

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

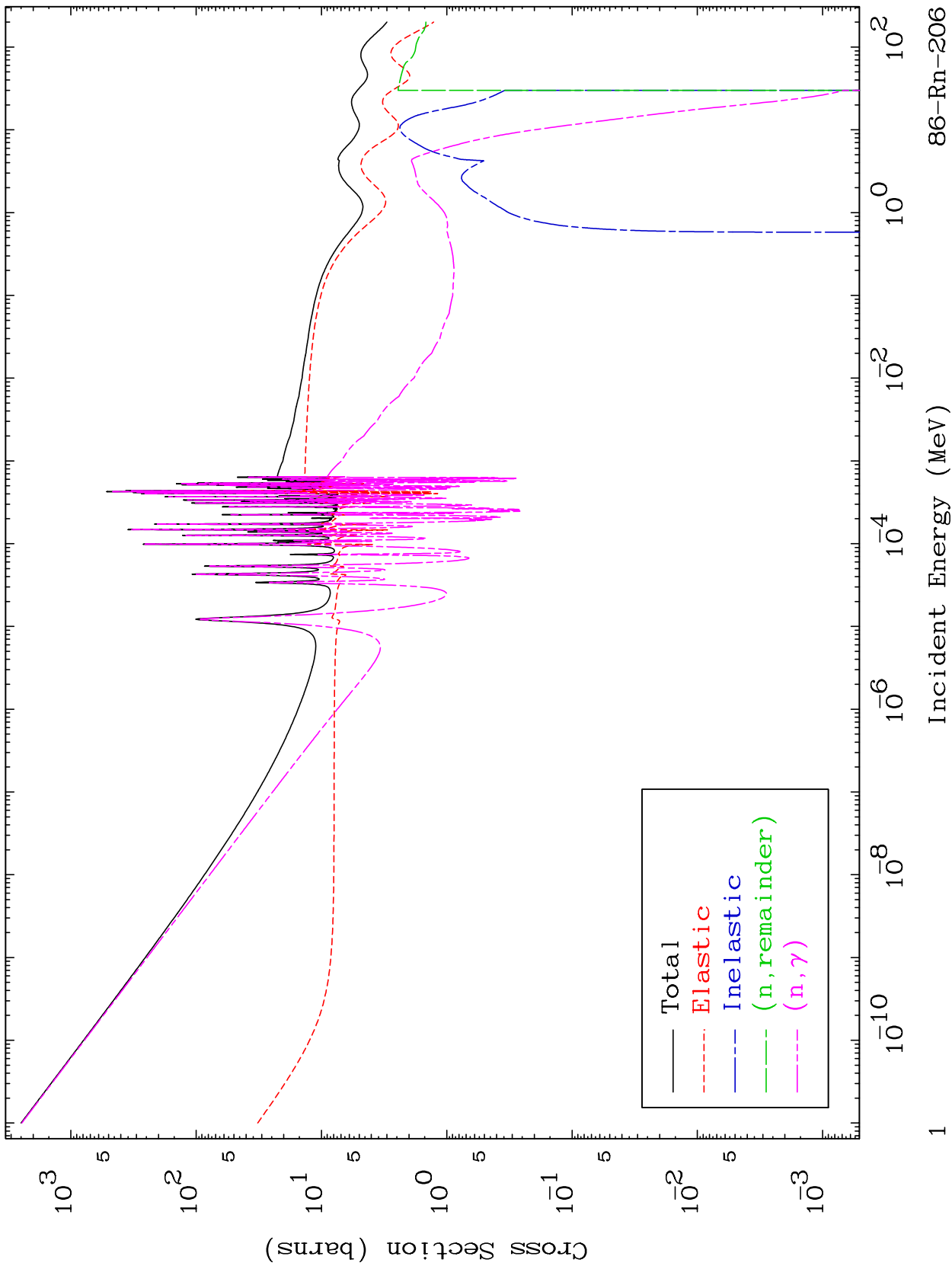
Press Mouse Button to Start

MAT 8610

Major

293 Kelvin Cross Sections

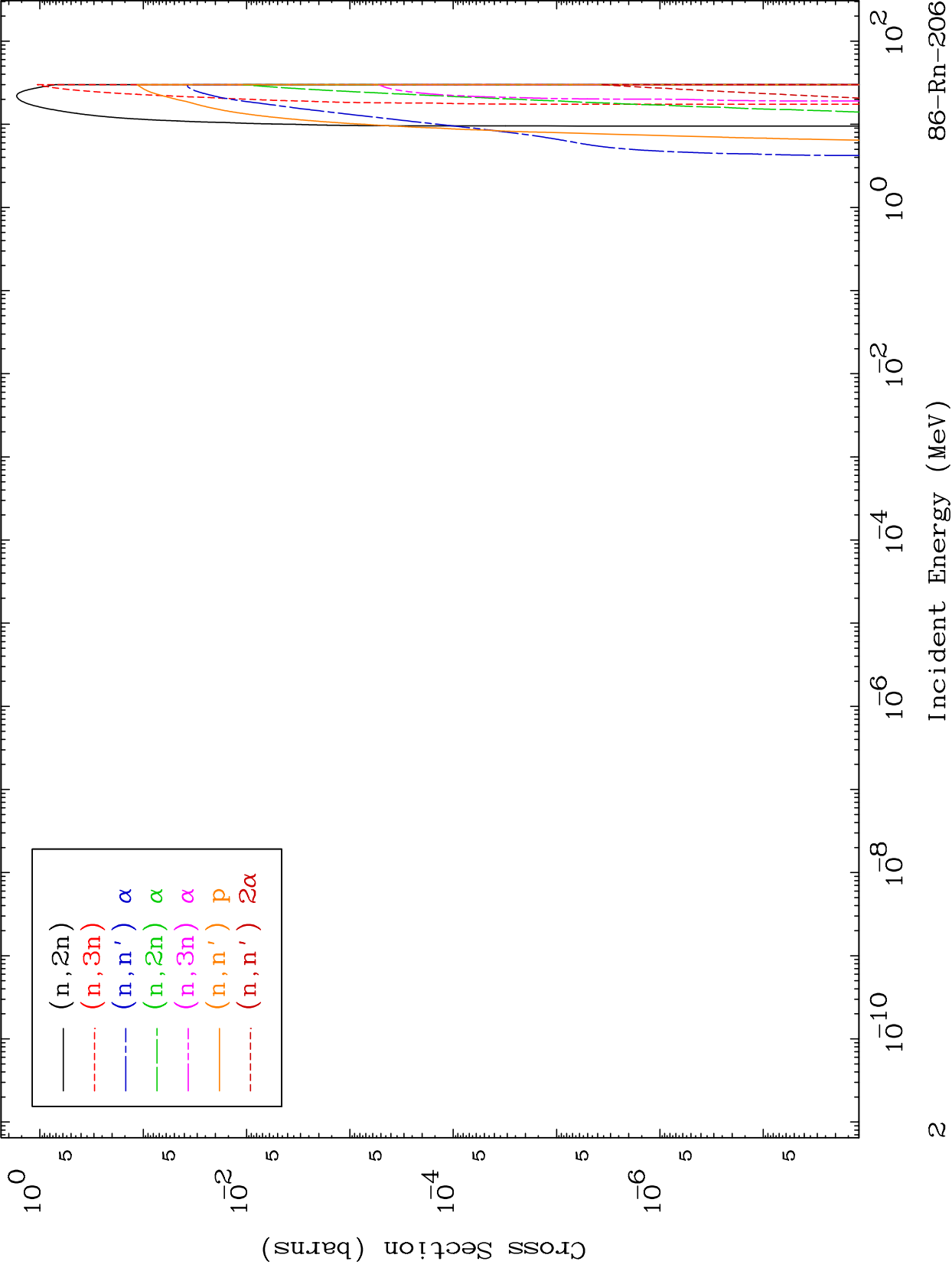
86-Rn-206



MAT 8610

Neutron Production
293 Kelvin Cross Sections

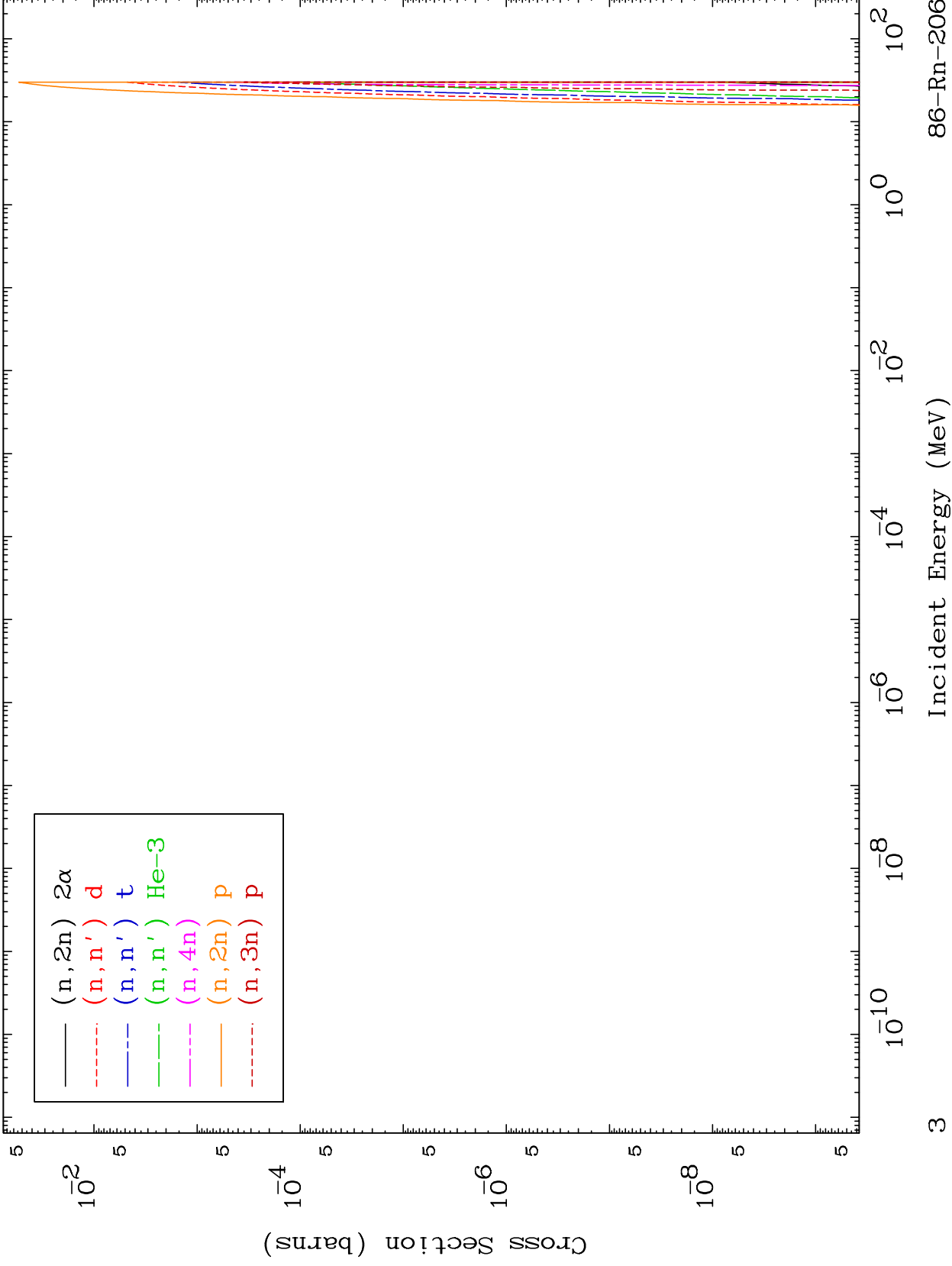
86-Rn-206



MAT 8610

Neutron Production
293 Kelvin Cross Sections

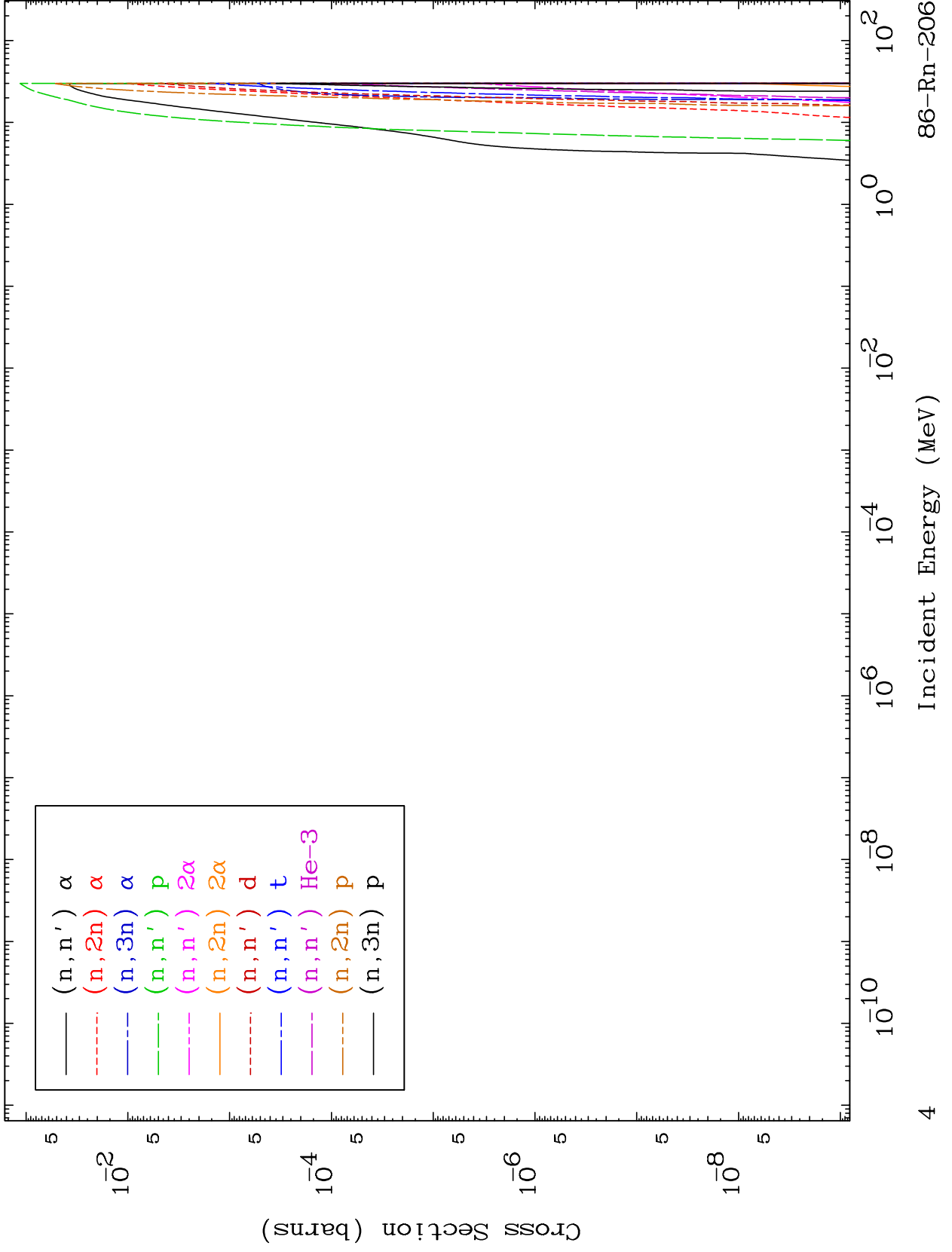
86-Rn-206



MAT 8610

Charged Particle
293 Kelvin Cross Sections

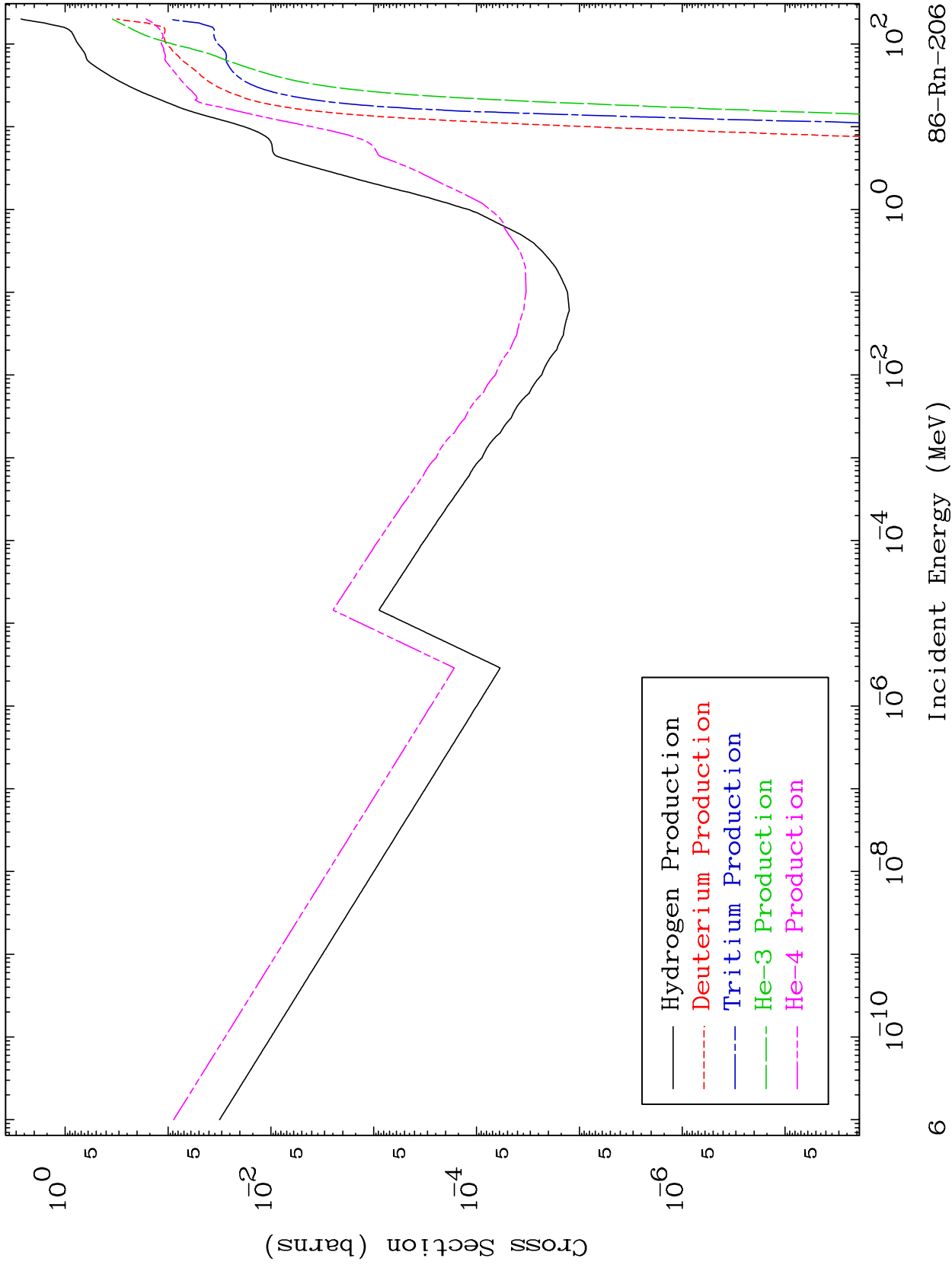
86-Rn-206



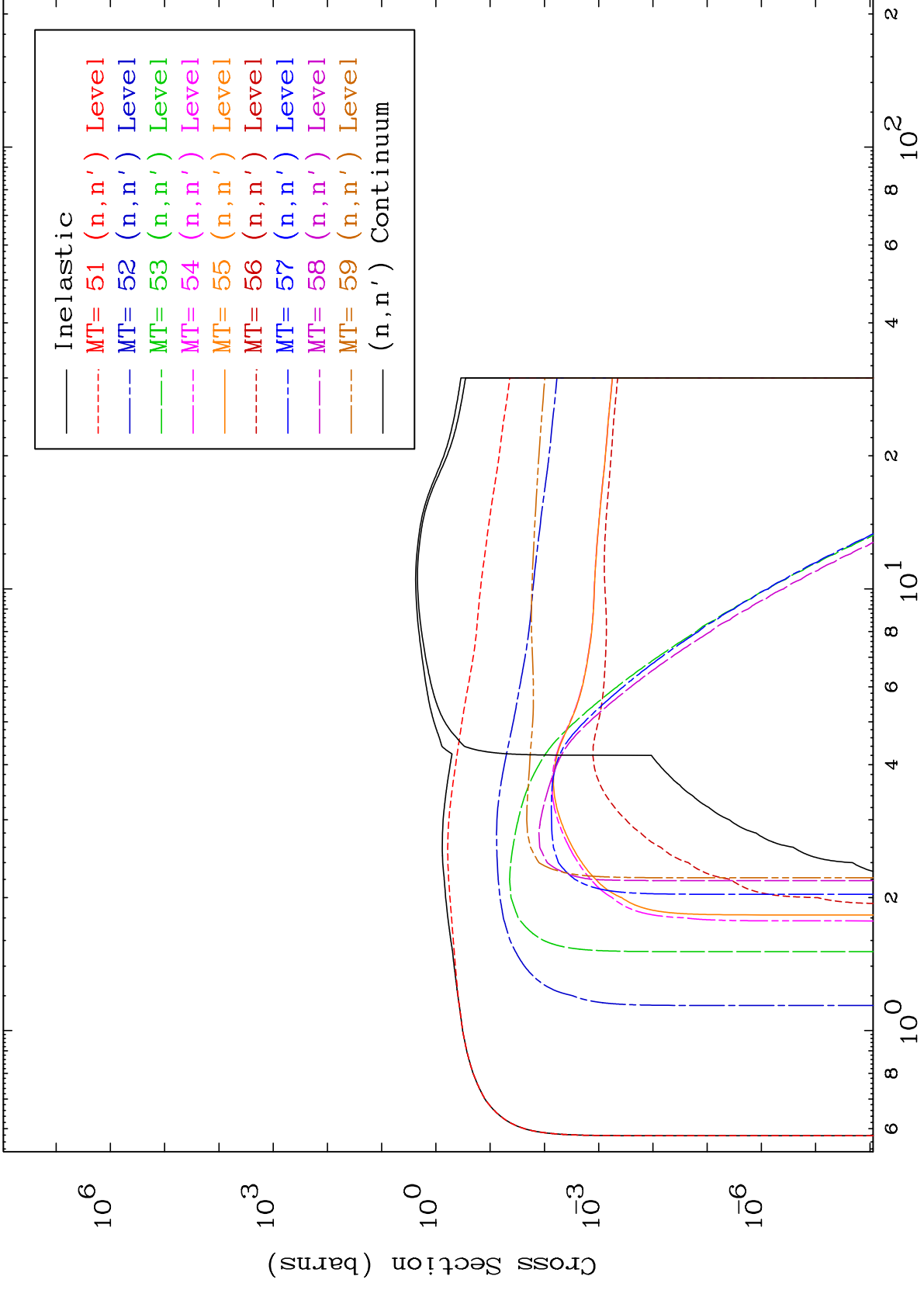
MAT 8610

Particle Production
293 Kelvin Cross Sections

86-Rn-206



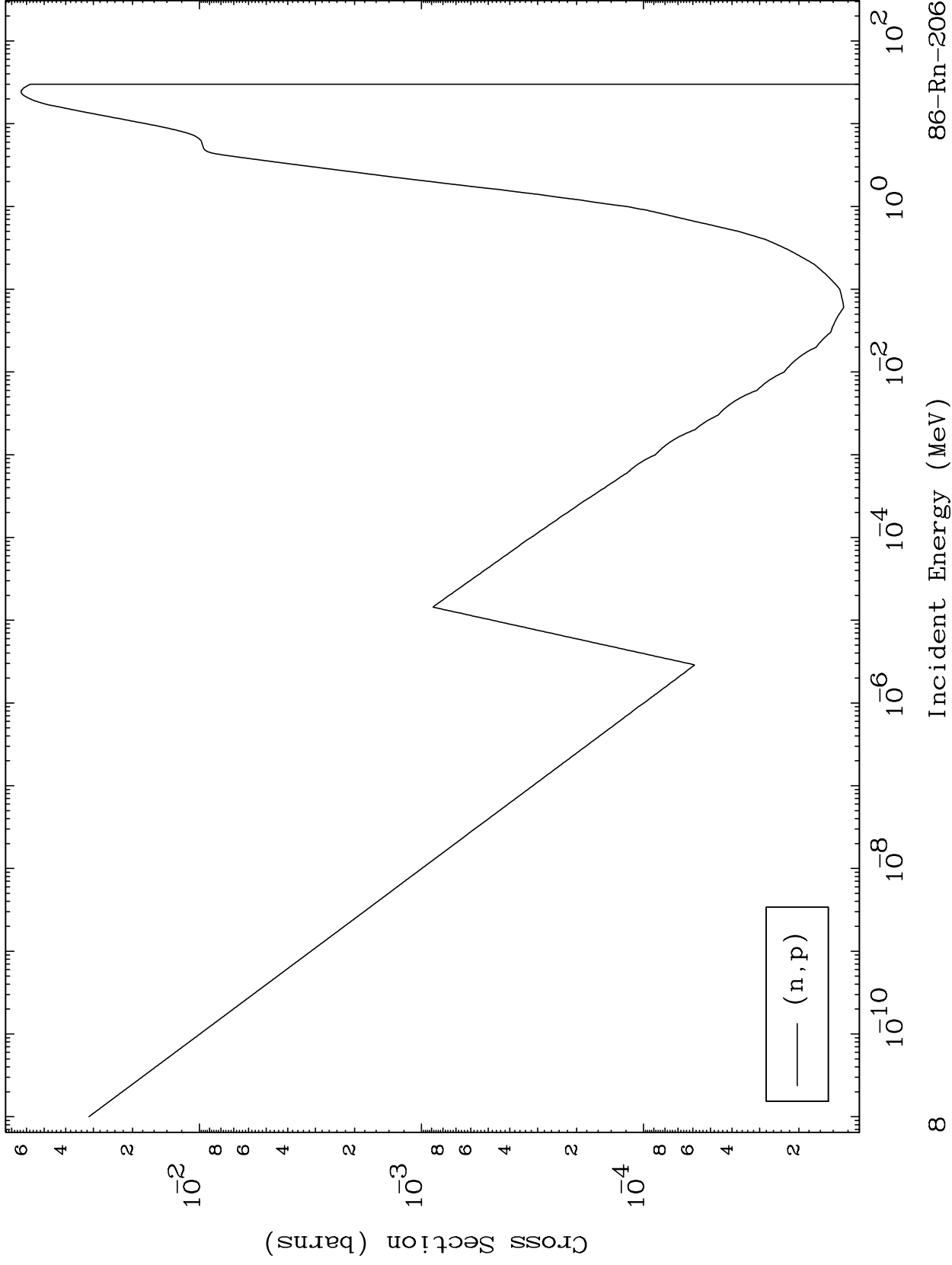
293 Kelvin Cross Sections



MAT 8610

(n,p) Levels
293 Kelvin Cross Sections

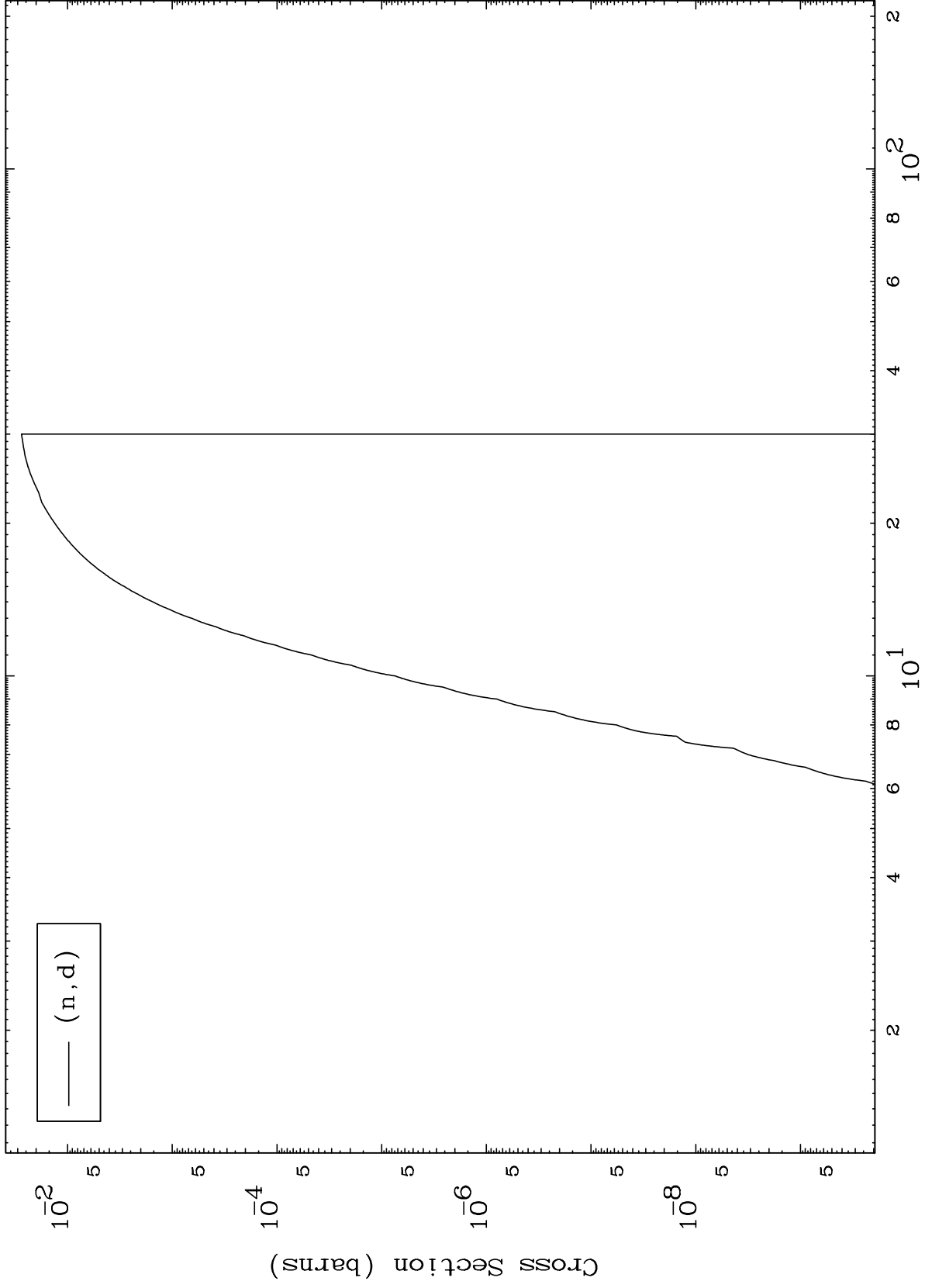
86-Rn-206



MAT 8610

(n,d) Levels
293 Kelvin Cross Sections

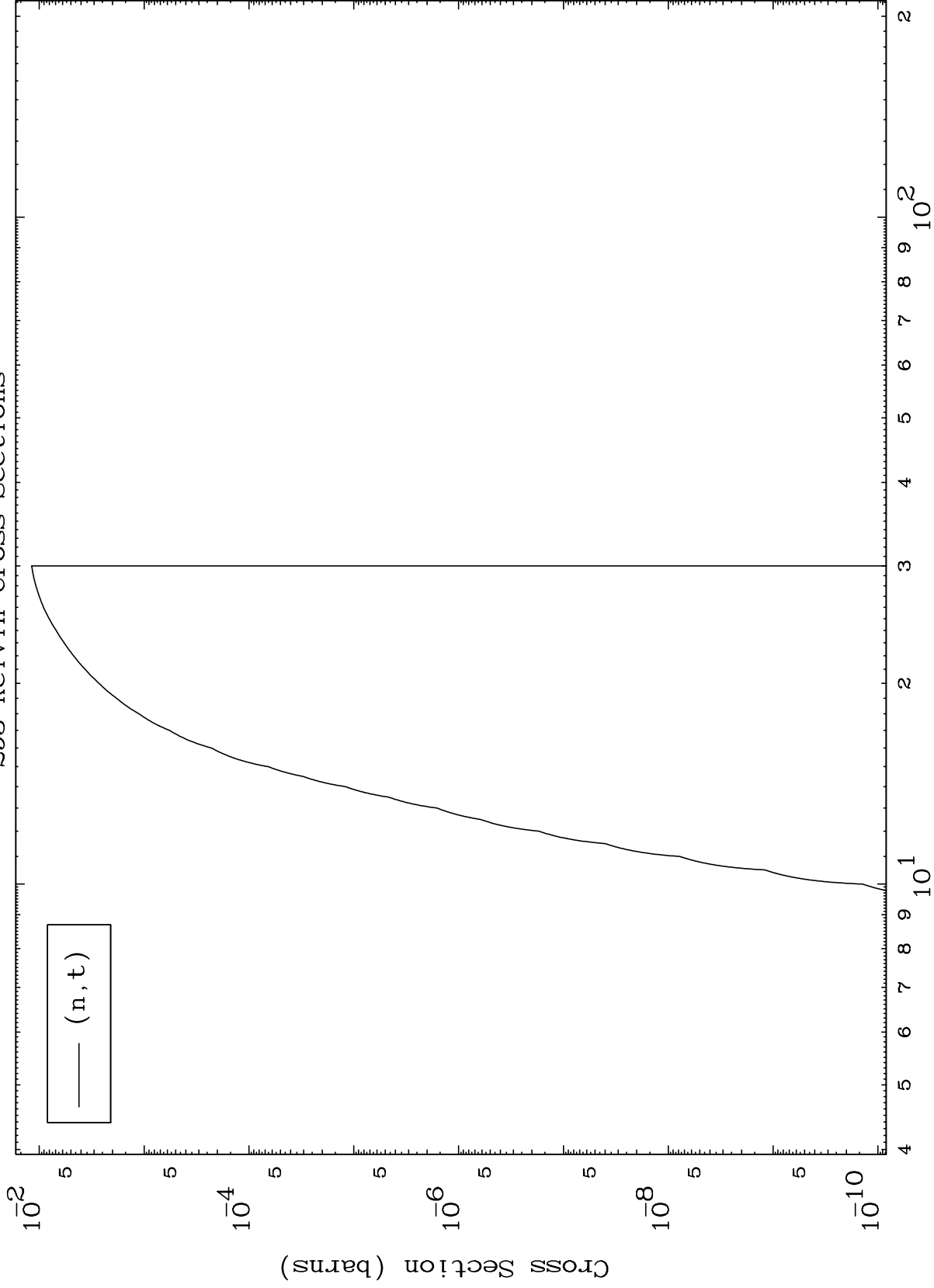
86-Rn-206



MAT 8610

(n,t) Levels
293 Kelvin Cross Sections

86-Rn-206



10

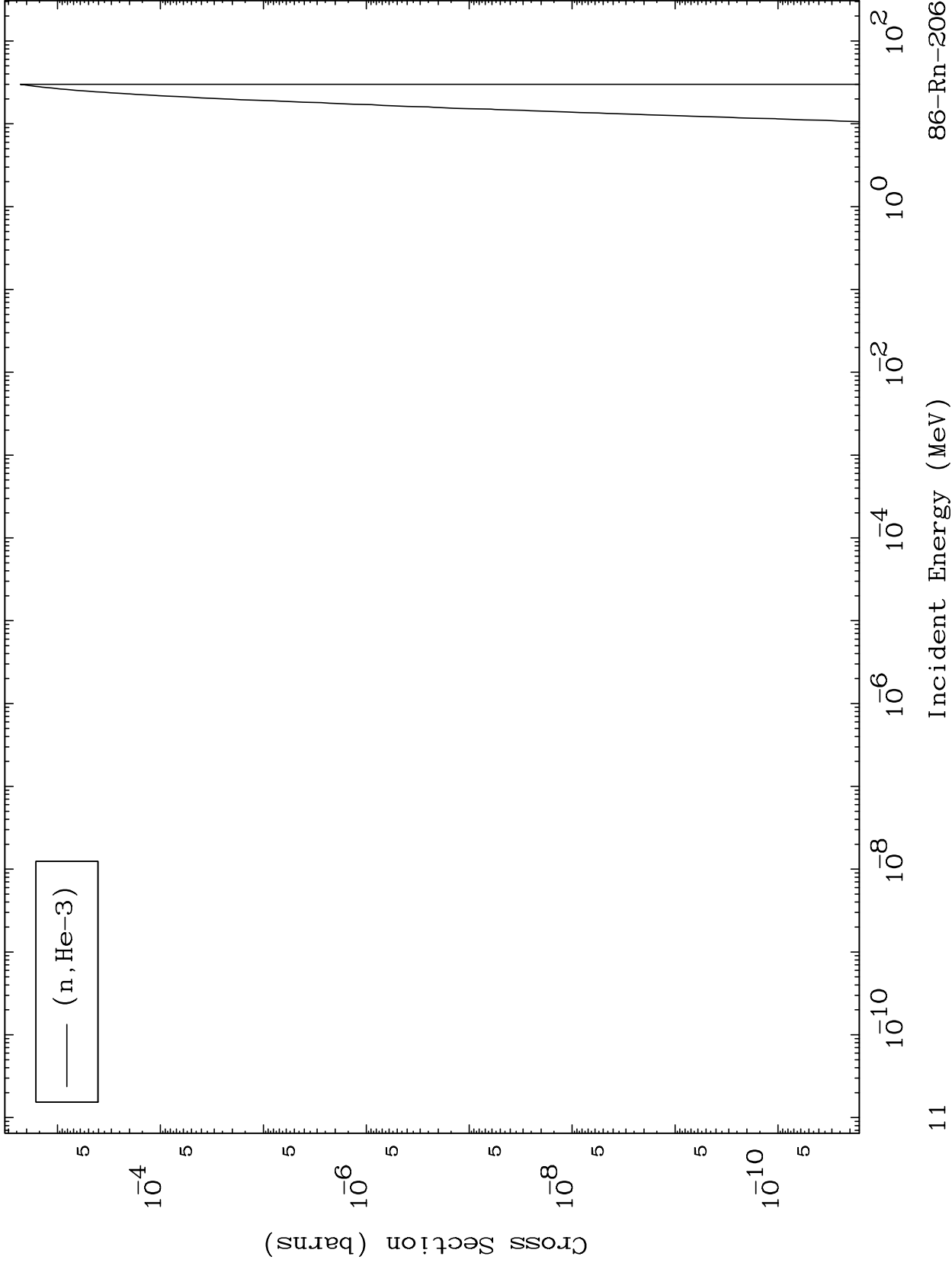
Incident Energy (MeV)

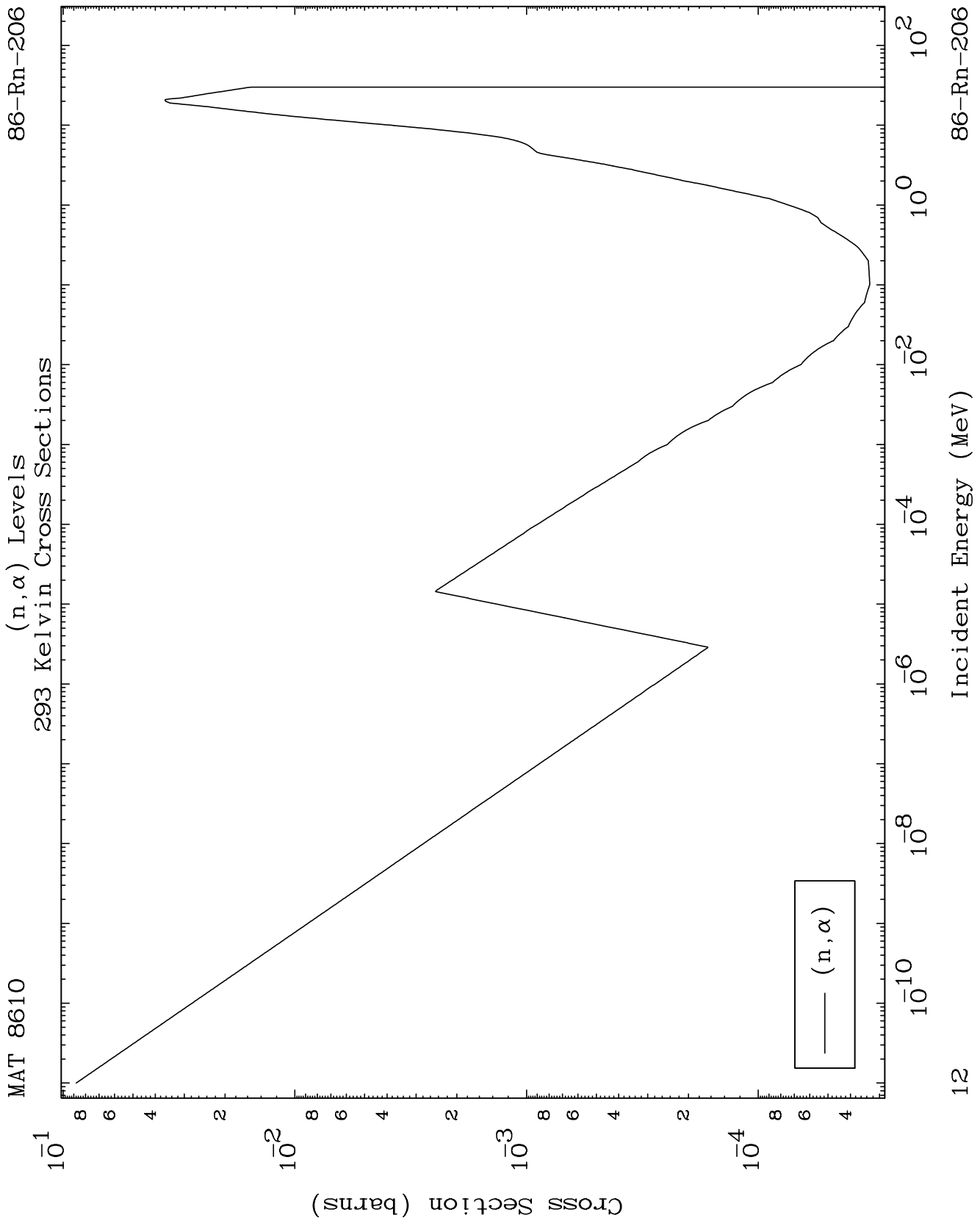
86-Rn-206

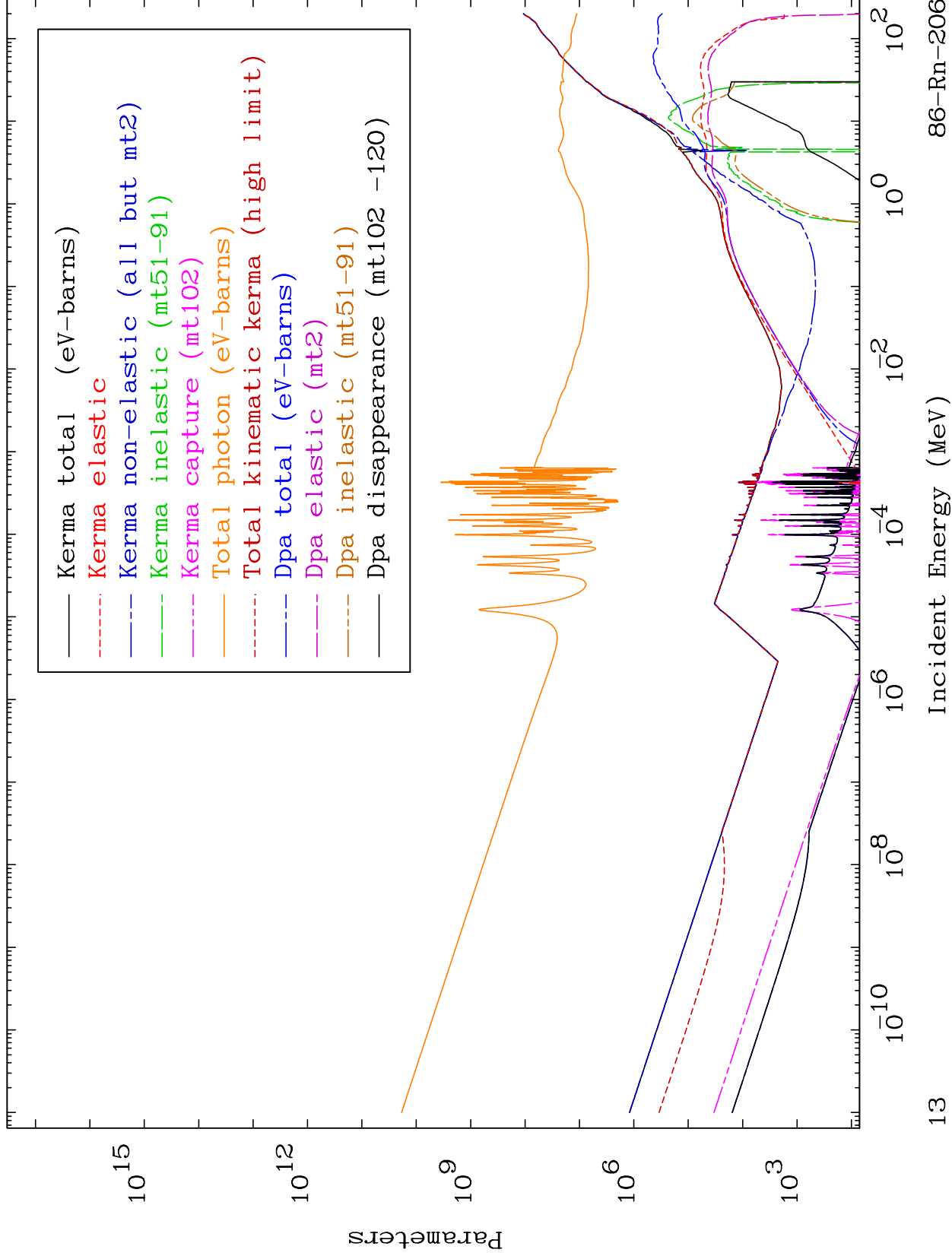
MAT 8610

(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-206



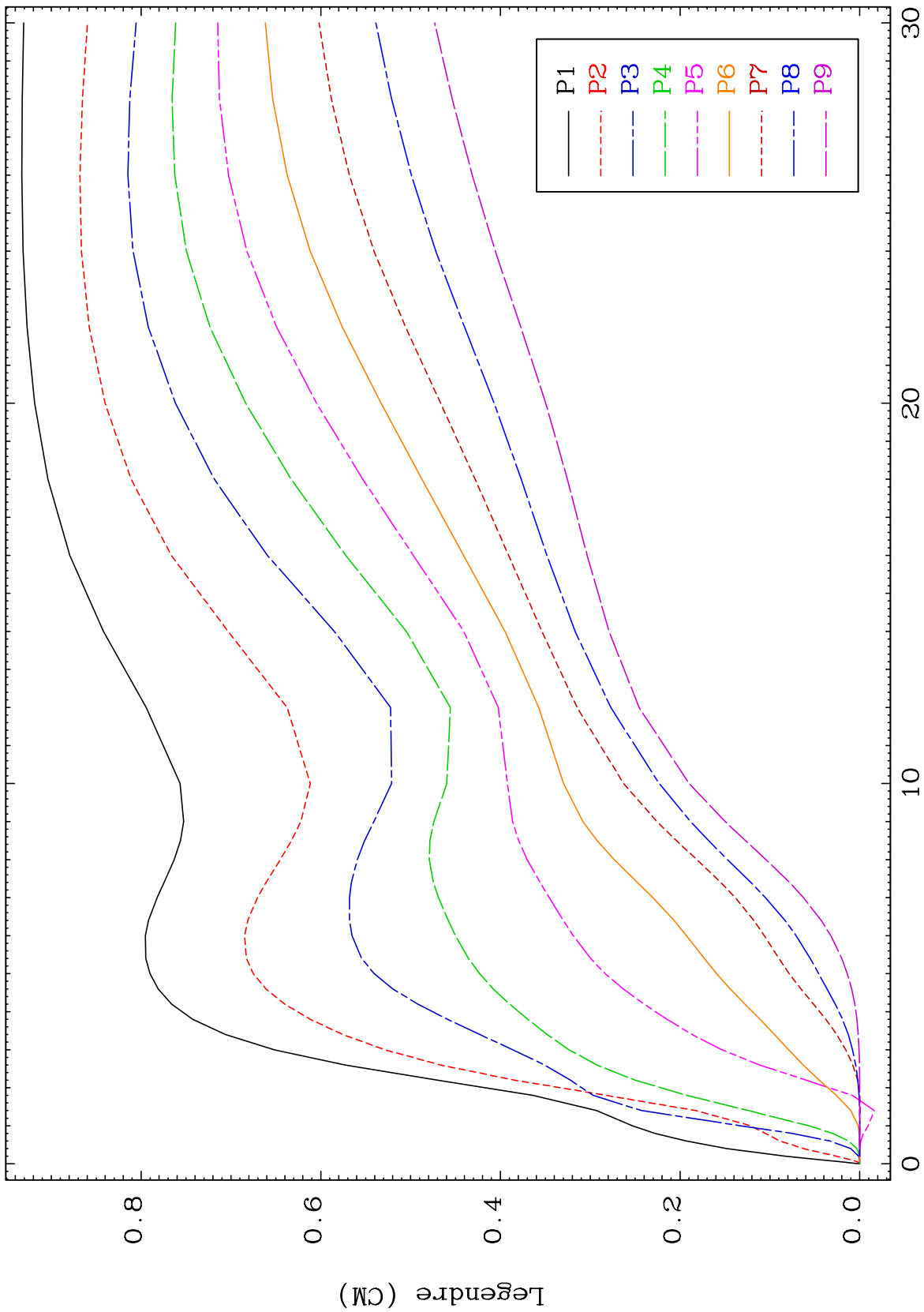




MAT 8610

86-Rn-206

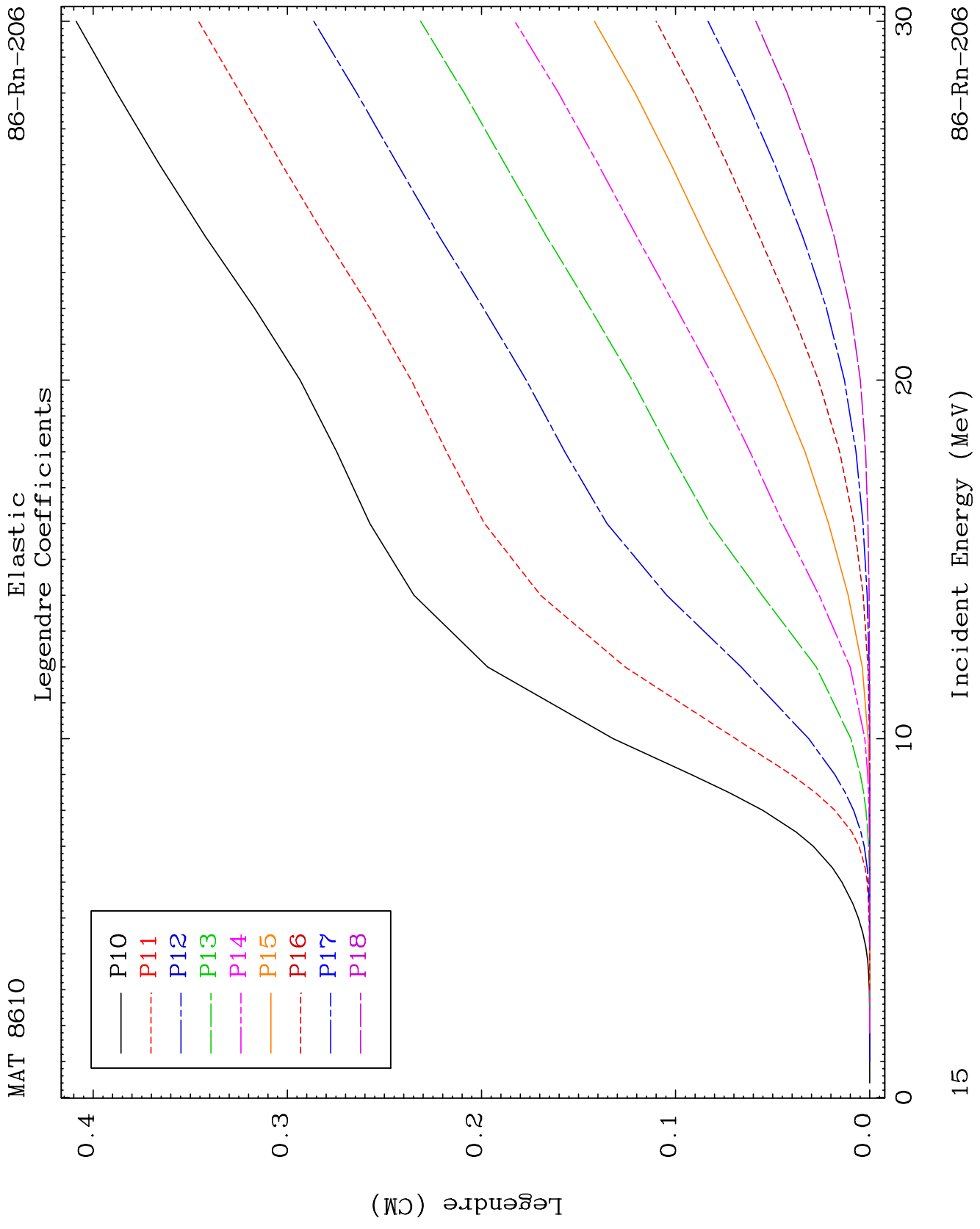
Elastic Legendre Coefficients



86-Rn-206

Incident Energy (MeV)

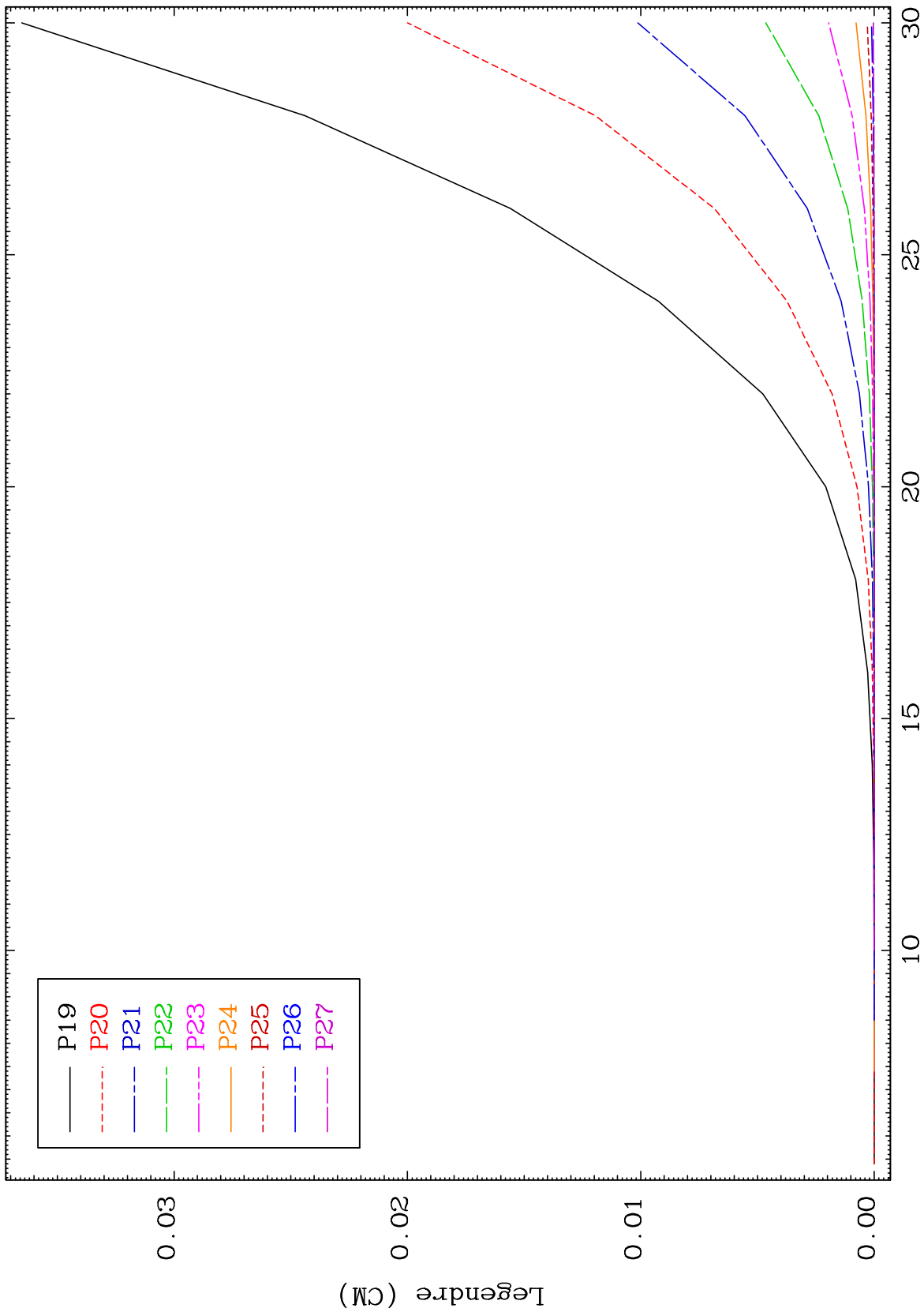
14



MAT 8610

Elastic
Legendre Coefficients

86-Rn-206



16

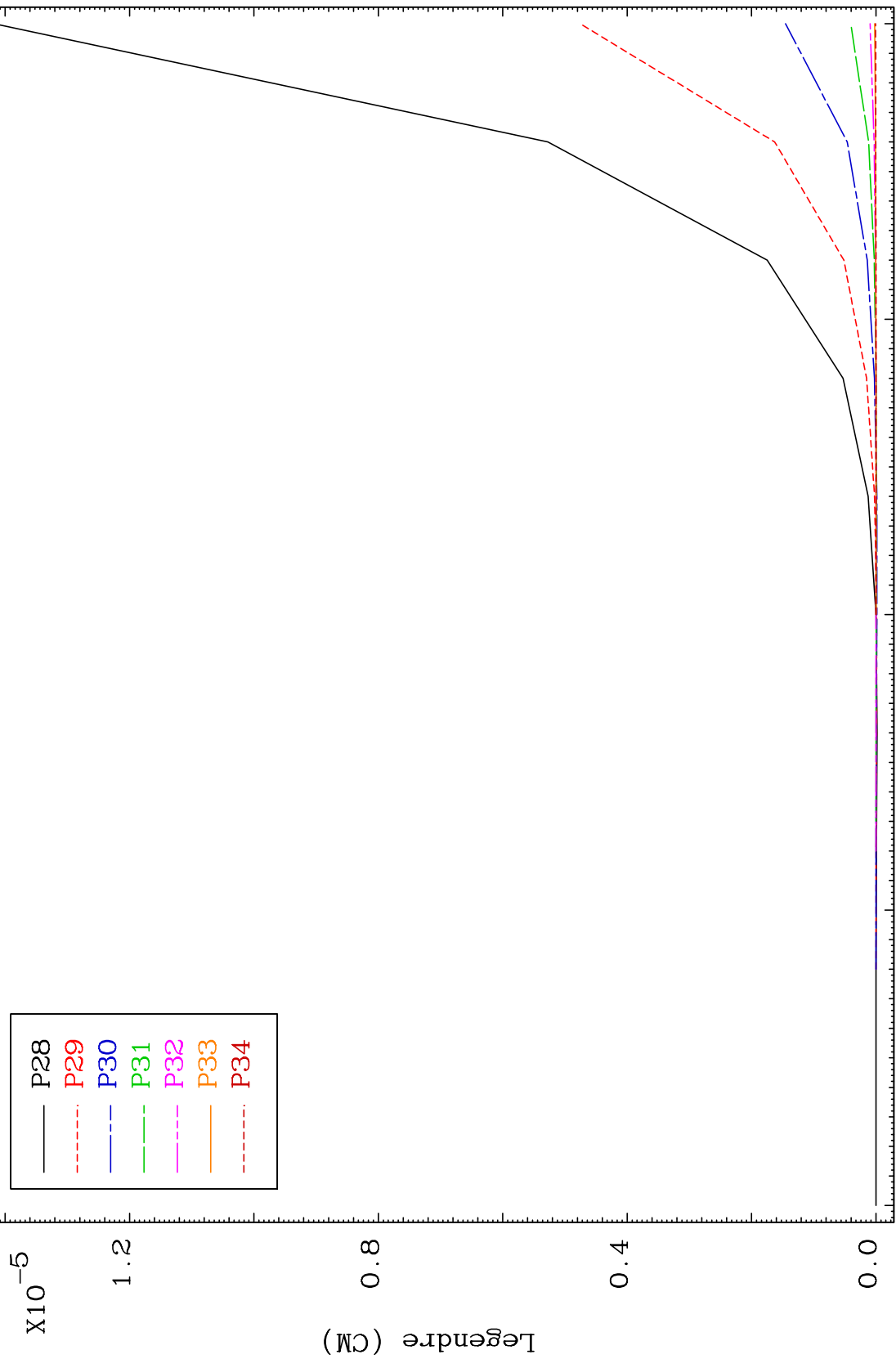
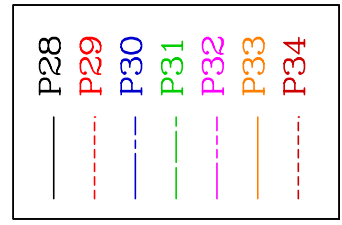
Incident Energy (MeV)

86-Rn-206

MAT 8610

Elastic Legendre Coefficients

86-Rn-206



17

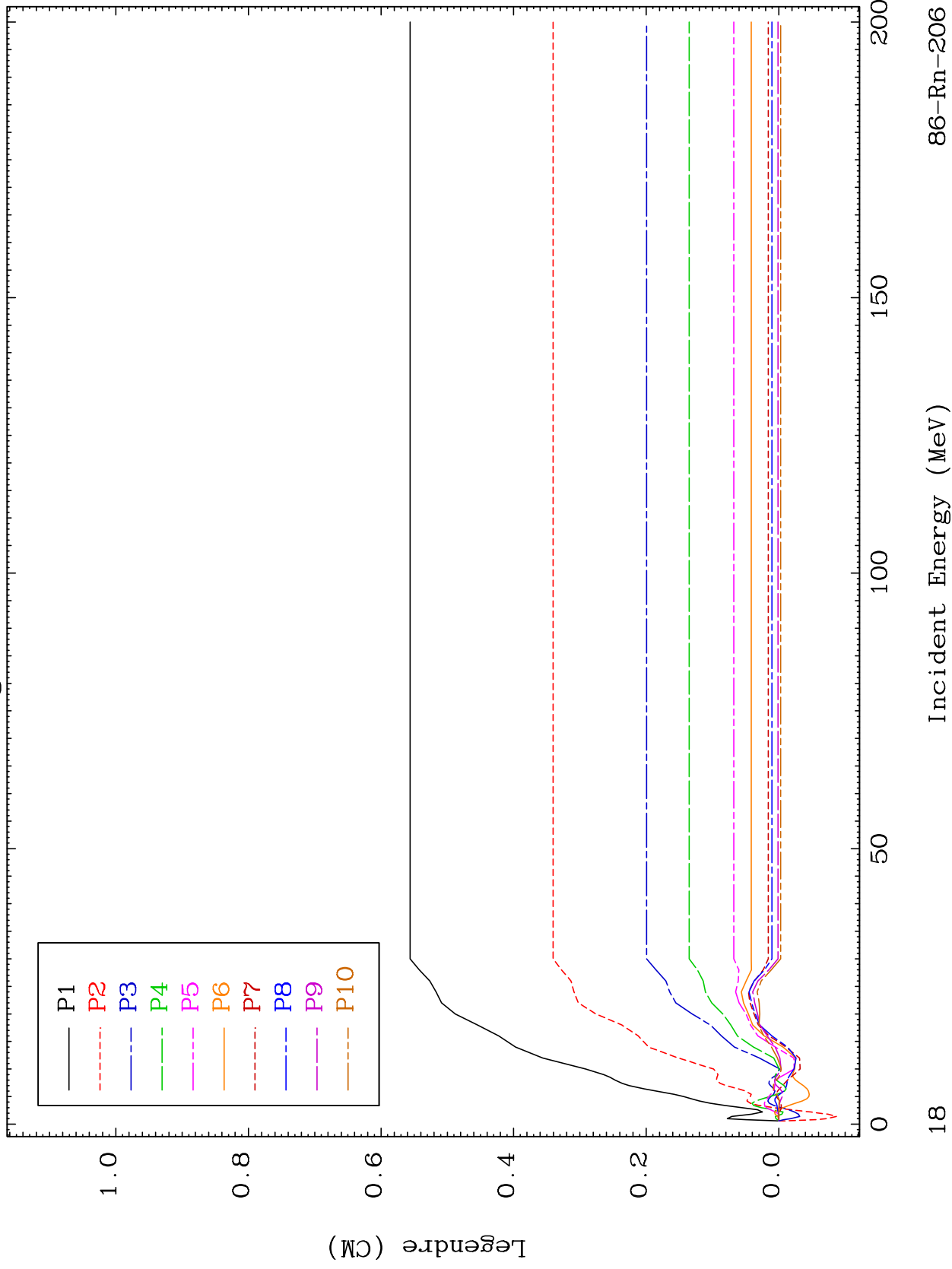
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 51 (n,n') Level
Legendre Coefficients

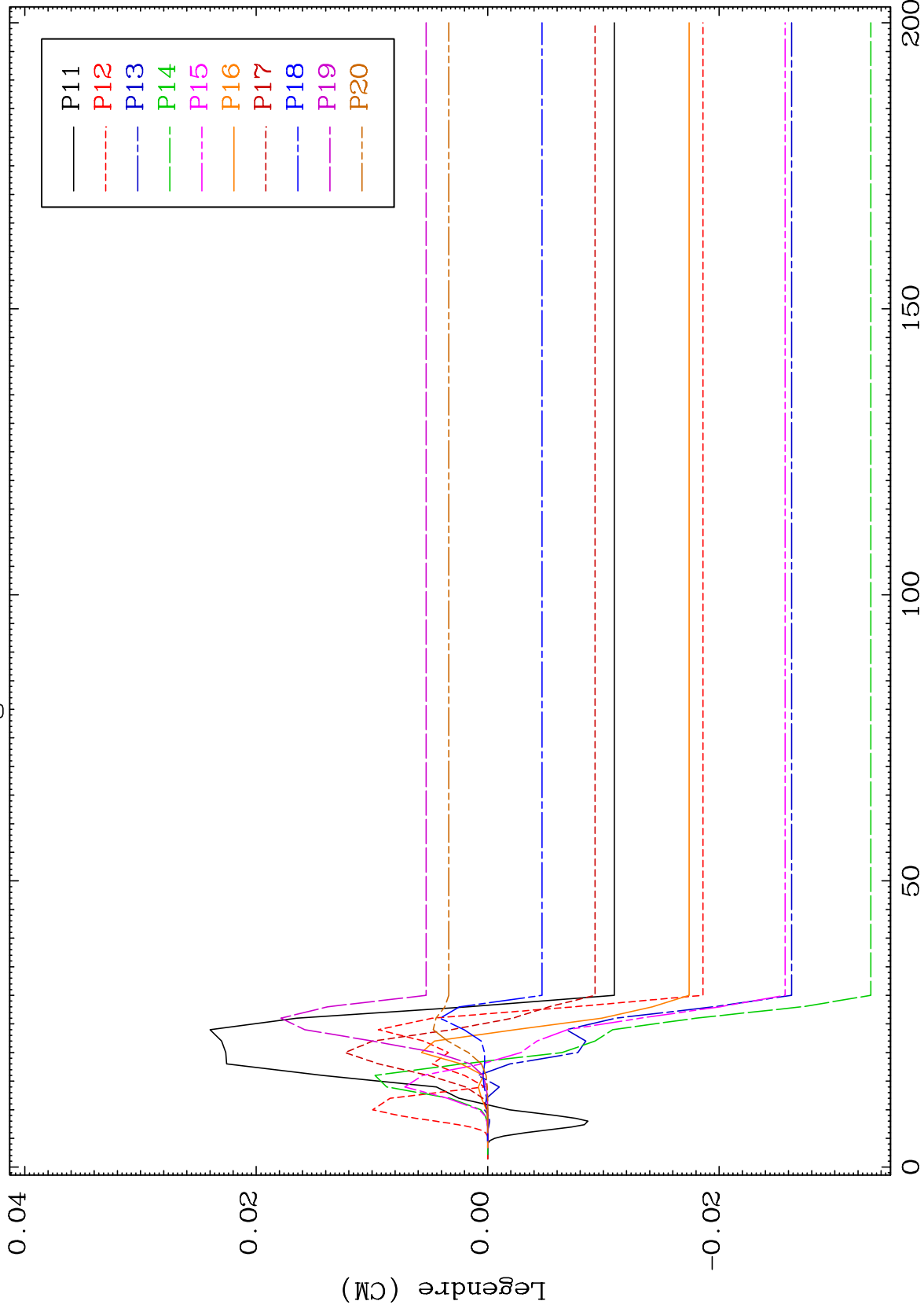
86-Rn-206



MAT 8610

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-206



19

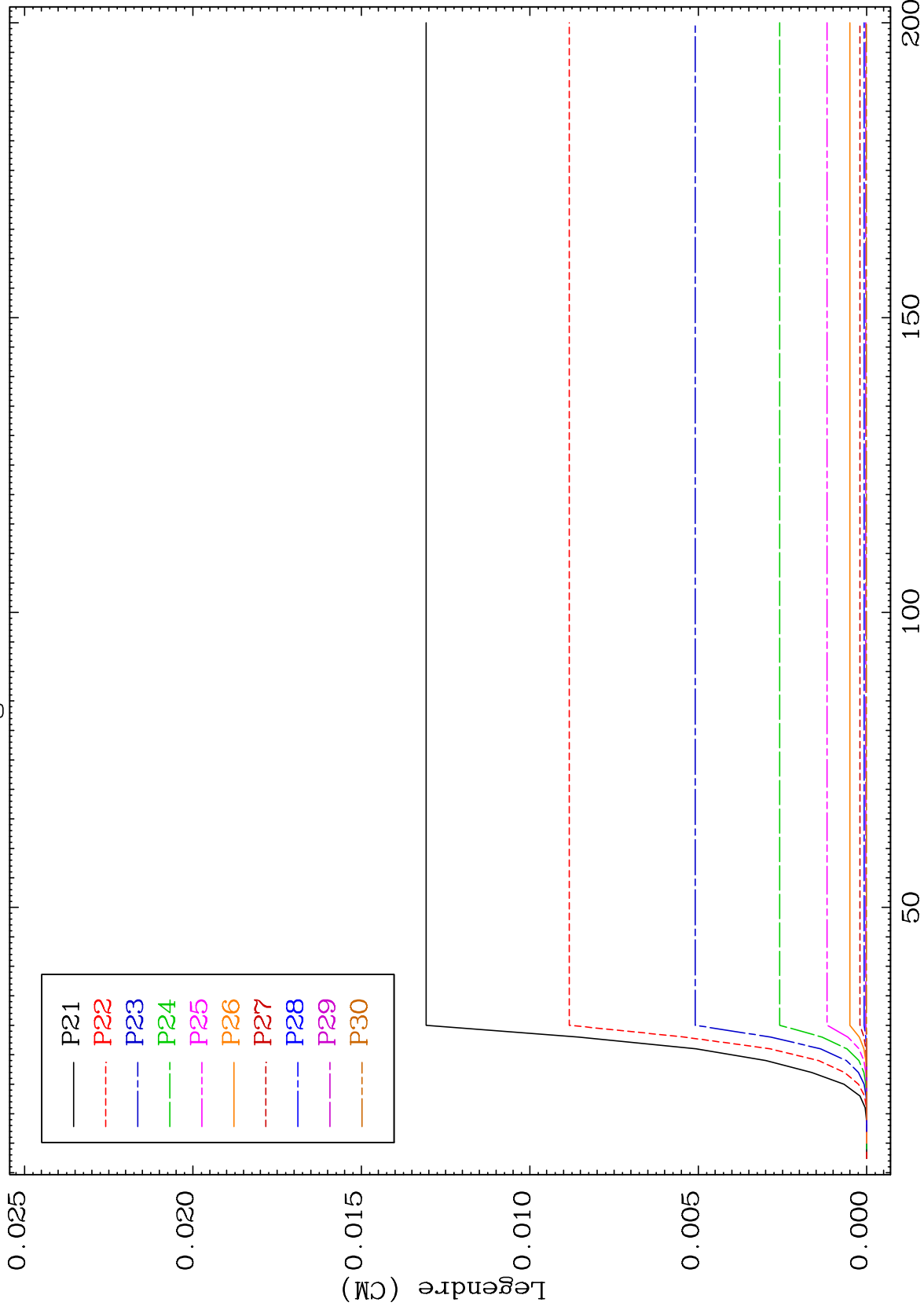
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-206



20

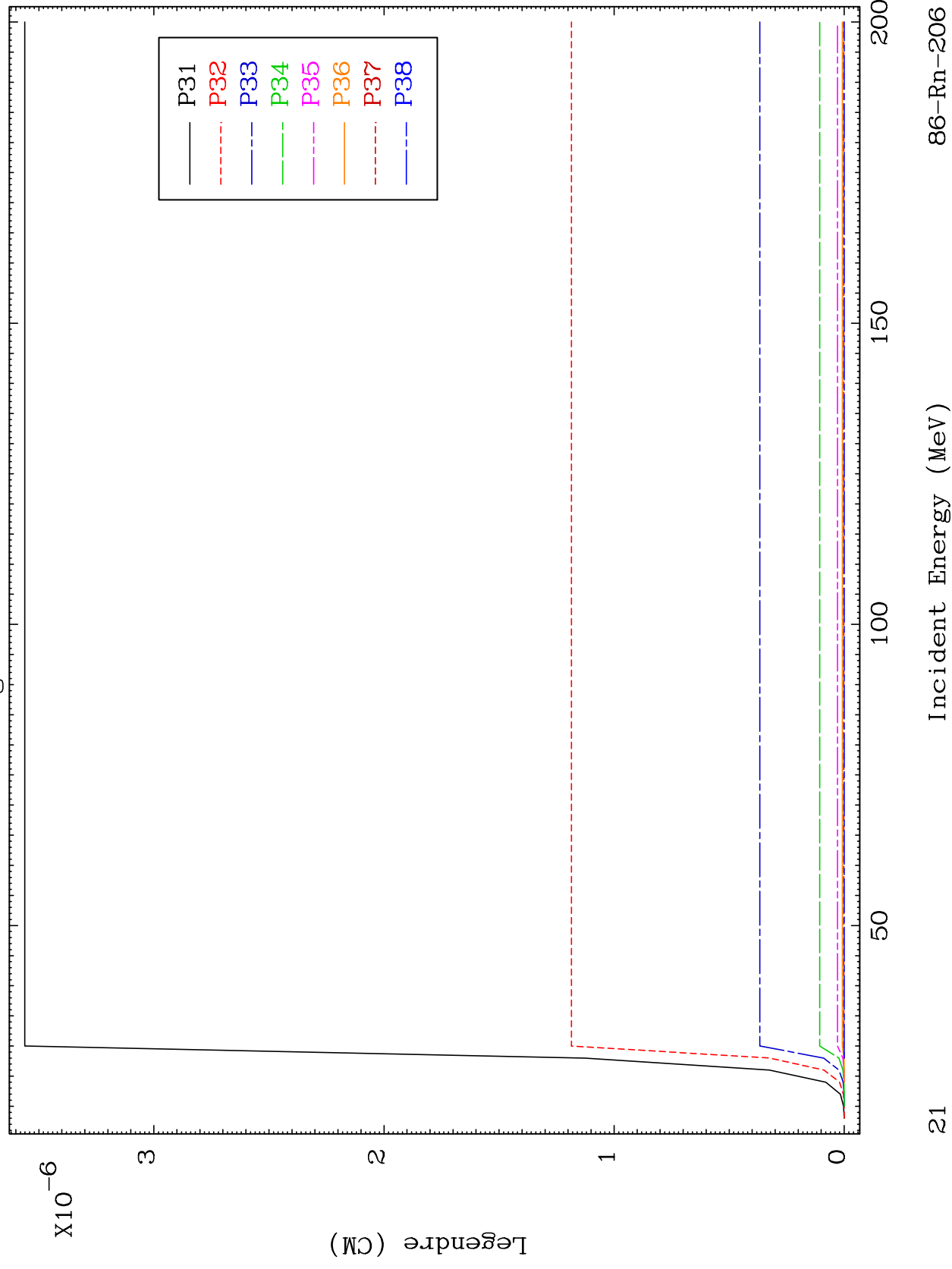
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-206



21

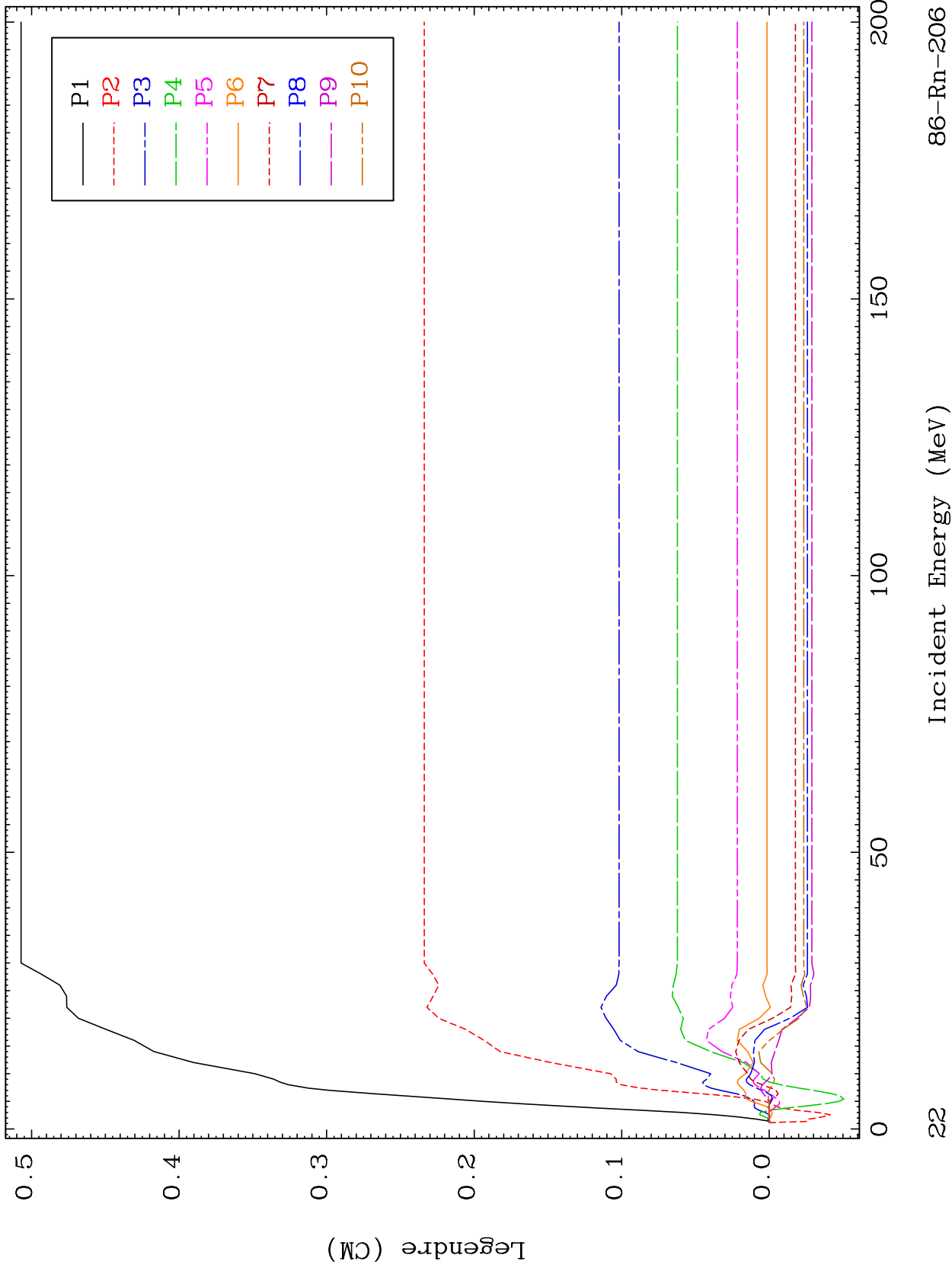
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-206



22

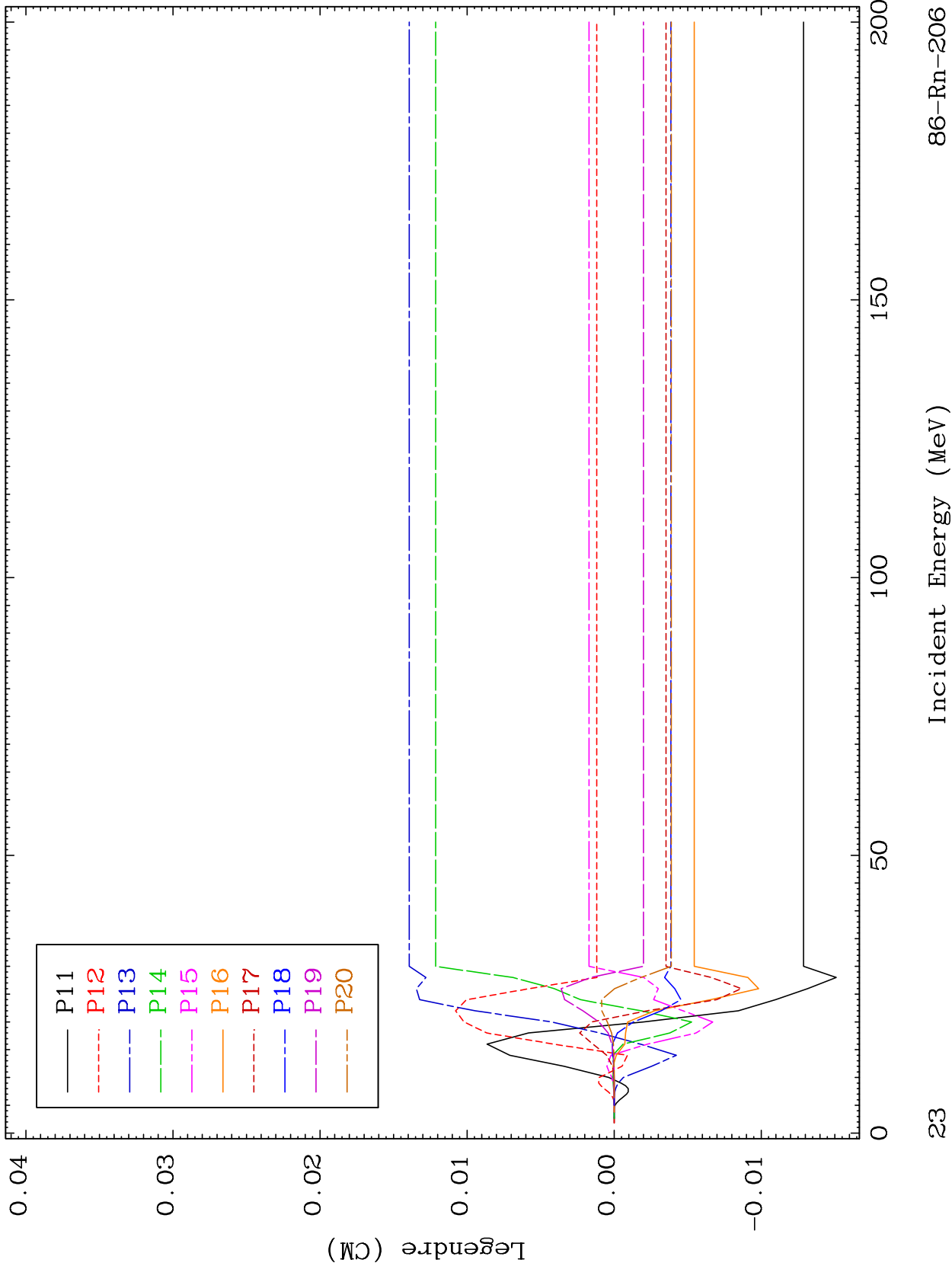
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-206



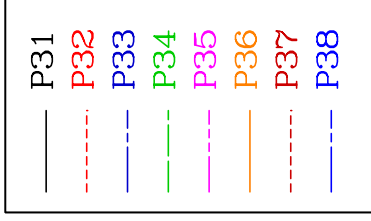
MAT 8610

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-206

$\times 10^{-6}$
1.5

Legendre (CM)



1.0

0.5

0.0

50

100

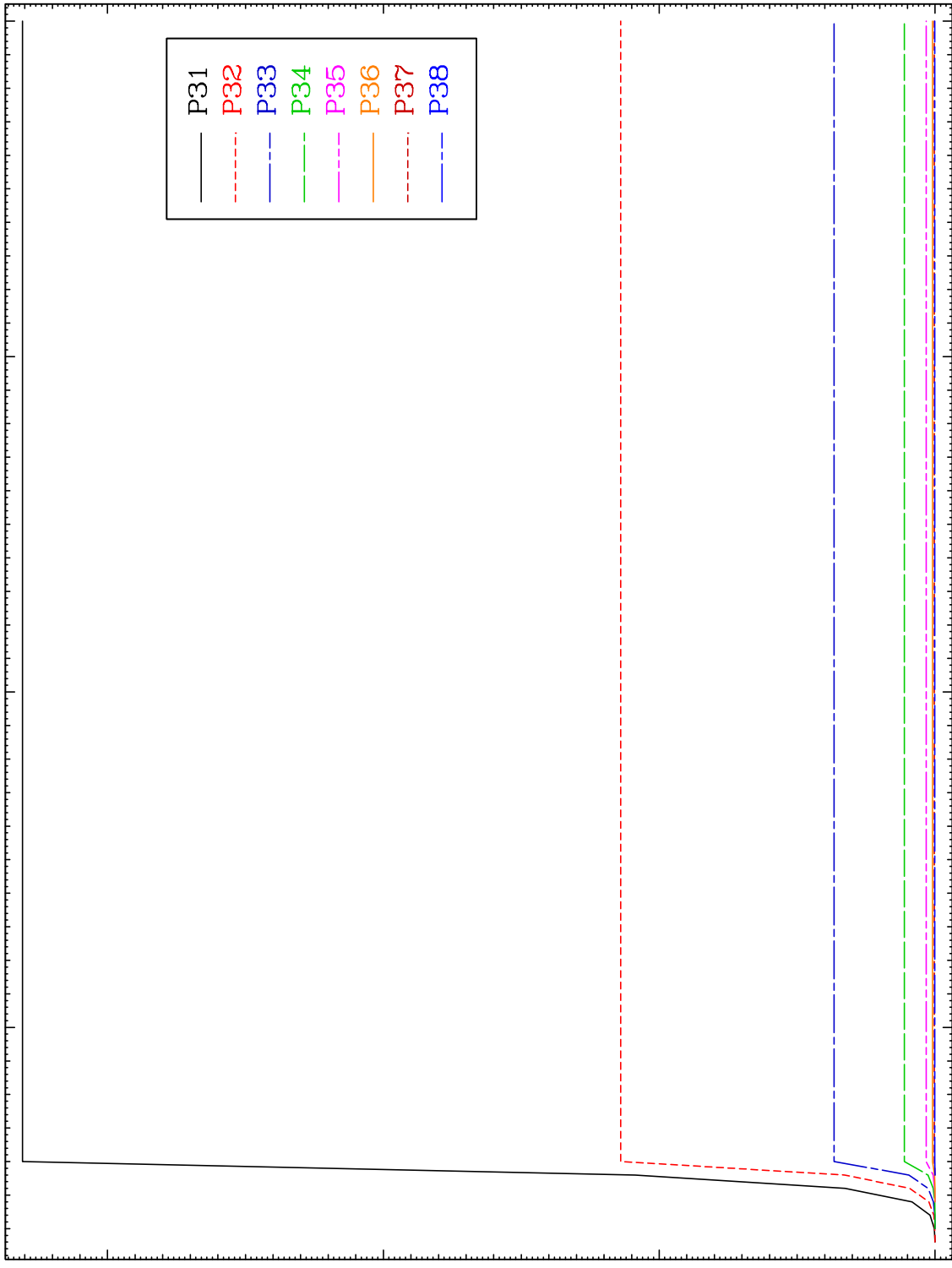
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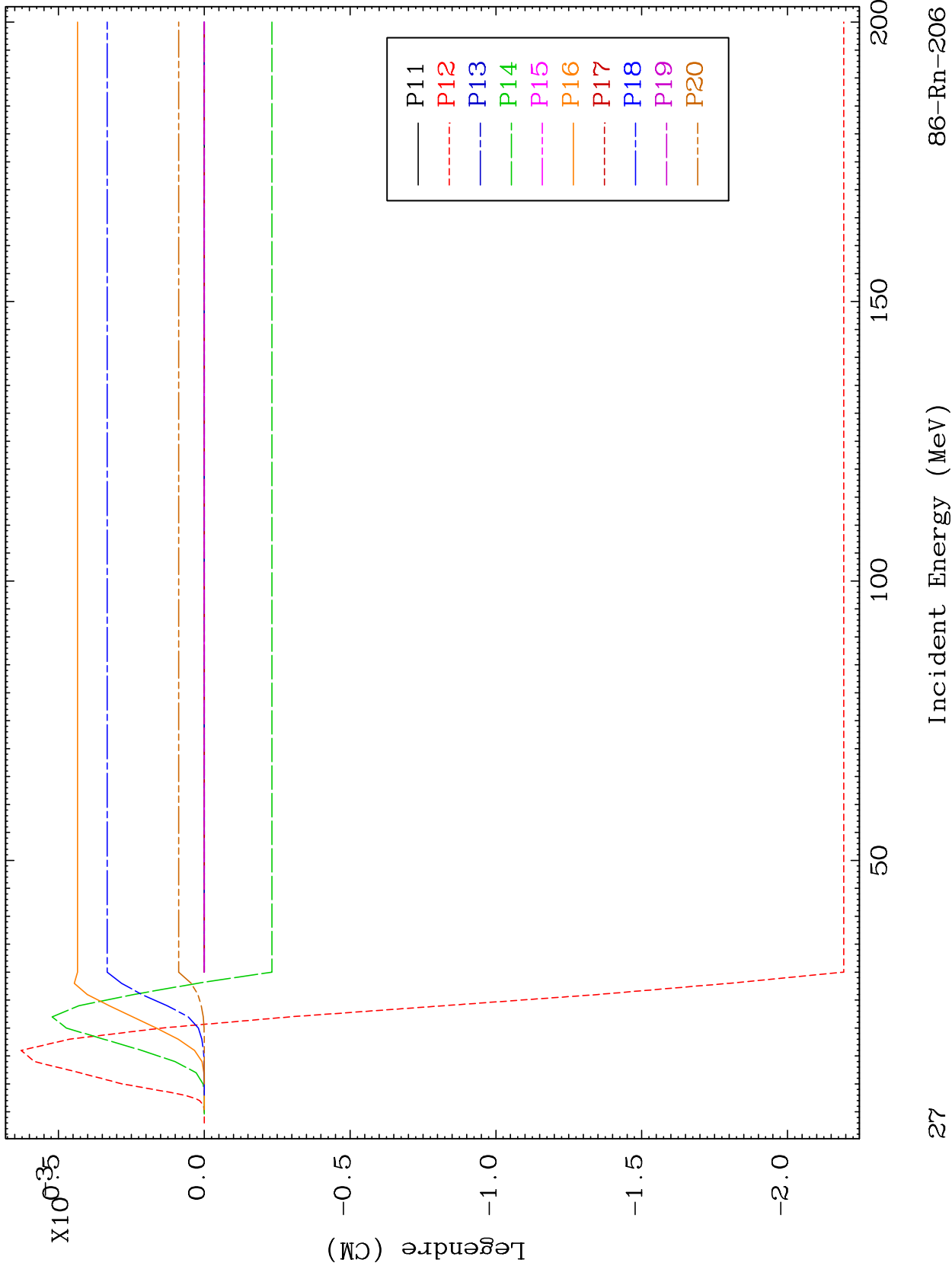
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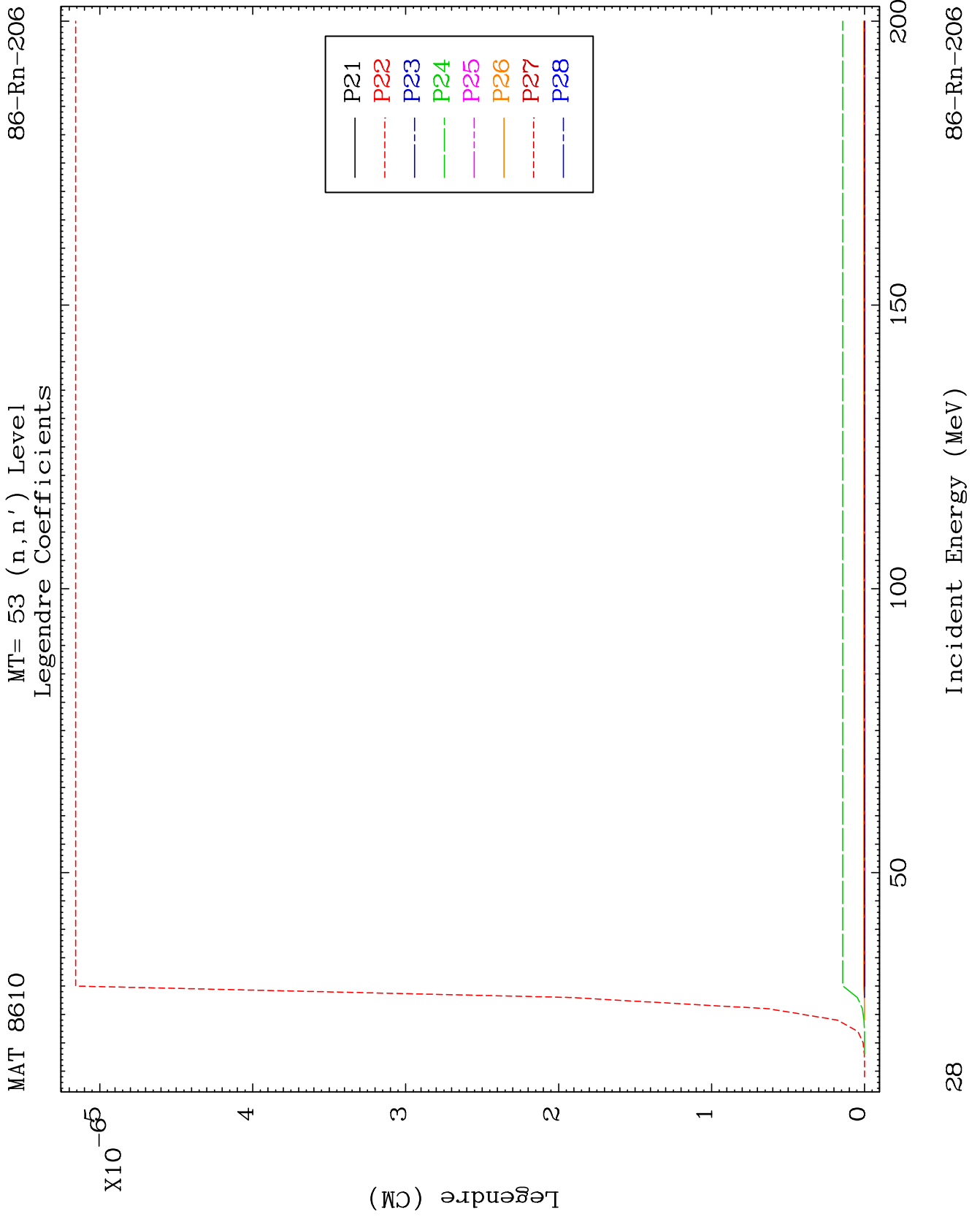
25

Incident Energy (MeV)

86-Rn-206



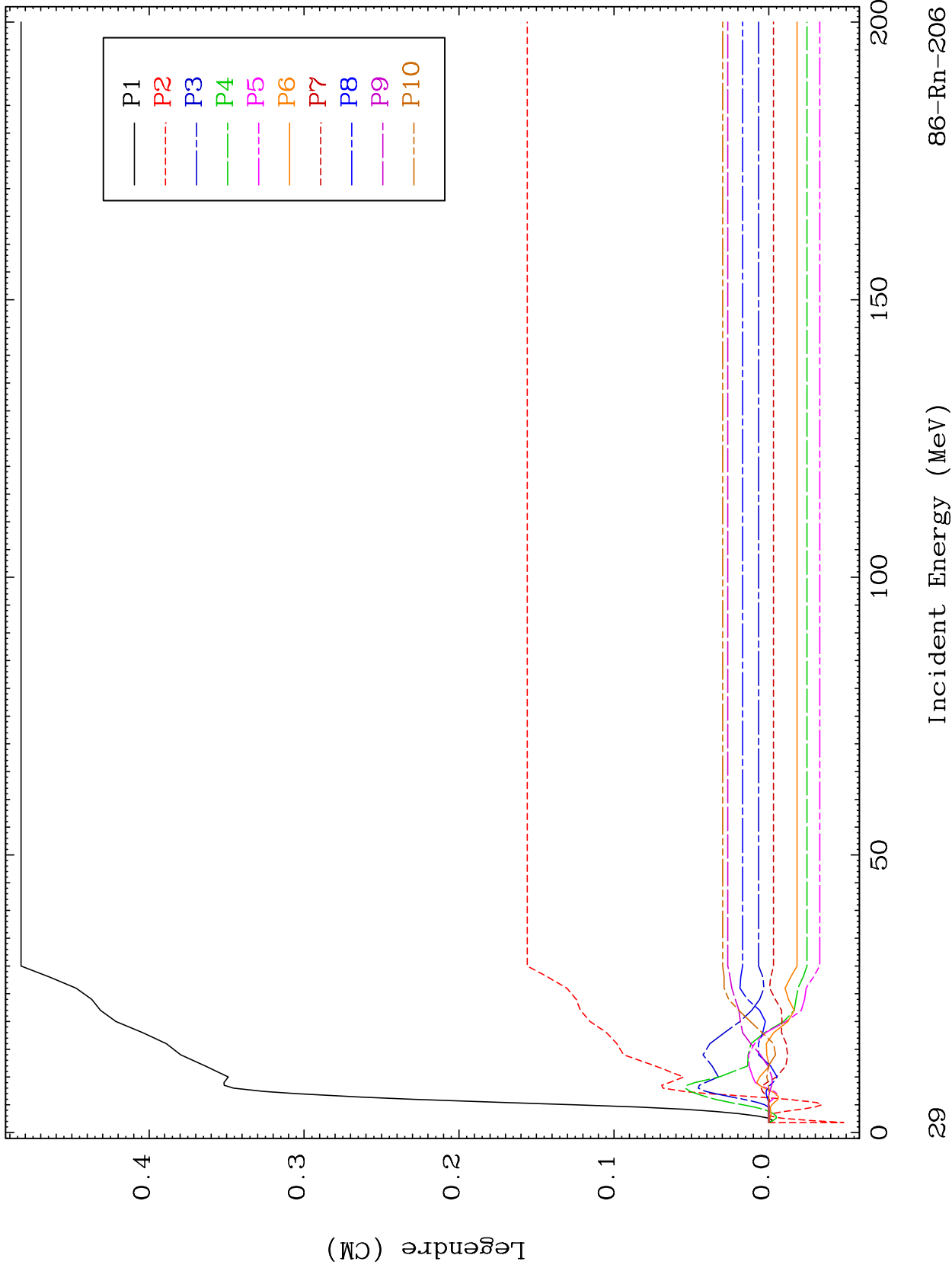




MAT 8610

MT= 54 (n,n') Level
Legendre Coefficients

86-Rn-206



29

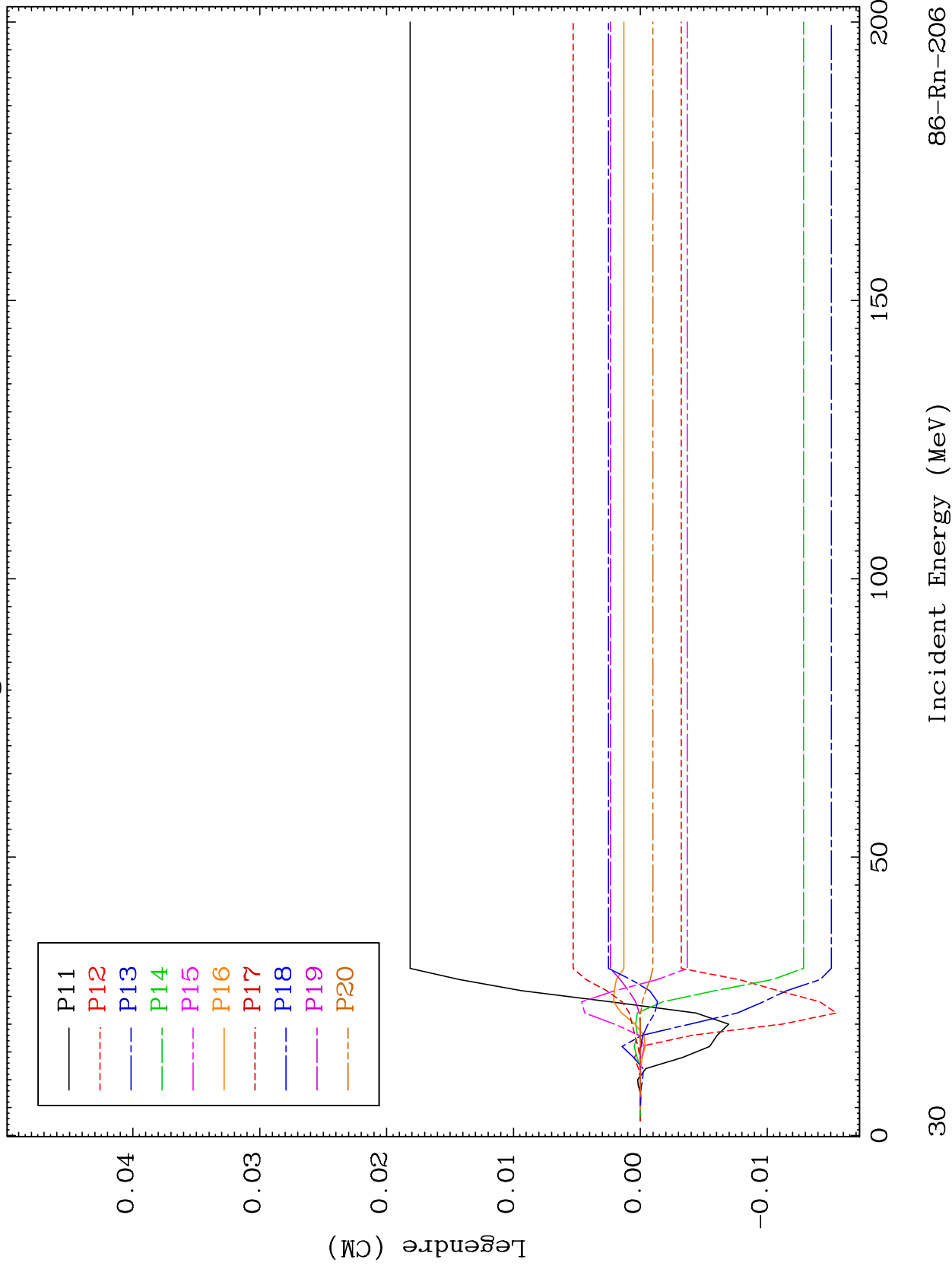
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 54 (n,n') Level
Legendre Coefficients

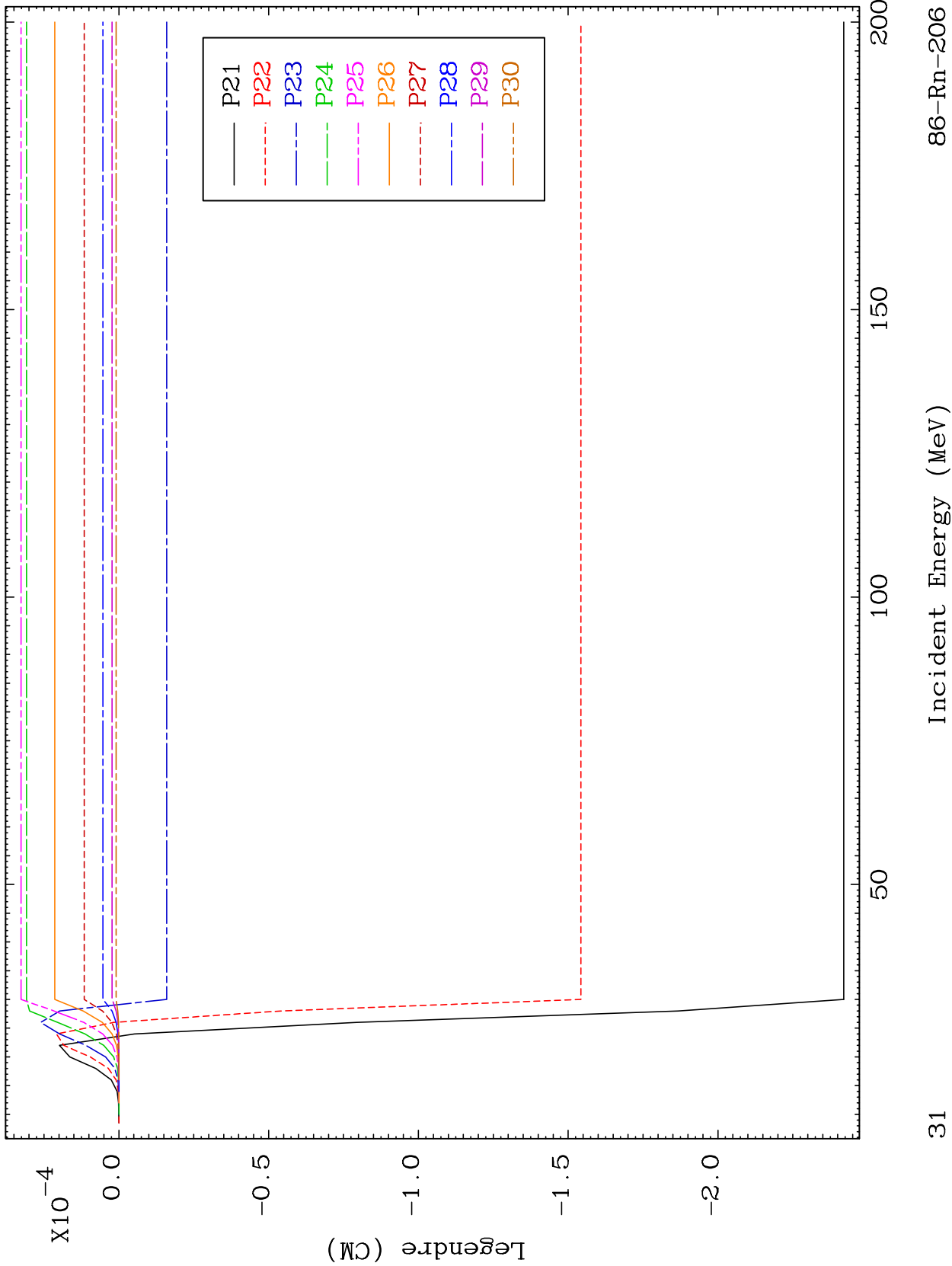
86-Rn-206



86-Rn-206

Incident Energy (MeV)

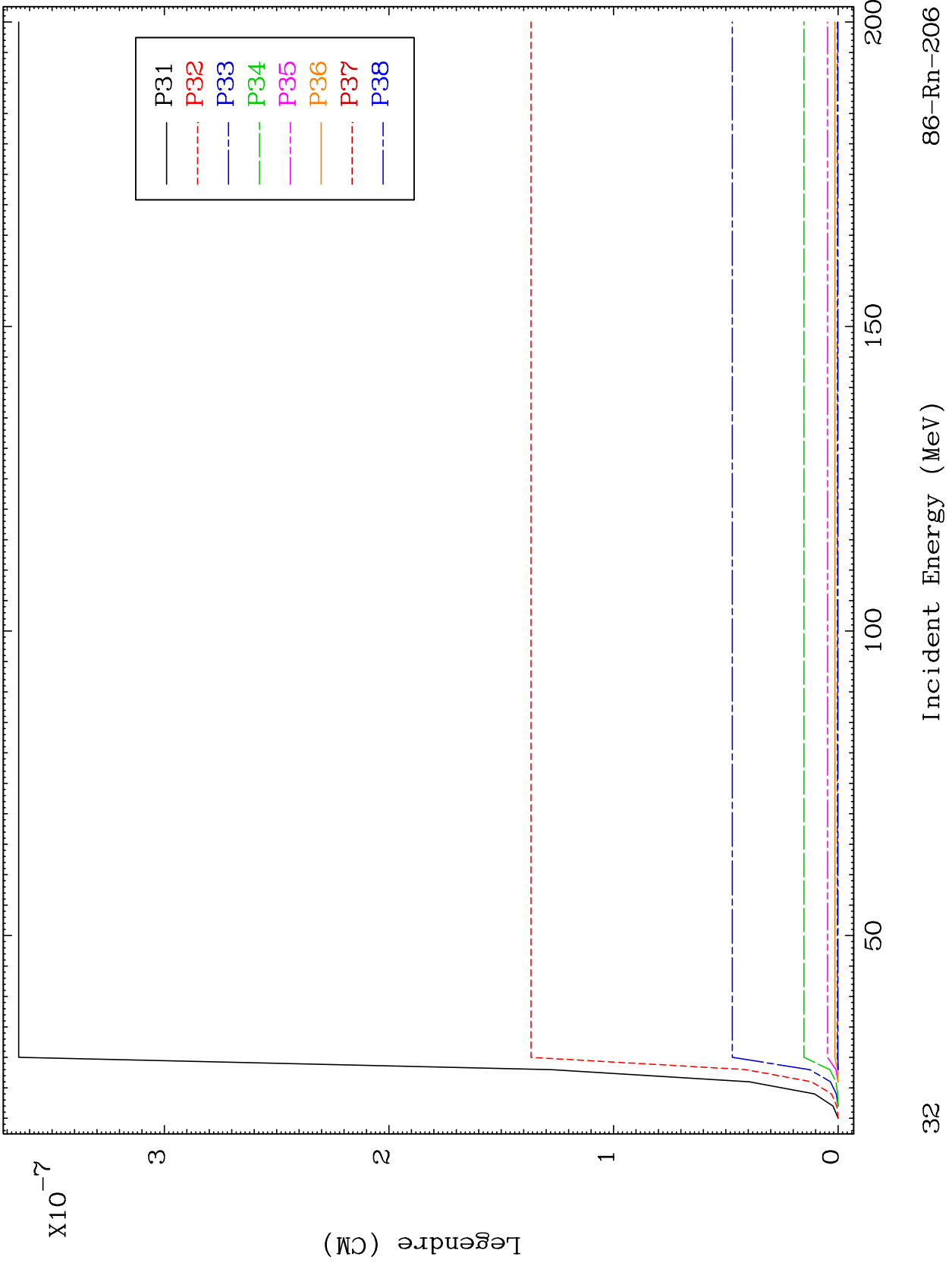
30



MAT 8610

MT= 54 (n,n') Level
Legendre Coefficients

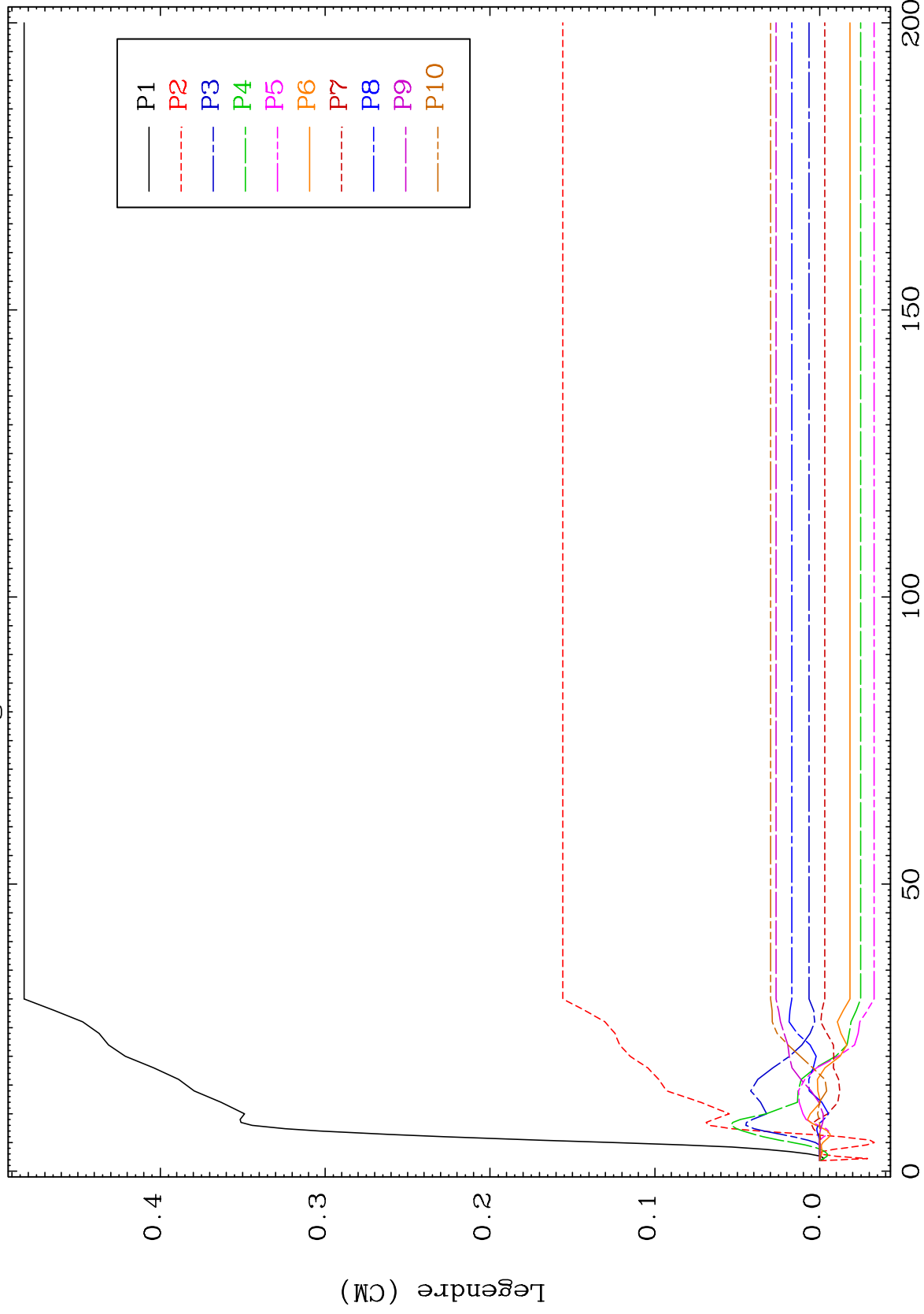
86-Rn-206



MAT 8610

MT= 55 (n,n') Level
Legendre Coefficients

86-Rn-206



33

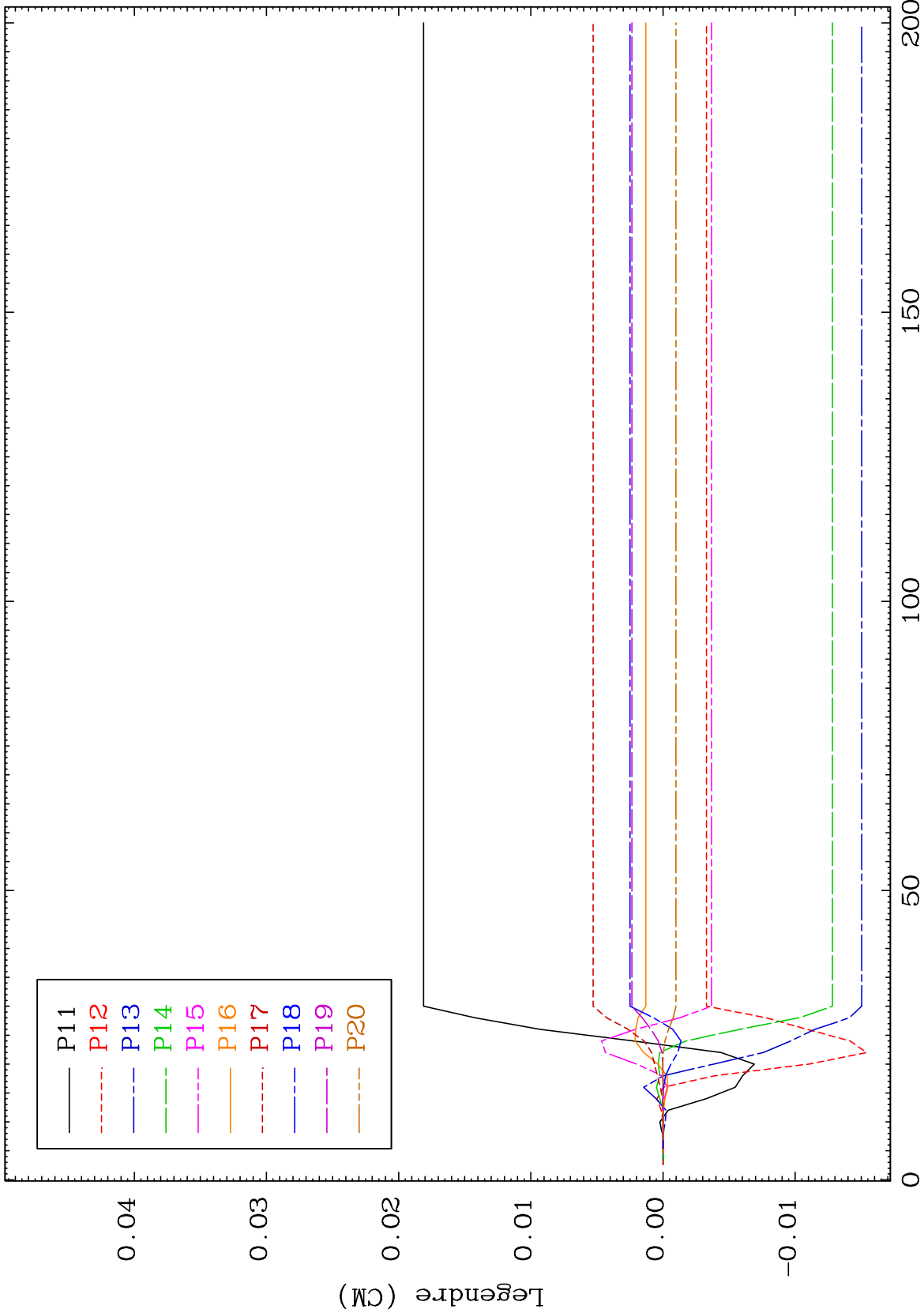
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 55 (n,n') Level
Legendre Coefficients

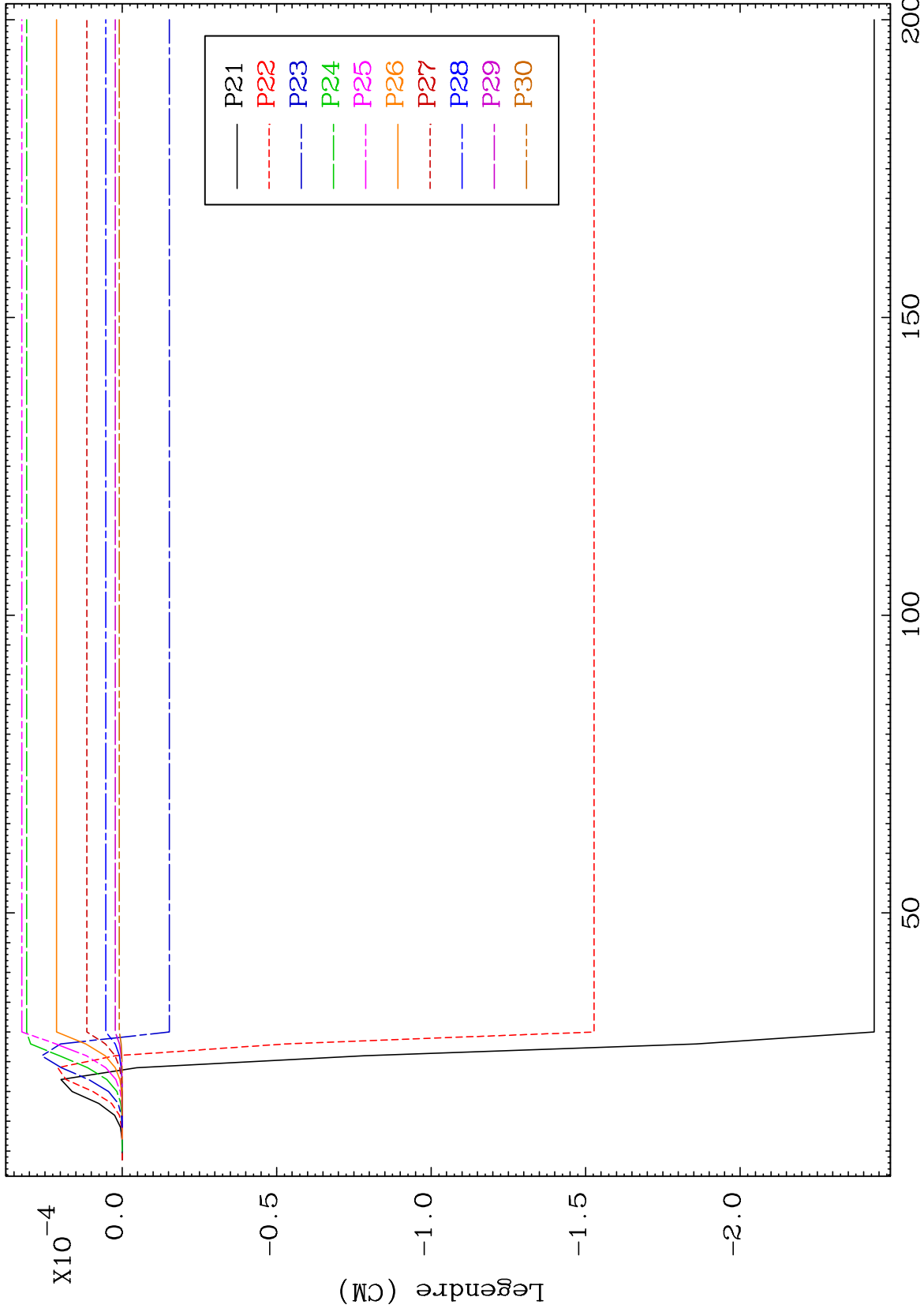
86-Rn-206



34

Incident Energy (MeV)

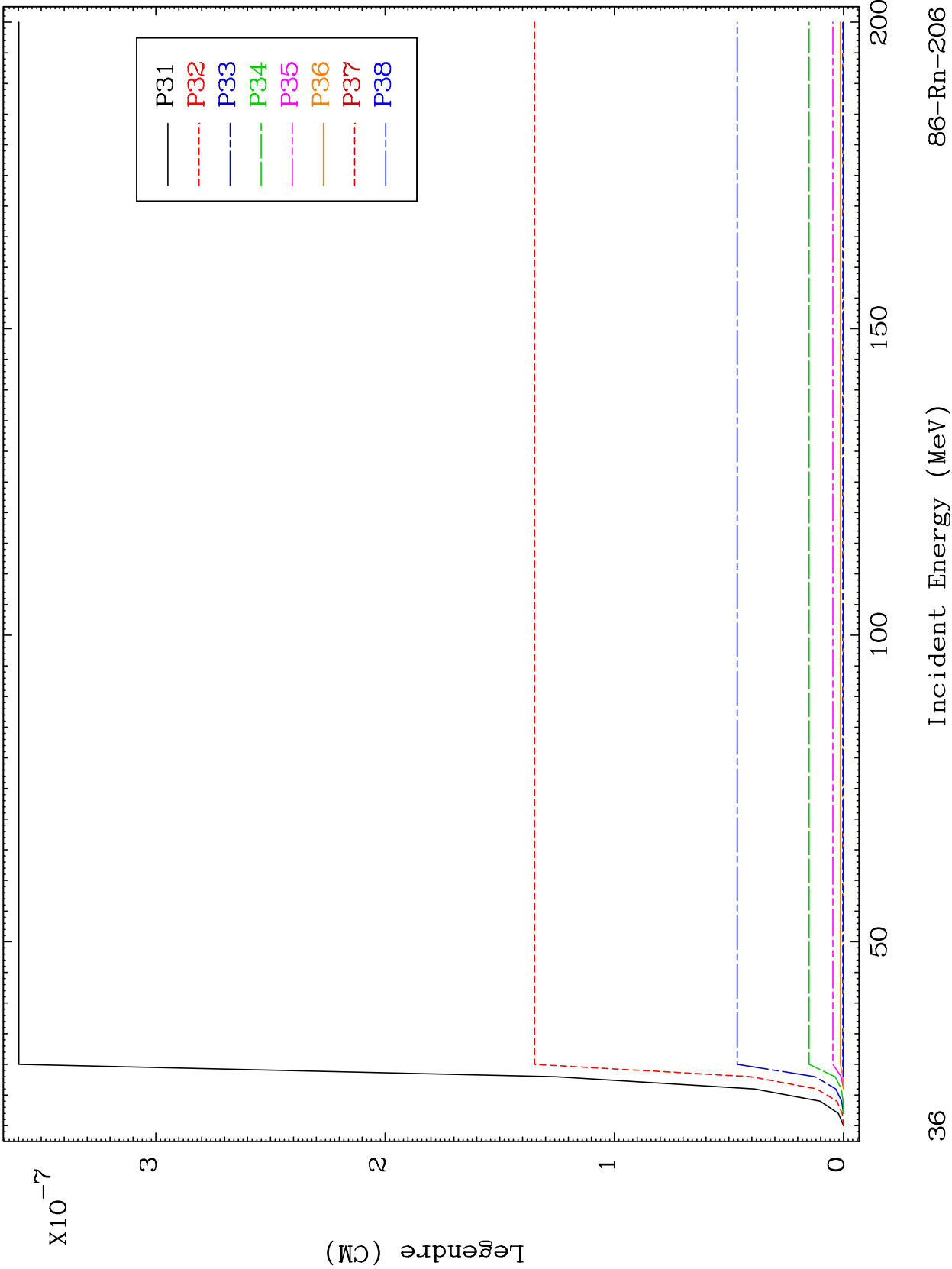
86-Rn-206



MAT 8610

MT= 55 (n,n') Level
Legendre Coefficients

86-Rn-206

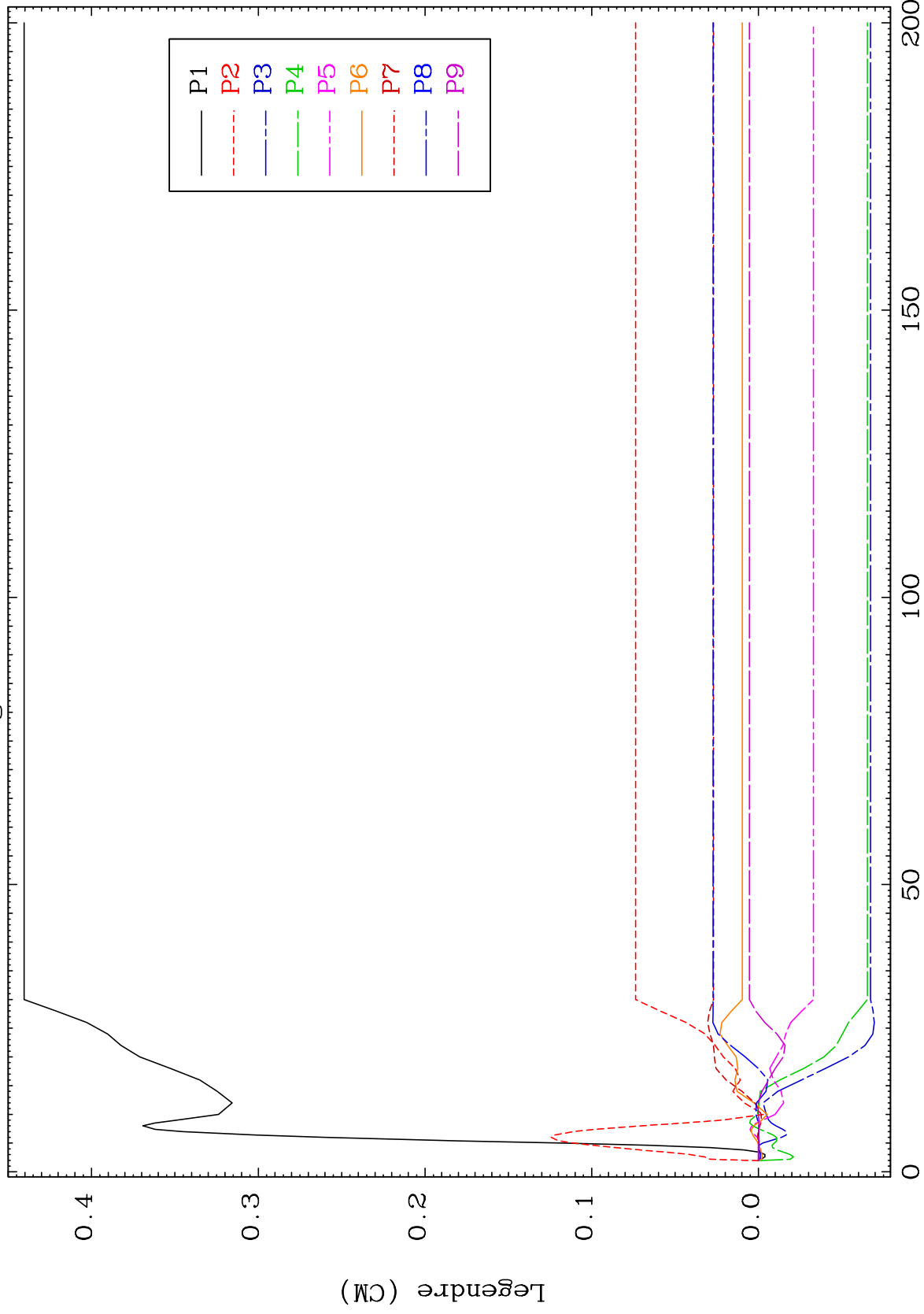


36

MAT 8610

MT= 56 (n,n') Level
Legendre Coefficients

86-Rn-206



37

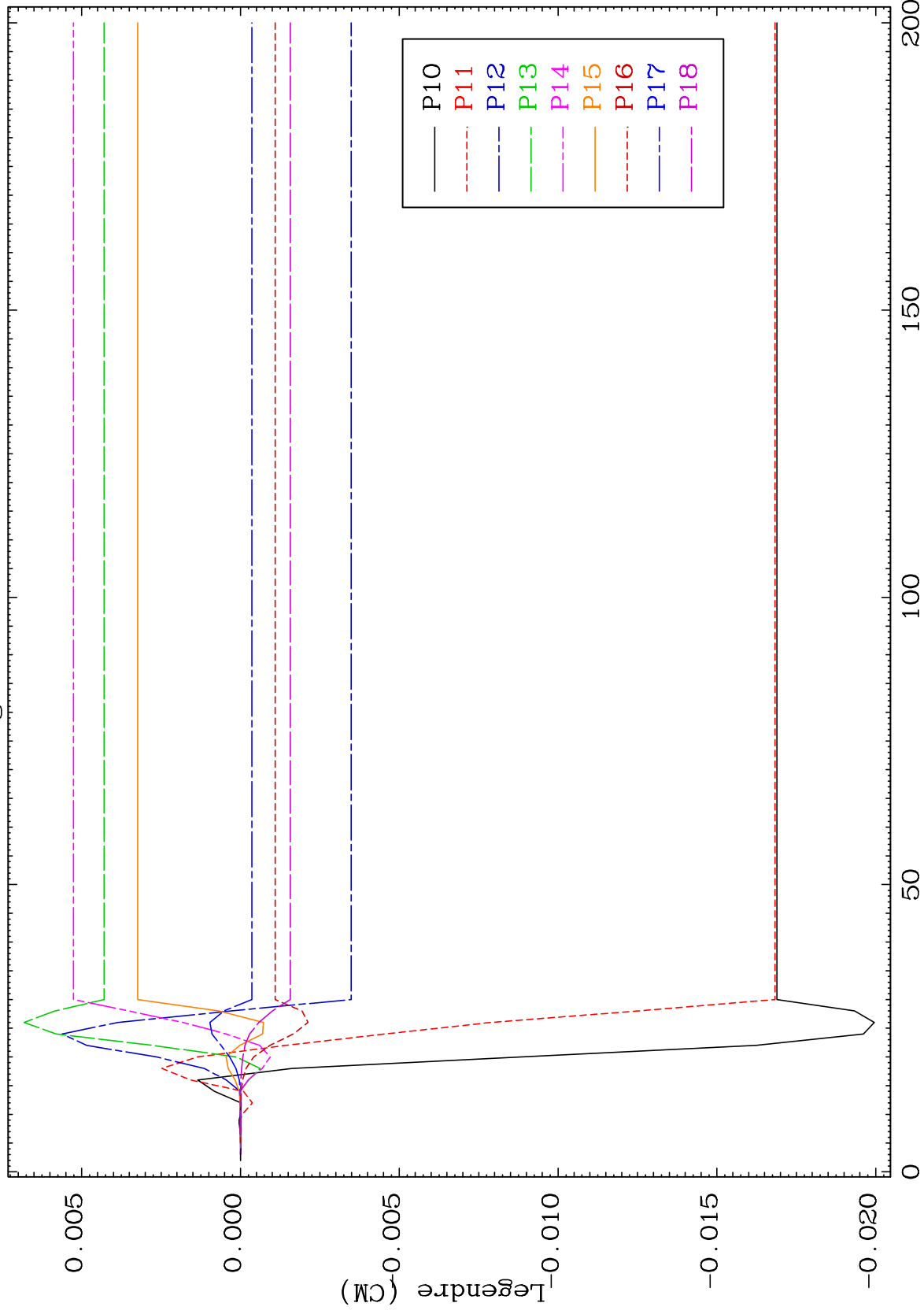
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 56 (n,n') Level
Legendre Coefficients

86-Rn-206



38

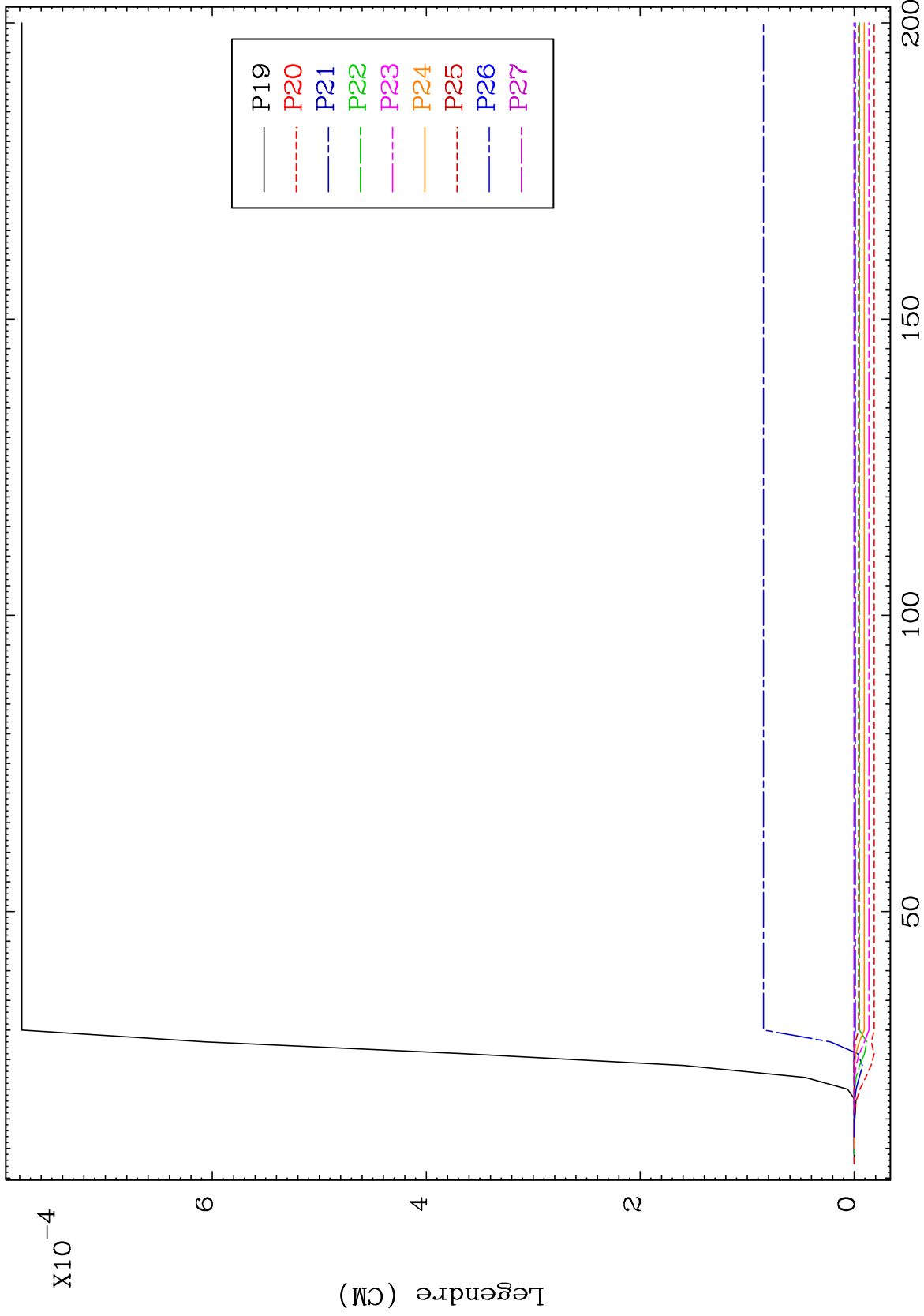
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 56 (n,n') Level
Legendre Coefficients

86-Rn-206



39

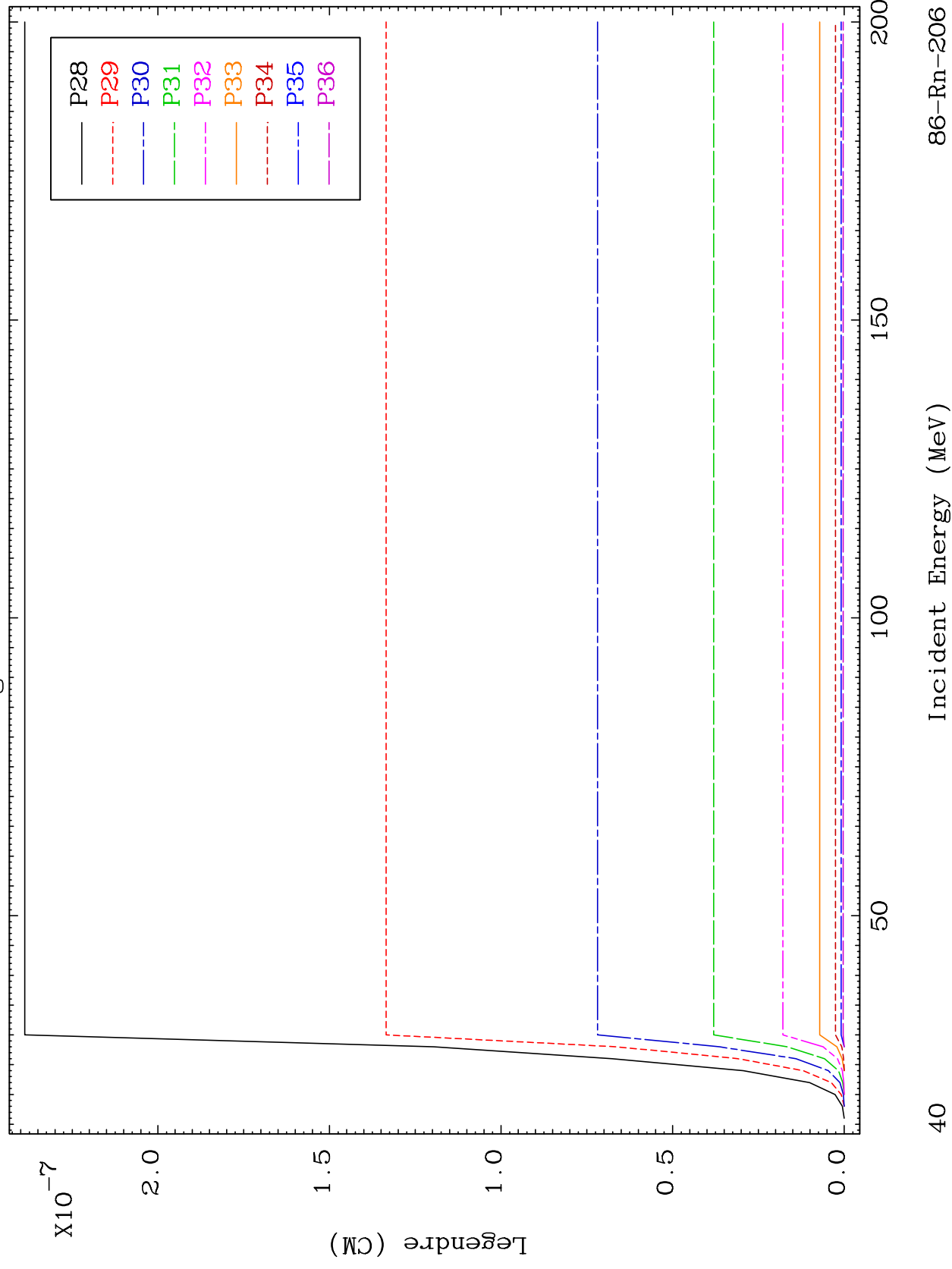
Incident Energy (MeV)

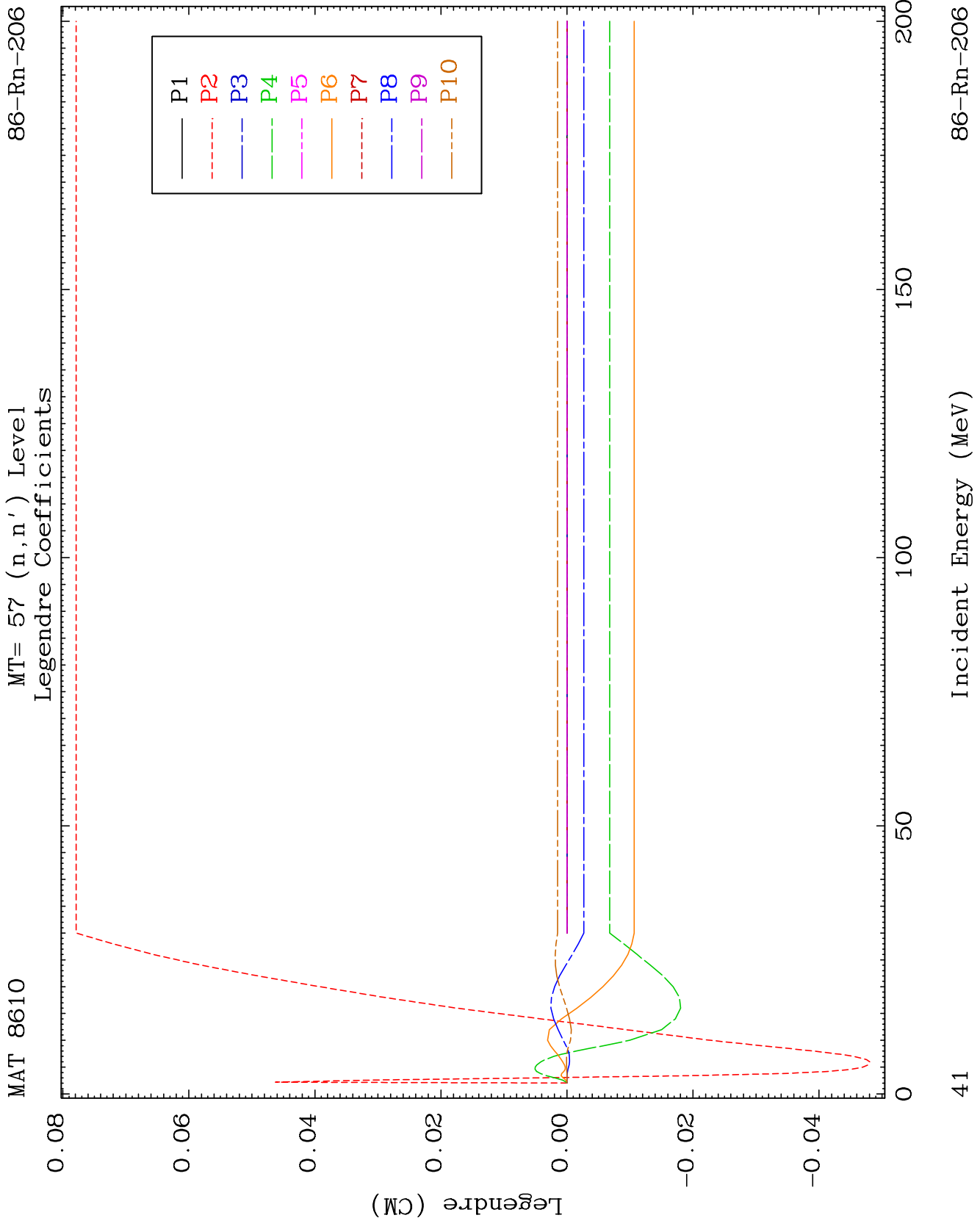
86-Rn-206

MAT 8610

MT= 56 (n,n') Level
Legendre Coefficients

86-Rn-206

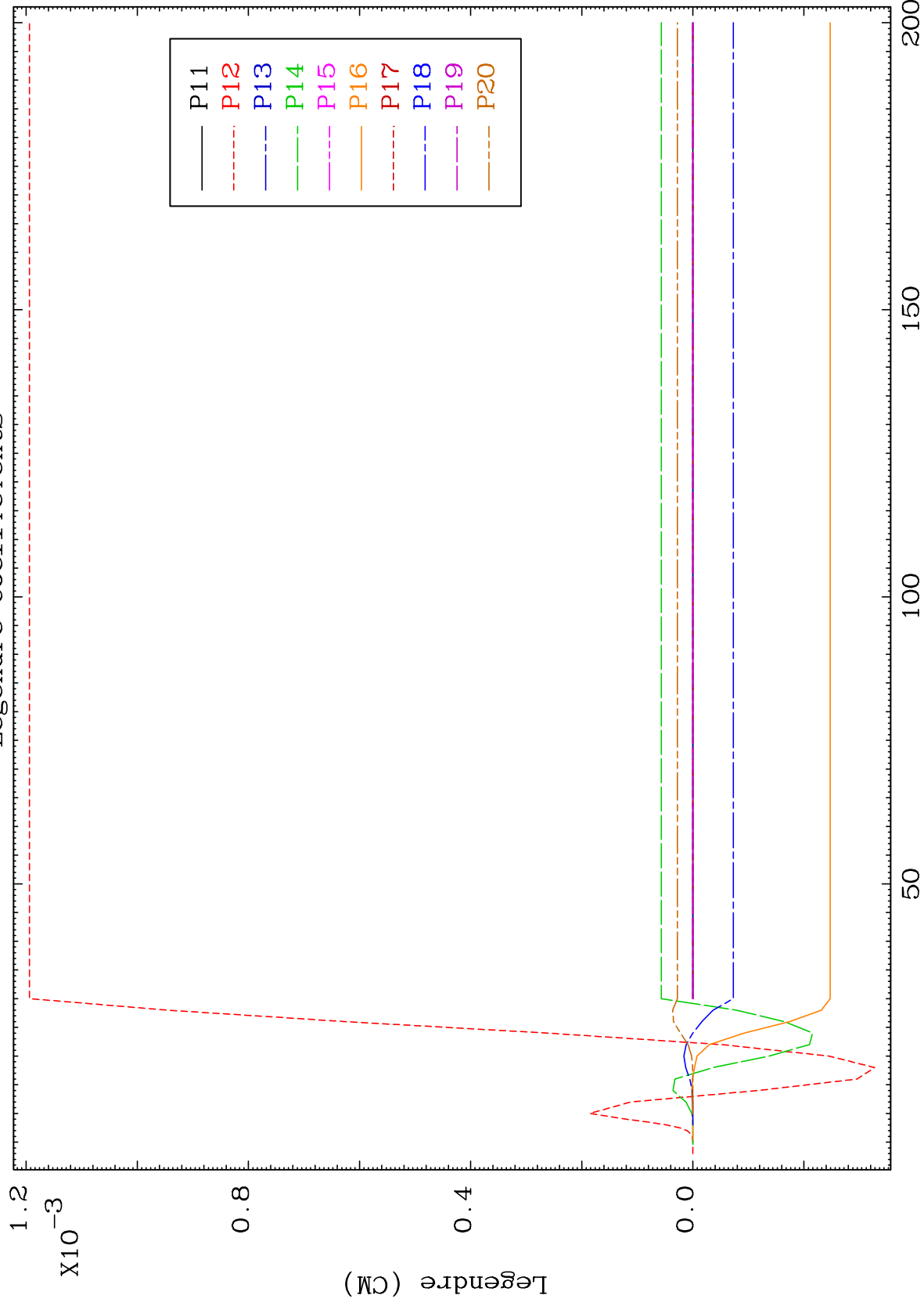




MAT 8610

MT= 57 (n,n') Level
Legendre Coefficients

86-Rn-206



42

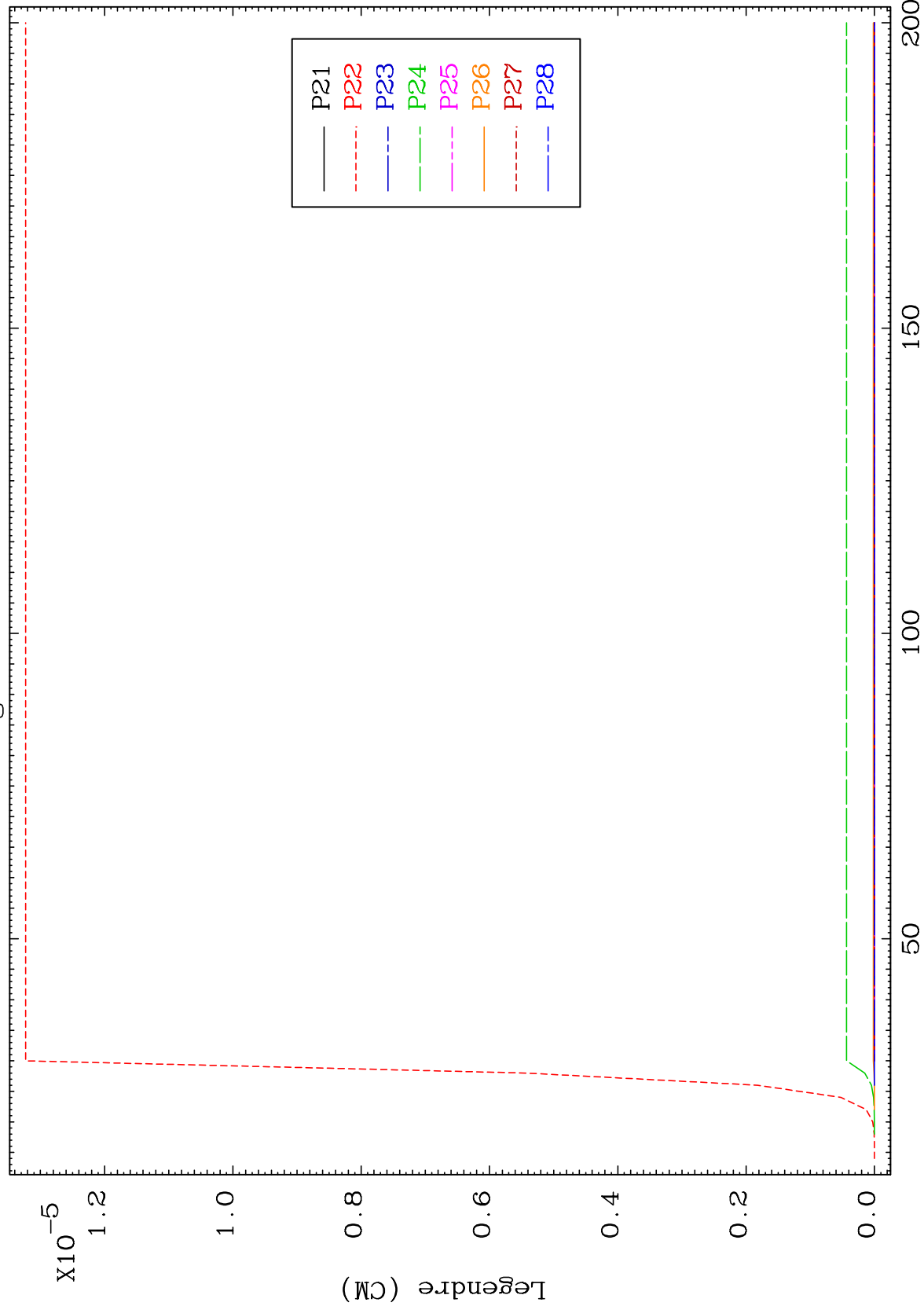
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 57 (n,n') Level
Legendre Coefficients

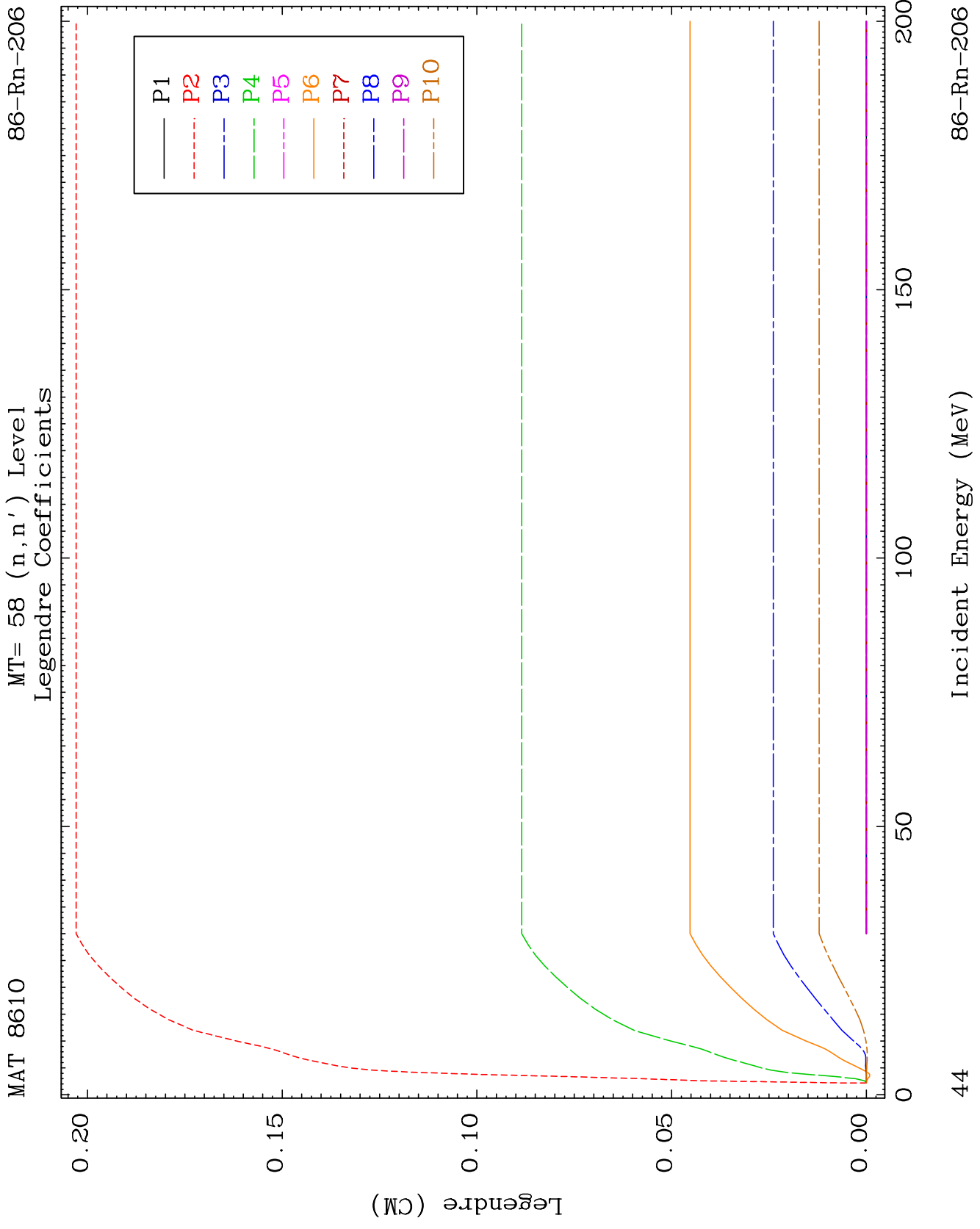
86-Rn-206



86-Rn-206

Incident Energy (MeV)

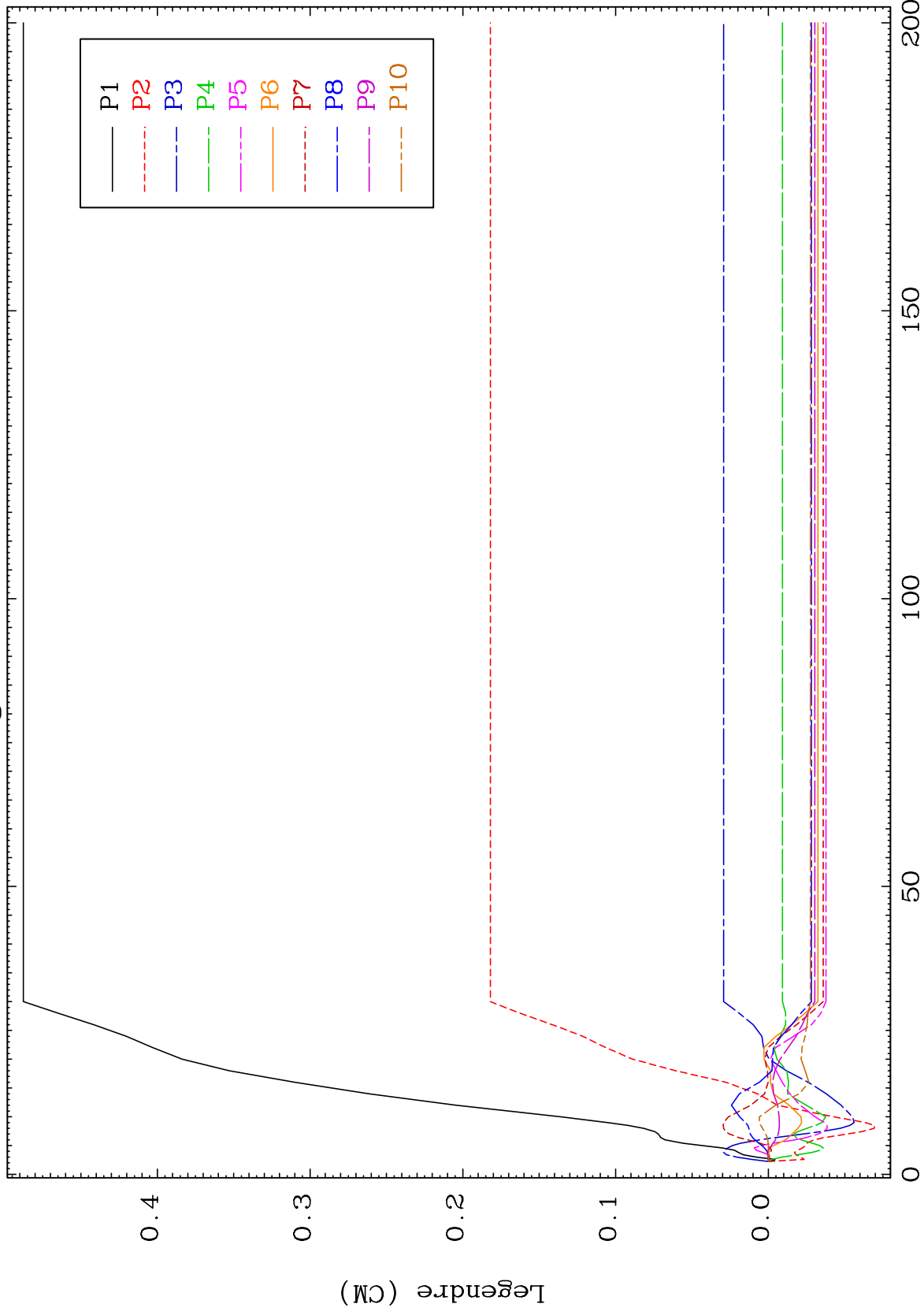
43



MAT 8610

MT= 59 (n,n') Level
Legendre Coefficients

86-Rn-206



47

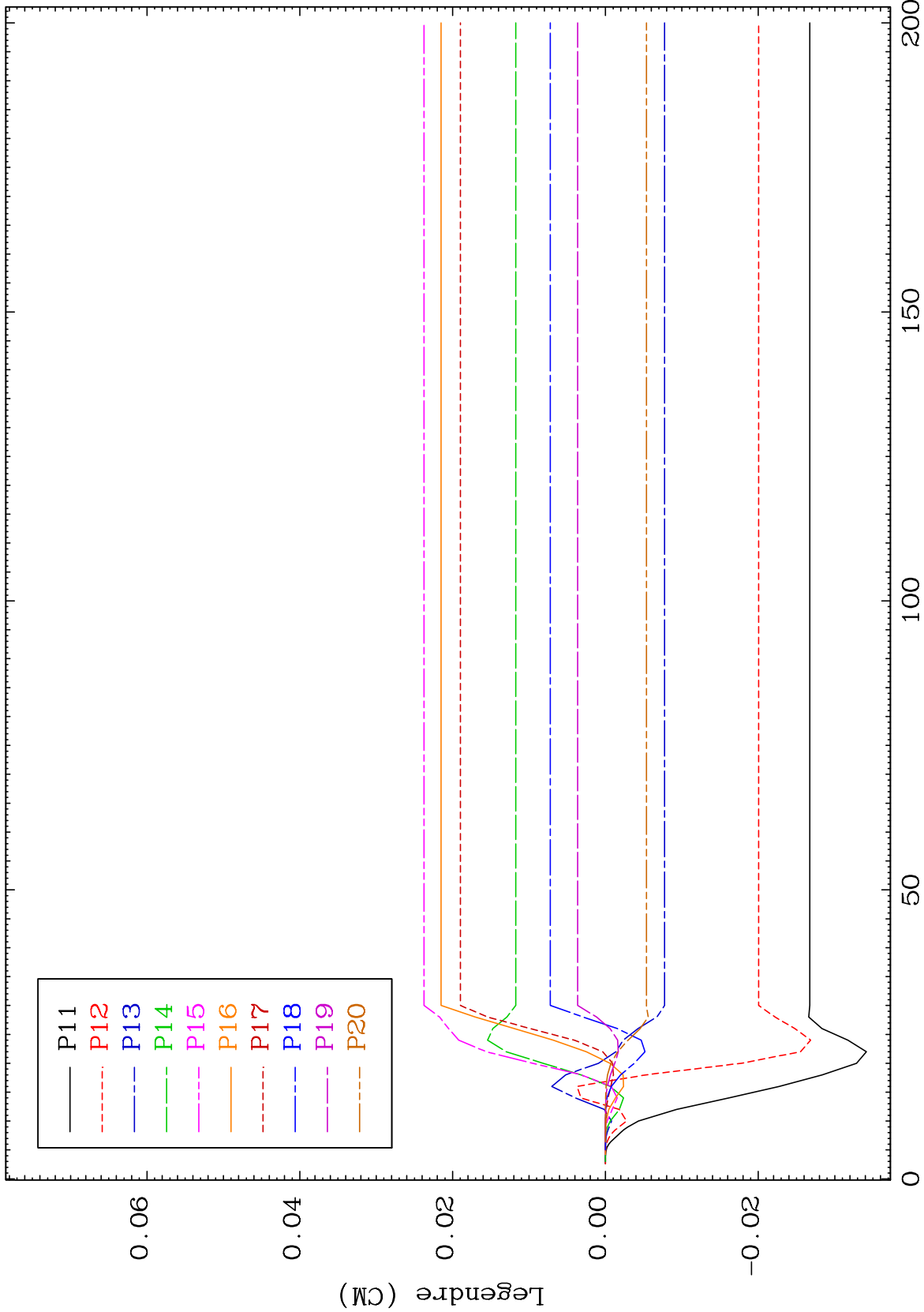
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 59 (n,n') Level Legendre Coefficients

86-Rn-206



48

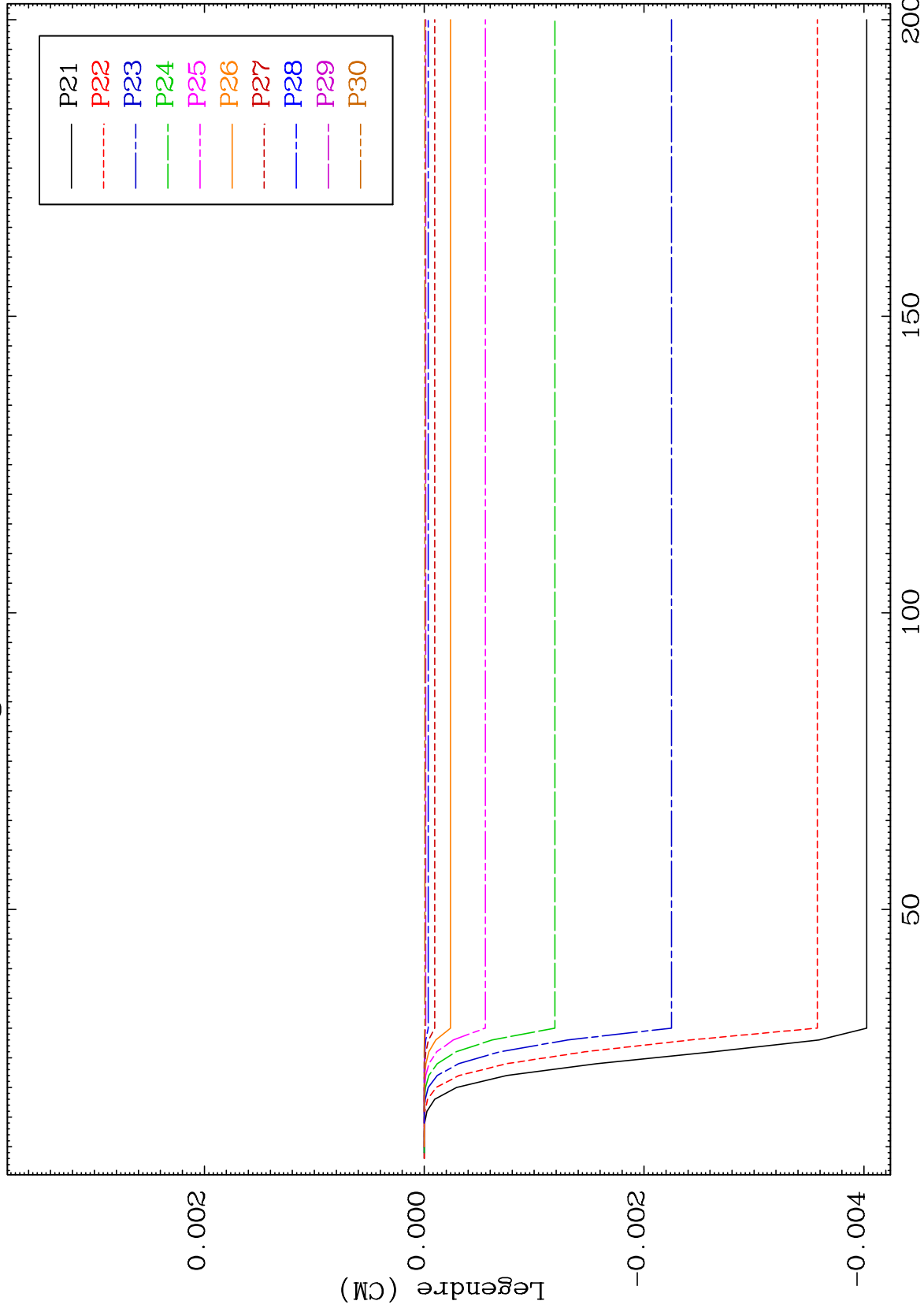
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 59 (n,n') Level
Legendre Coefficients

86-Rn-206



49

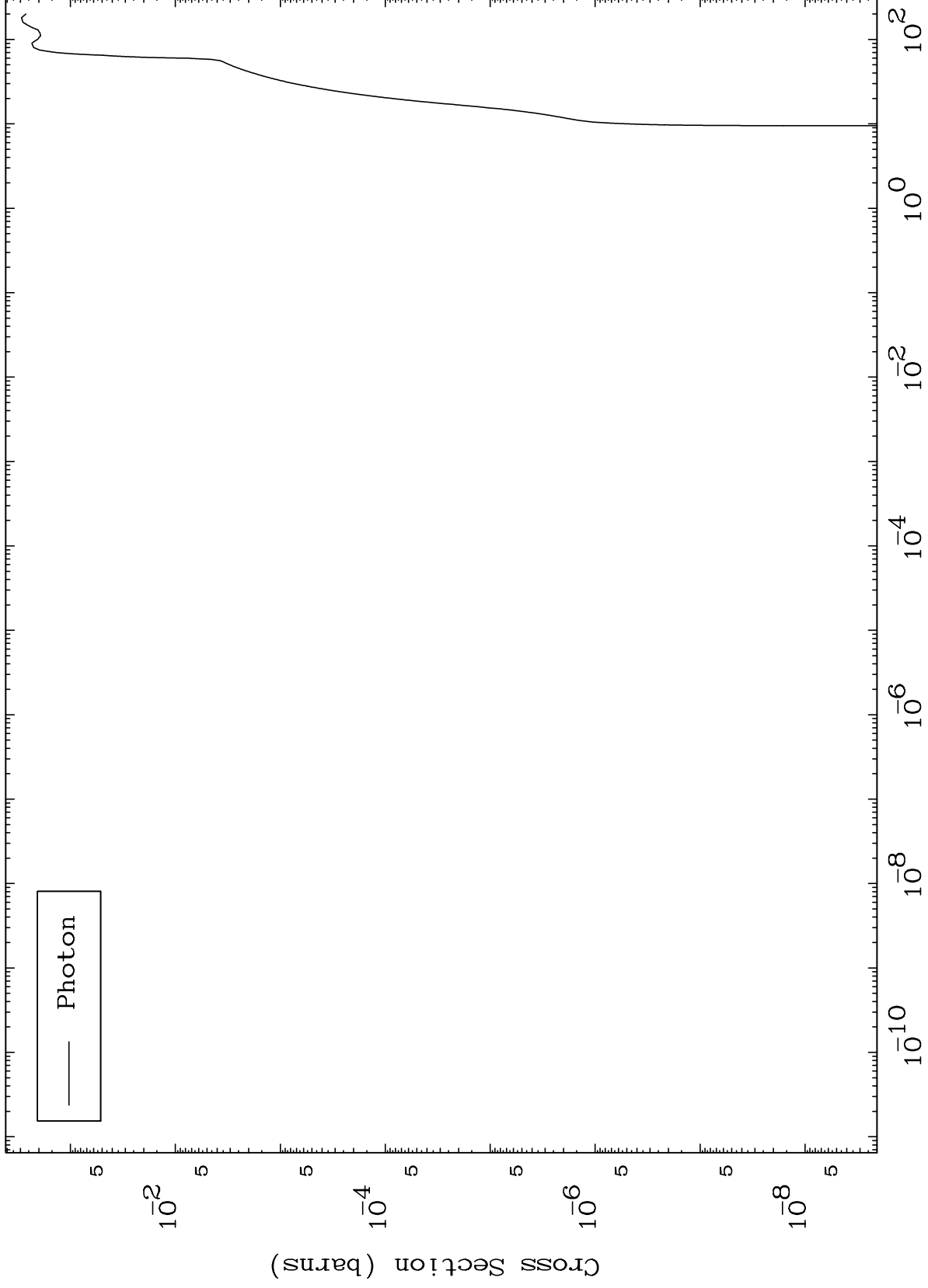
Incident Energy (MeV)

86-Rn-206

MAT 8610

Fission
Radionuclide Production Cross Section

86-Rn-206

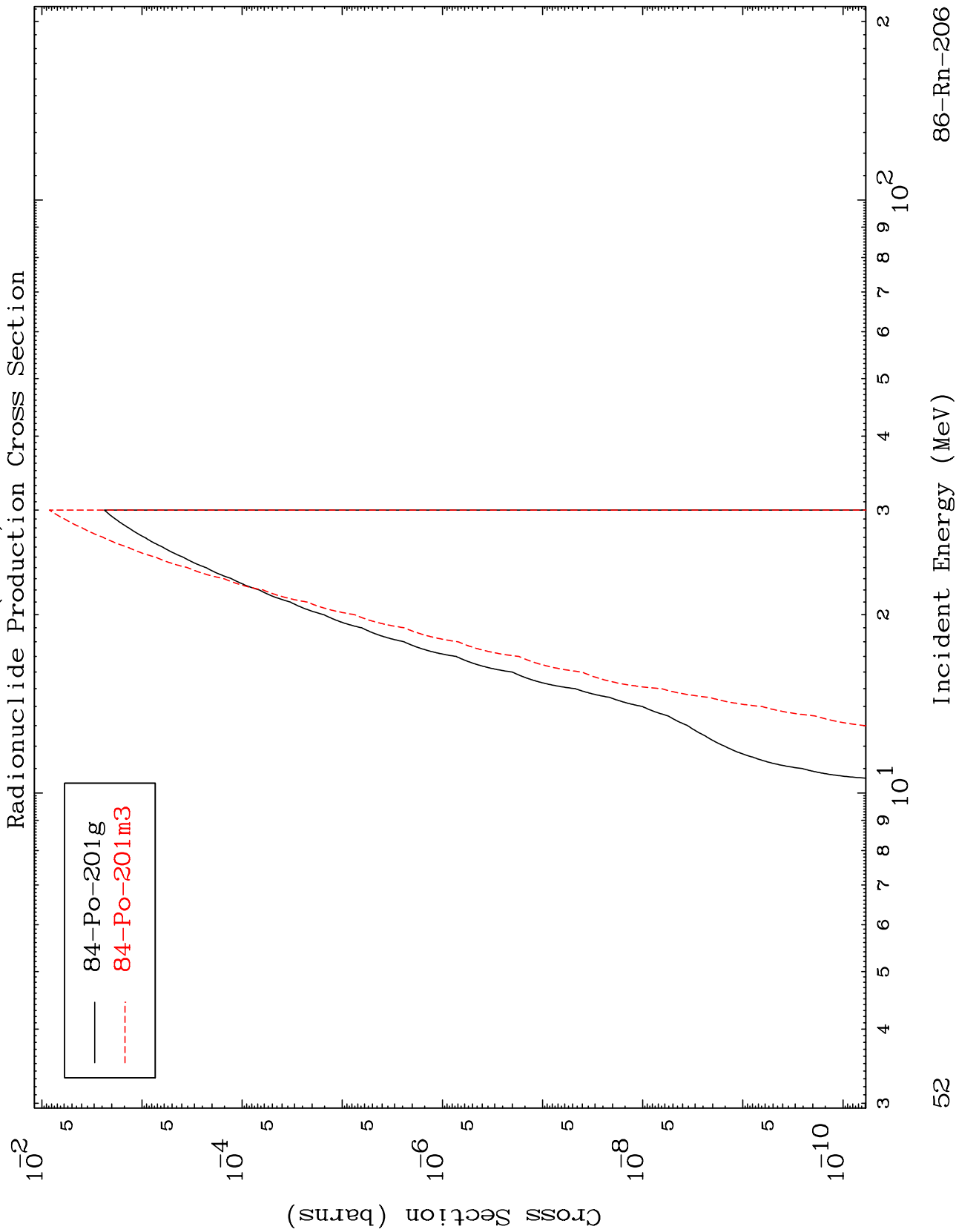


86-Rn-206

MAT 8610

$(n,2n) \alpha$

86-Rn-206

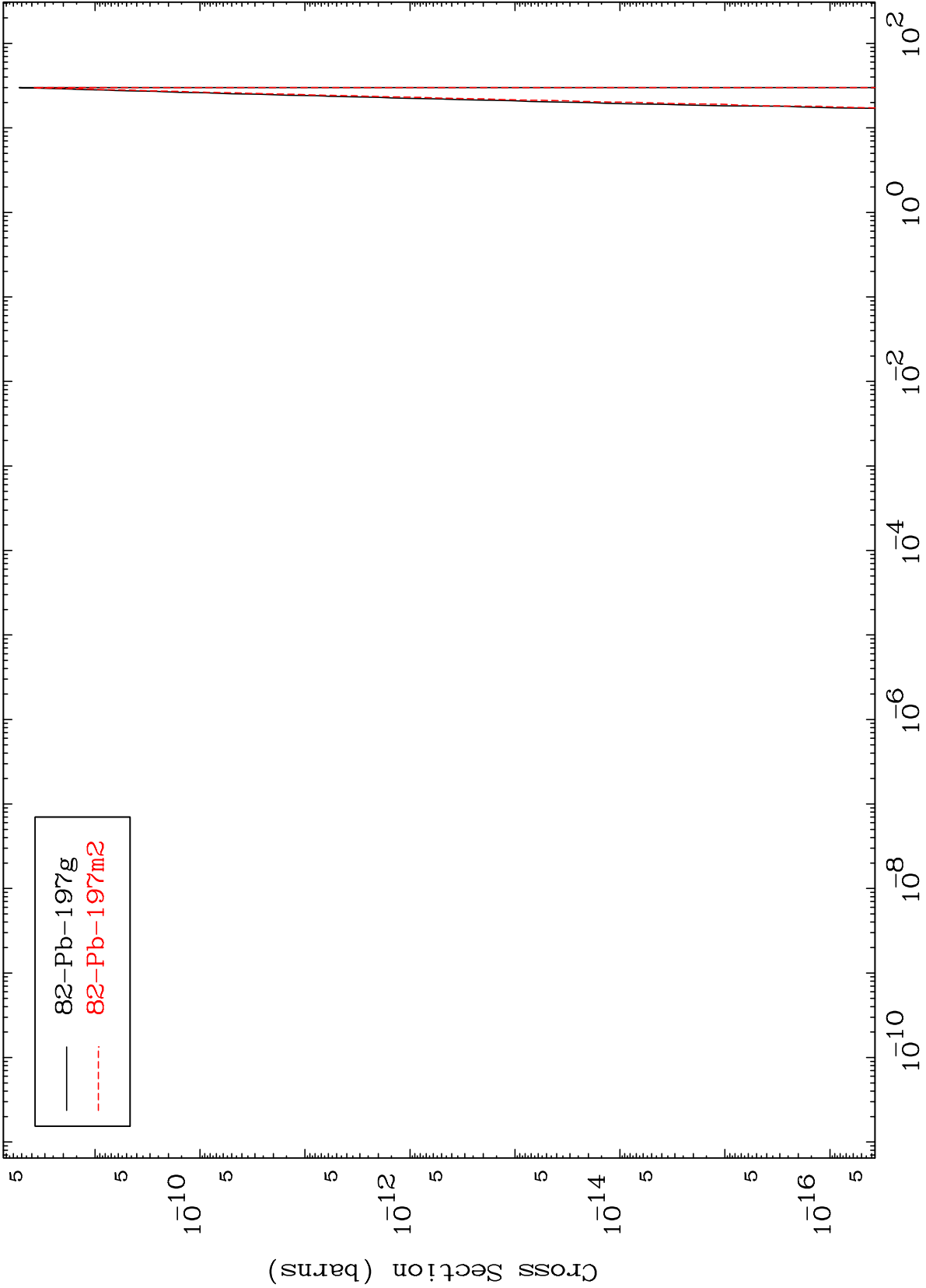


MAT 8610

$(n,2n) 2\alpha$

86-Rn-206

Radionuclide Production Cross Section

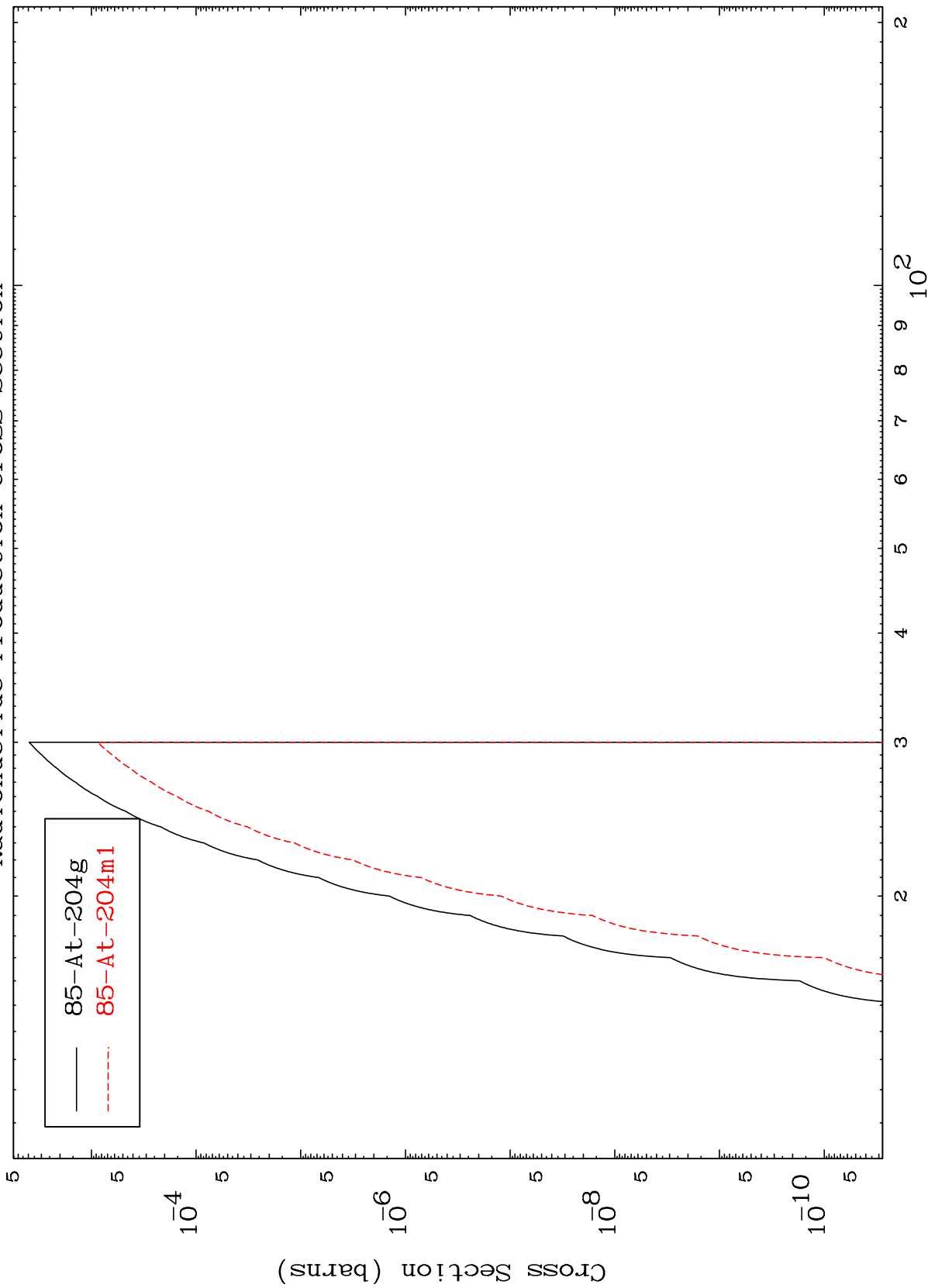


MAT 8610

(n,n') d

86-Rn-206

Radionuclide Production Cross Section



54

Incident Energy (MeV)

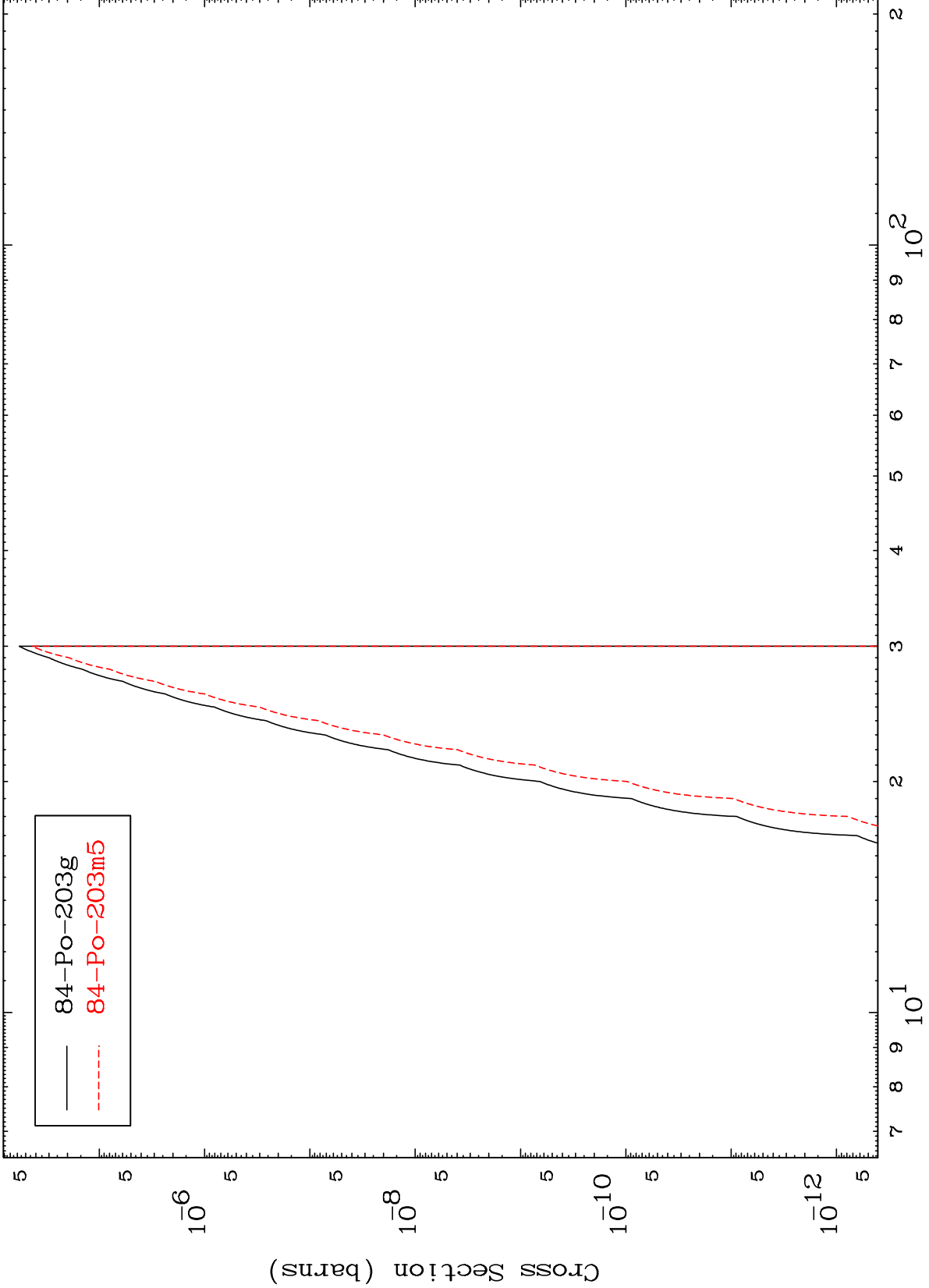
86-Rn-206

MAT 8610

(n,n') He-3

86-Rn-206

Radionuclide Production Cross Section



84-Po-203g
84-Po-203m5

55

Incident Energy (MeV)

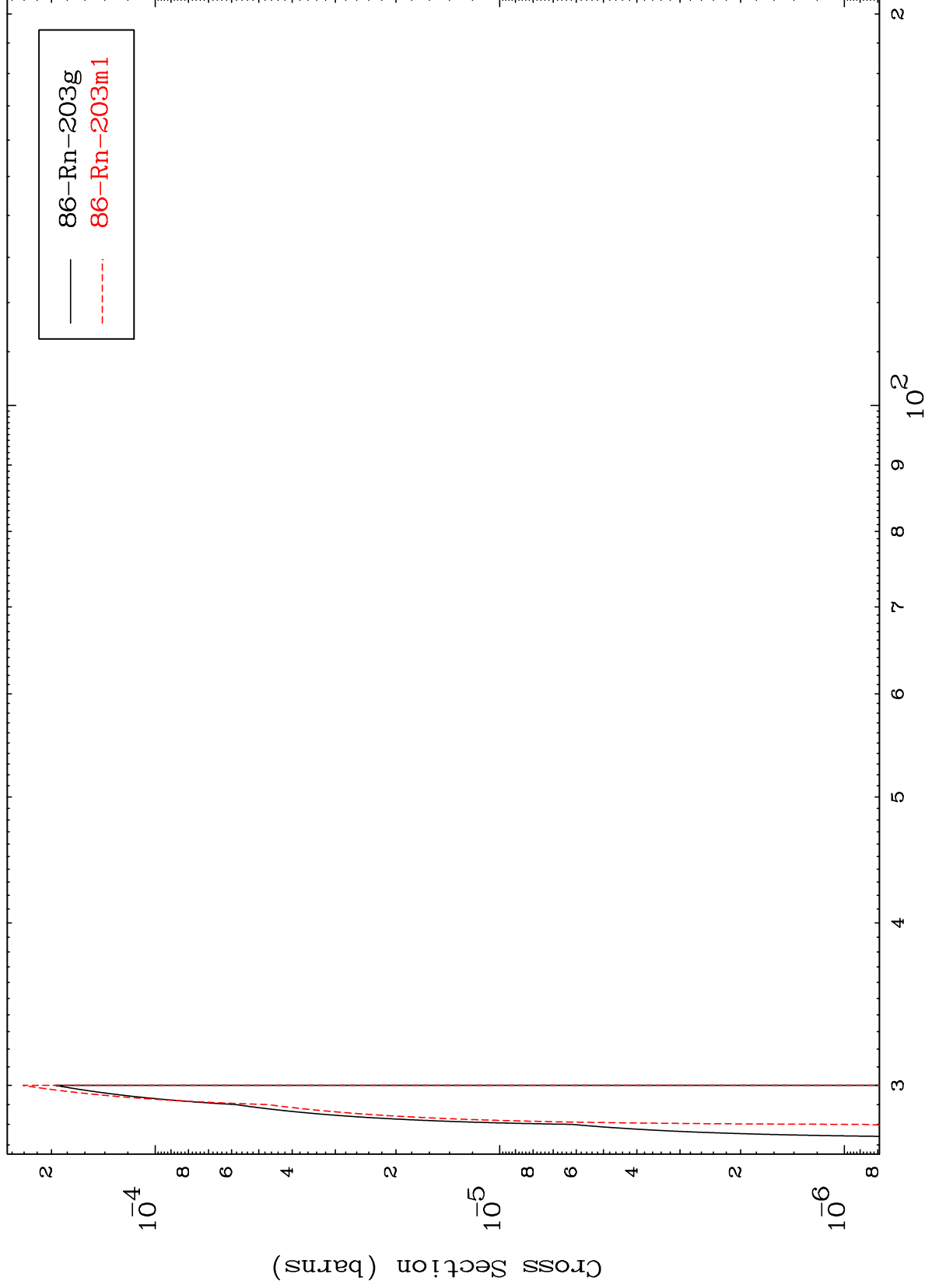
86-Rn-206

MAT 8610

(n,4n)

86-Rn-206

Radionuclide Production Cross Section



56

Incident Energy (MeV)

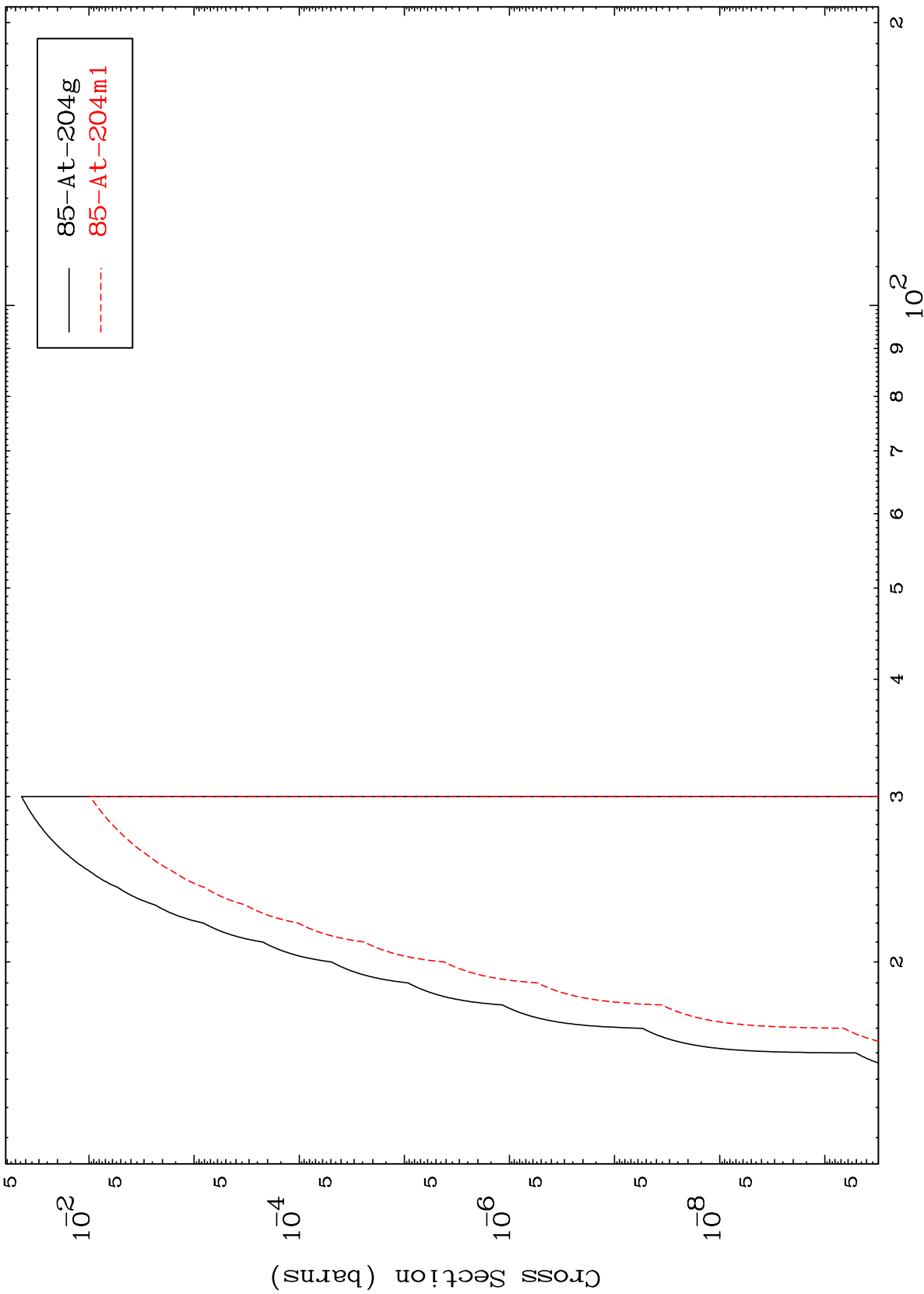
86-Rn-206

MAT 8610

(n,2n) p

86-Rn-206

Radionuclide Production Cross Section



57

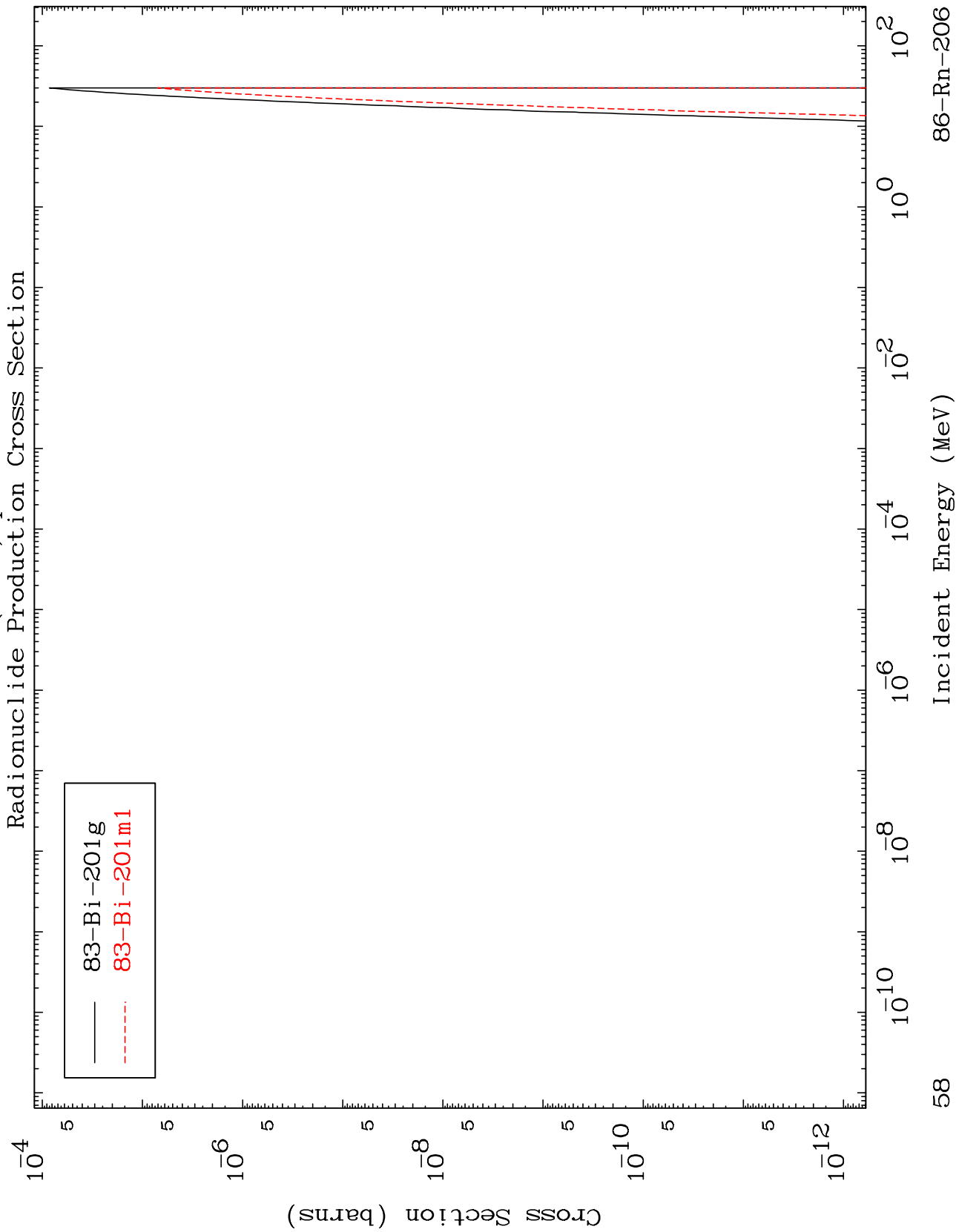
Incident Energy (MeV)

86-Rn-206

MAT 8610

(n,n') p α

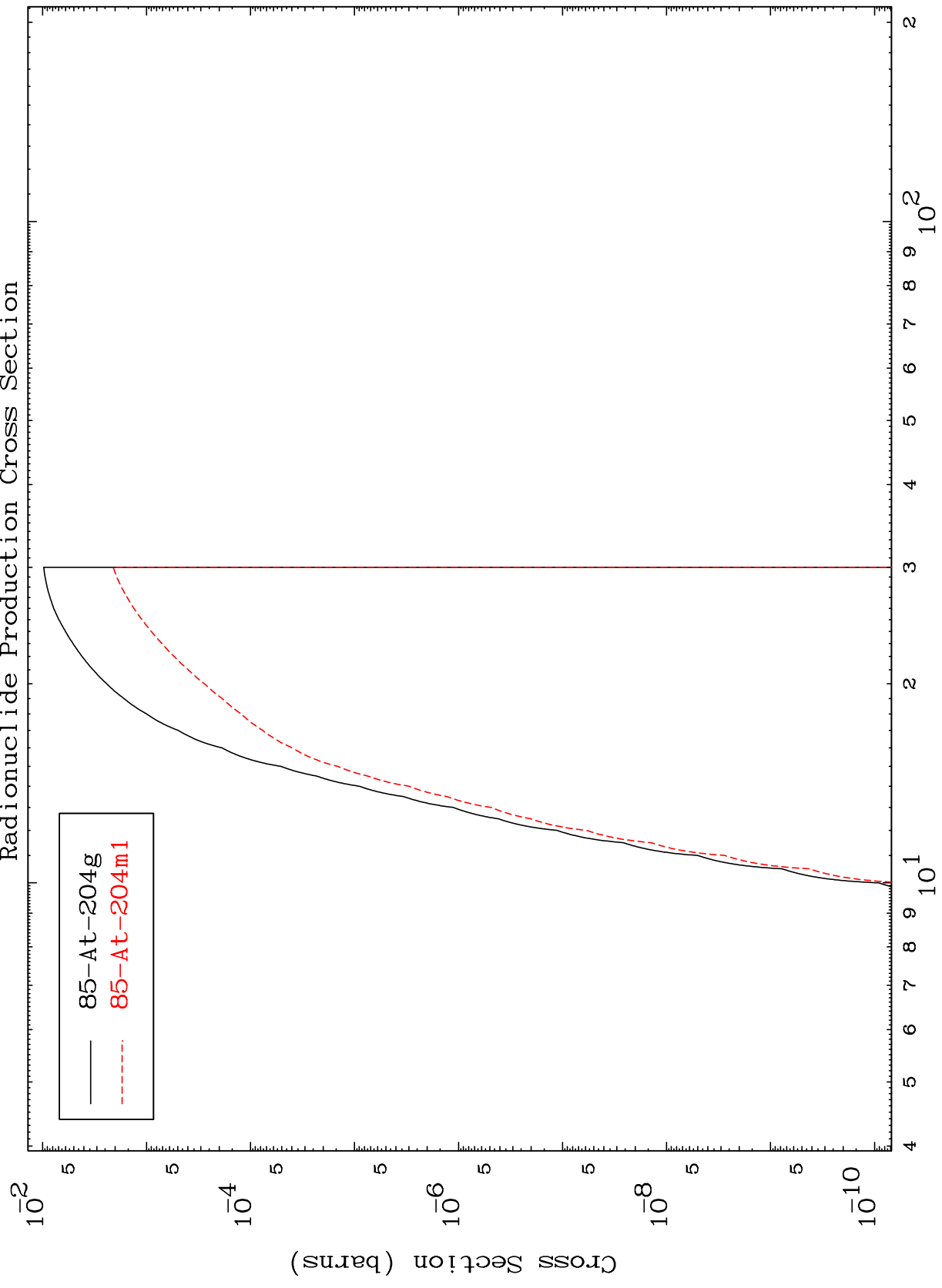
86-Rn-206



MAT 8610

86-Rn-206

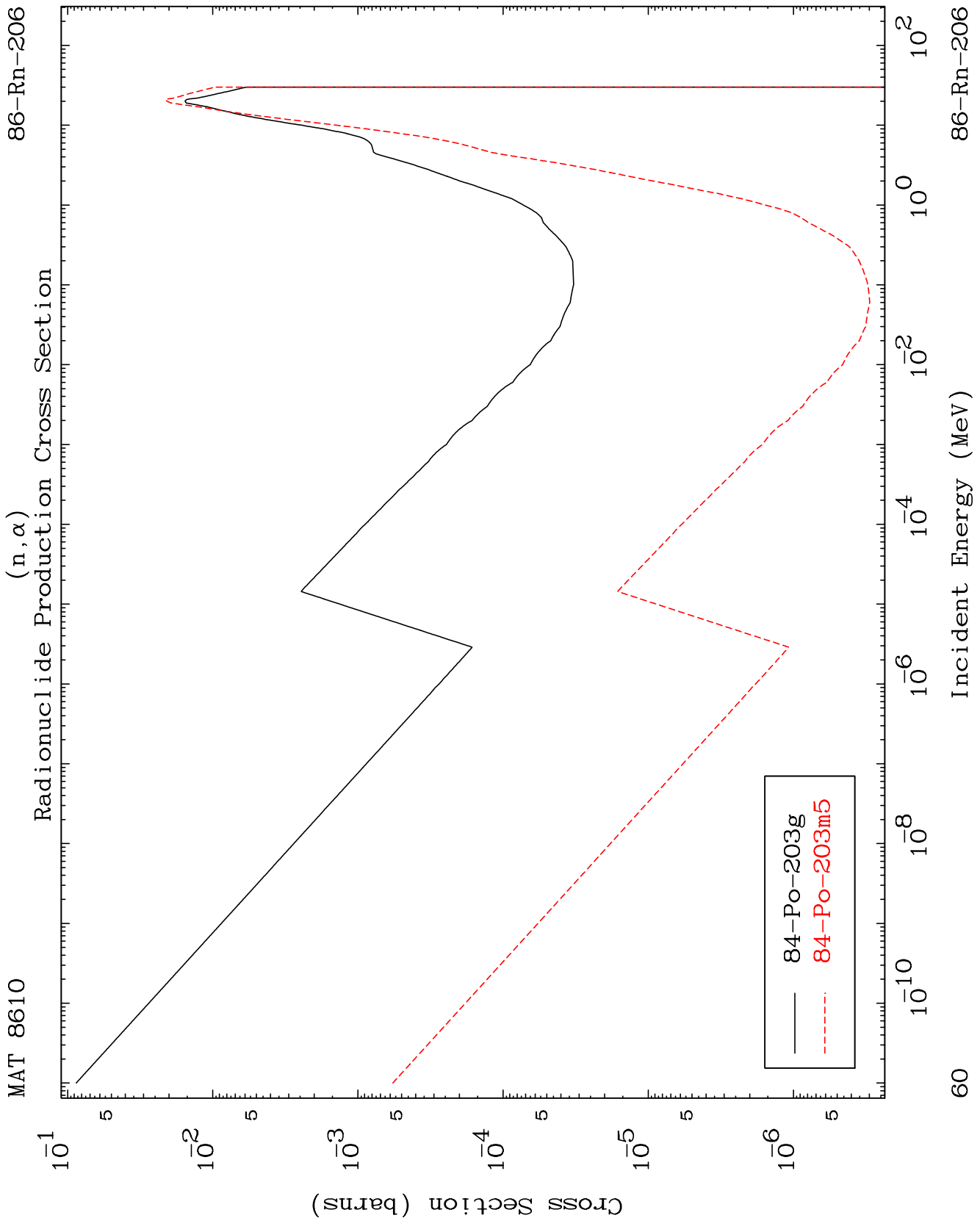
Radionuclide Production Cross Section (n,t)



Incident Energy (MeV)

86-Rn-206

59

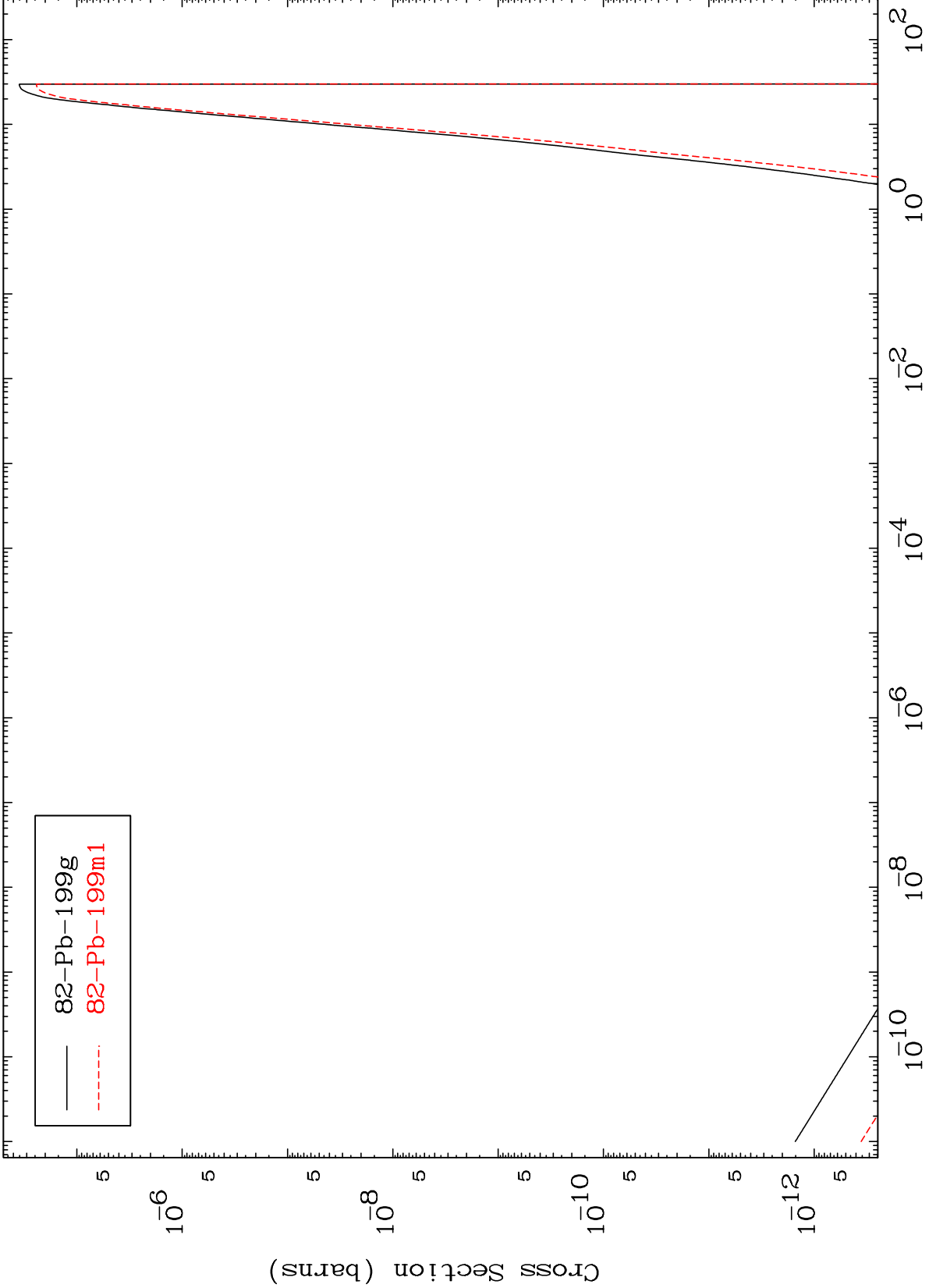
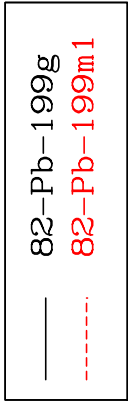


MAT 8610

(n,2α)

86-Rn-206

Radionuclide Production Cross Section



61

Incident Energy (MeV)

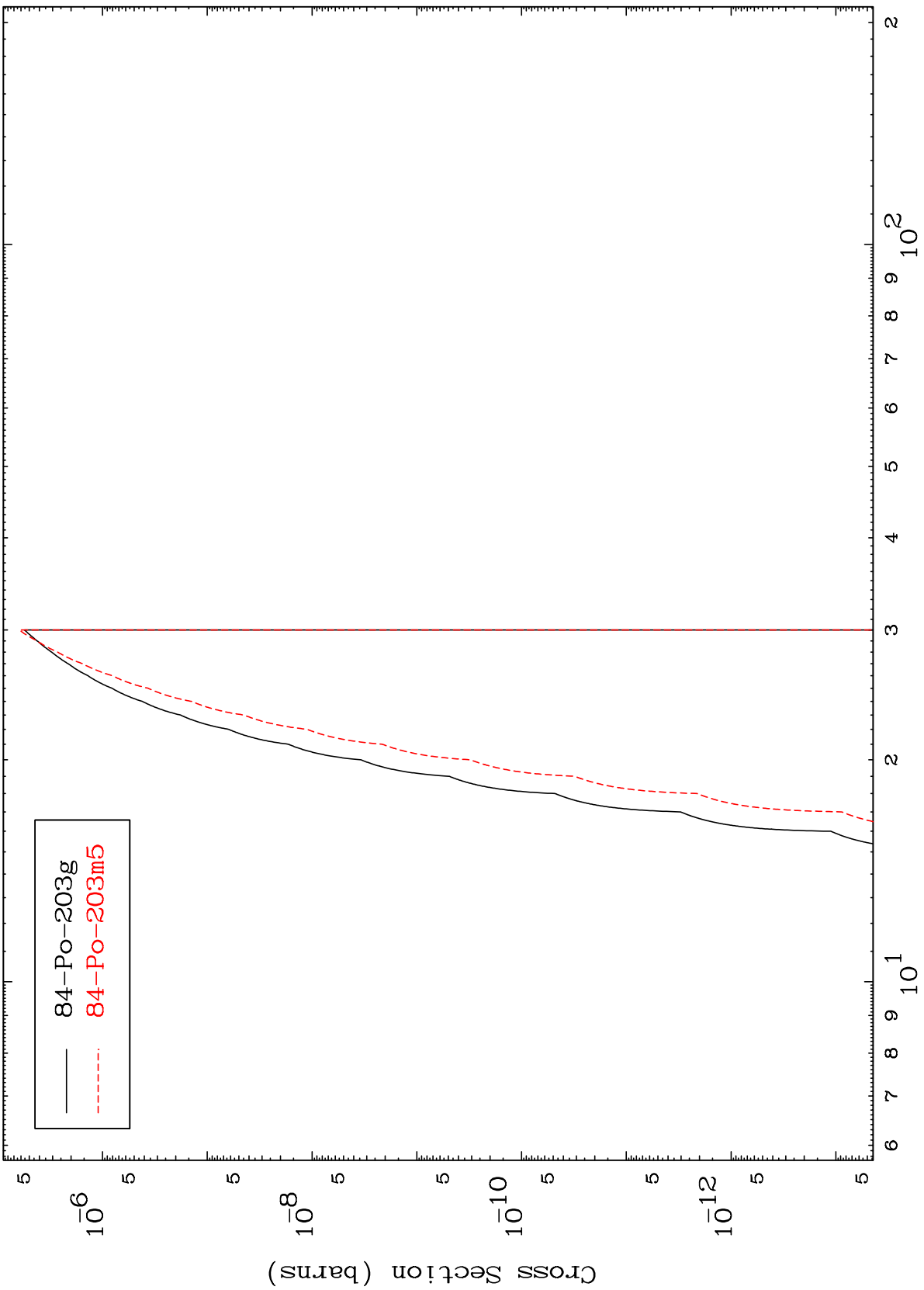
86-Rn-206

MAT 8610

(n,p) t

86-Rn-206

Radionuclide Production Cross Section



84-Po-203g
84-Po-203m5

62

Incident Energy (MeV)

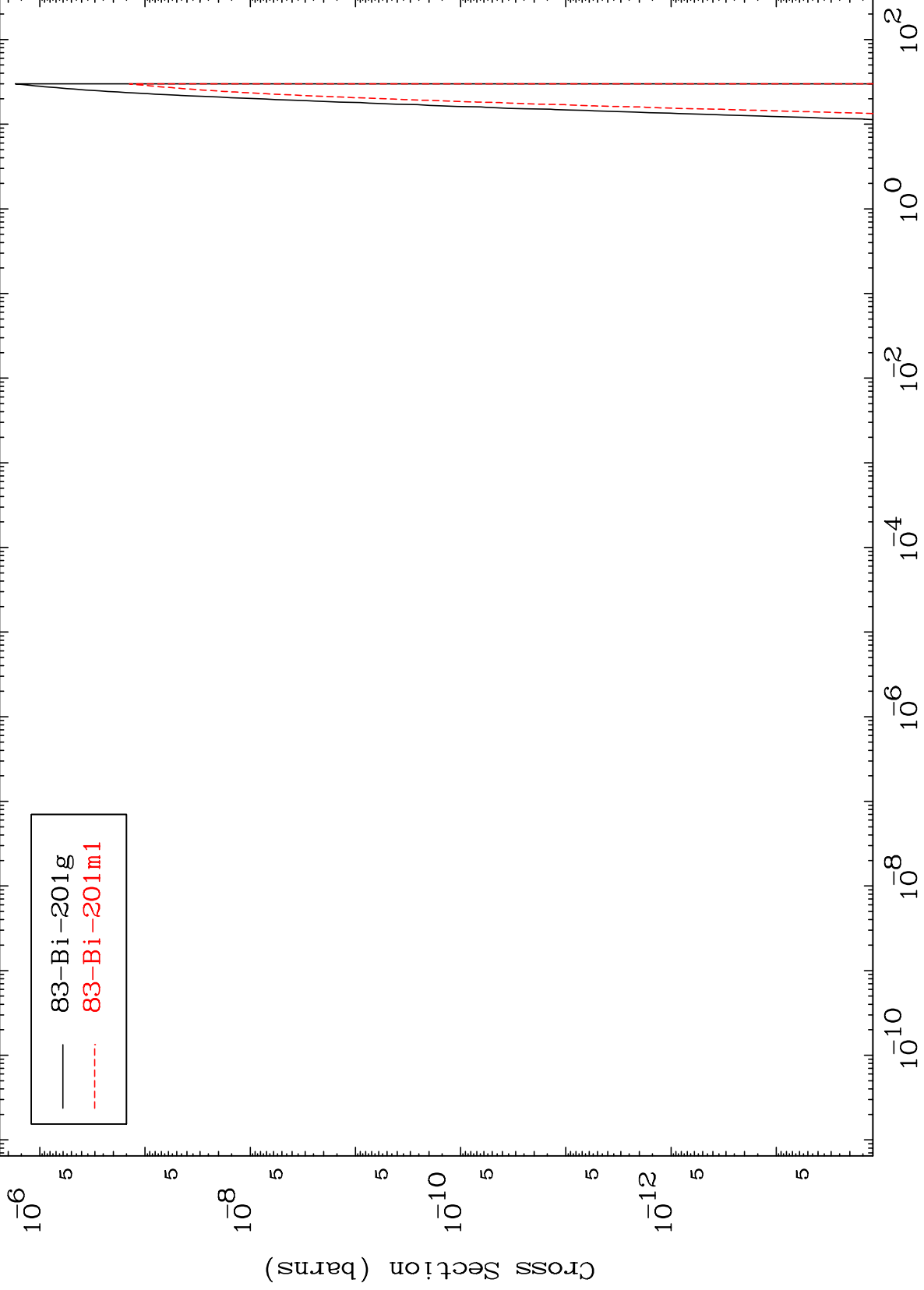
86-Rn-206

MAT 8610

(n,d) α

86-Rn-206

Radionuclide Production Cross Section



63

Incident Energy (MeV)

86-Rn-206