

Program EVALPLOT  
(Version 2017-1)

by

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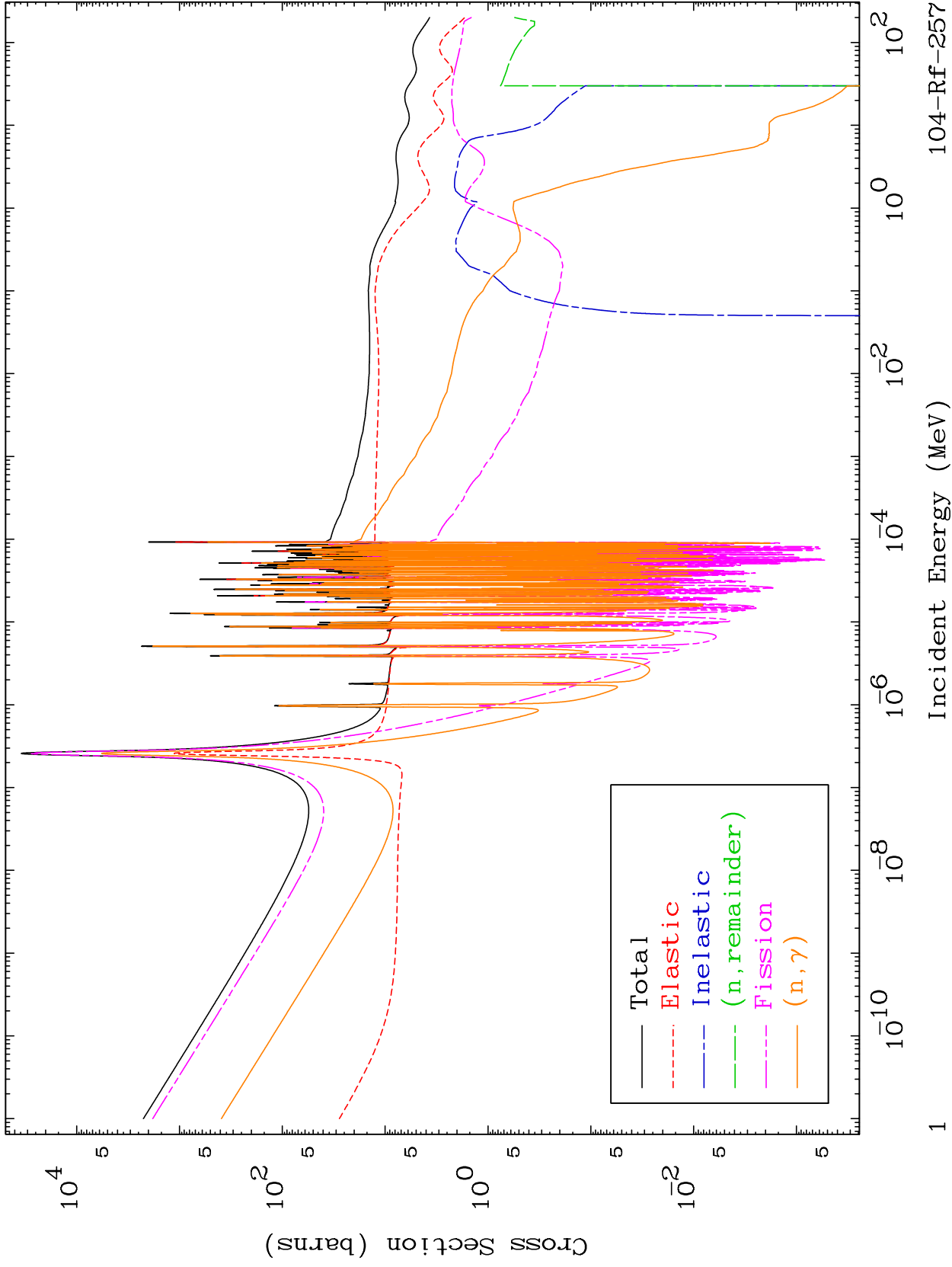
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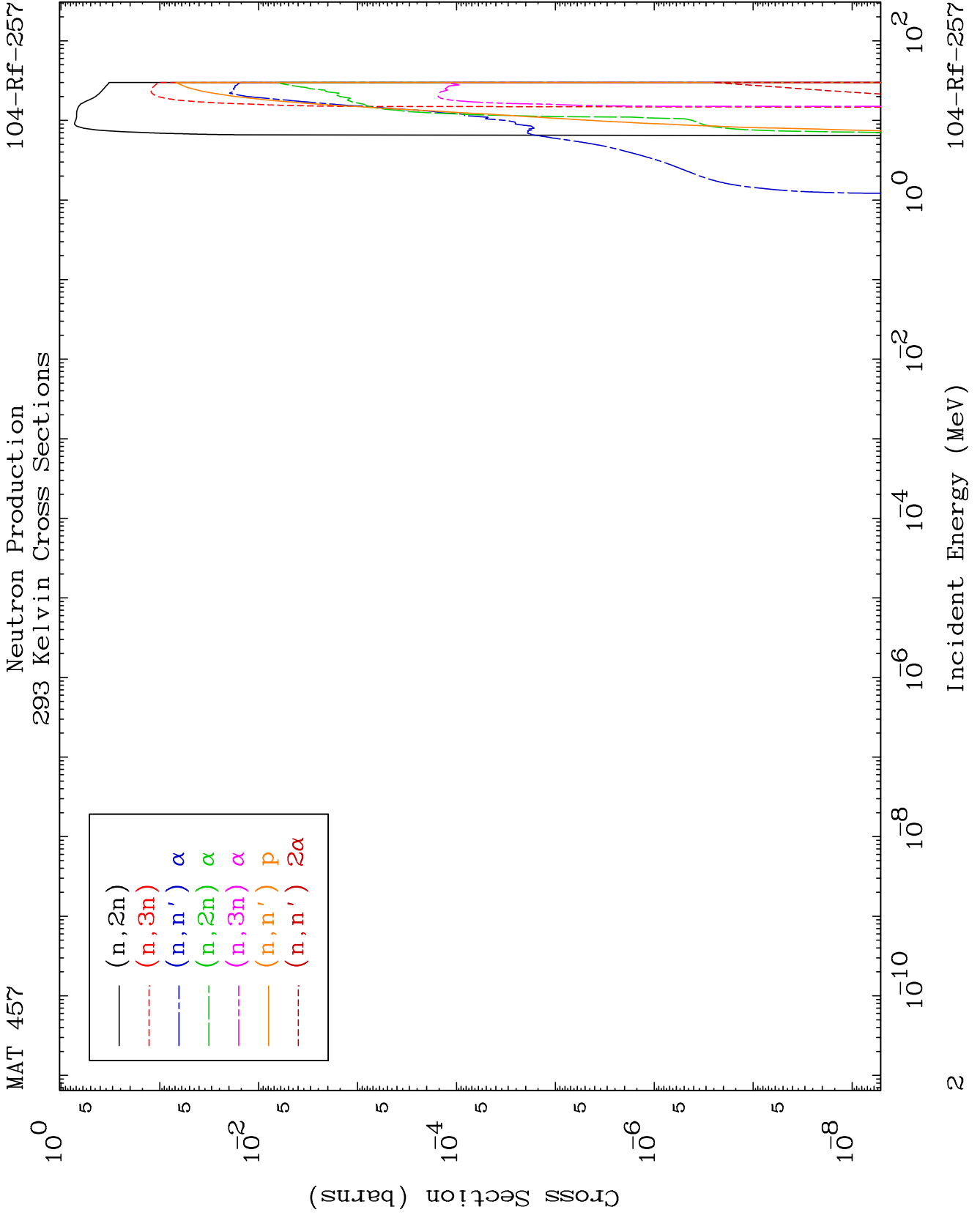
Press Mouse Button to Start

MAT 457

Major  
293 Kelvin Cross Sections

104-Rf-257

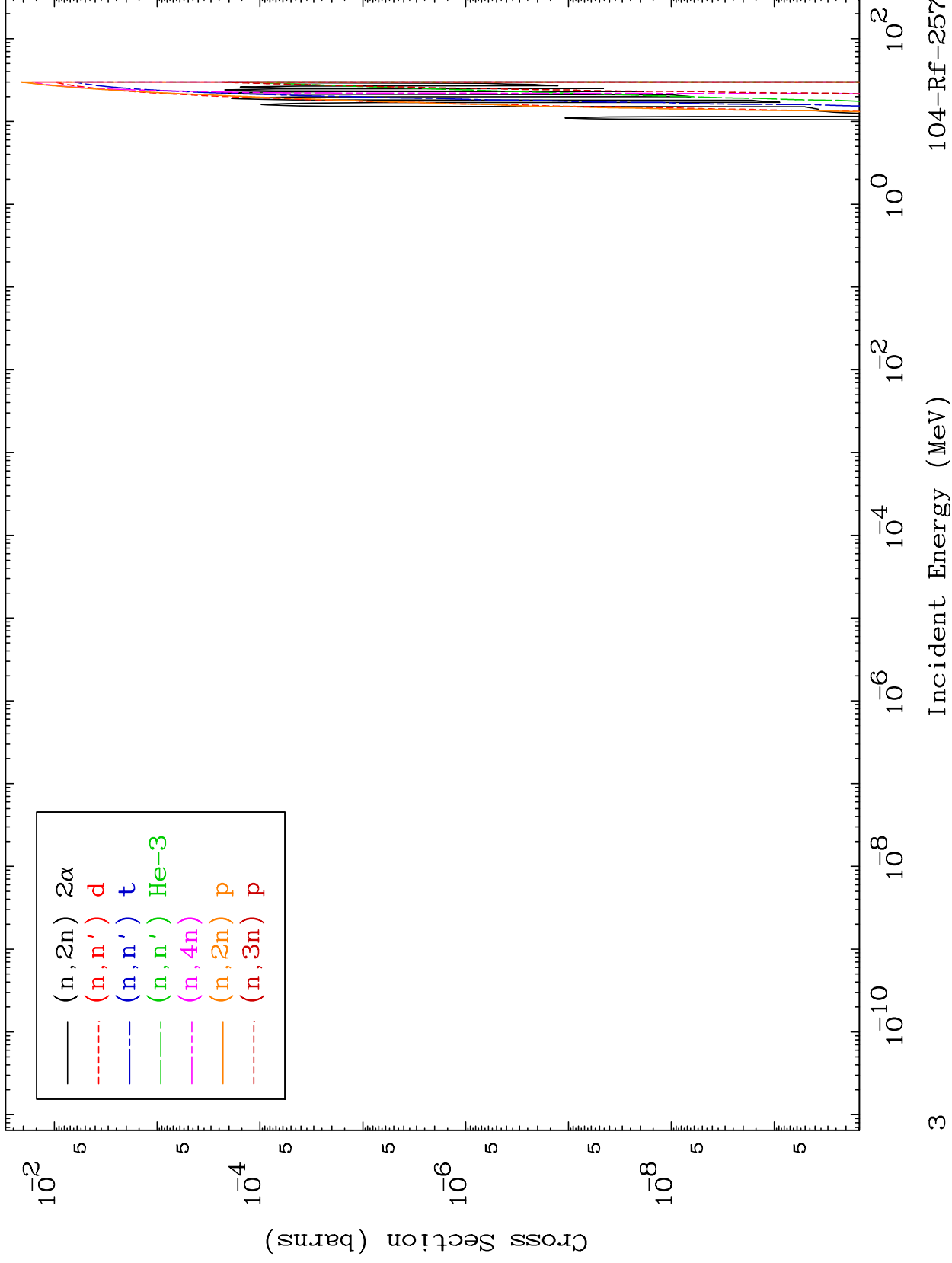




MAT 457

Neutron Production  
293 Kelvin Cross Sections

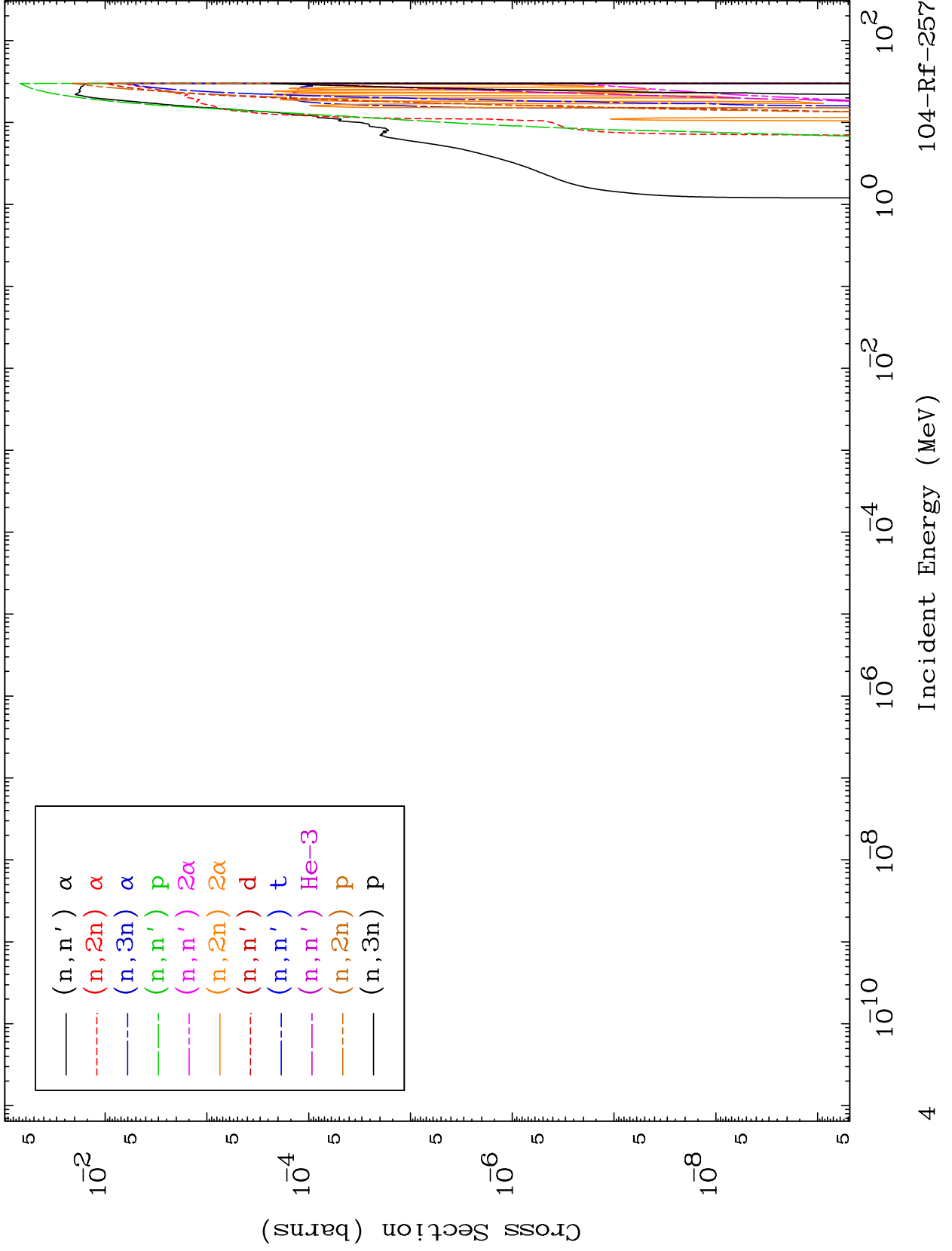
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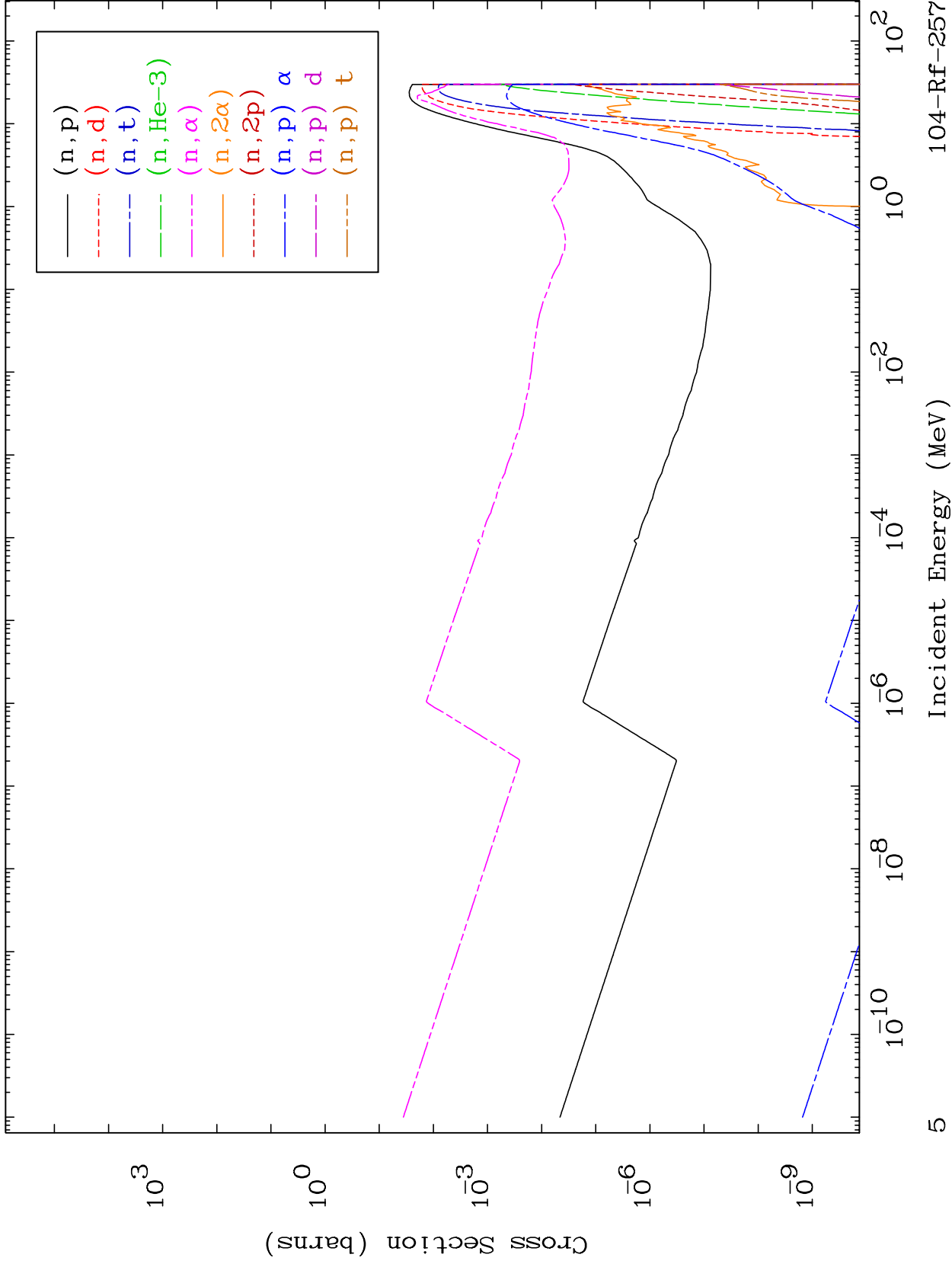


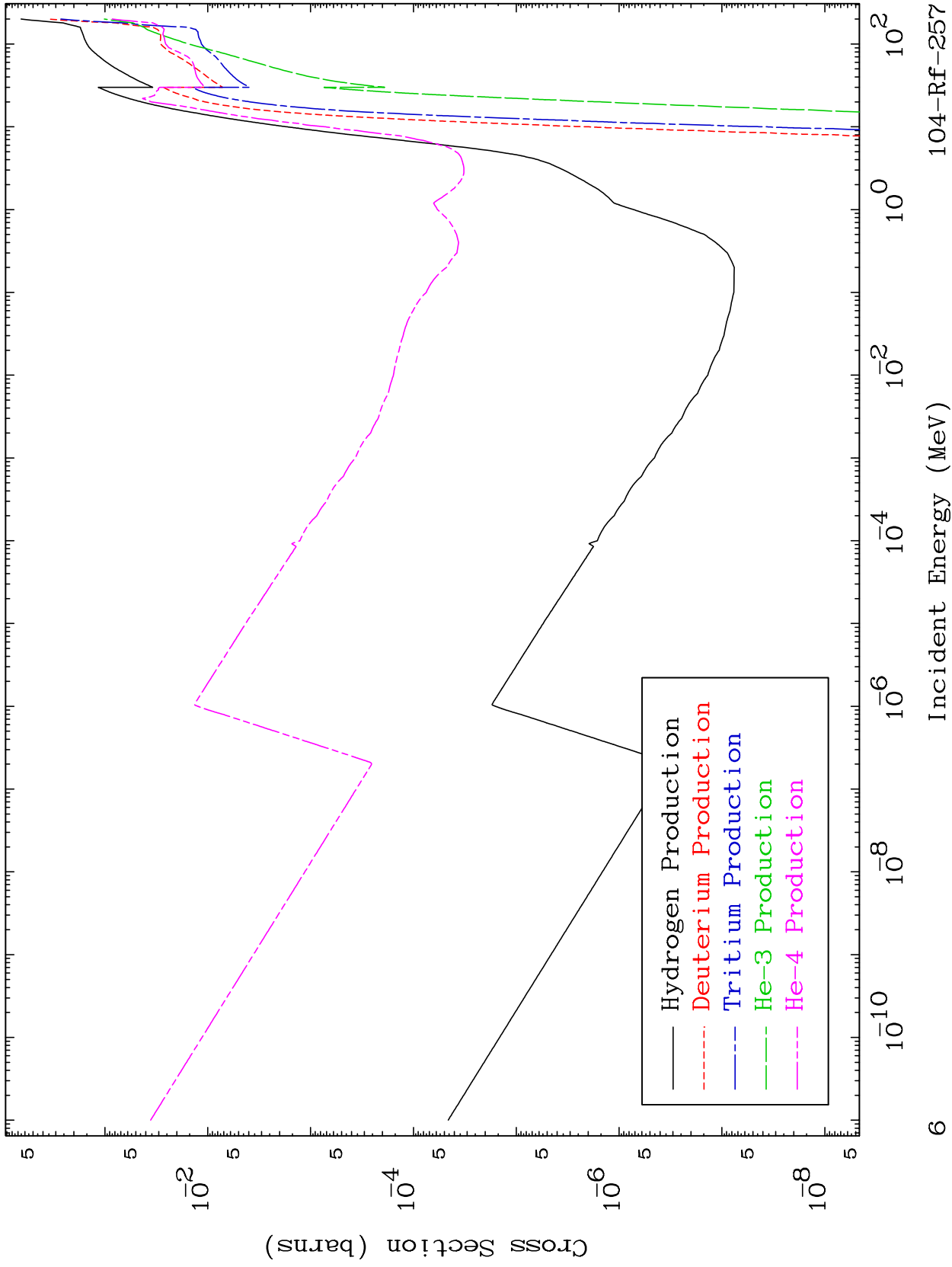
MAT 457

Charged Particle  
293 Kelvin Cross Sections

104-Rf-257



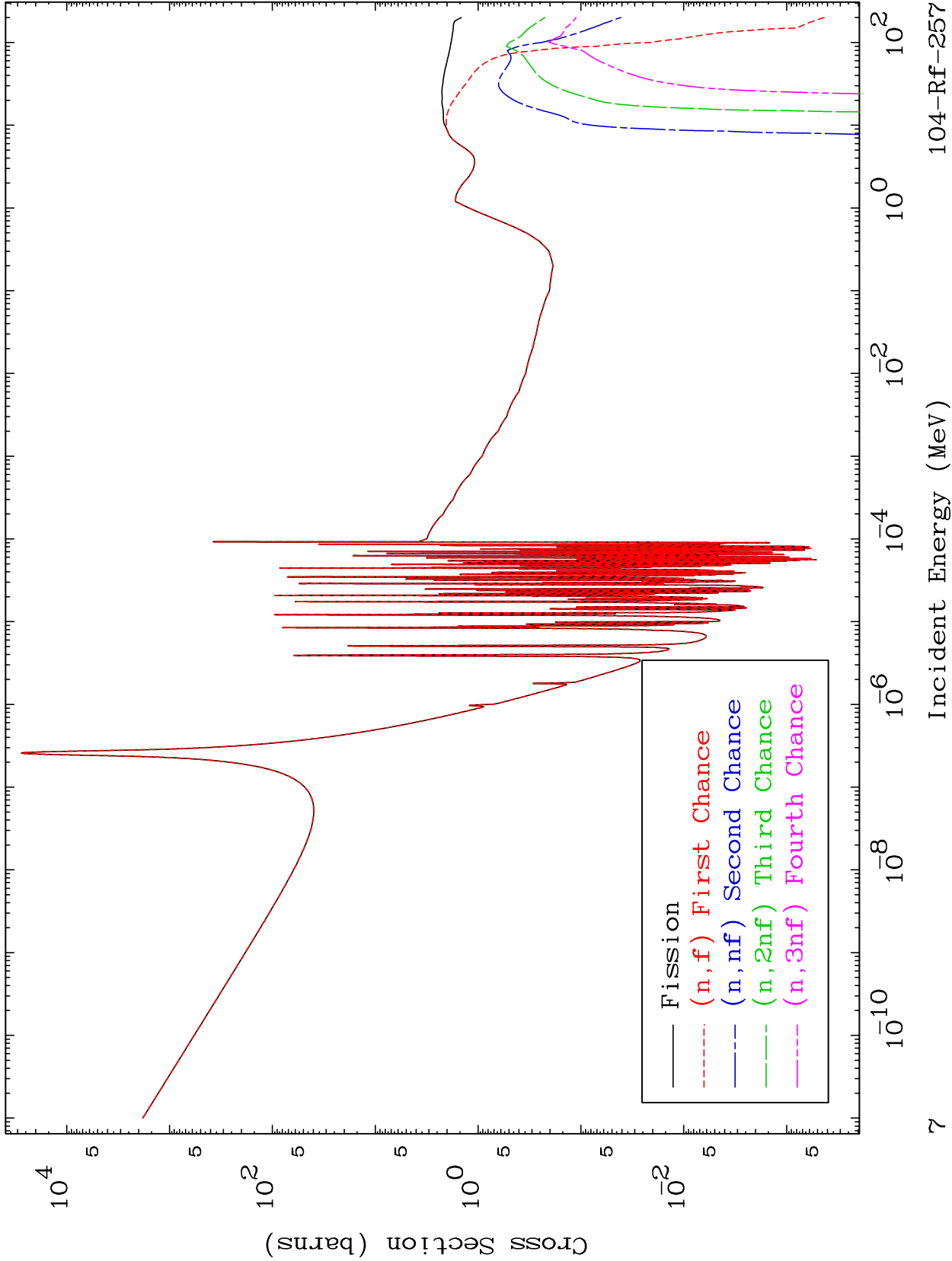




MAT 457

Fission  
293 Kelvin Cross Sections

104-Rf-257



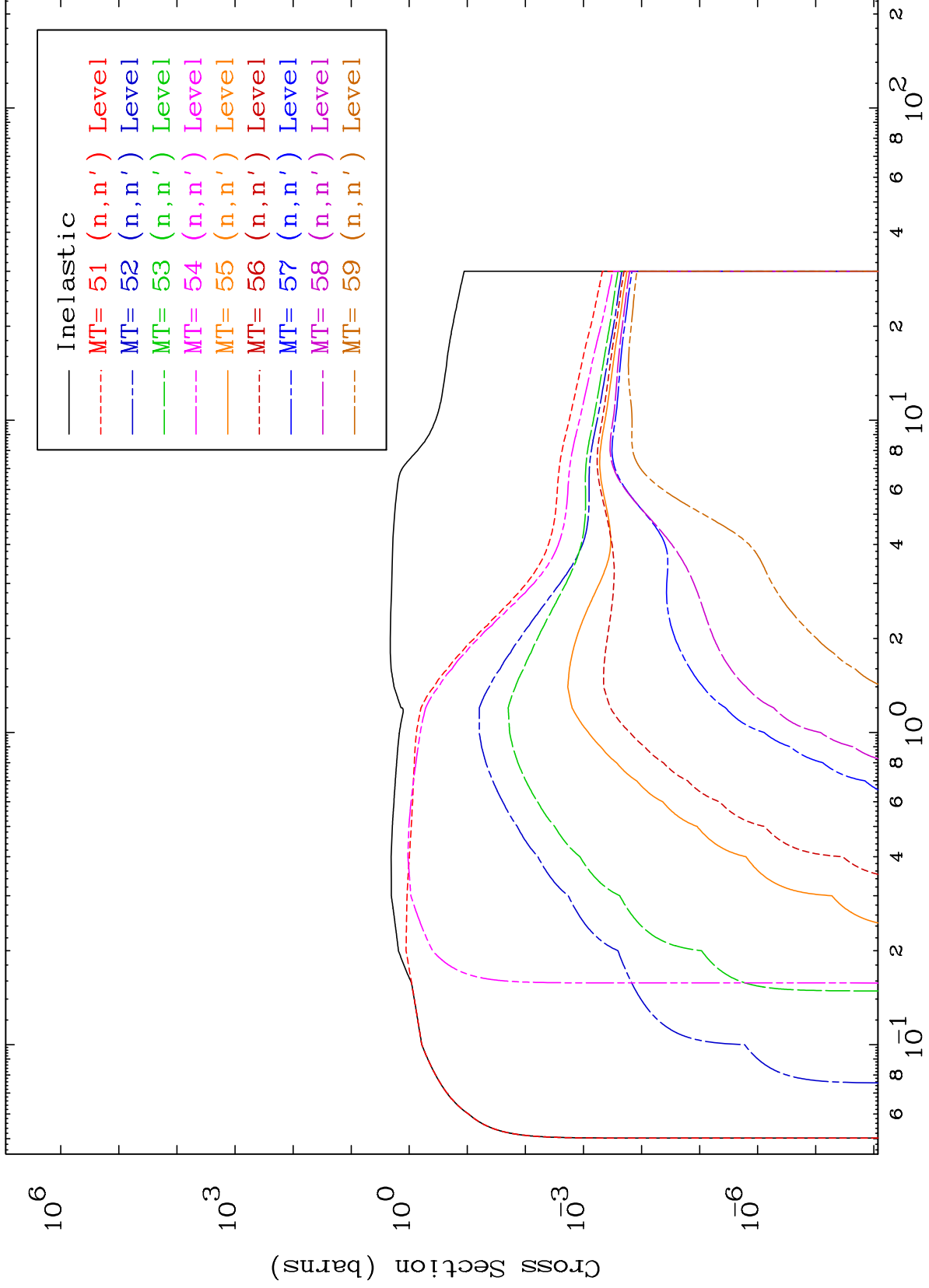


MAT 457

(n,n') Level

104-Rf-257

293 Kelvin Cross Sections

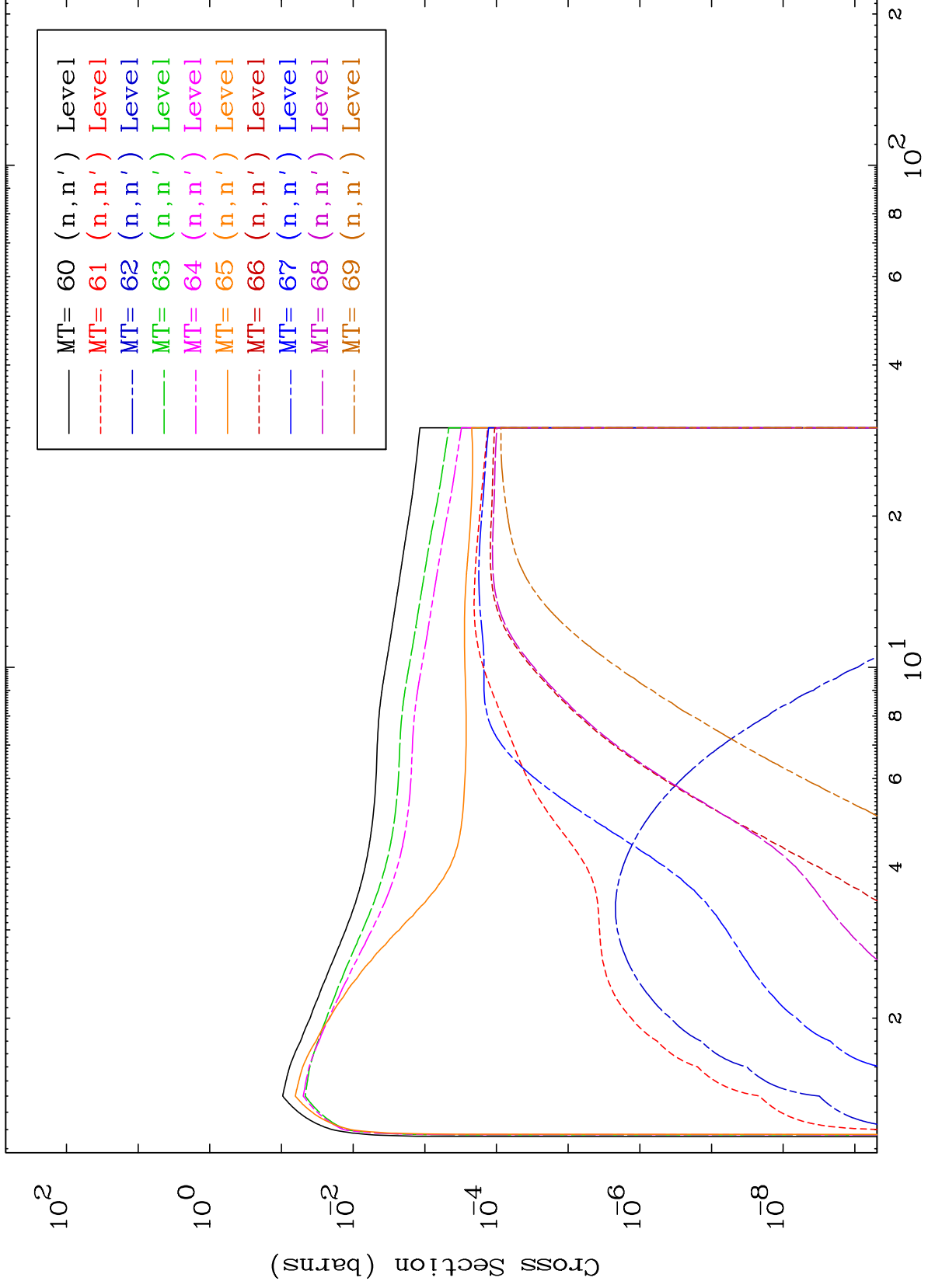


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Incident Energy (MeV)

104-Rf-257

293 Kelvin Cross Sections

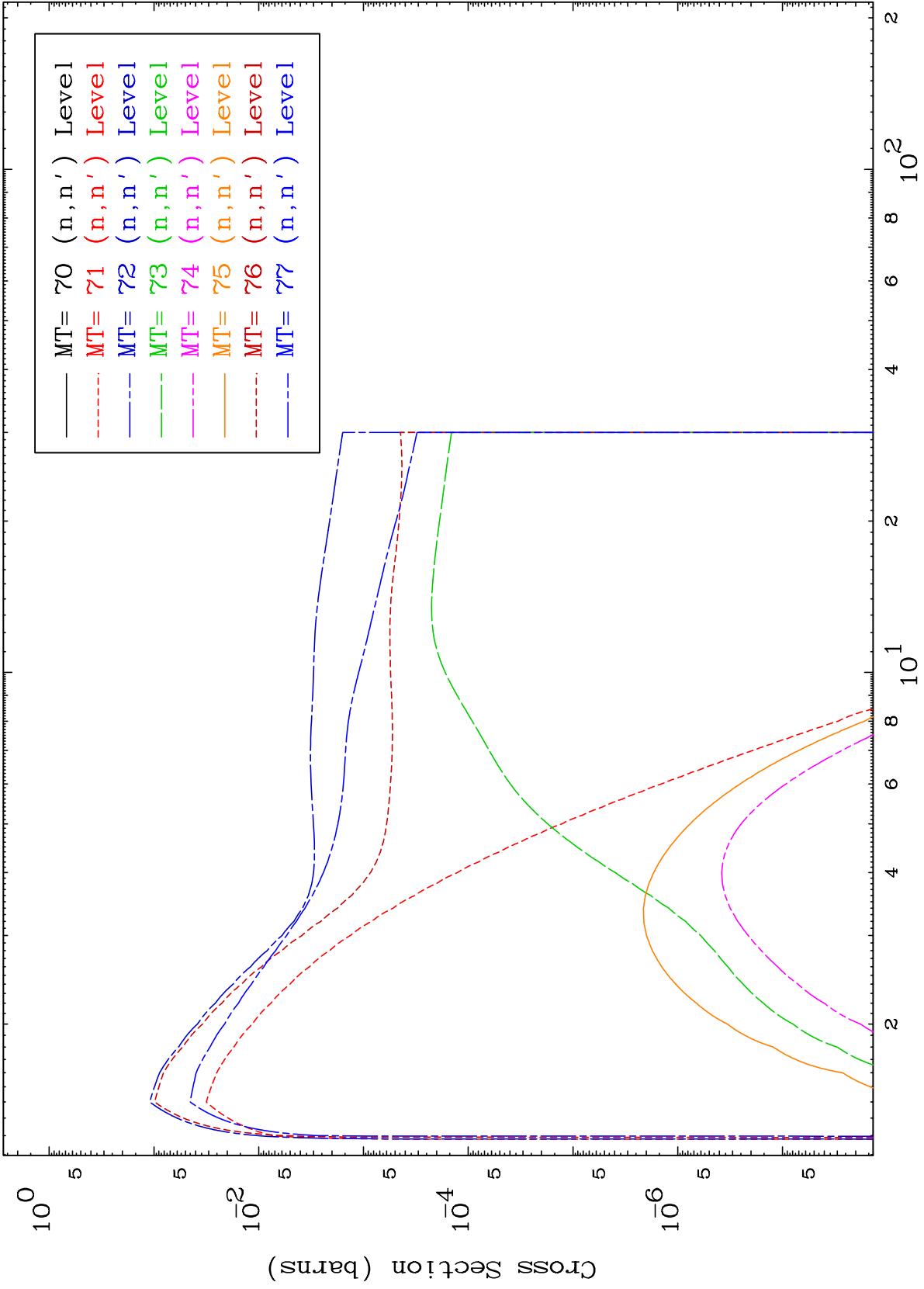


MAT 457

(n,n') Level

293 Kelvin Cross Sections

104-Rf-257



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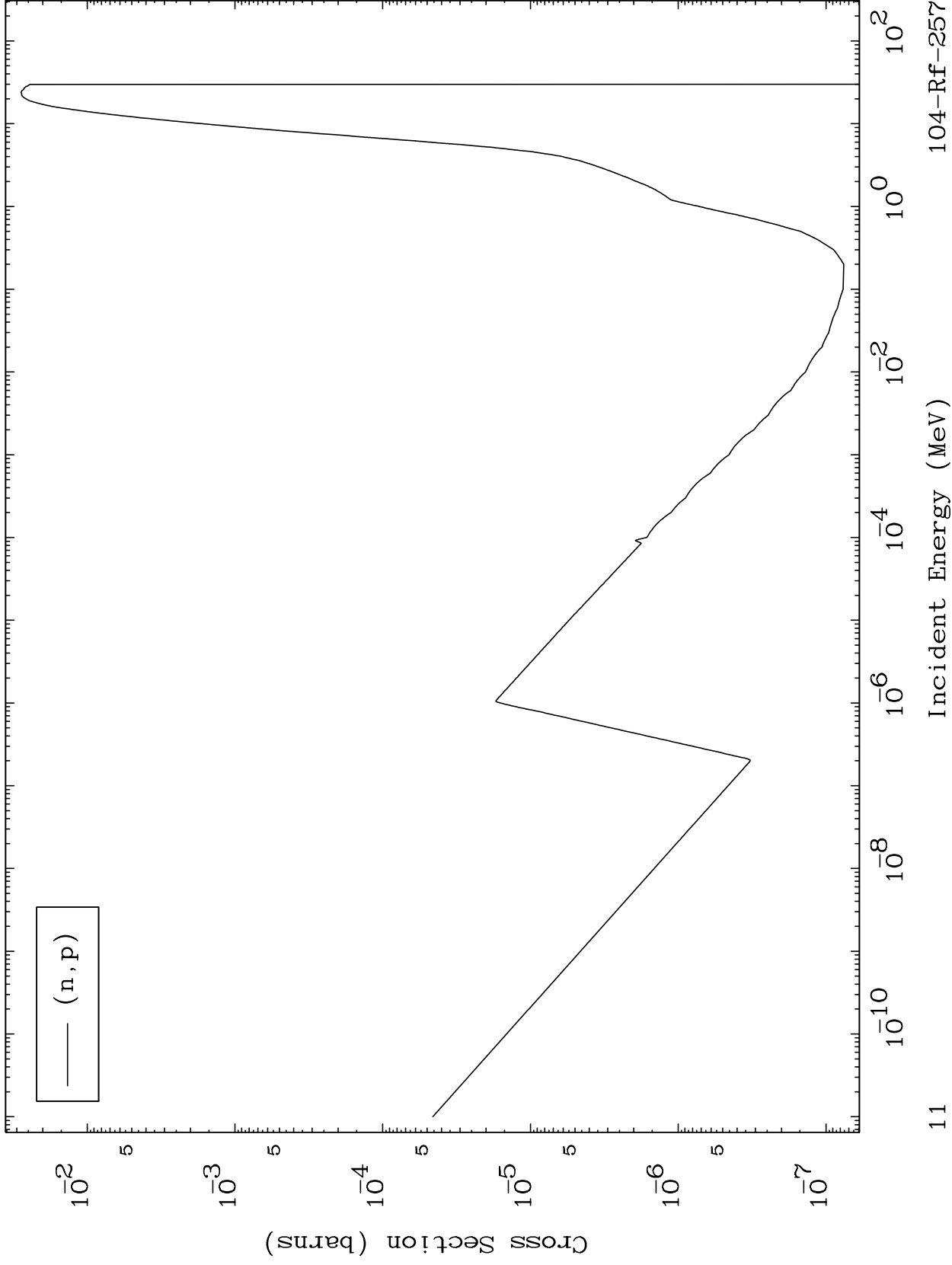
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels  
293 Kelvin Cross Sections

104-Rf-257



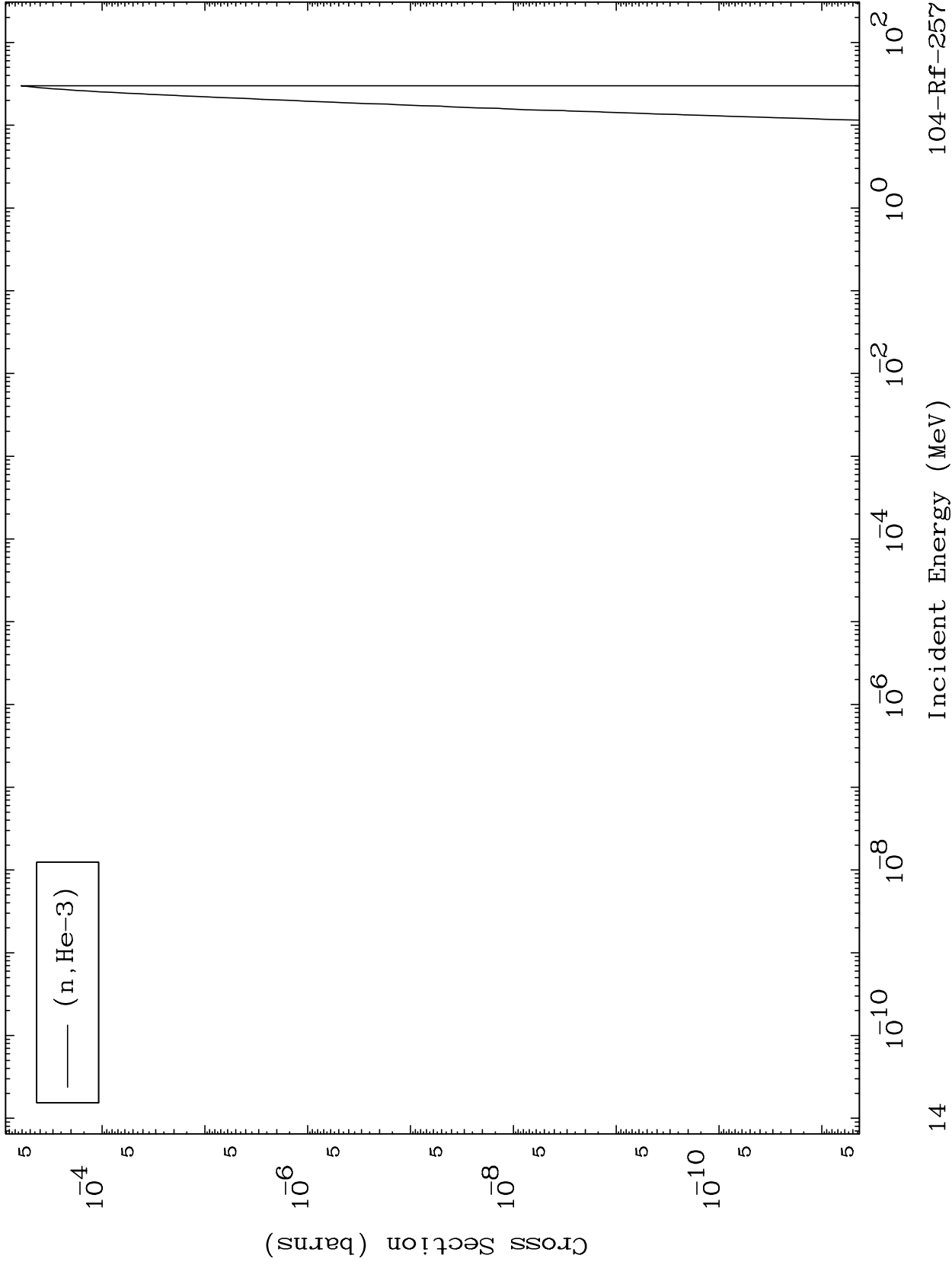


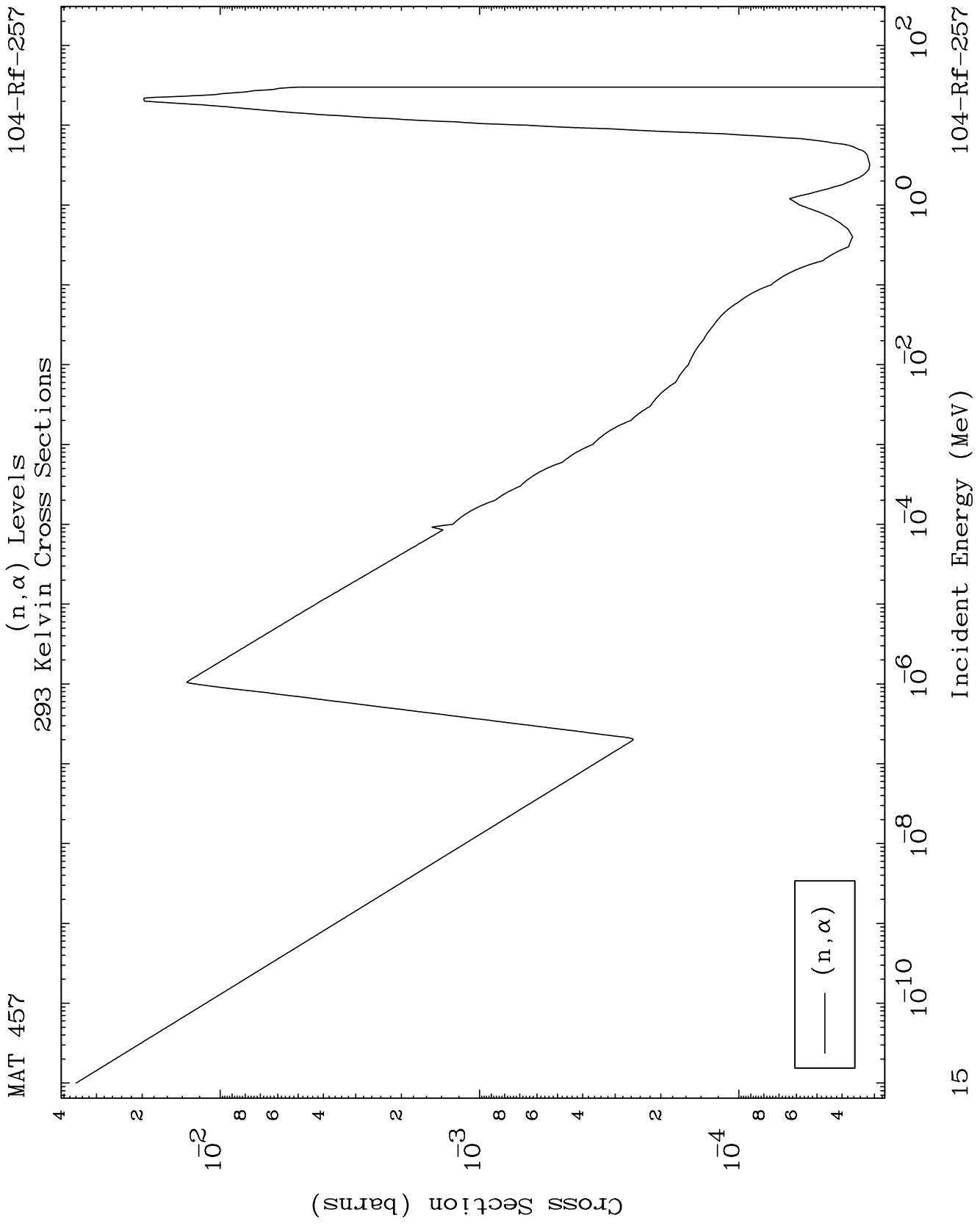


MAT 457

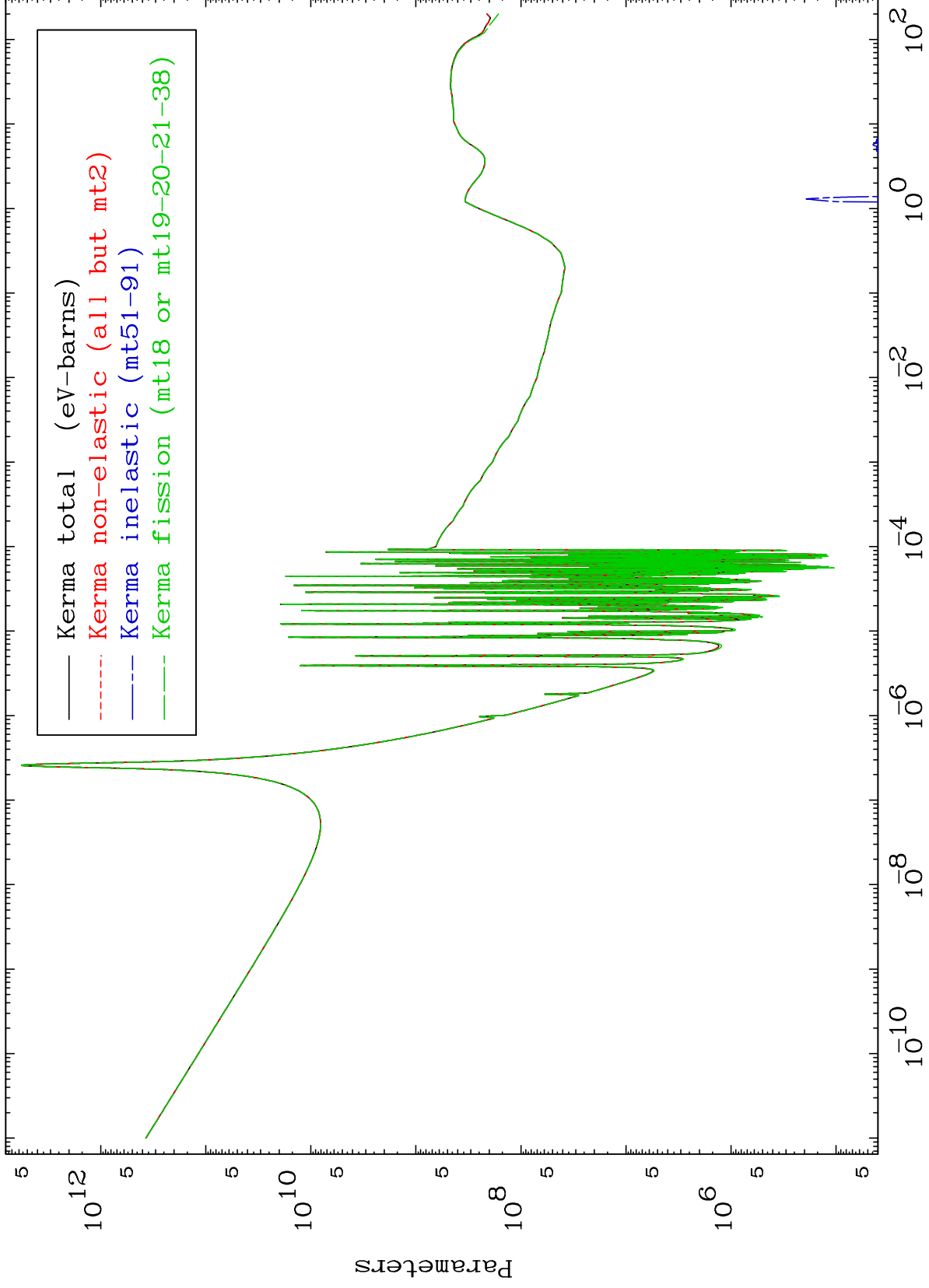
(n,He3) Levels  
293 Kelvin Cross Sections

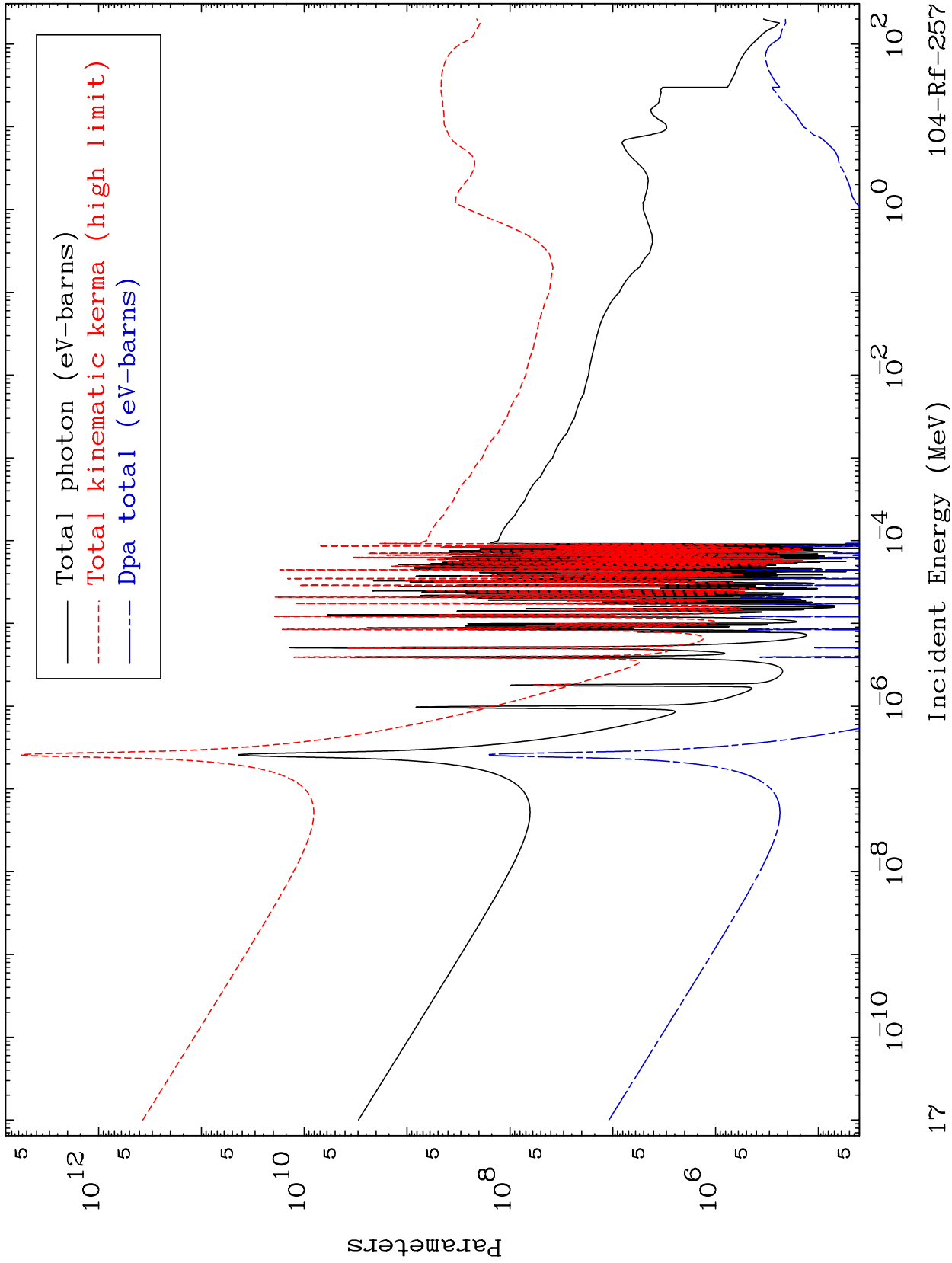
104-Rf-257

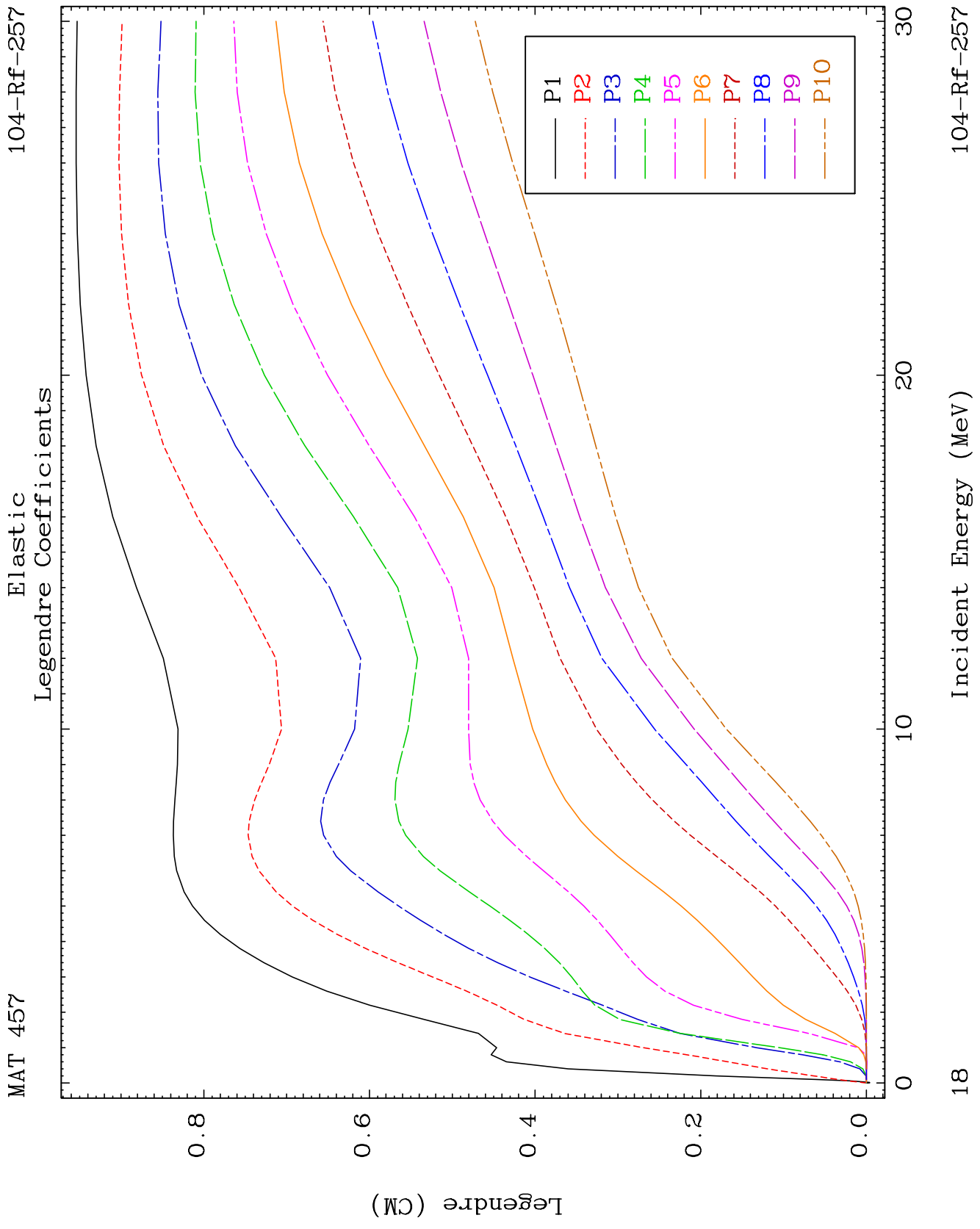








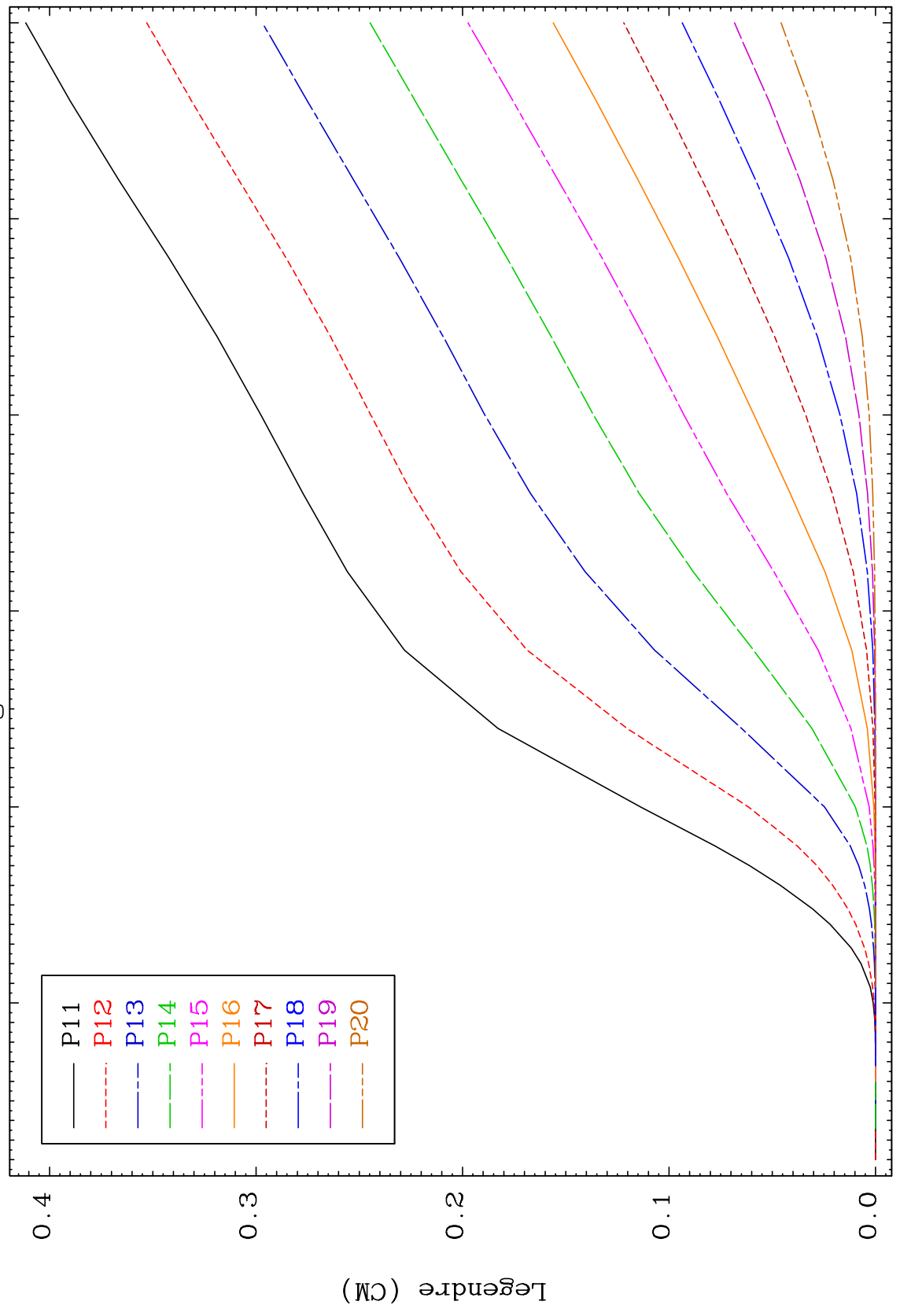




MAT 457

Elastic Legendre Coefficients

104-Rf-257



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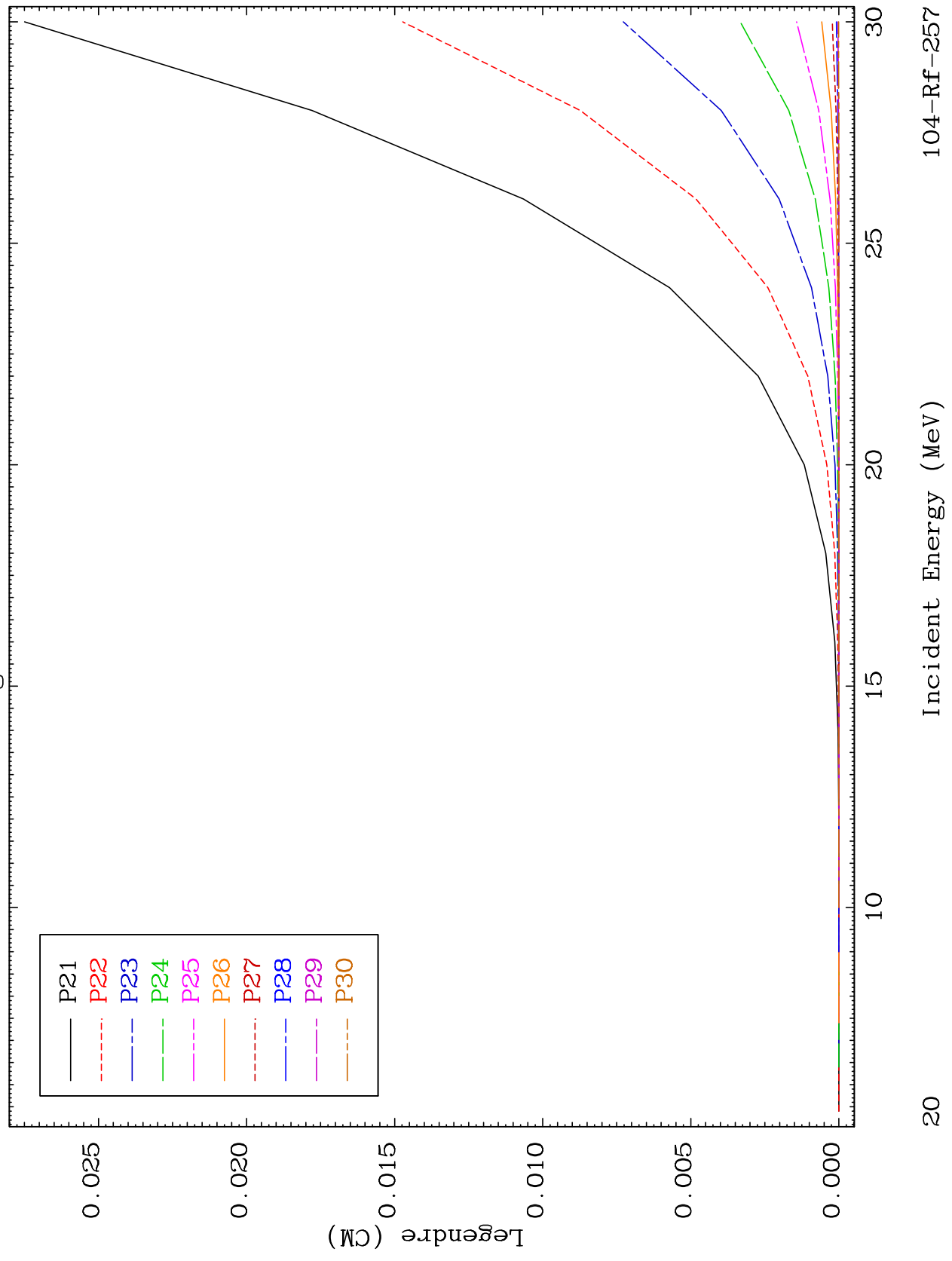
Incident Energy (MeV)

104-Rf-257

MAT 457

### Elastic Legendre Coefficients

104-Rf-257



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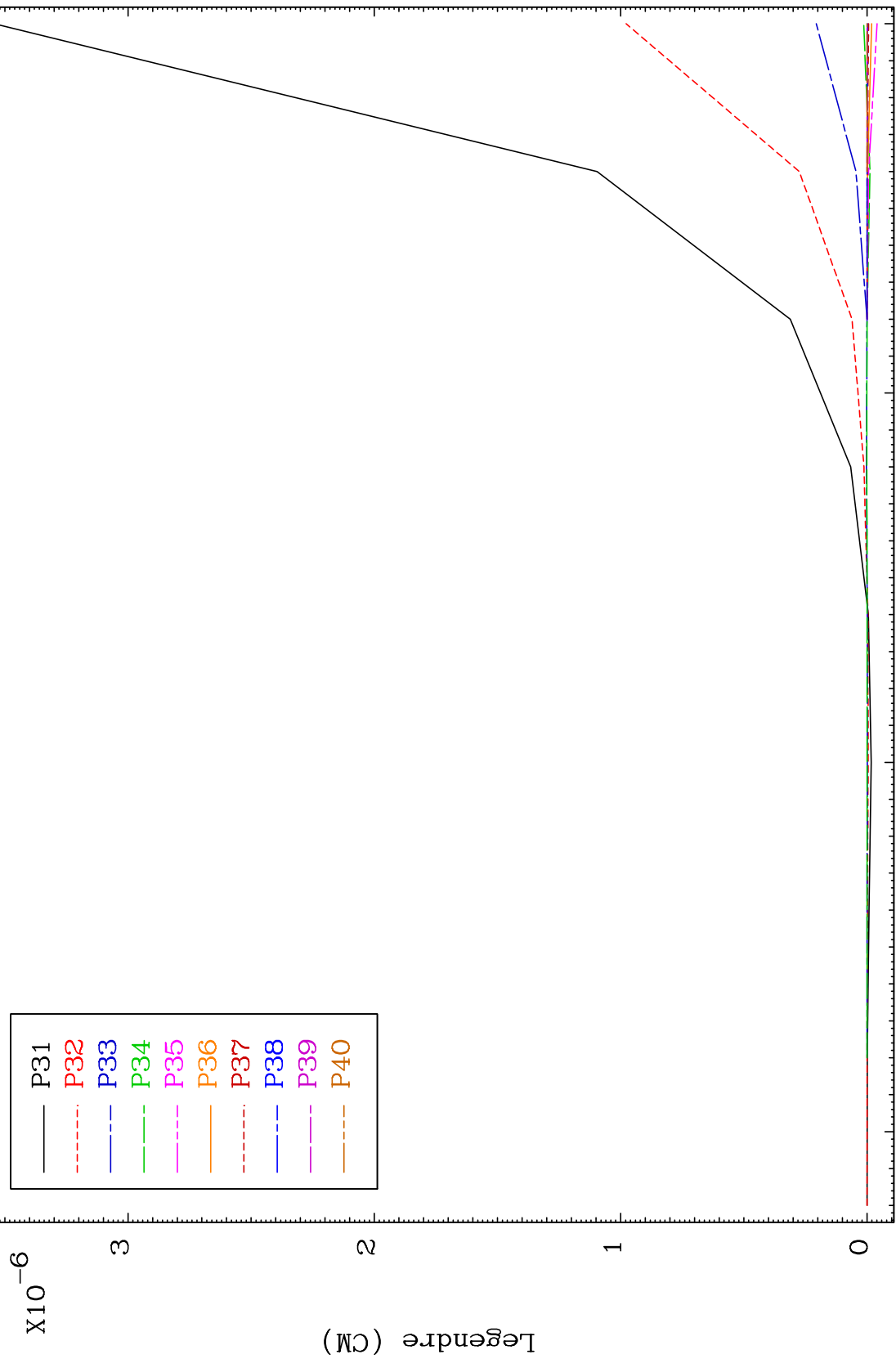
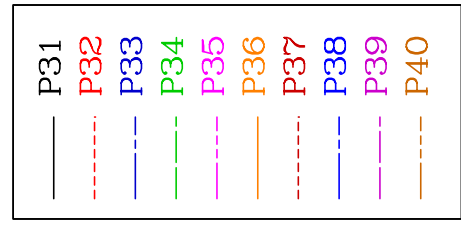
Incident Energy (MeV)

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257

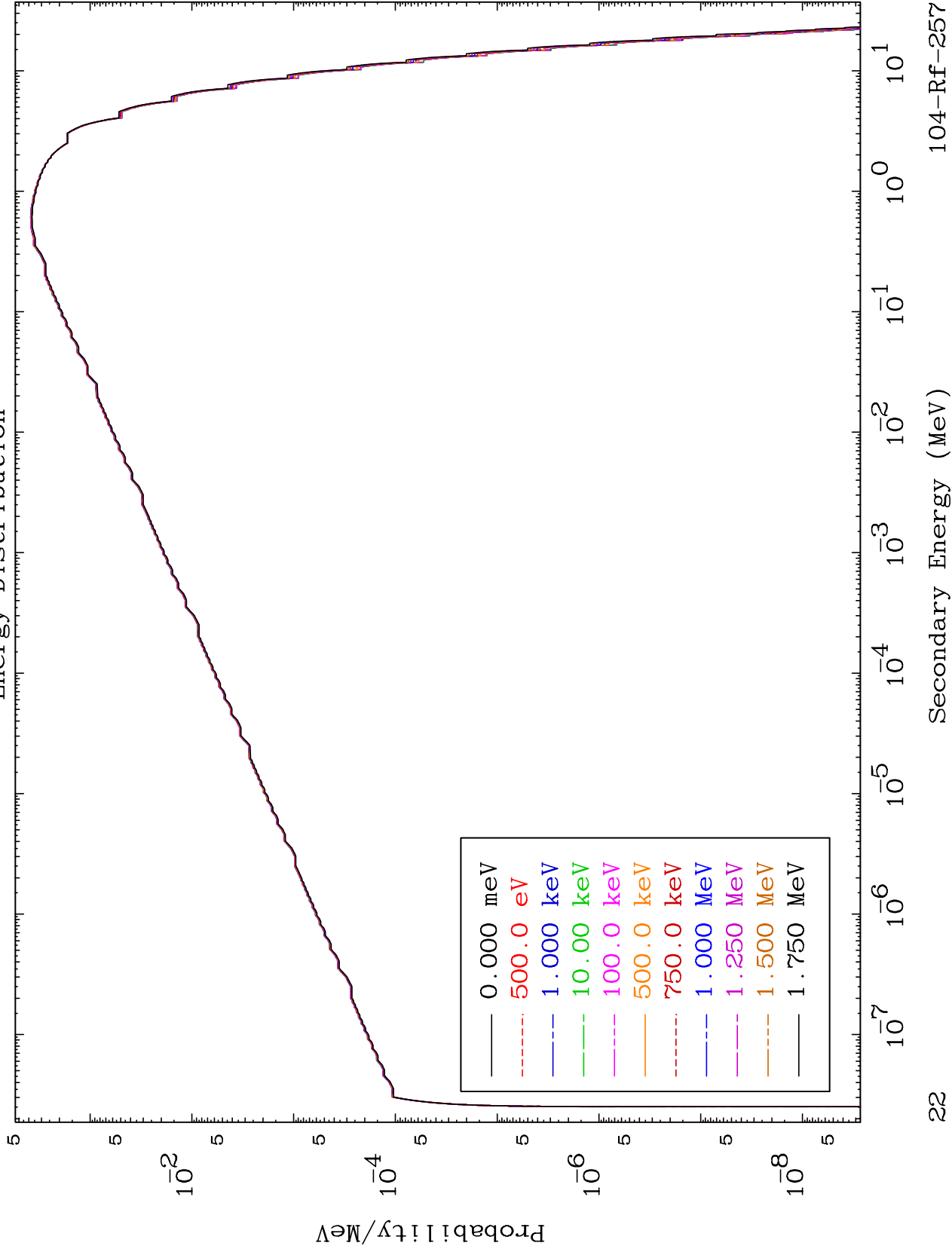


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Incident Energy (MeV)

104-Rf-257

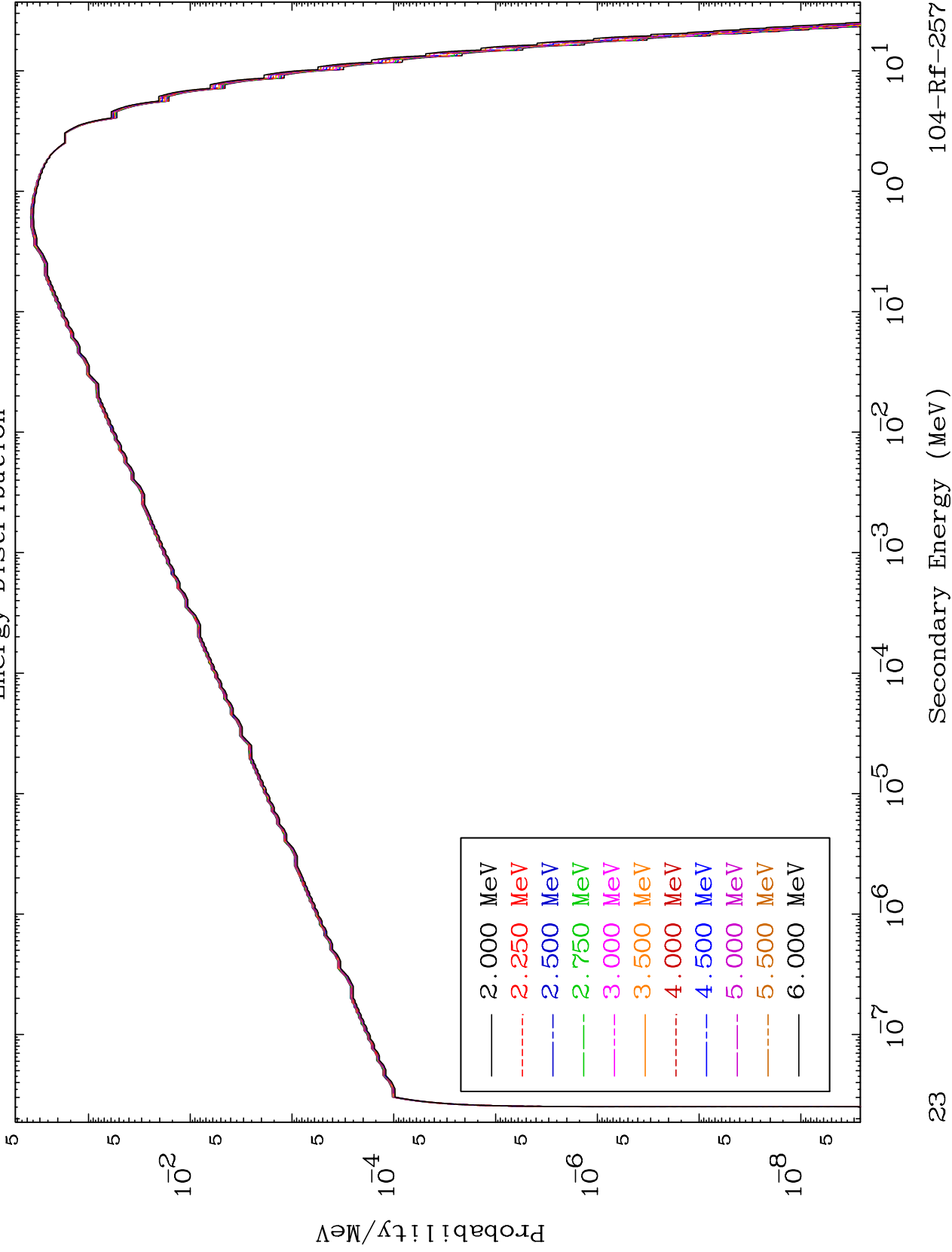
# Fission Energy Distribution



MAT 457

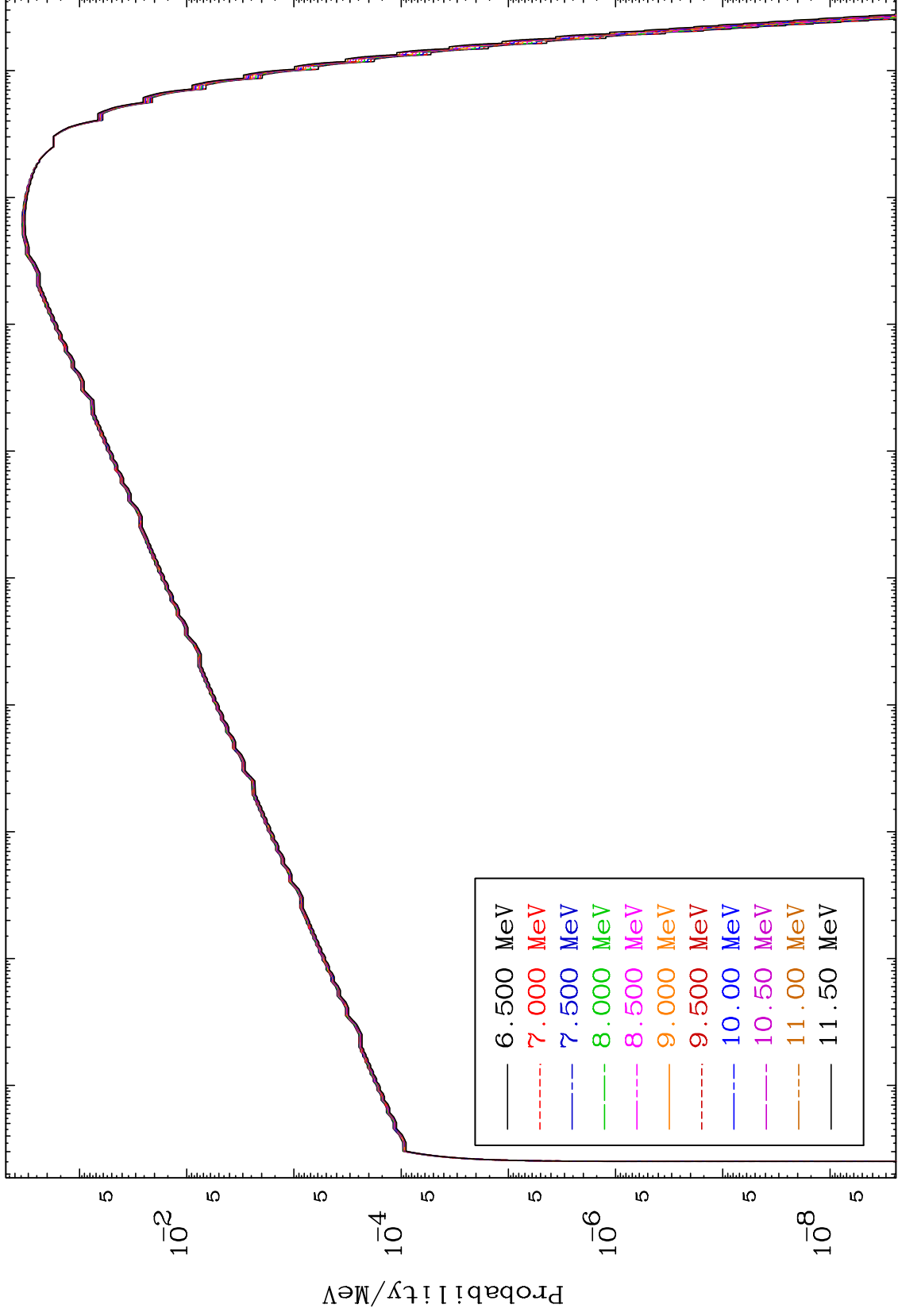
# Fission Energy Distribution

104-Rf-257

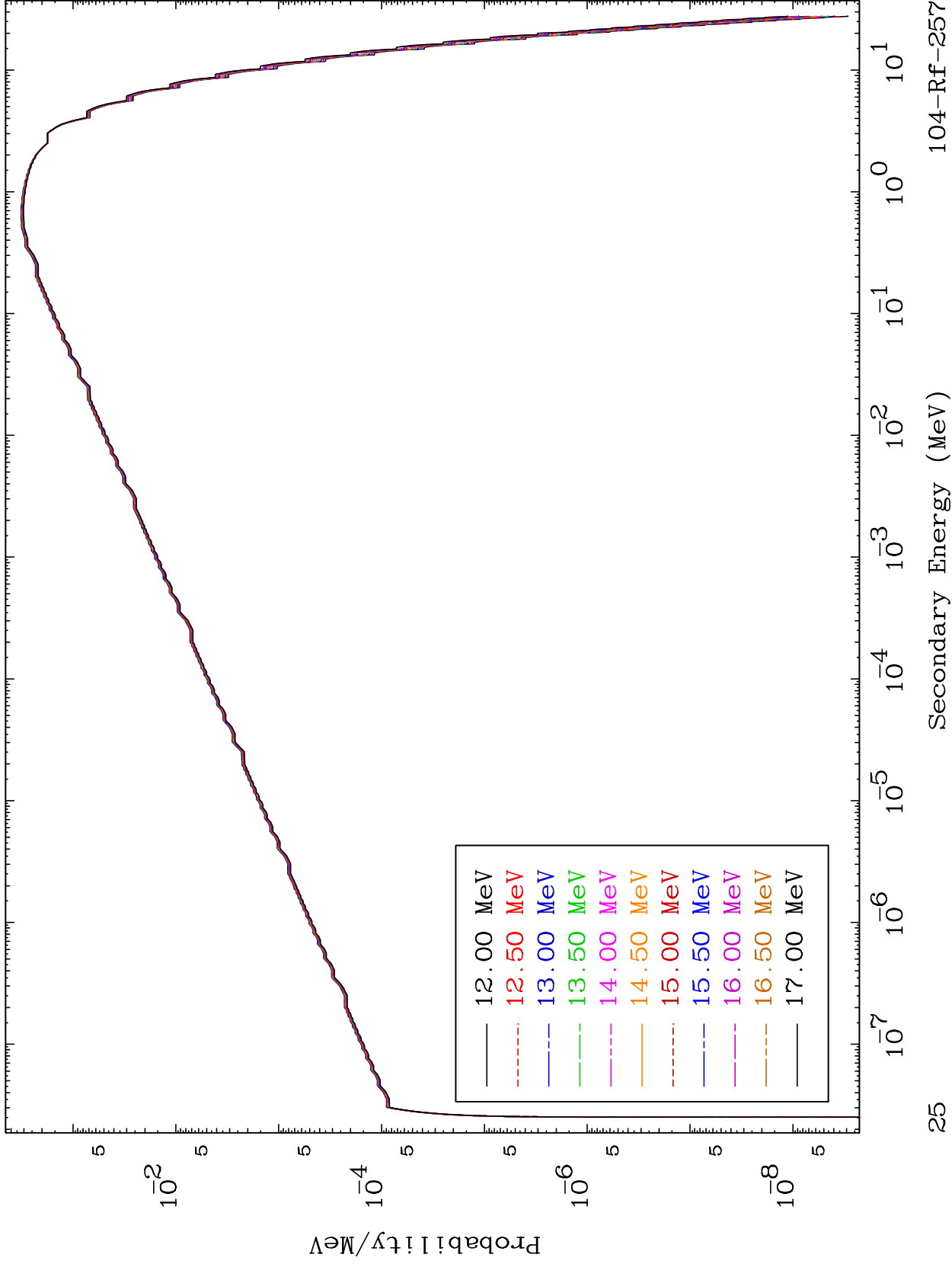




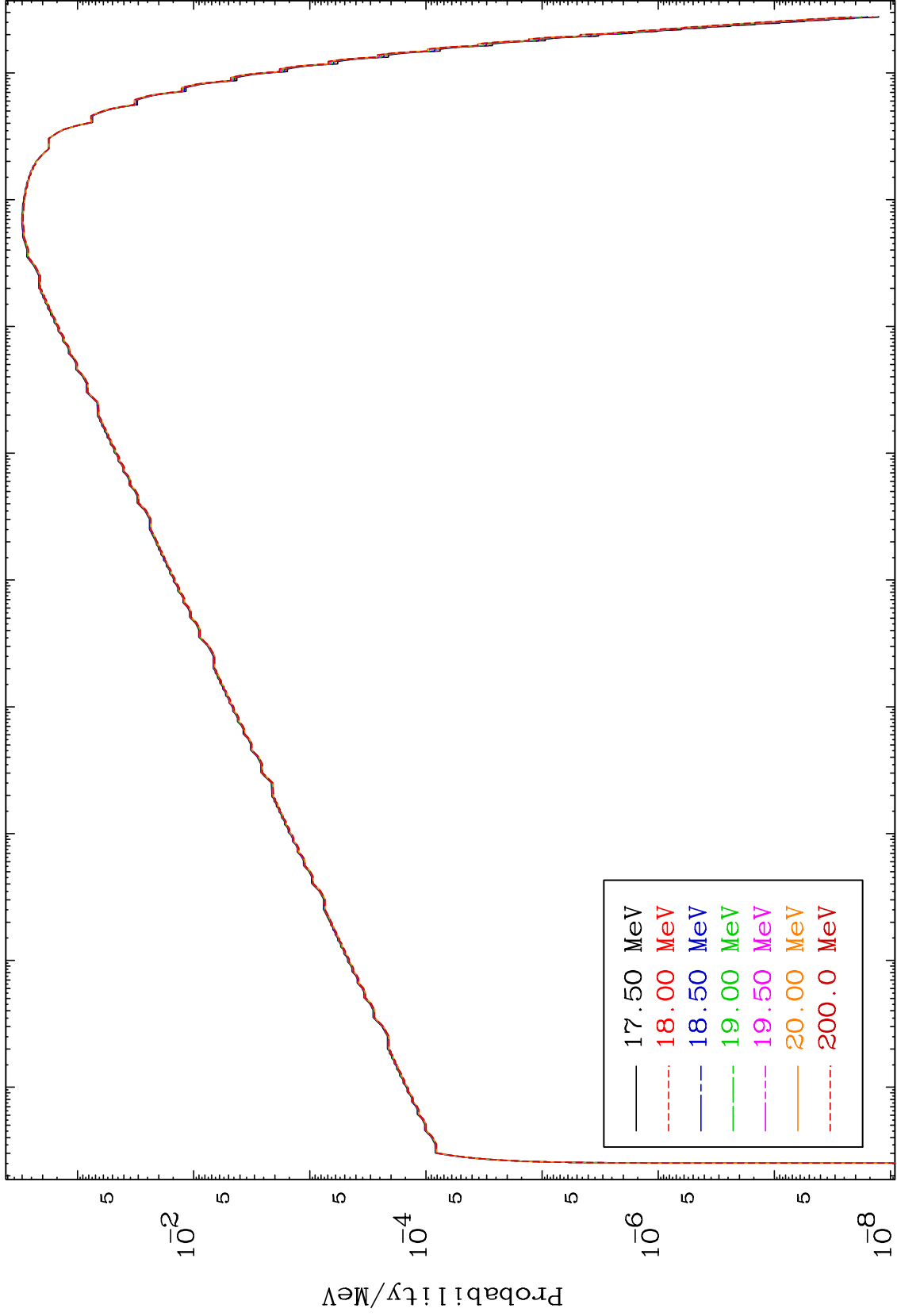
Fission  
Energy Distribution



Fission  
Energy Distribution



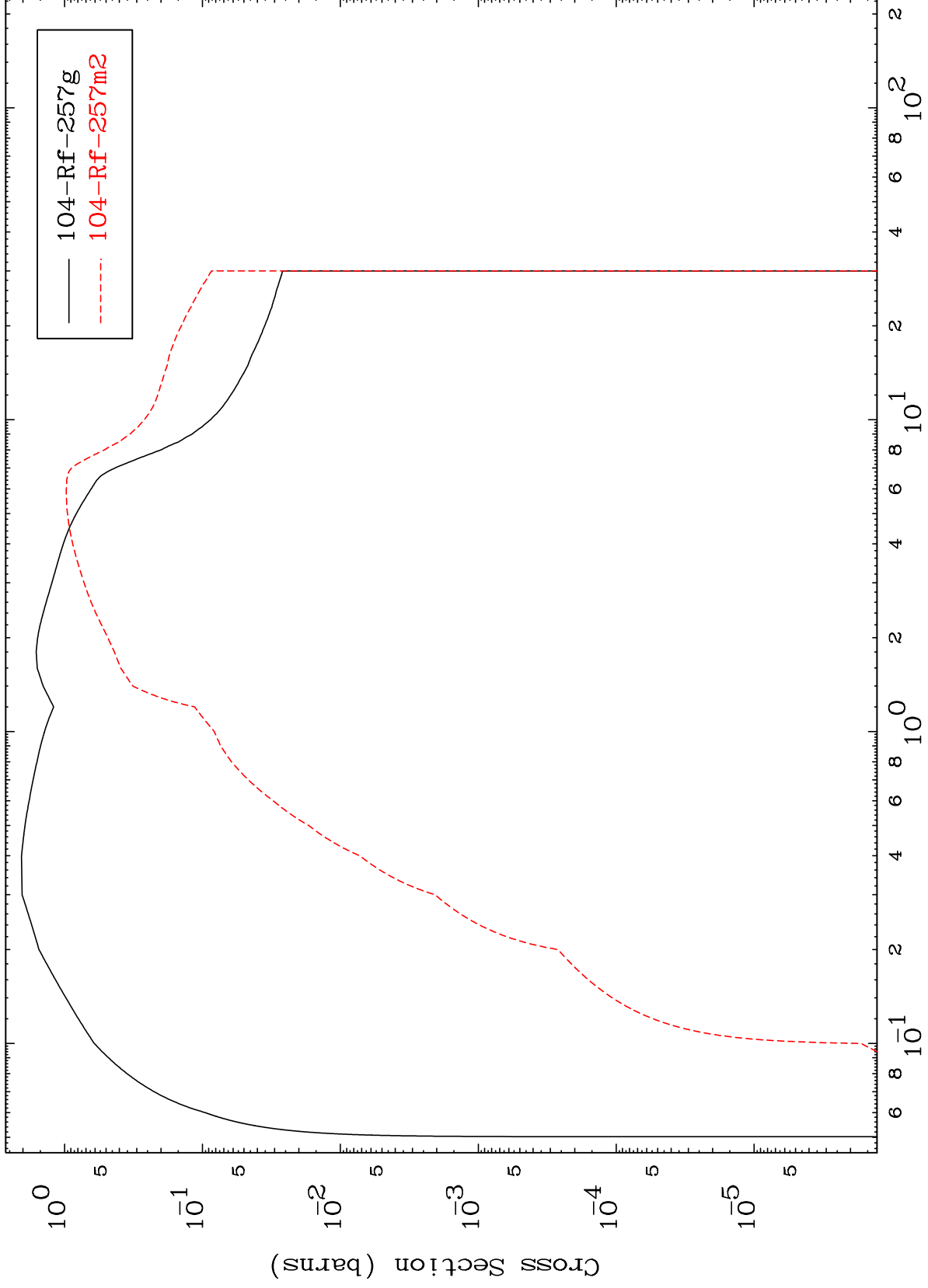
### Fission Energy Distribution



MAT 457

104-Rf-257

Inelastic  
Radionuclide Production Cross Section



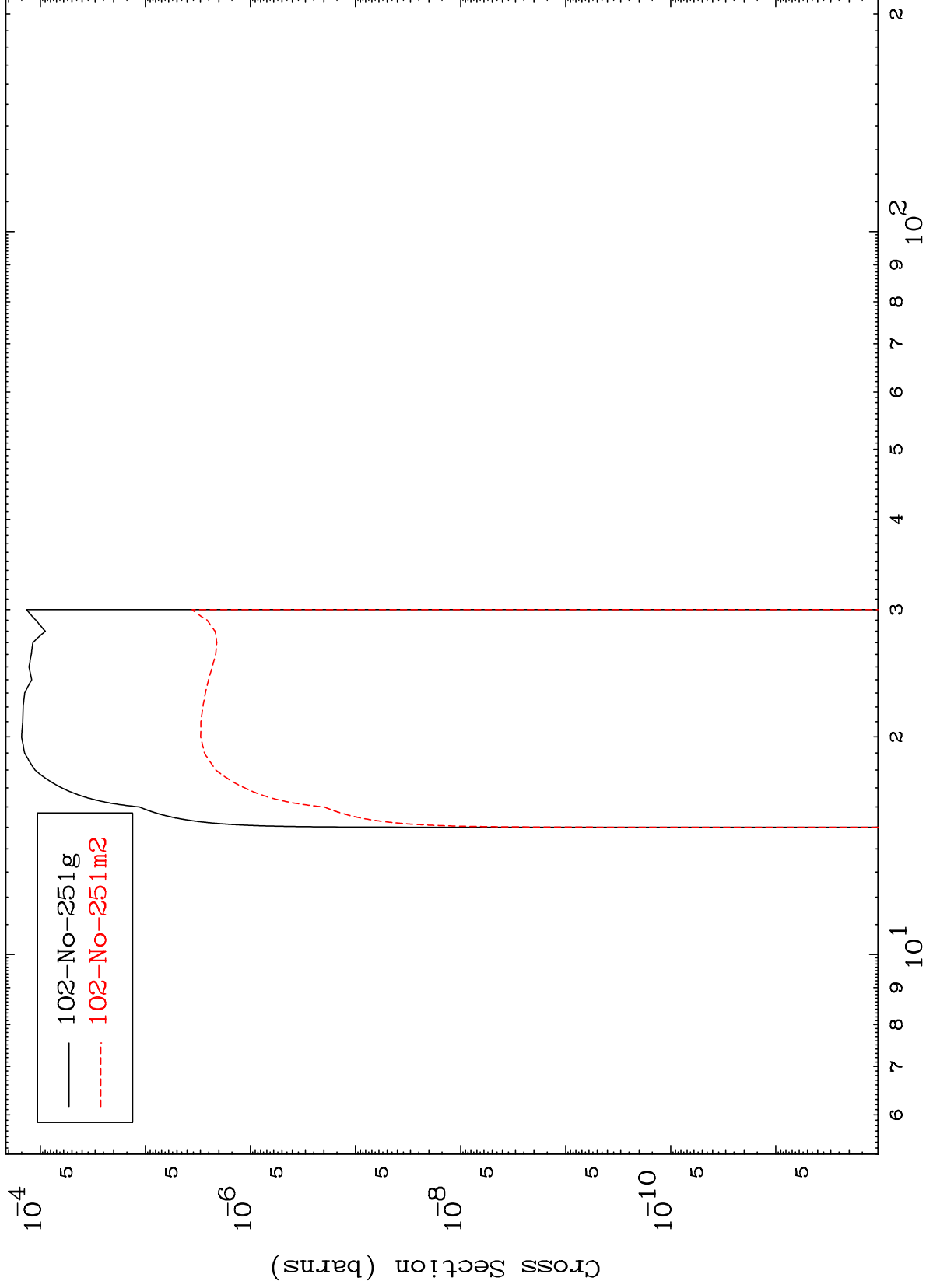
104-Rf-257g  
104-Rf-257m2

MAT 457

(n,3n)  $\alpha$

104-Rf-257

Radionuclide Production Cross Section

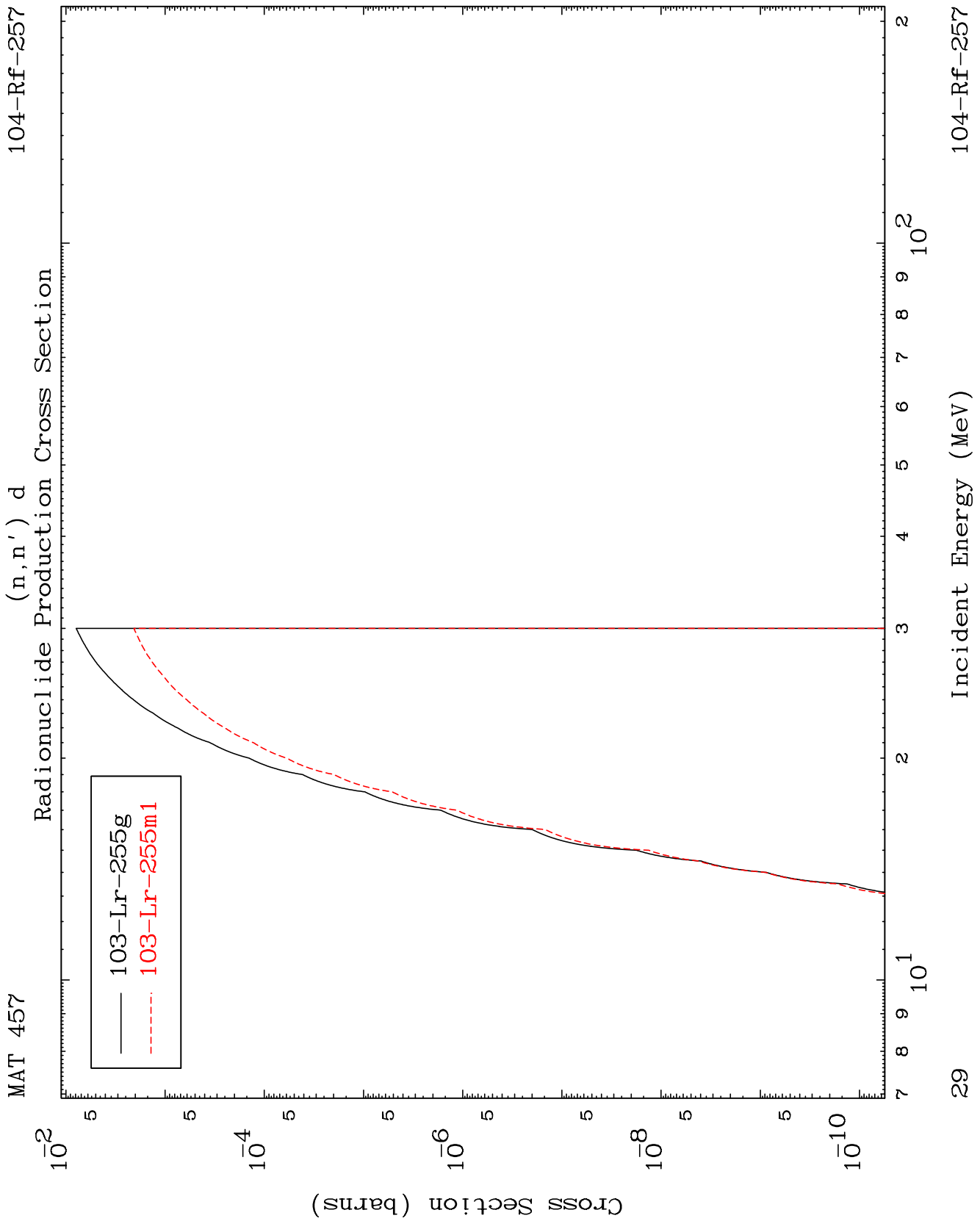


— 102-No-251 g  
- - - 102-No-251 m2

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Incident Energy (MeV)

104-Rf-257

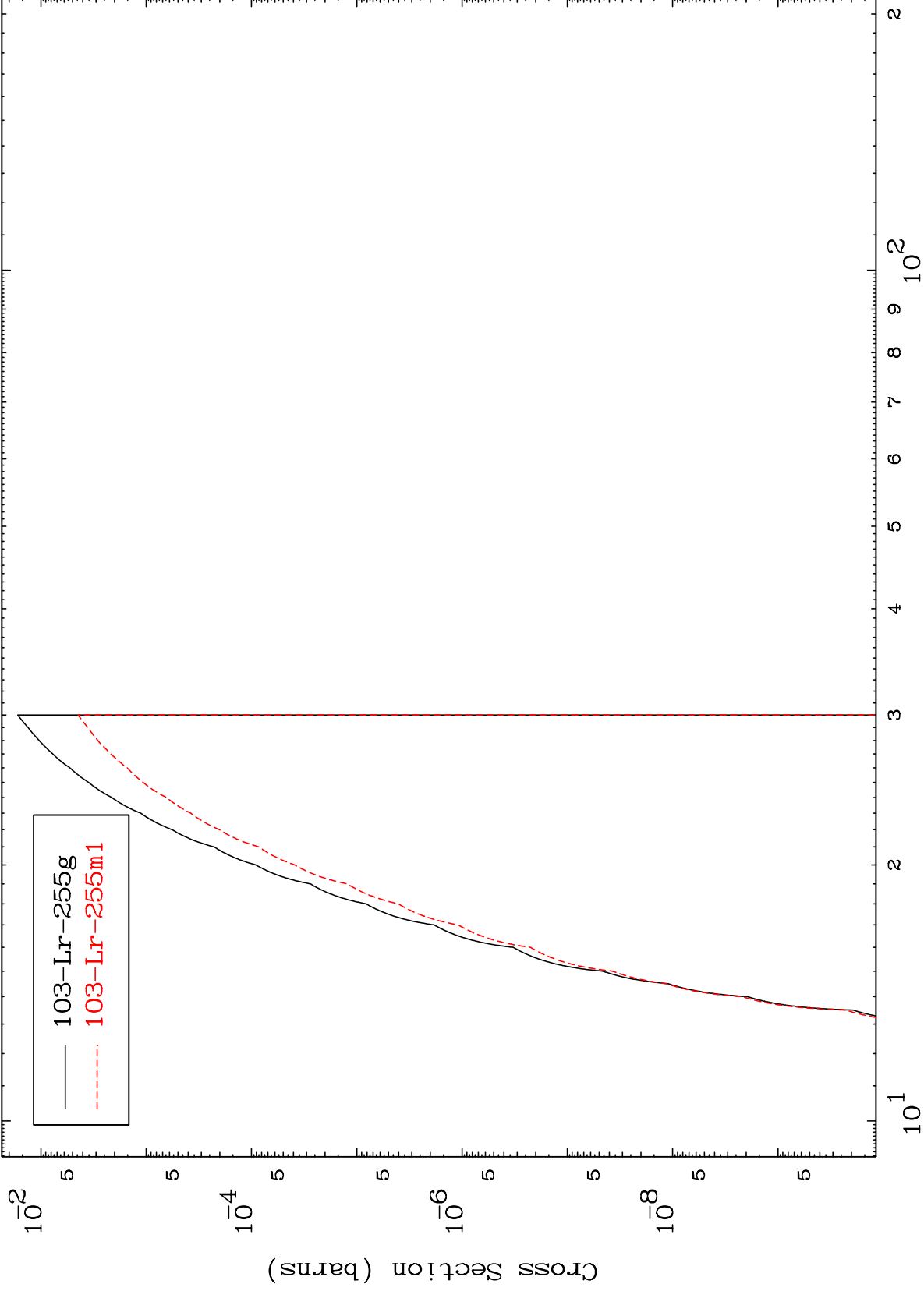


MAT 457

(n,2n) p

104-Rf-257

Radionuclide Production Cross Section



Incident Energy (MeV)

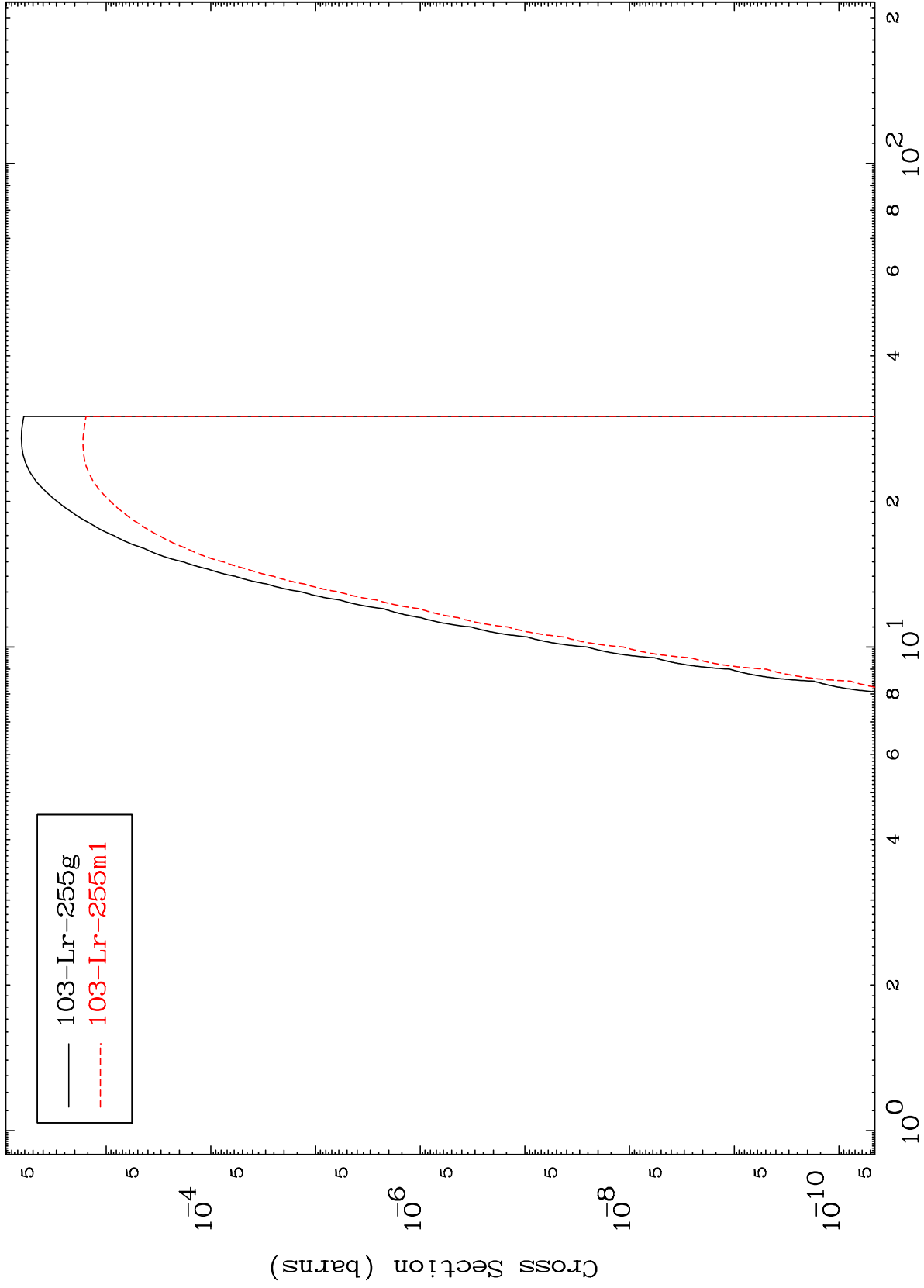
104-Rf-257

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MAT 457

104-Rf-257

(n, t)  
Radionuclide Production Cross Section



31

Incident Energy (MeV)

104-Rf-257

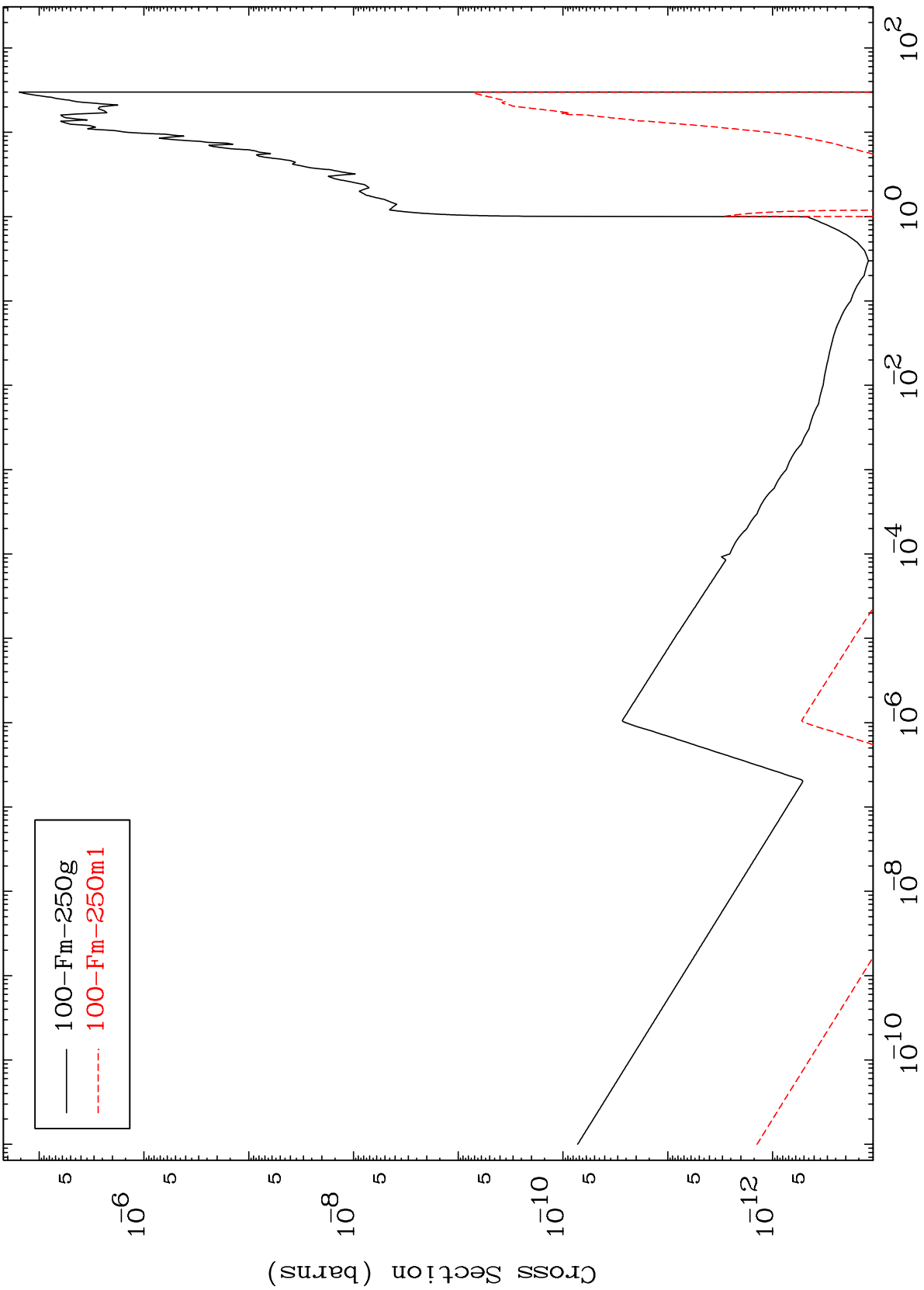


MAT 457

(n,2α)

104-Rf-257

Radionuclide Production Cross Section



104-Rf-257

Incident Energy (MeV)