

Program EVALPLOT
(Version 2017-1)

by

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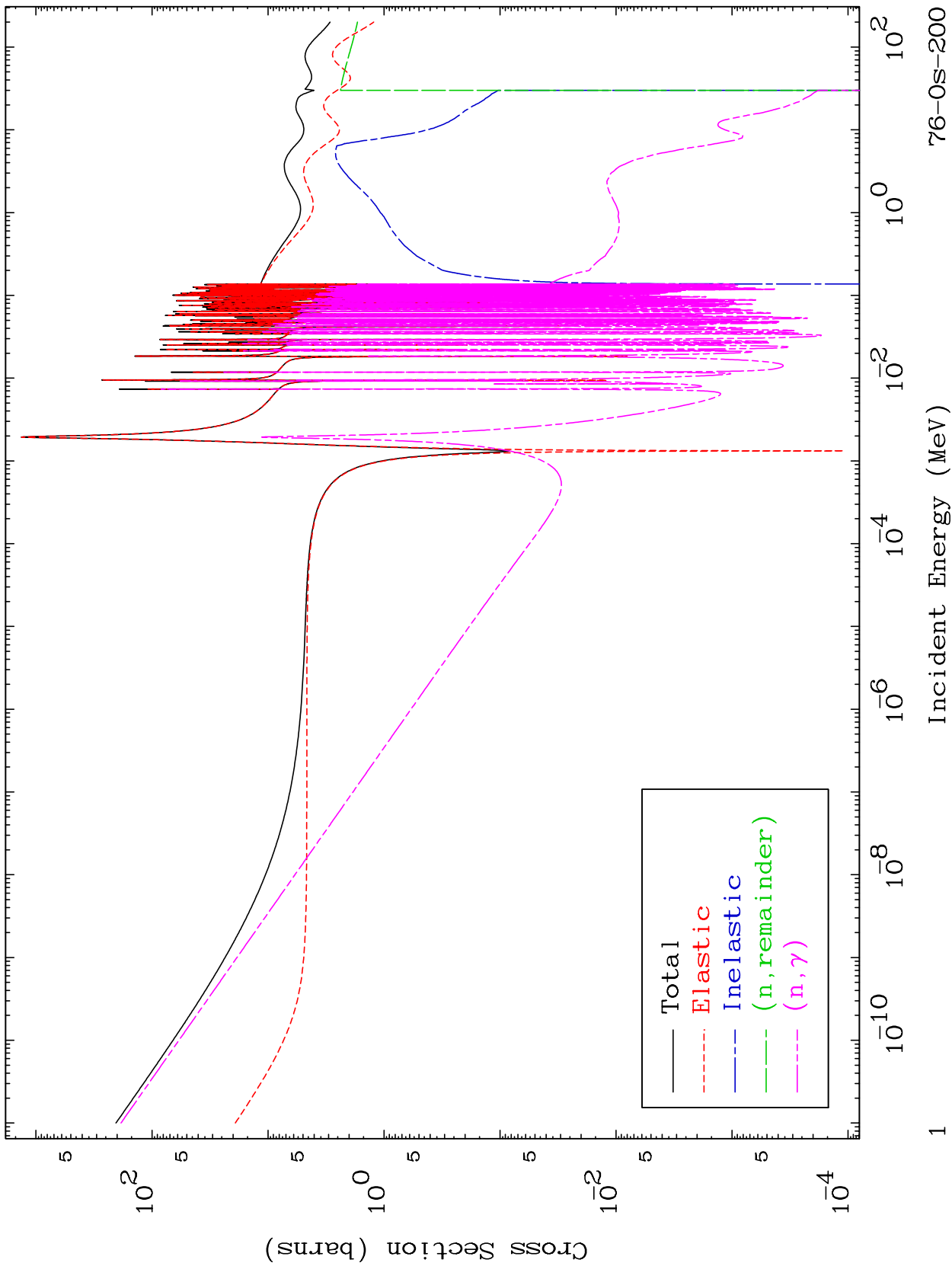
Press Mouse Button to Start

MAT 7673

Major

293 Kelvin Cross Sections

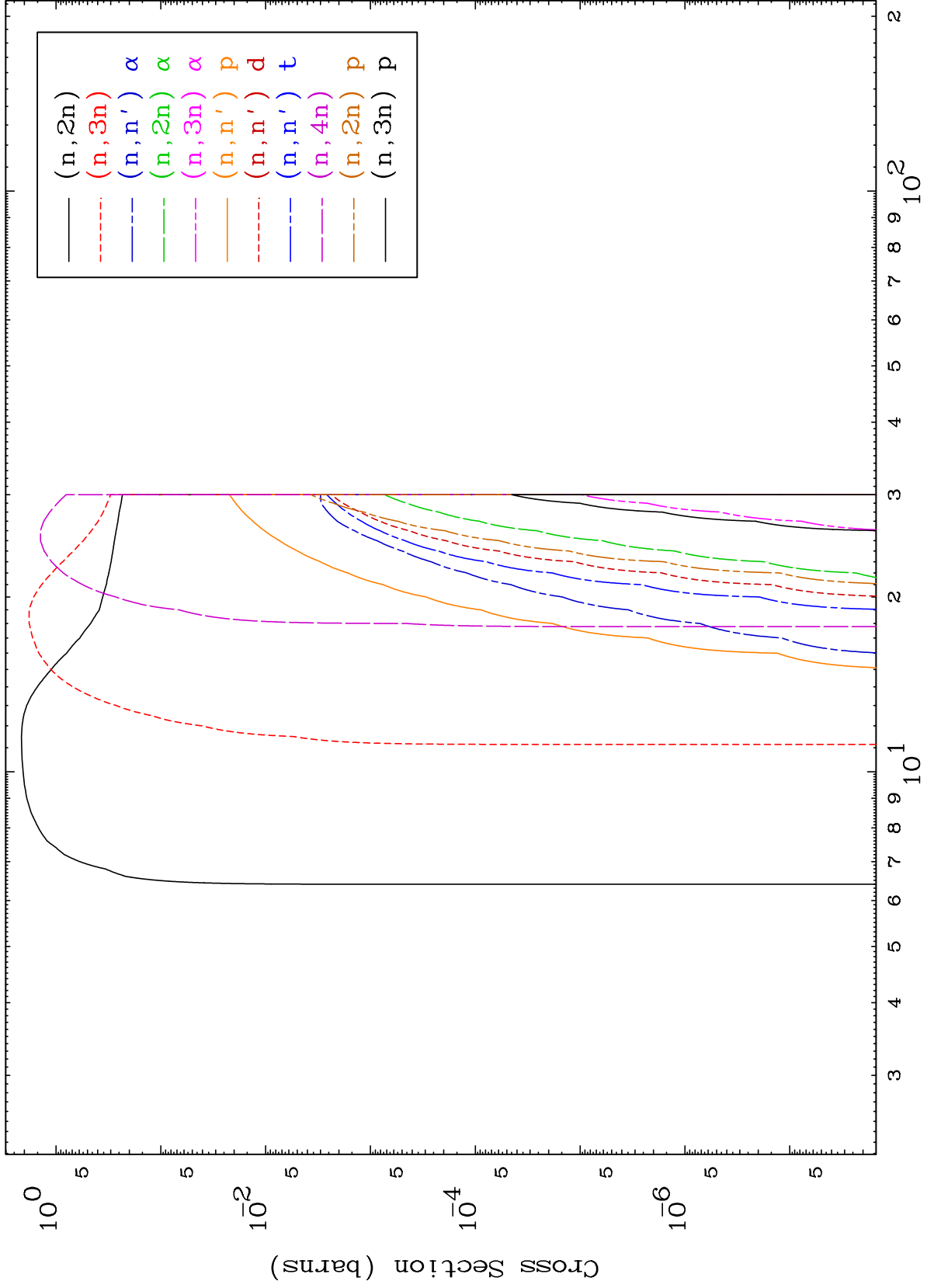
76-Os-200



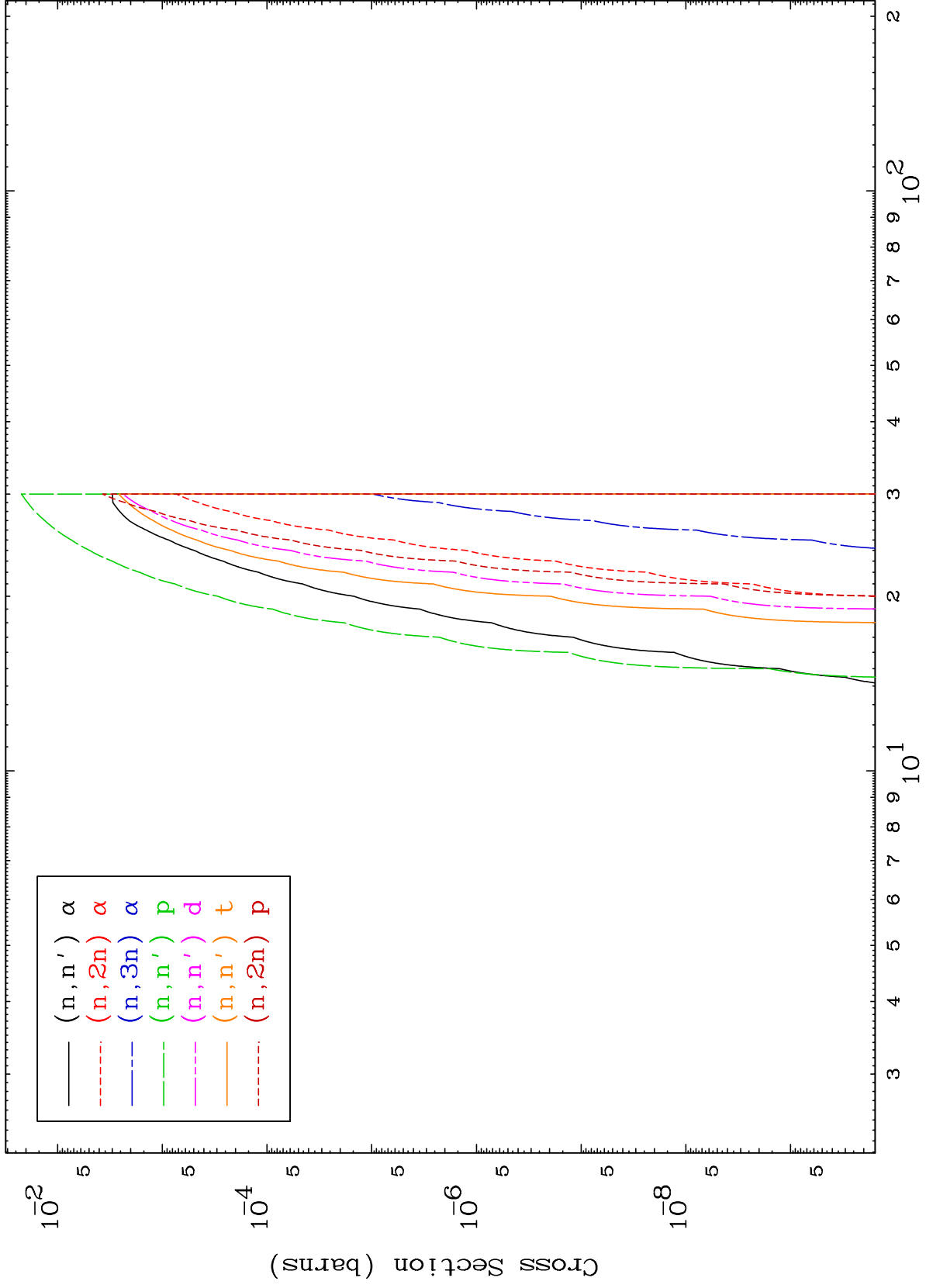
MAT 7673

Neutron Production
293 Kelvin Cross Sections

76-0s-200



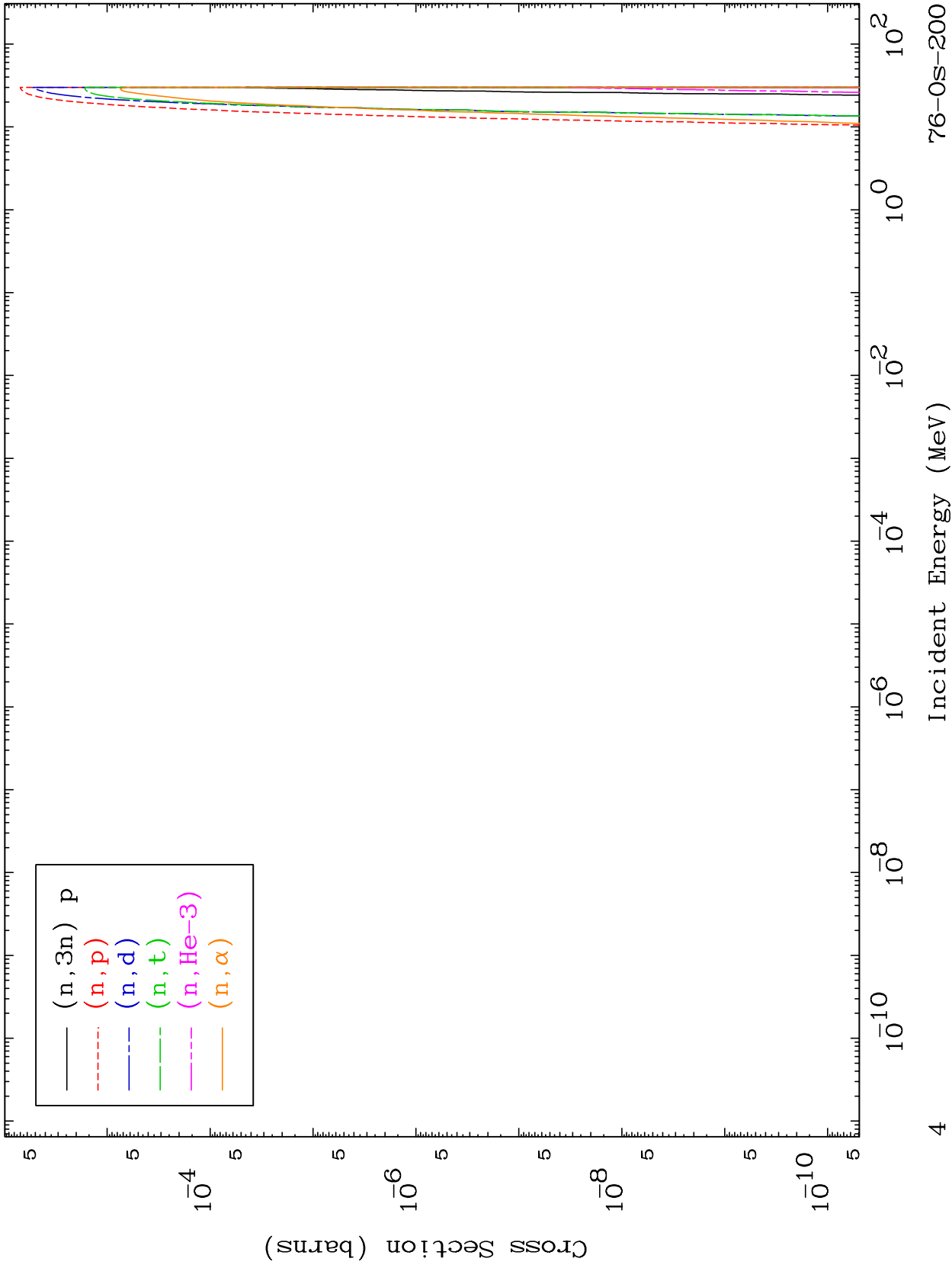
76-0s-200



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Charged Particle
293 Kelvin Cross Sections

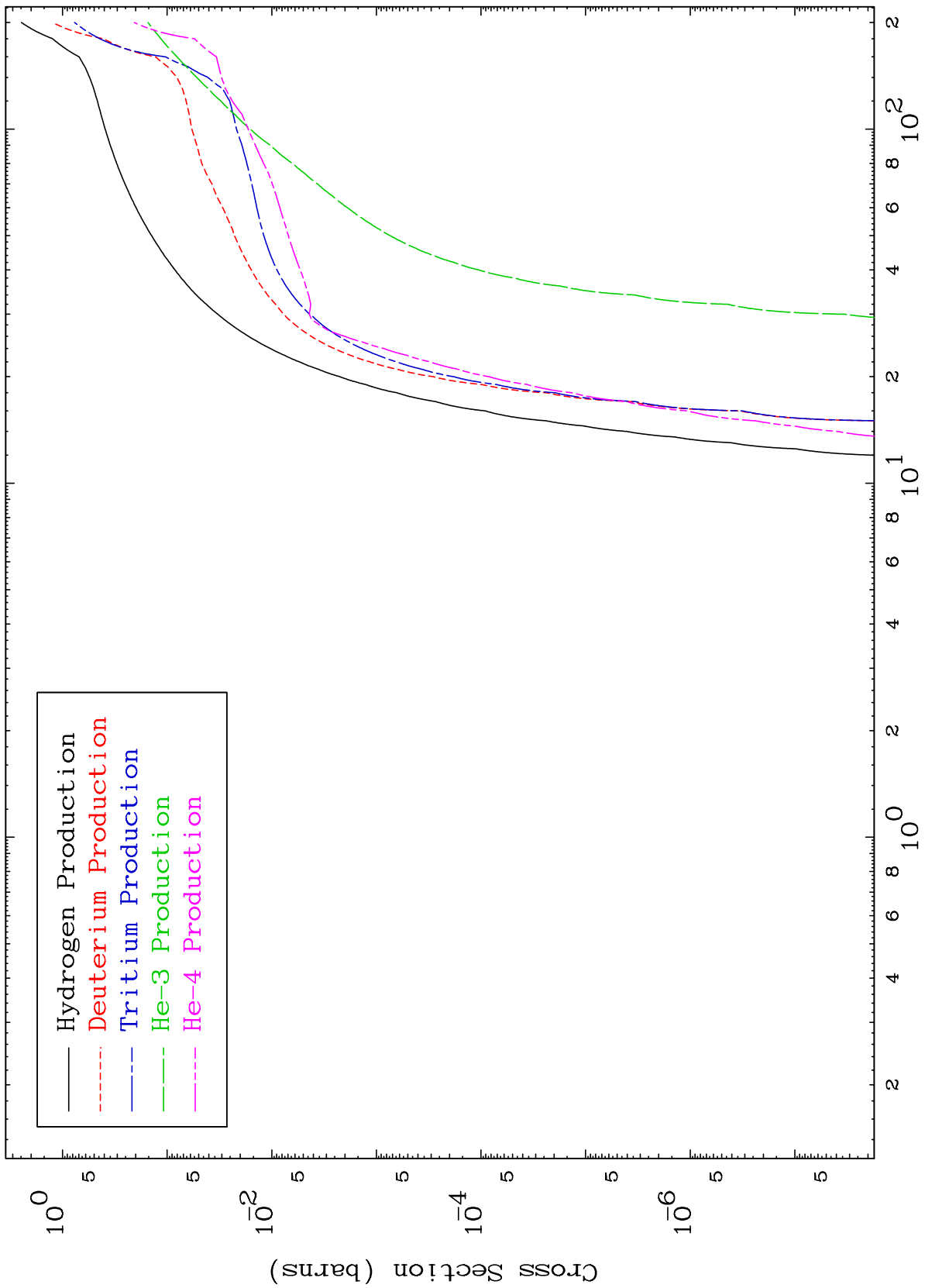
76-Os-200



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Particle Production
293 Kelvin Cross Sections

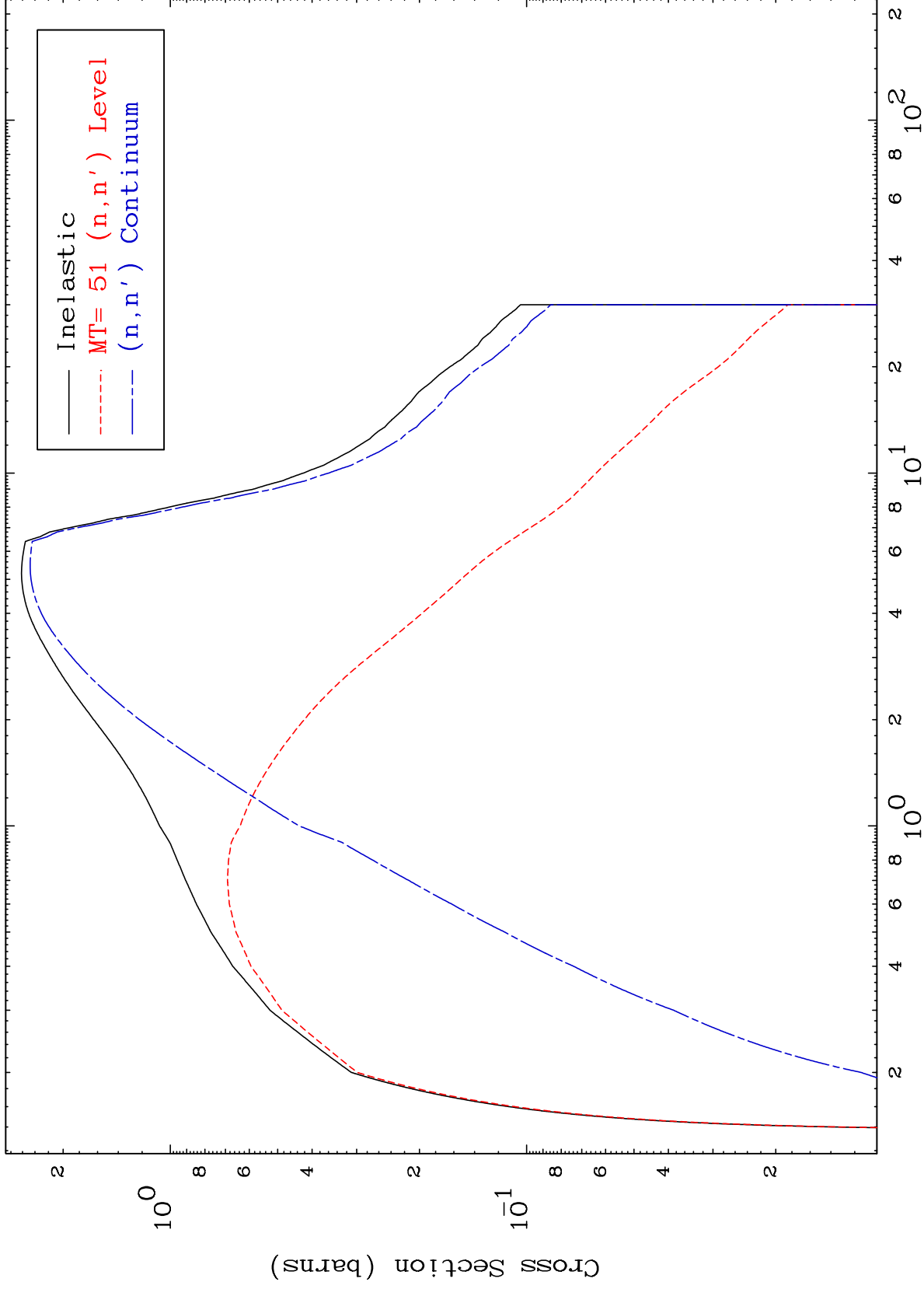
76-0s-200



MAT 7673

(n,n') Level
293 Kelvin Cross Sections

76-0s-200



6

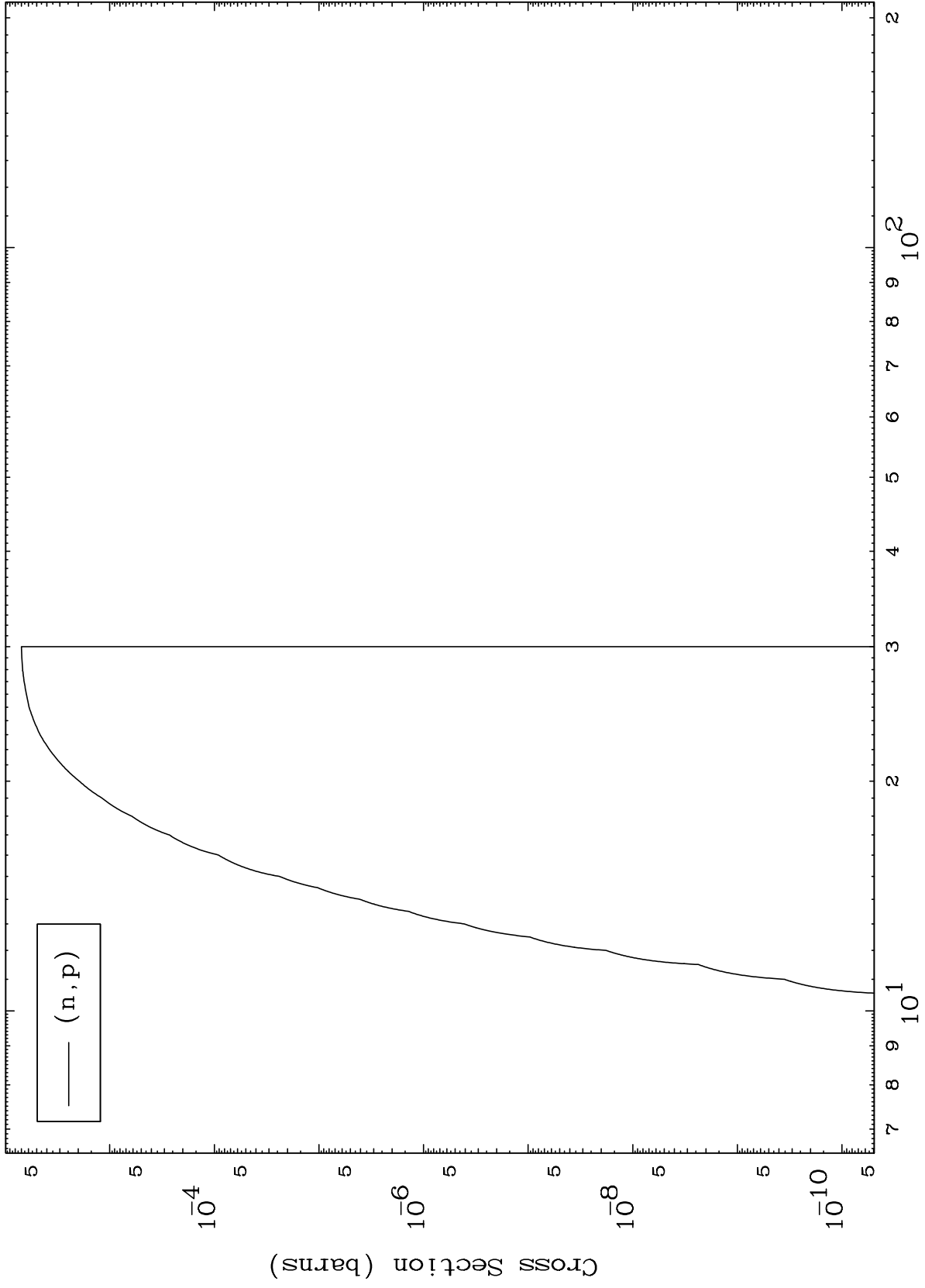
Incident Energy (MeV)

76-0s-200

MAT 7673

(n,p) Levels
293 Kelvin Cross Sections

76-0s-200



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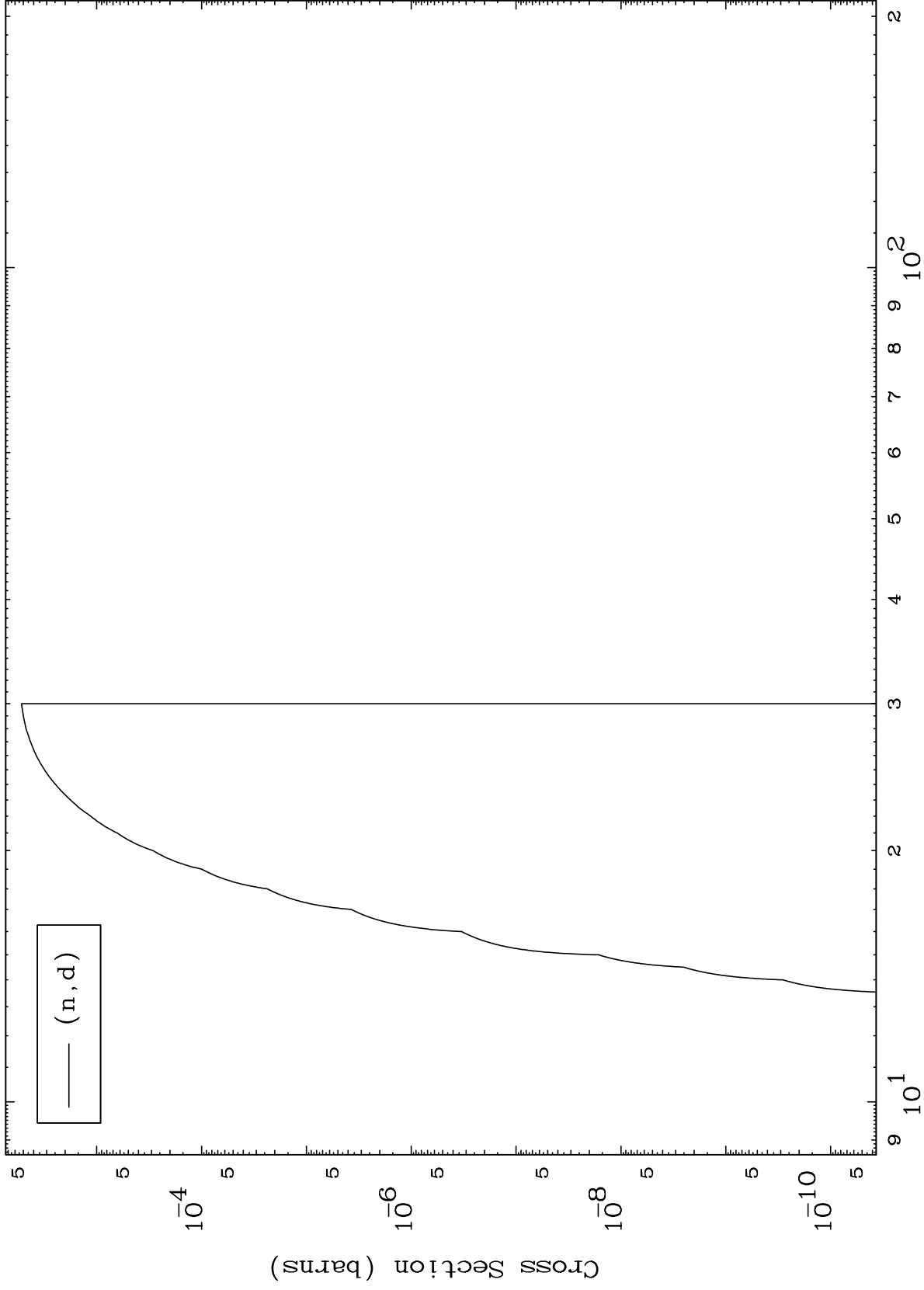
Incident Energy (MeV)

76-0s-200

MAT 7673

(n,d) Levels
293 Kelvin Cross Sections

76-0s-200



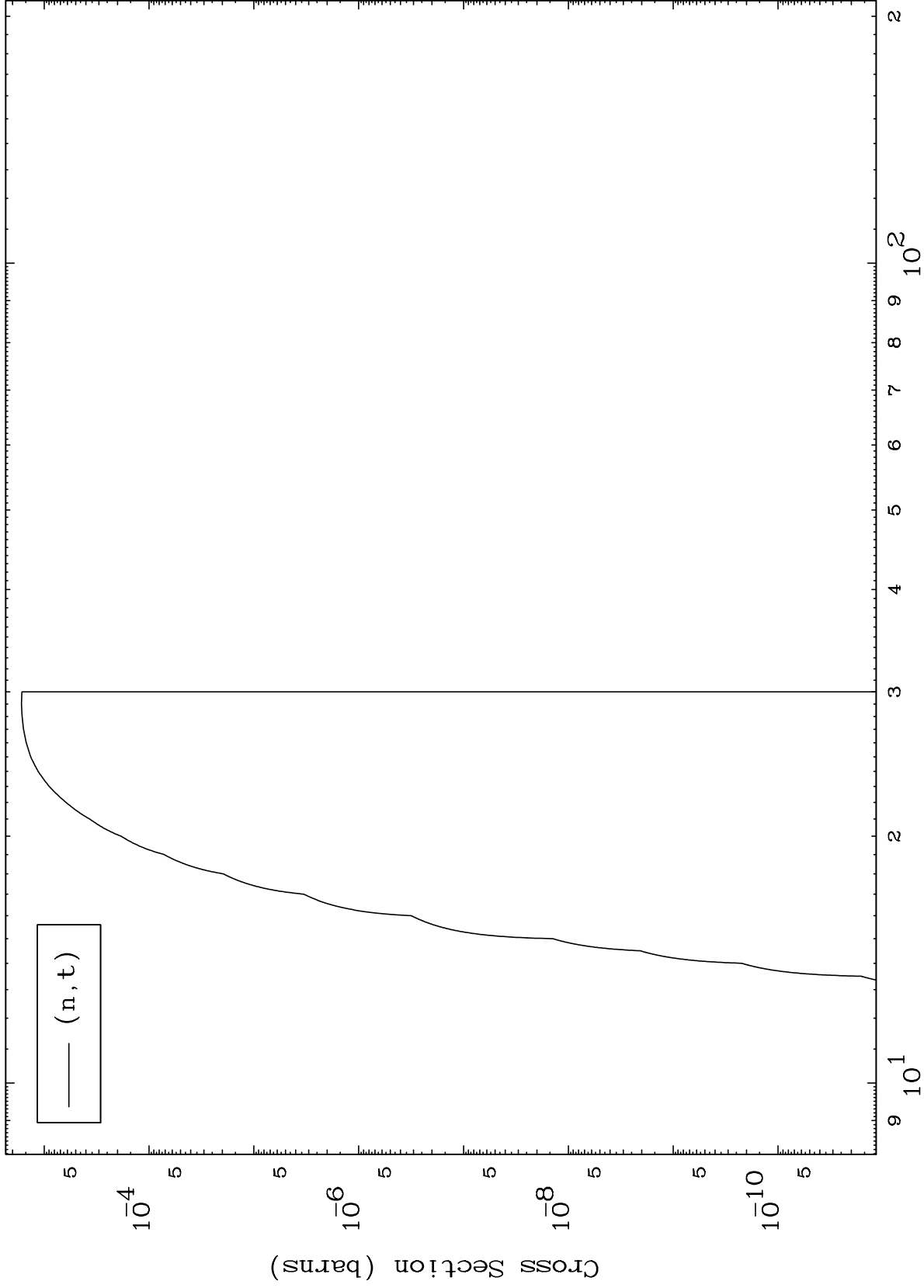
Incident Energy (MeV)

76-0s-200

MAT 7673

(n,t) Levels
293 Kelvin Cross Sections

76-0s-200



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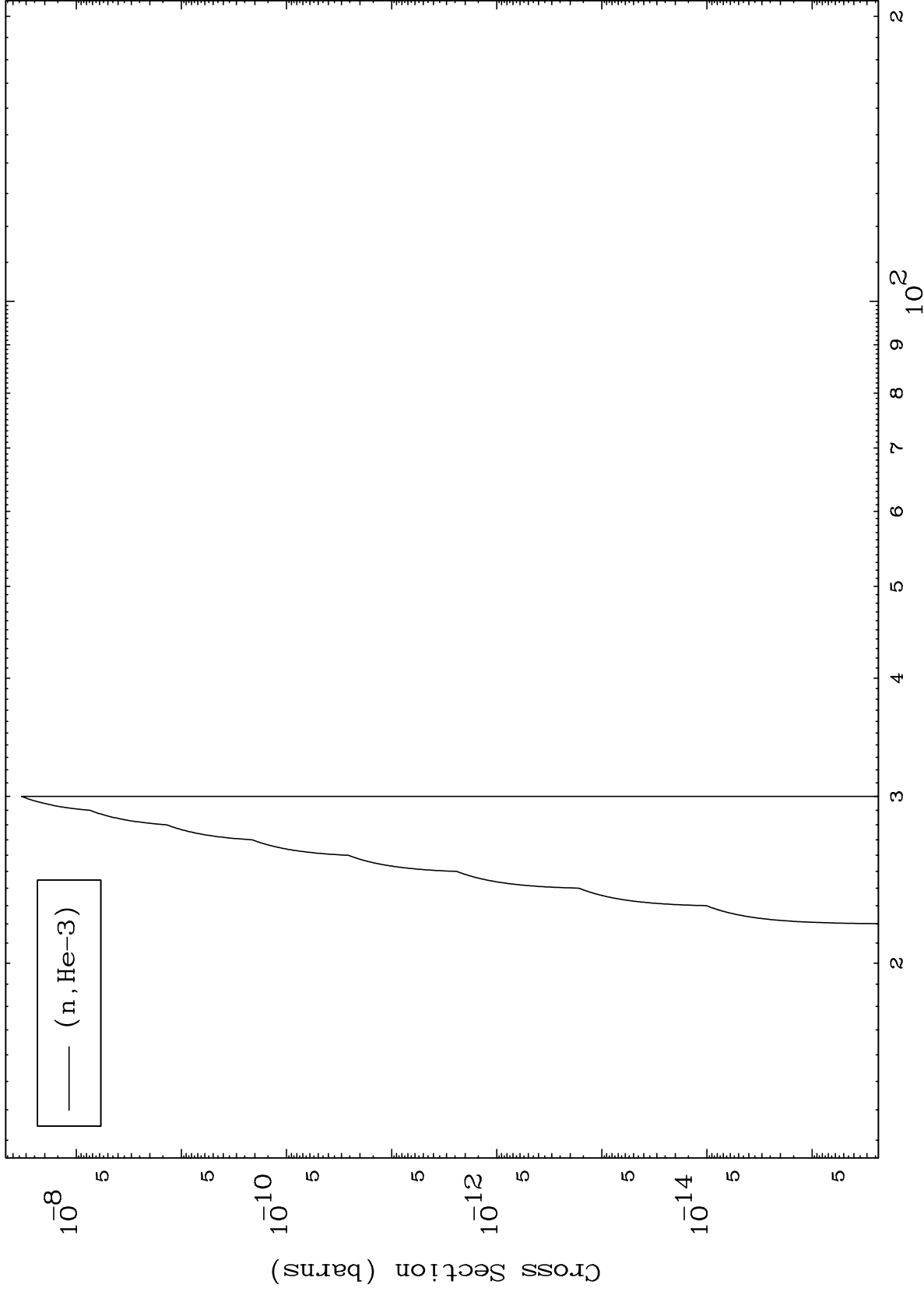
Incident Energy (MeV)

76-0s-200

MAT 7673

(n,He3) Levels
293 Kelvin Cross Sections

76-0s-200



10

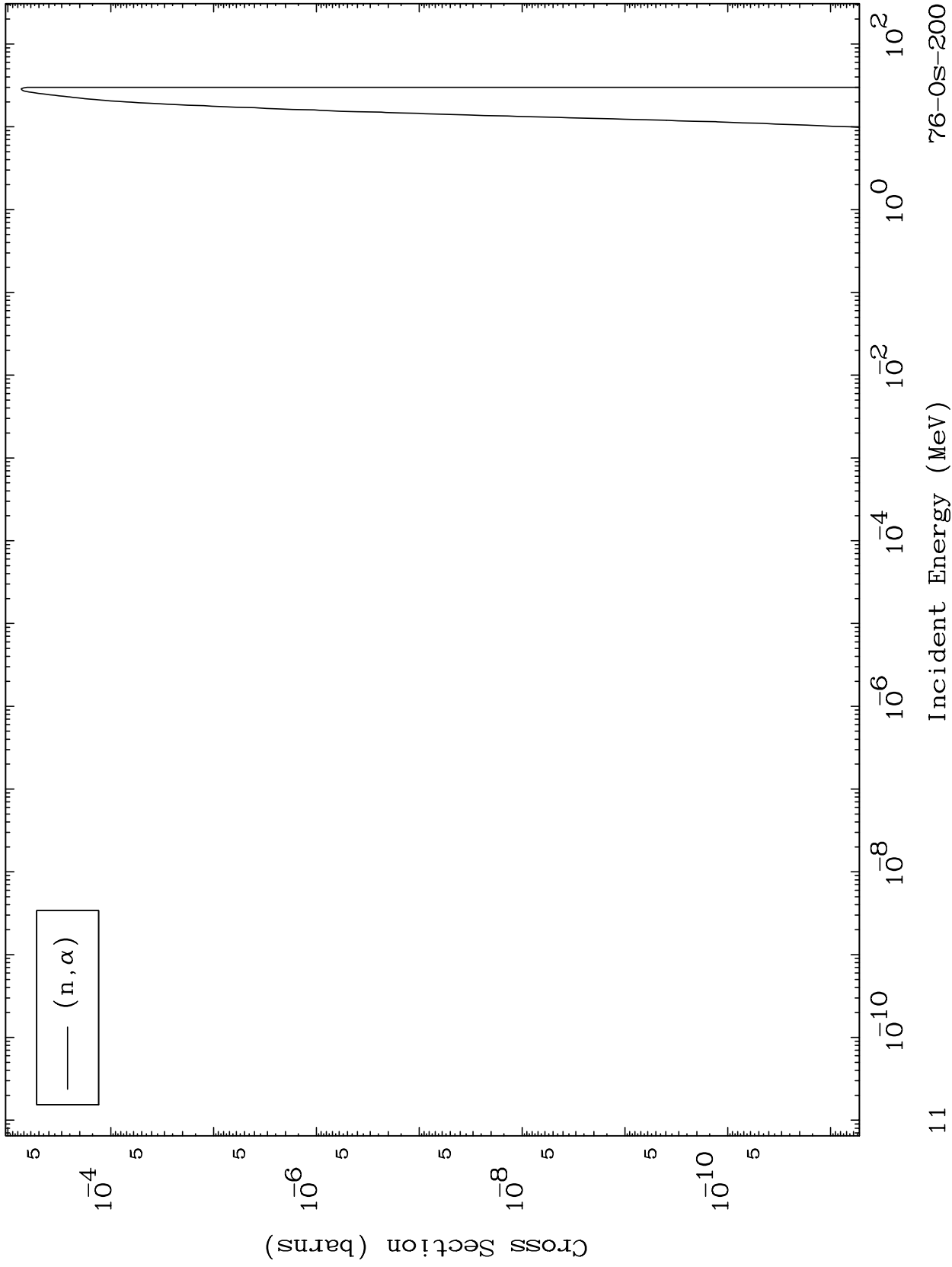
Incident Energy (MeV)

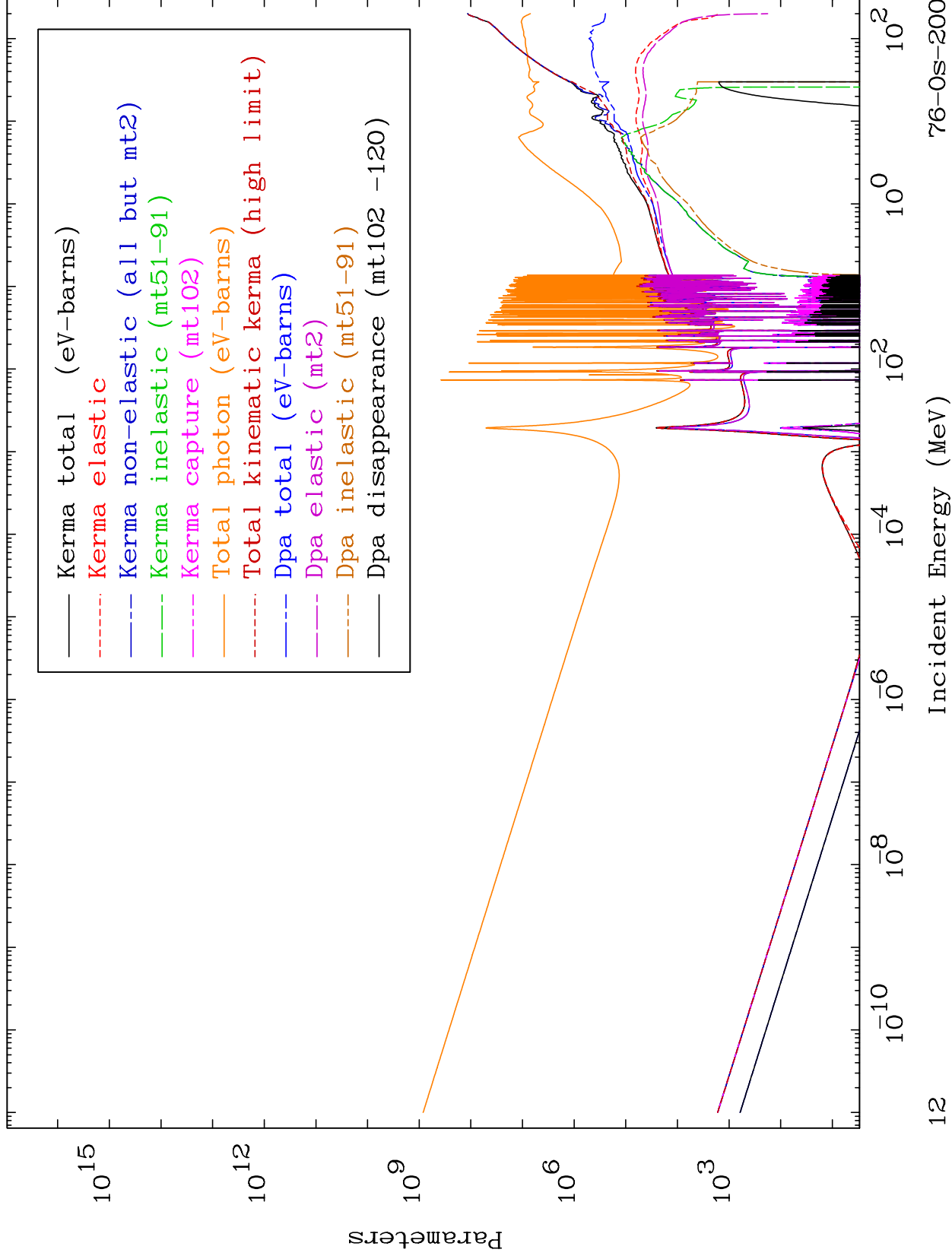
76-0s-200

MAT 7673

(n, α) Levels
293 Kelvin Cross Sections

76-0s-200

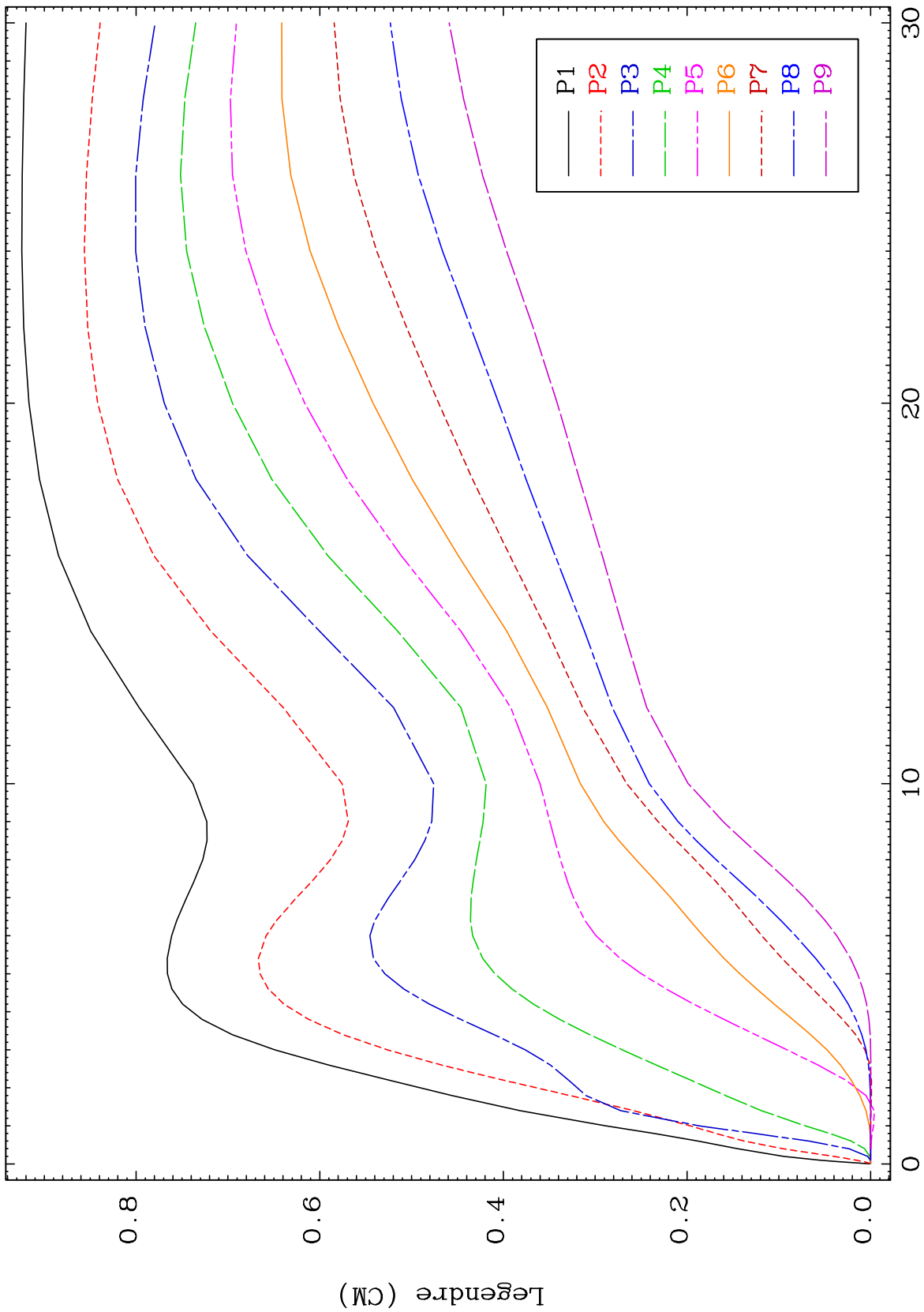




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Elastic Legendre Coefficients

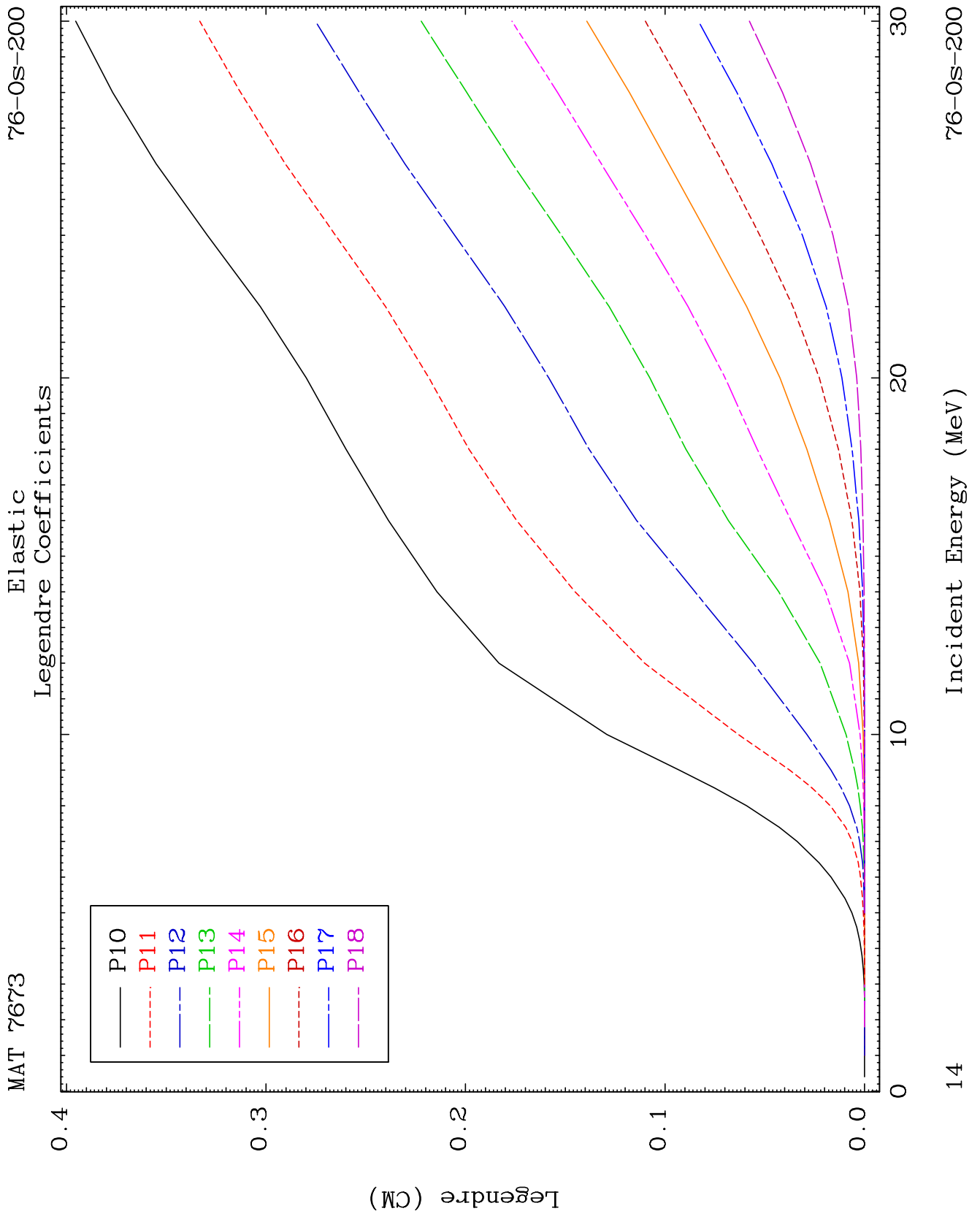
76-0s-200



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Incident Energy (MeV)

76-0s-200



76-0s-200

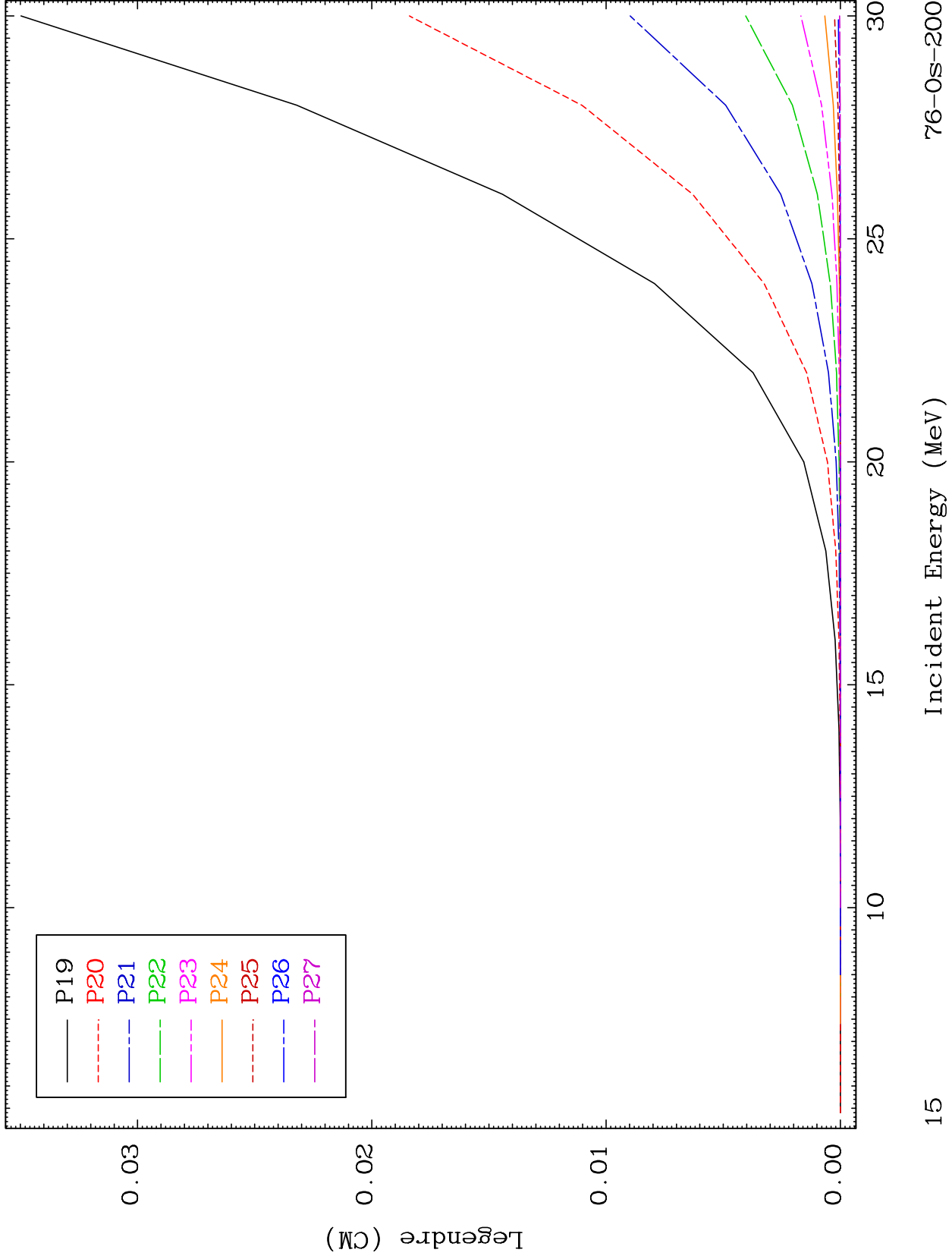
76-0s-200

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MAT 7673

Elastic Legendre Coefficients

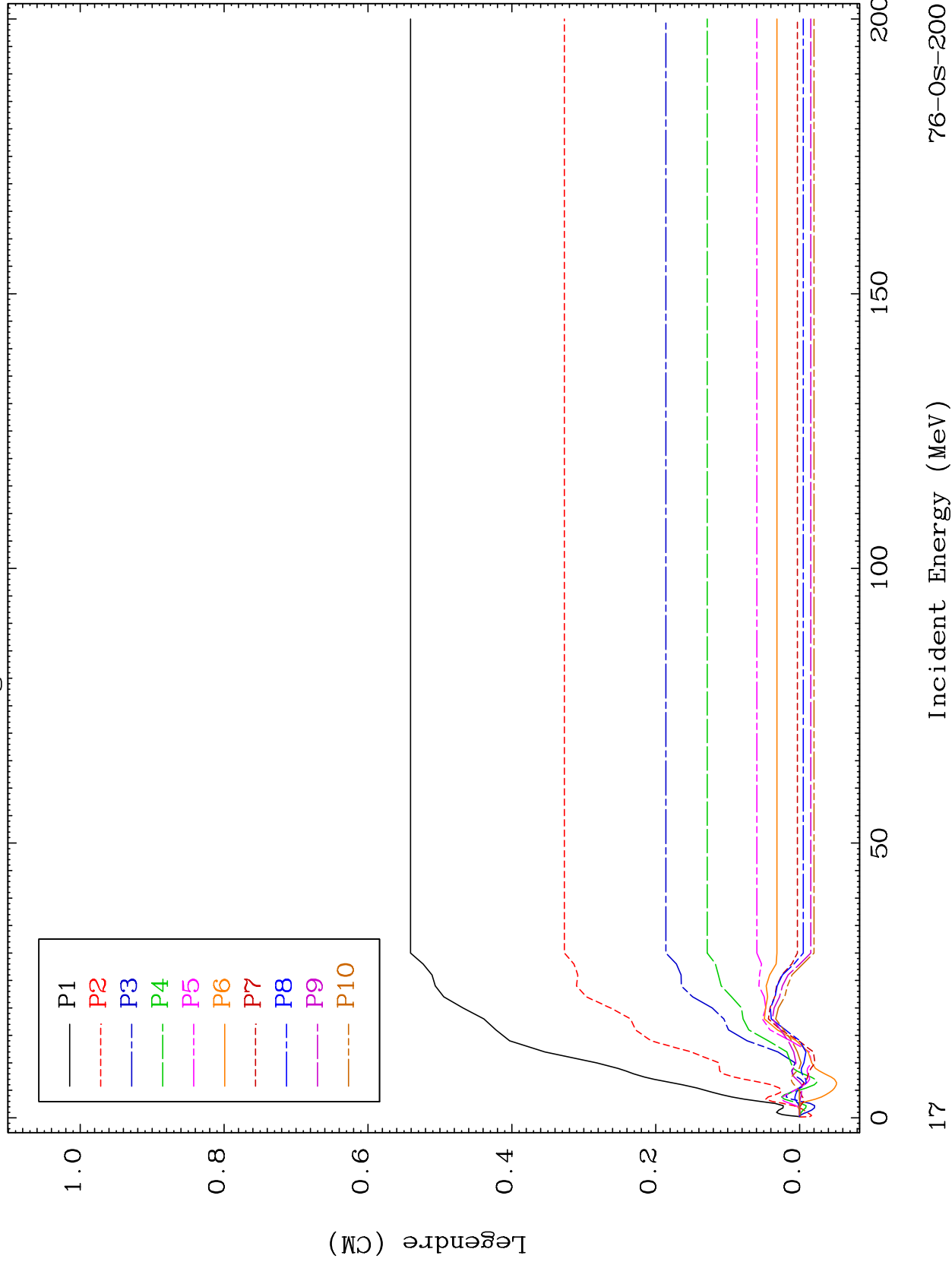
76-0s-200



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MT= 51 (n,n') Level
Legendre Coefficients

76-0s-200



17

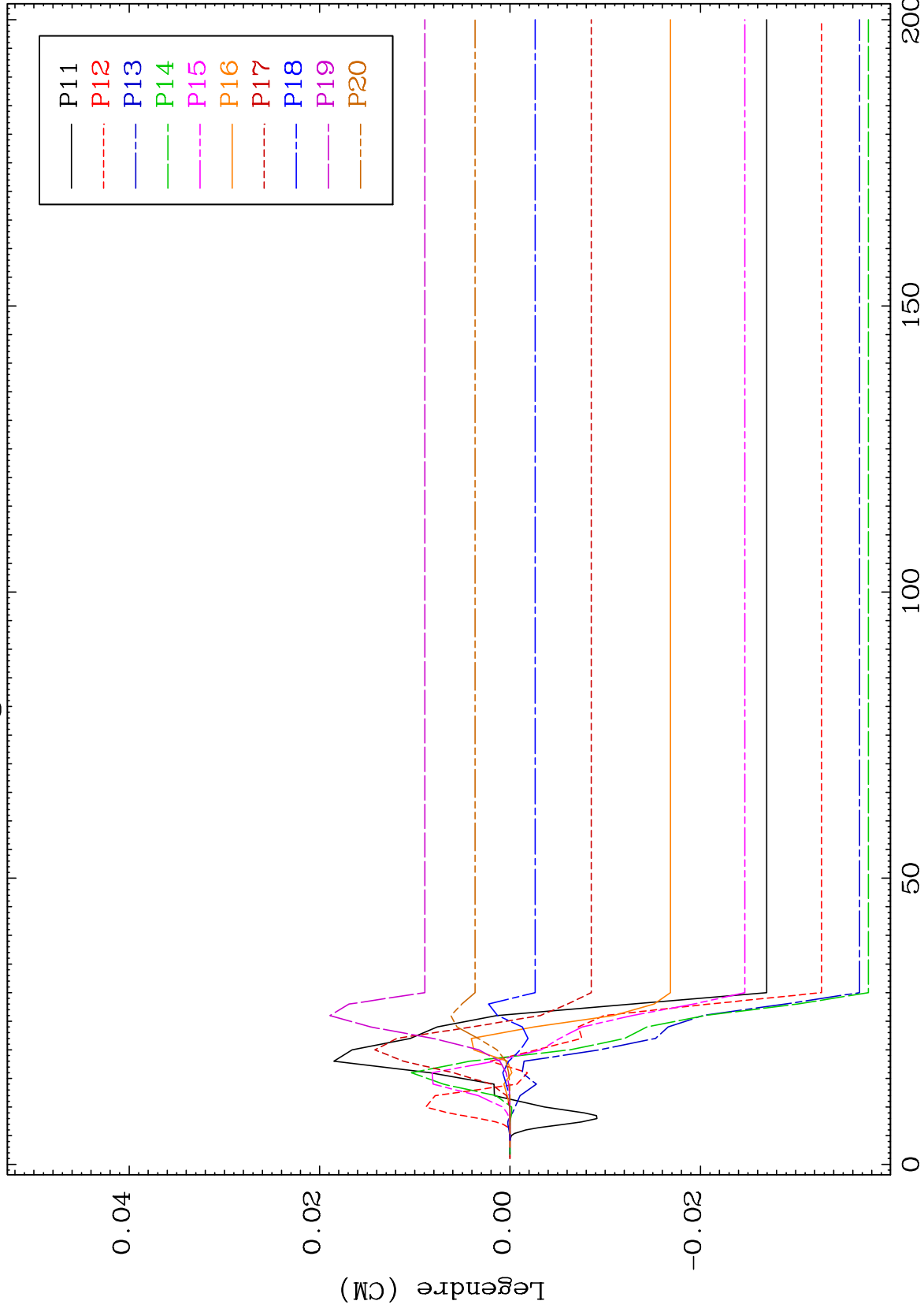
Incident Energy (MeV)

76-0s-200

MAT 7673

MT= 51 (n,n') Level
Legendre Coefficients

76-0s-200



18

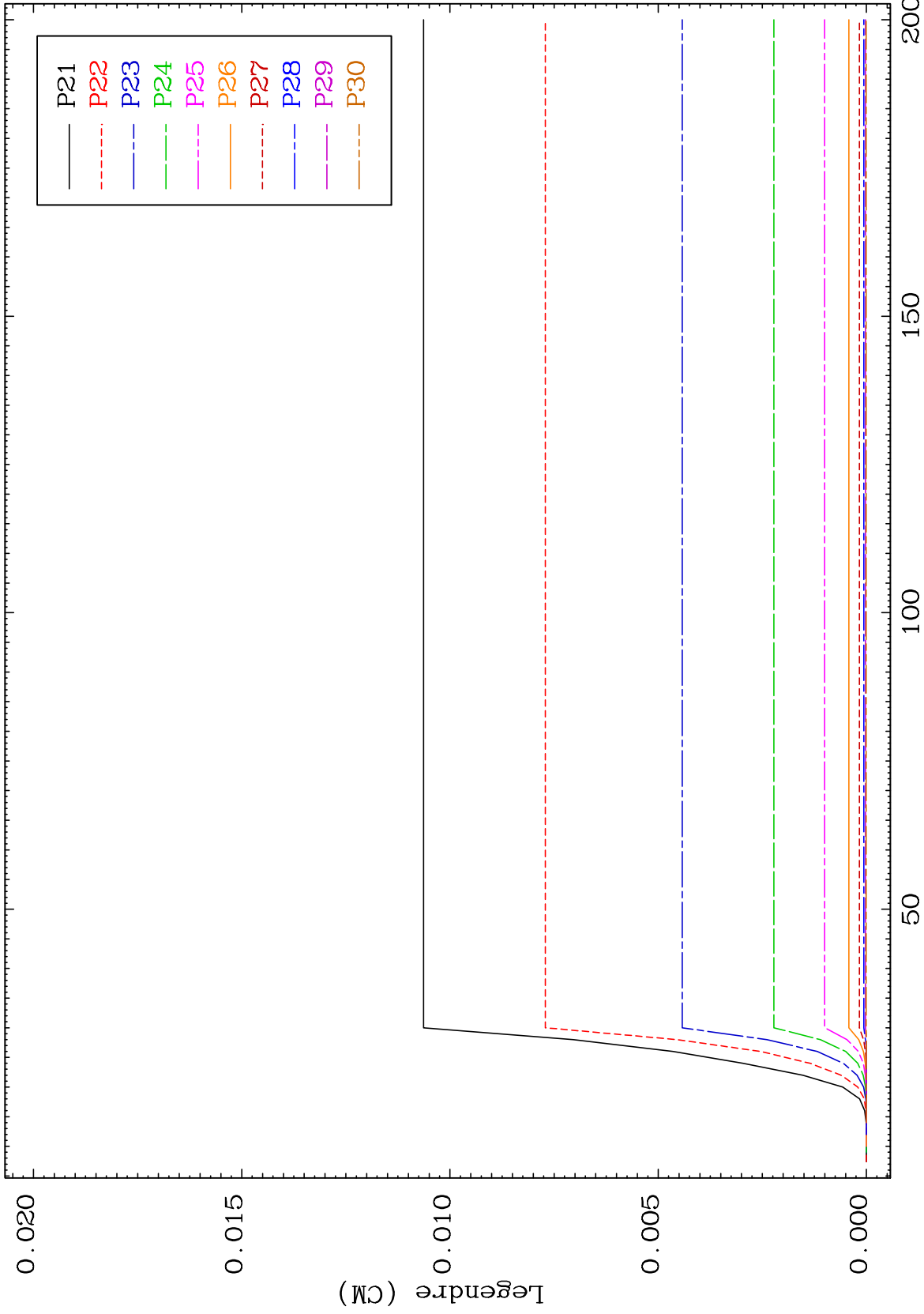
Incident Energy (MeV)

76-0s-200

MAT 7673

MT= 51 (n,n') Level
Legendre Coefficients

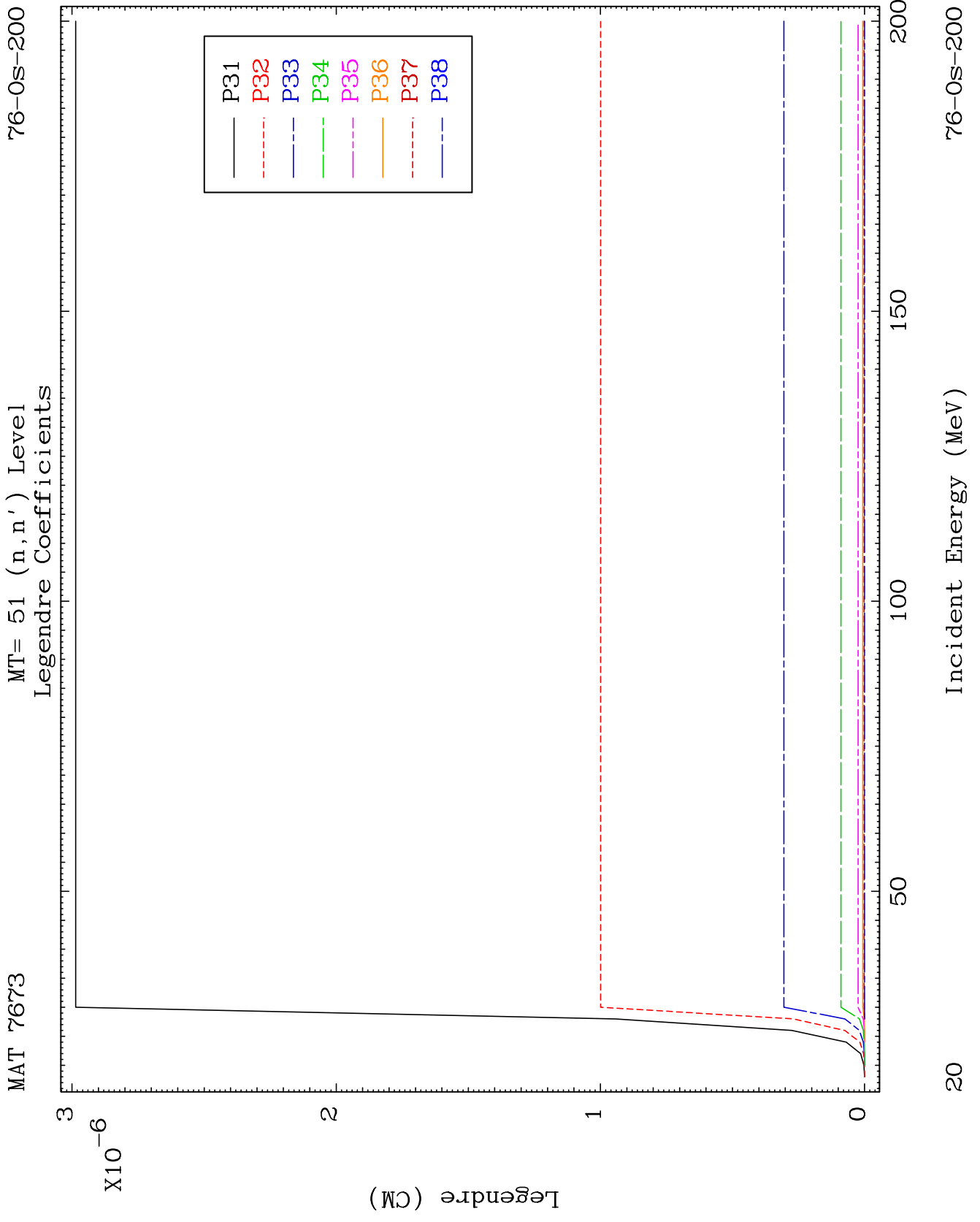
76-0s-200



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Incident Energy (MeV)

76-0s-200



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Fission

76-0s-200

Radionuclide Production Cross Section

