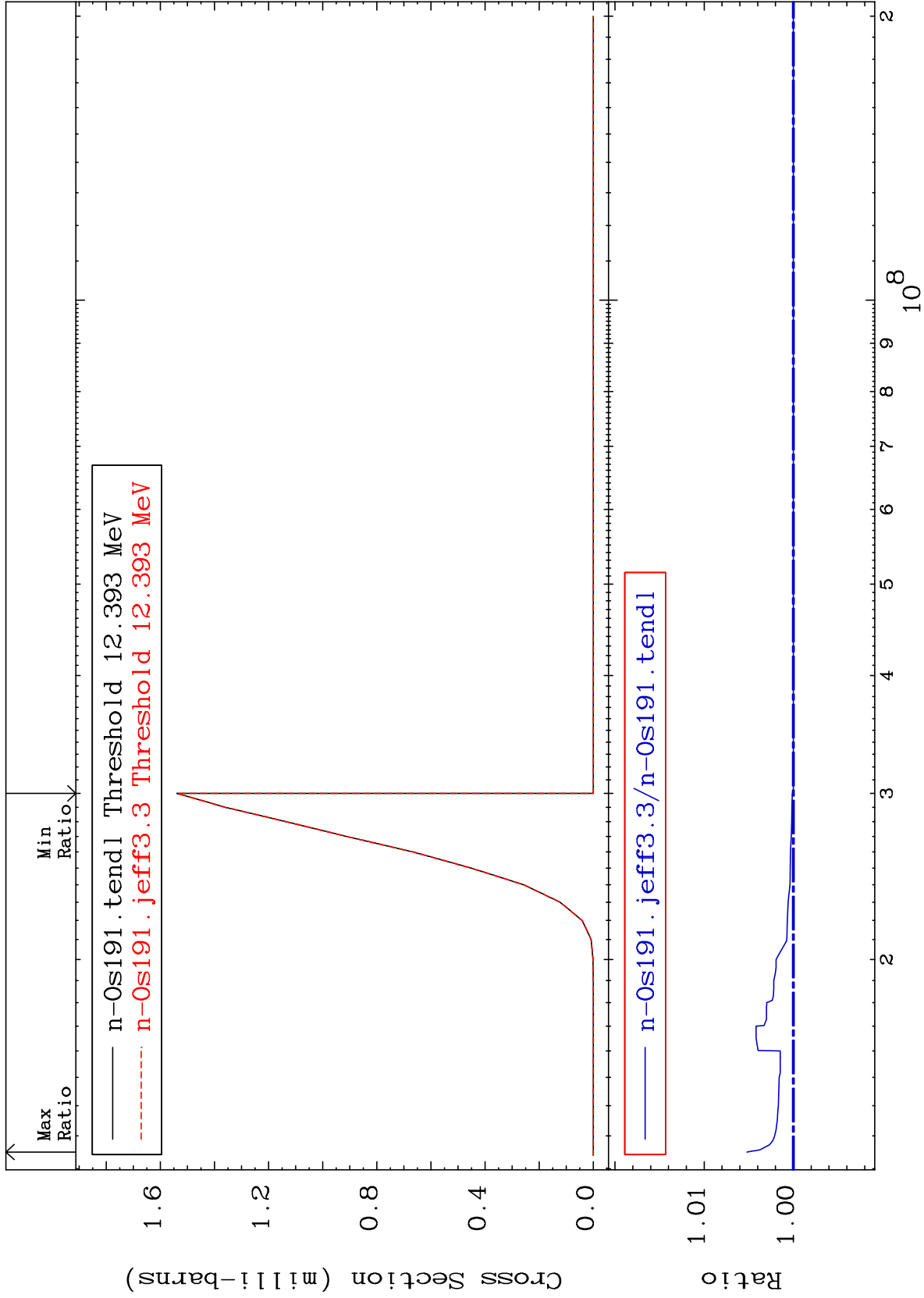


72

Incident Energy (eV)

76-0s-191

Radionuclide Production Cross Section 0.000 To 0.520 %

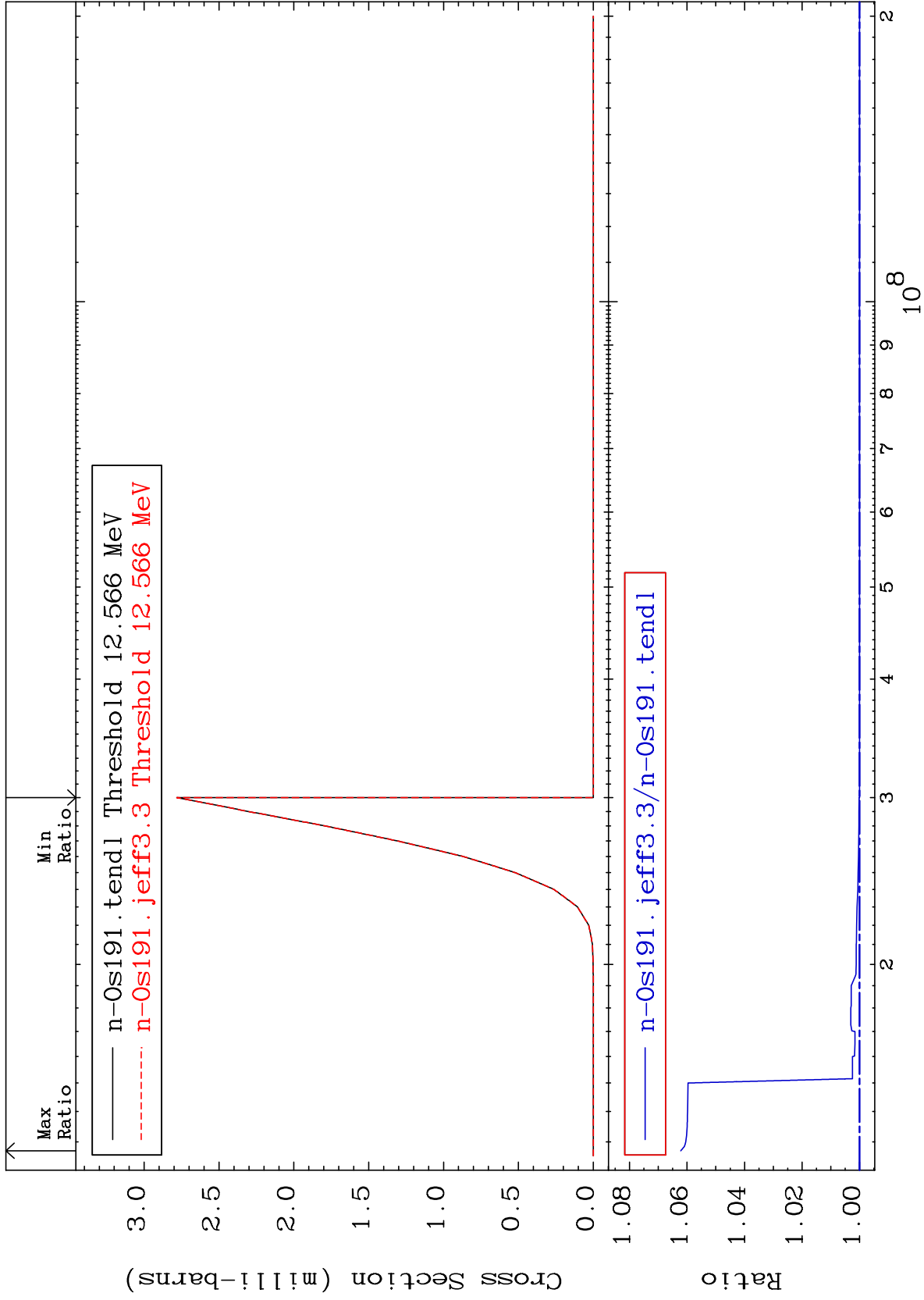


MAT 7646

(n, n') t: 75-Re-188m7

76-0s-191

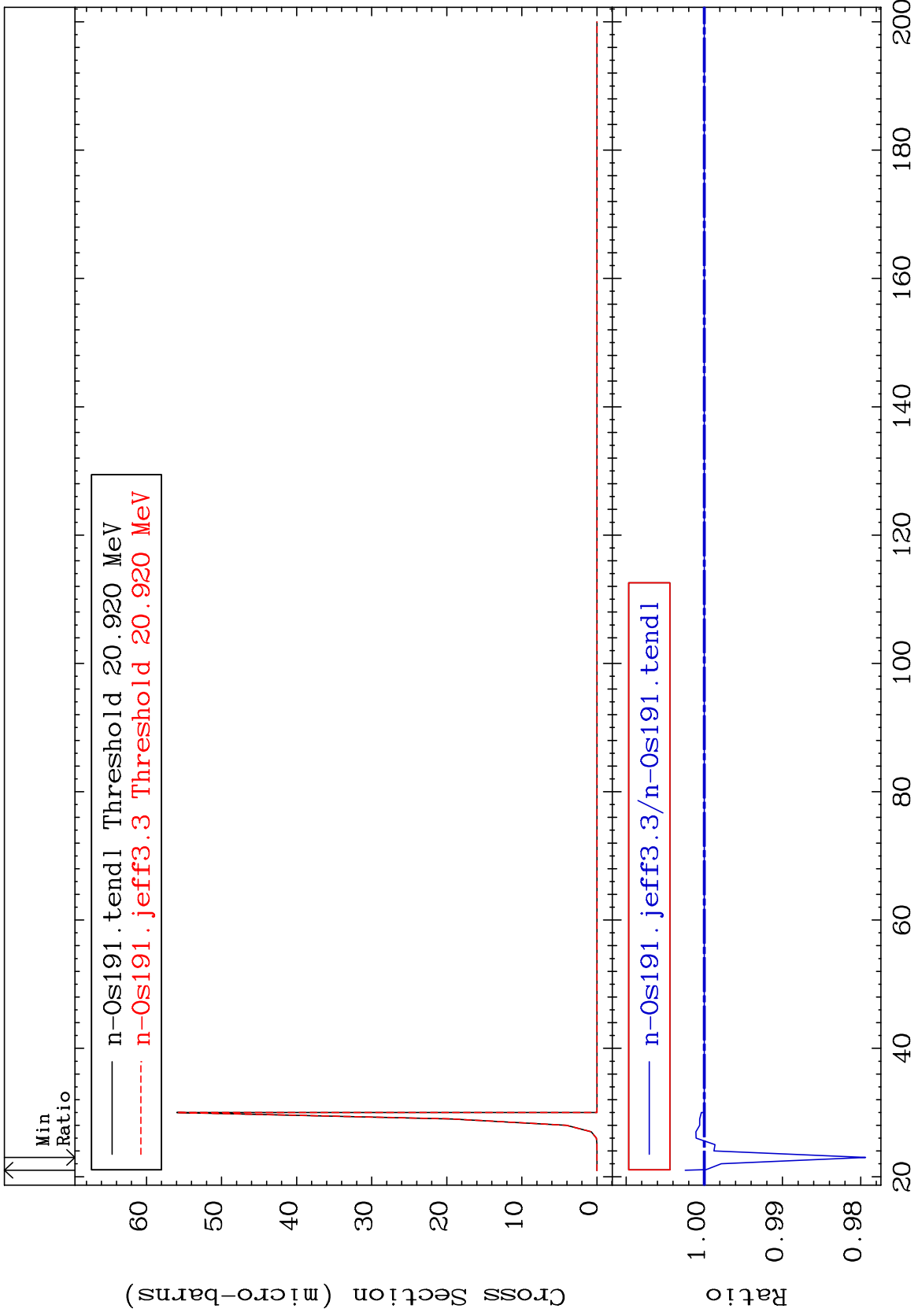
Radionuclide Production Cross Section 0.000 To 6.218 %



74

Incident Energy (eV)

76-0s-191

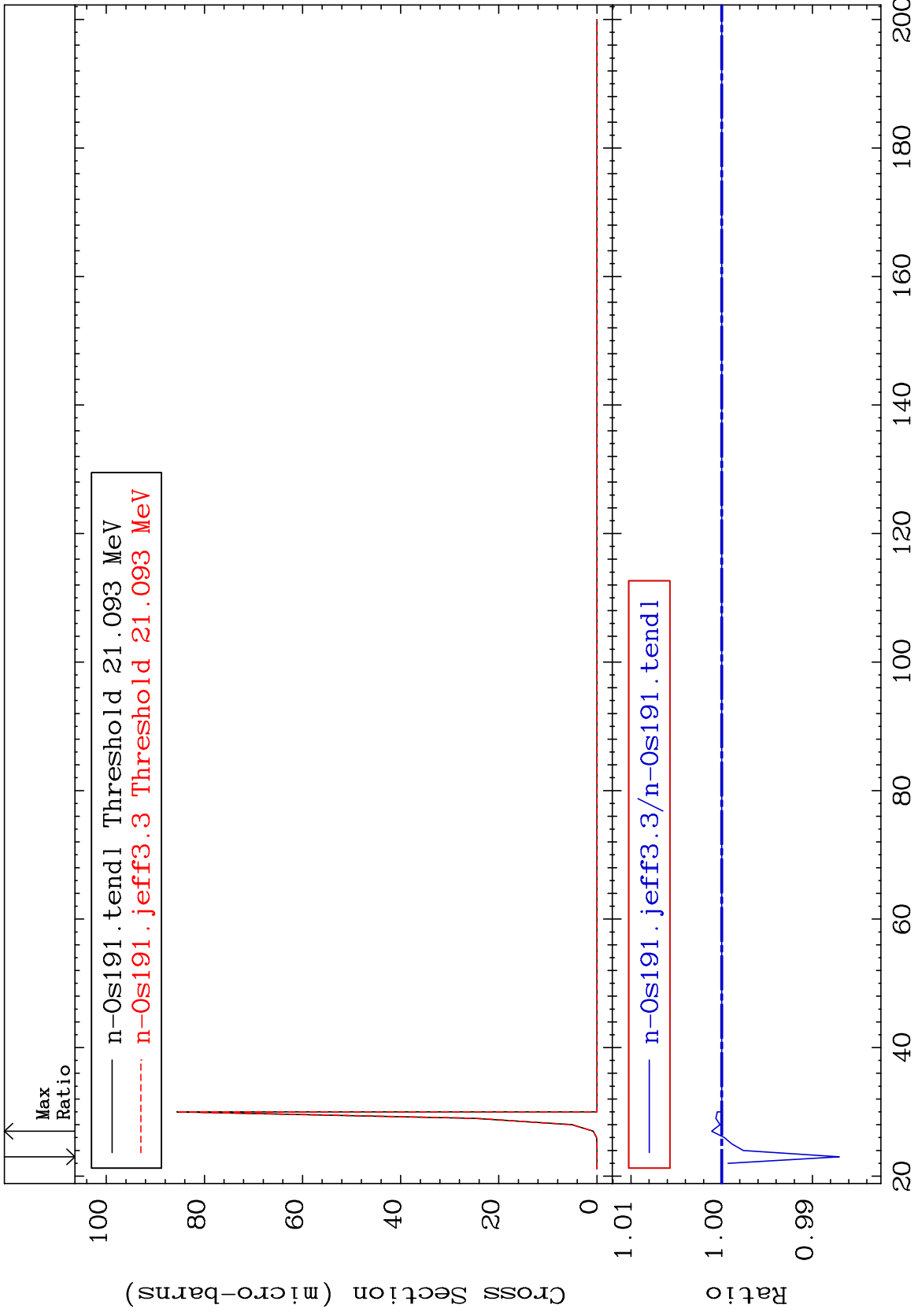


MAT 7646

(n,3n) p:75-Re-188m7

76-0s-191

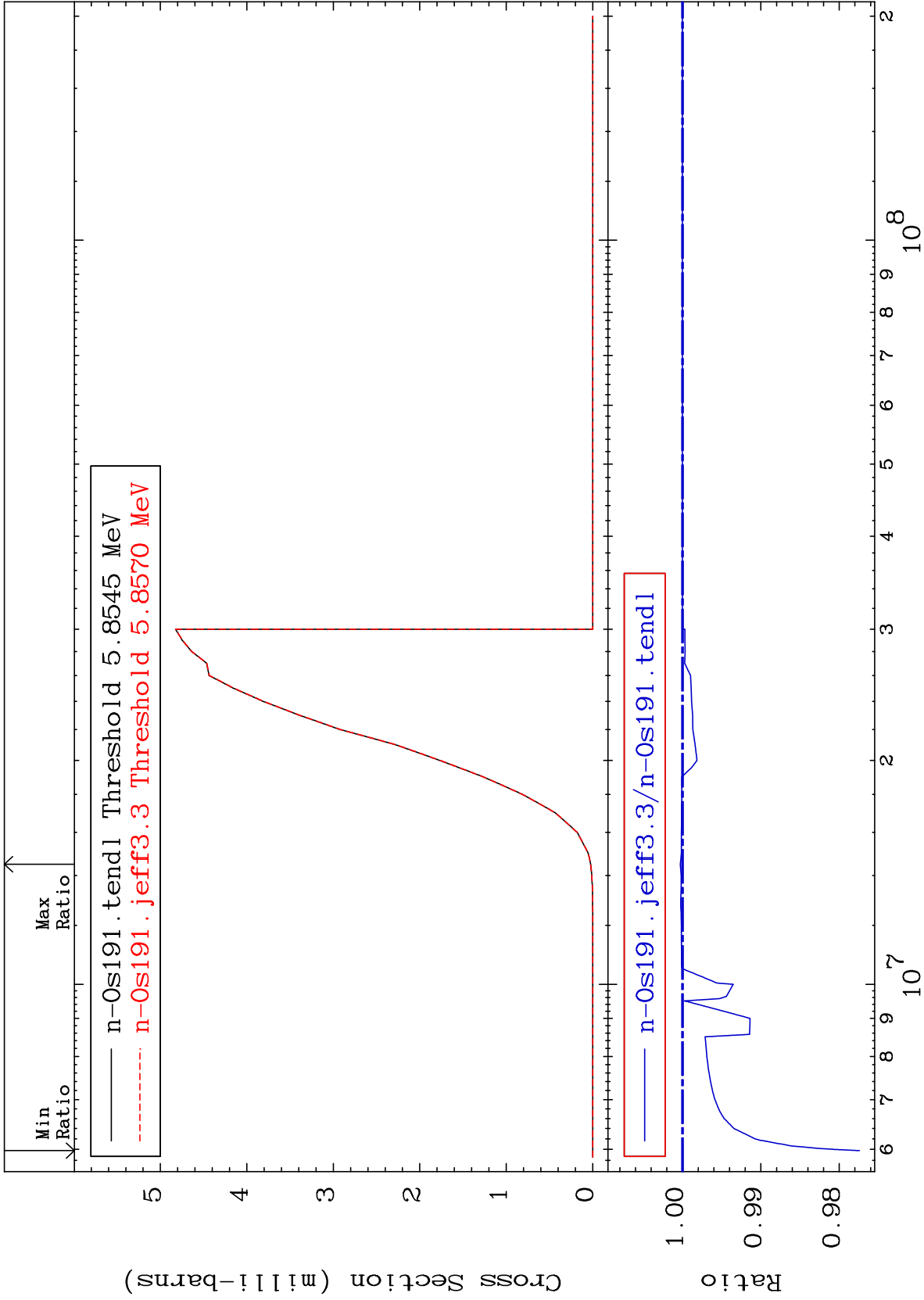
Radionuclide Production Cross Section -1.296 To 0.112 %



76

Incident Energy (MeV)

76-0s-191

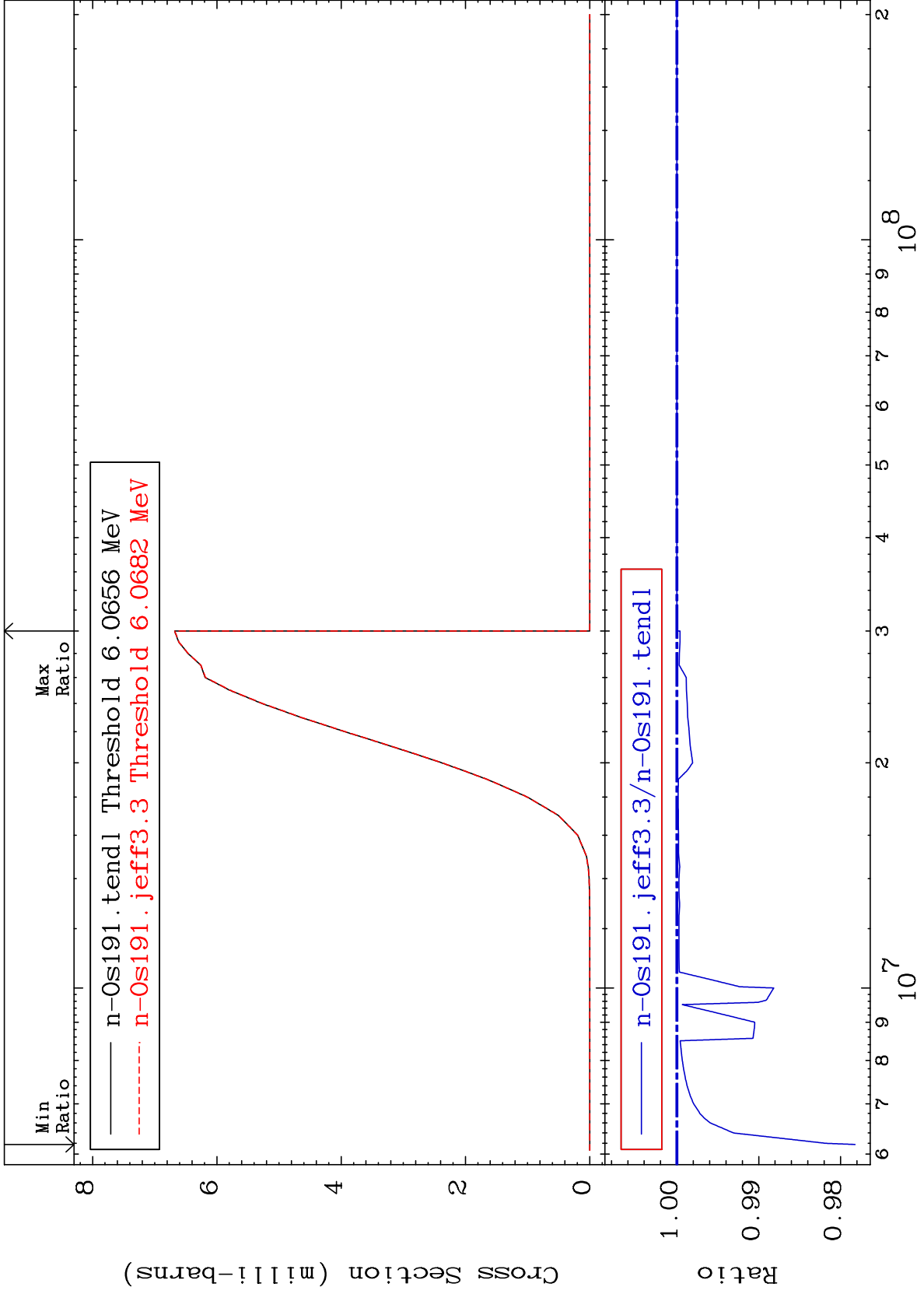


MAT 7646

(n, d) : 75-Re-190m3

76-0s-191

Radionuclide Production Cross Section -2.178 To 0.000 %



78

Incident Energy (eV)

76-0s-191