

Program EVALPLOT  
(Version 2017-1)

by

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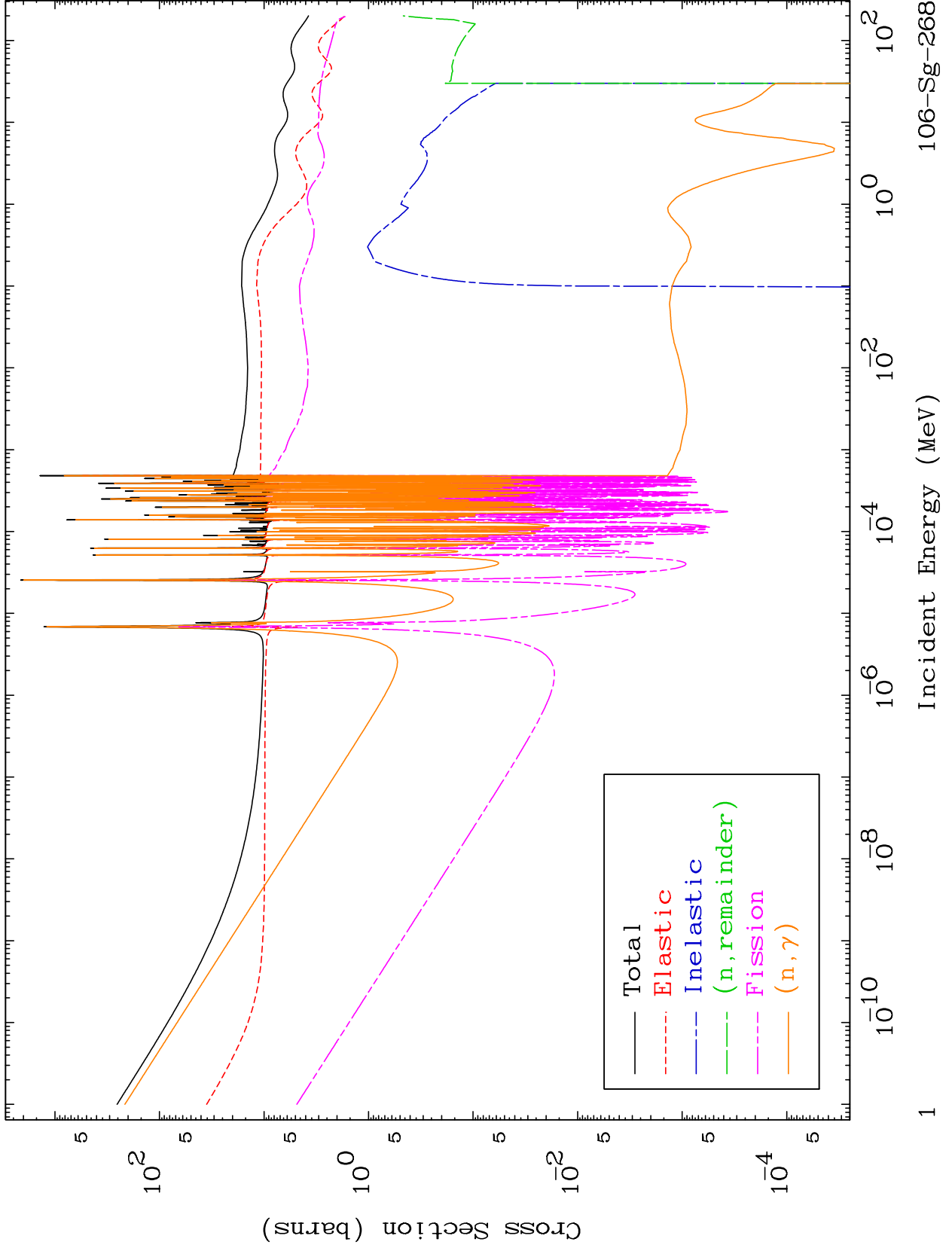
Press Mouse Button to Start

MAT 668

Major

293 Kelvin Cross Sections

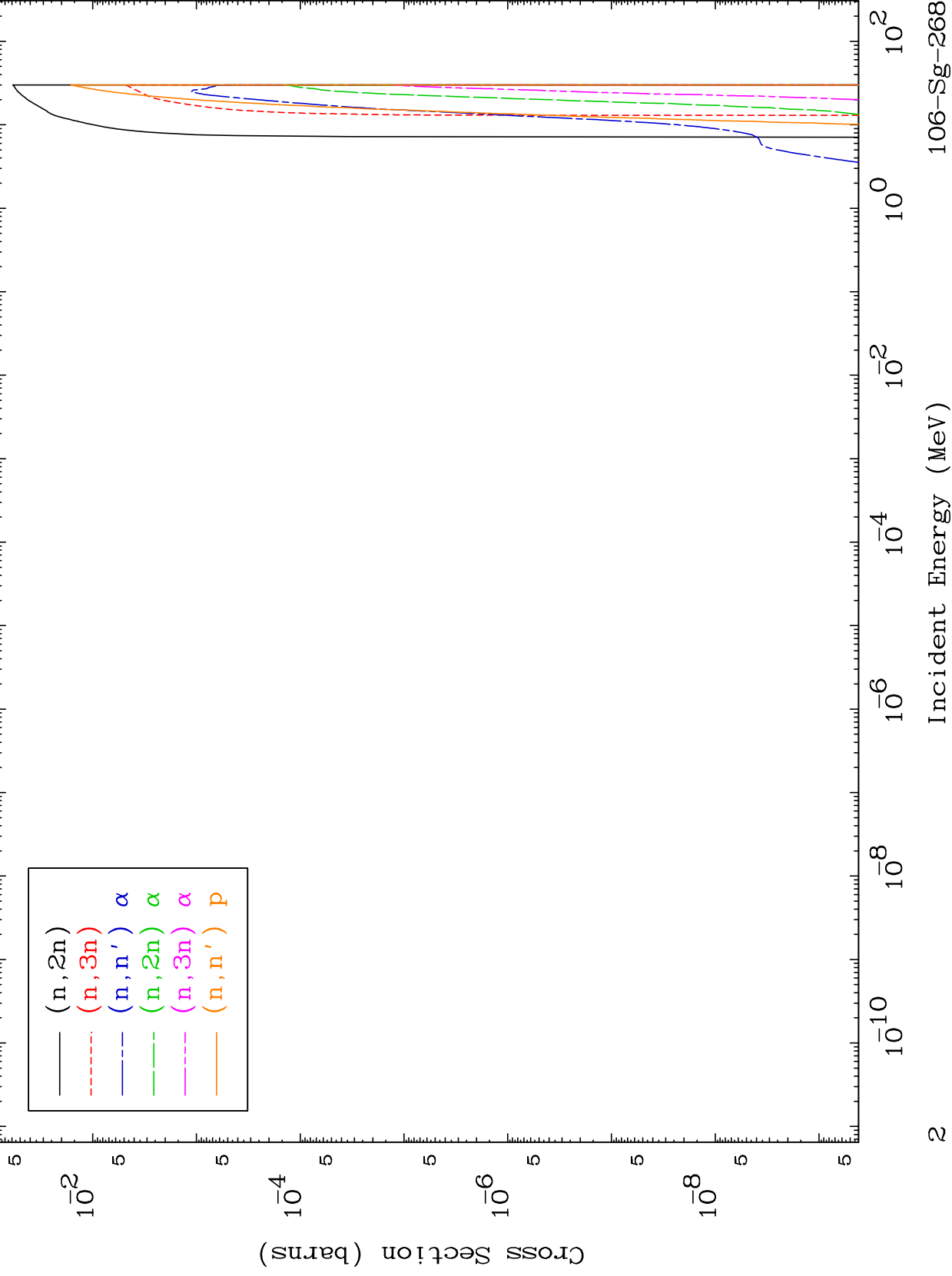
106-Sg-268



MAT 668

Neutron Production  
293 Kelvin Cross Sections

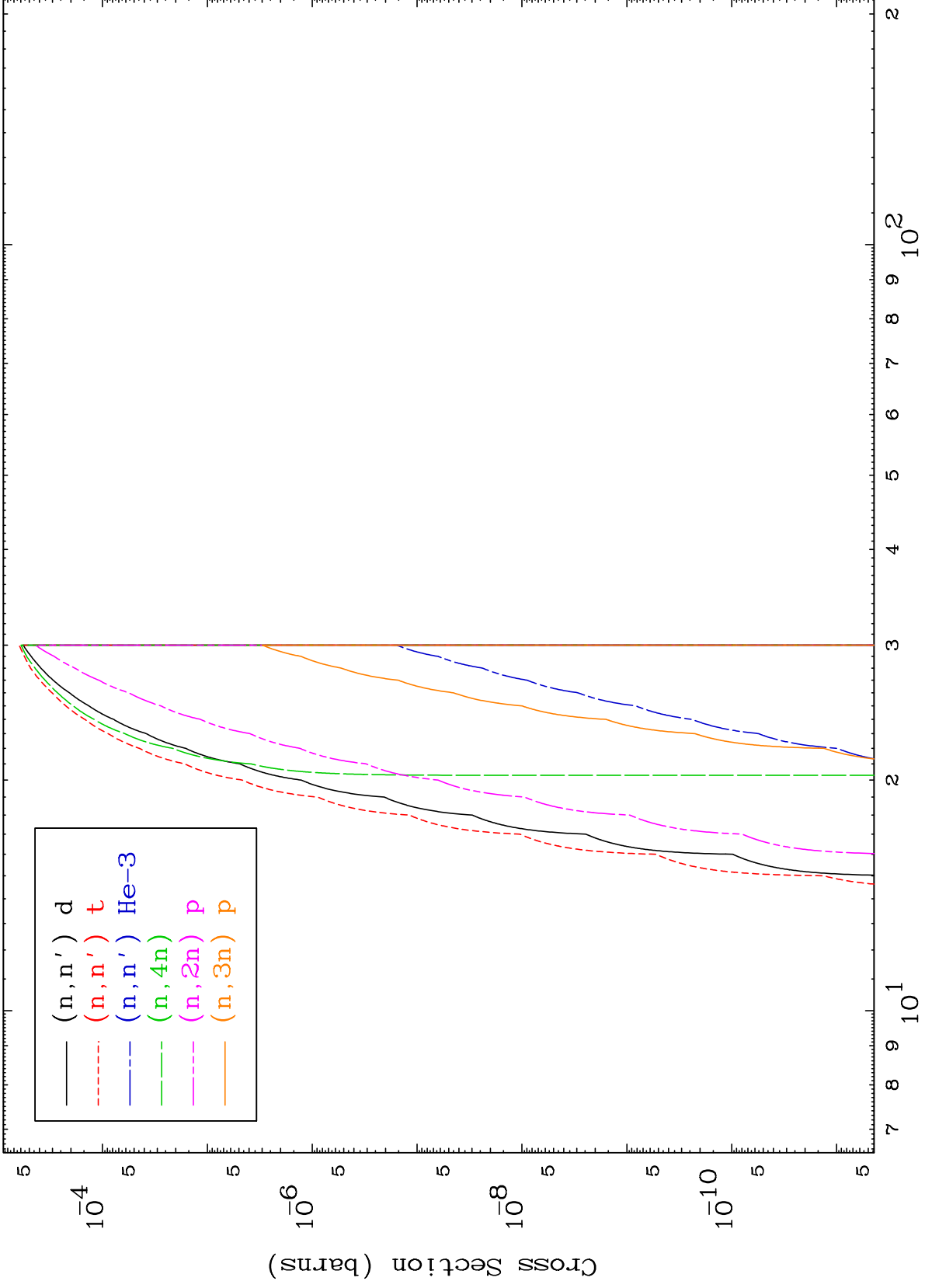
106-Sg-268



MAT 668

Neutron Production  
293 Kelvin Cross Sections

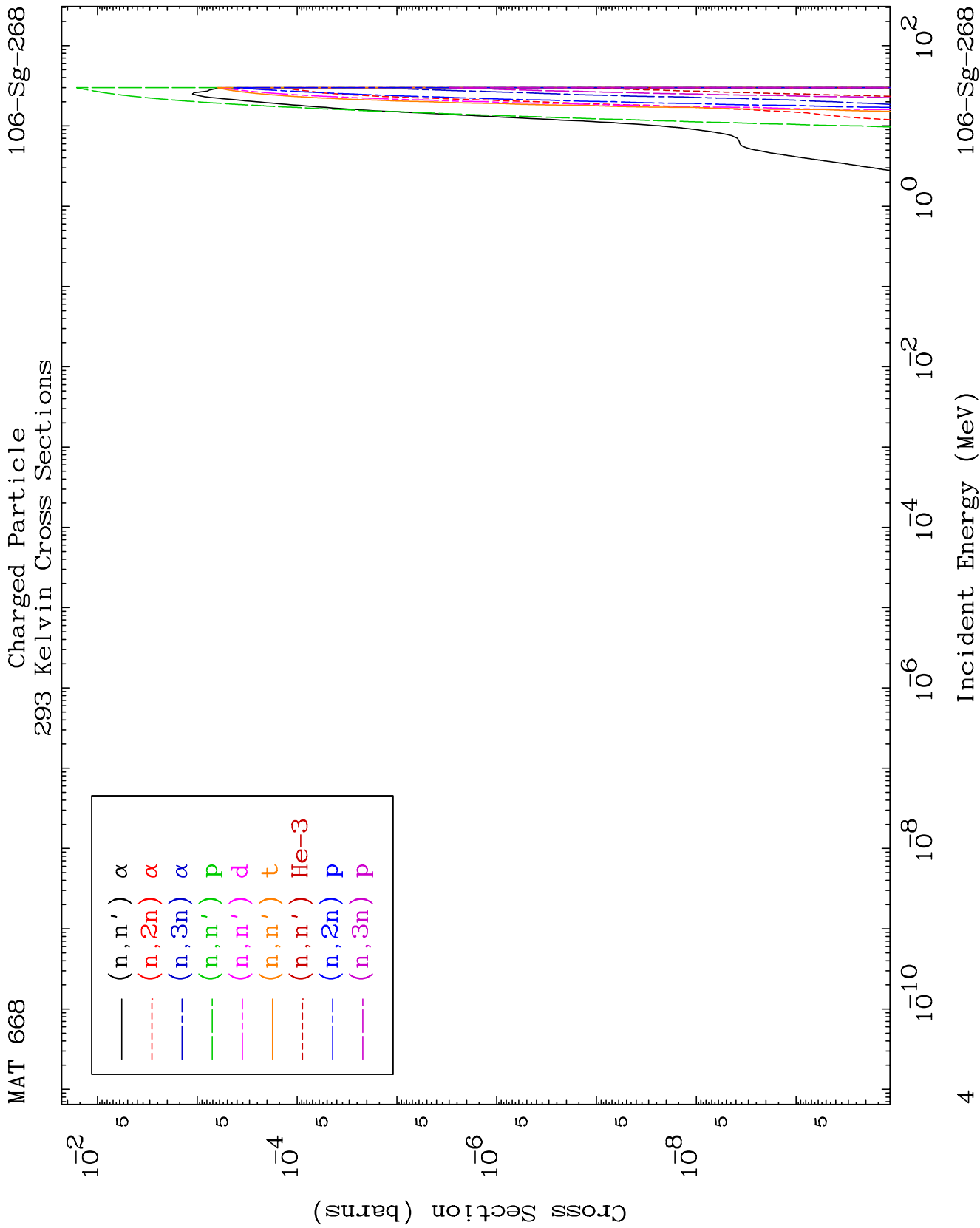
106-Sg-268

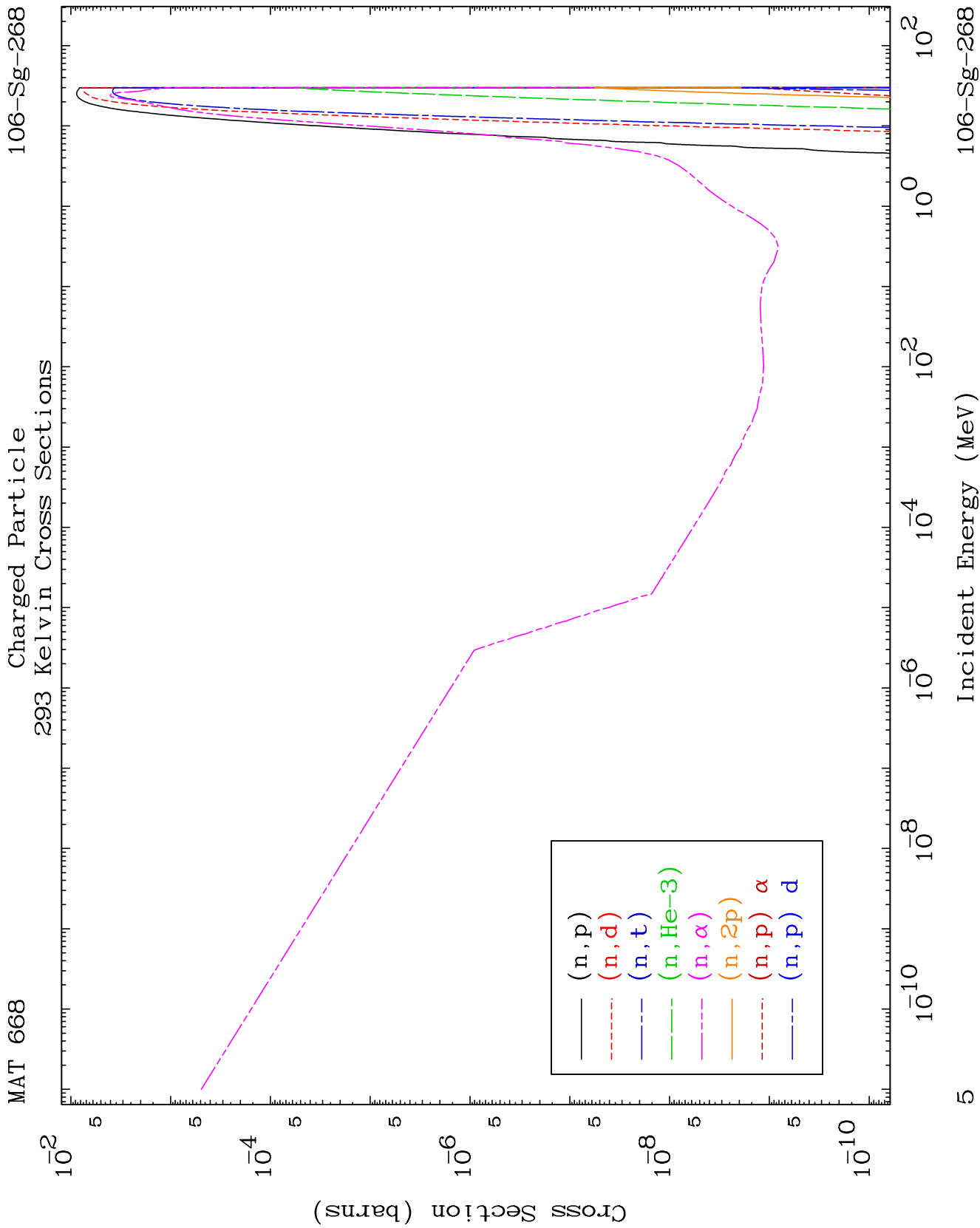


3

Incident Energy (MeV)

106-Sg-268

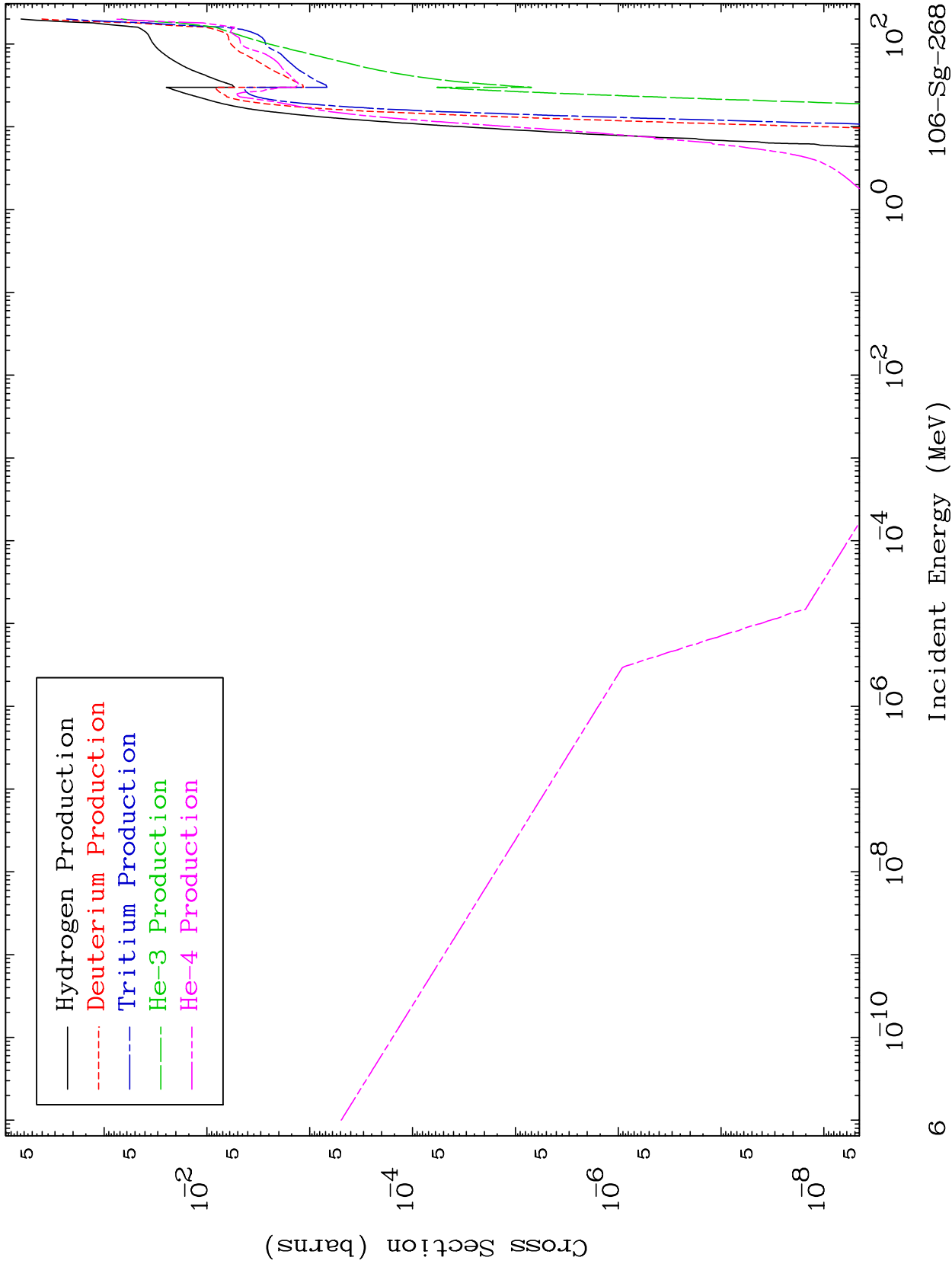




MAT 668

Particle Production  
293 Kelvin Cross Sections

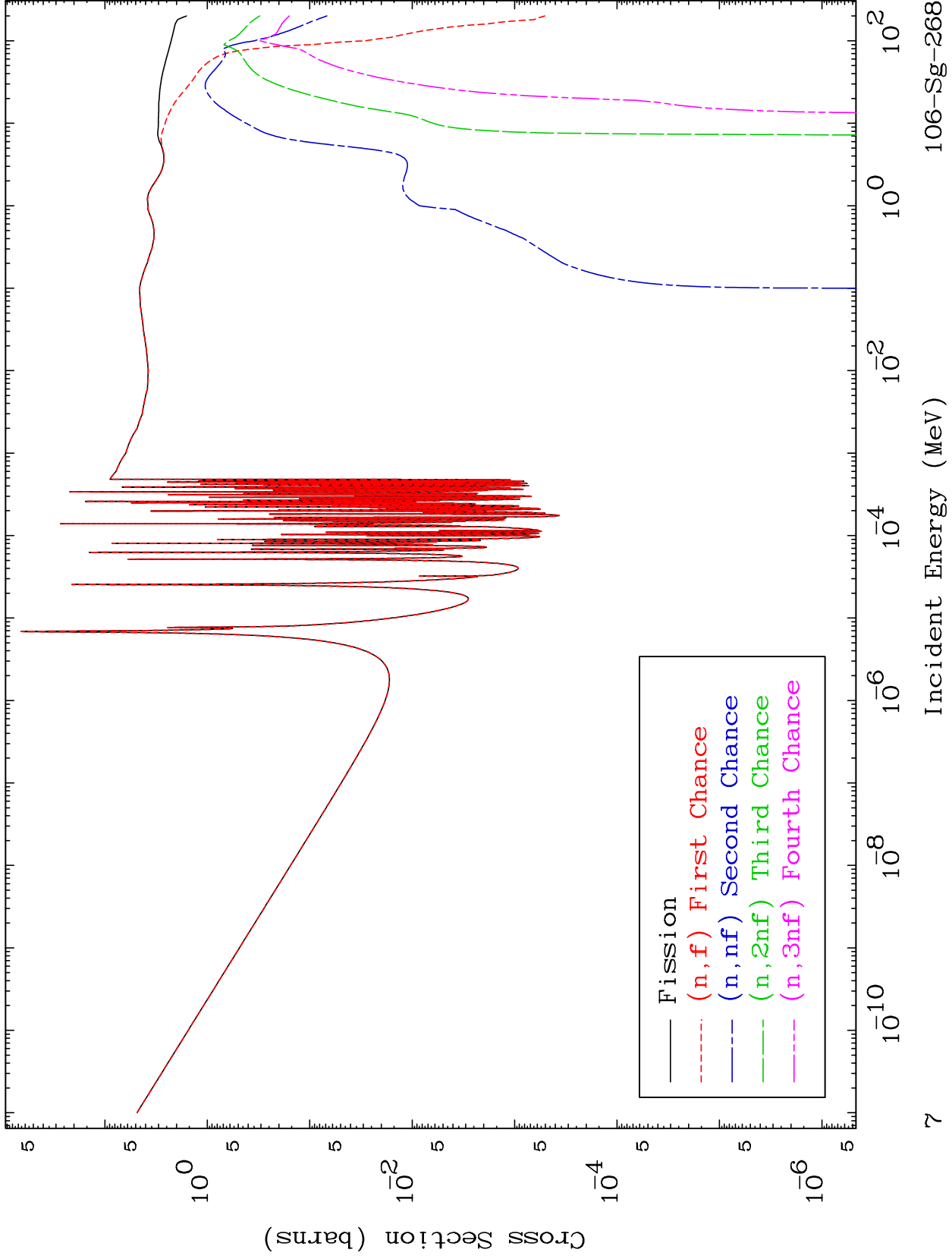
106-Sg-268



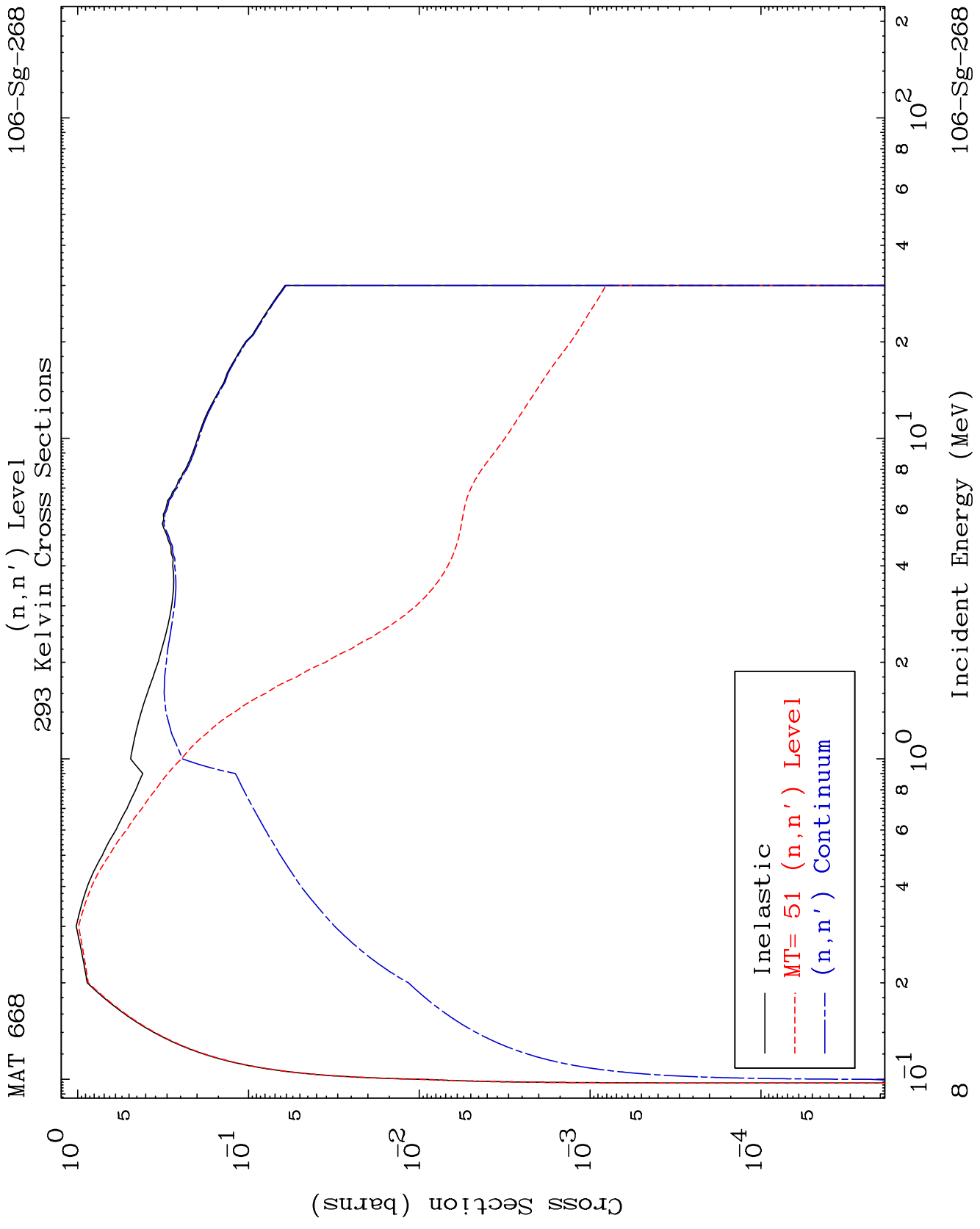
MAT 668

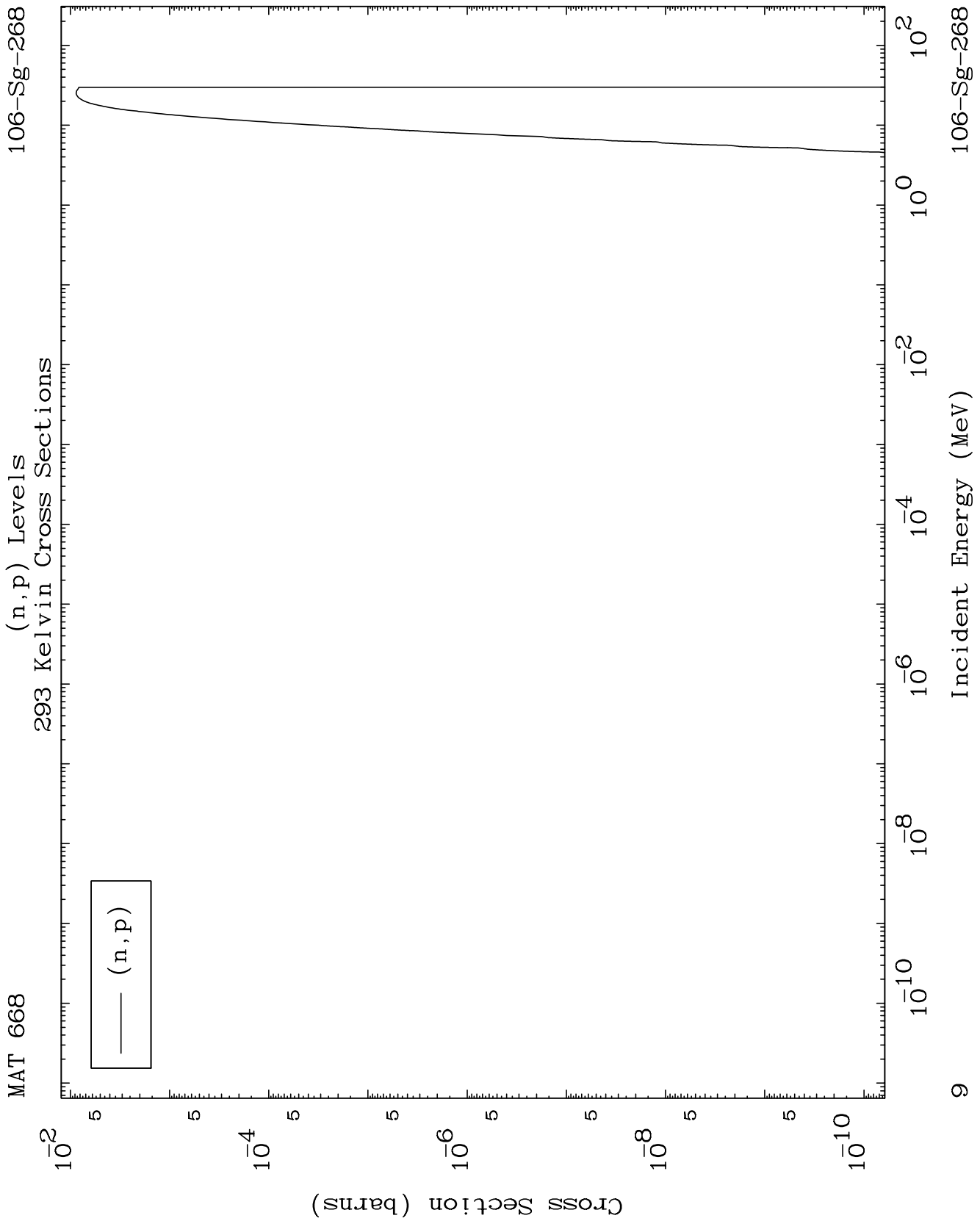
Fission  
293 Kelvin Cross Sections

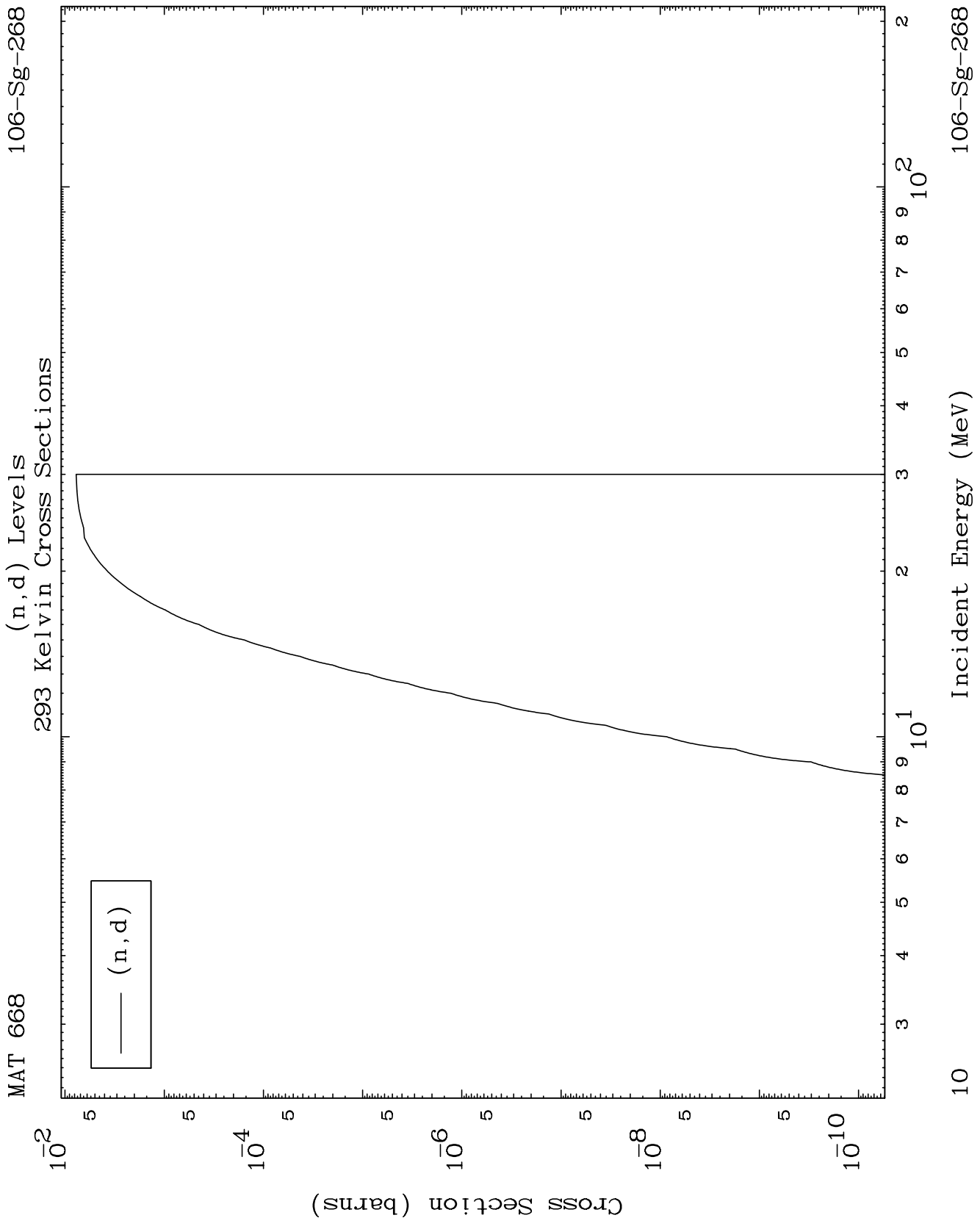
106-Sg-268







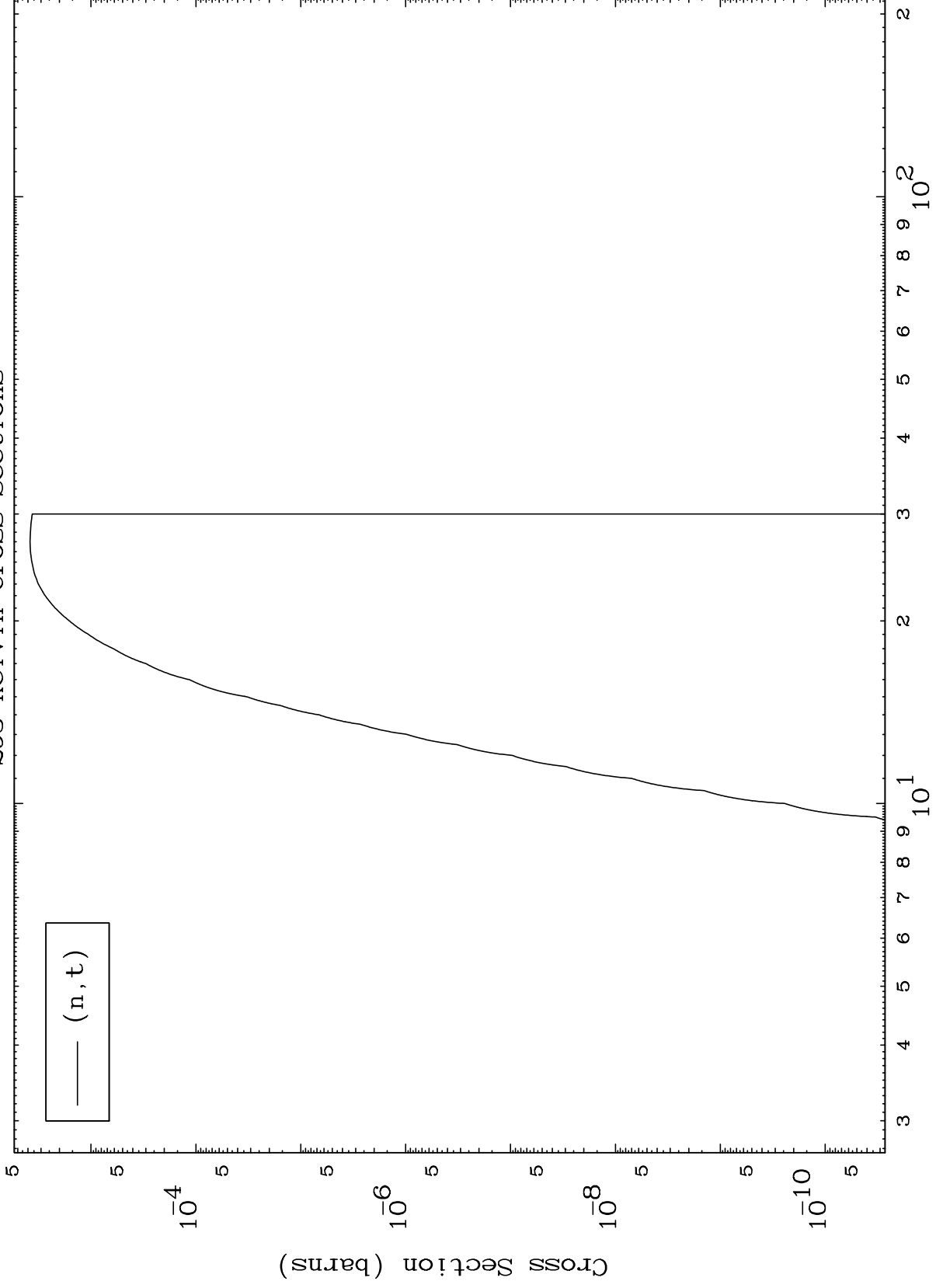




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(n,t) Levels  
293 Kelvin Cross Sections

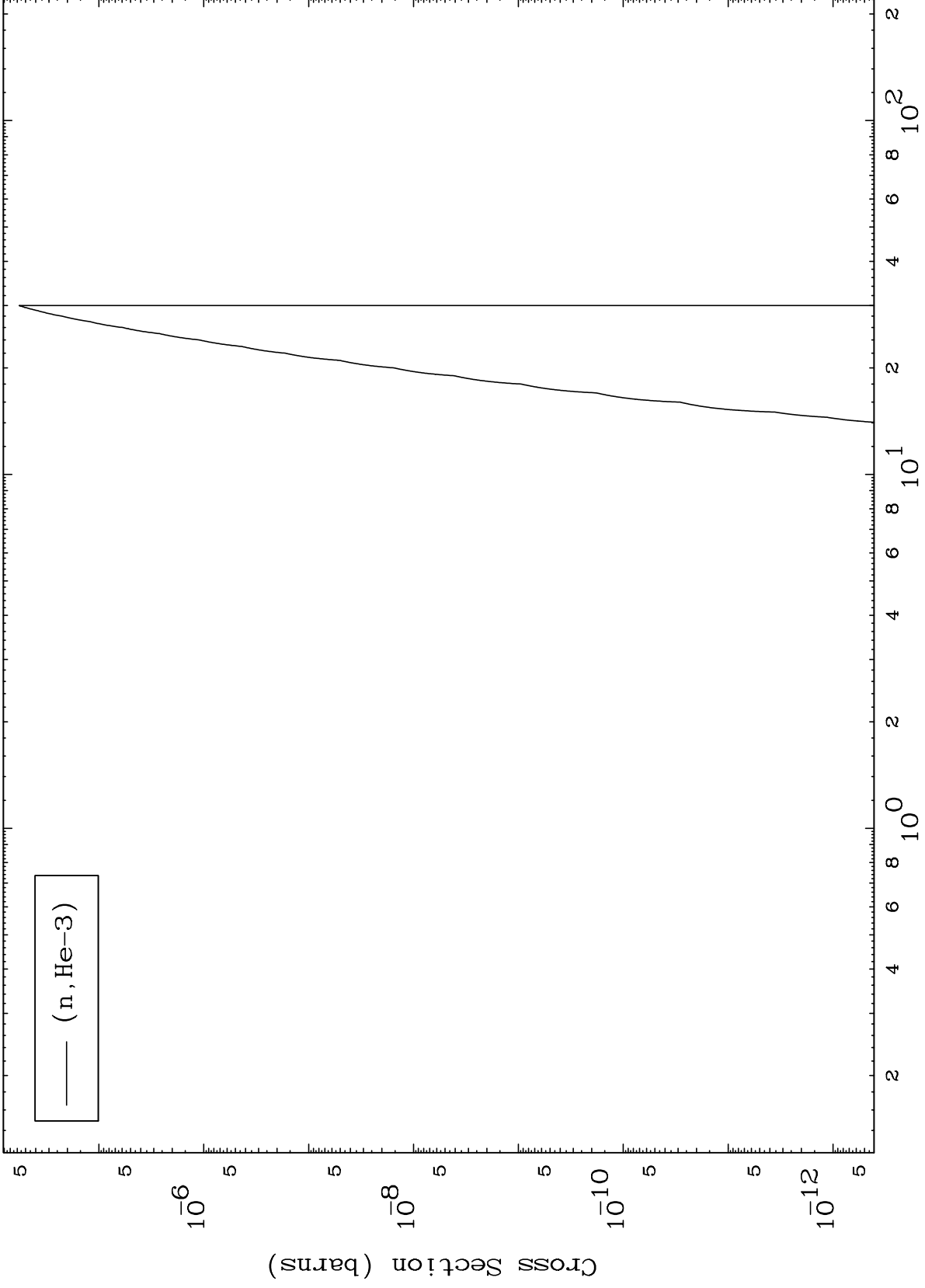
106-Sg-268



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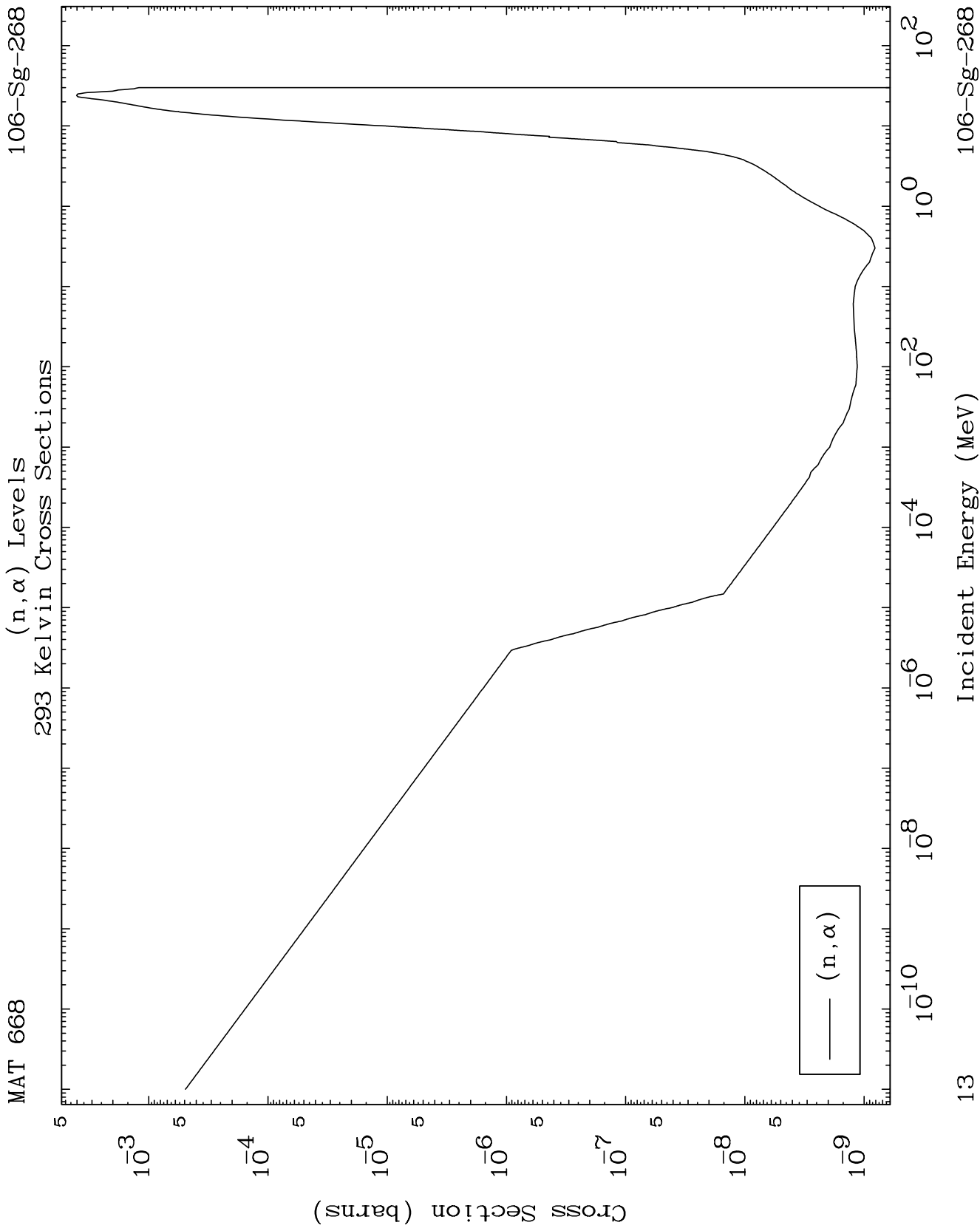
(n,He3) Levels  
293 Kelvin Cross Sections

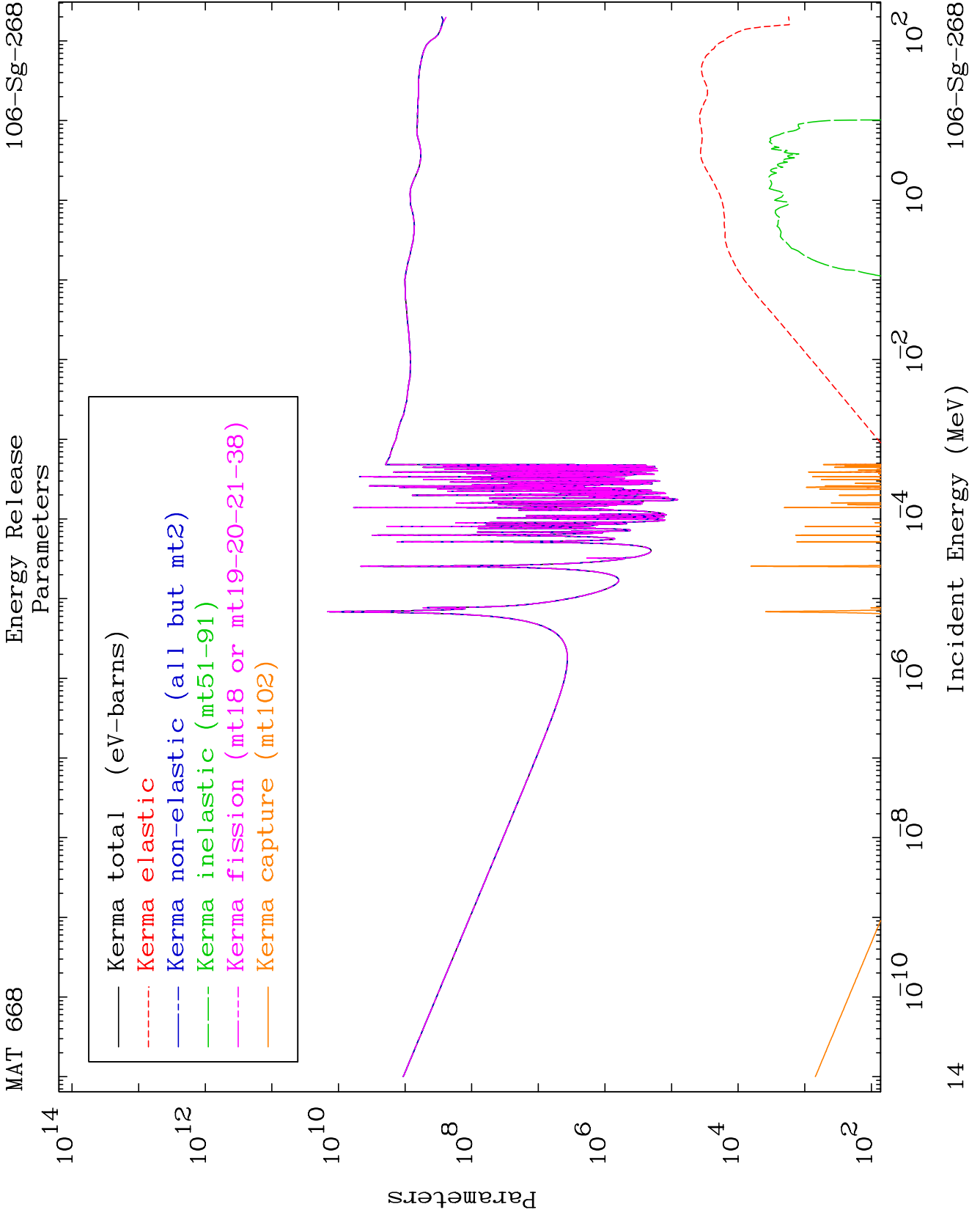
106-Sg-268

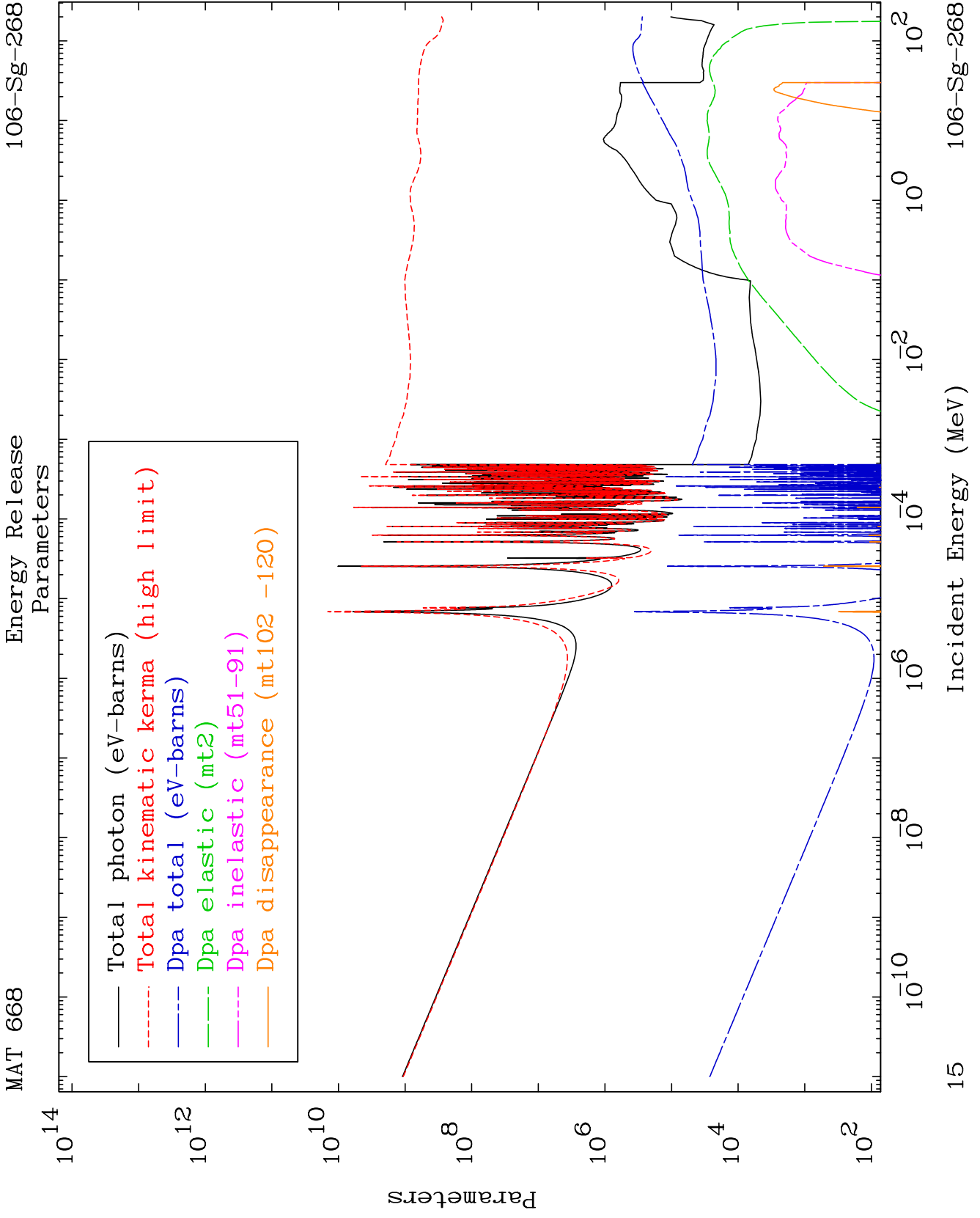


12

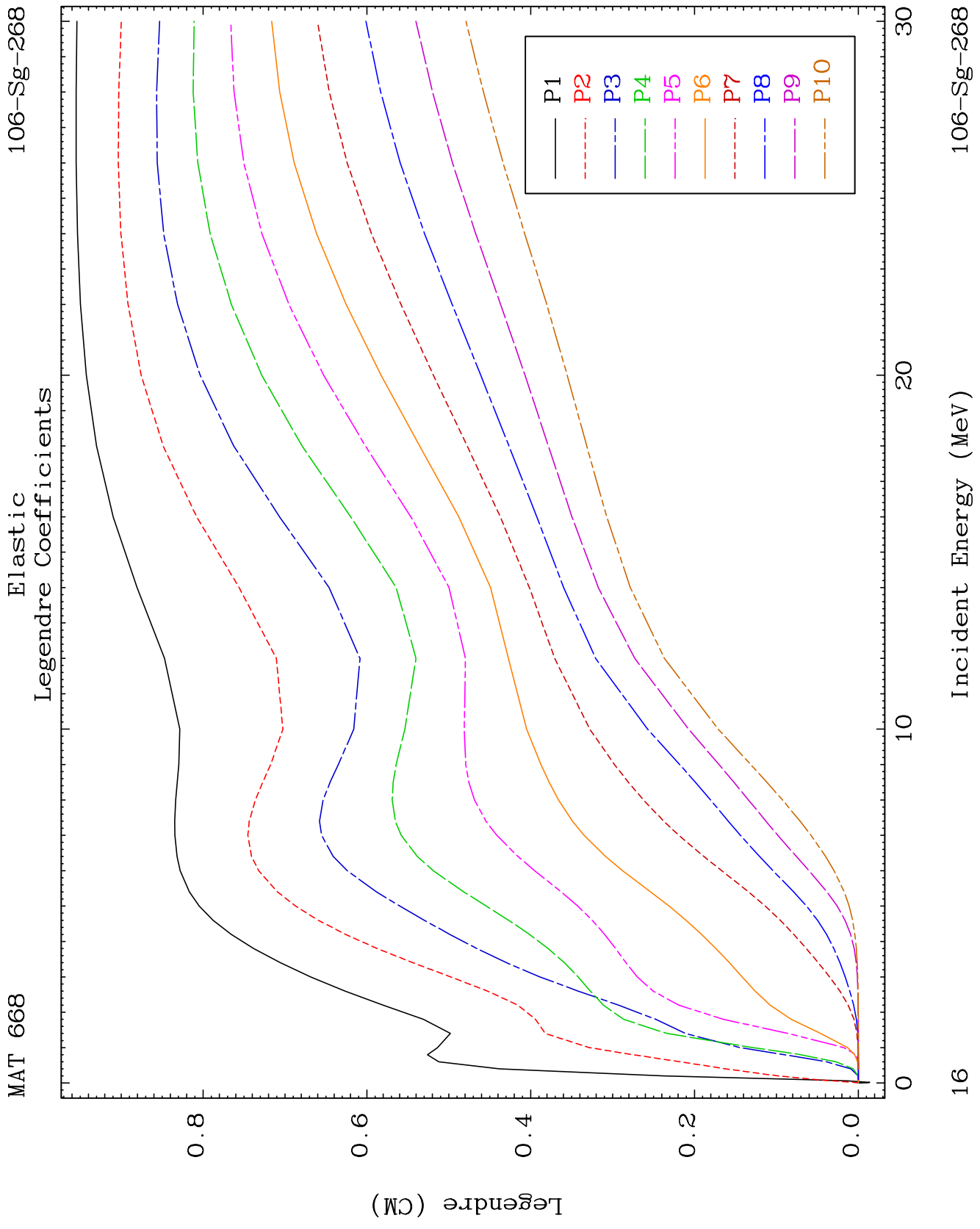
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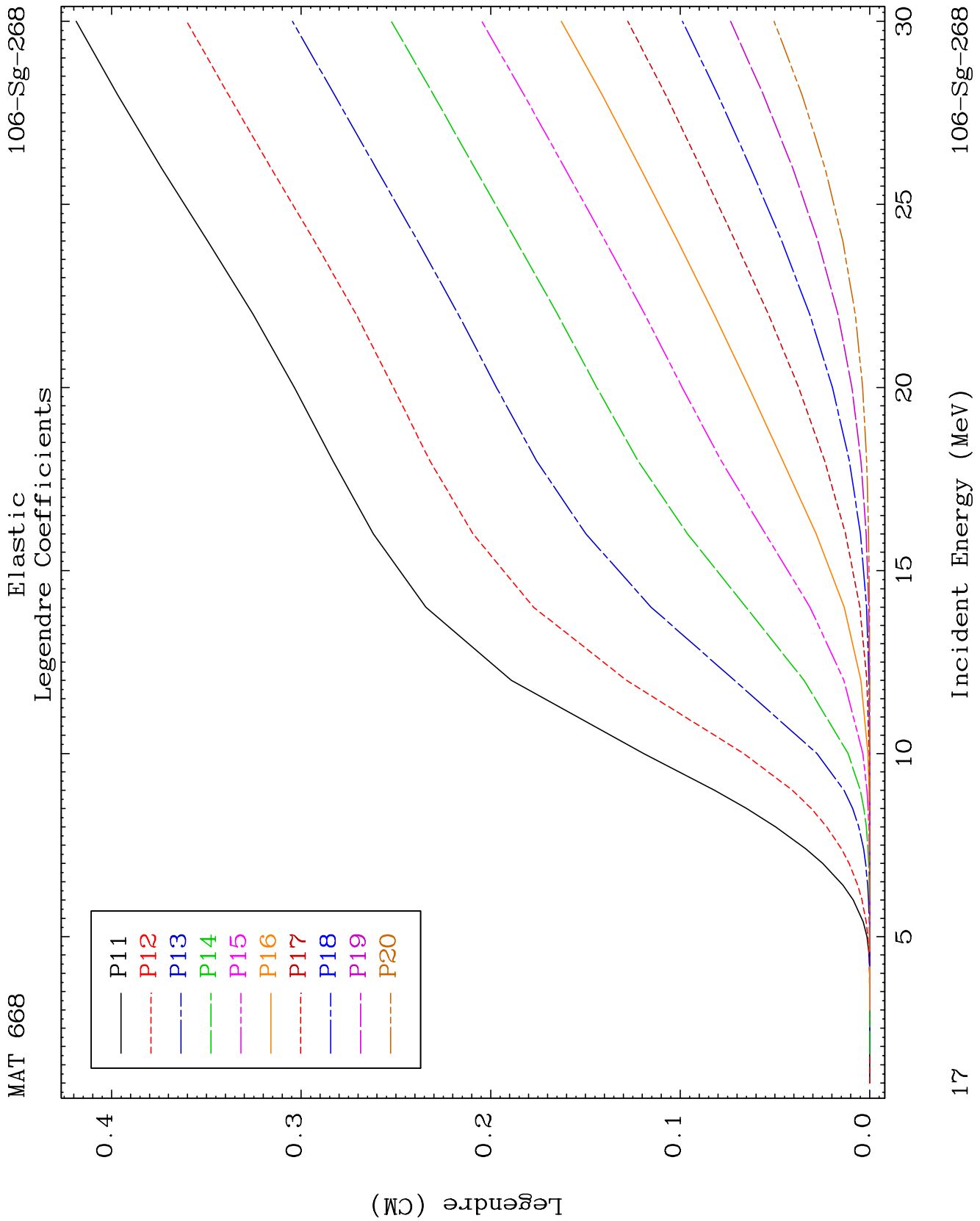












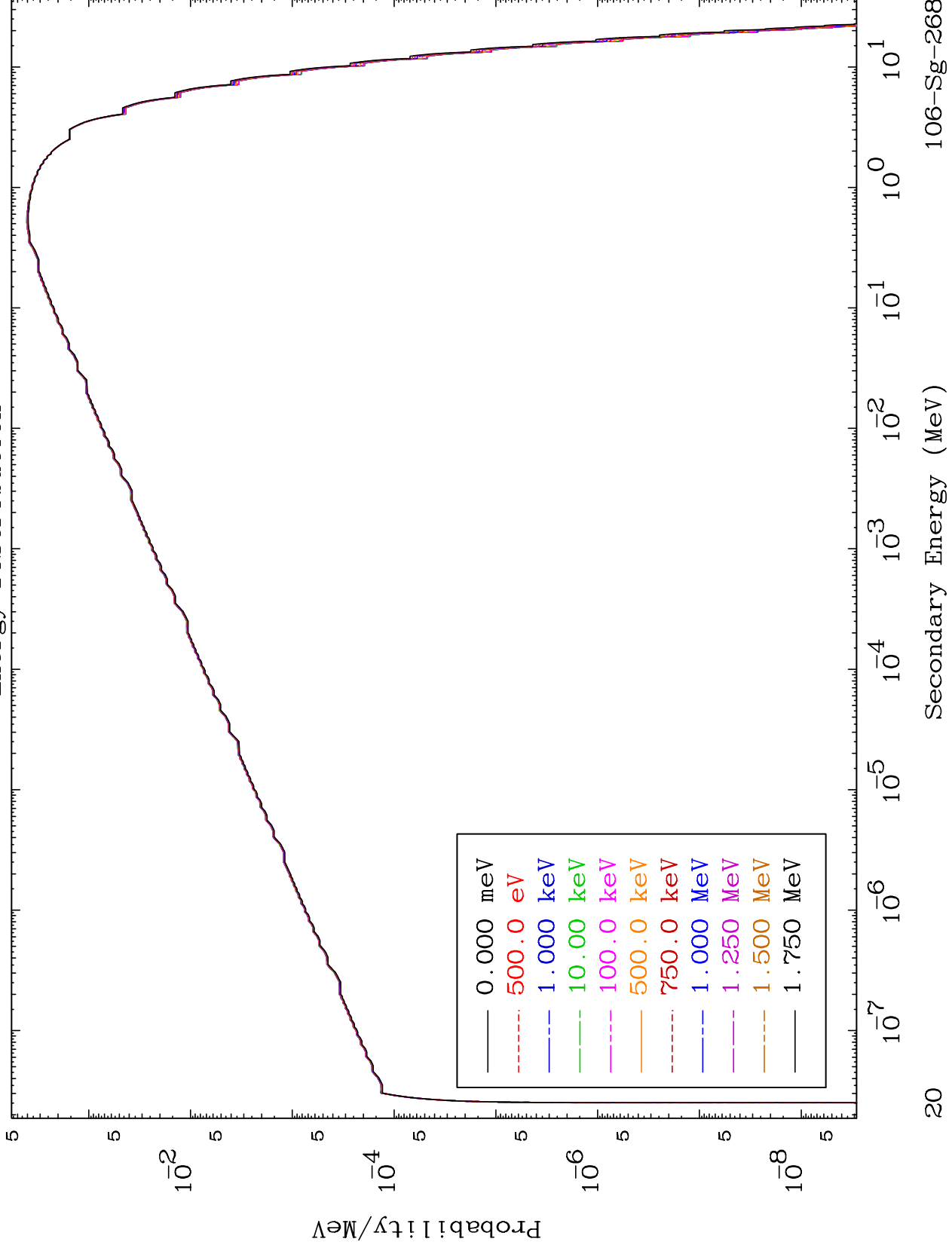




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### Fission Energy Distribution

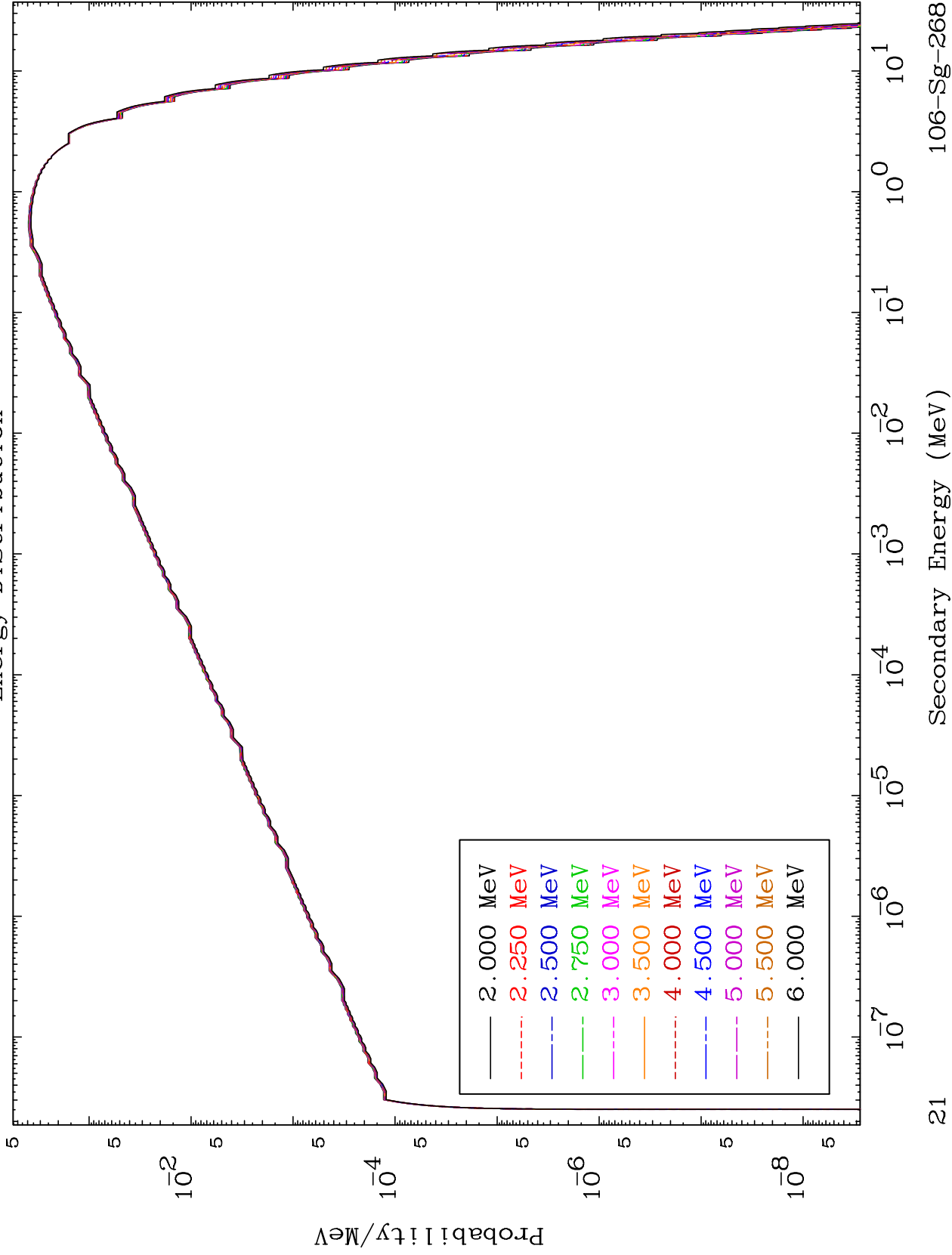
106-Sg-268



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### Fission Energy Distribution

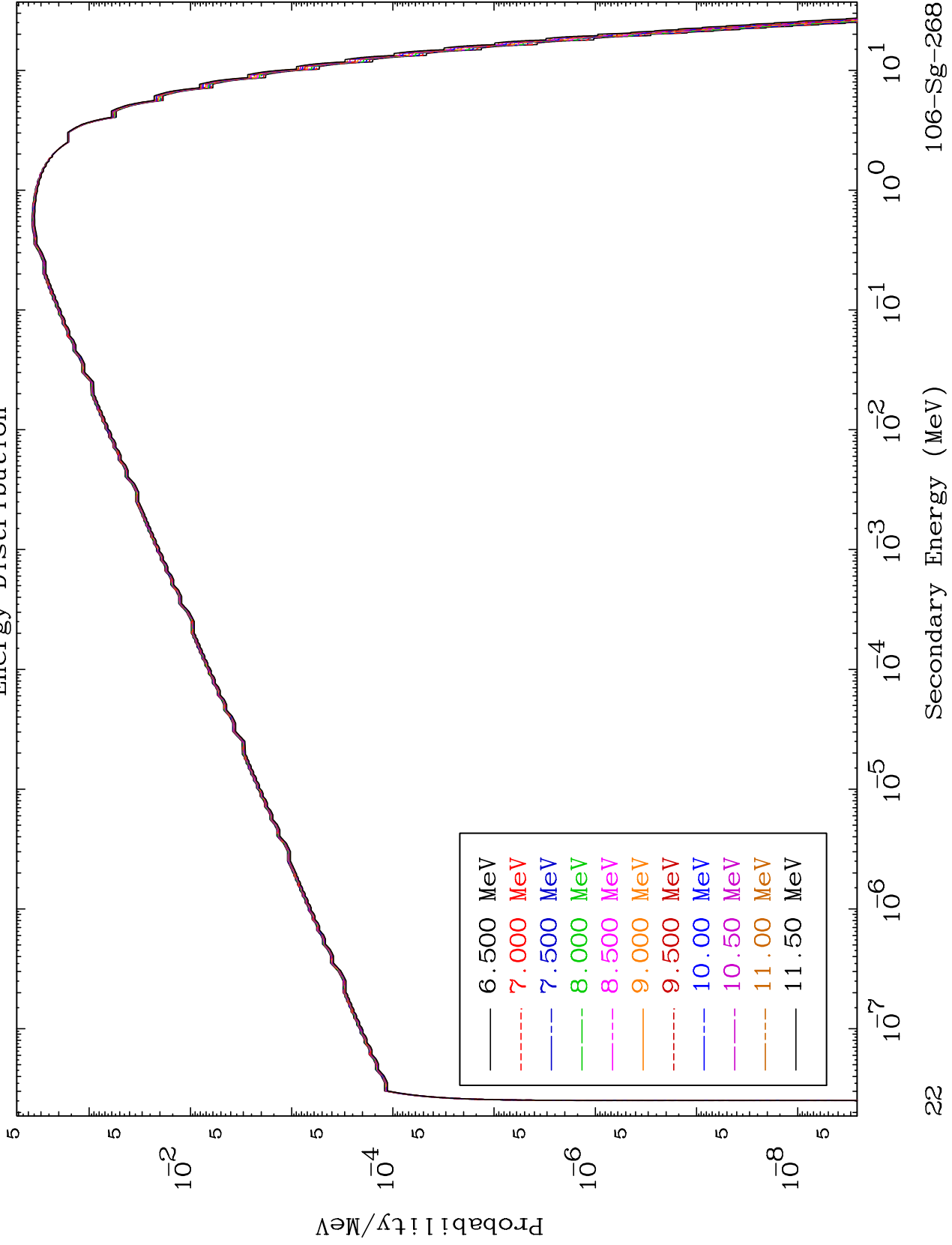
106-Sg-268



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### Fission Energy Distribution

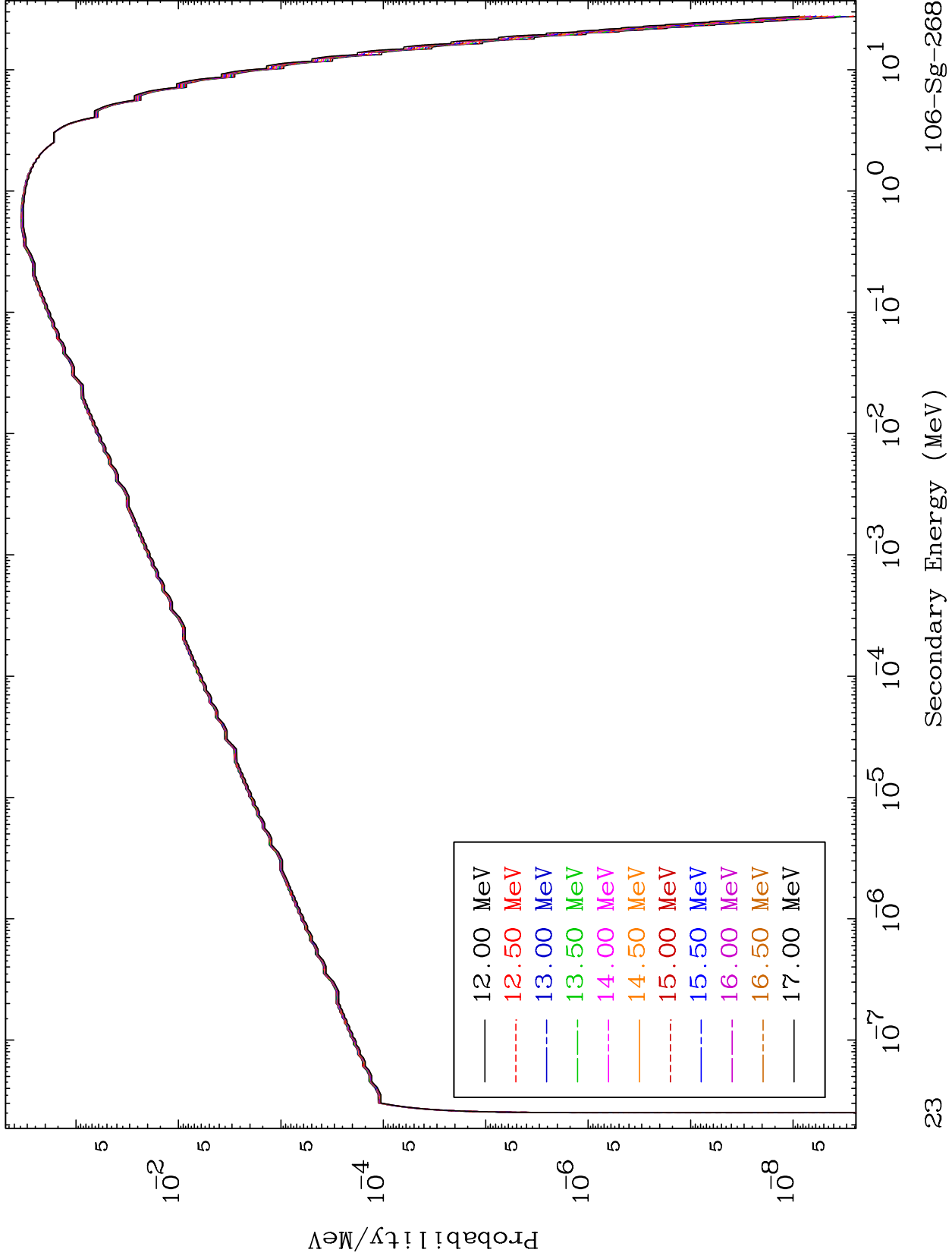
106-Sg-268



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### Fission Energy Distribution

106-Sg-268

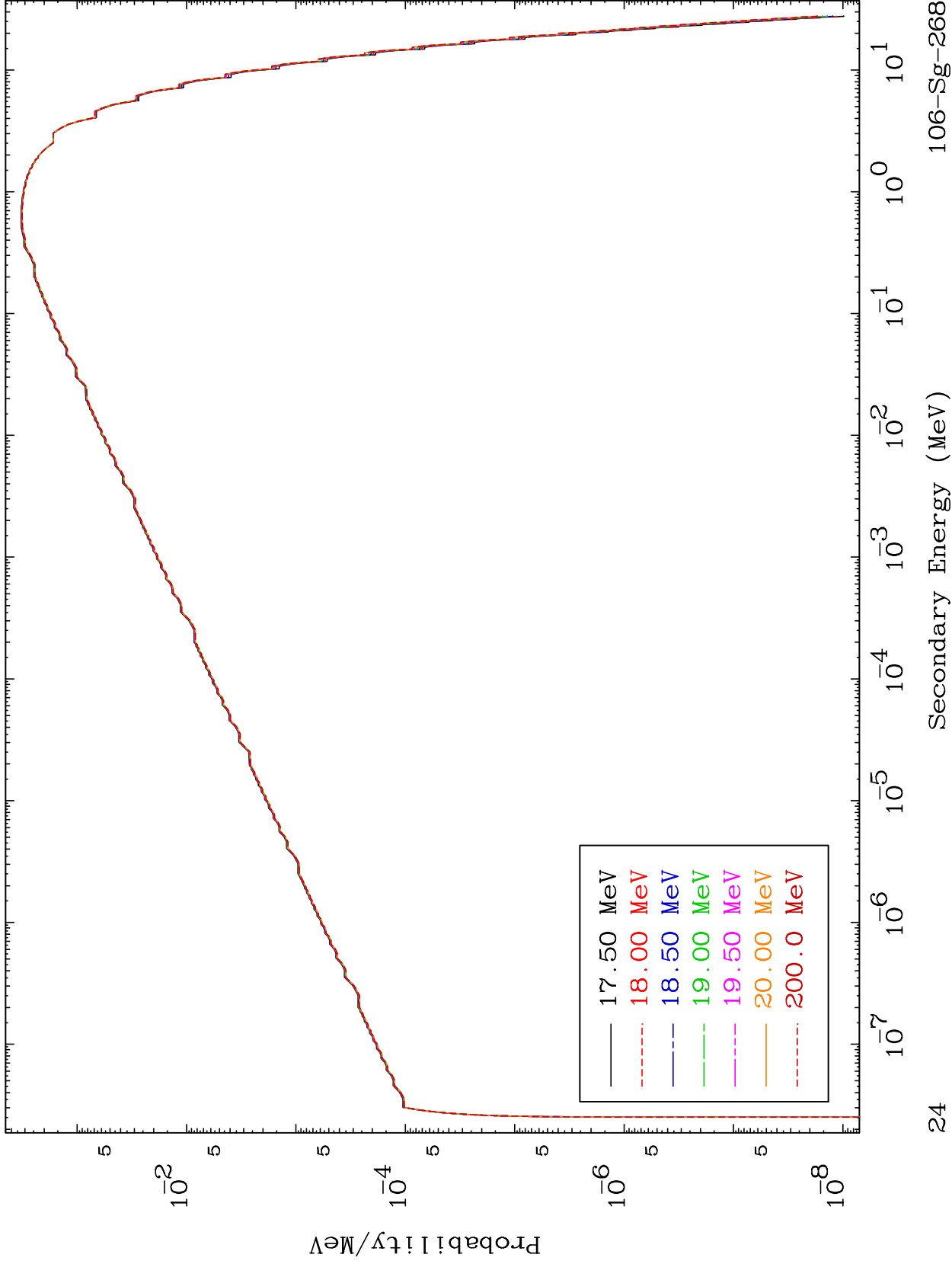




MAT 668

# Fission Energy Distribution

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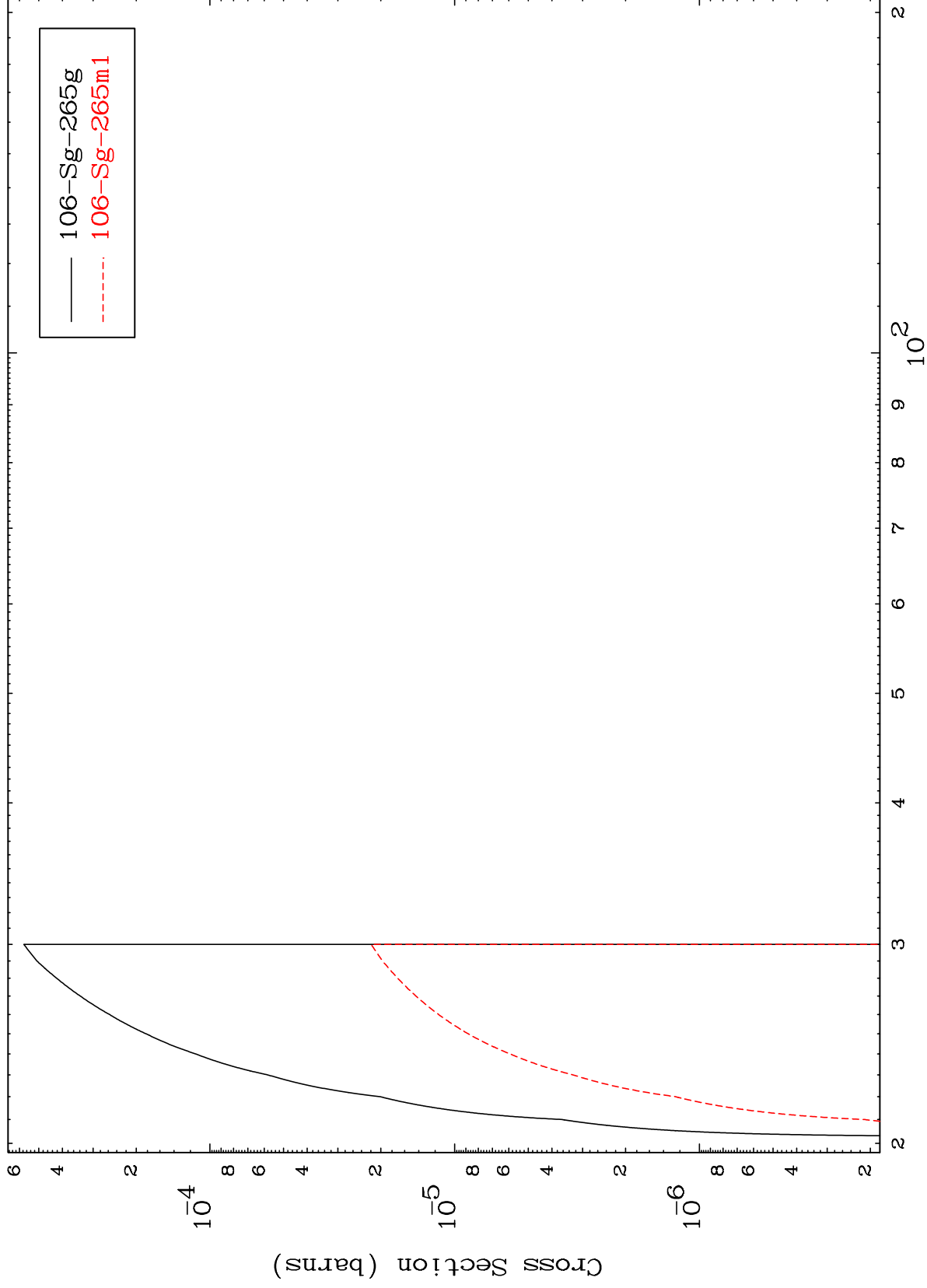


MAT 668

(n,4n)

106-Sg-268

Radionuclide Production Cross Section



25

Incident Energy (MeV)

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