

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

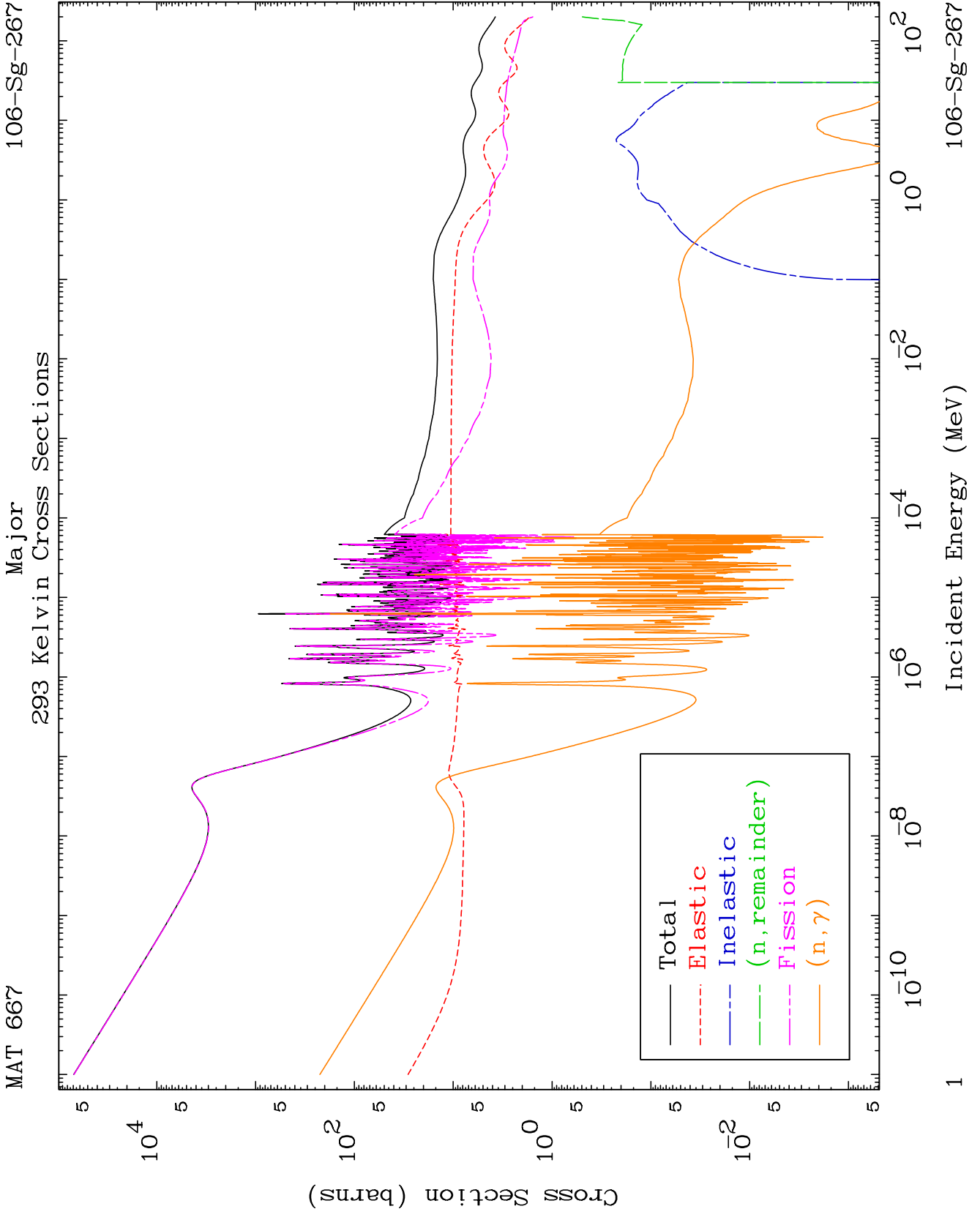
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Web:redcullen1.net/HOMEPAGE.NEW

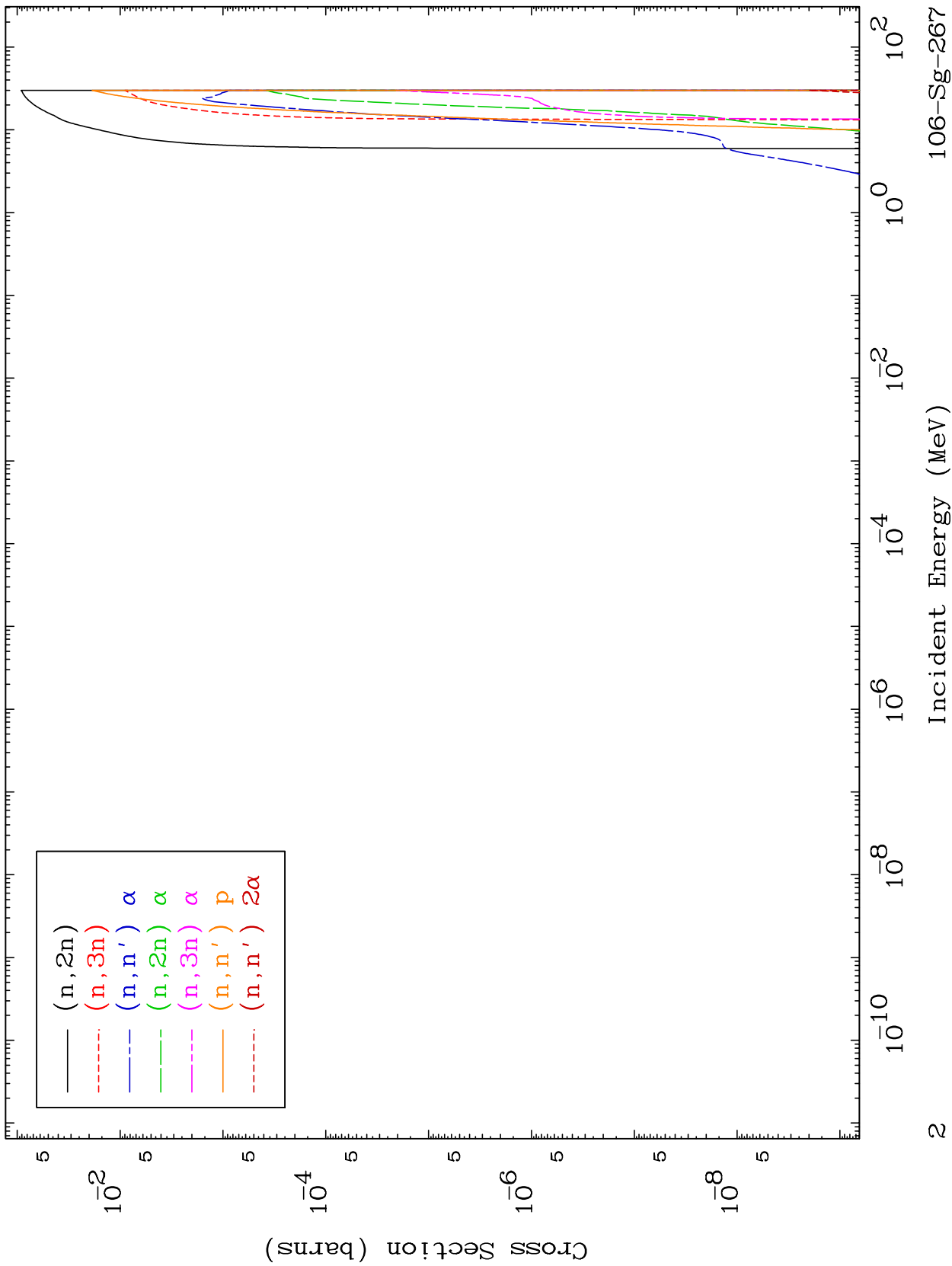
Press Mouse Button to Start

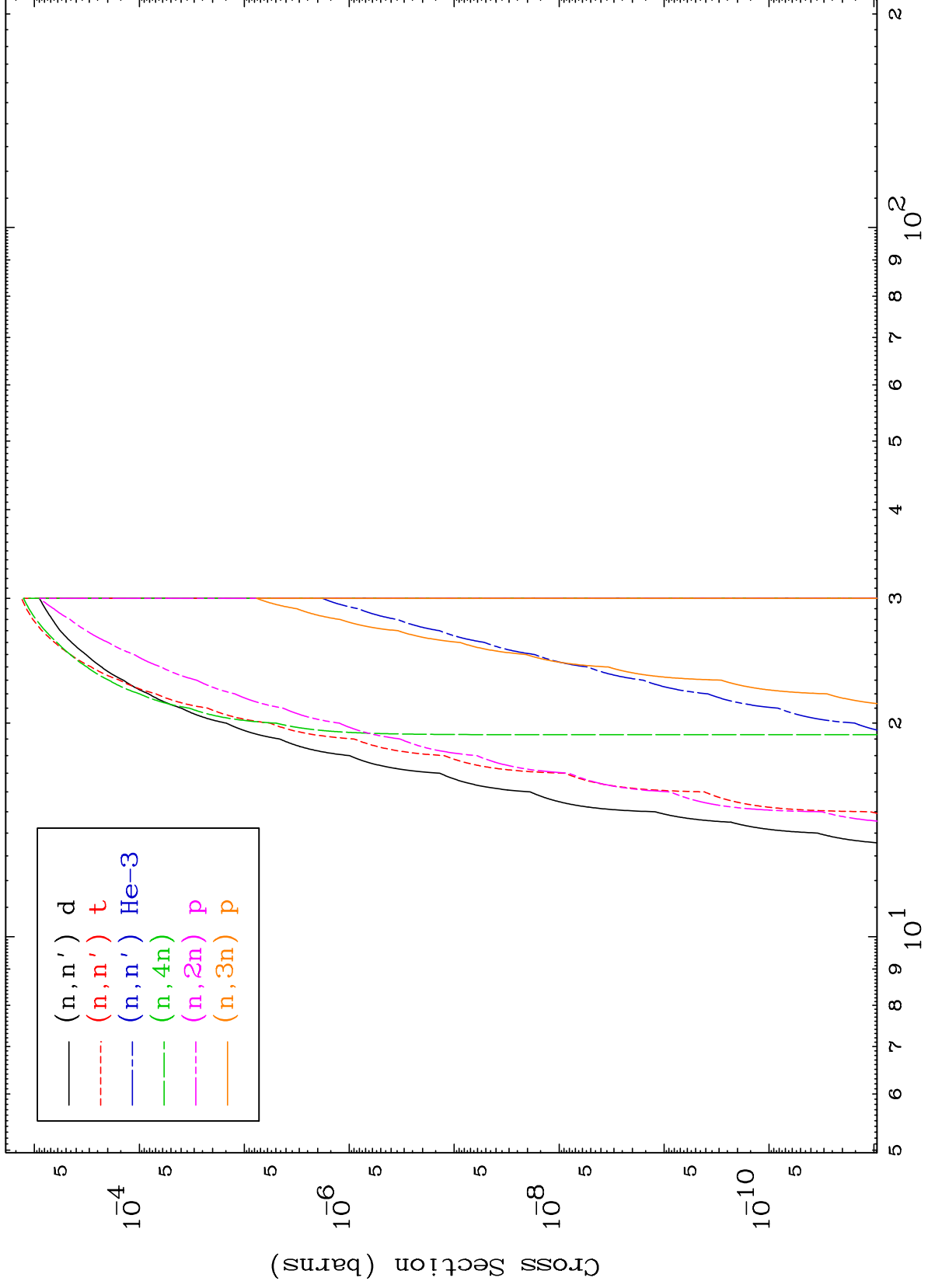


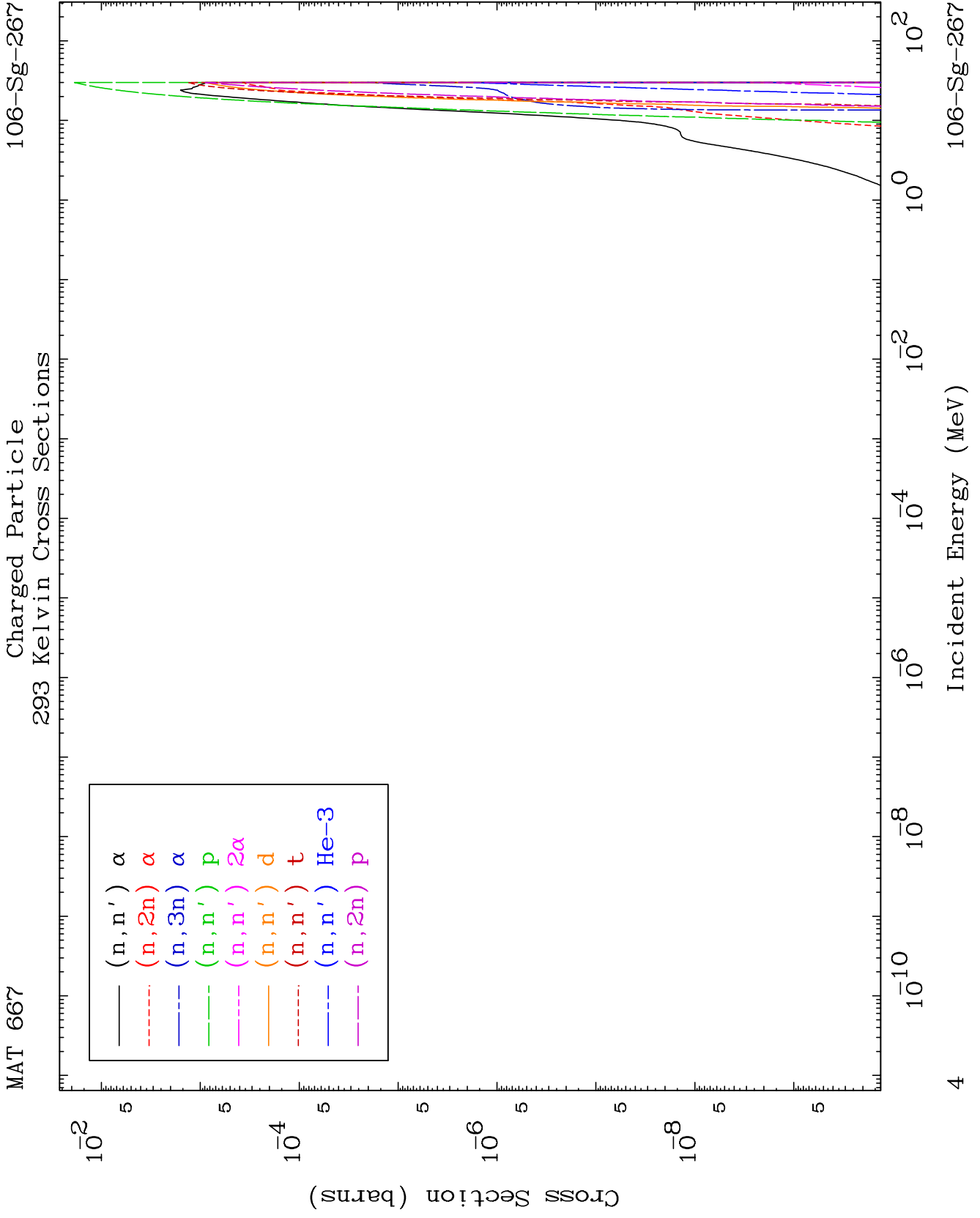
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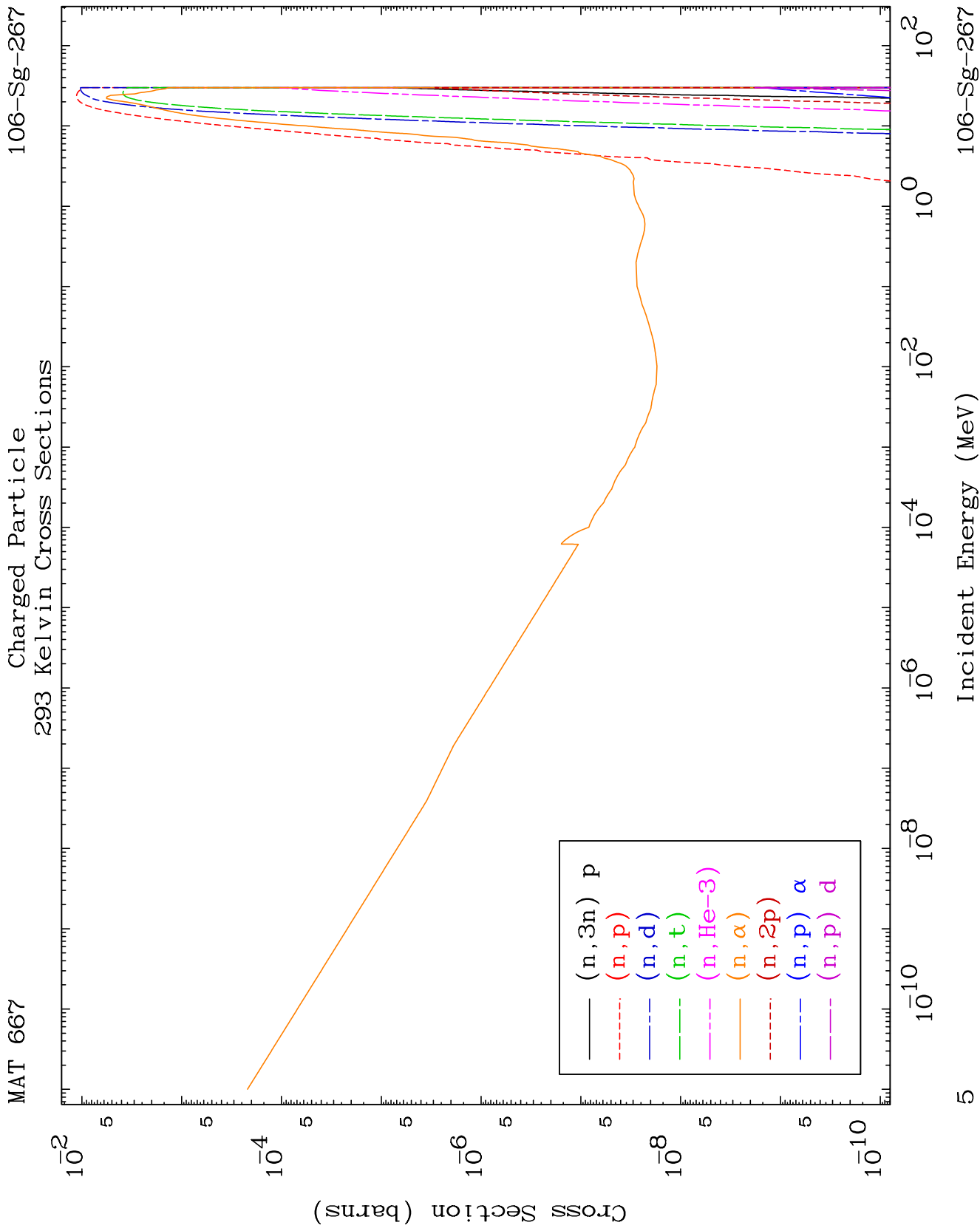
Neutron Production  
293 Kelvin Cross Sections

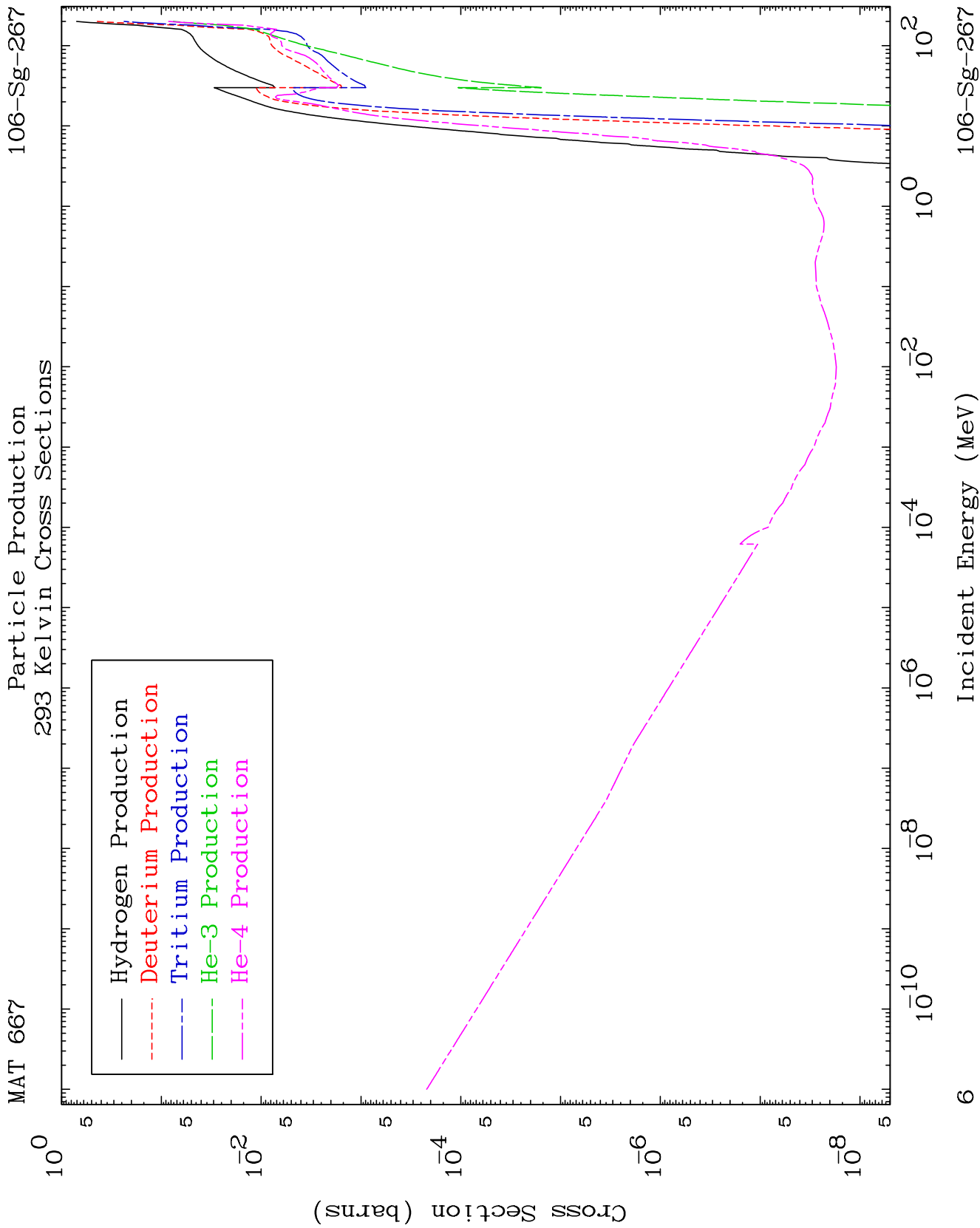
106-Sg-267







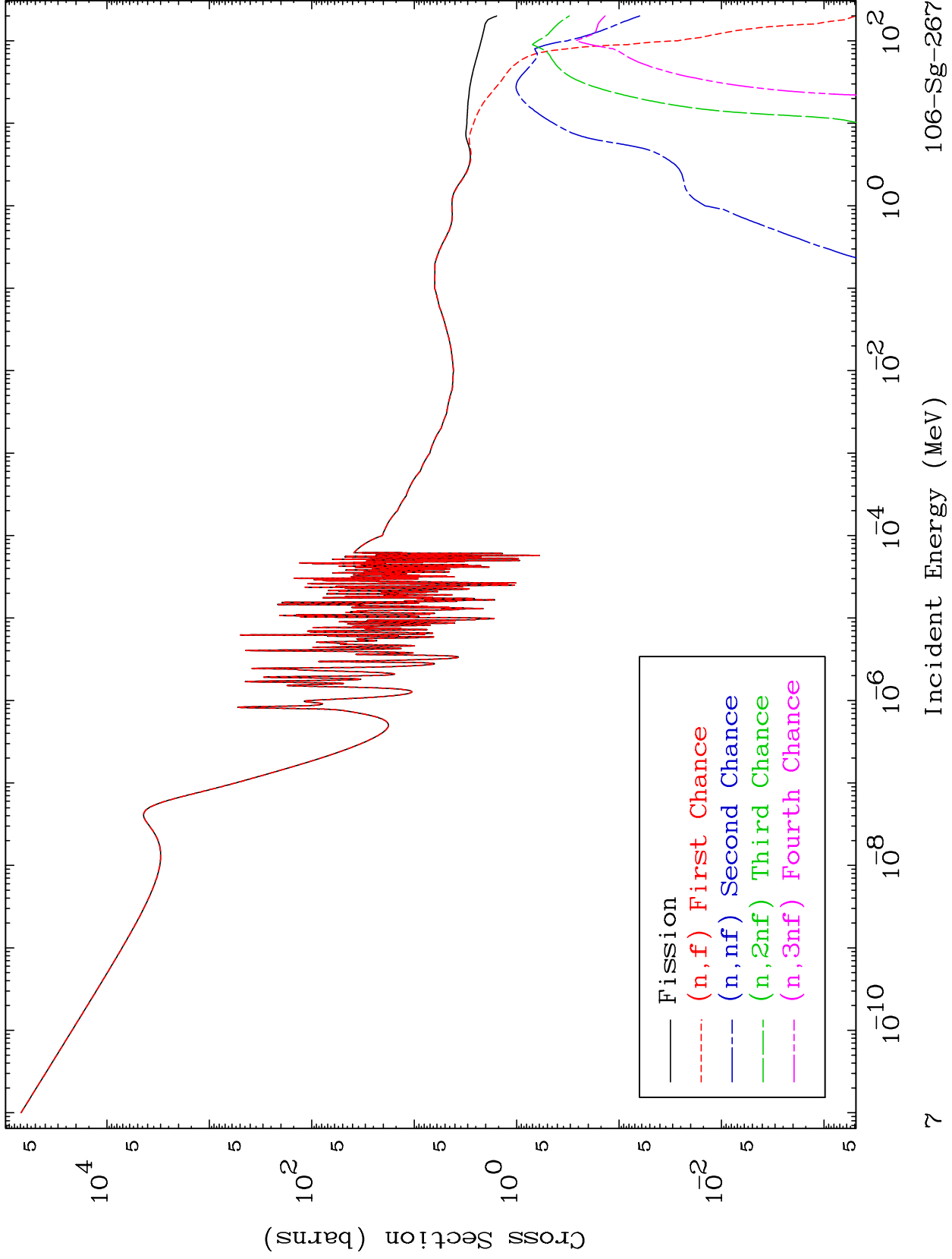




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Fission  
293 Kelvin Cross Sections

106-Sg-267

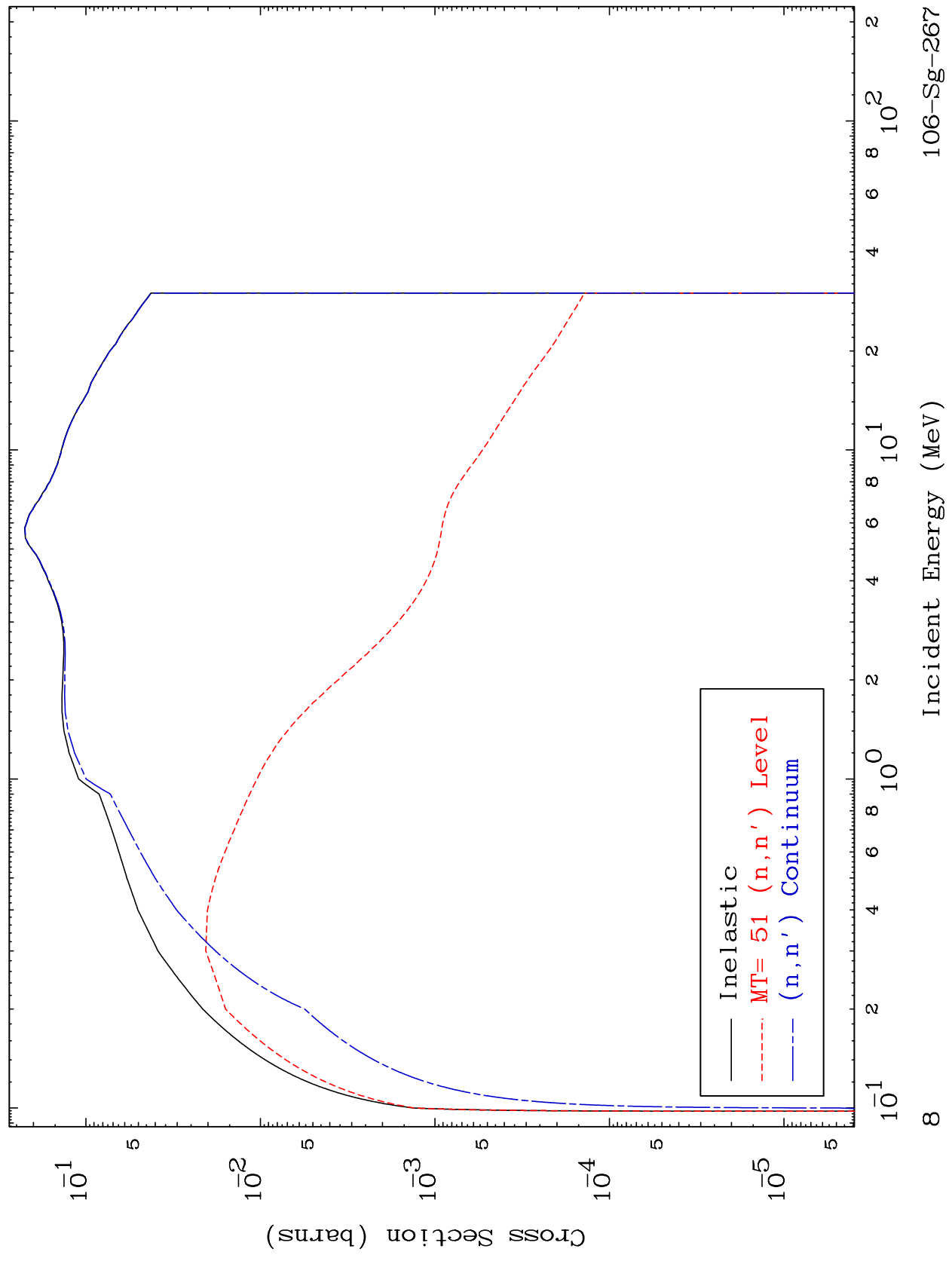




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293 Kelvin Cross Sections  
(n,n') Level

106-Sg-267



106-Sg-267

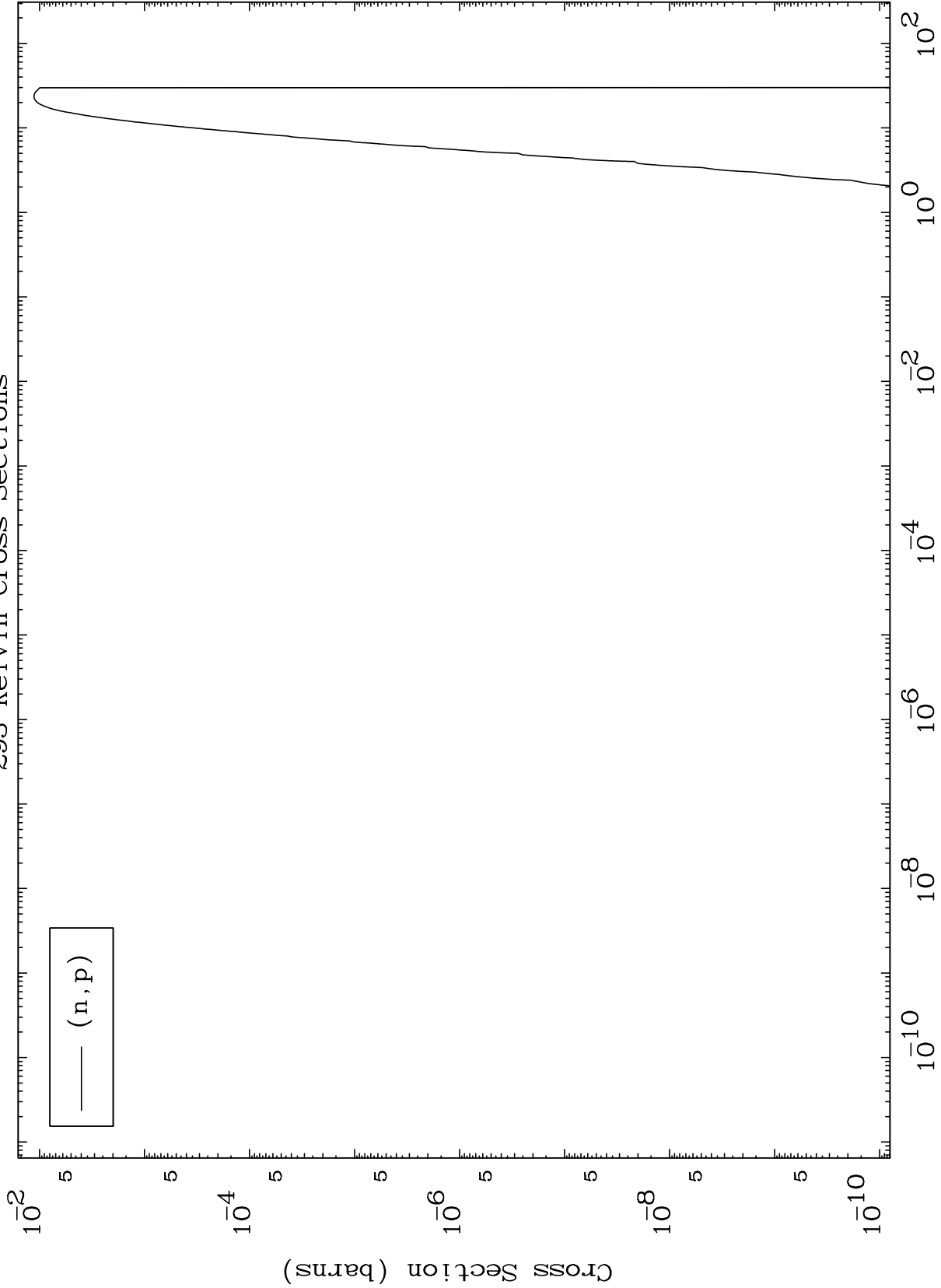
Incident Energy (MeV)

8

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(n,p) Levels  
293 Kelvin Cross Sections

106-Sg-267



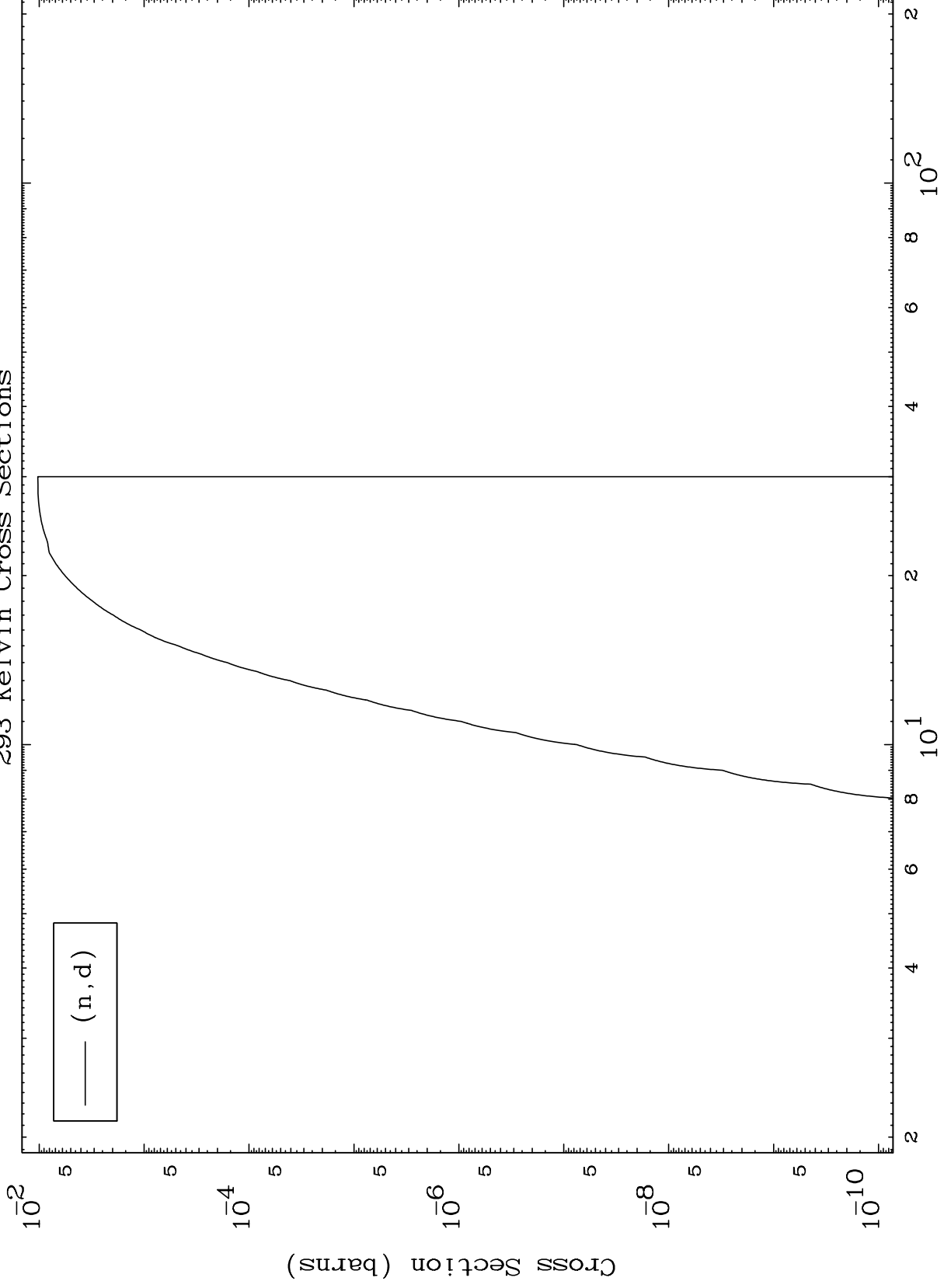
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Incident Energy (MeV)

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(n,d) Levels  
293 Kelvin Cross Sections

106-Sg-267



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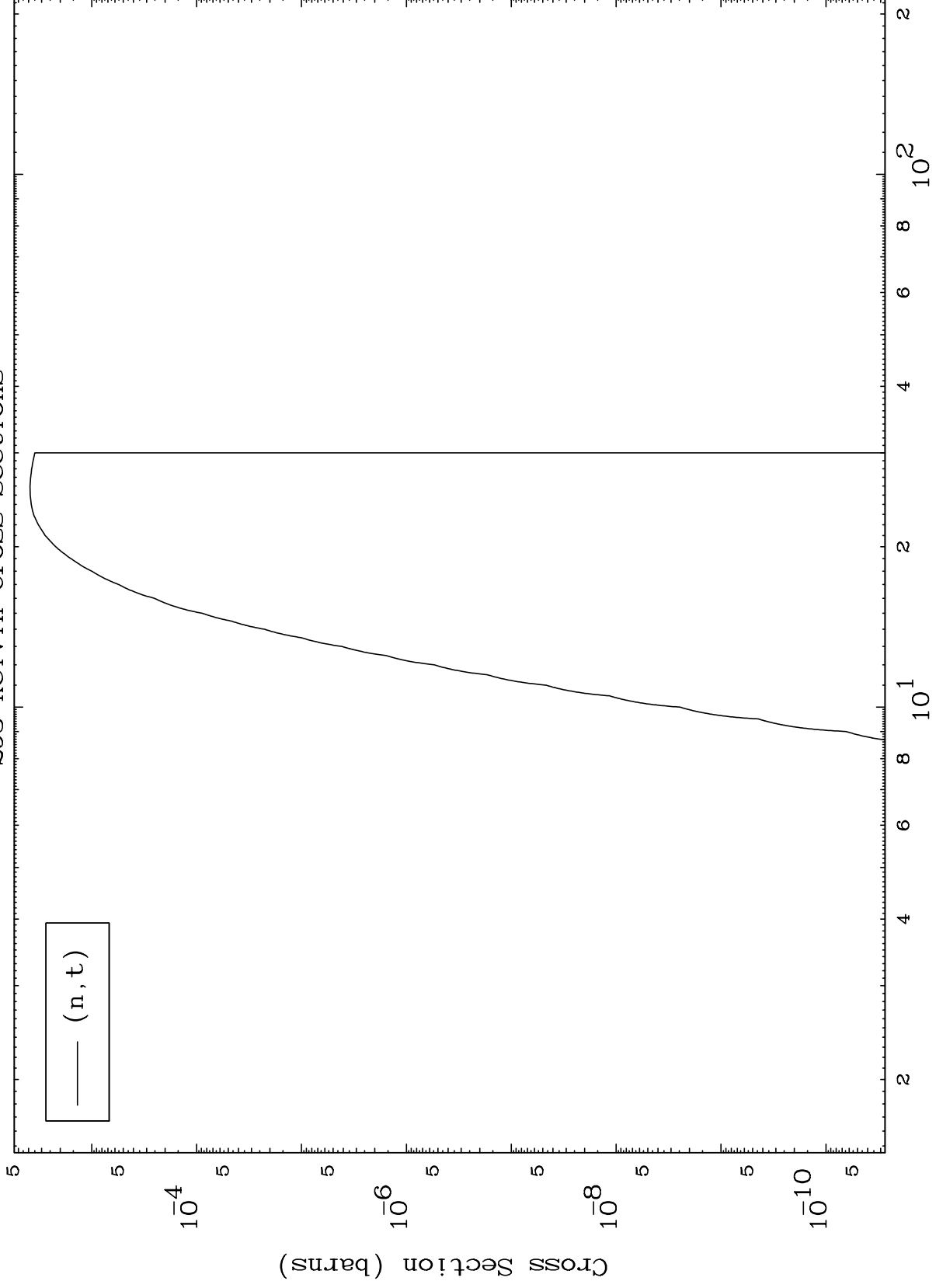
Incident Energy (MeV)

106-Sg-267

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(n,t) Levels  
293 Kelvin Cross Sections

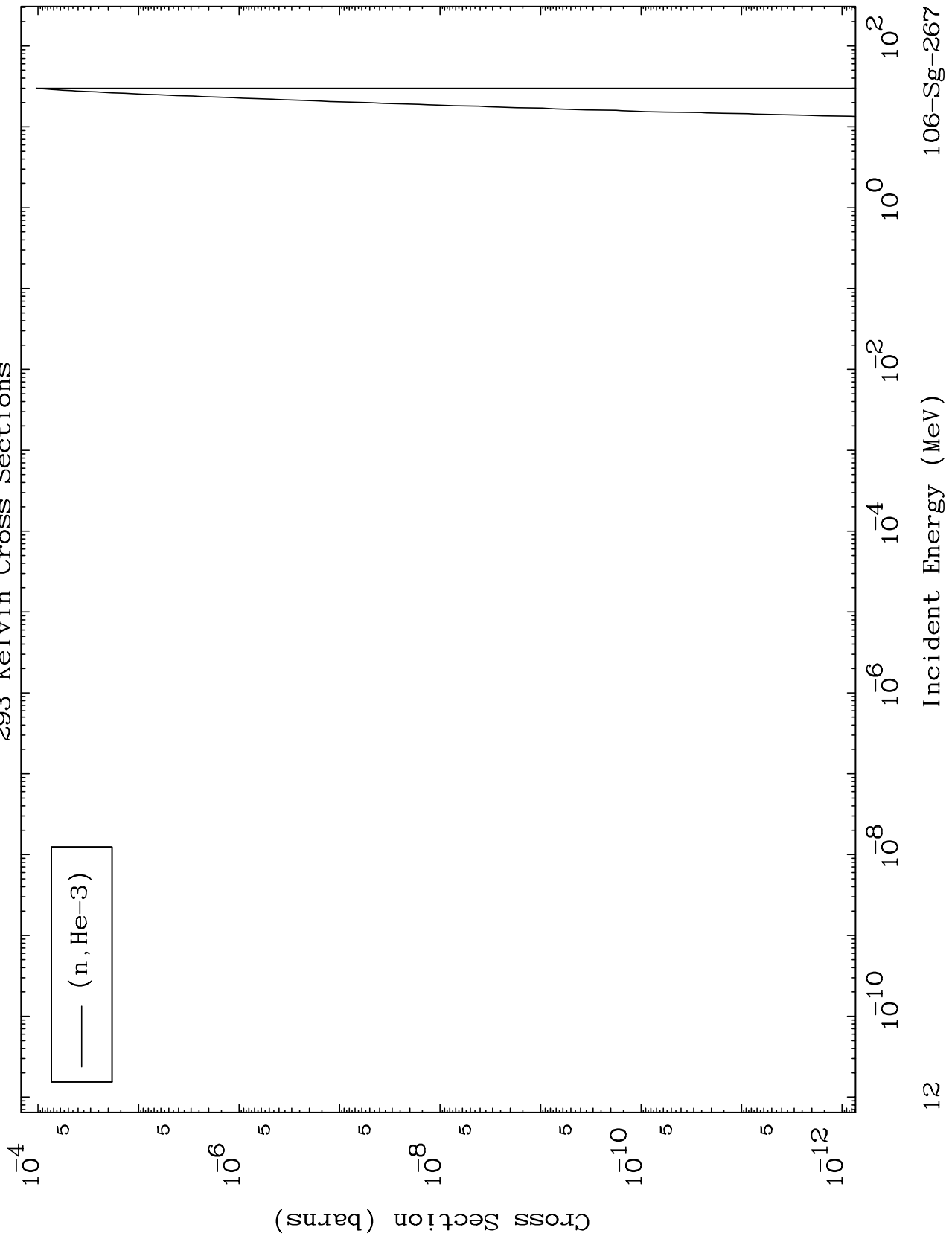
106-Sg-267



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(n,He3) Levels  
293 Kelvin Cross Sections

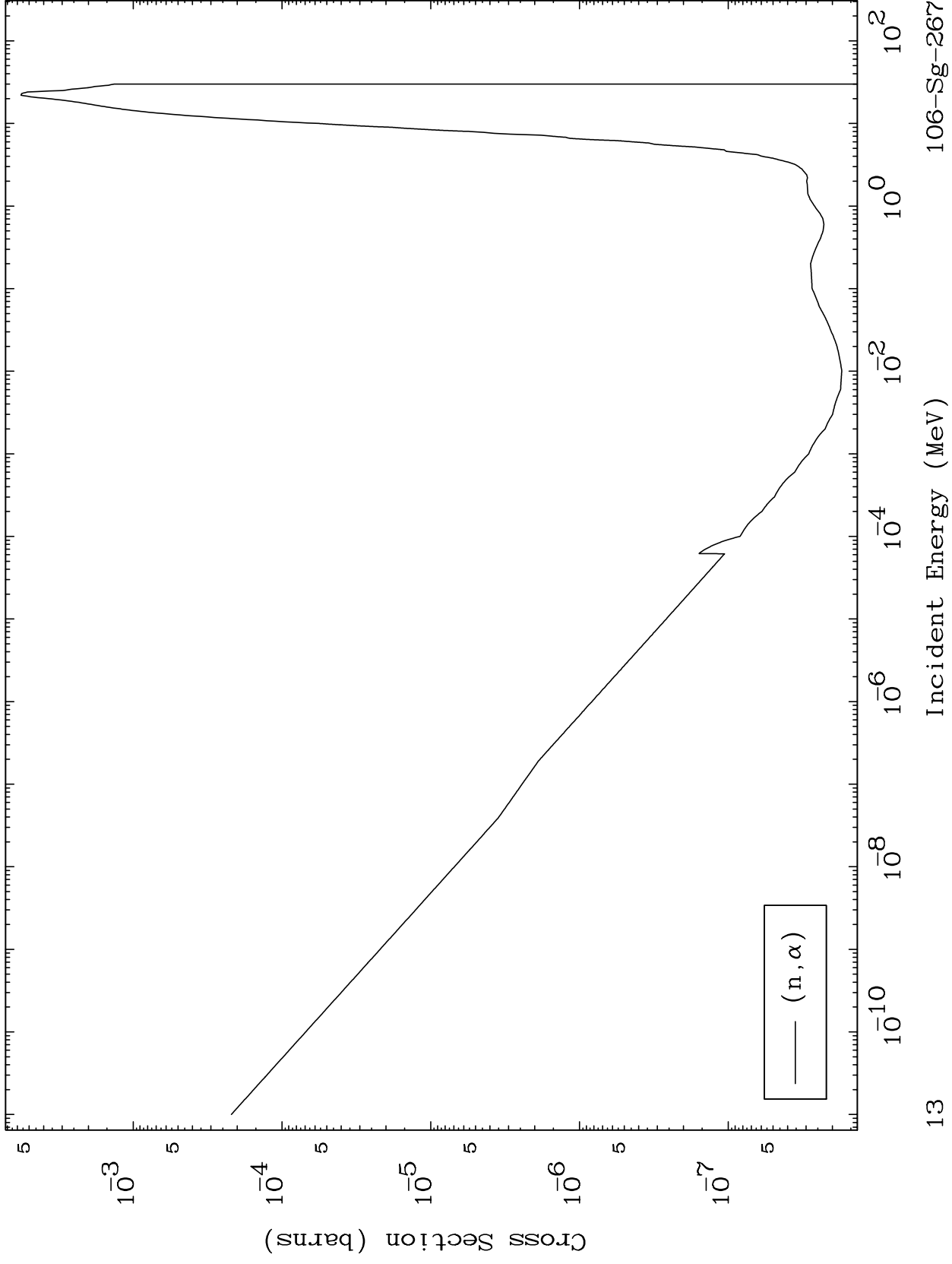
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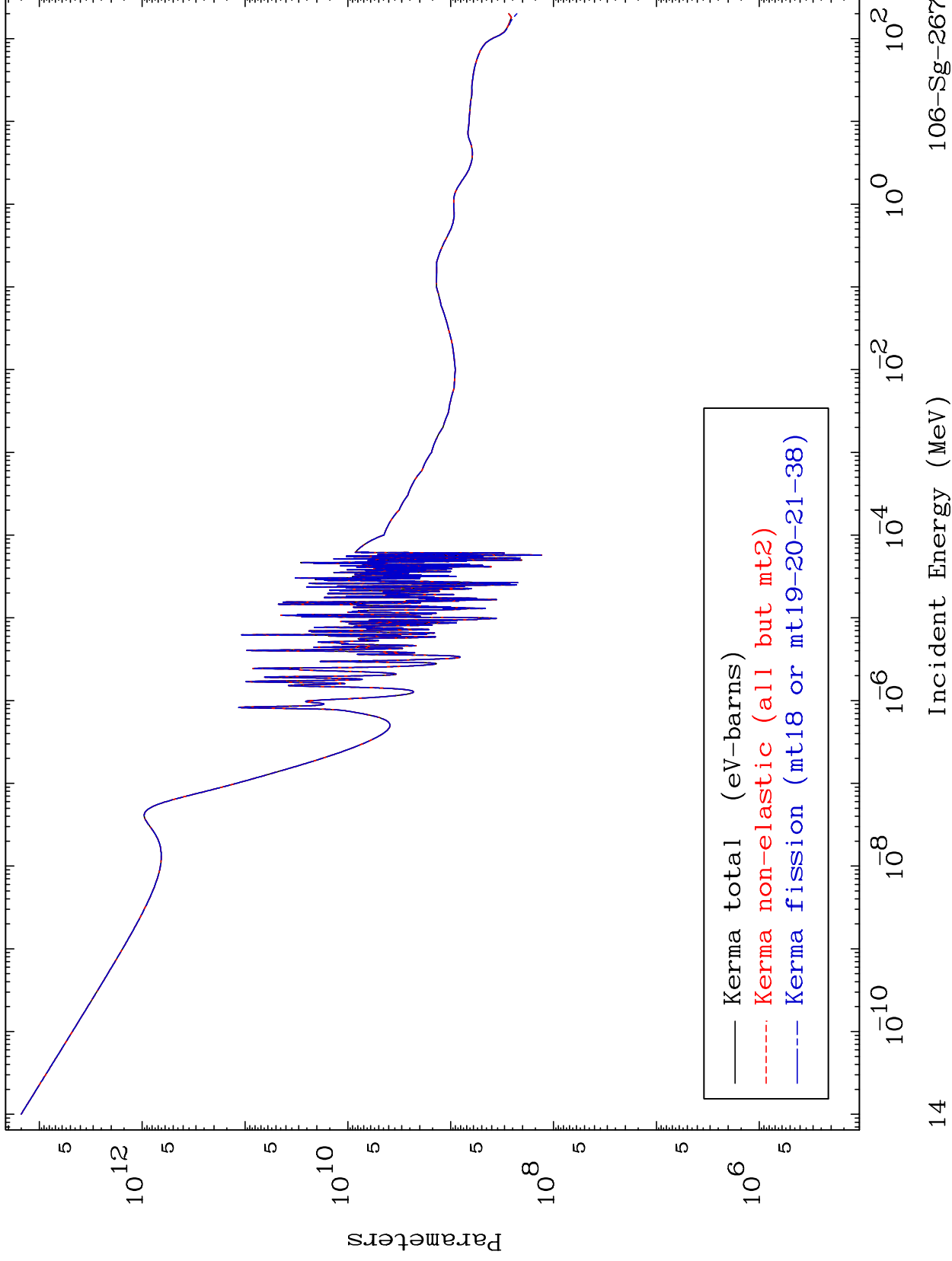
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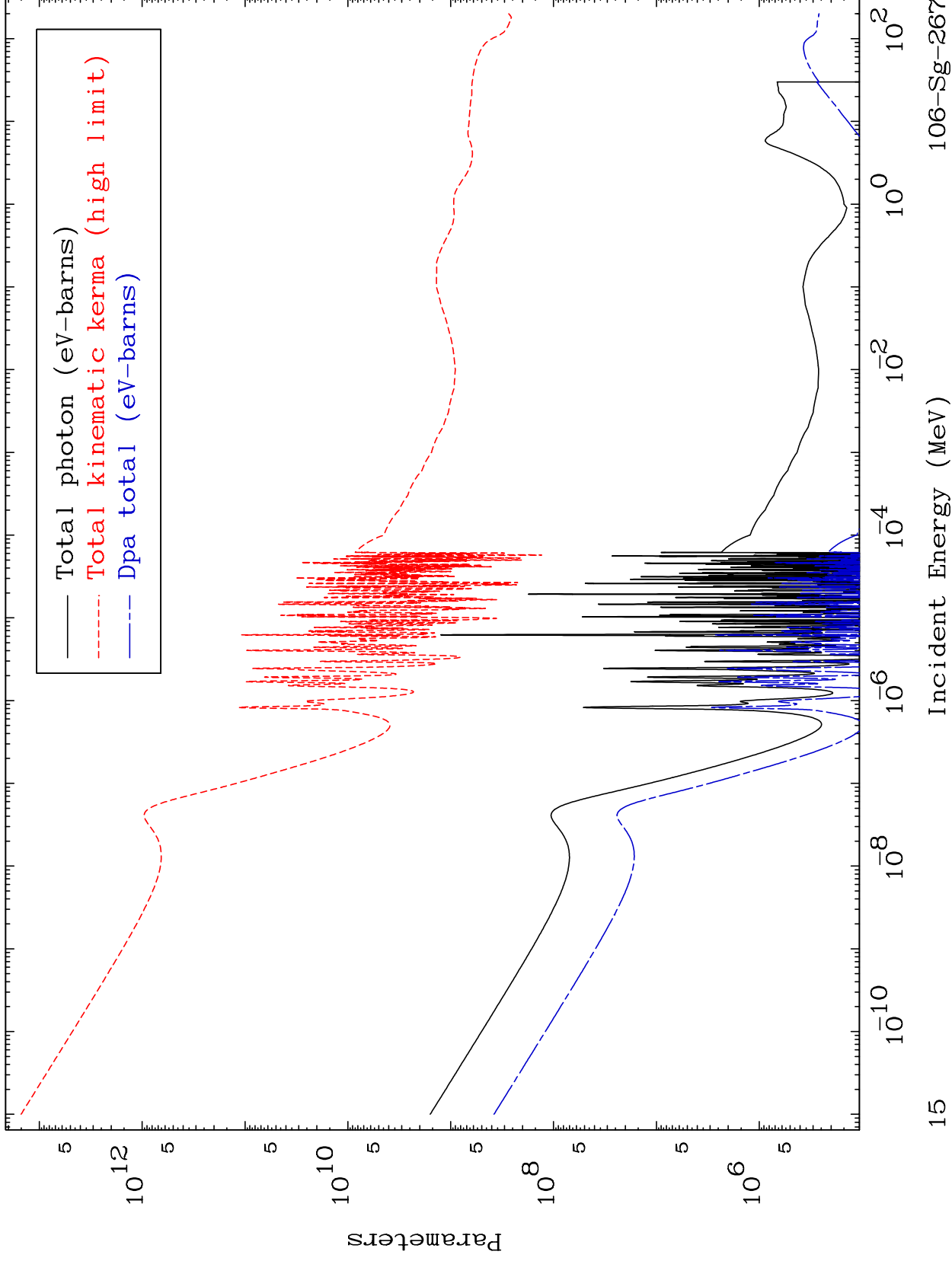
(n,α) Levels  
293 Kelvin Cross Sections

106-Sg-267



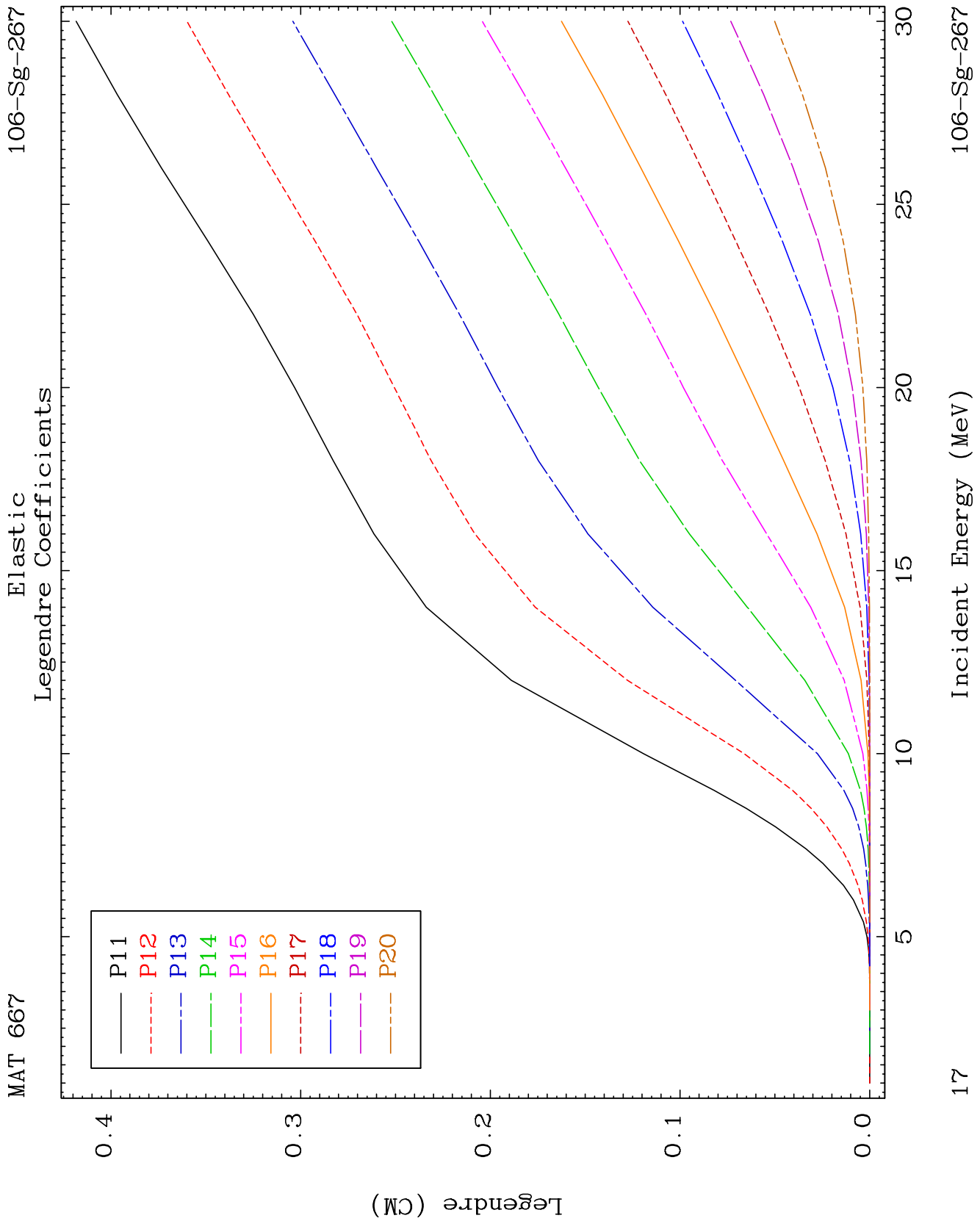
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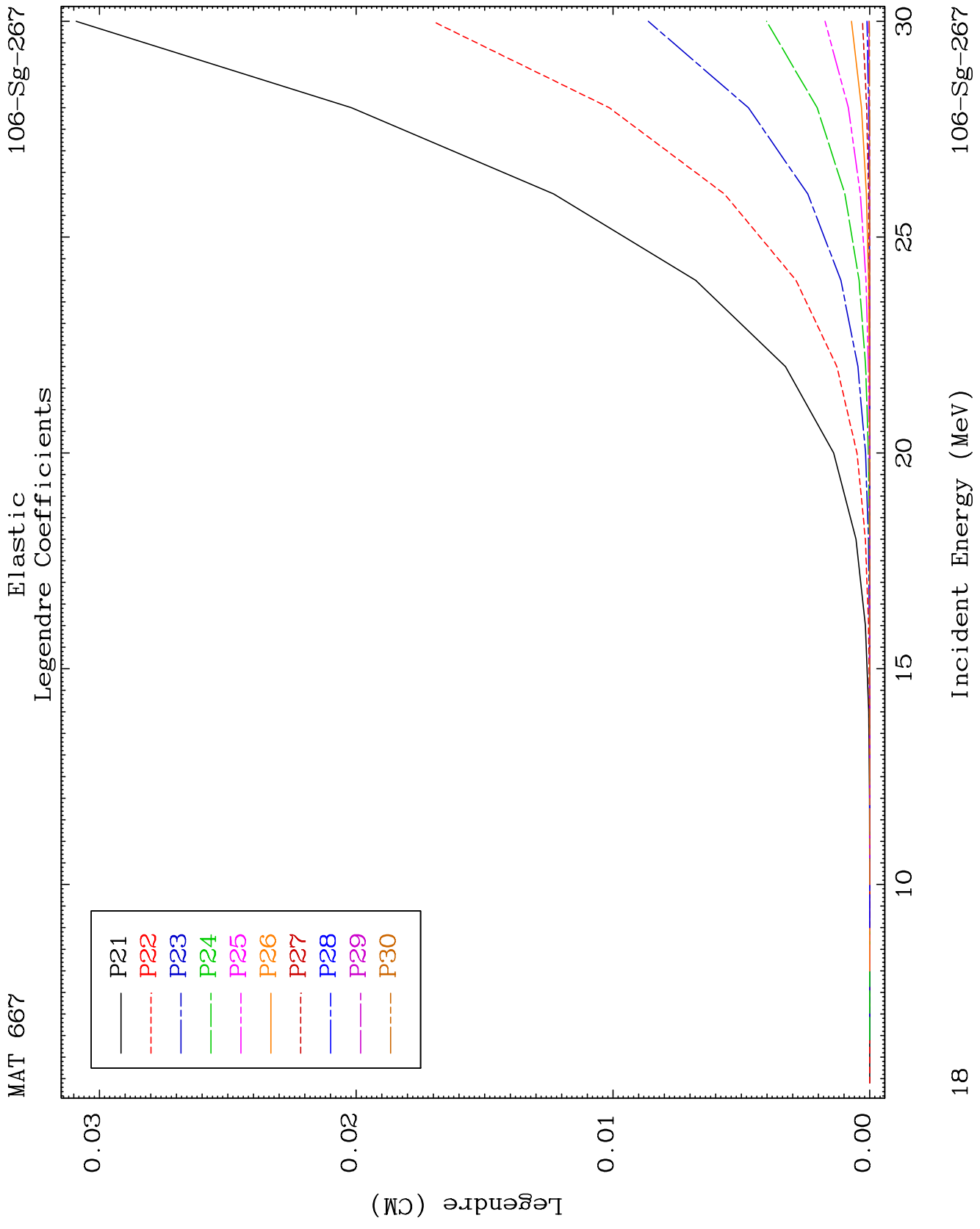










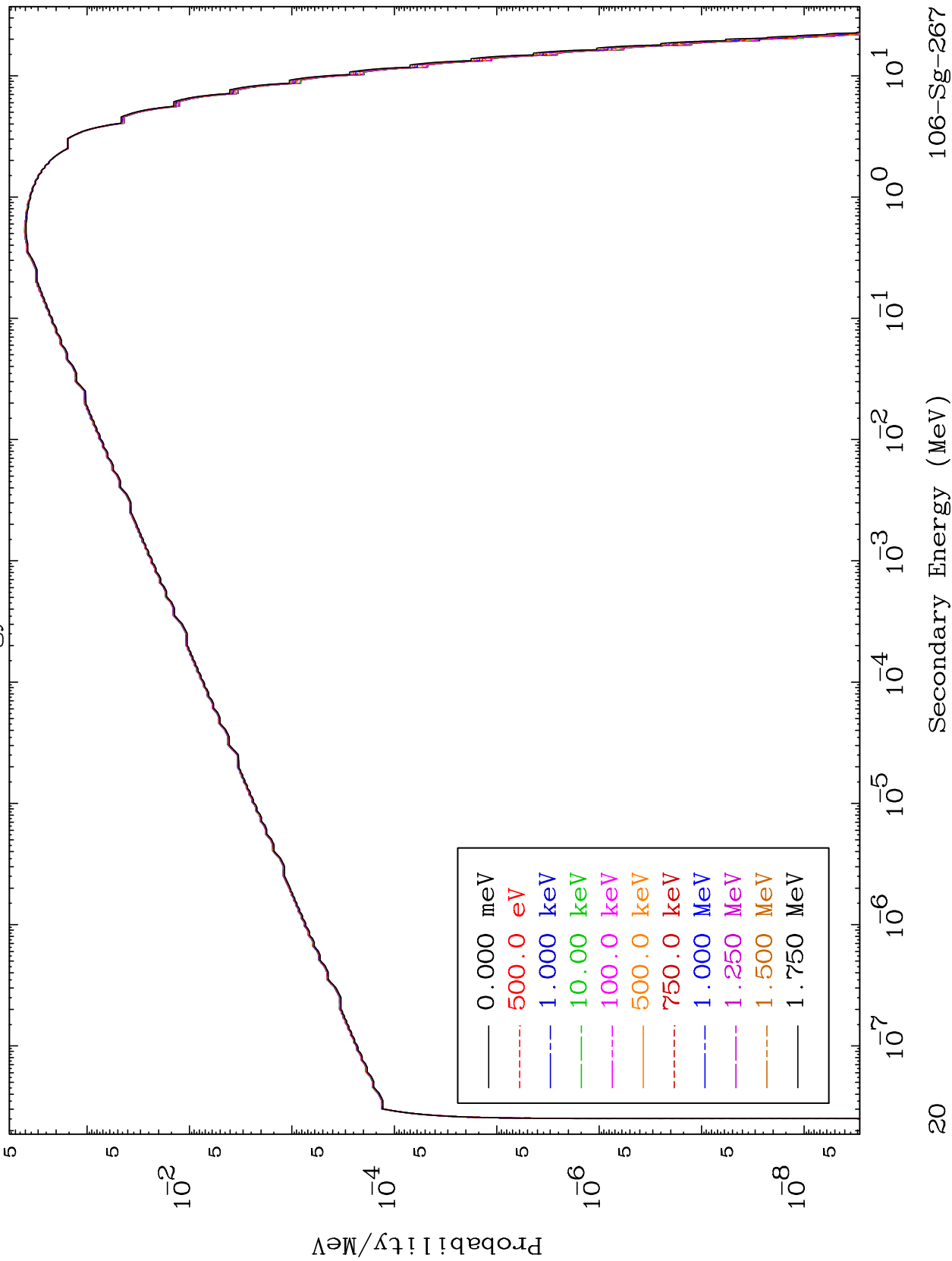




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Fission  
Energy Distribution

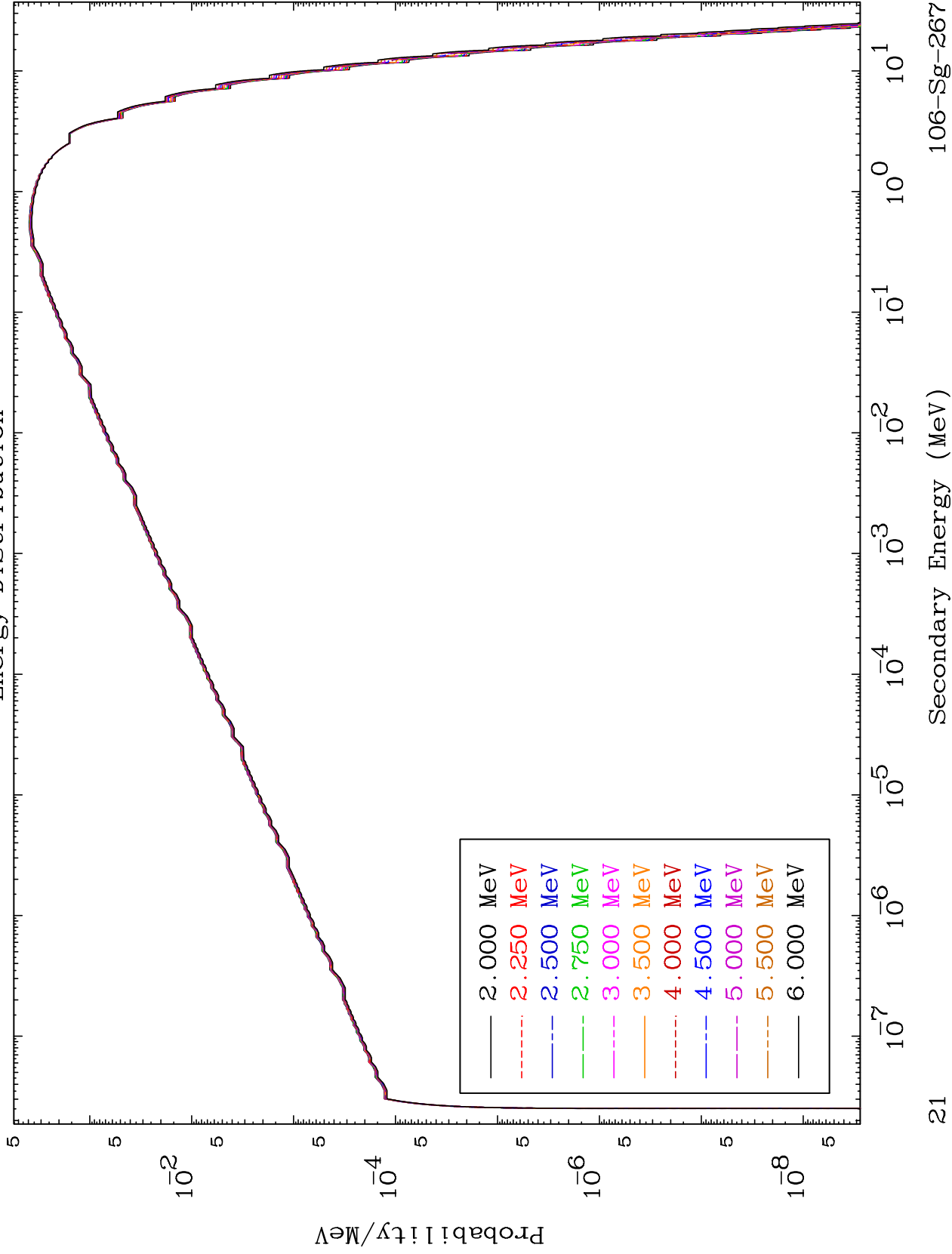
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# Fission Energy Distribution

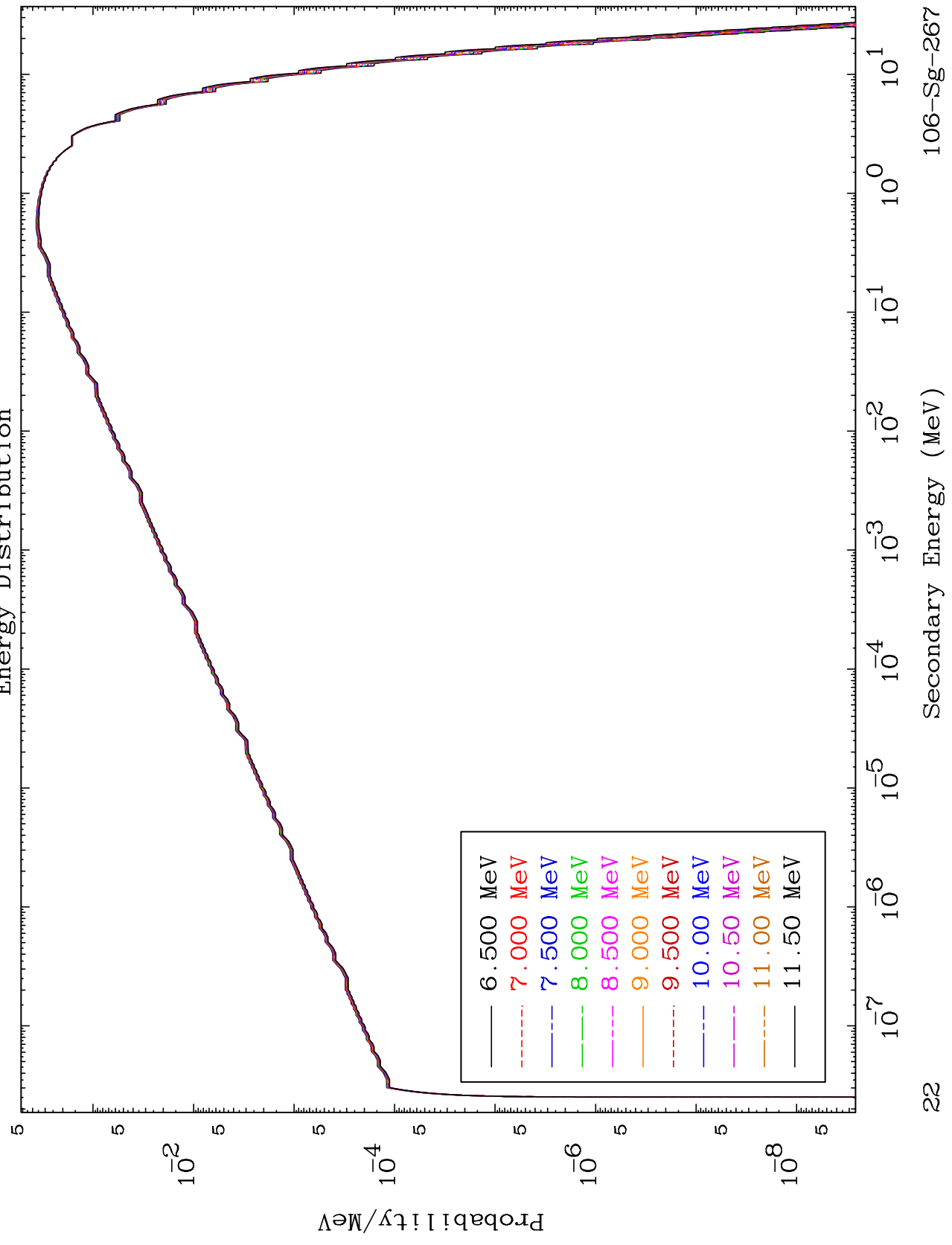
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### Fission Energy Distribution

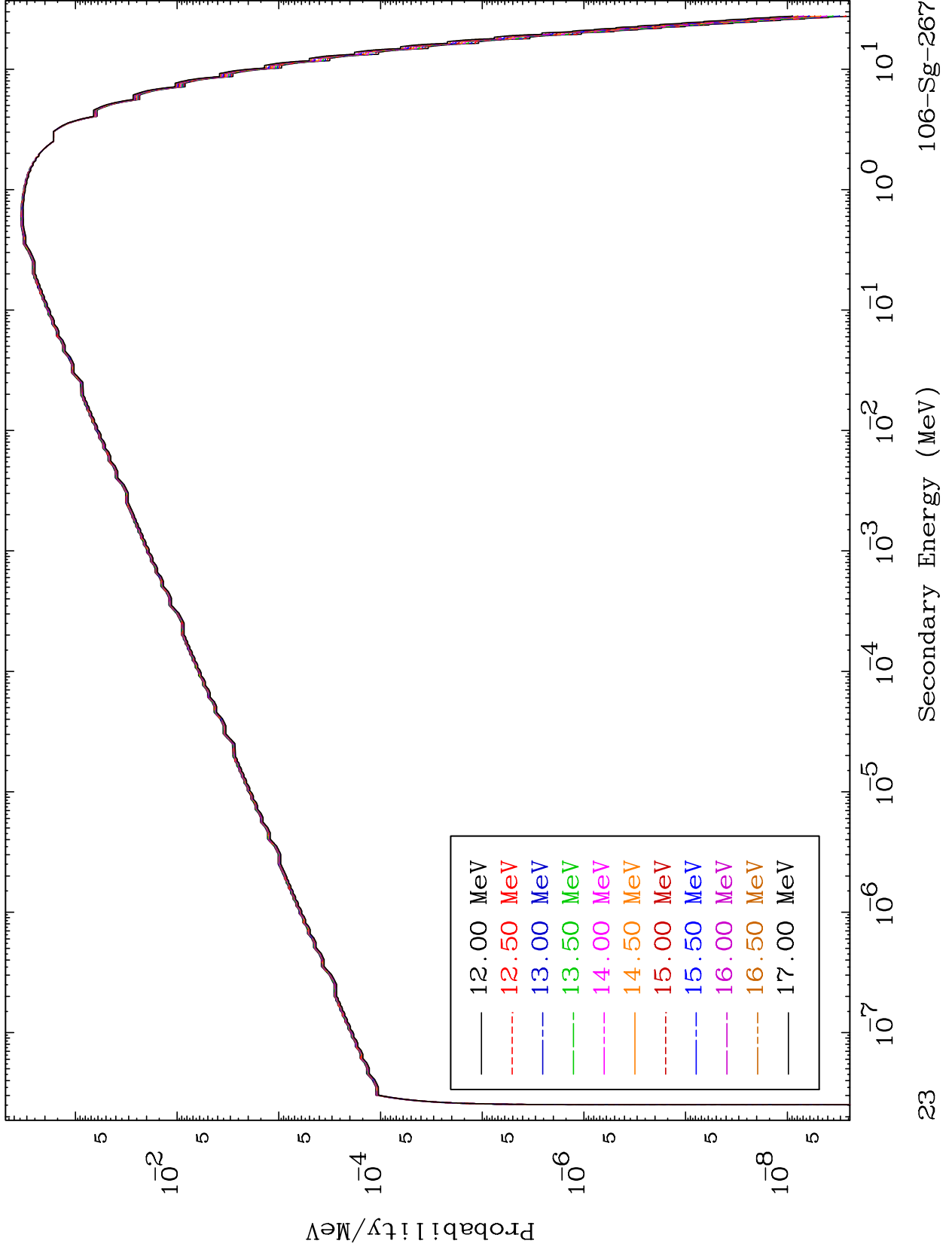
106-Sg-267



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### Fission Energy Distribution

106-Sg-267

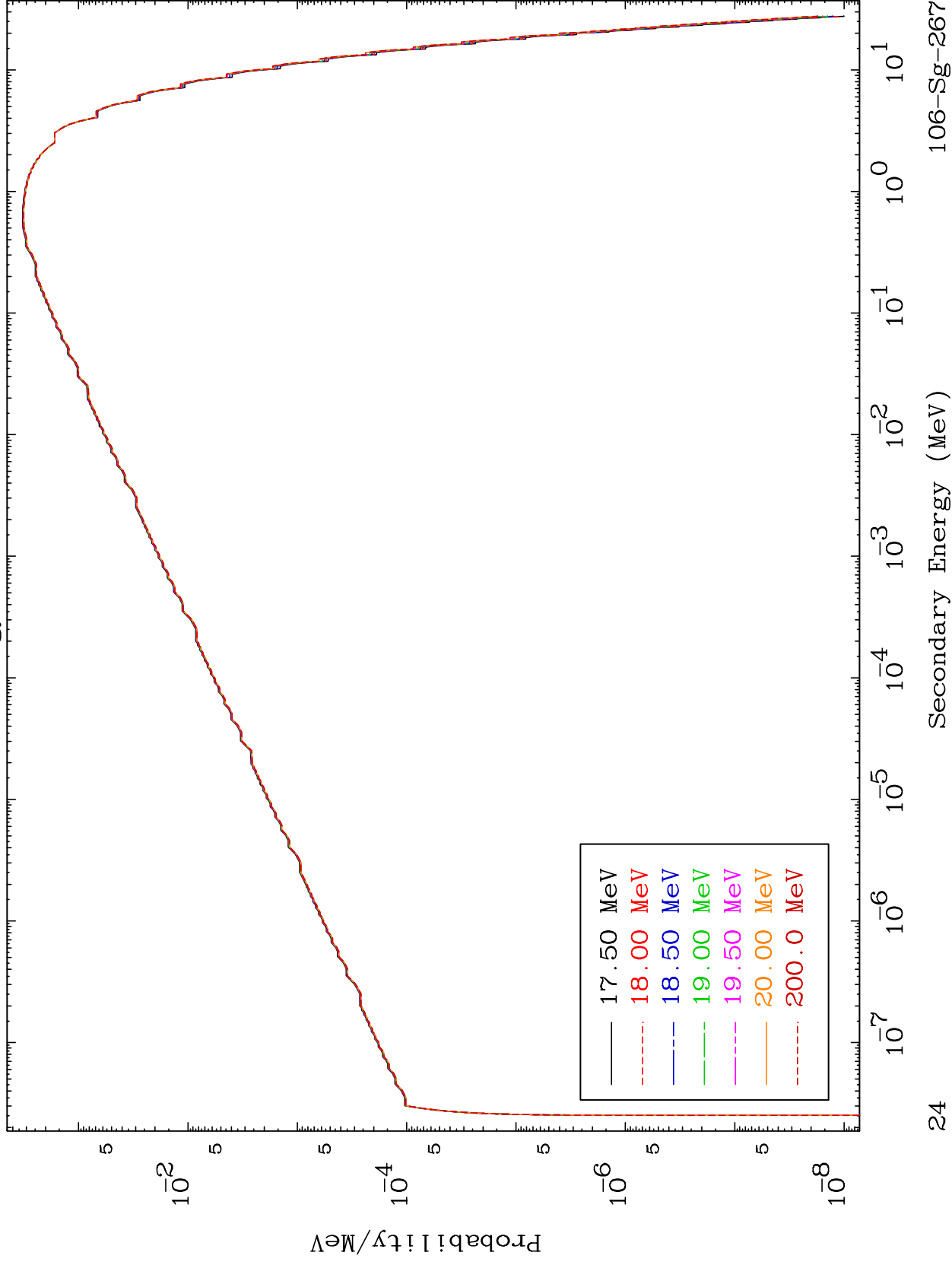


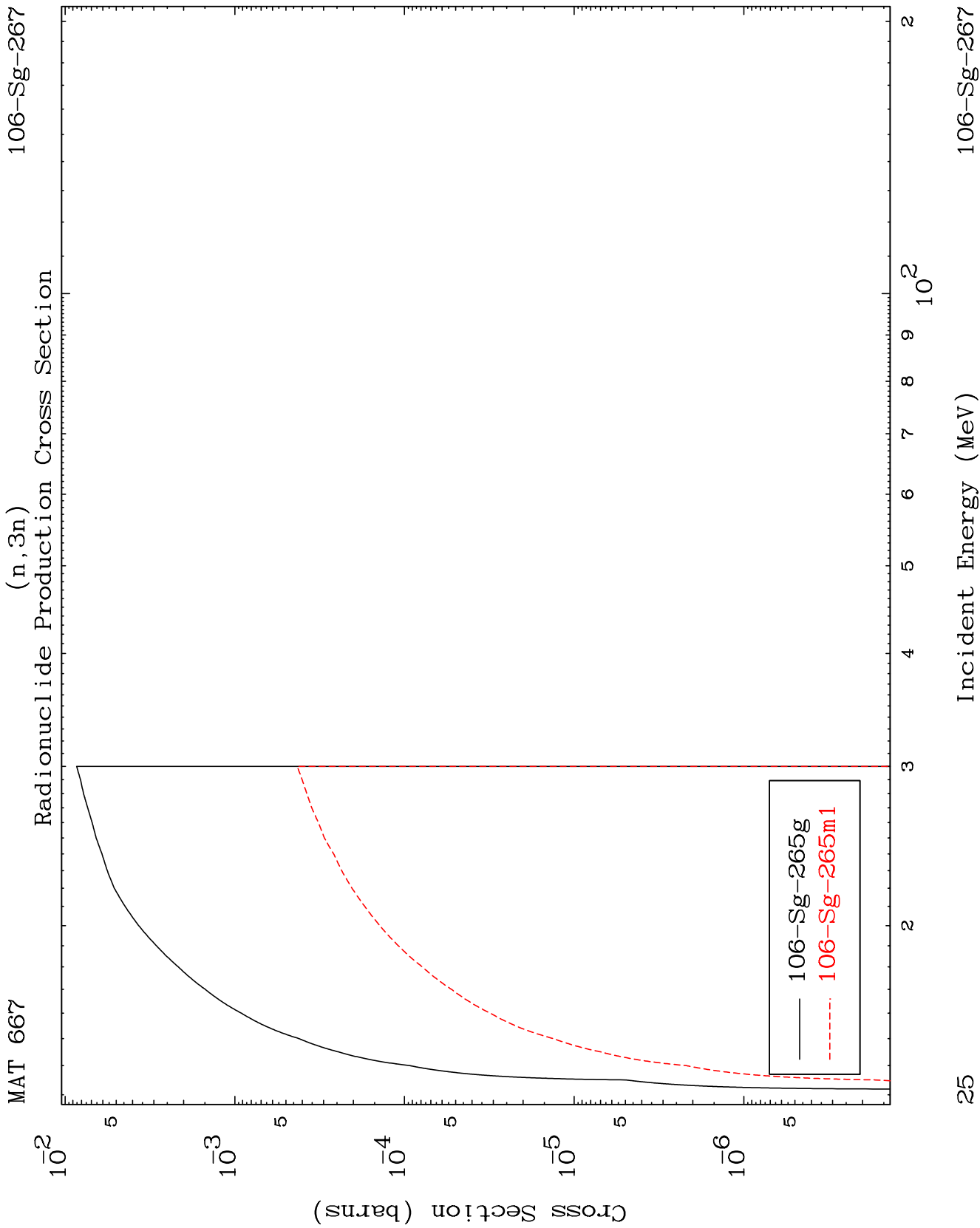


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### Fission Energy Distribution

106-Sg-267



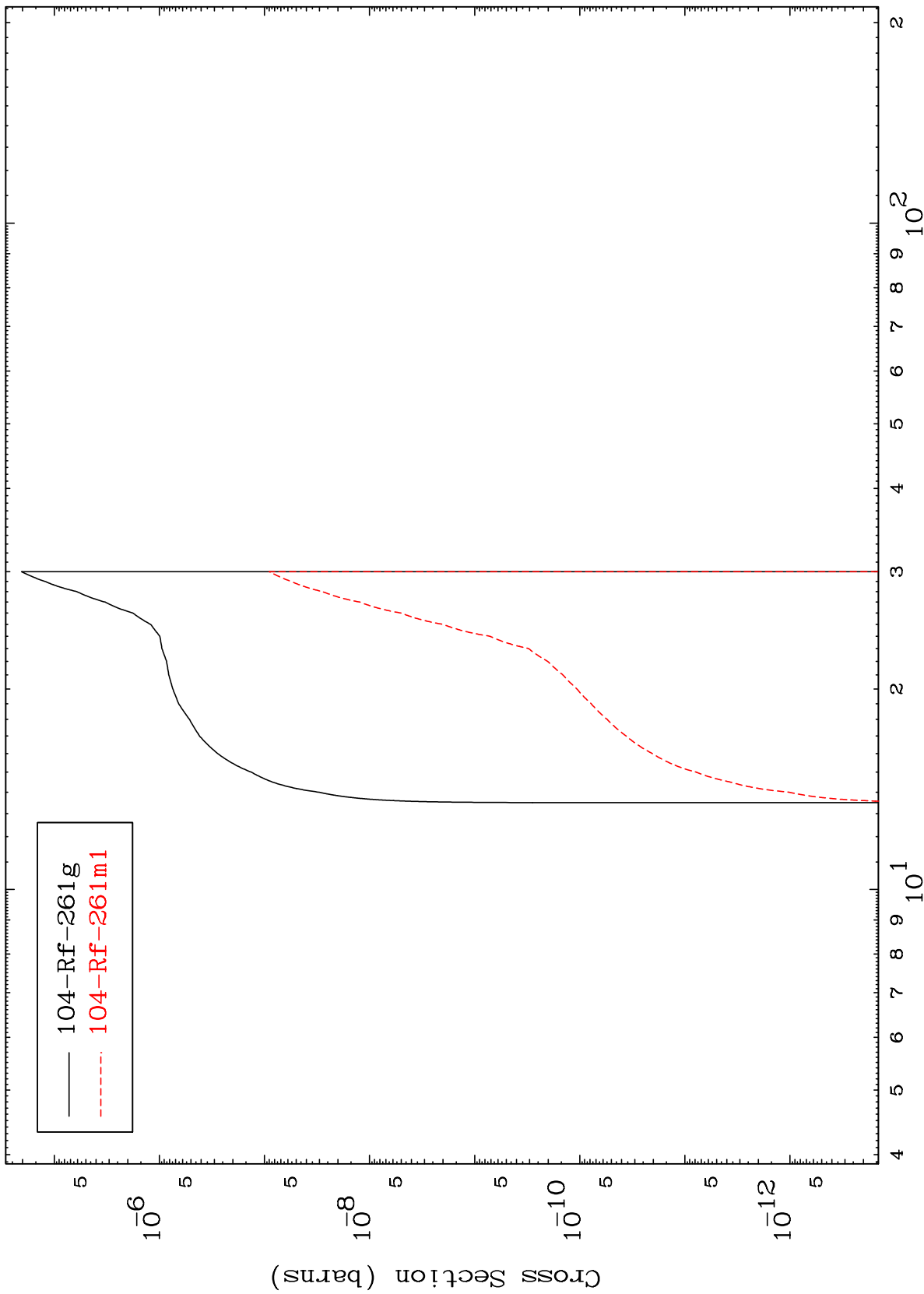


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(n,3n)  $\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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