

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

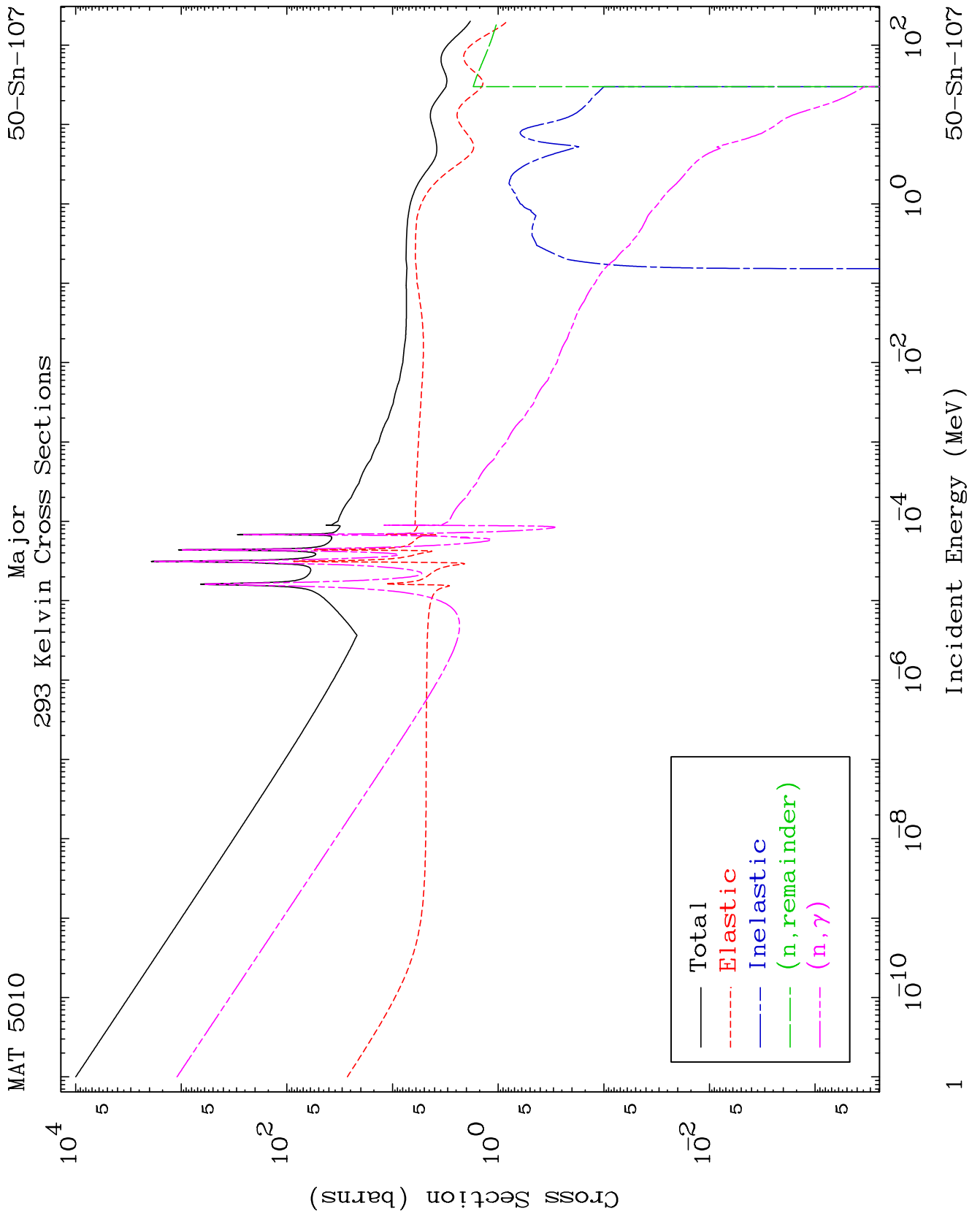
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

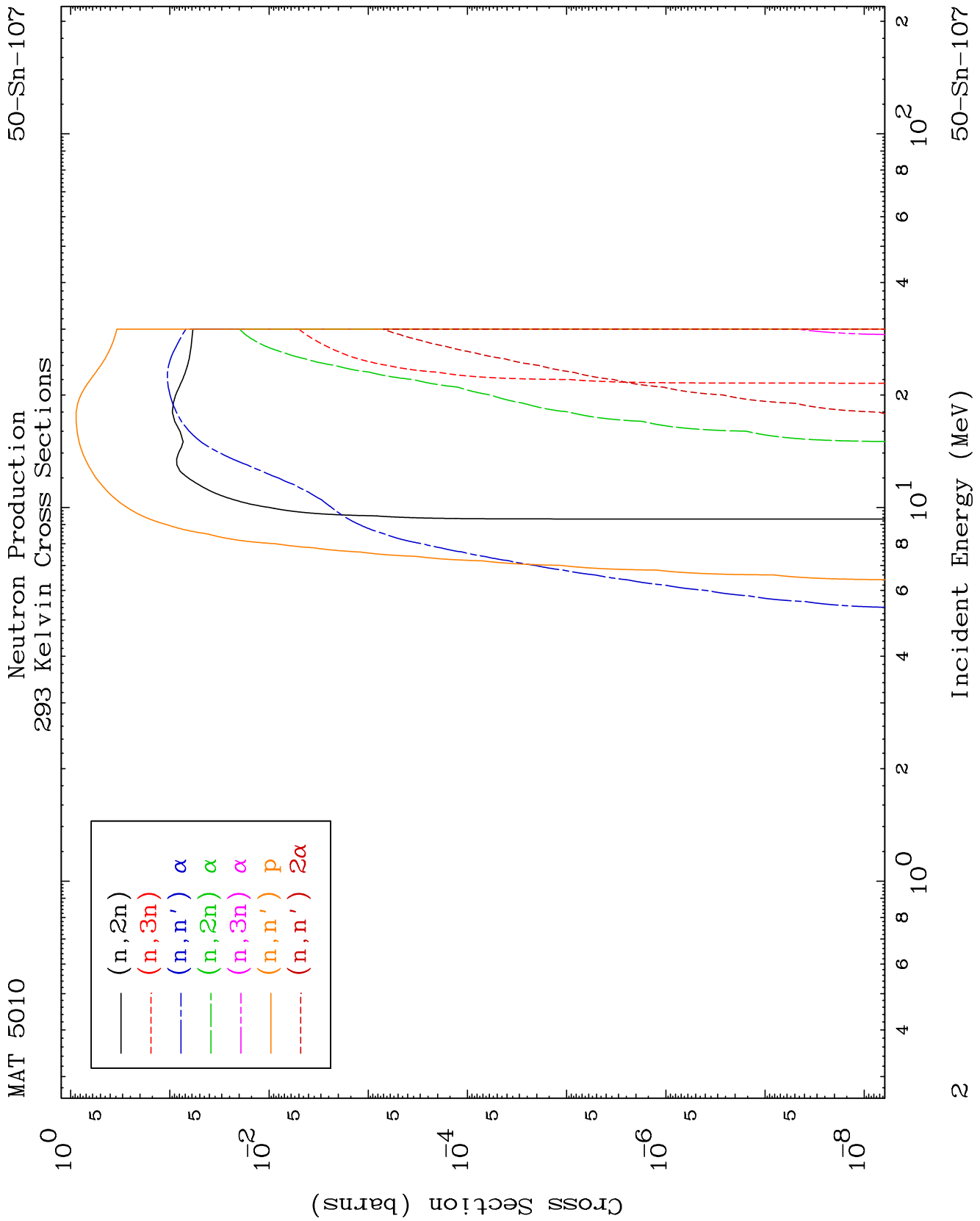
Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

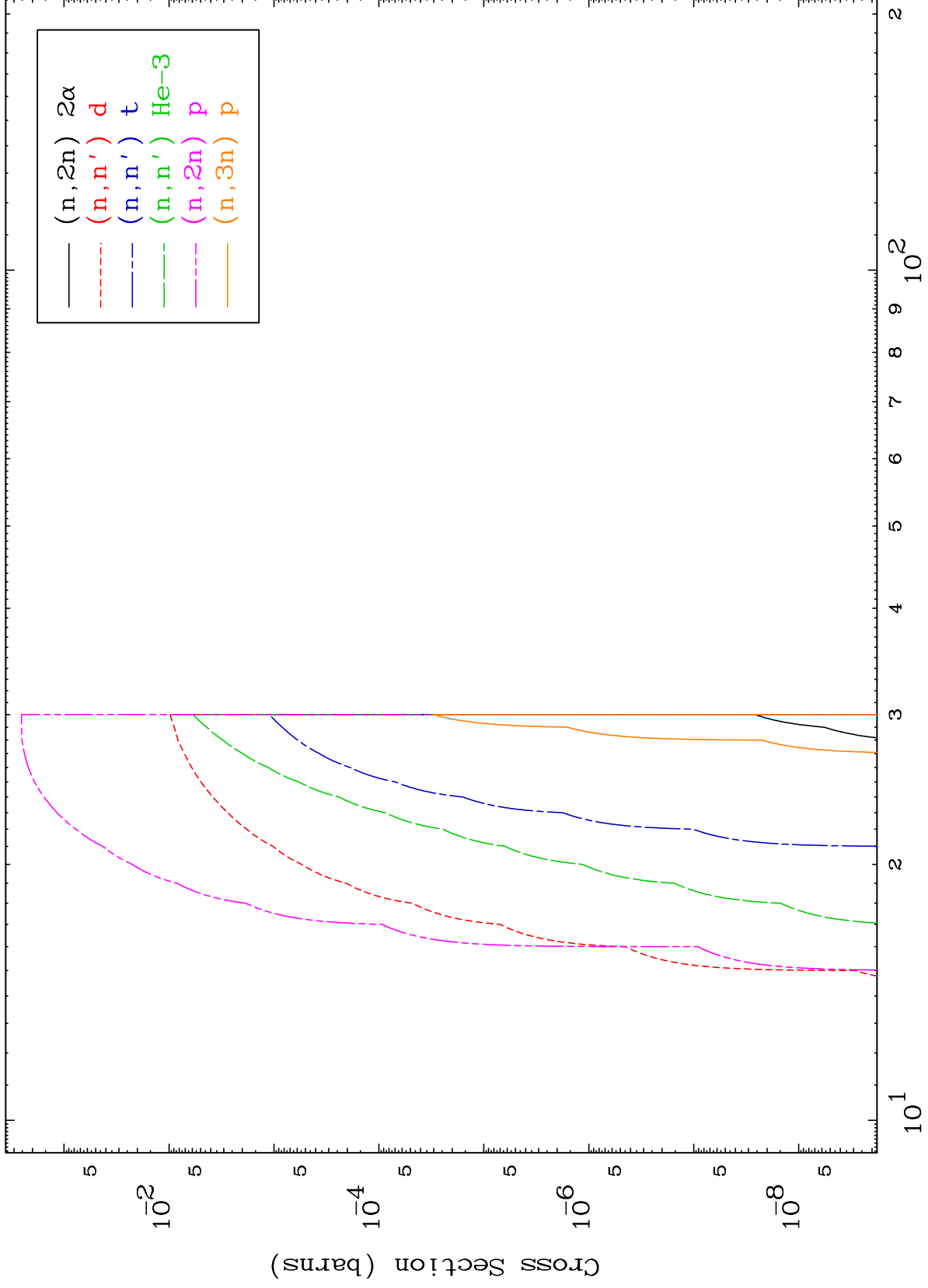




MAT 5010

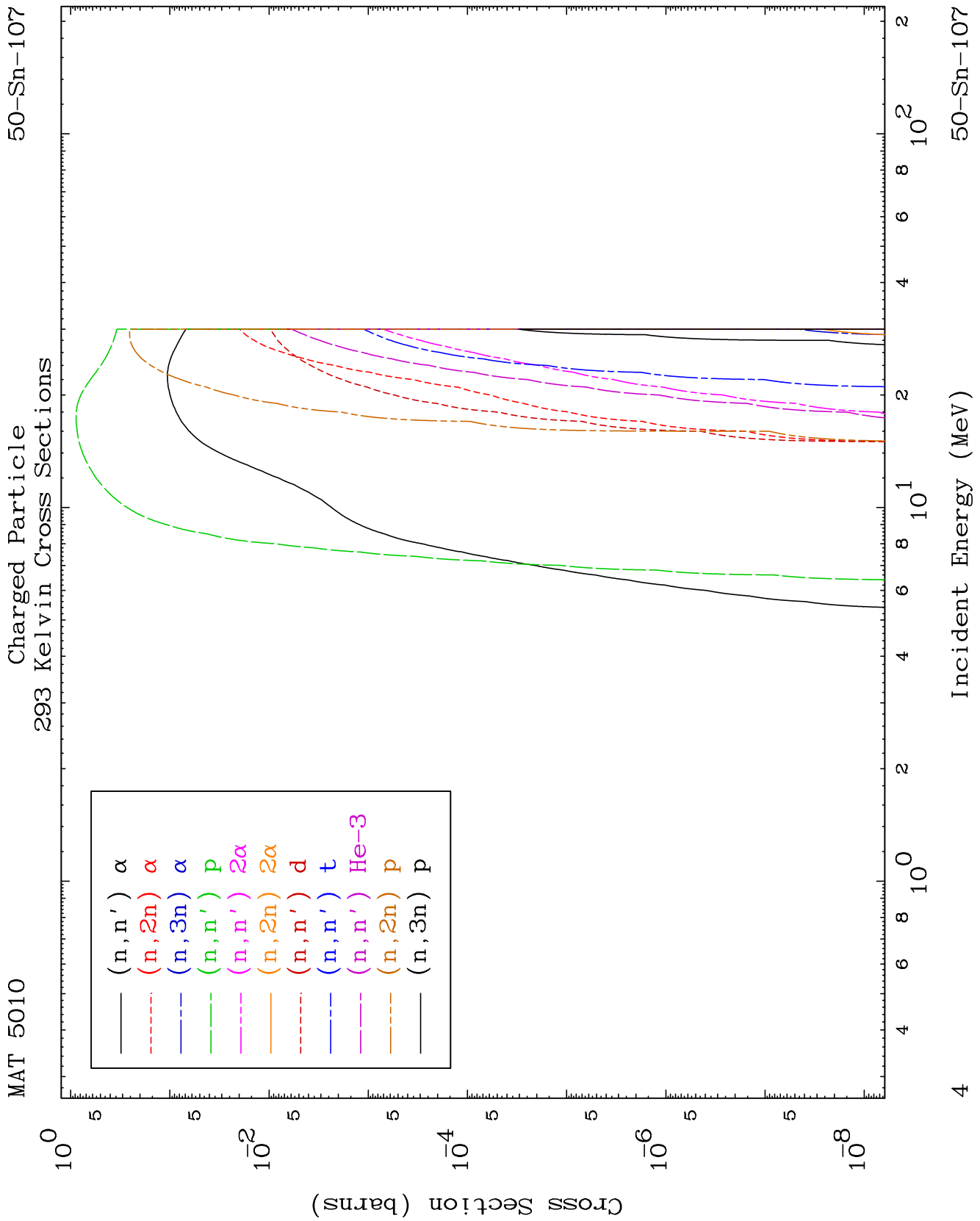
Neutron Production
293 Kelvin Cross Sections

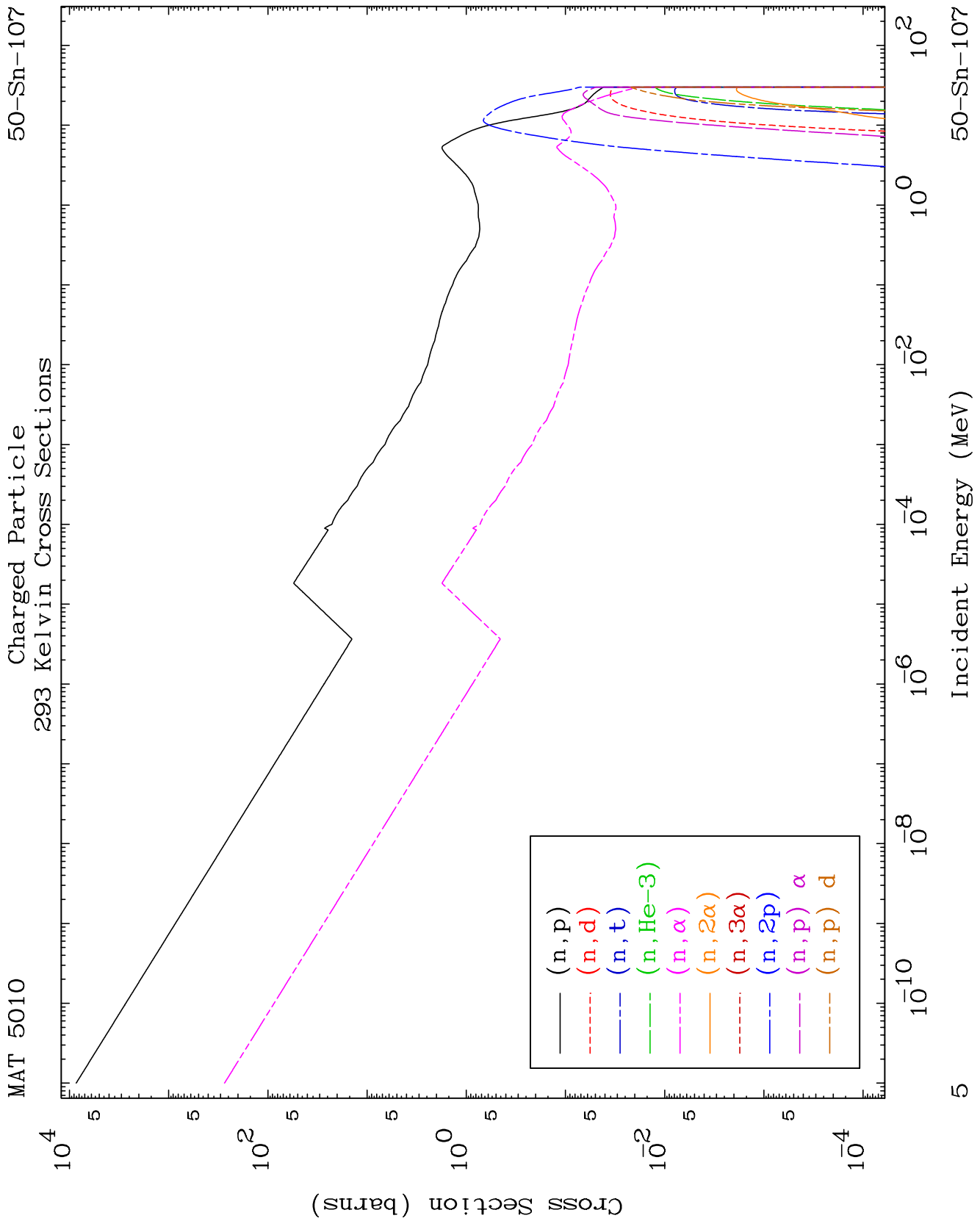
50-Sn-107

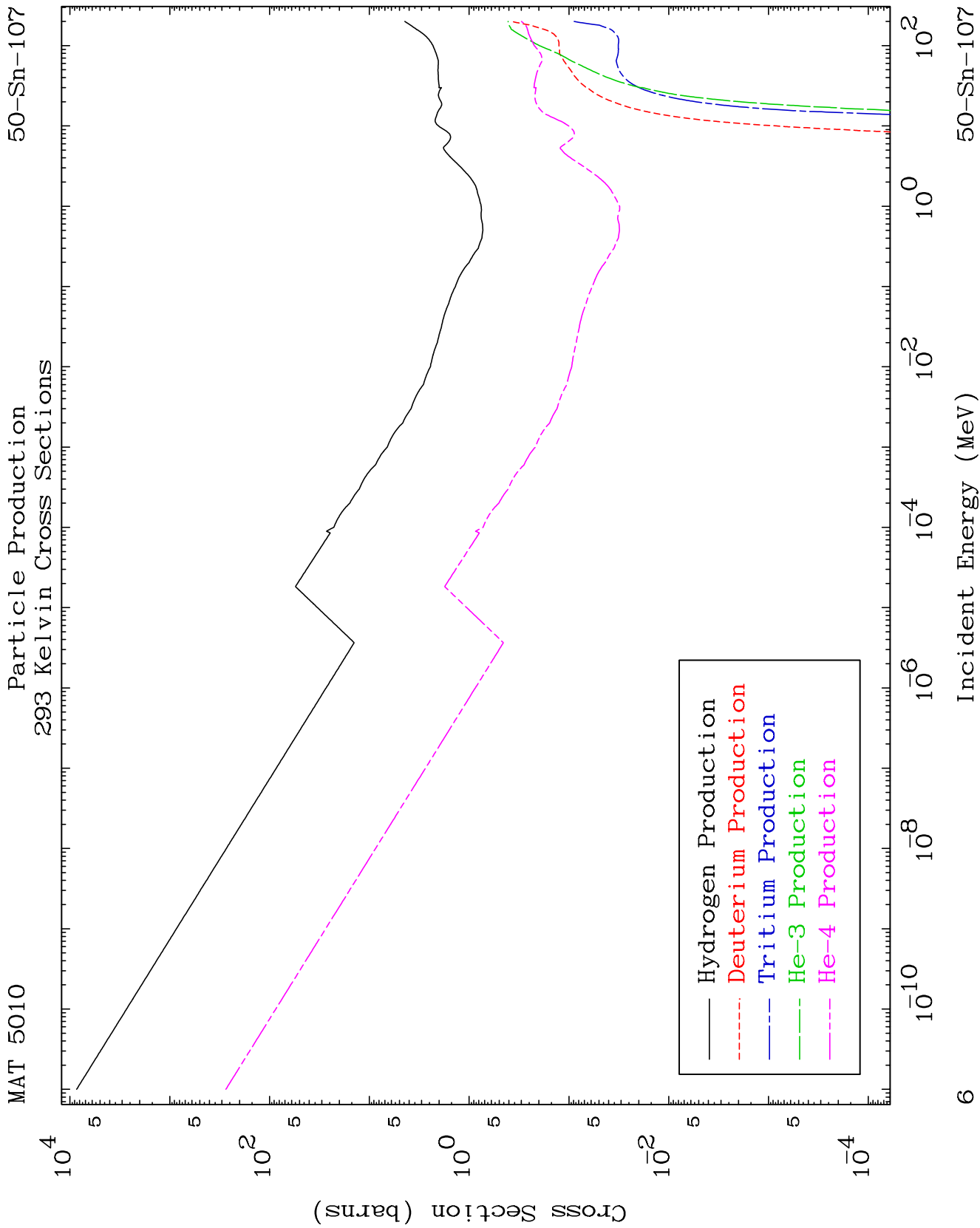


Incident Energy (MeV)

50-Sn-107





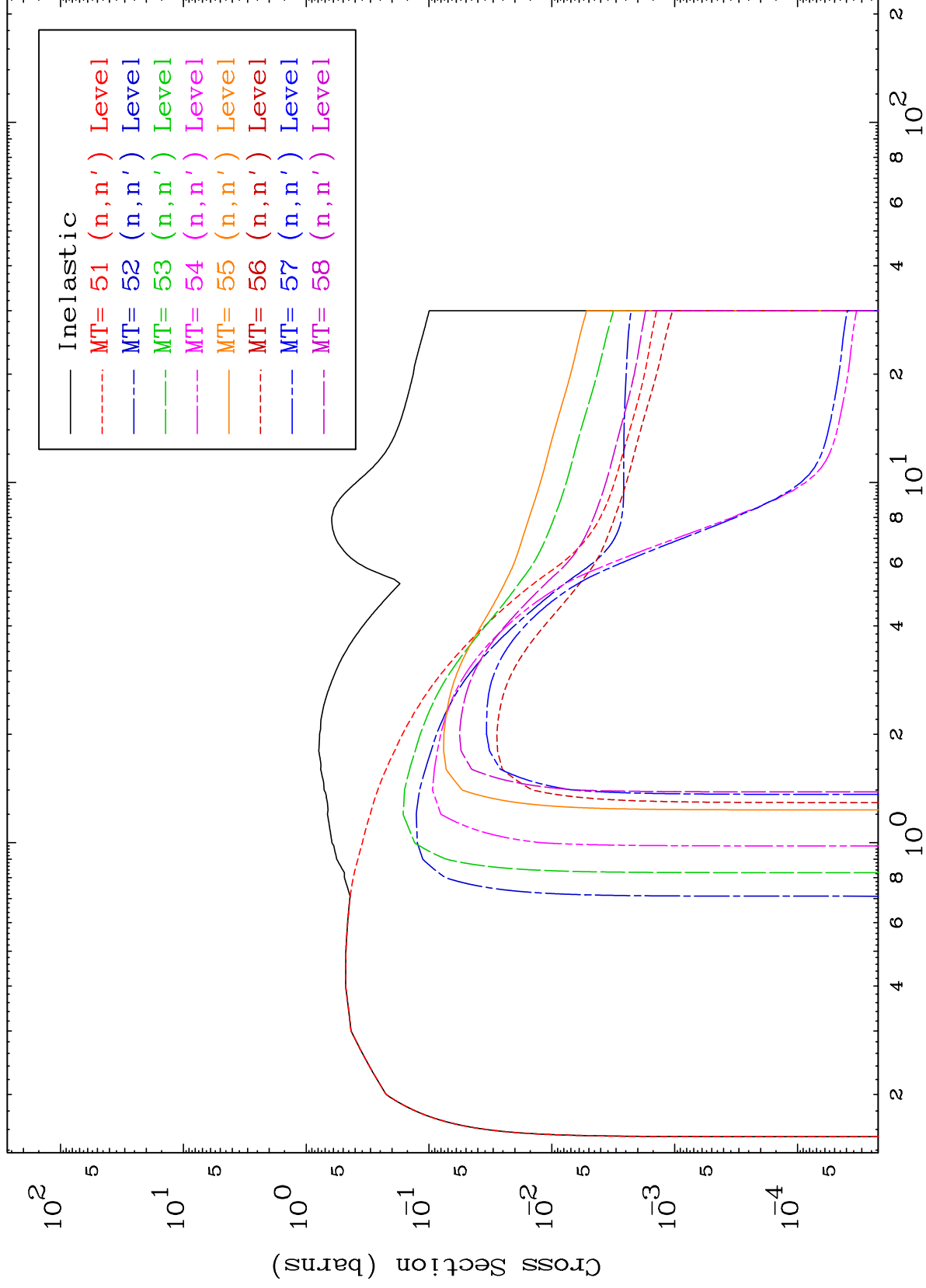


MAT 5010

(n,n') Level

50-Sn-107

293 Kelvin Cross Sections



7

Incident Energy (MeV)

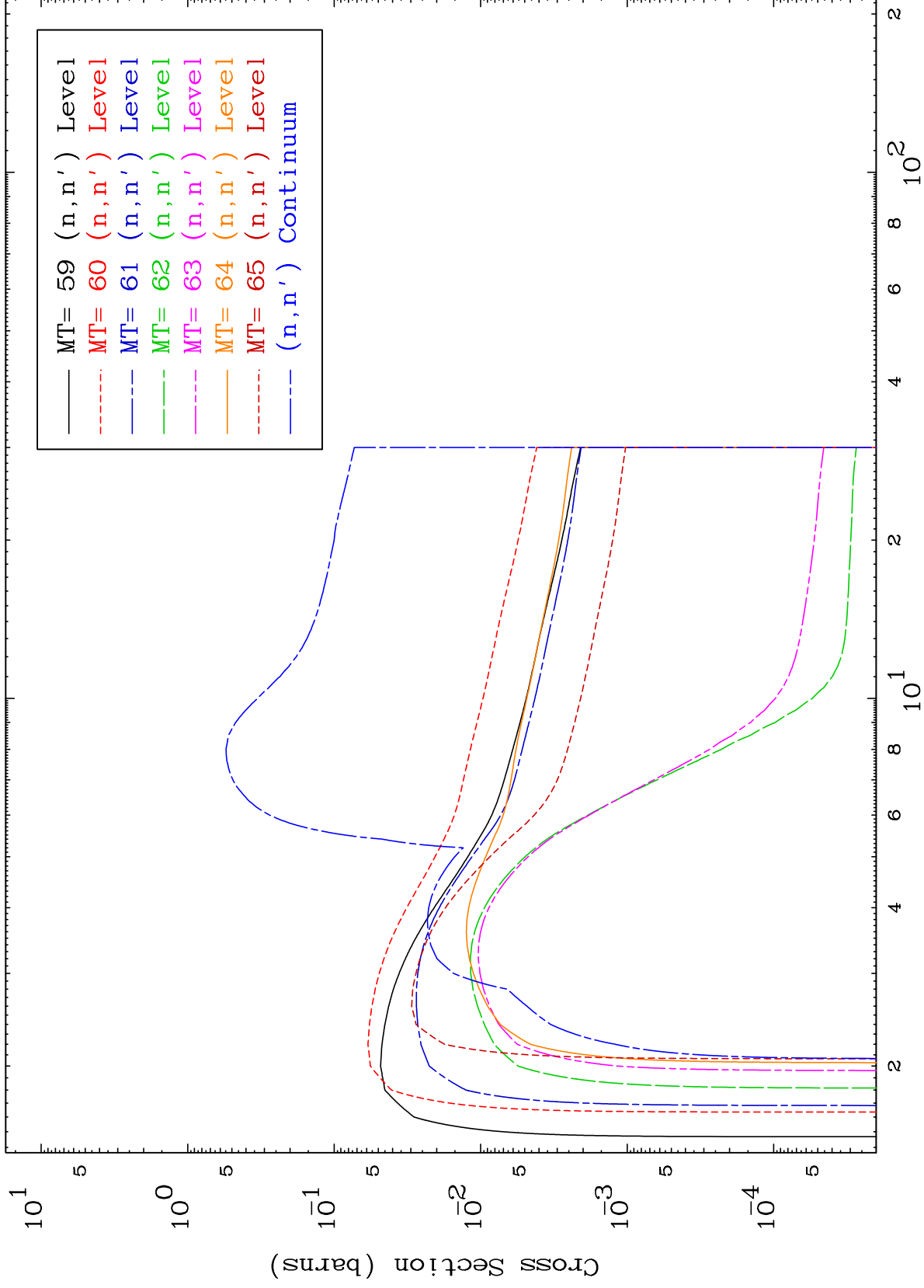
50-Sn-107

MAT 5010

(n,n') Level

50-Sn-107

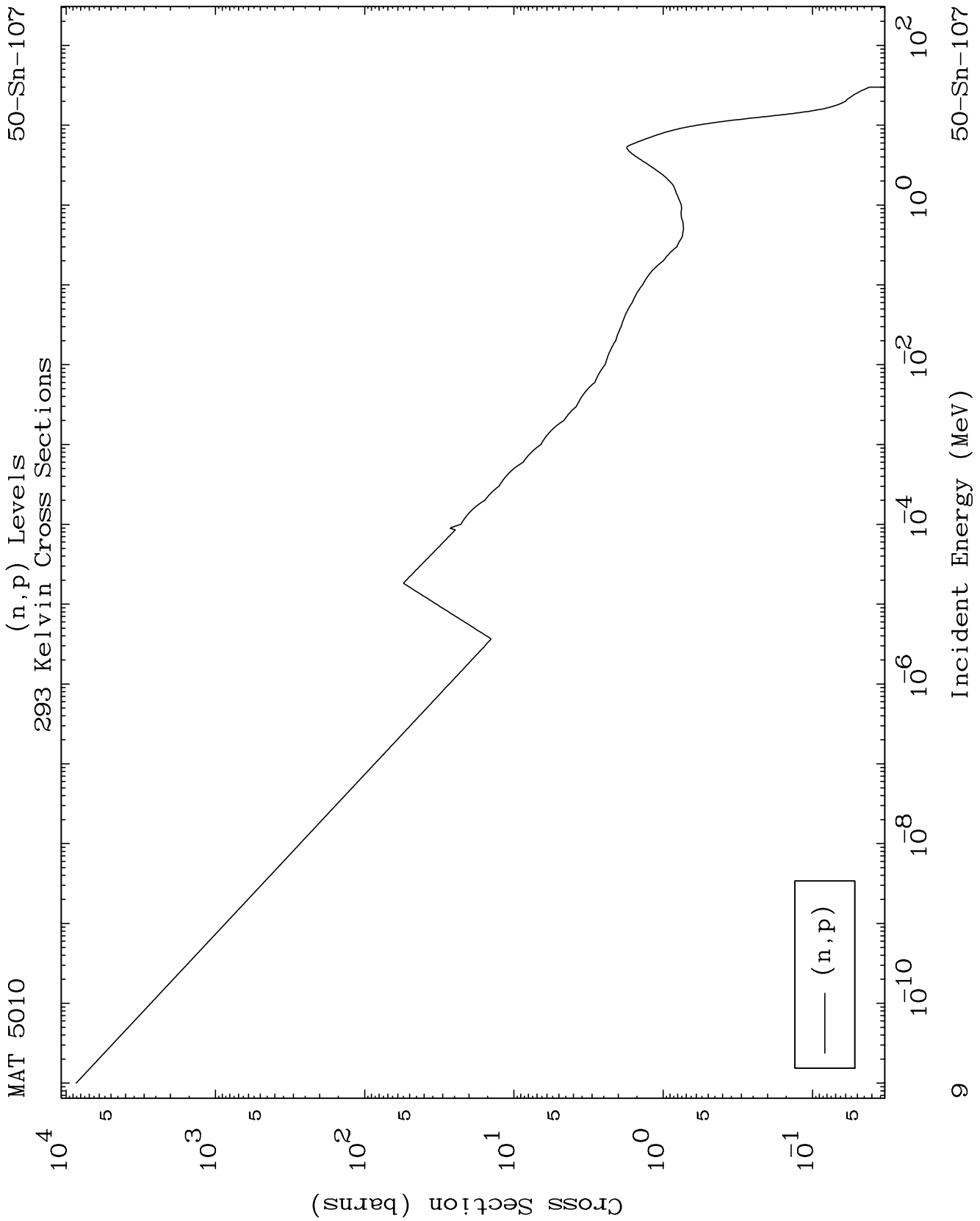
293 Kelvin Cross Sections



8

Incident Energy (MeV)

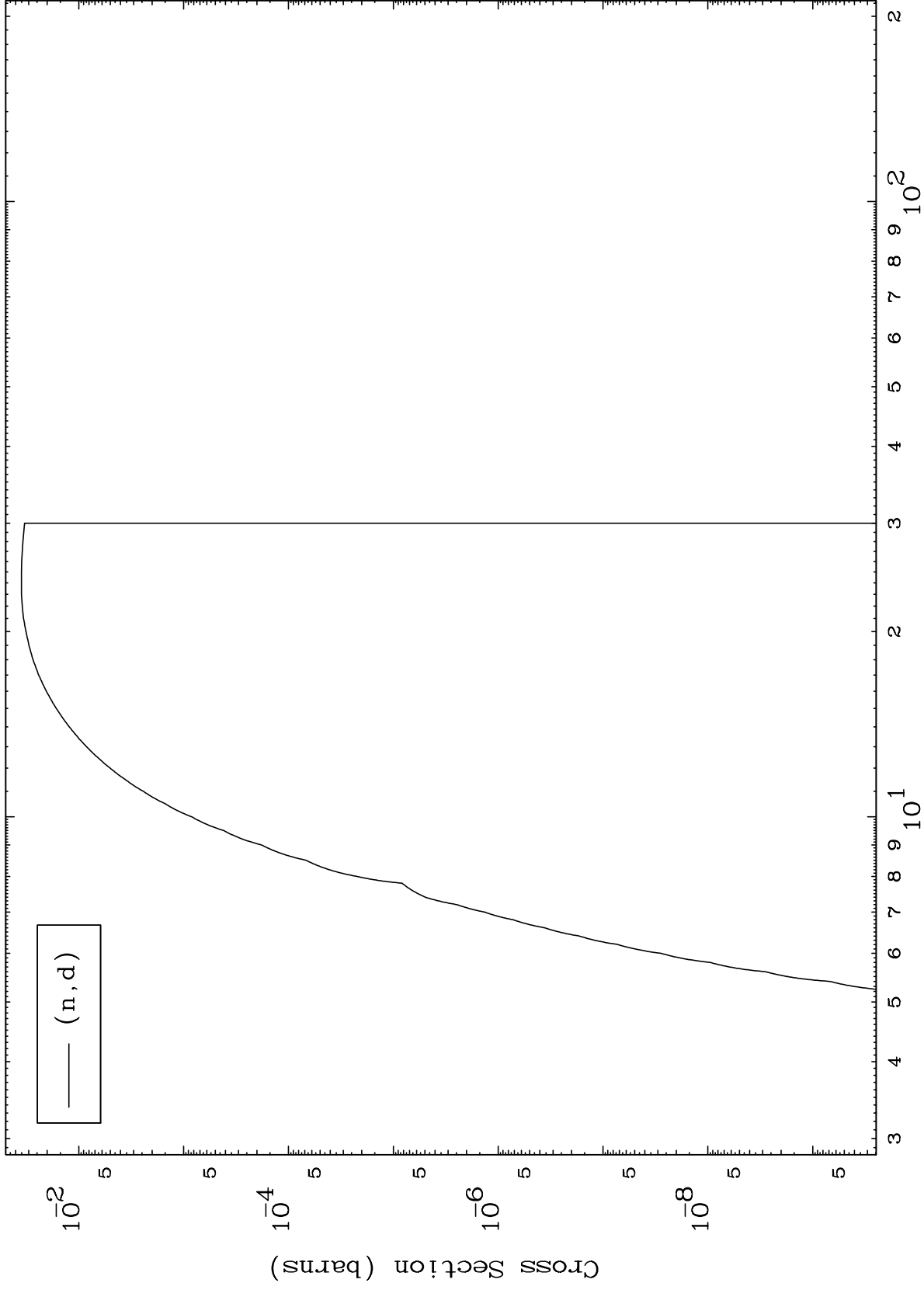
50-Sn-107



MAT 5010

(n,d) Levels
293 Kelvin Cross Sections

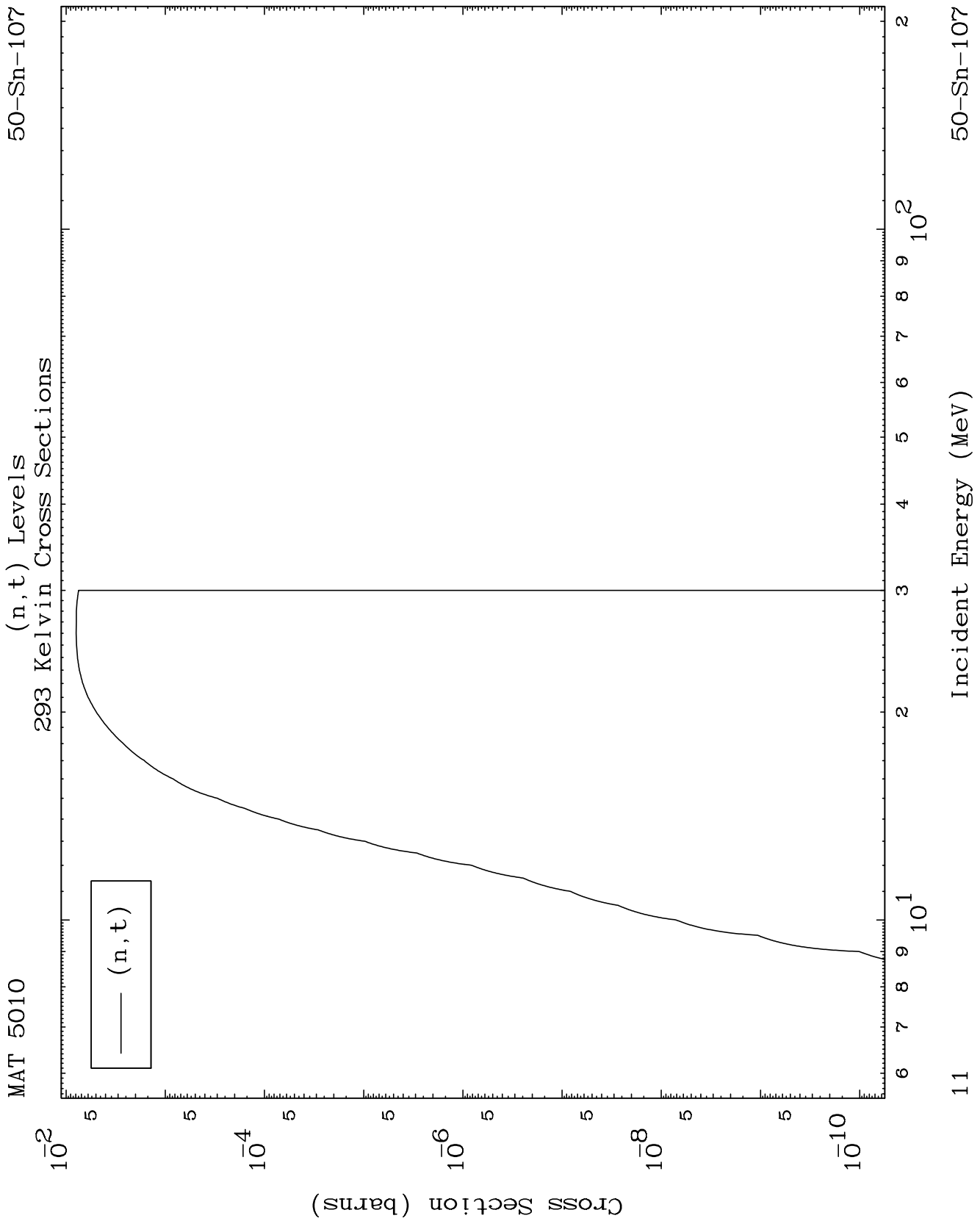
50-Sn-107



50-Sn-107

Incident Energy (MeV)

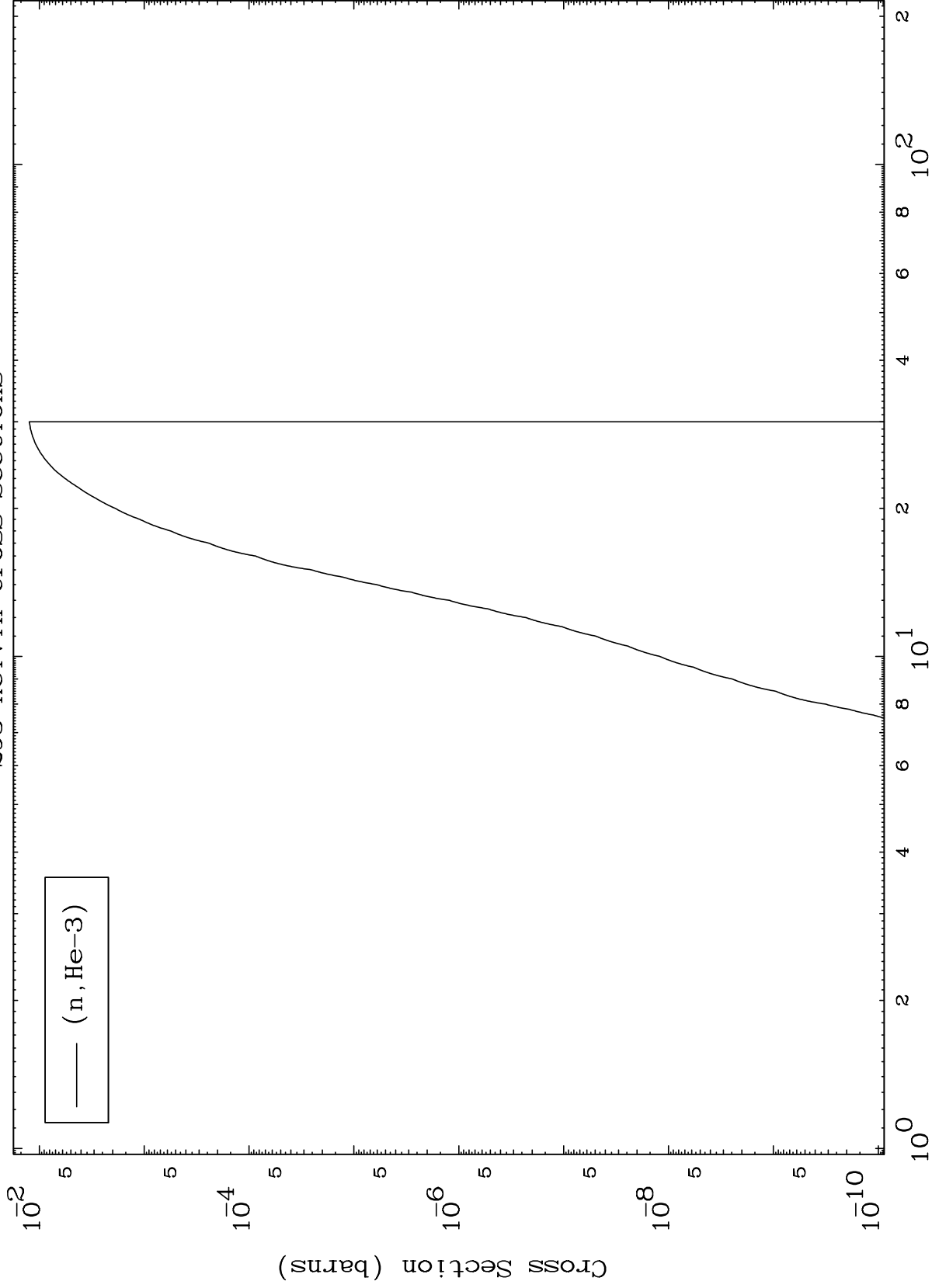
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MAT 5010

(n,He3) Levels
293 Kelvin Cross Sections

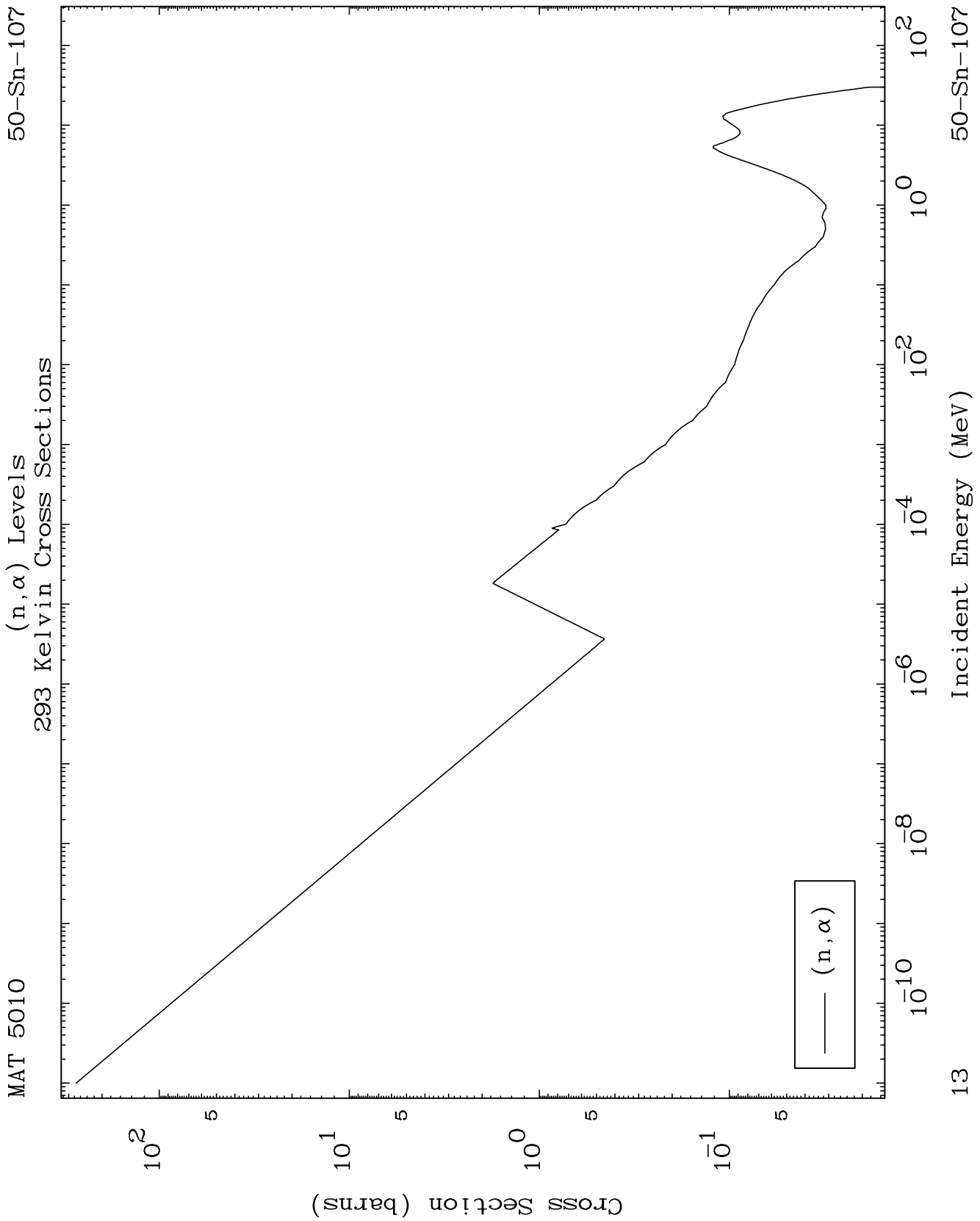
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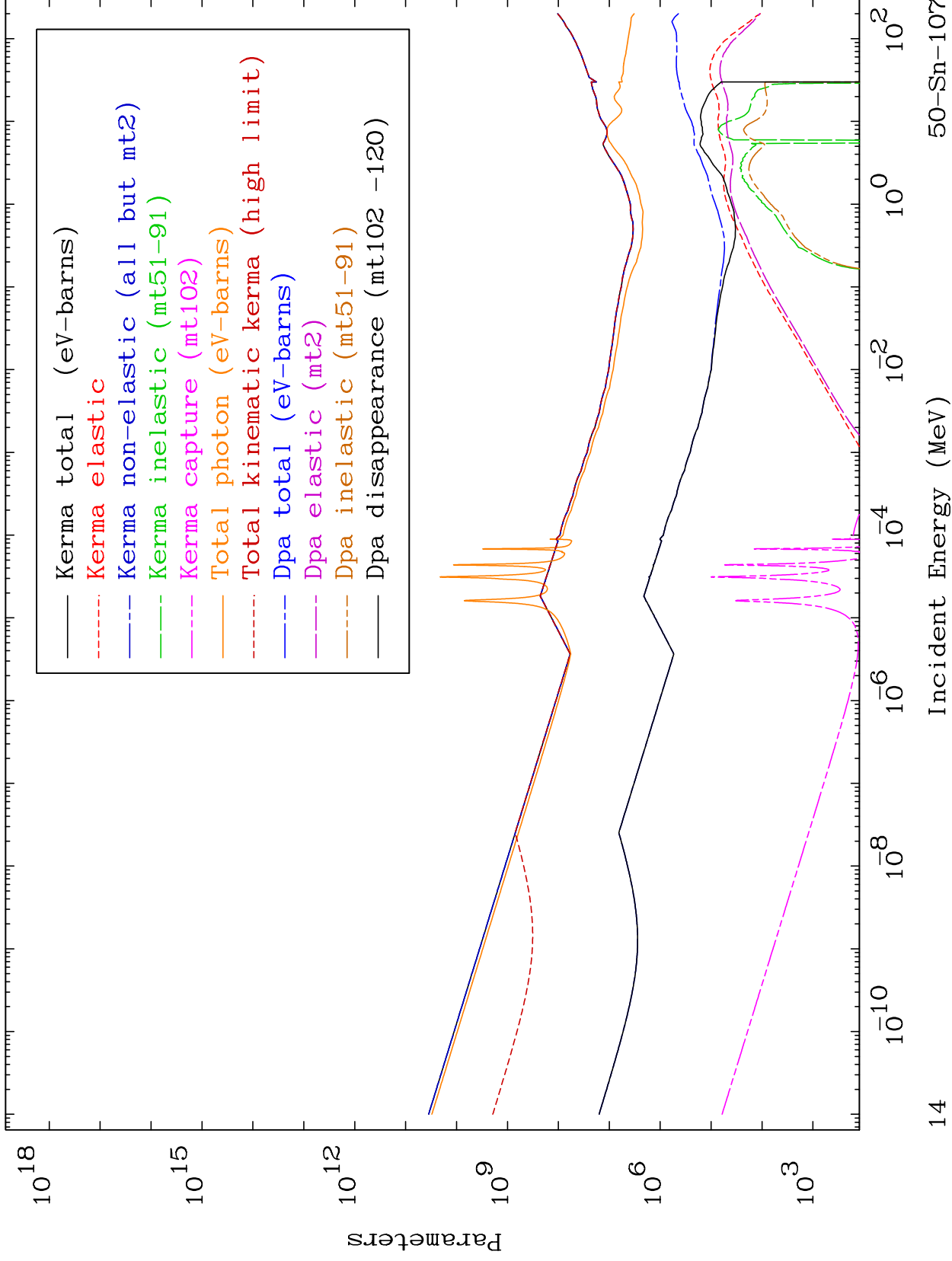


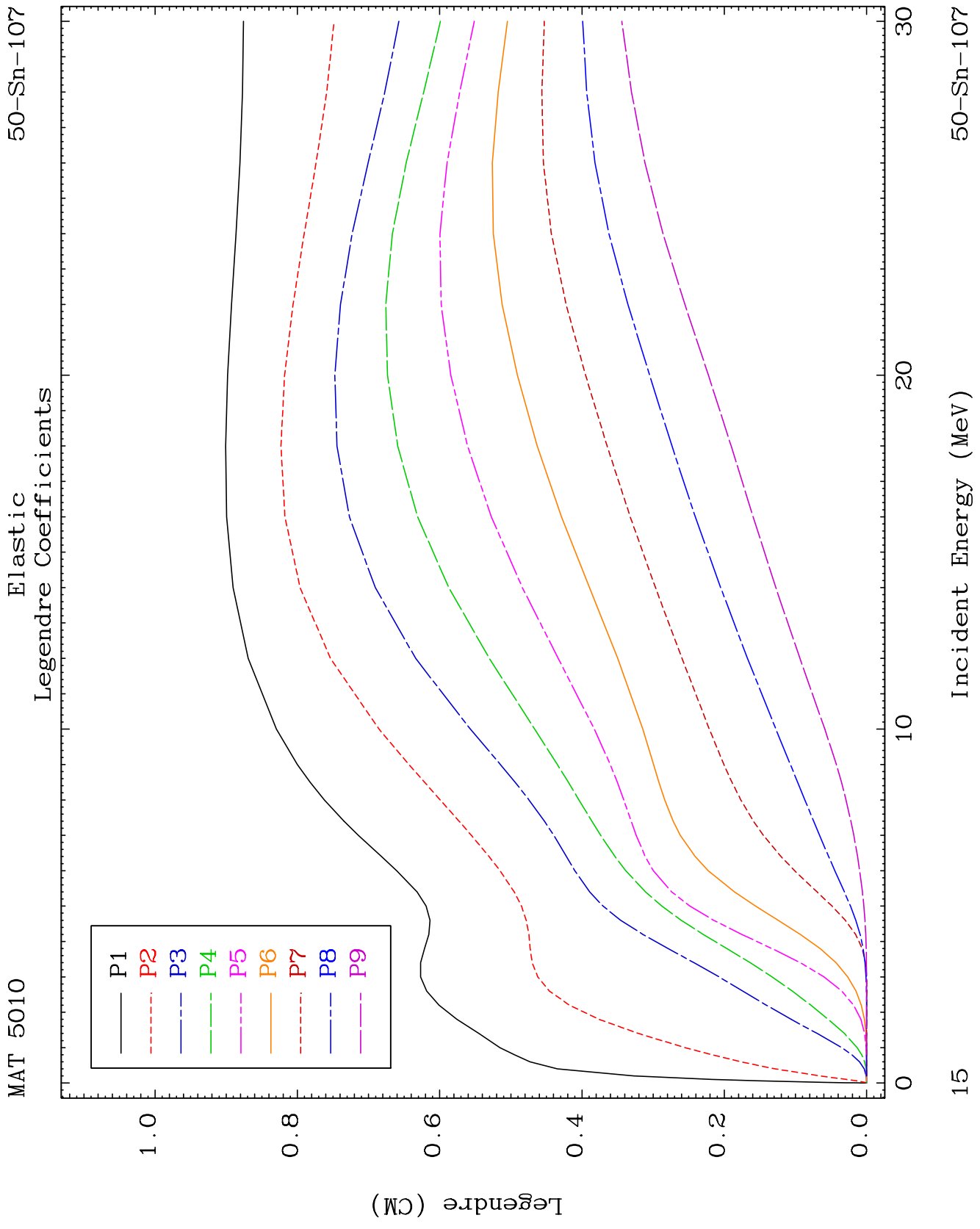
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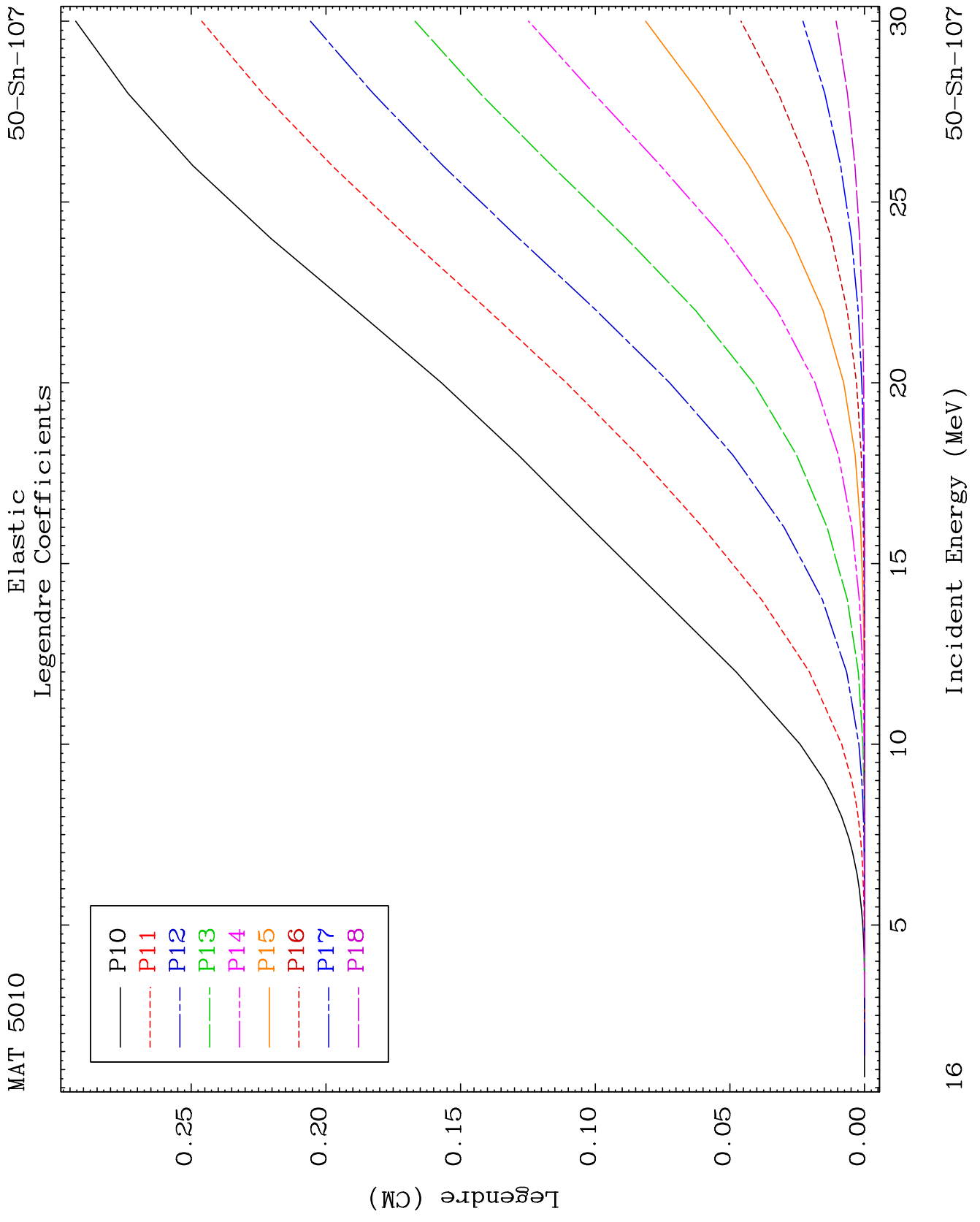
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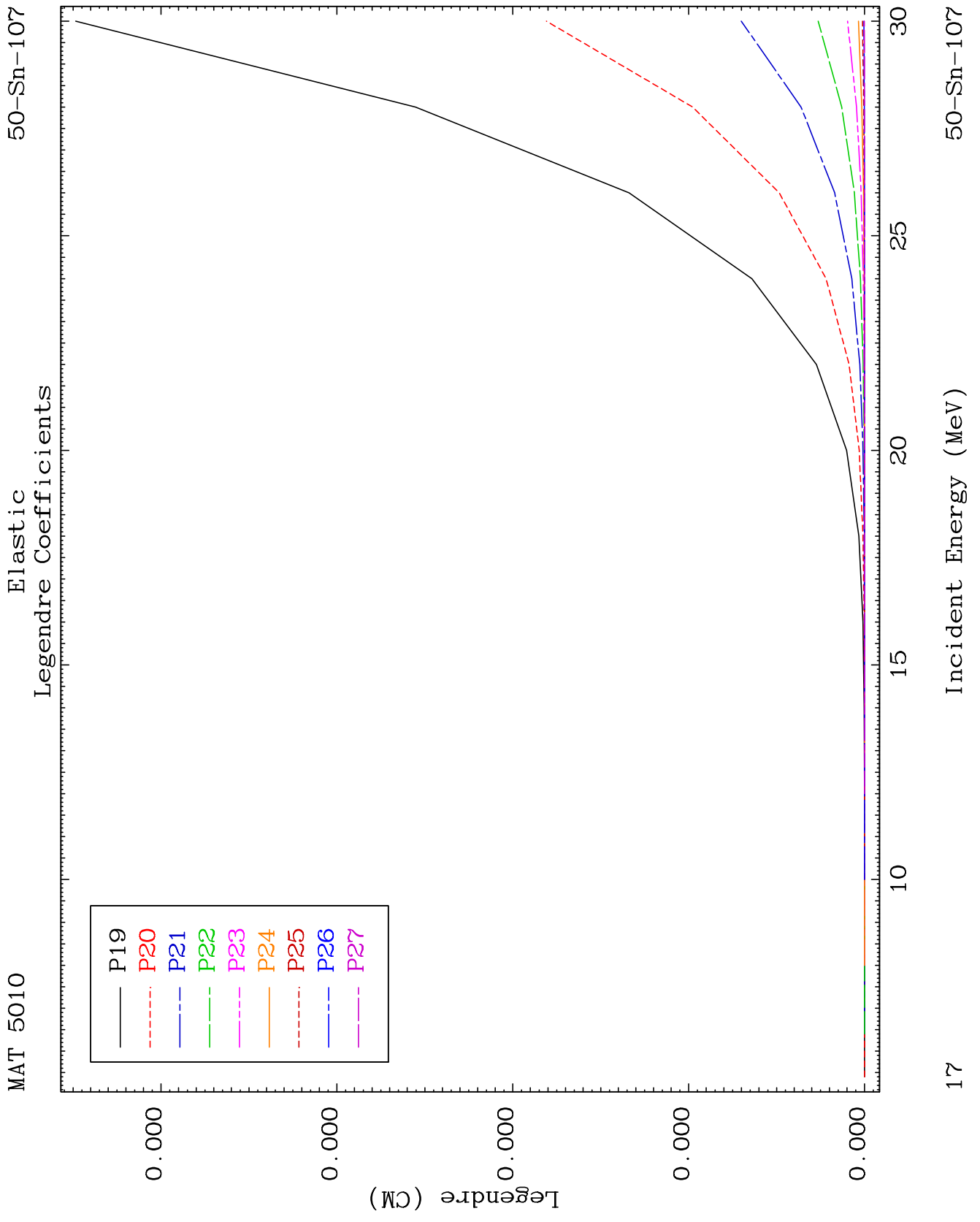
50-Sn-107

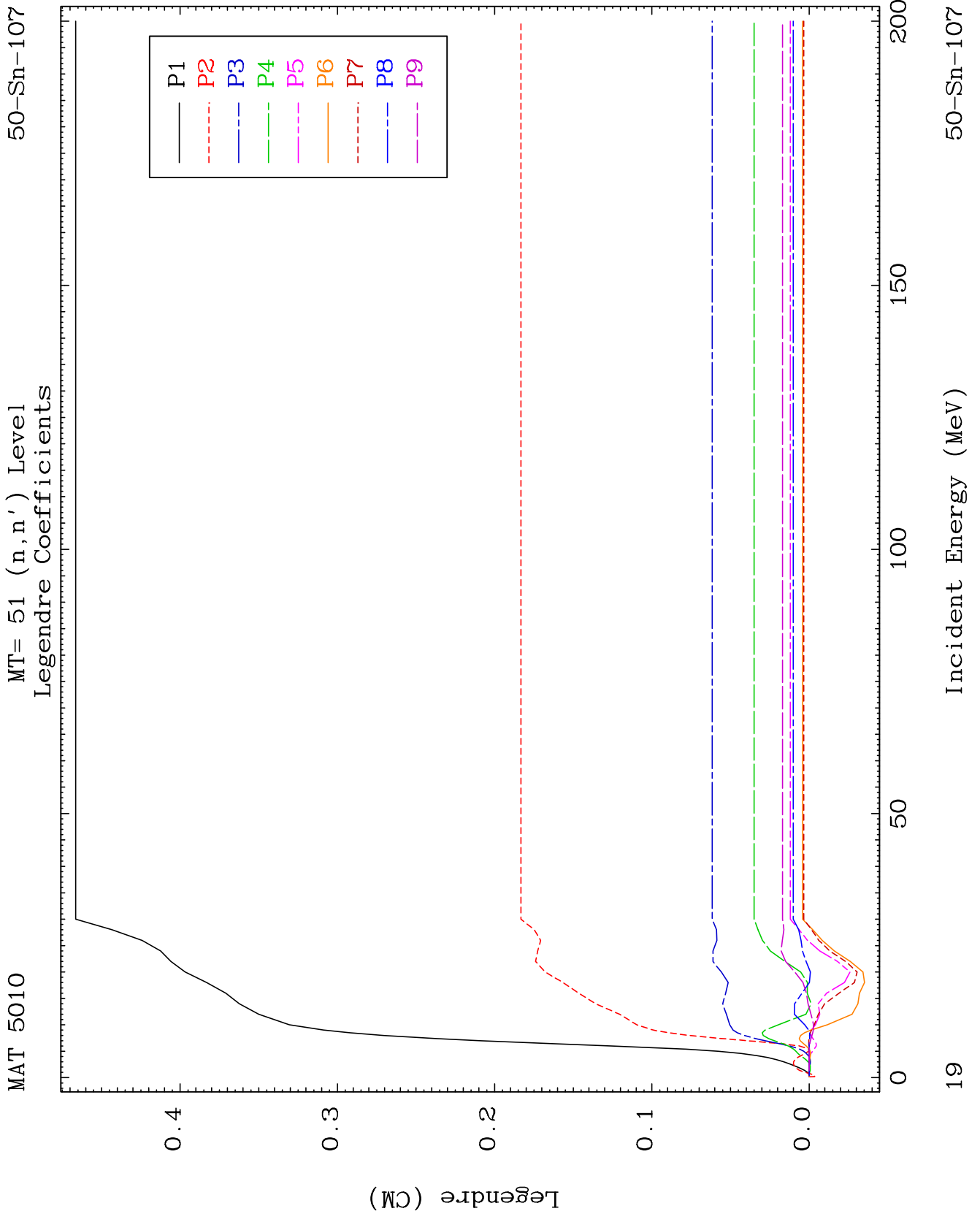


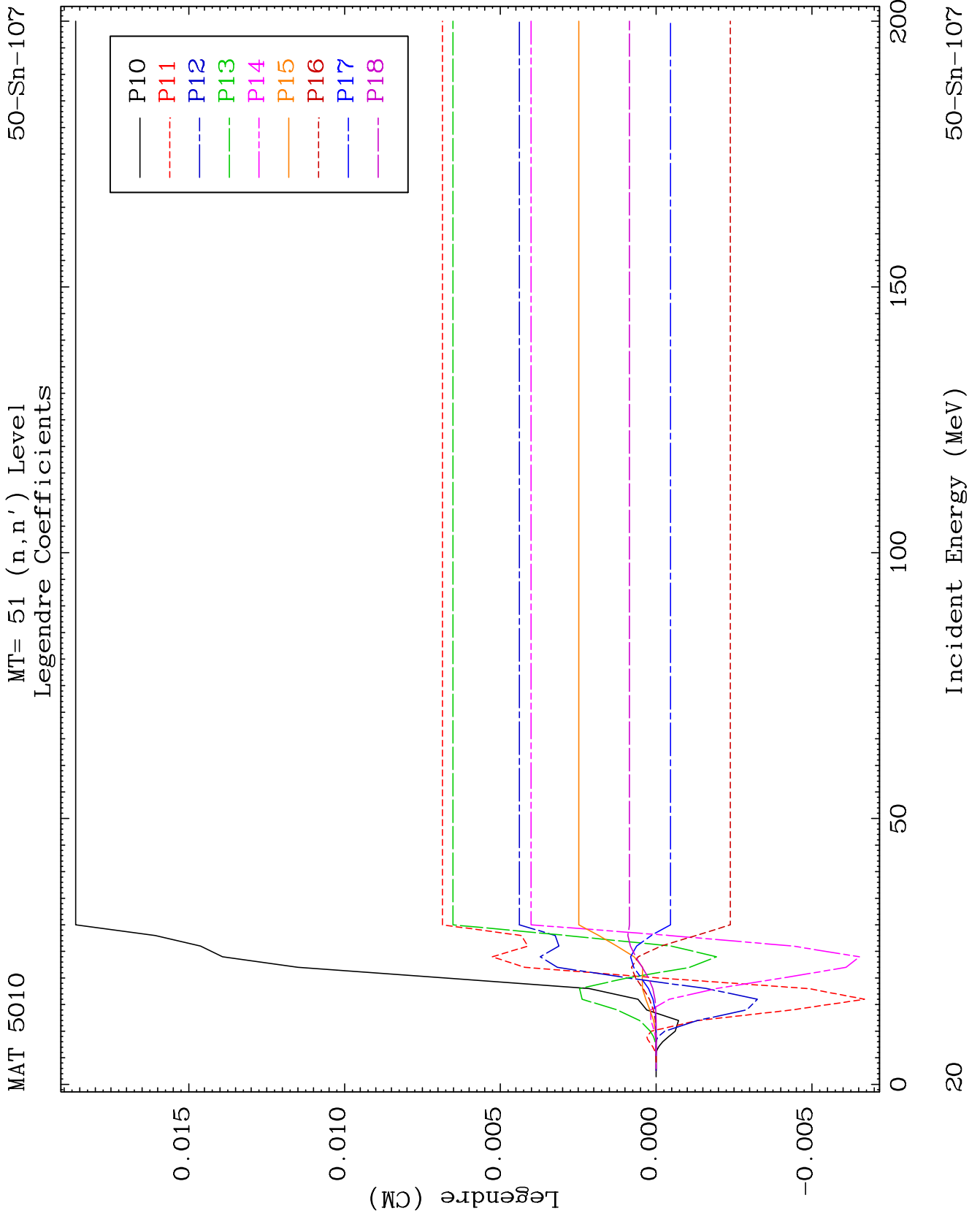


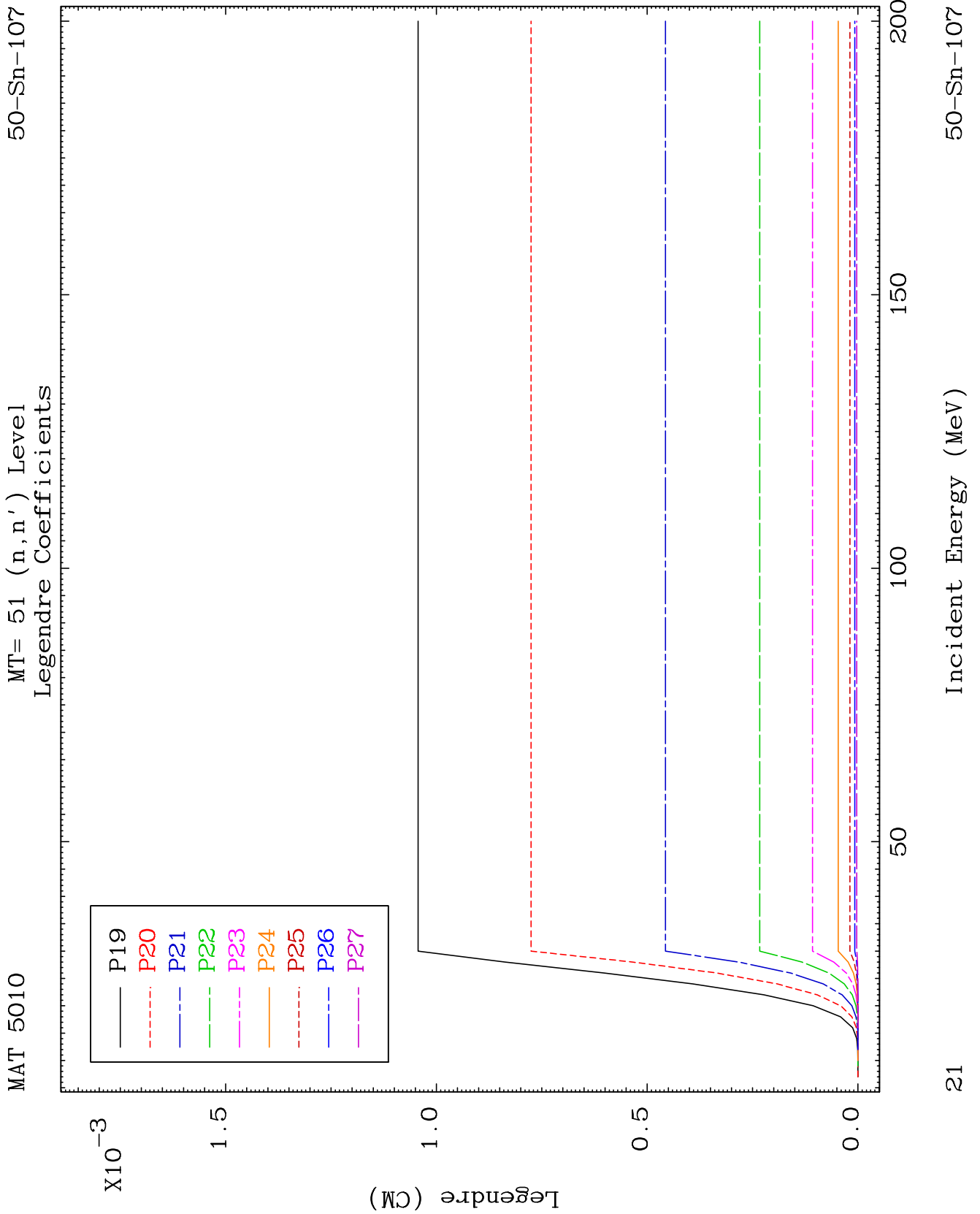


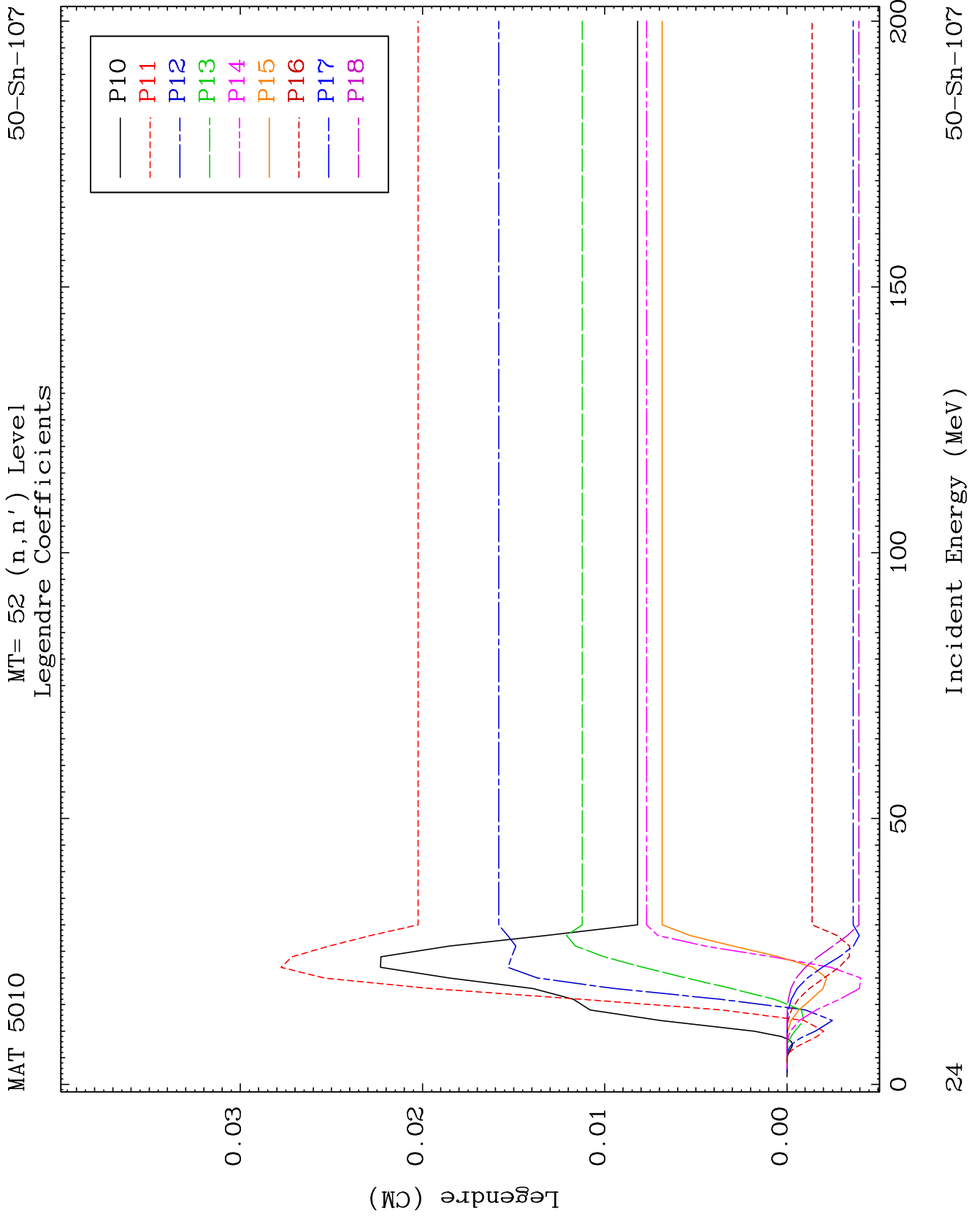


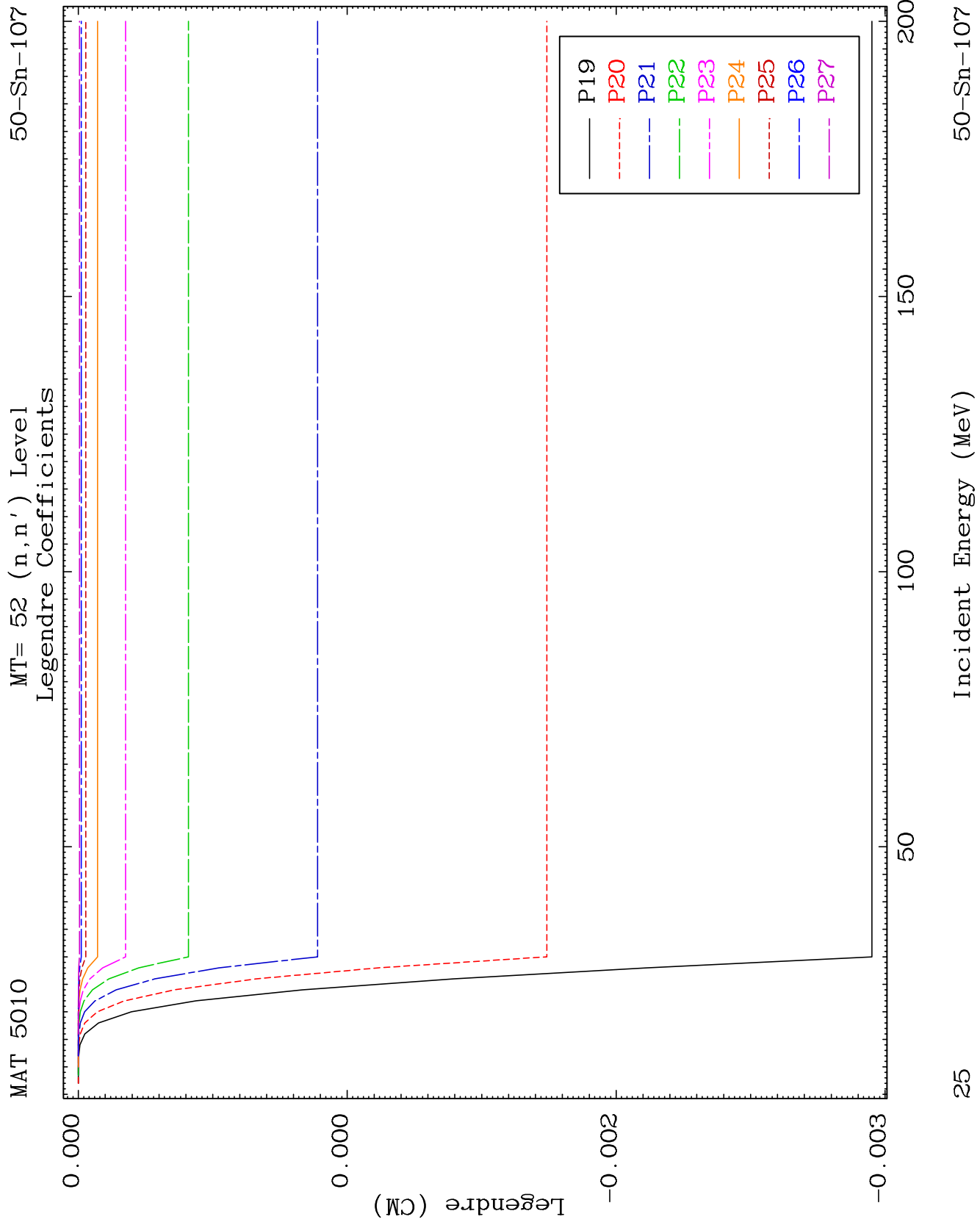


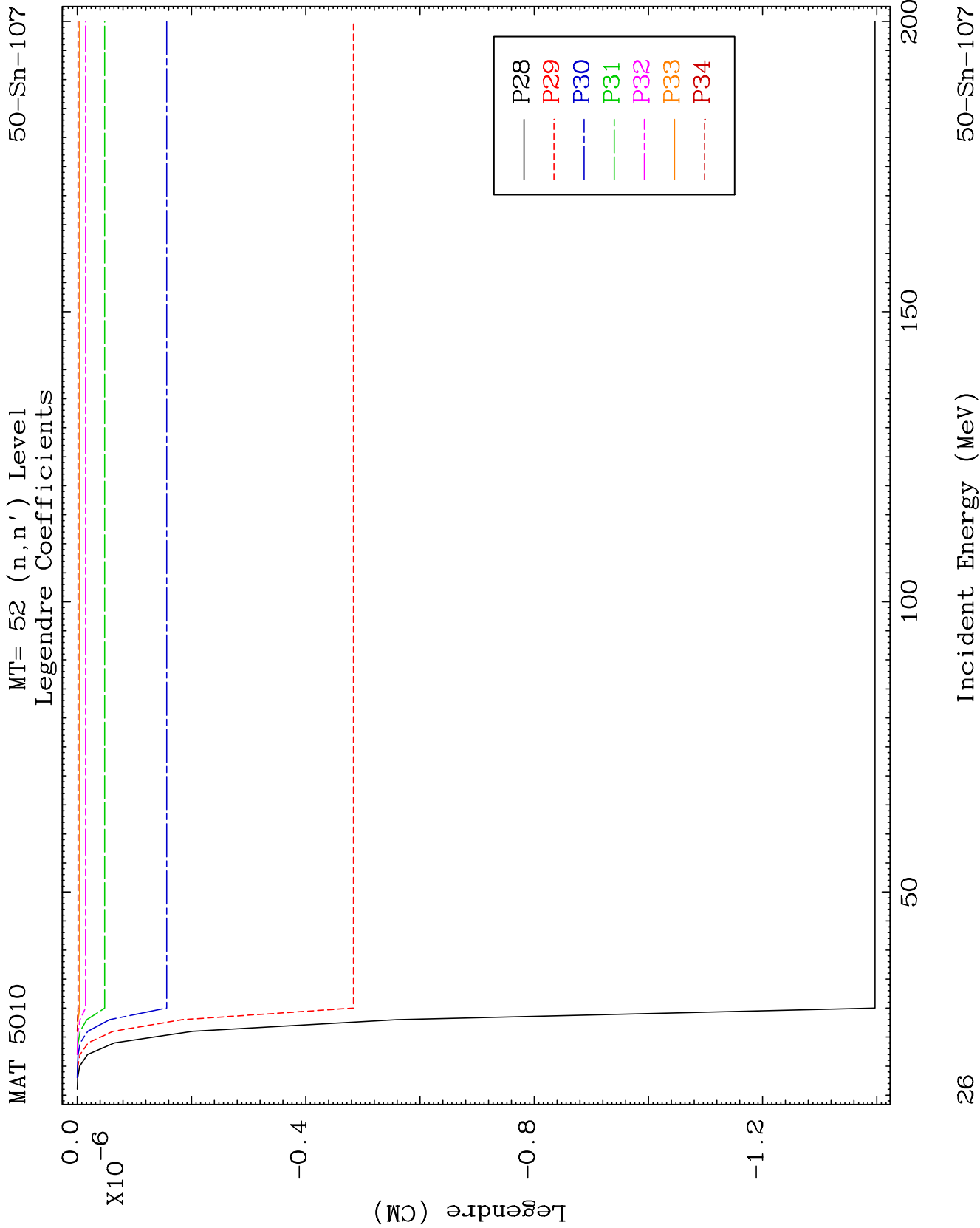


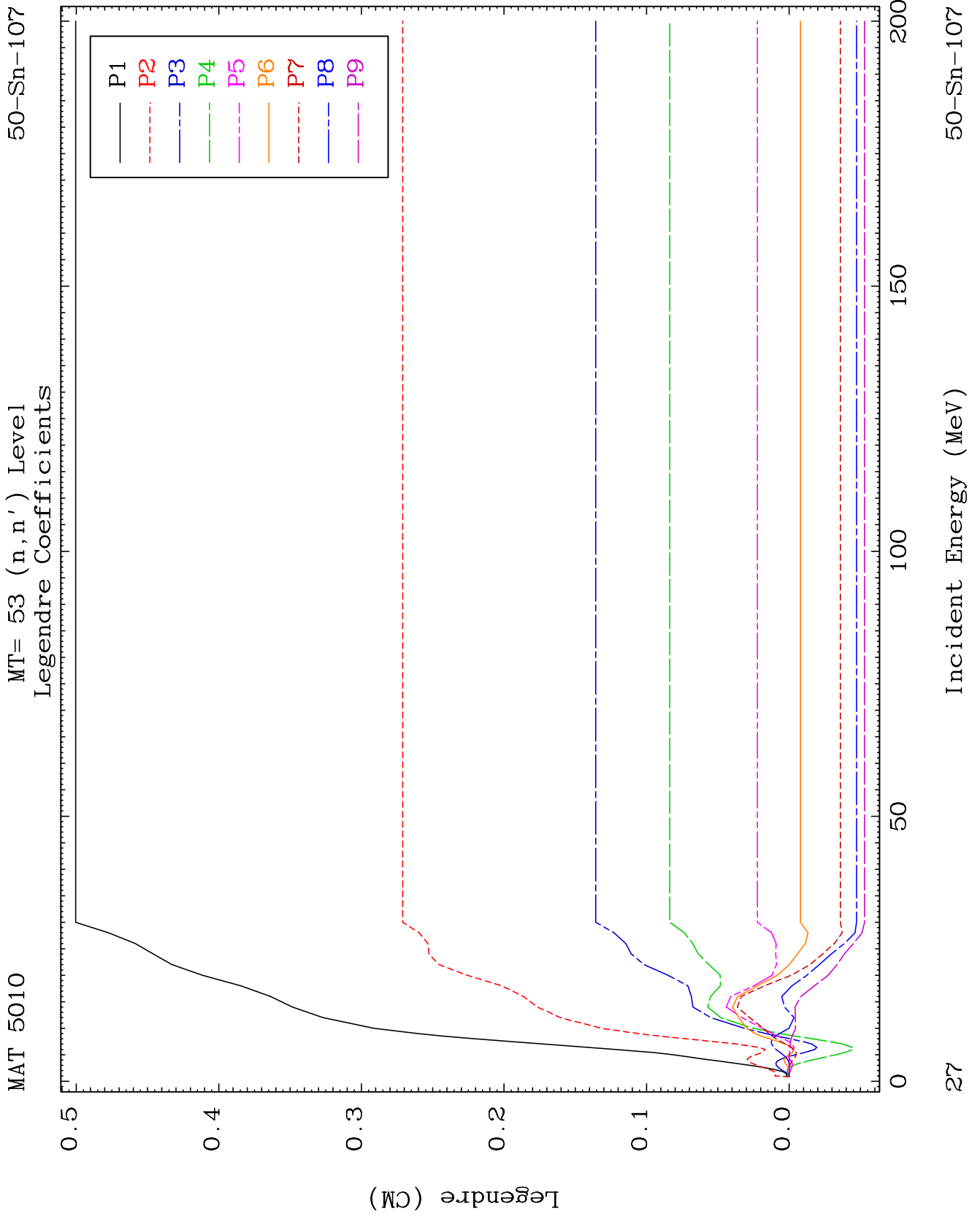


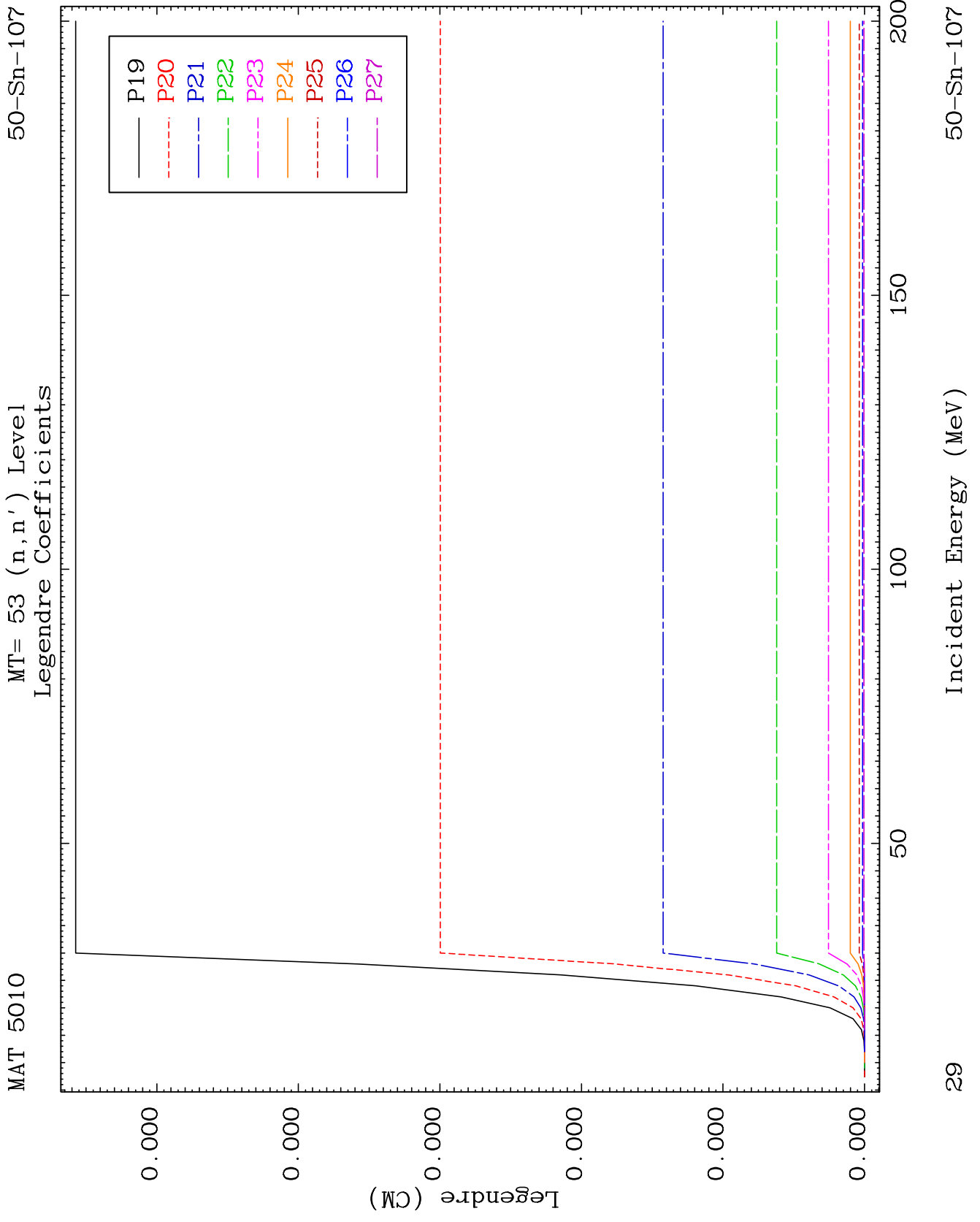


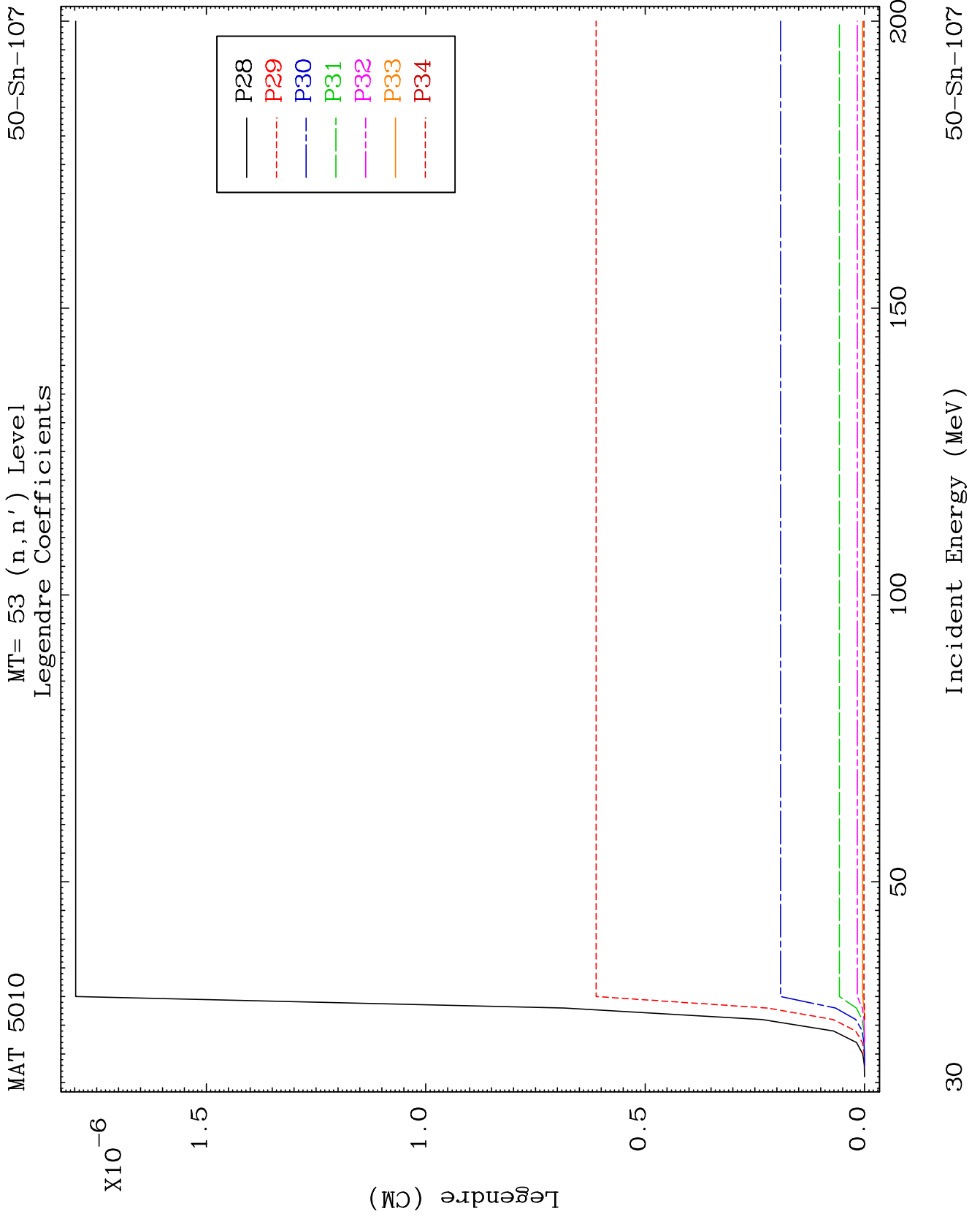


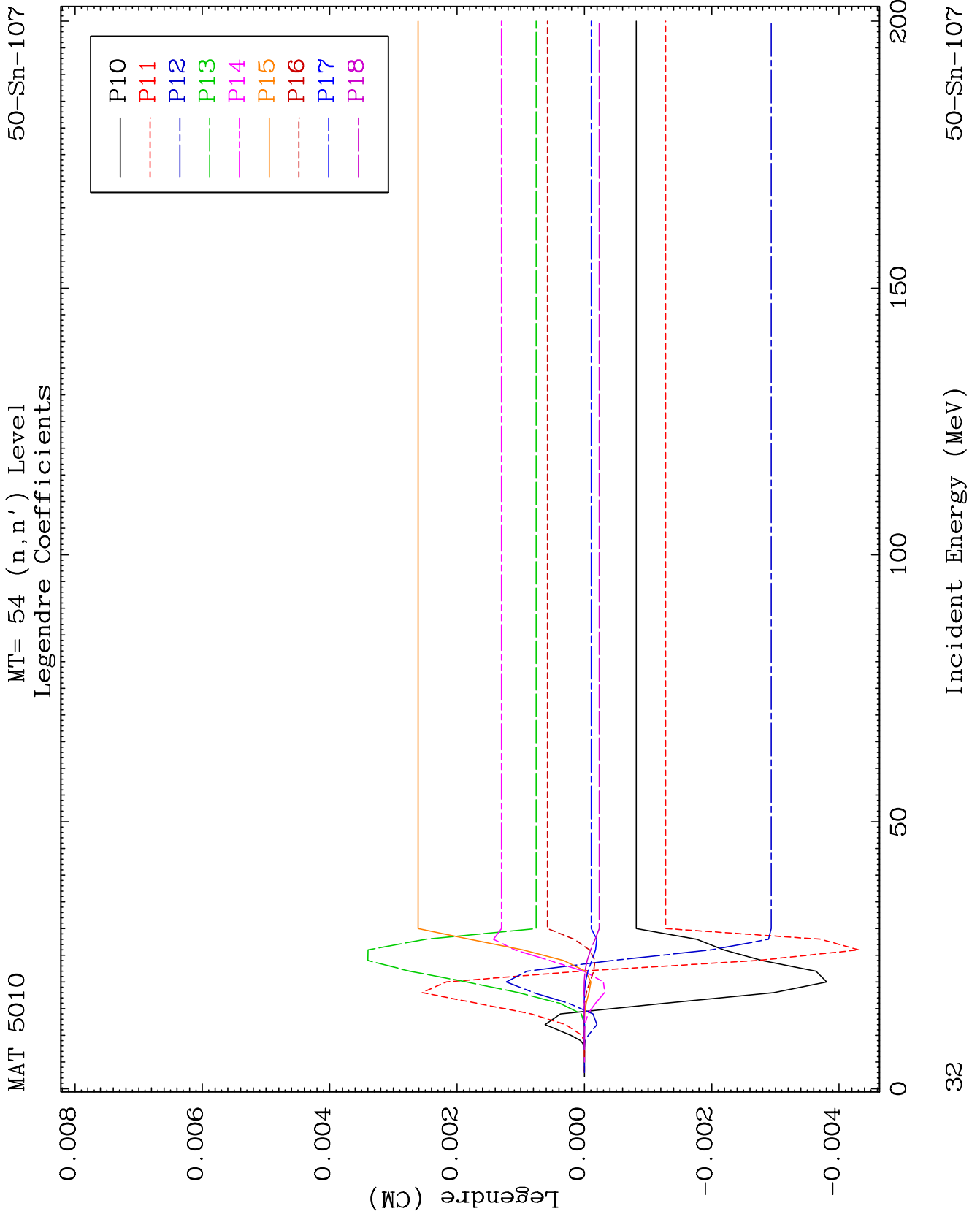


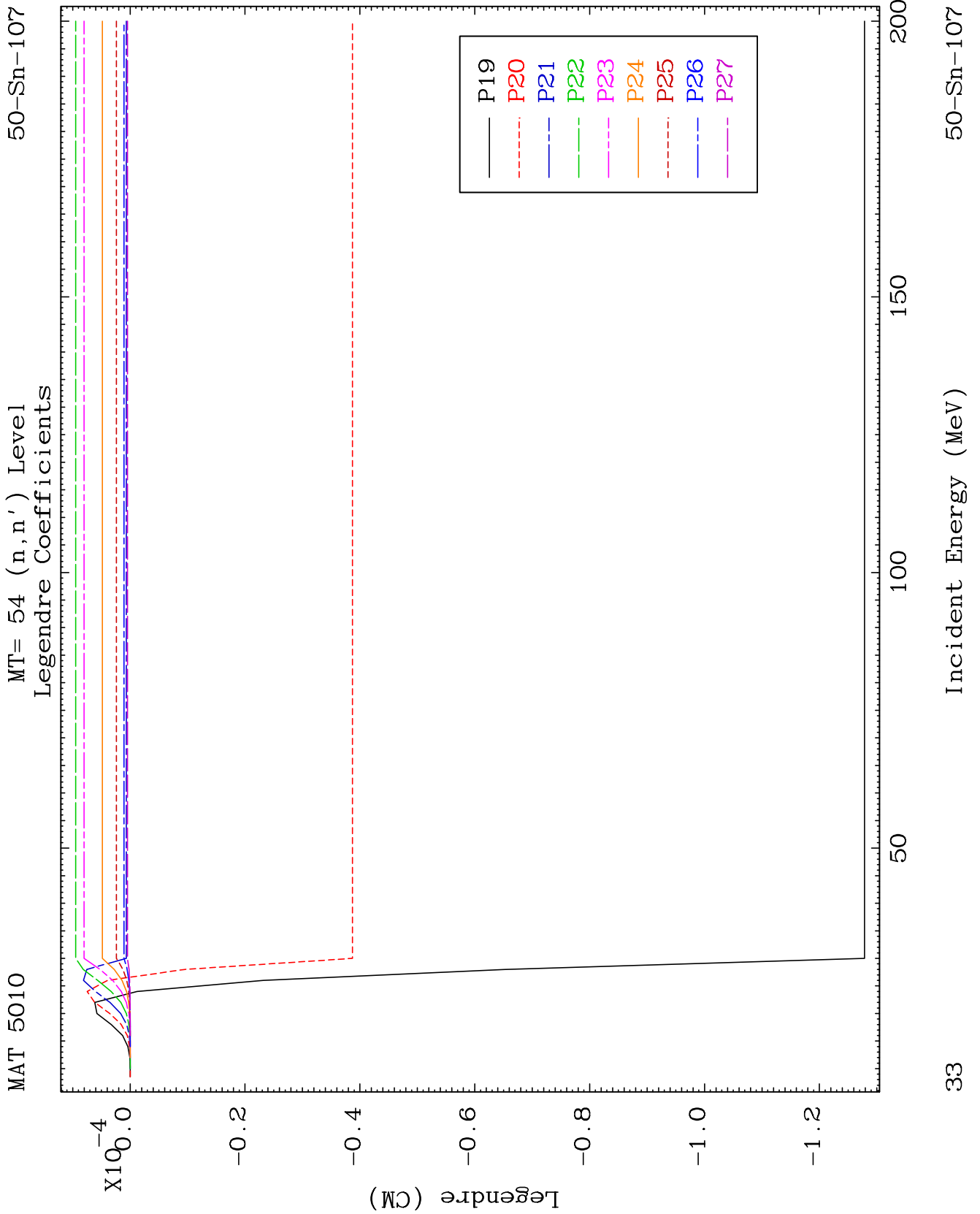


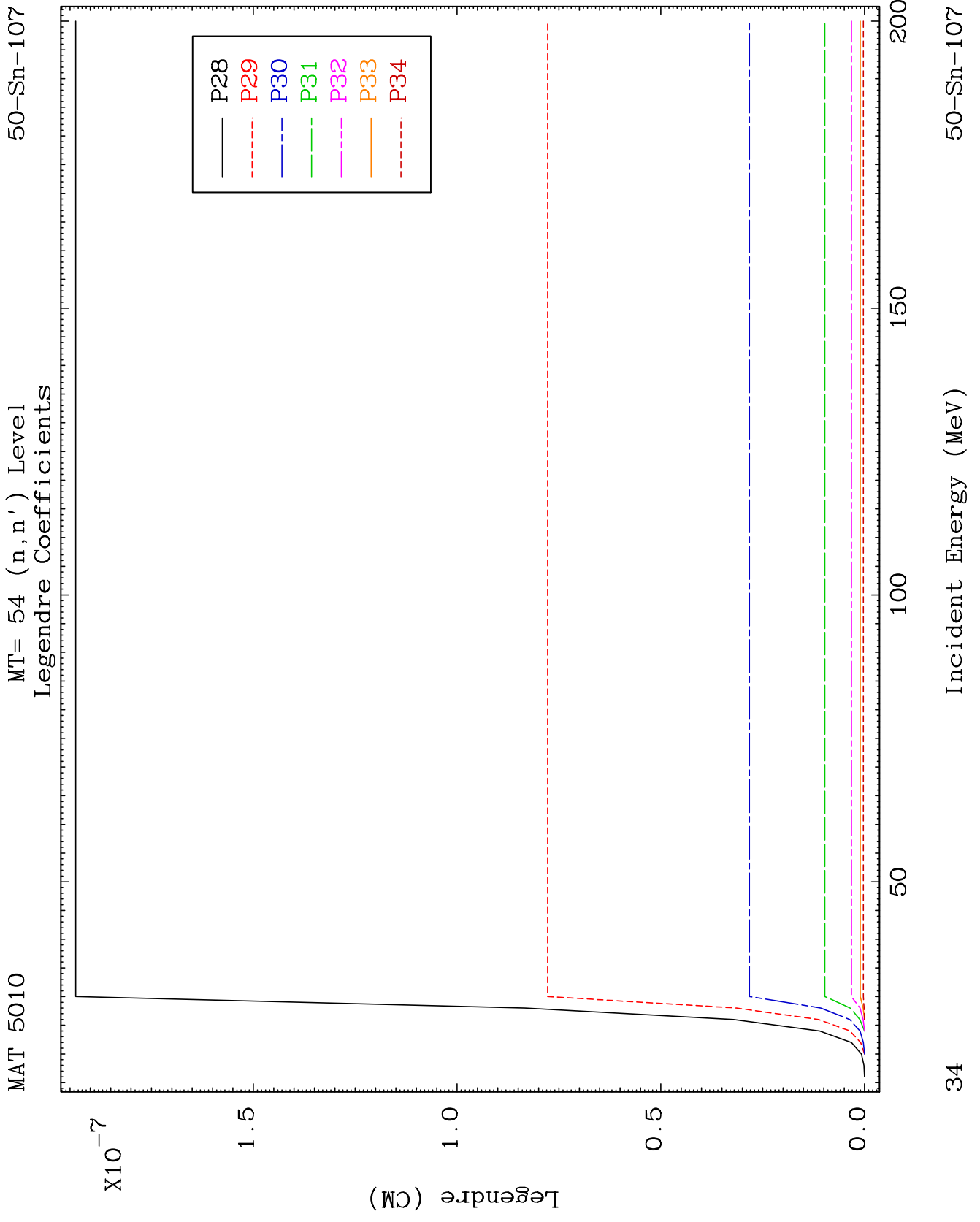


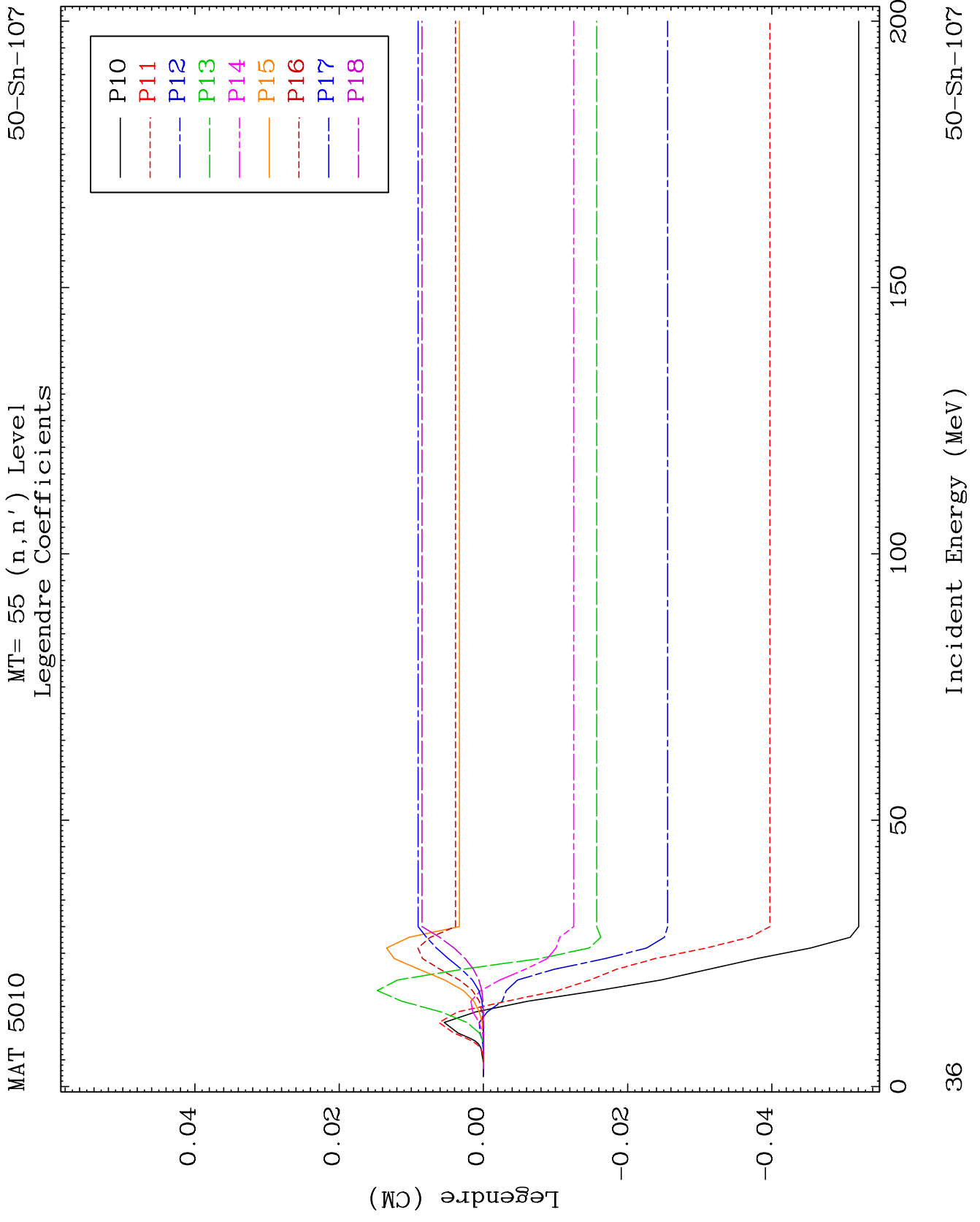


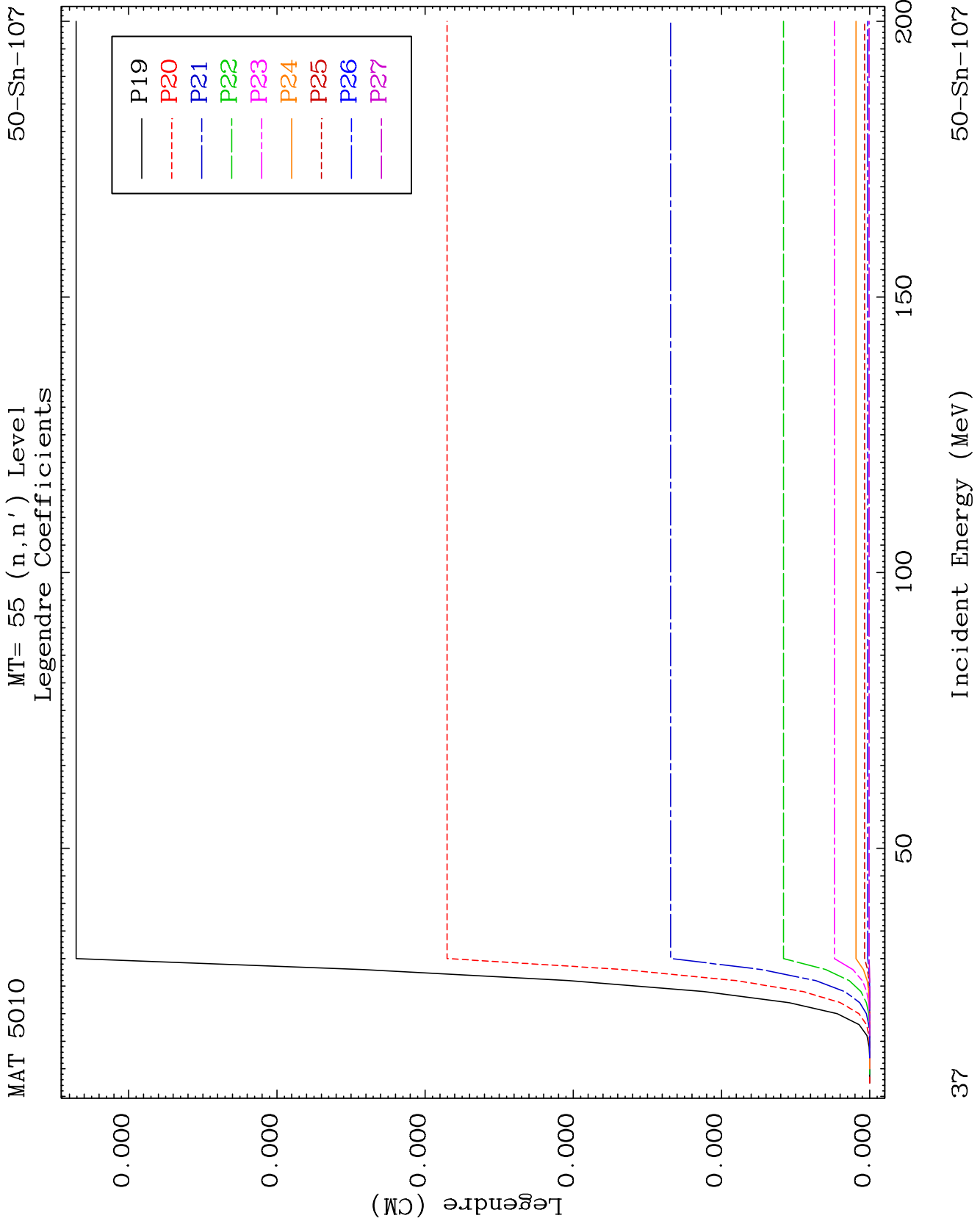


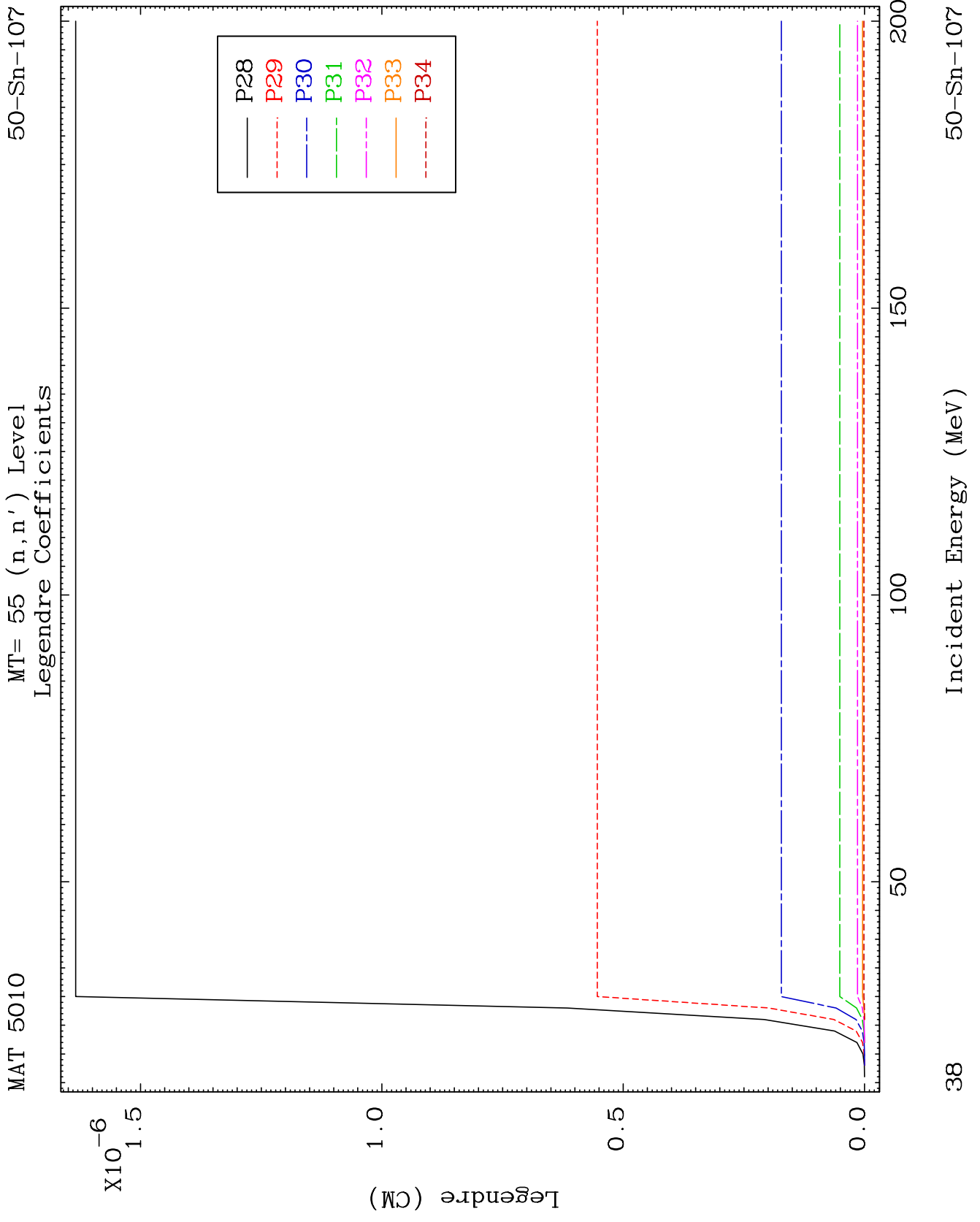


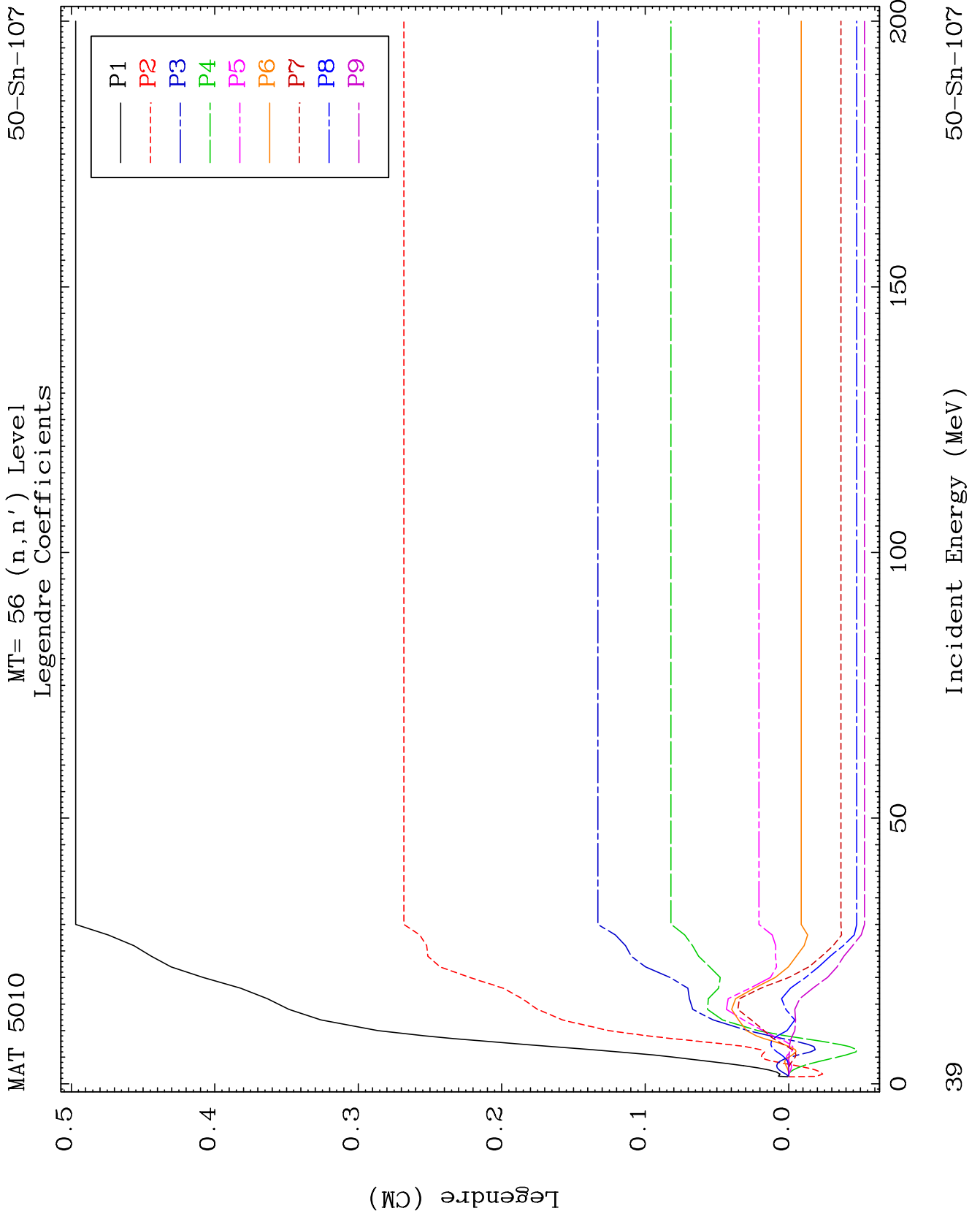


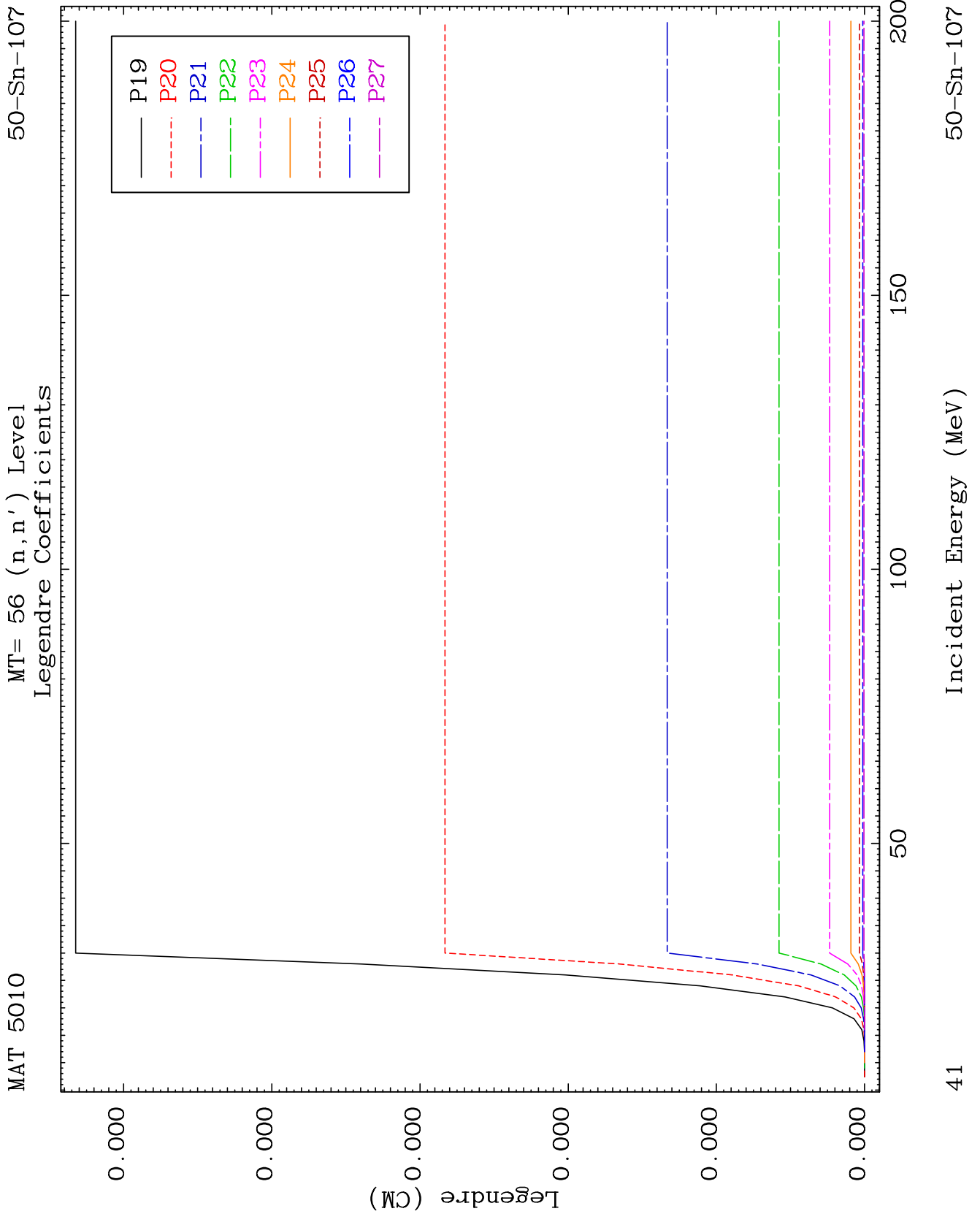


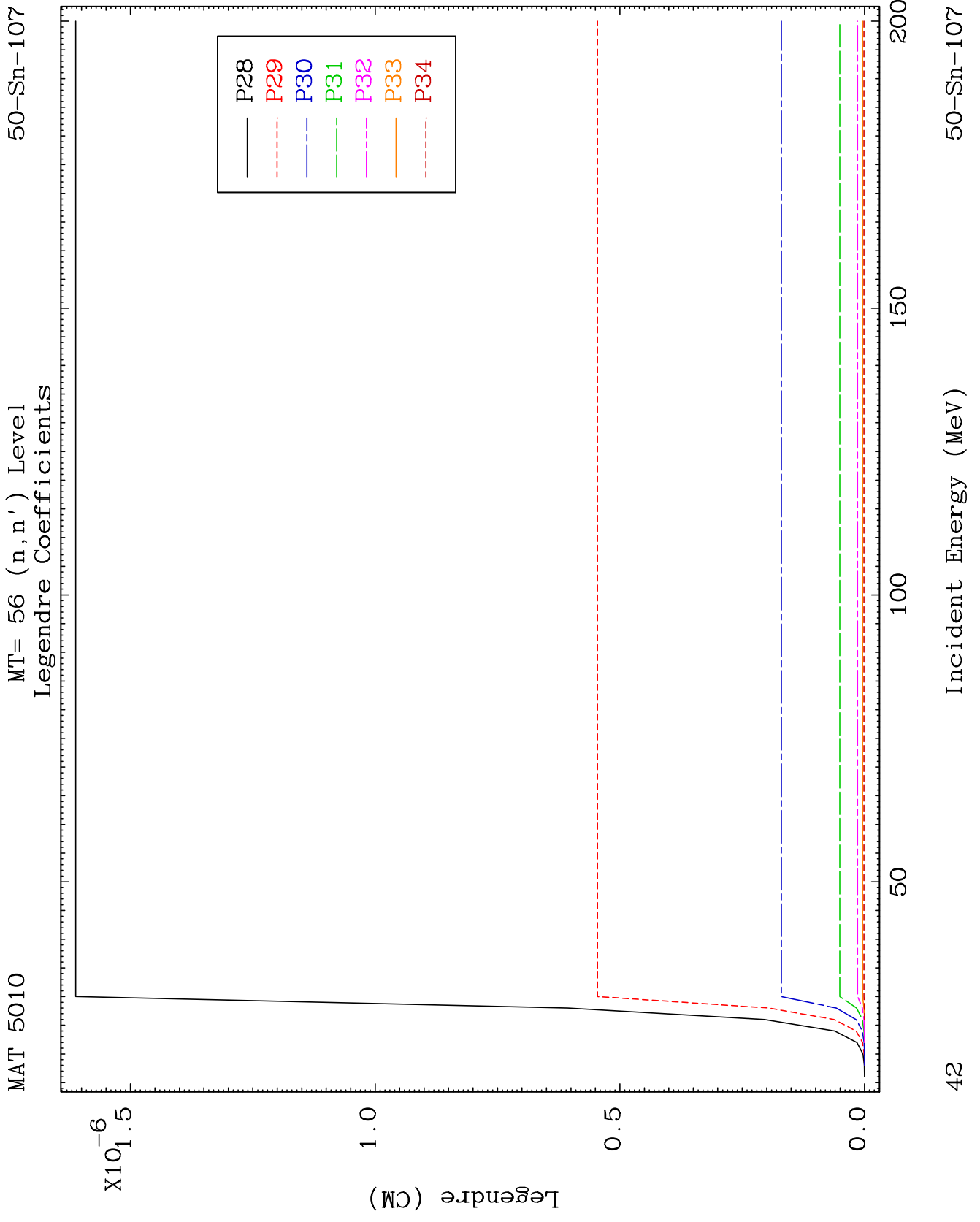


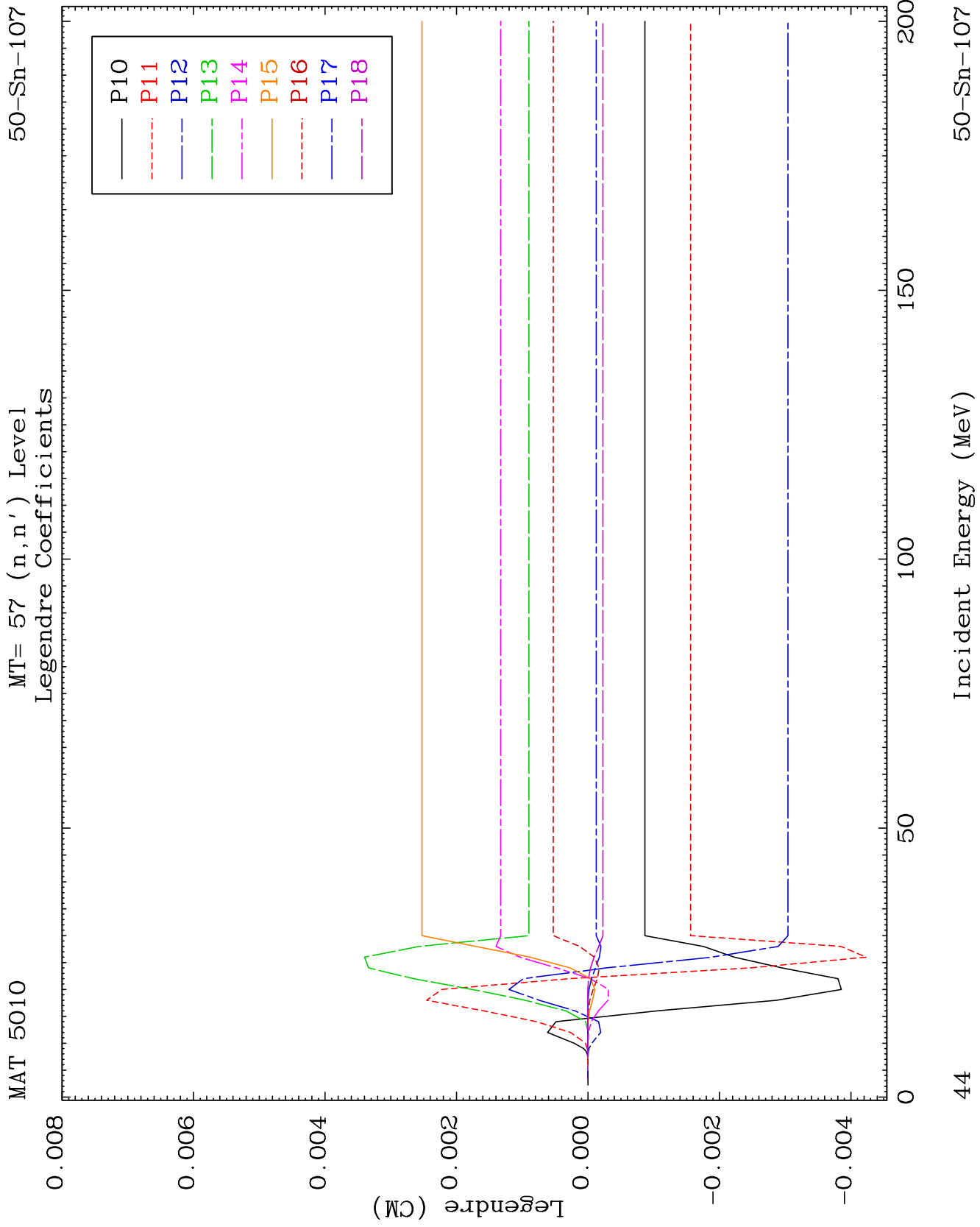


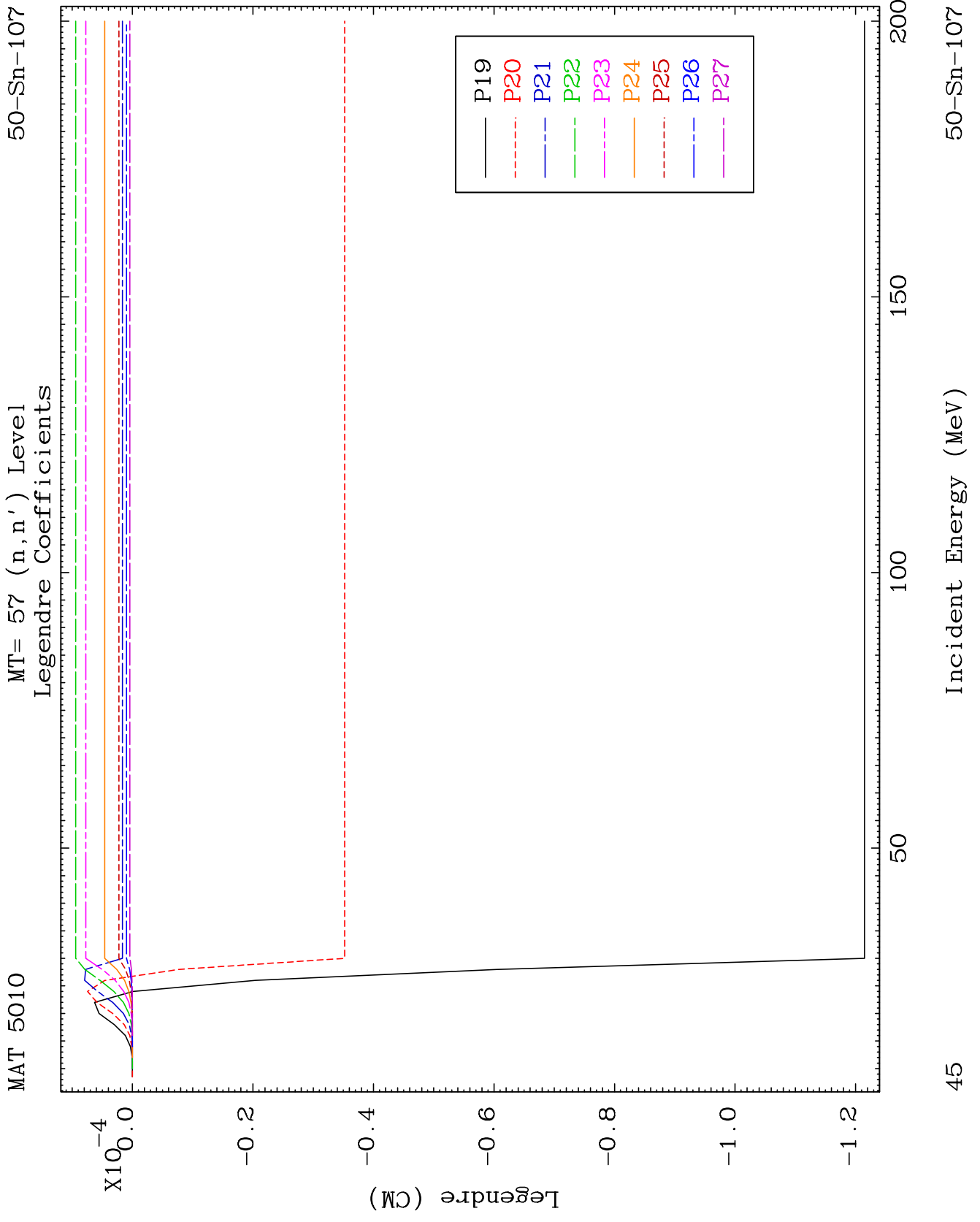


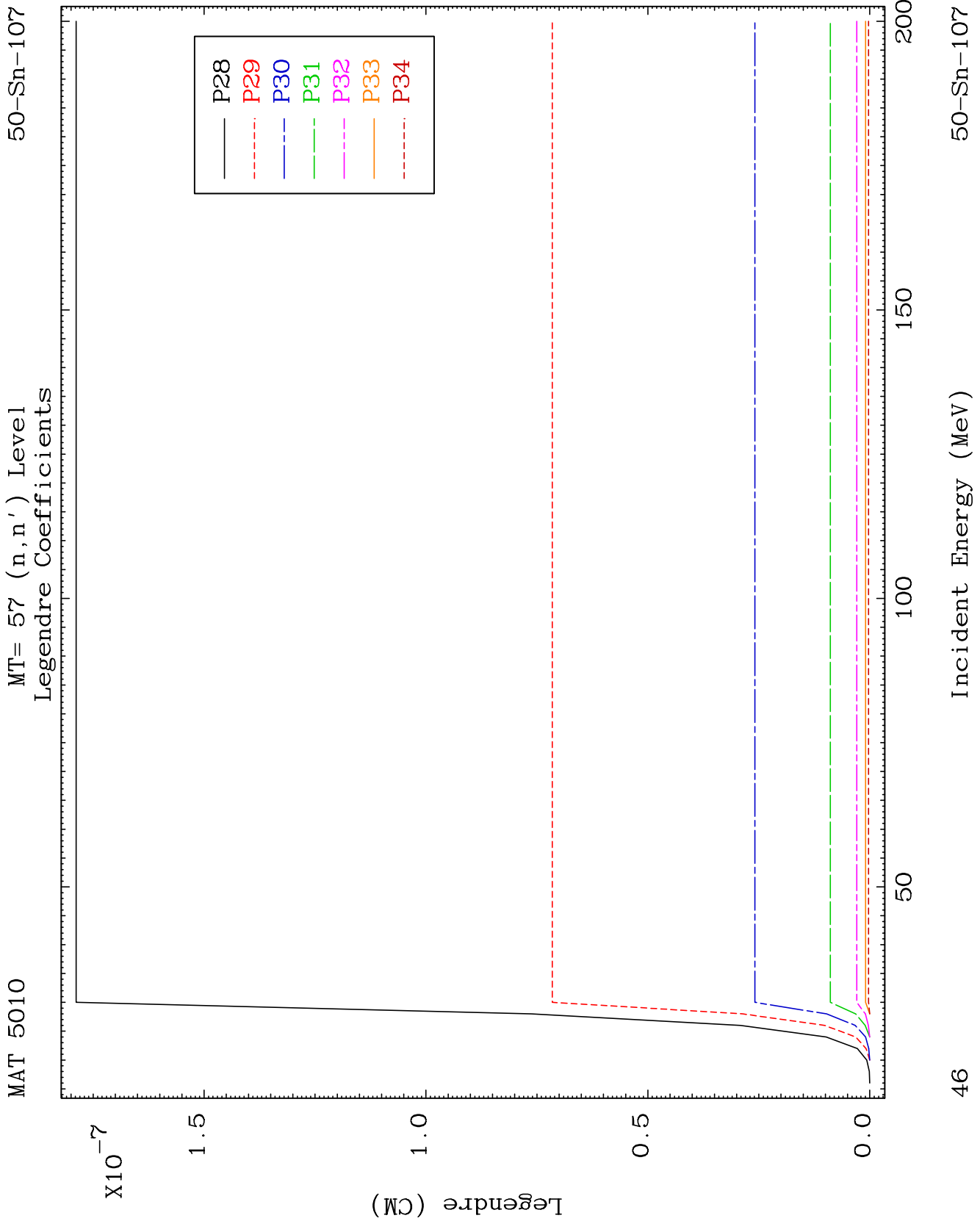


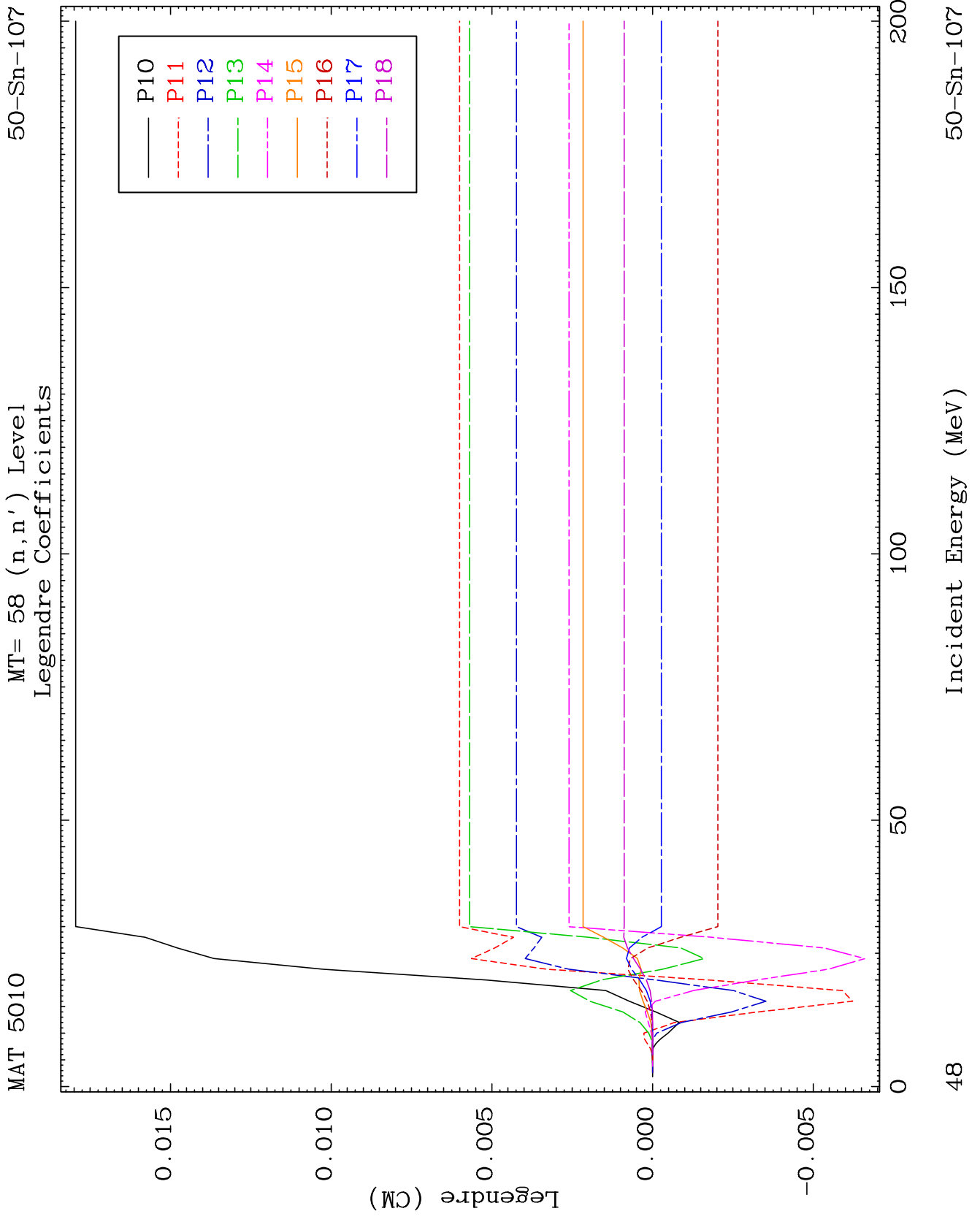


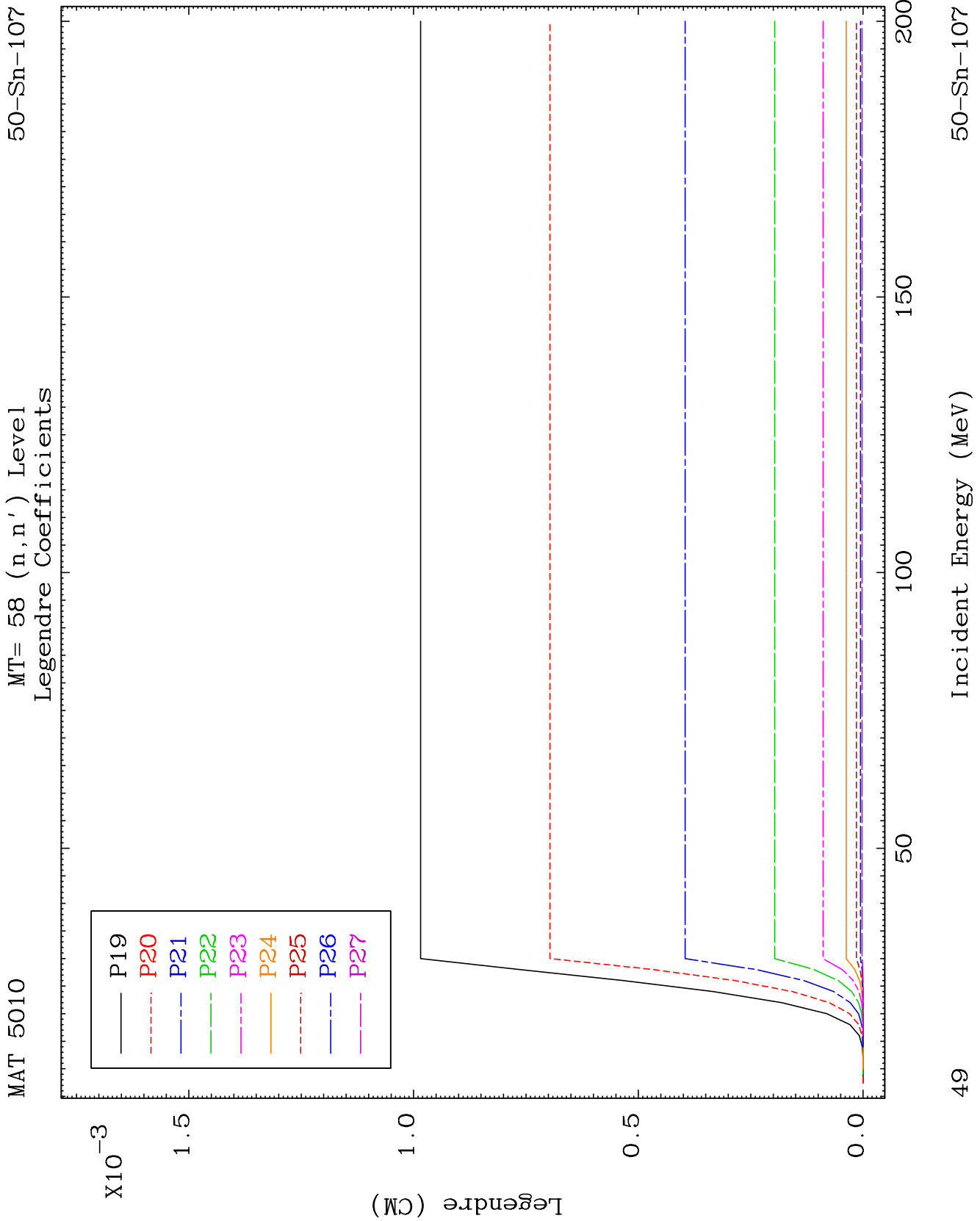


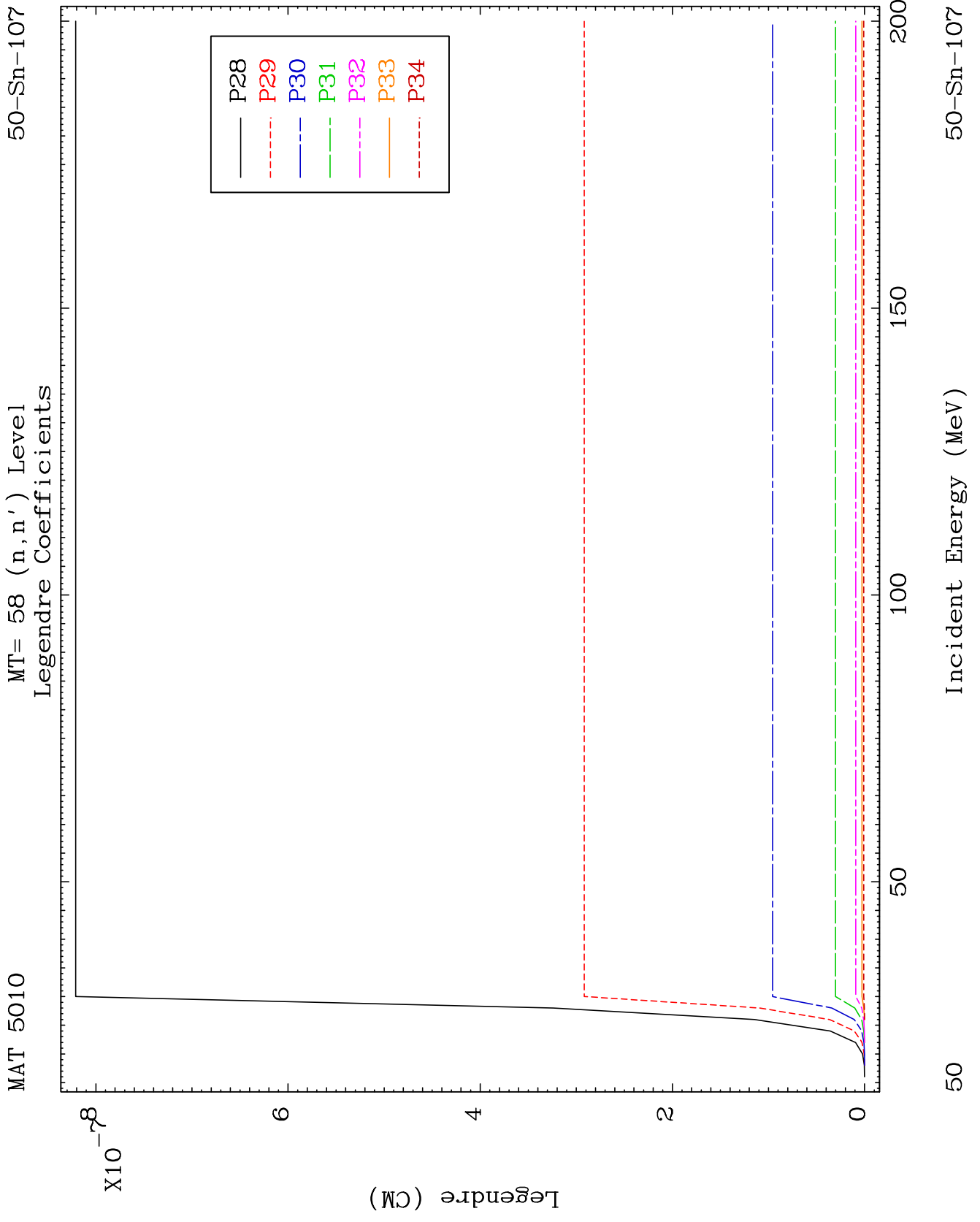


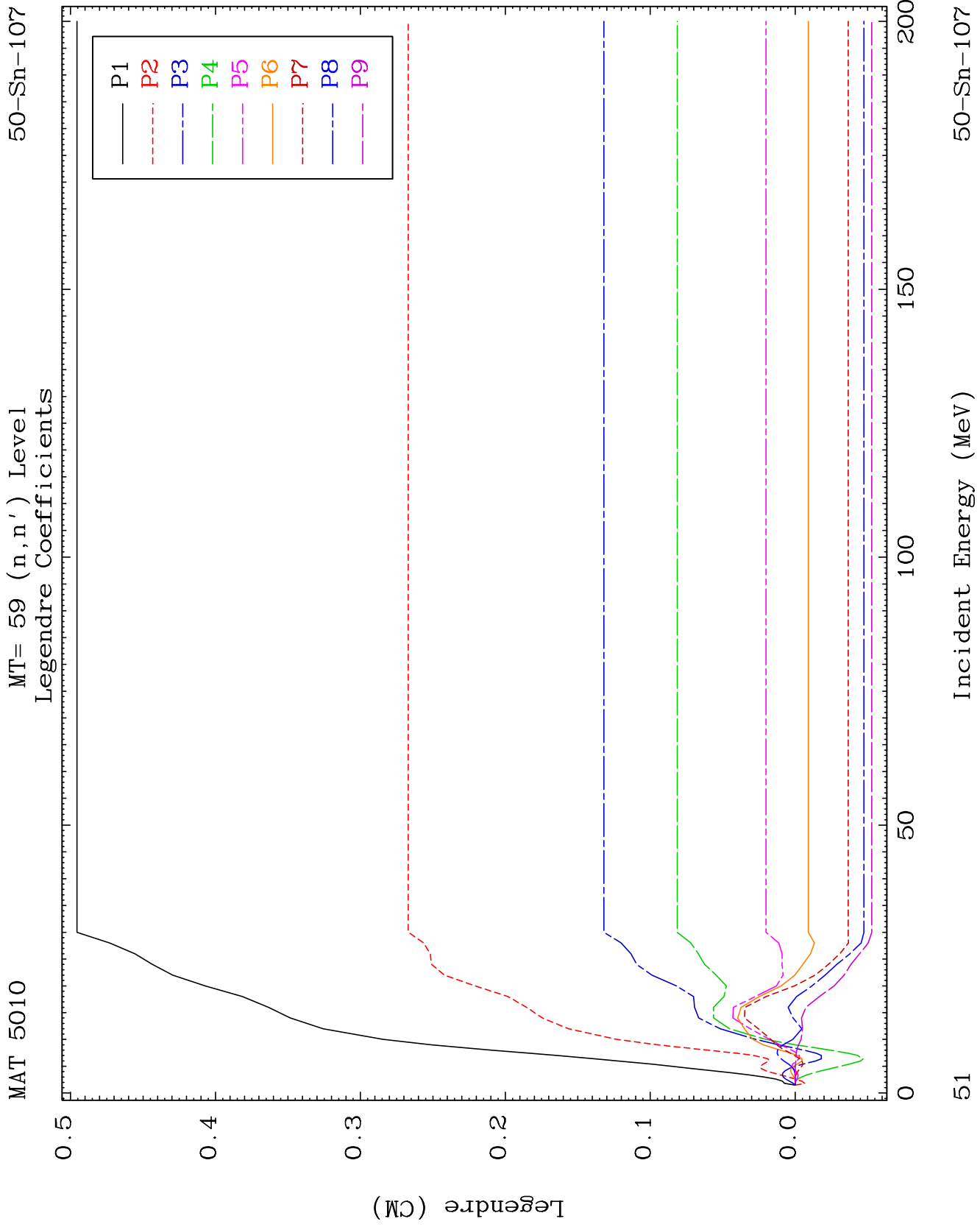


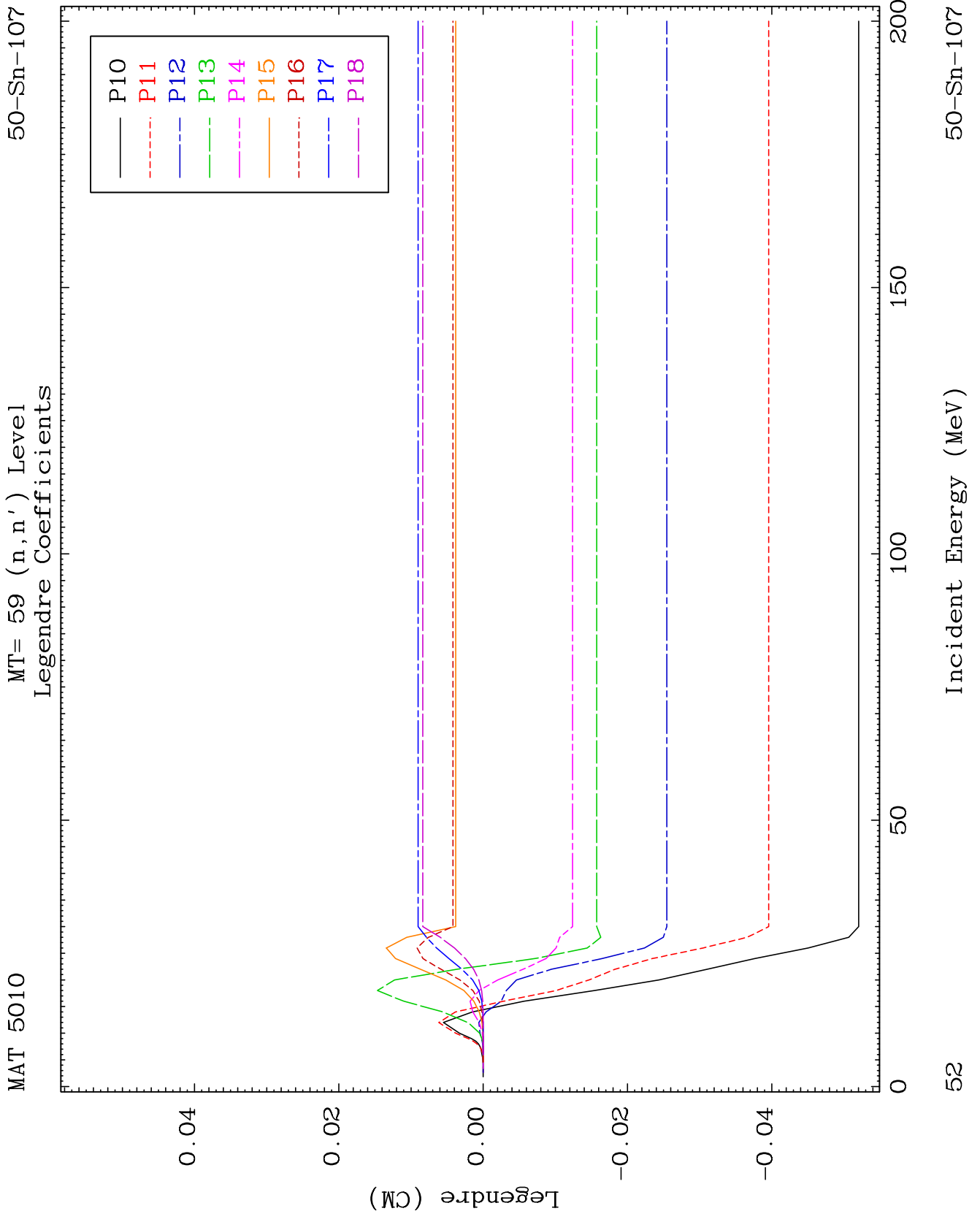


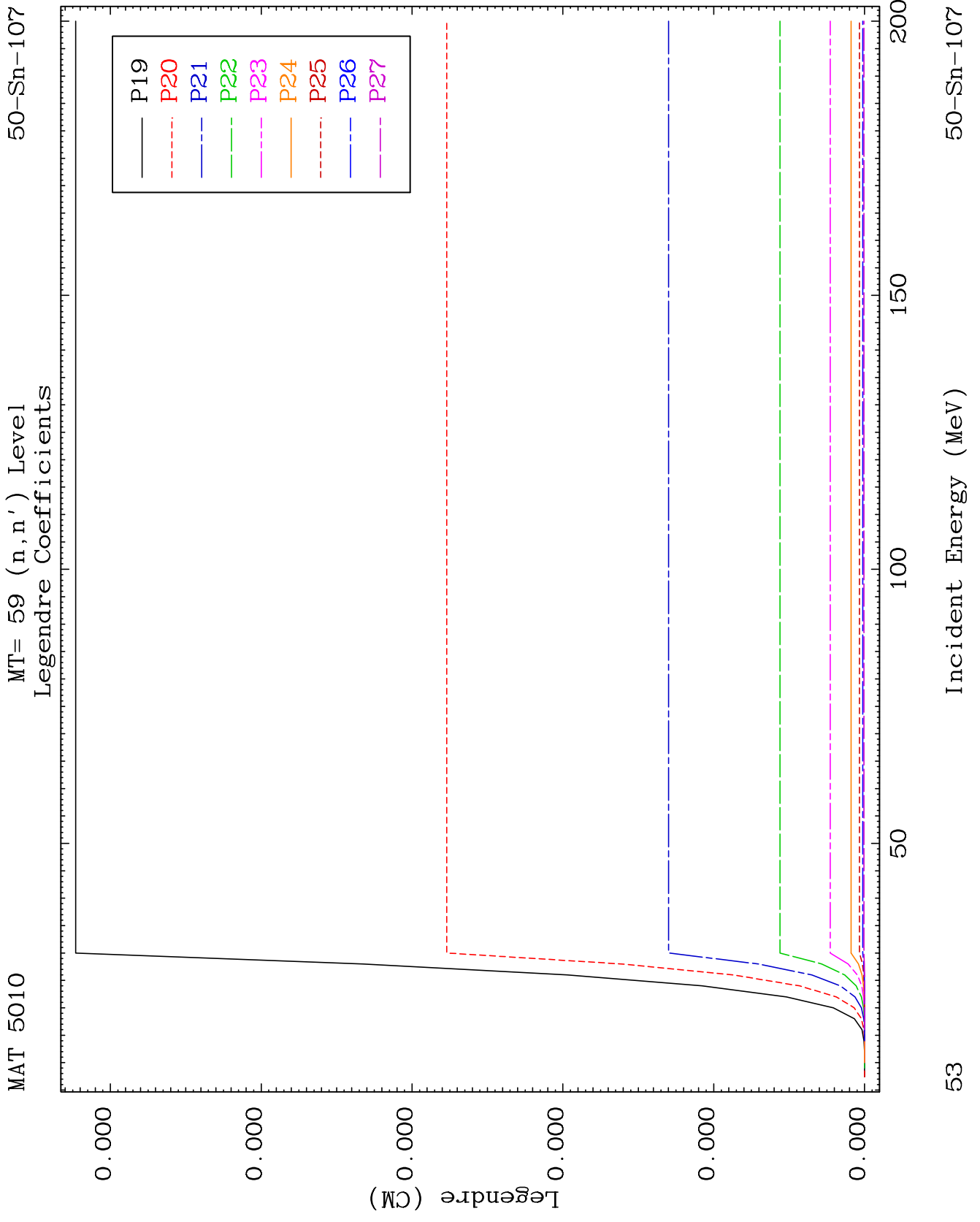


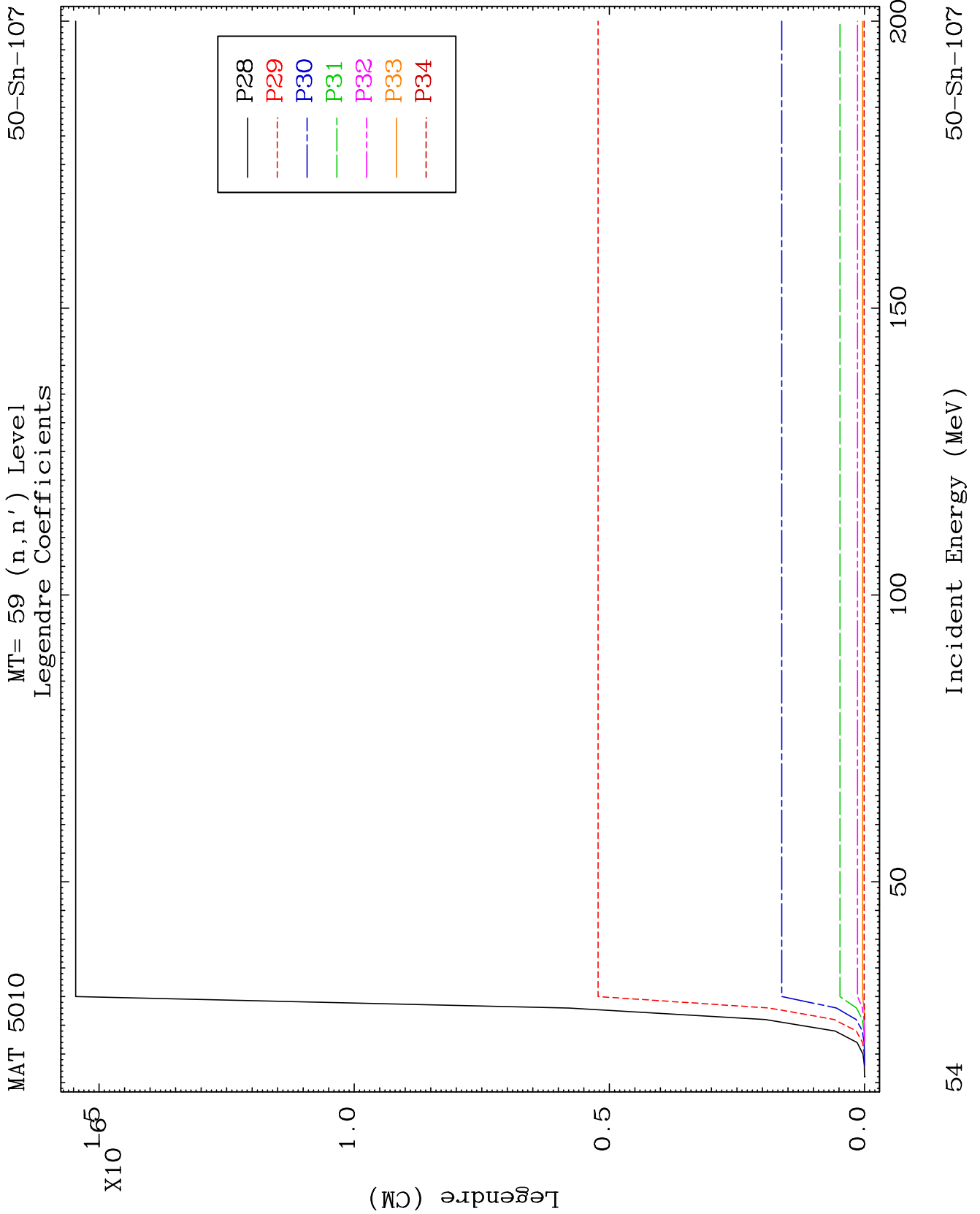


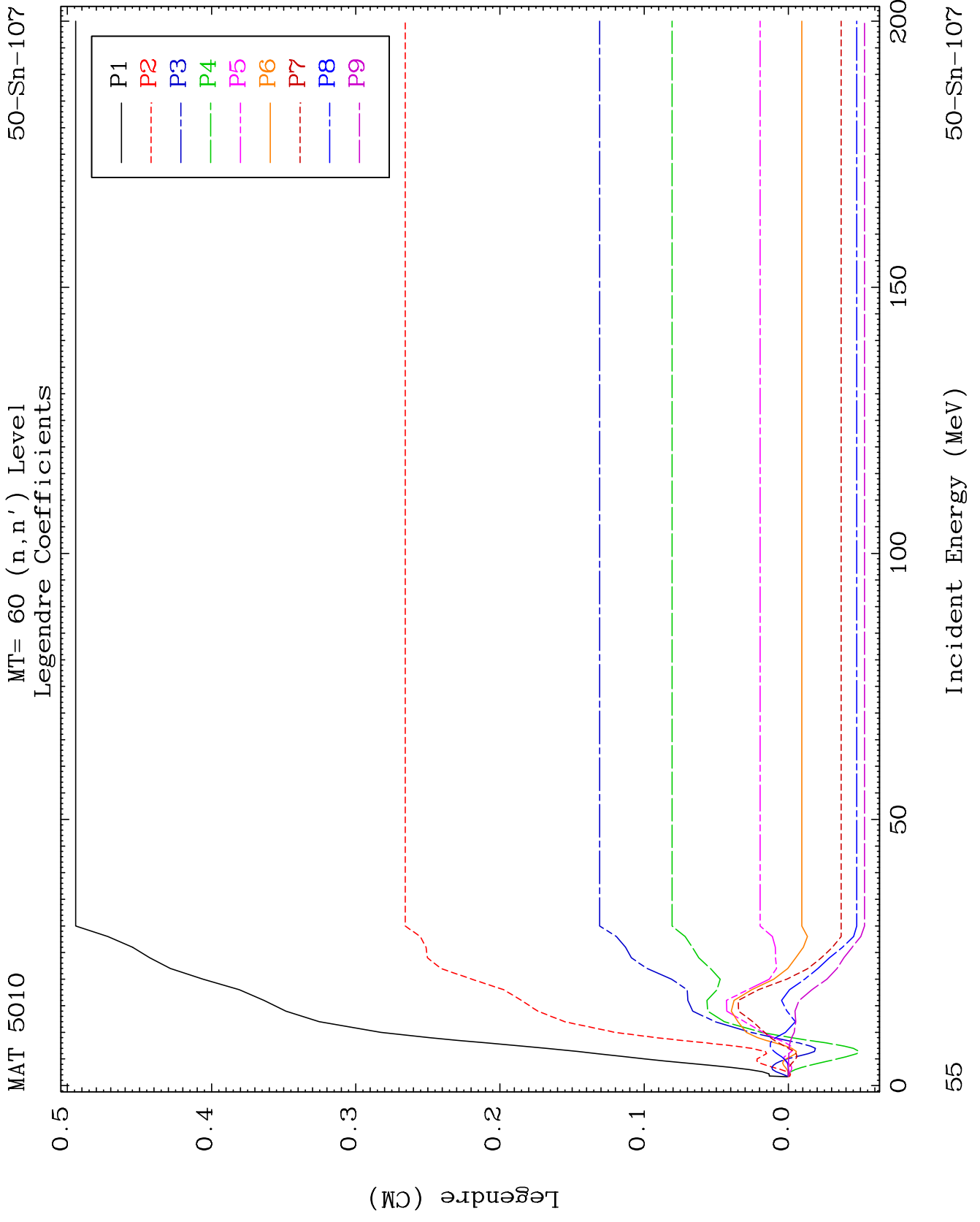


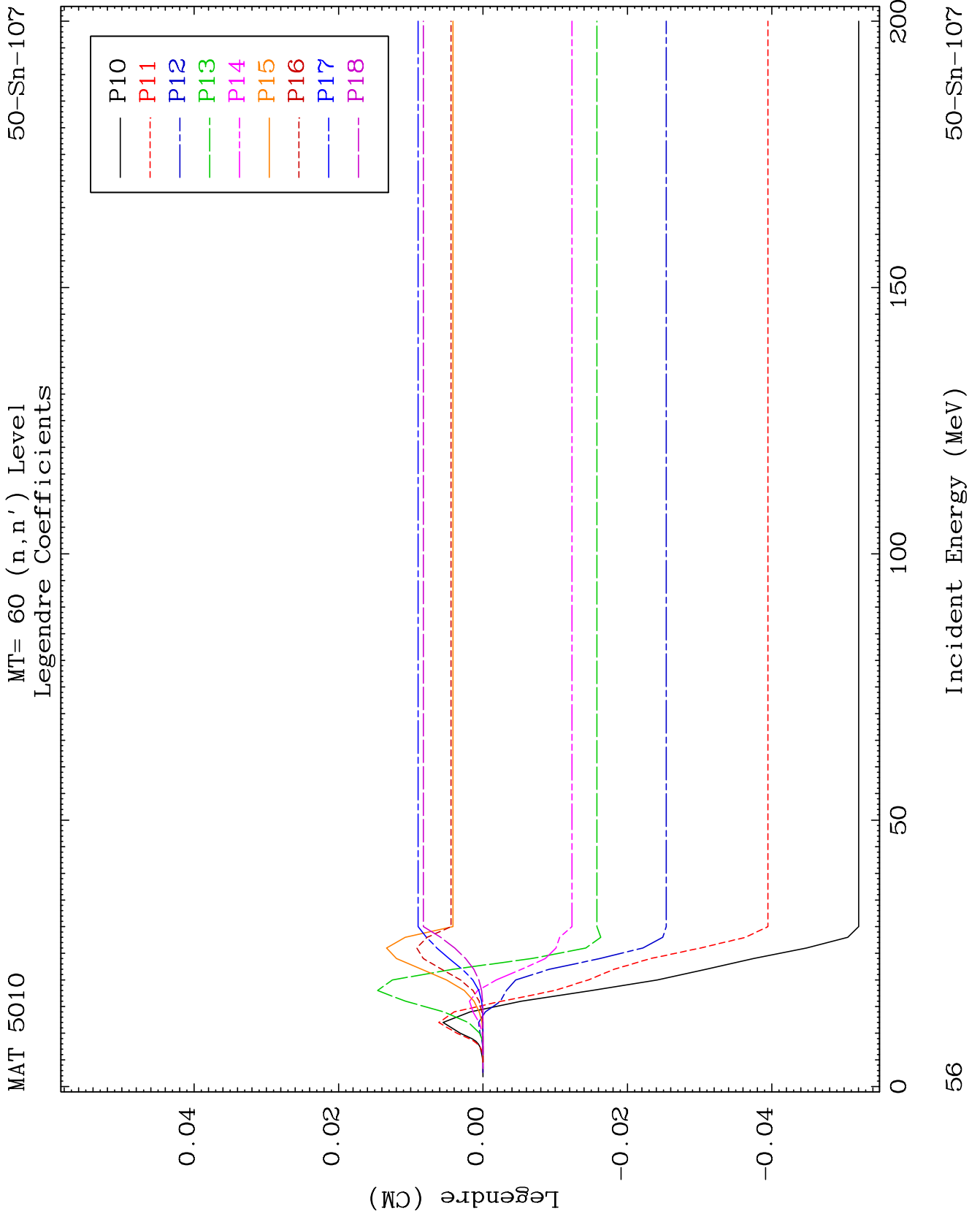


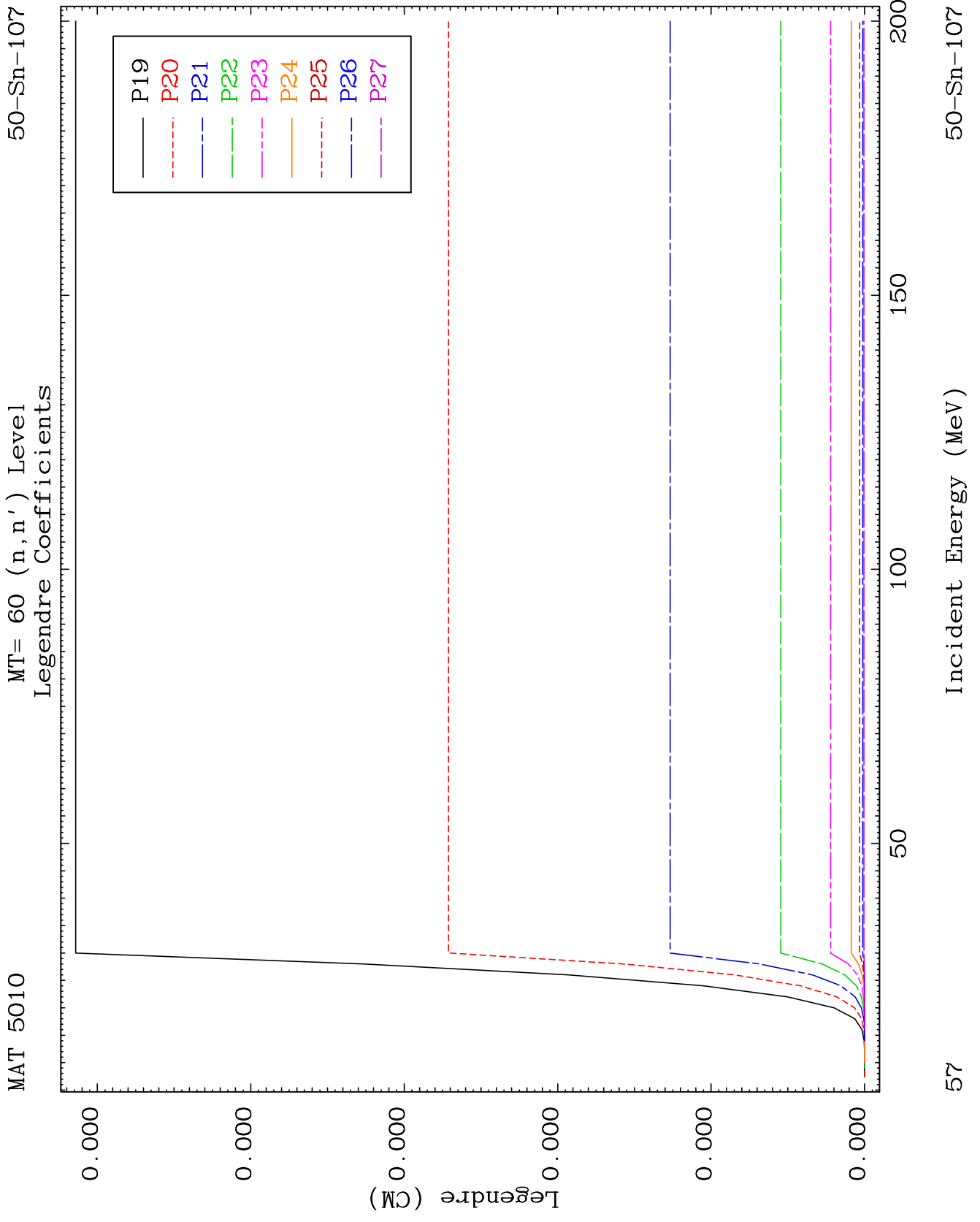


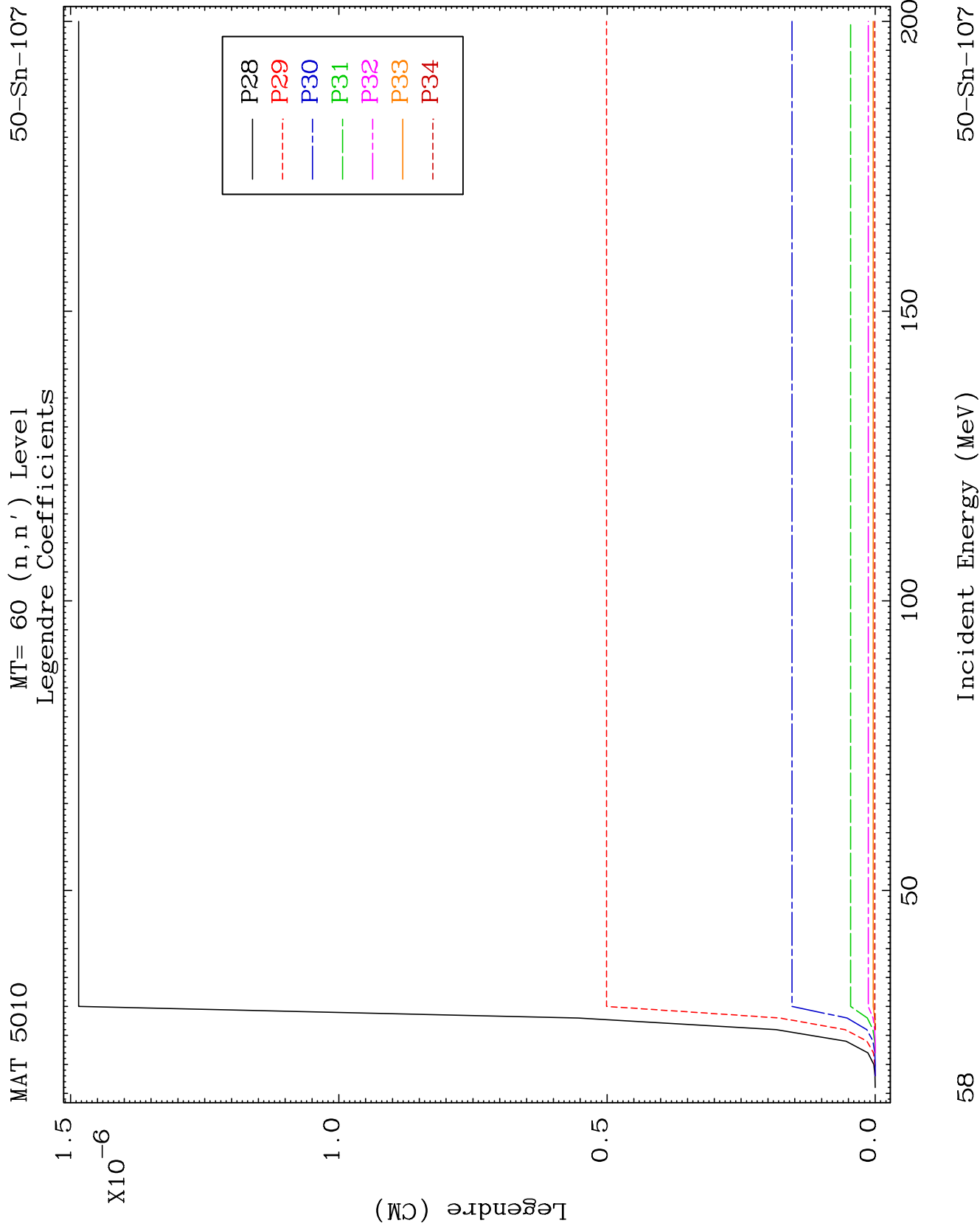


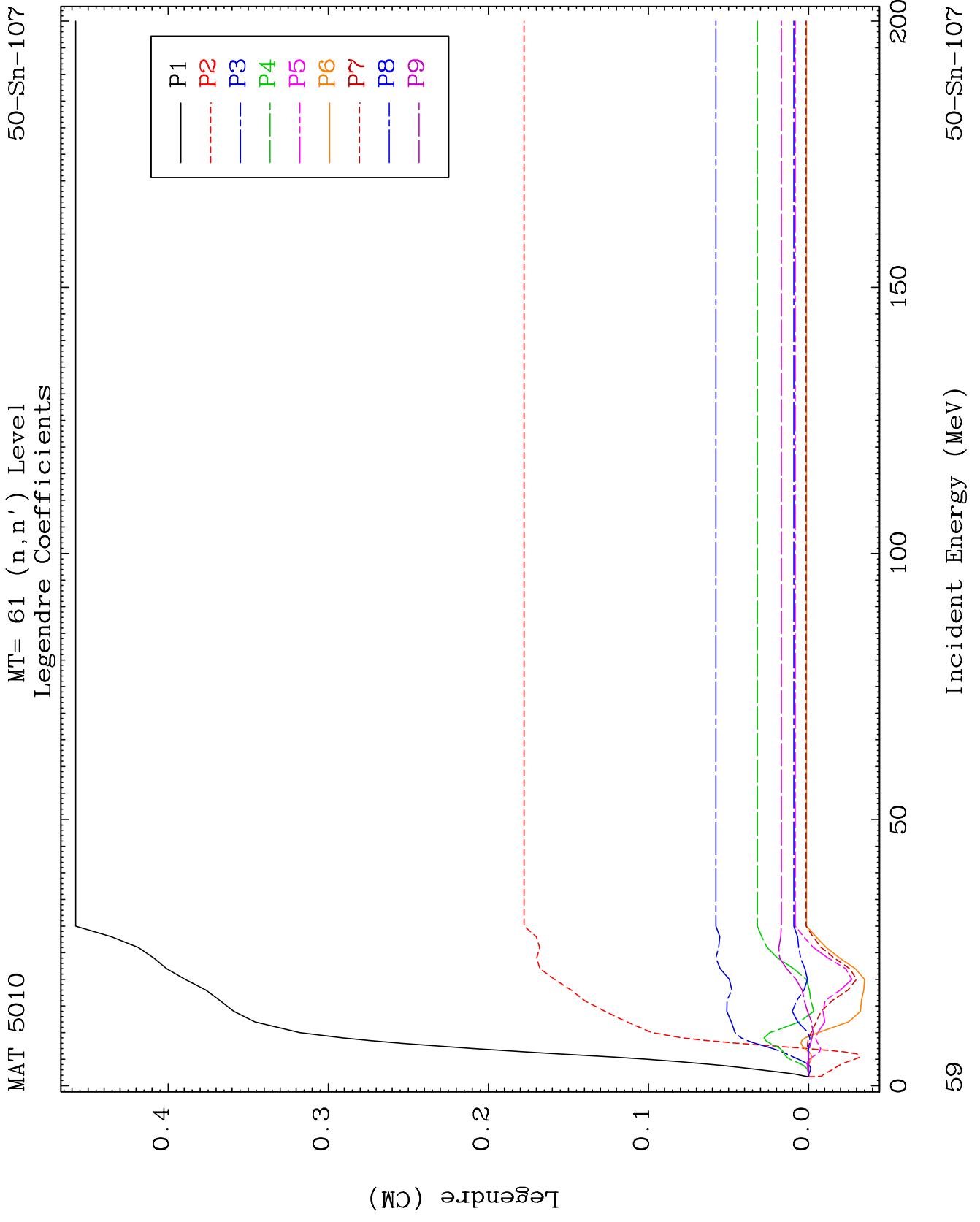


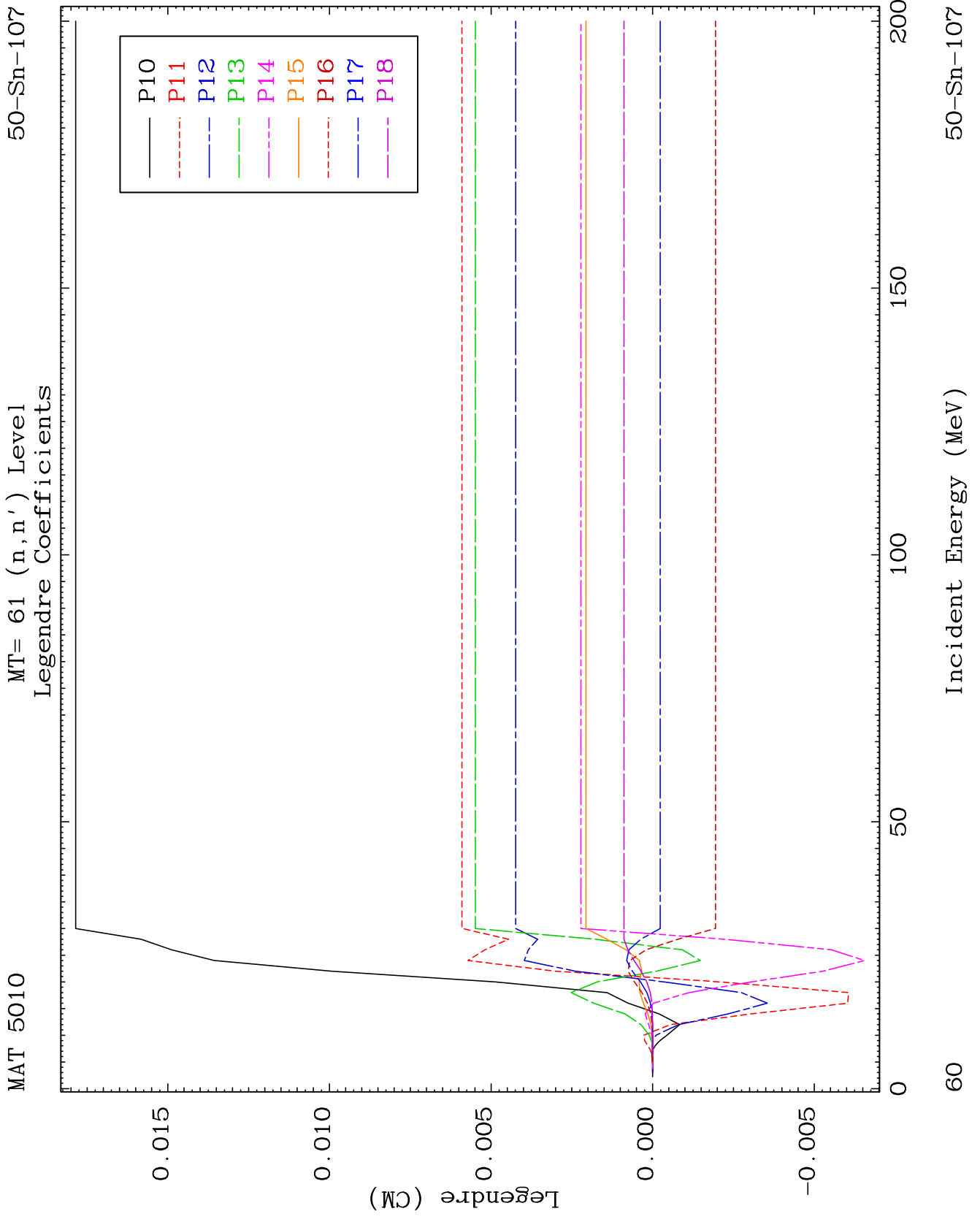


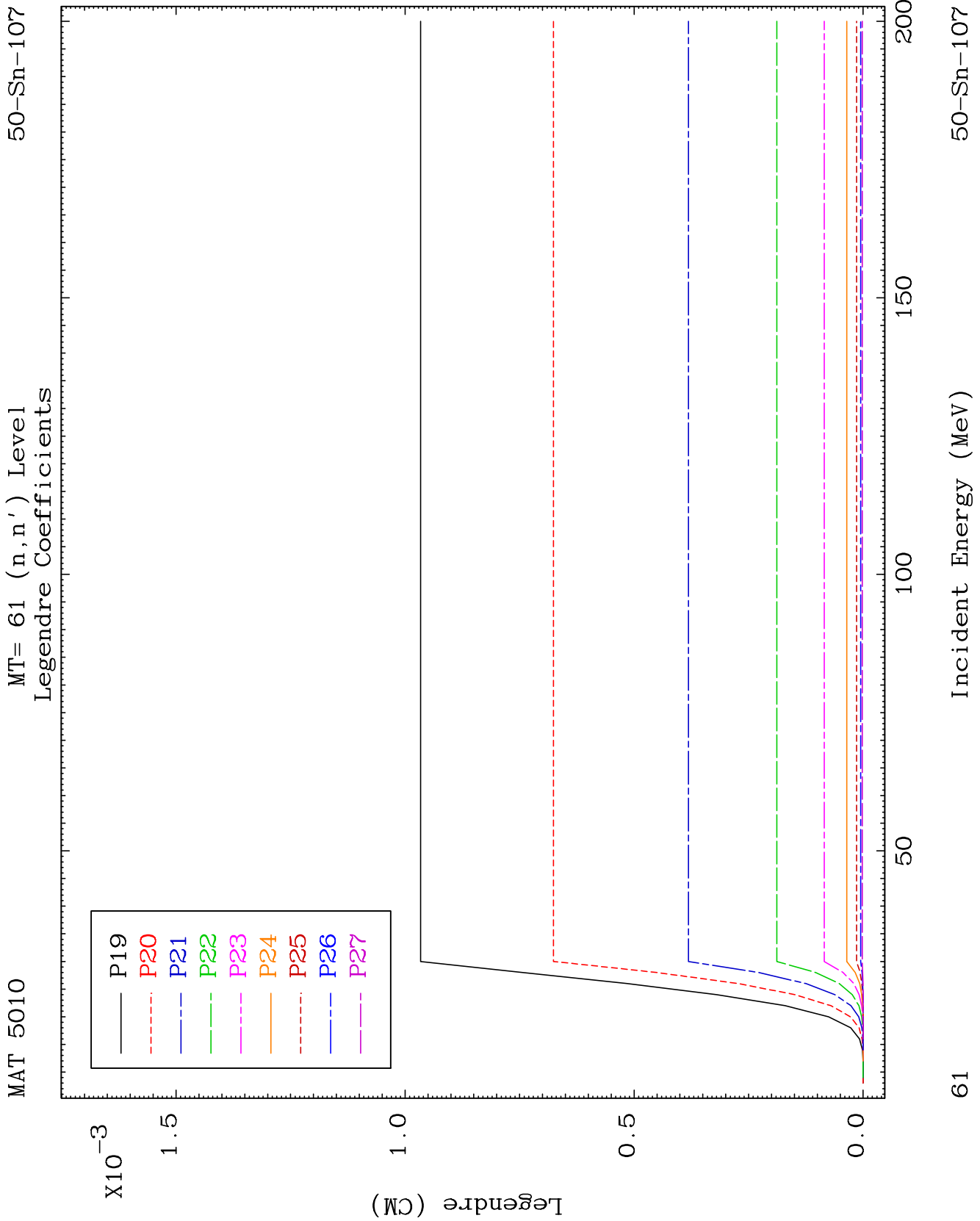


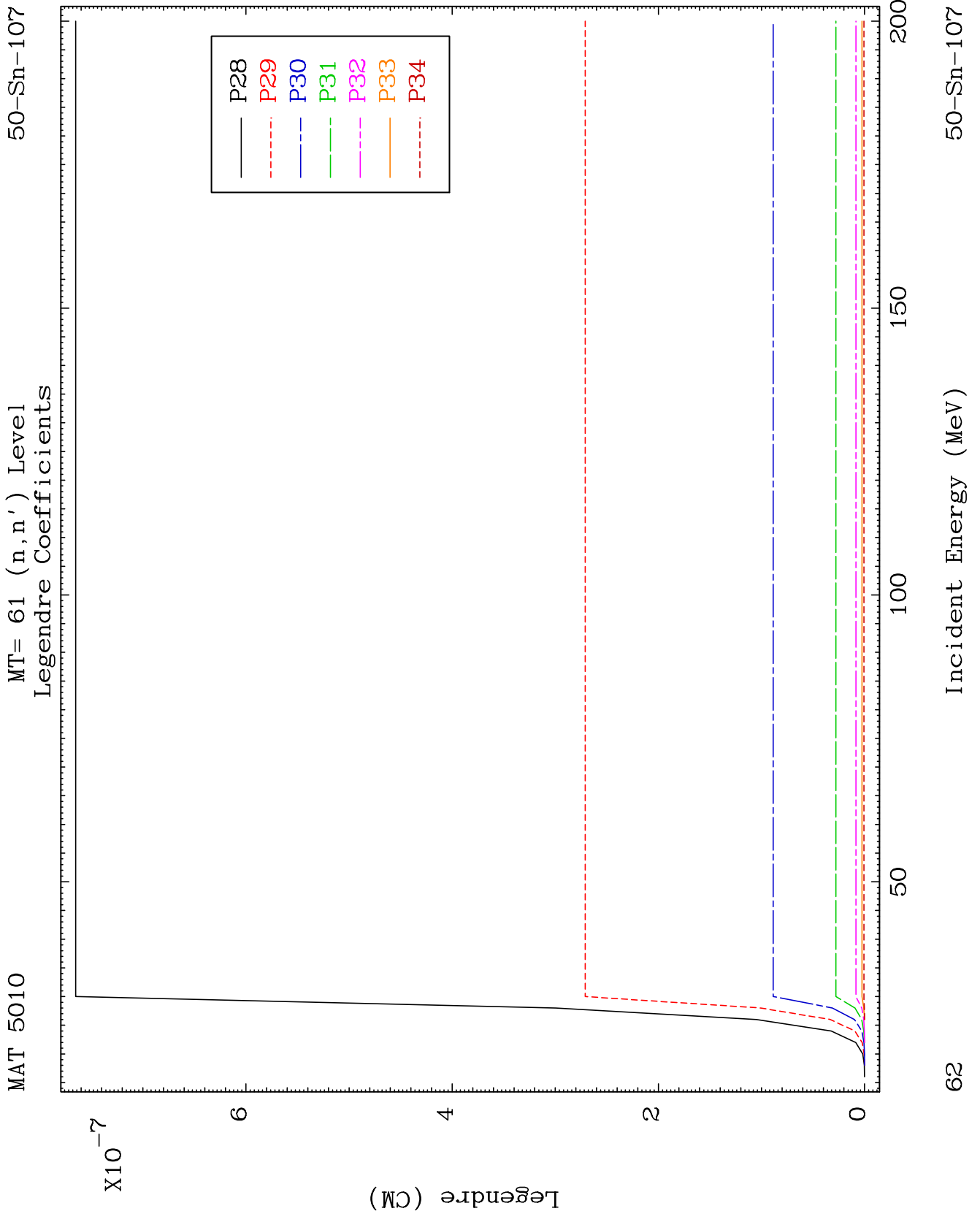


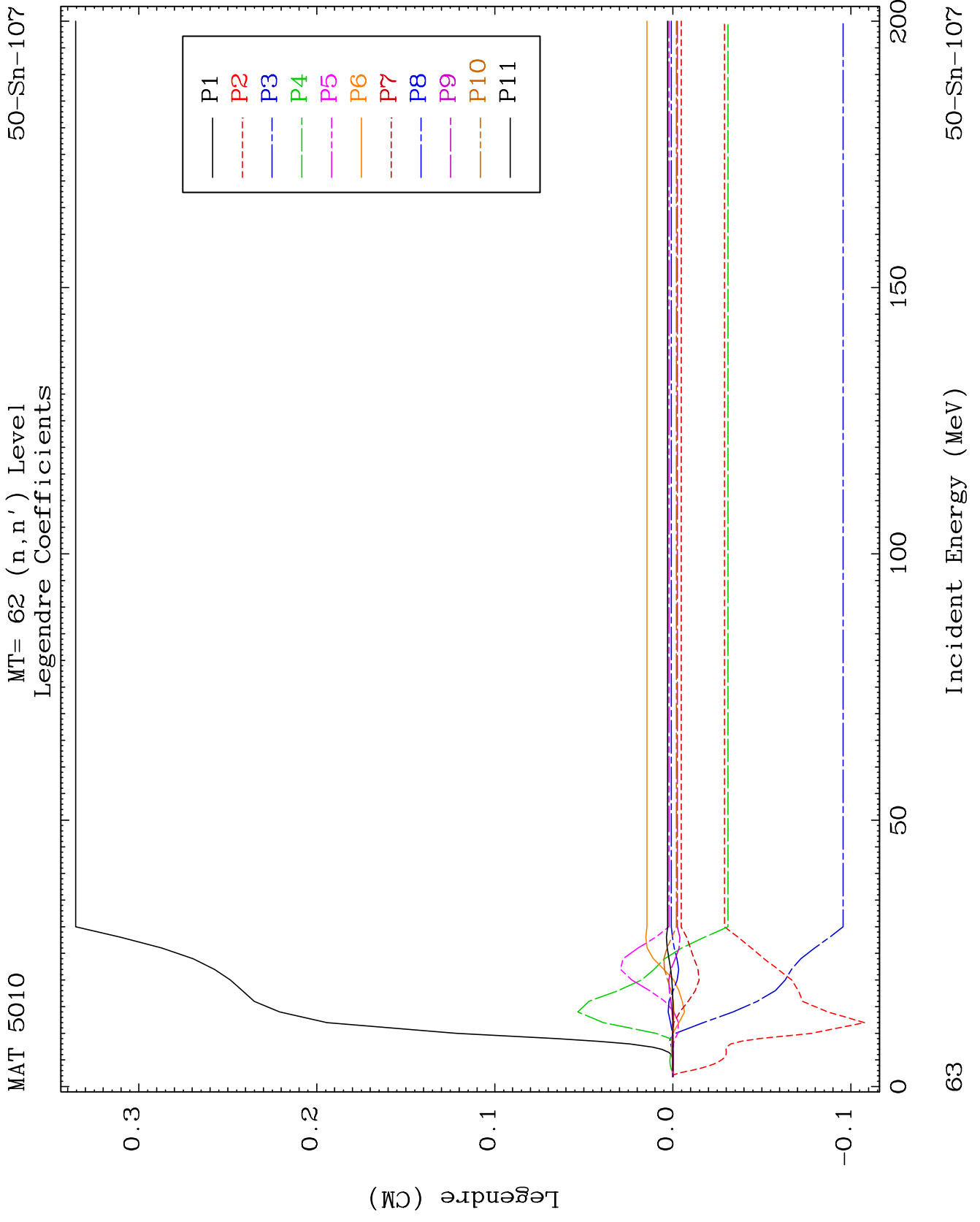








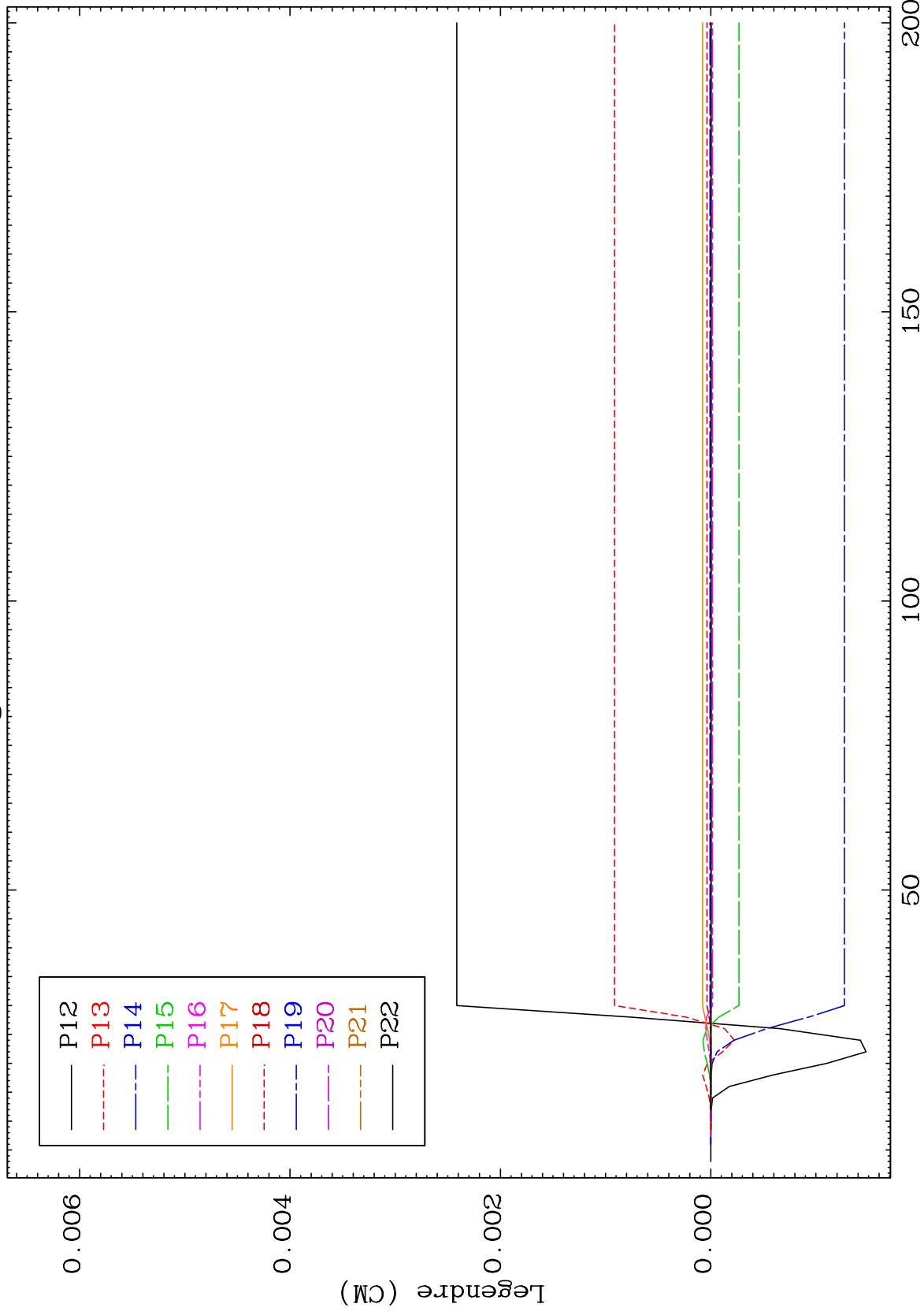


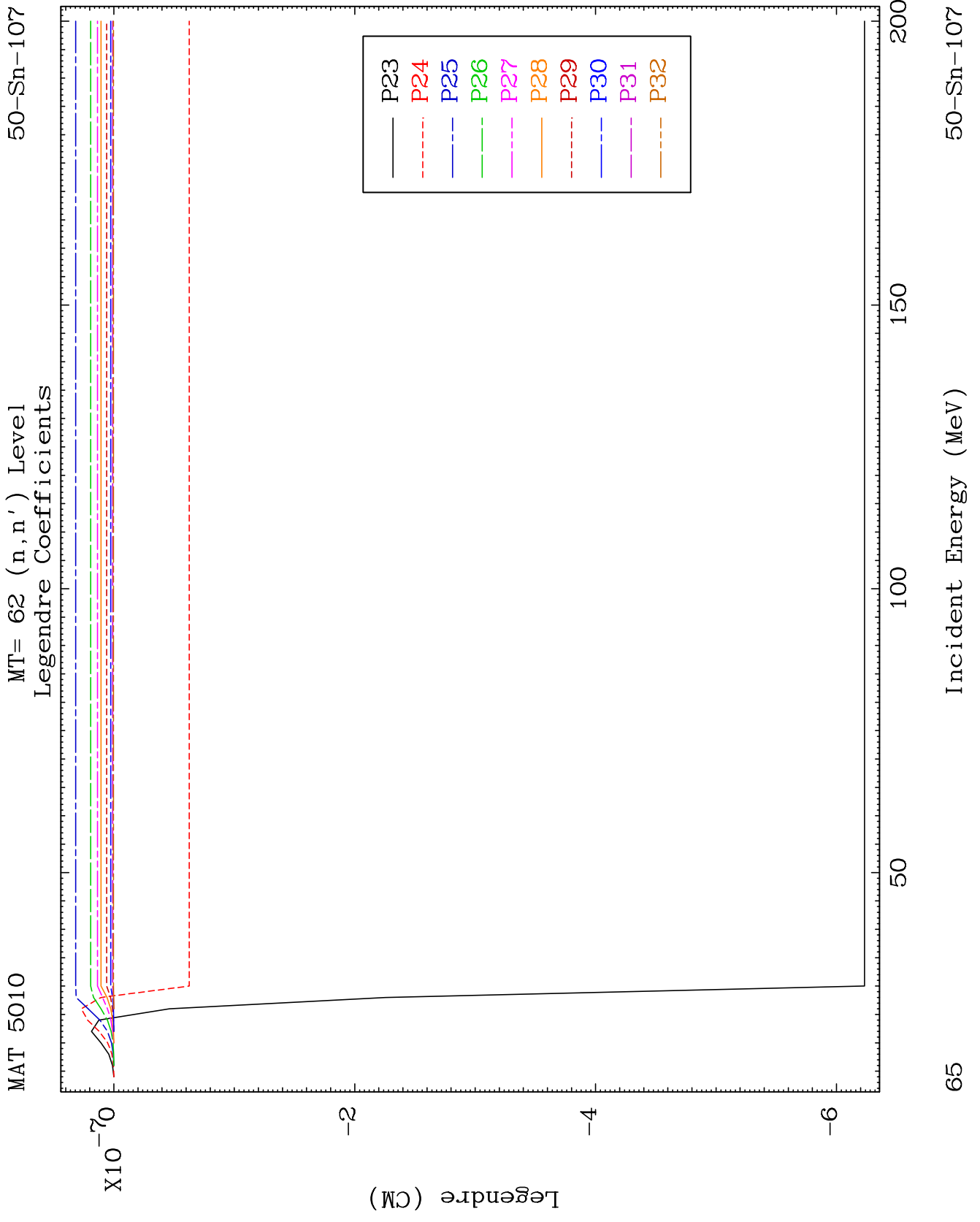


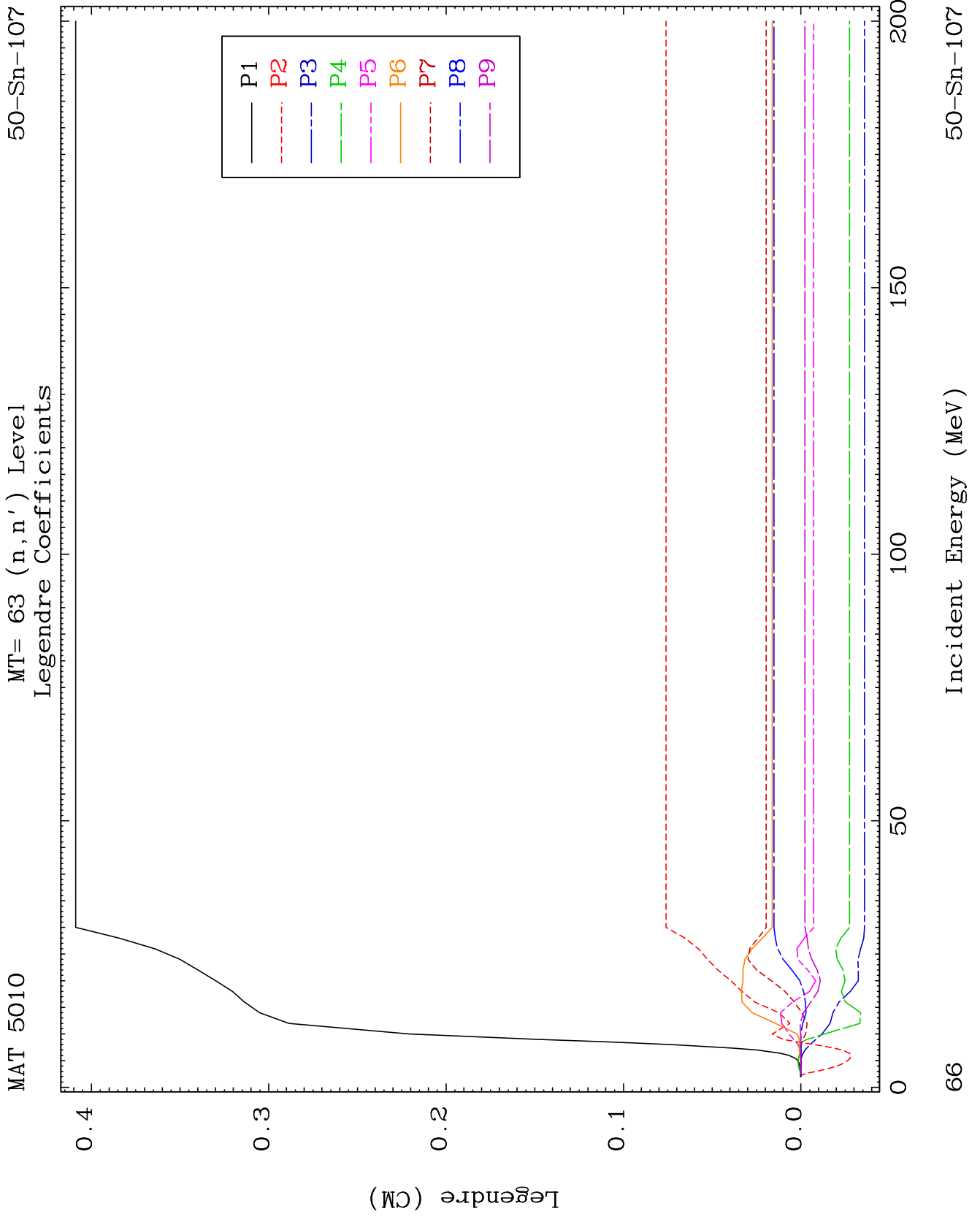
MAT 5010

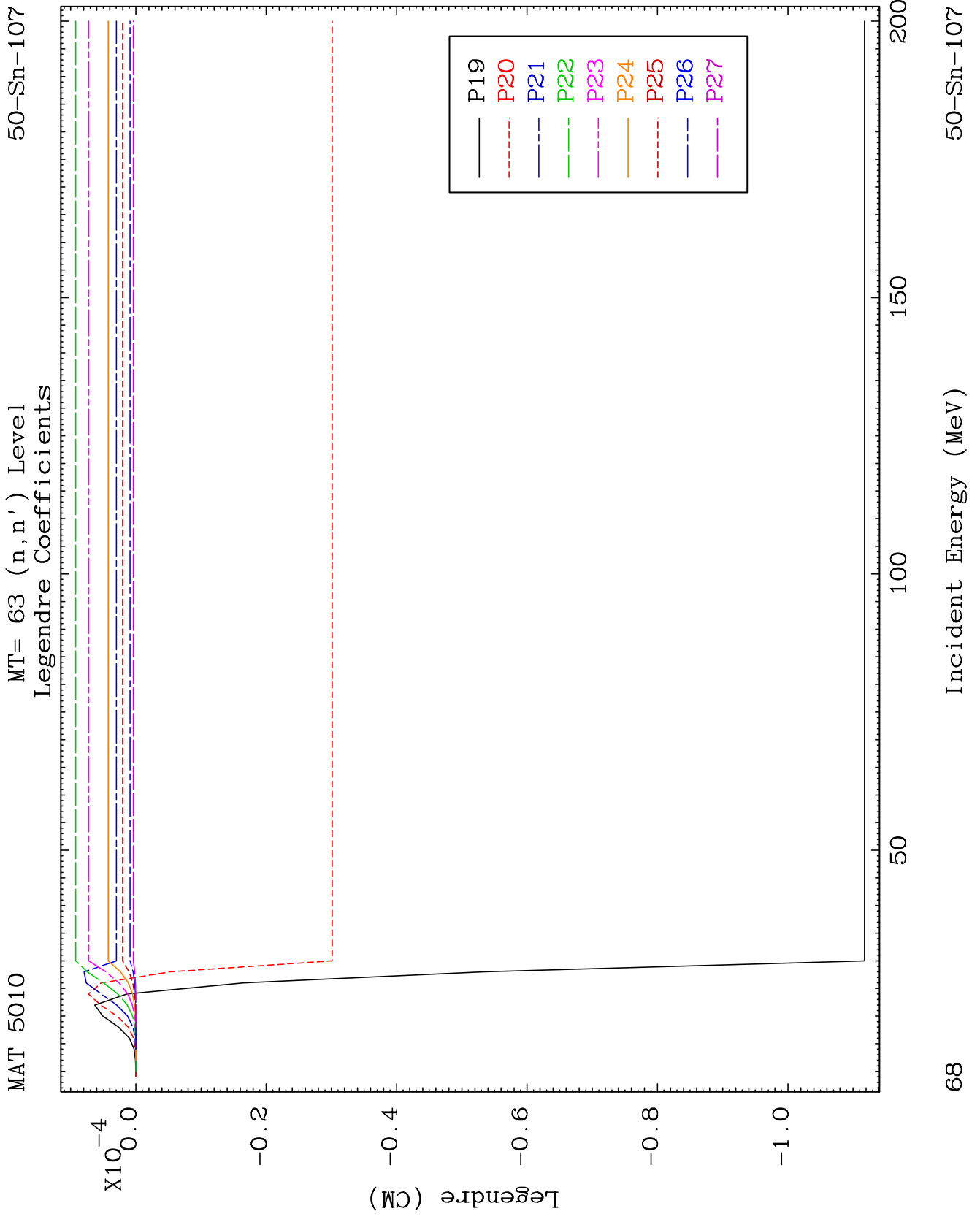
MT= 62 (n,n') Level
Legendre Coefficients

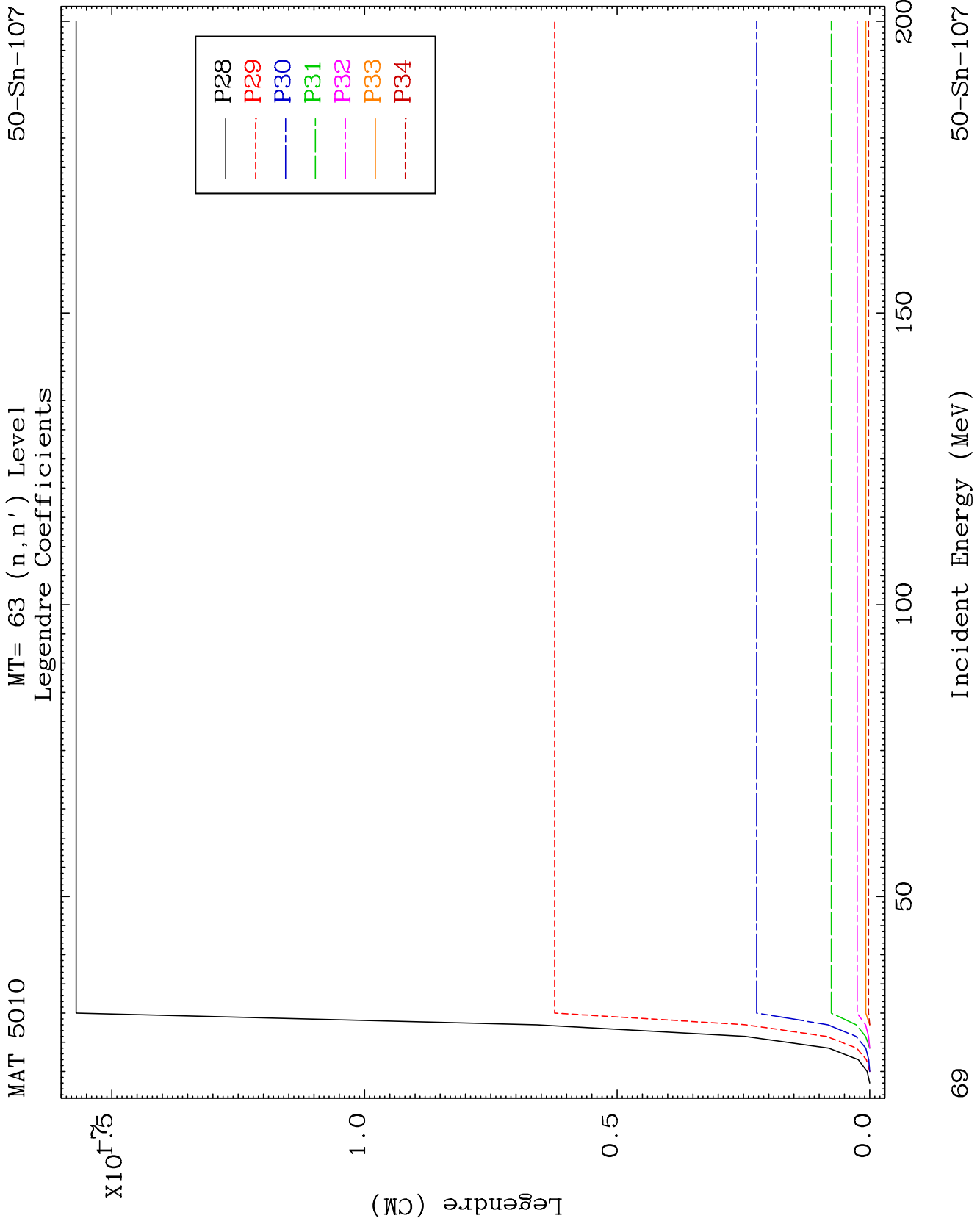
50-Sn-107

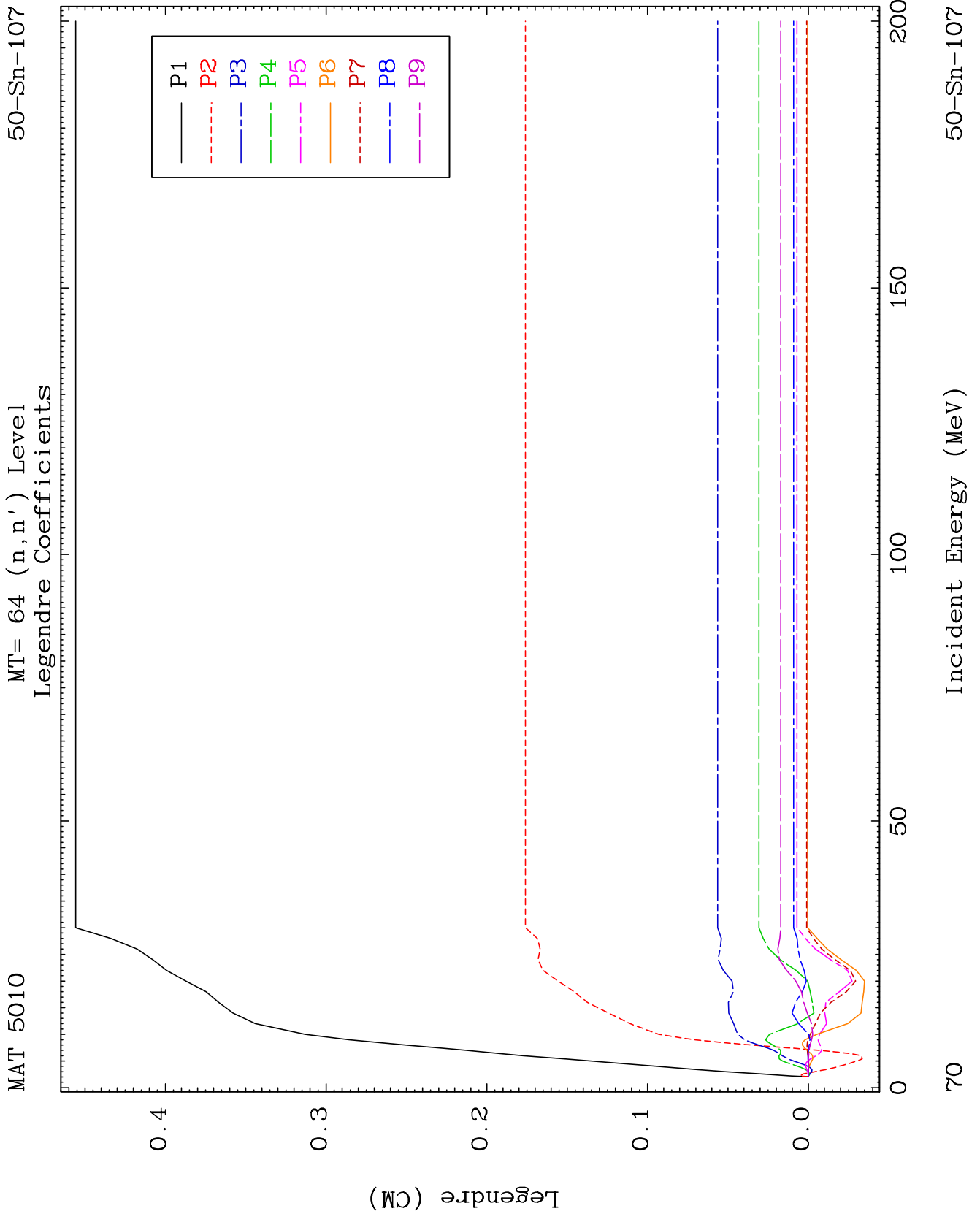


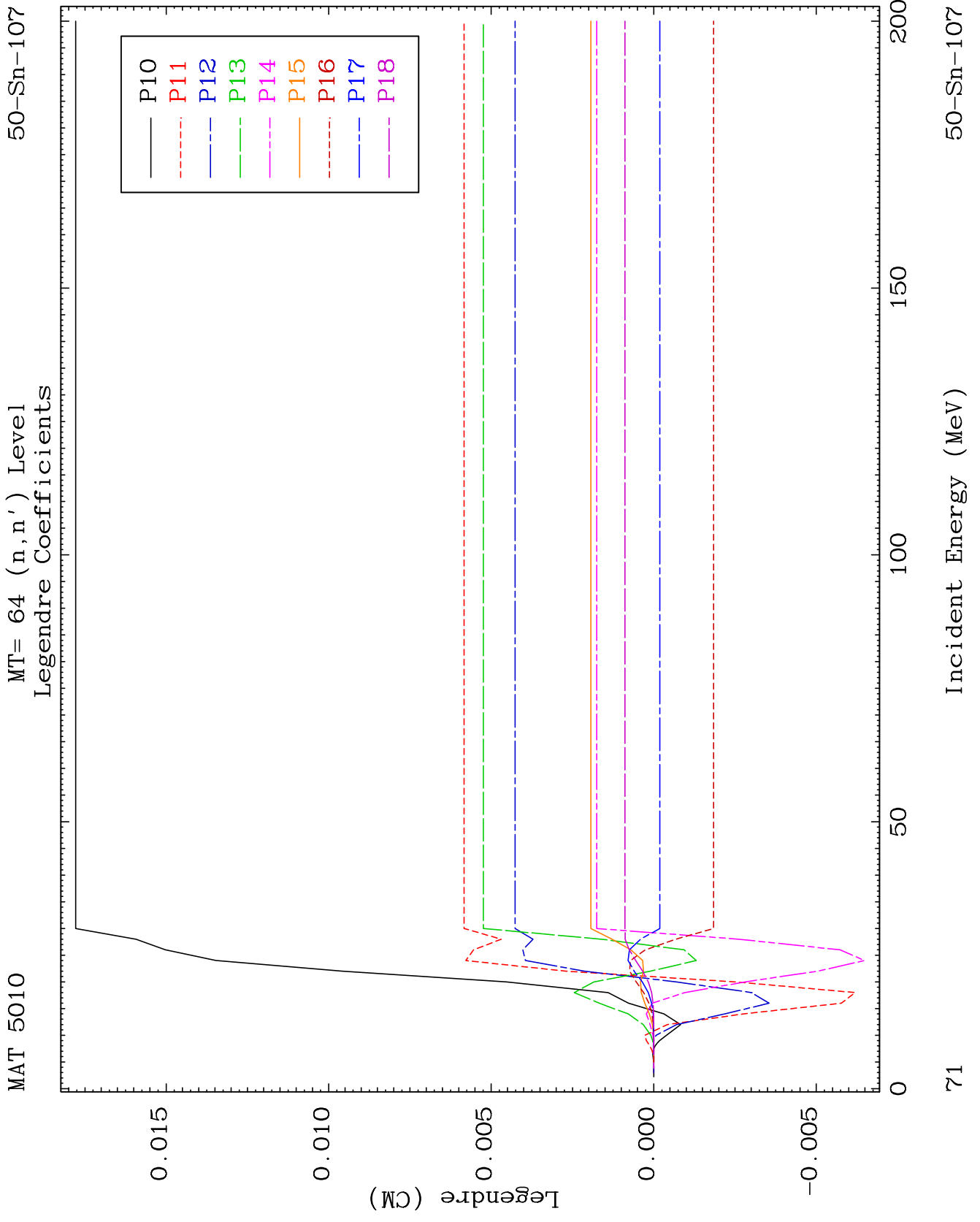


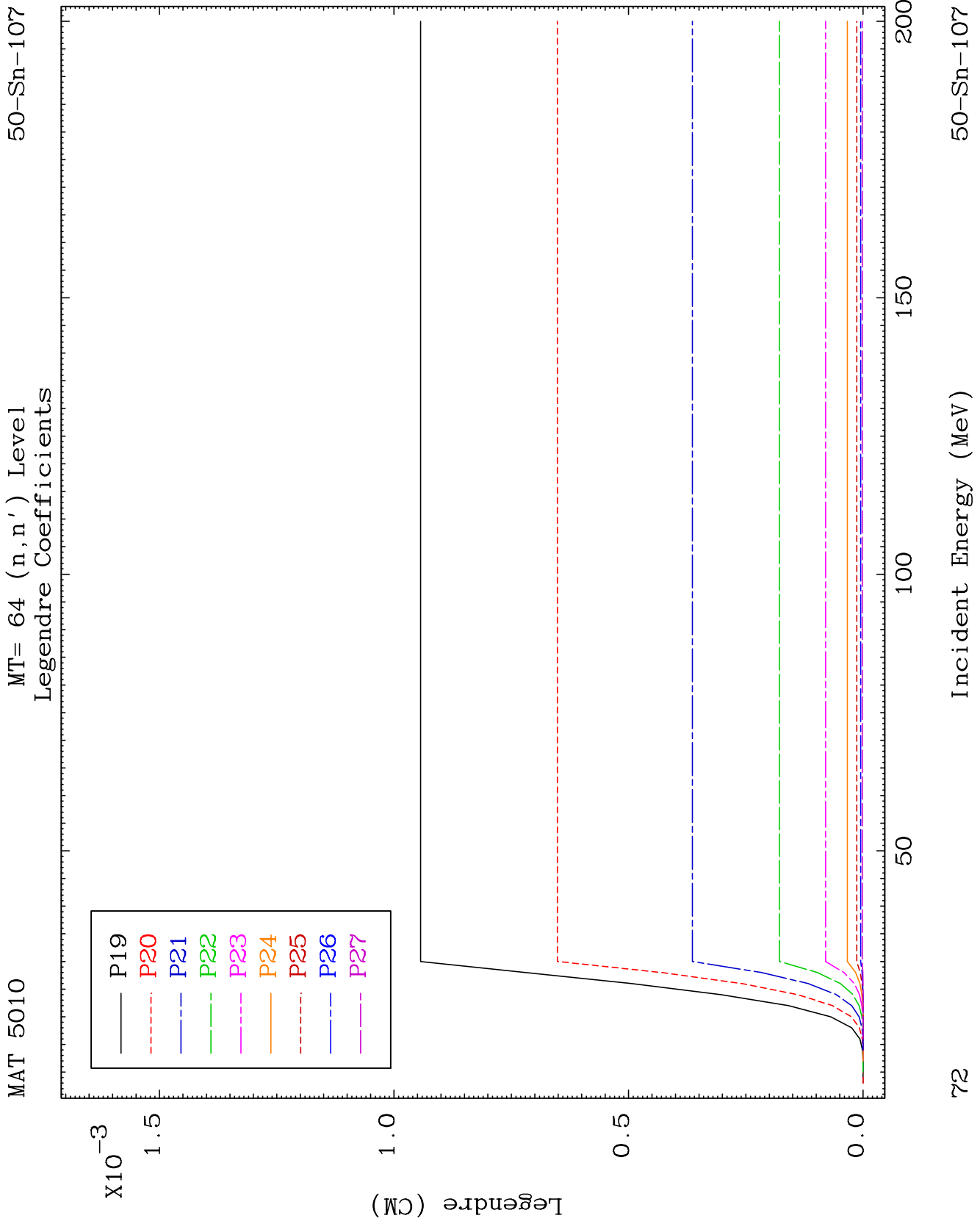


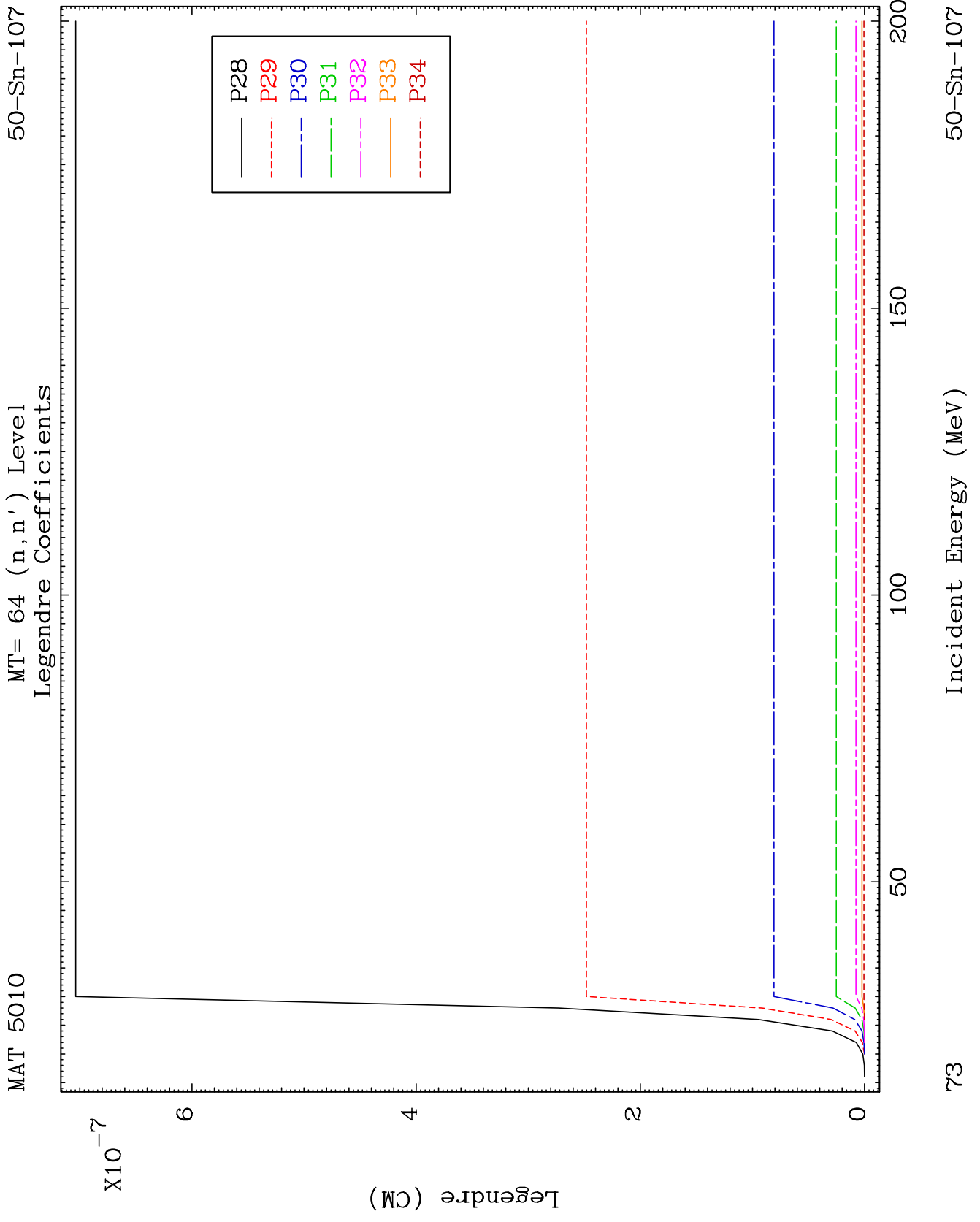


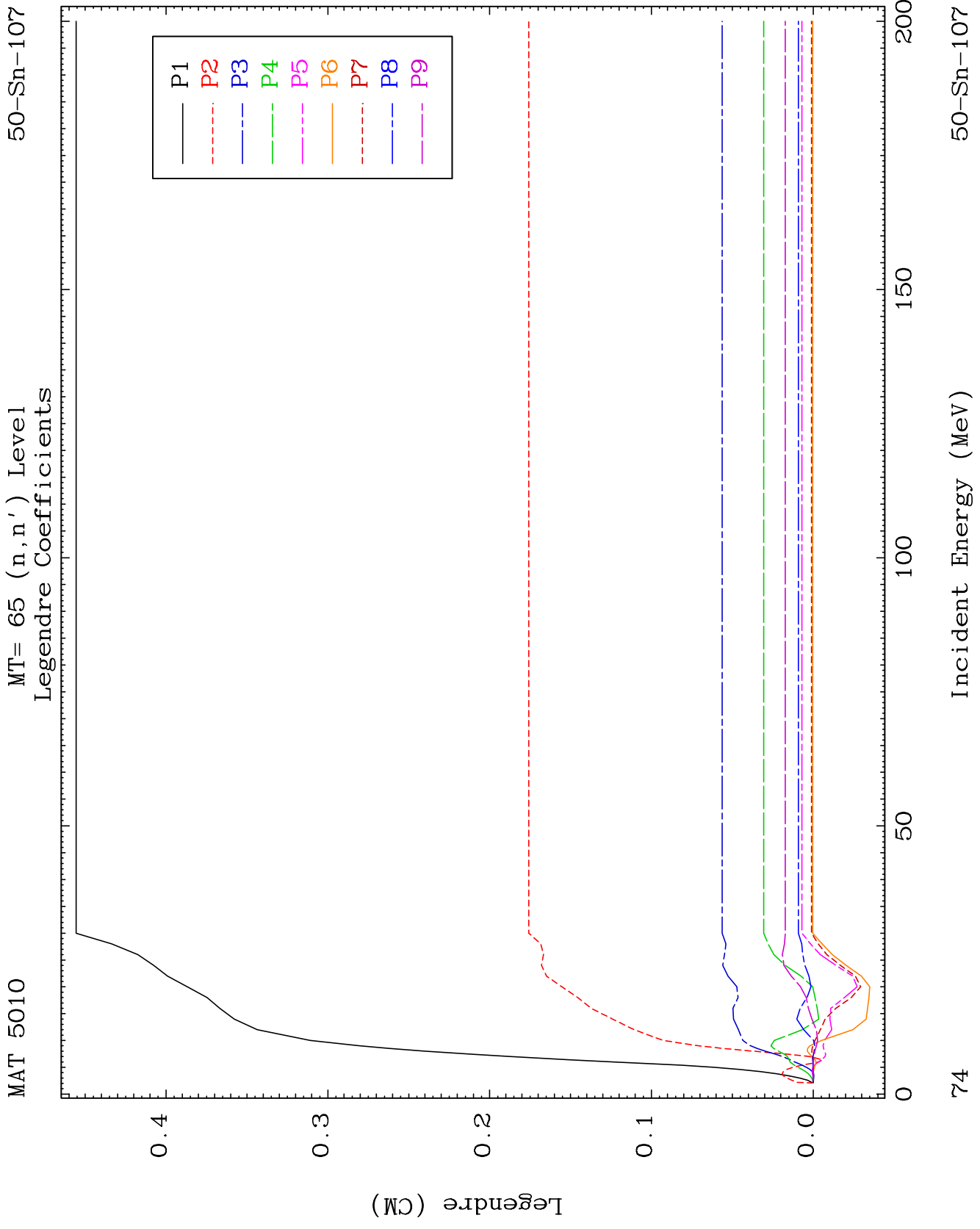


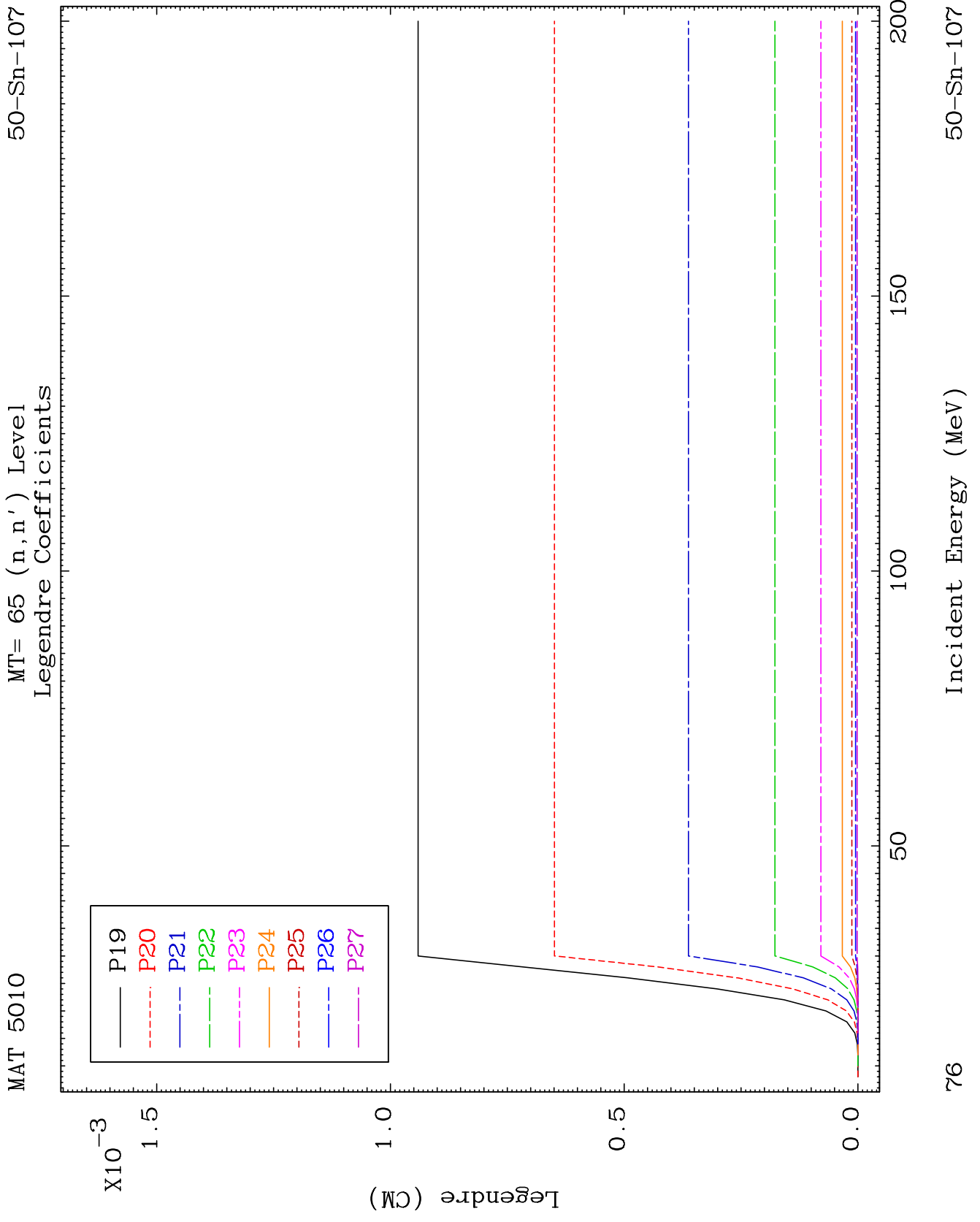


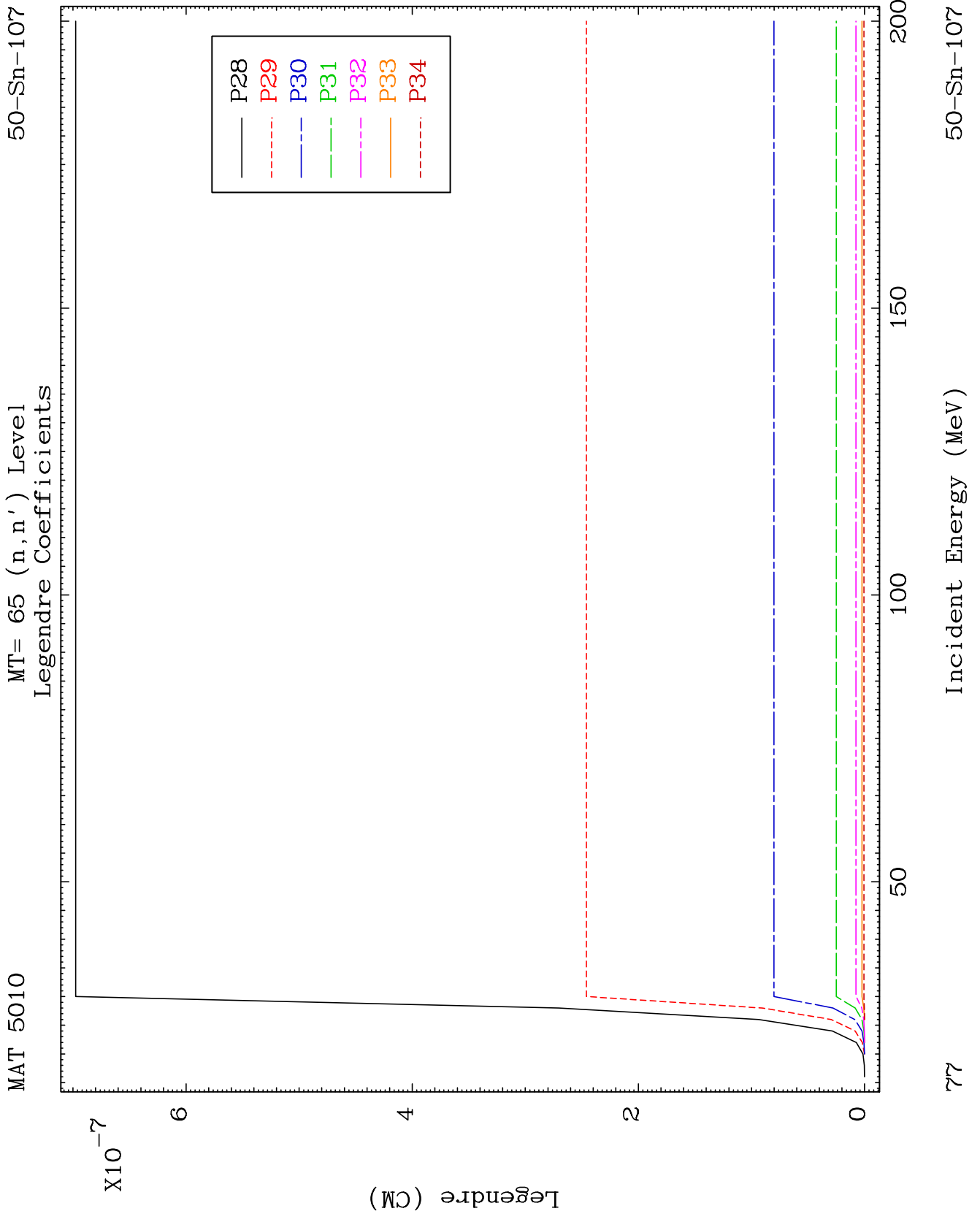








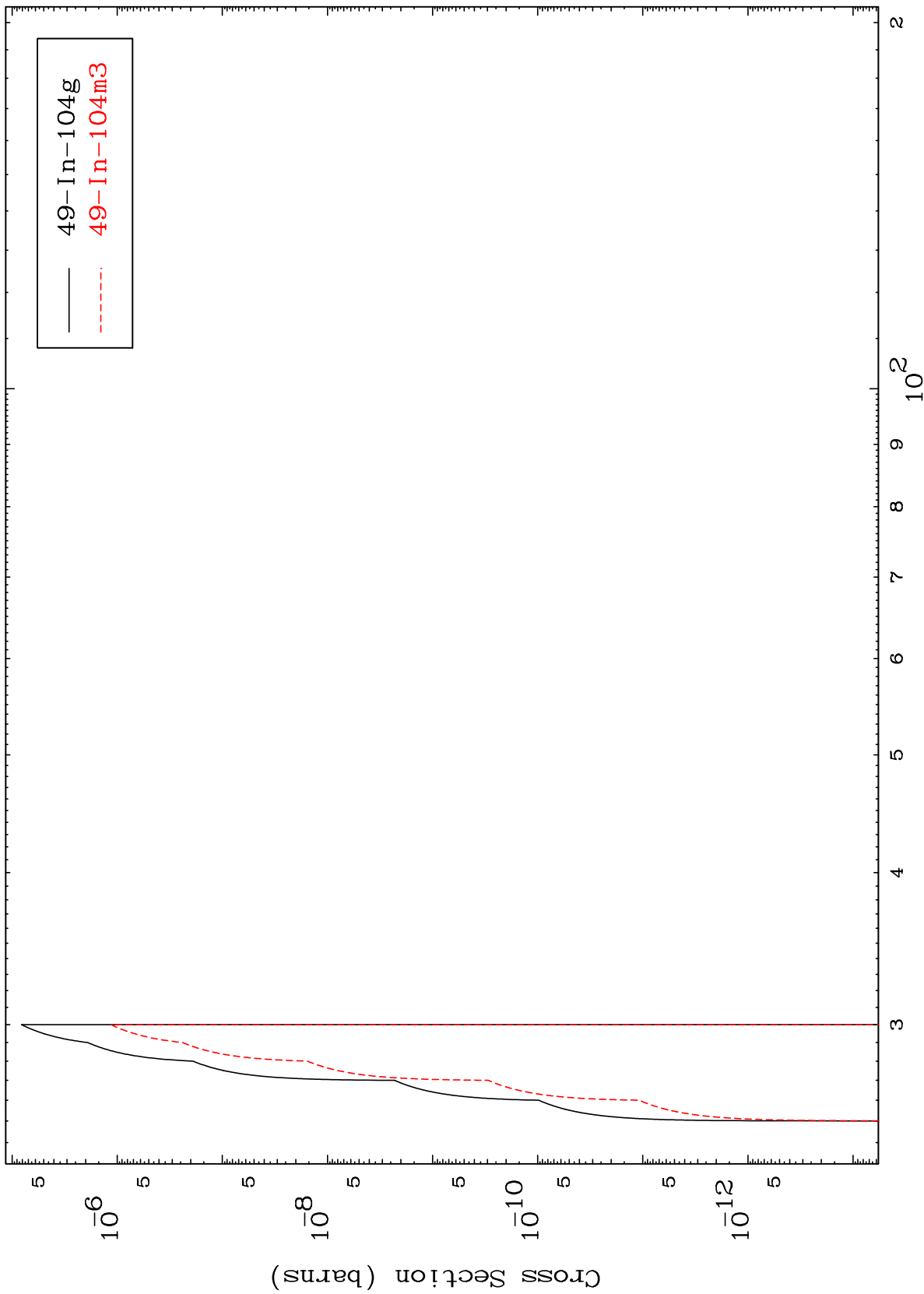




MAT 5010

50-Sn-107

(n,2n) d
Radionuclide Production Cross Section



78

Incident Energy (MeV)

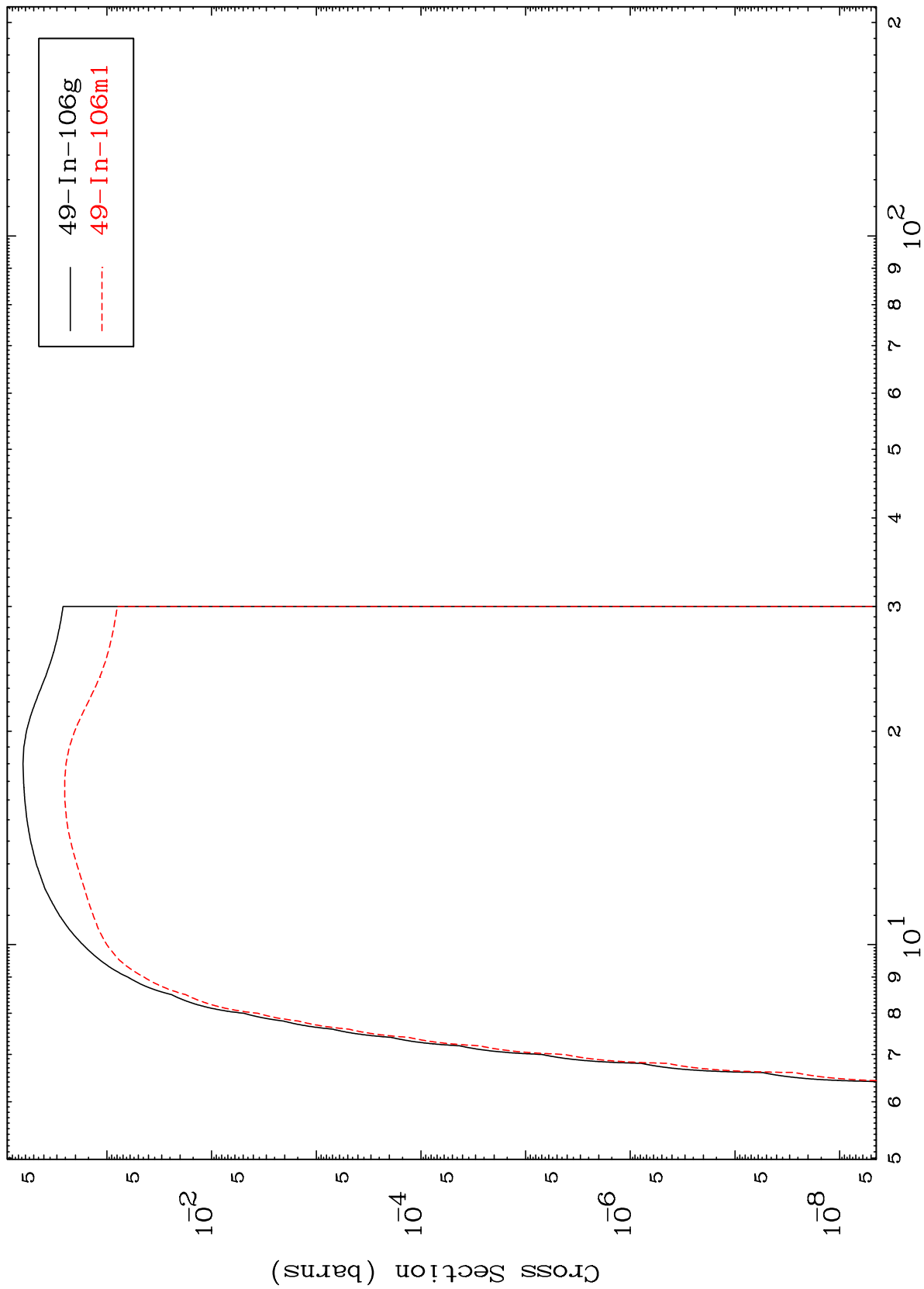
50-Sn-107

MAT 5010

(n,n') p

50-Sn-107

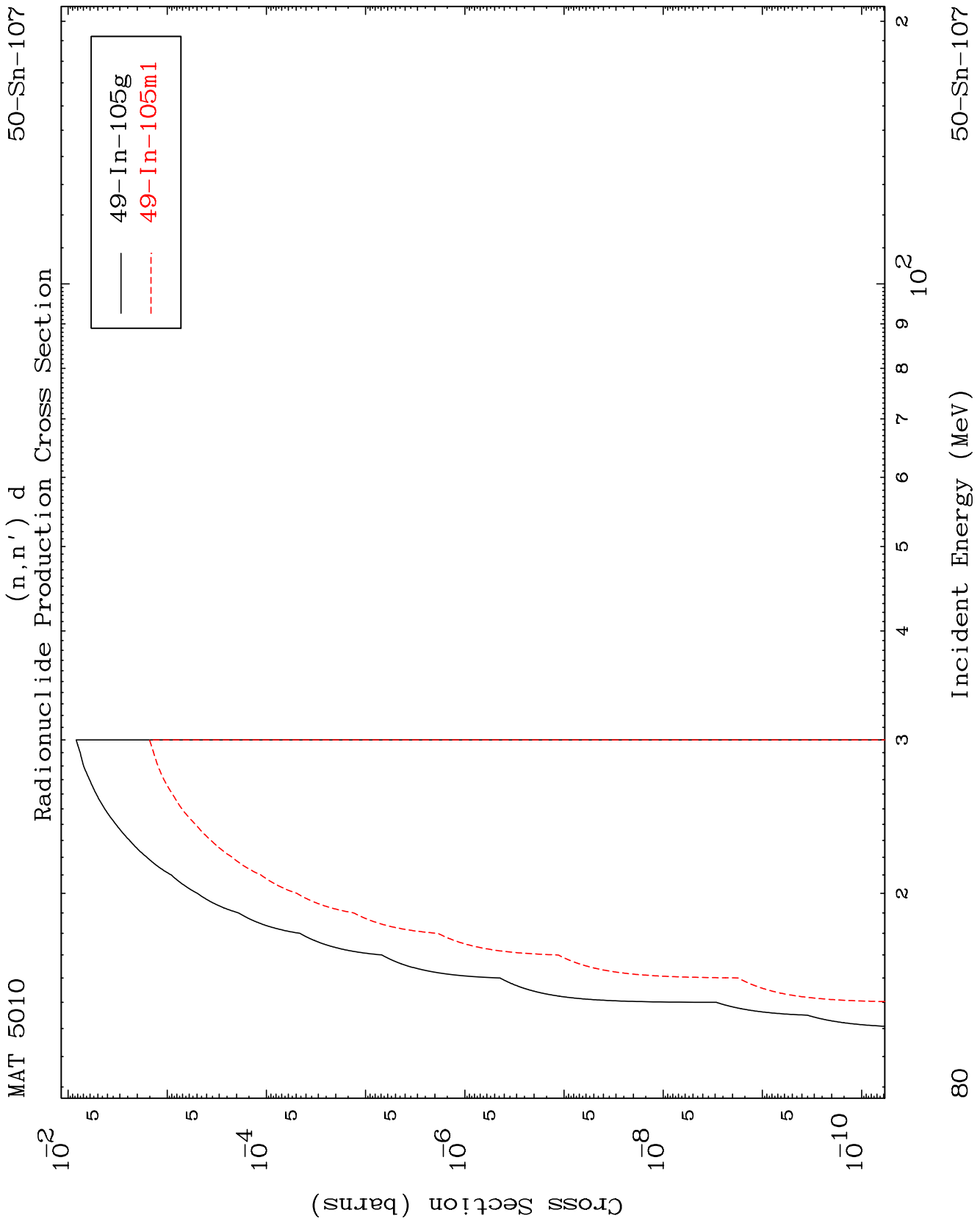
Radionuclide Production Cross Section



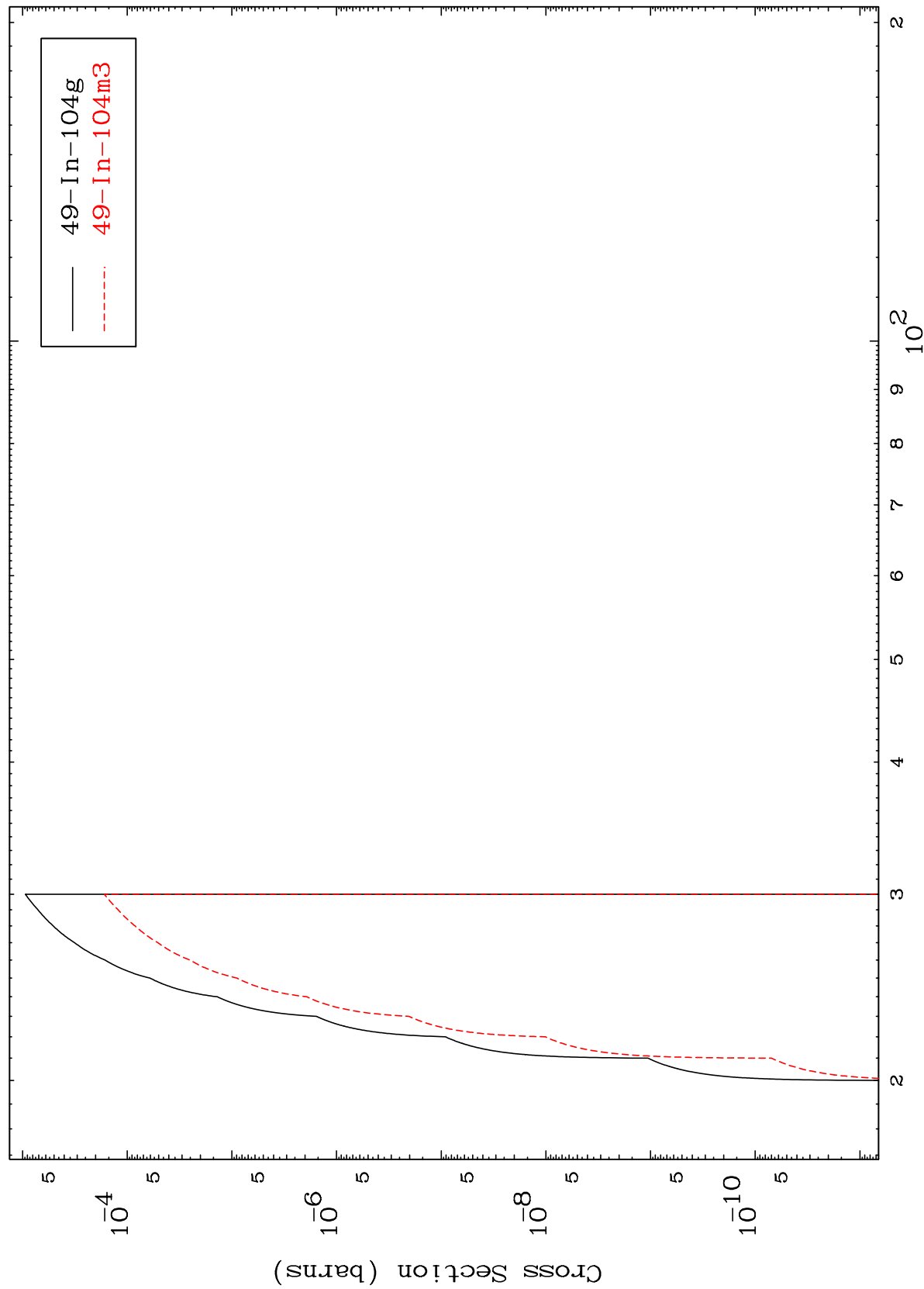
79

Incident Energy (MeV)

50-Sn-107



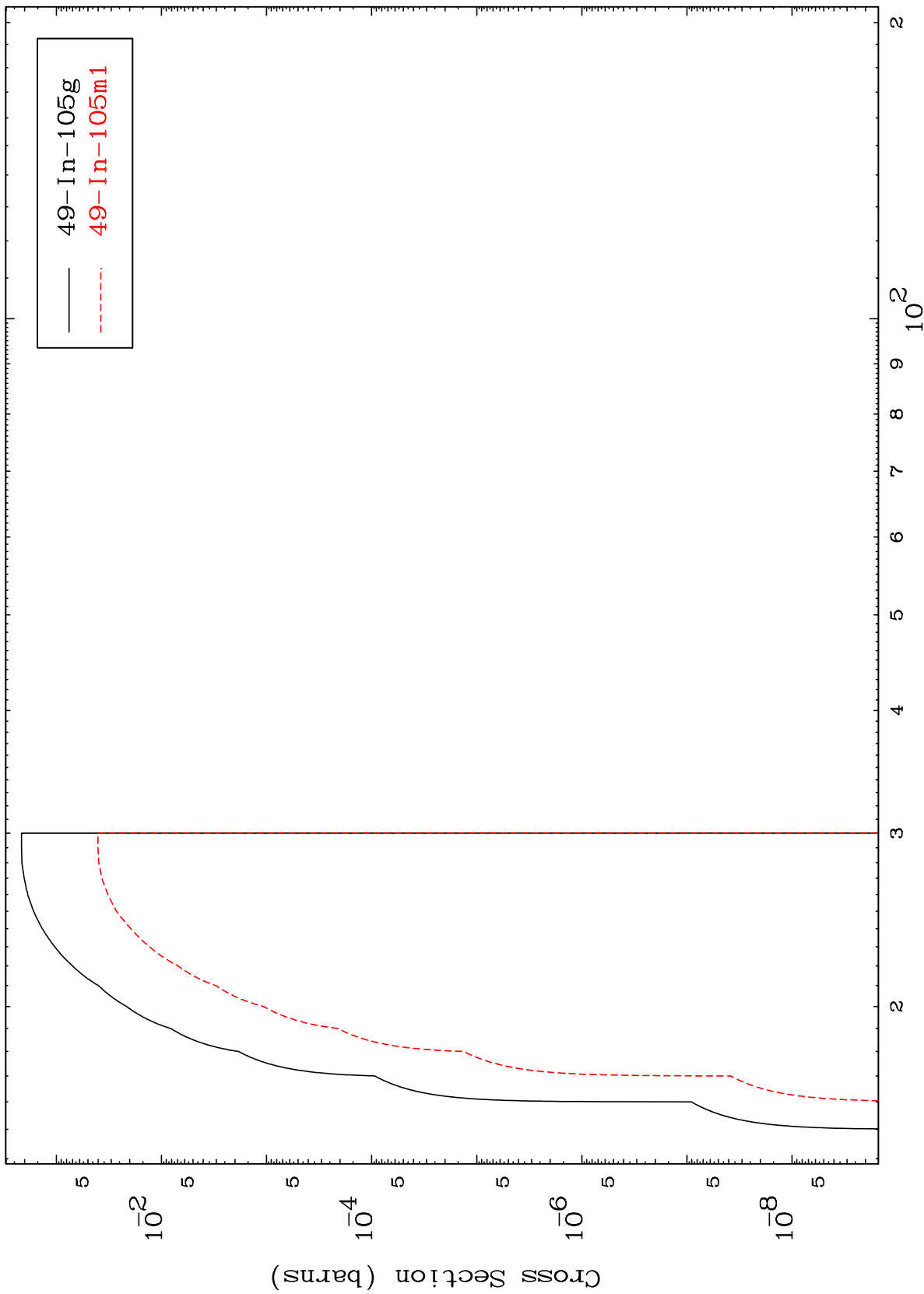
Radionuclide Production Cross Section



MAT 5010

50-Sn-107

(n,2n) p
Radionuclide Production Cross Section



82

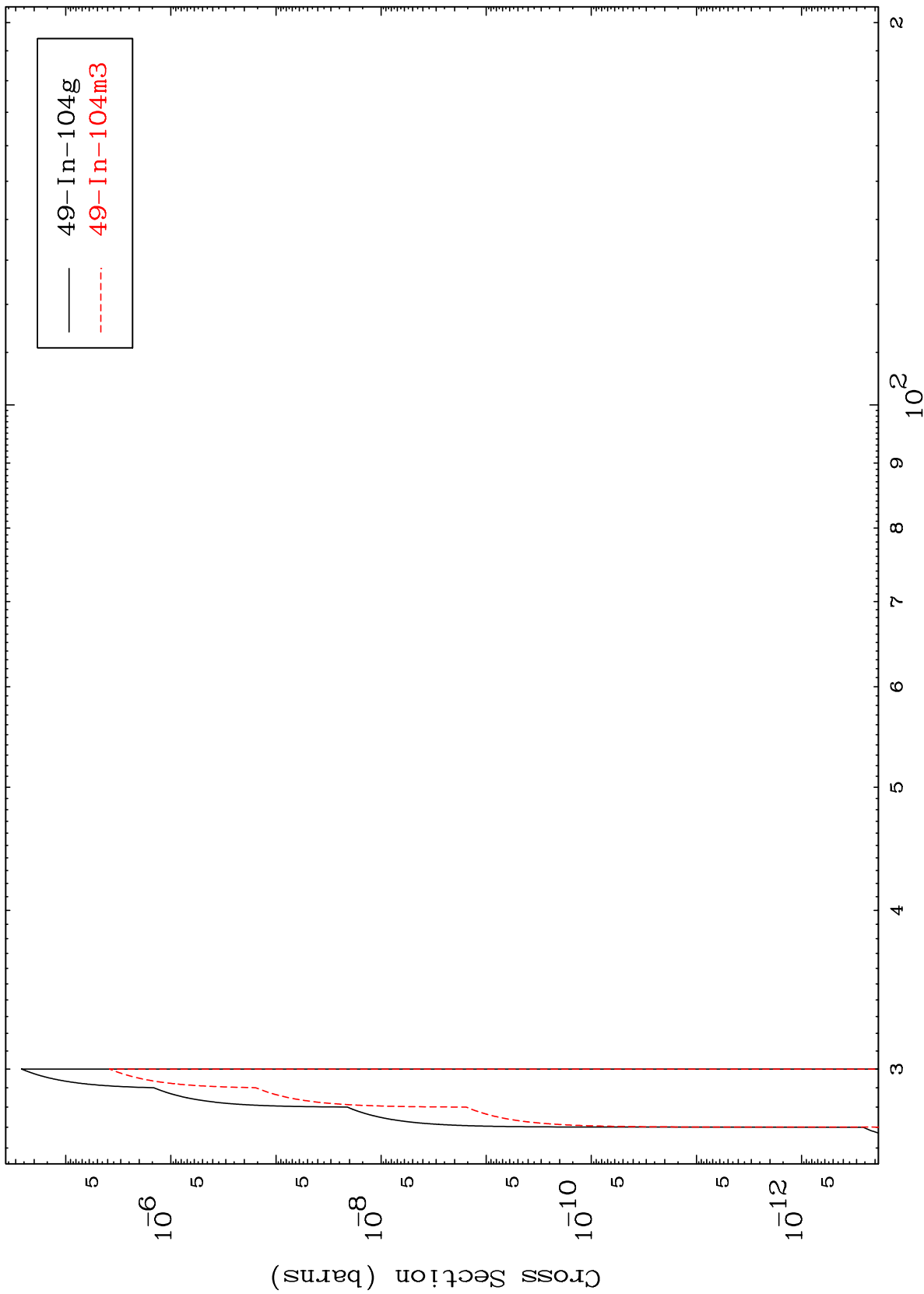
50-Sn-107

Incident Energy (MeV)

MAT 5010

50-Sn-107

(n,3n) p
Radionuclide Production Cross Section



83

50-Sn-107

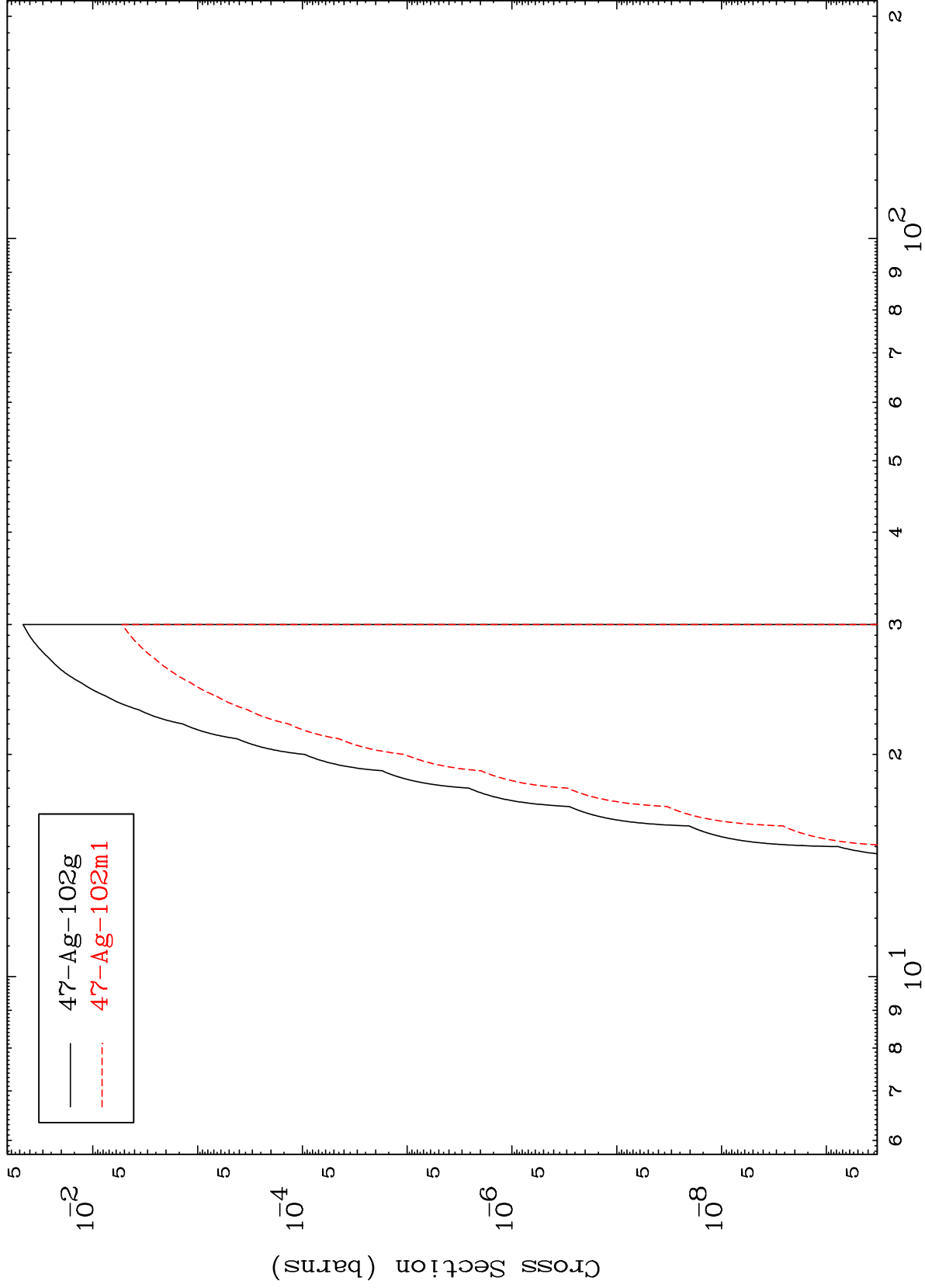
Incident Energy (MeV)

MAT 5010

(n,n') p α

50-Sn-107

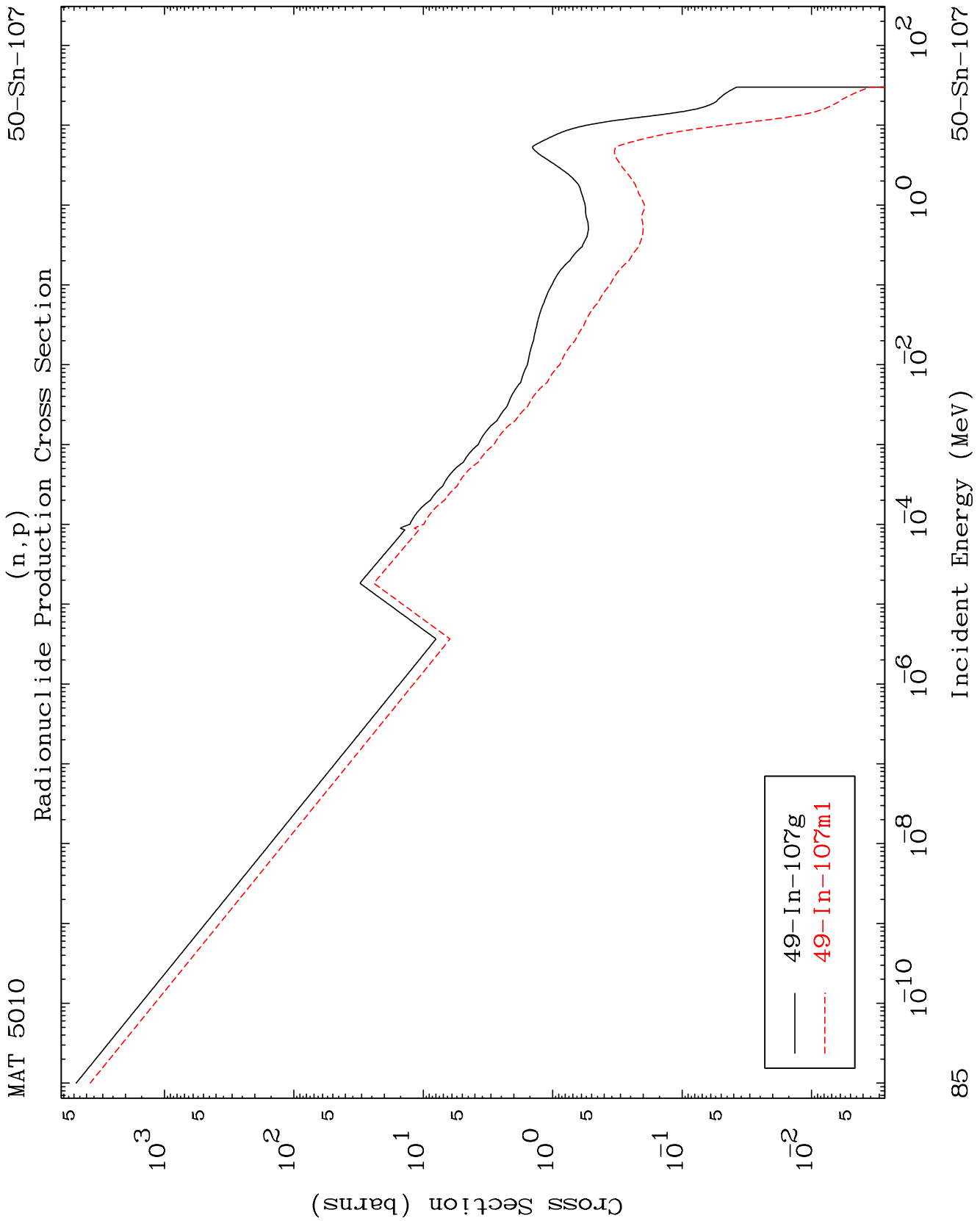
Radionuclide Production Cross Section



84

Incident Energy (MeV)

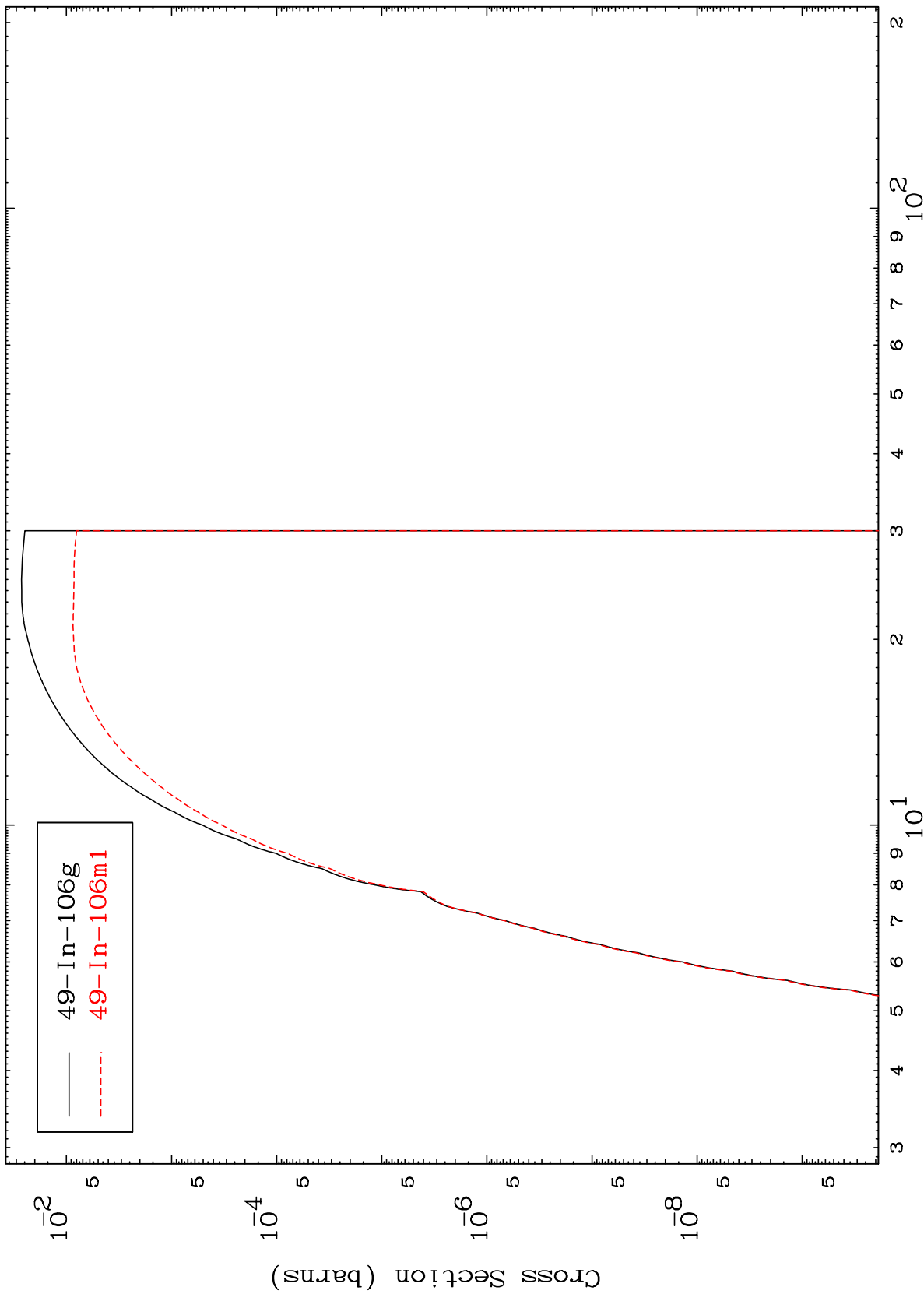
50-Sn-107



MAT 5010

50-Sn-107

(n,d)
Radionuclide Production Cross Section



86

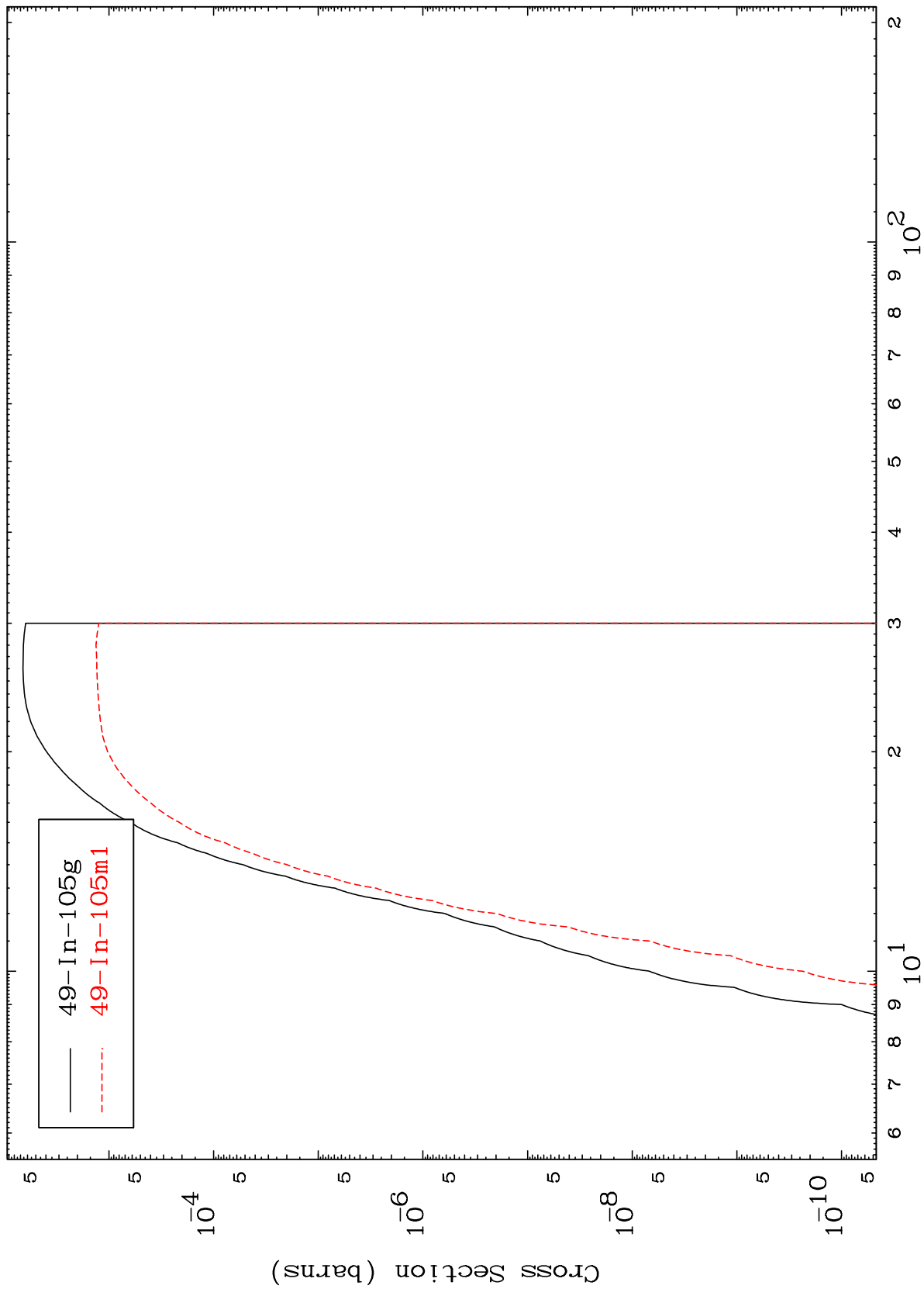
Incident Energy (MeV)

50-Sn-107

MAT 5010

50-Sn-107

(n, t)
Radionuclide Production Cross Section



50-Sn-107

Incident Energy (MeV)

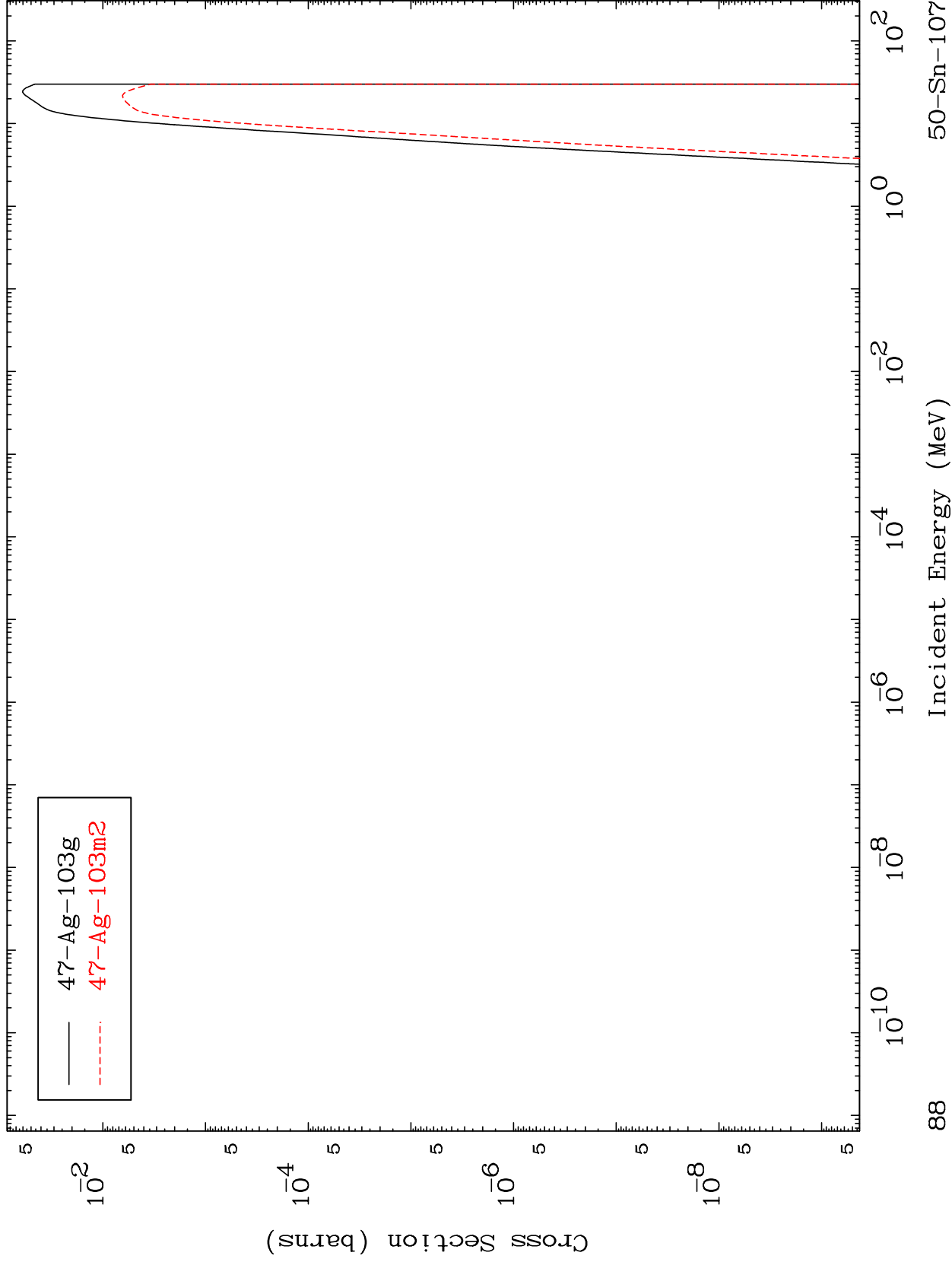
87

MAT 5010

(n,p) α

50-Sn-107

Radionuclide Production Cross Section

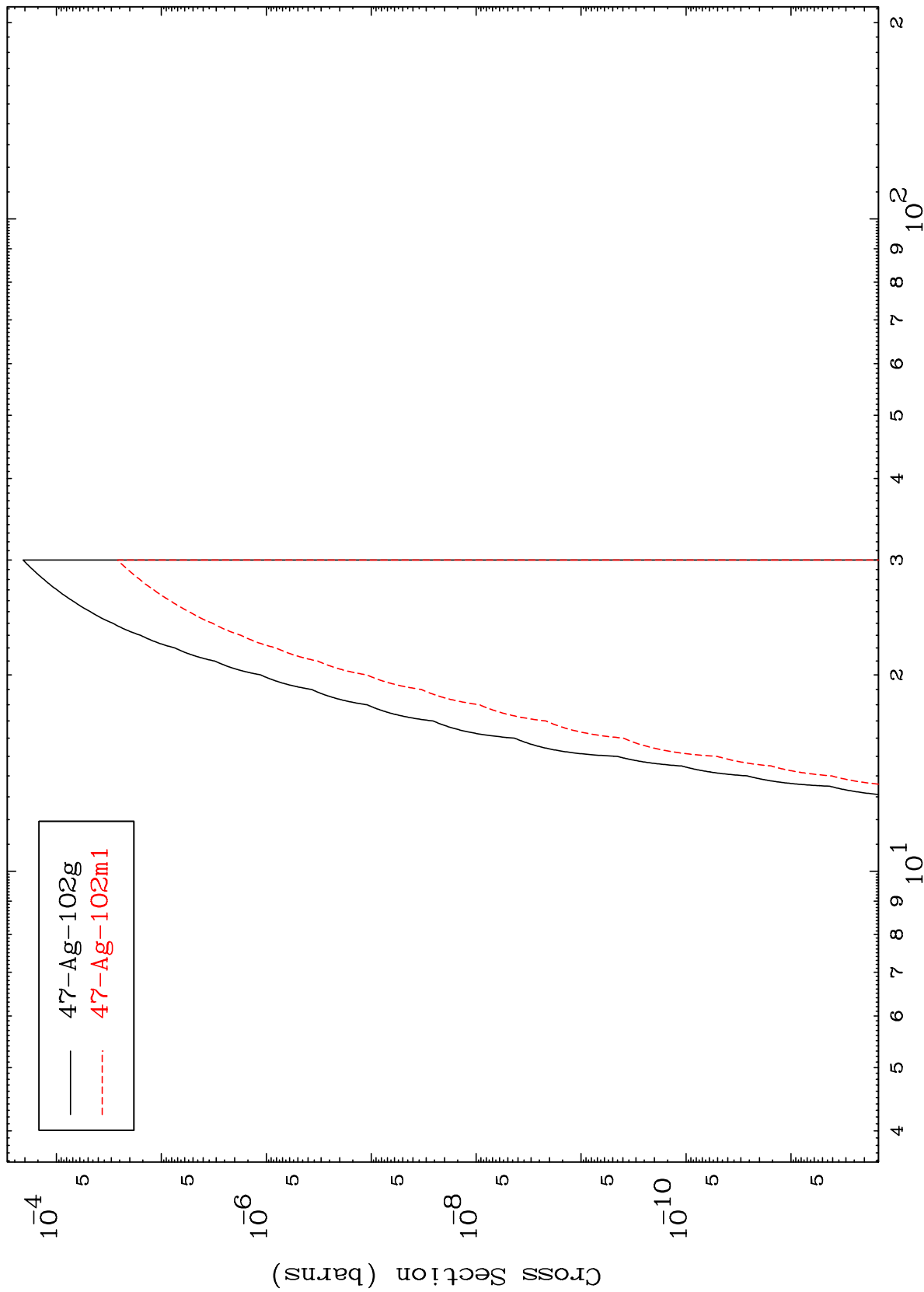


MAT 5010

(n,d) α

50-Sn-107

Radionuclide Production Cross Section



— 47-Ag-102g
- - - 47-Ag-102m1

89

Incident Energy (MeV)

50-Sn-107