

Program EVALPLOT  
(Version 2017-1)

by

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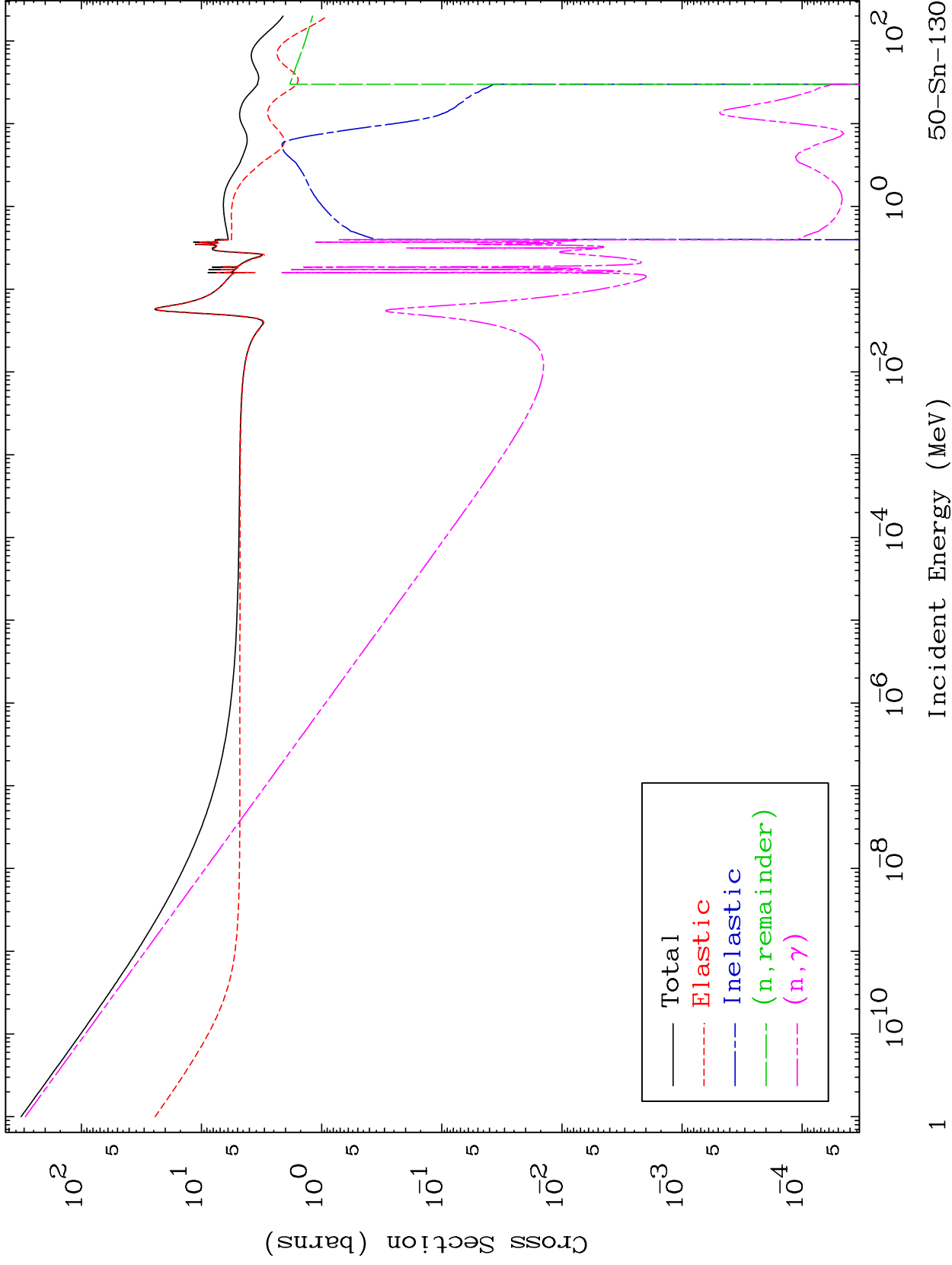
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 5080

Major  
293 Kelvin Cross Sections

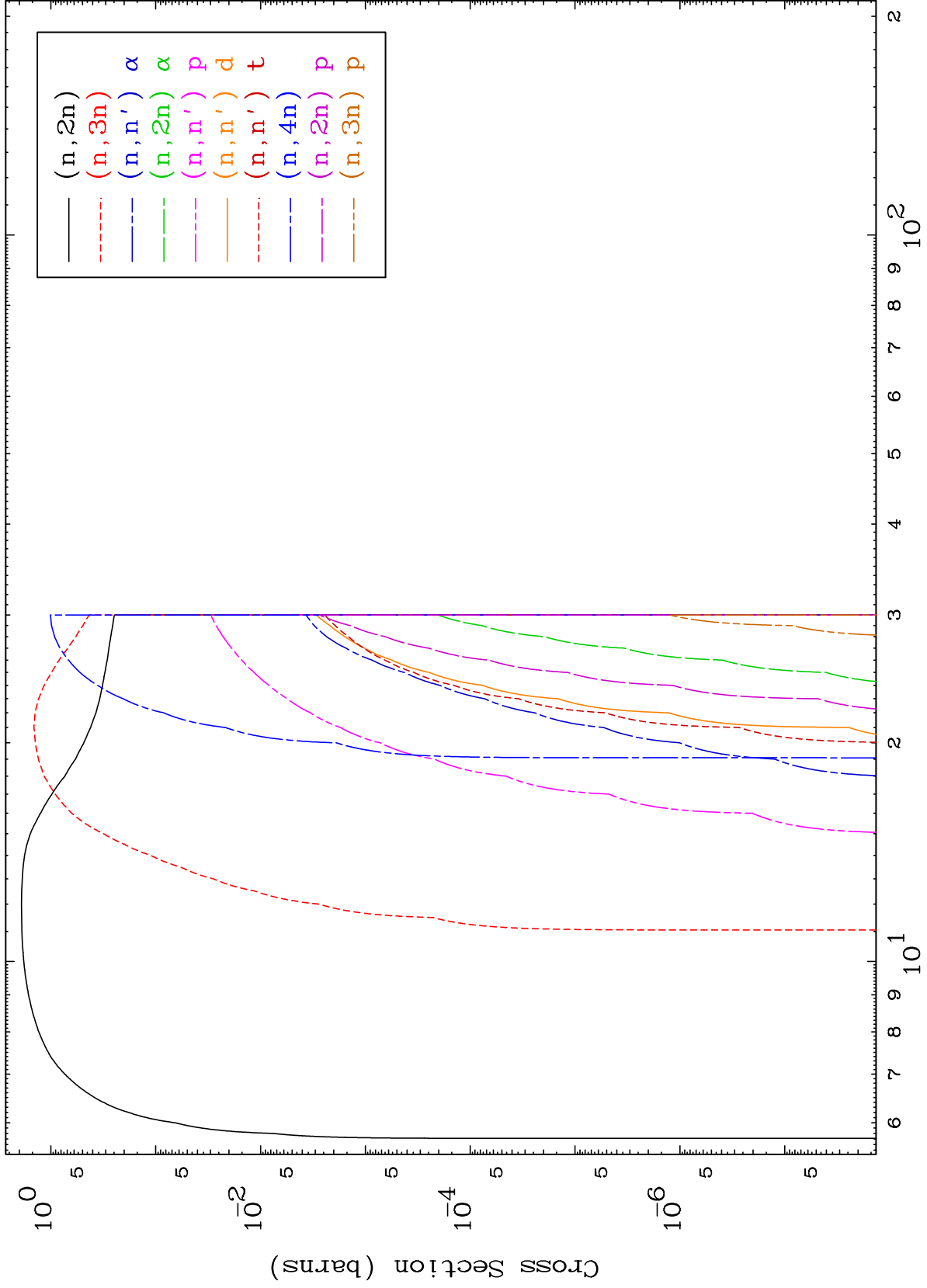
50-Sn-130



MAT 5080

Neutron Production  
293 Kelvin Cross Sections

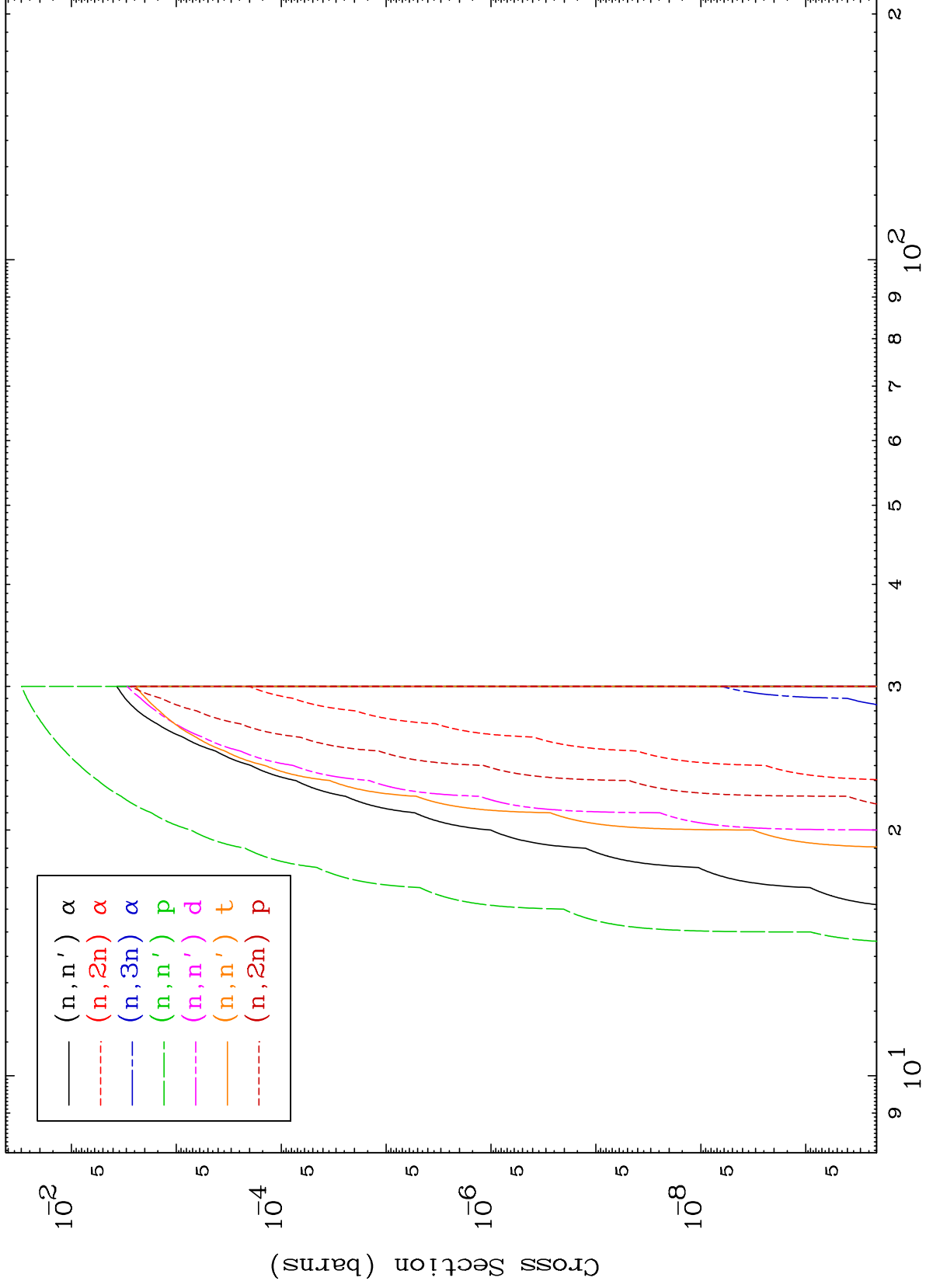
50-Sn-130



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Incident Energy (MeV)

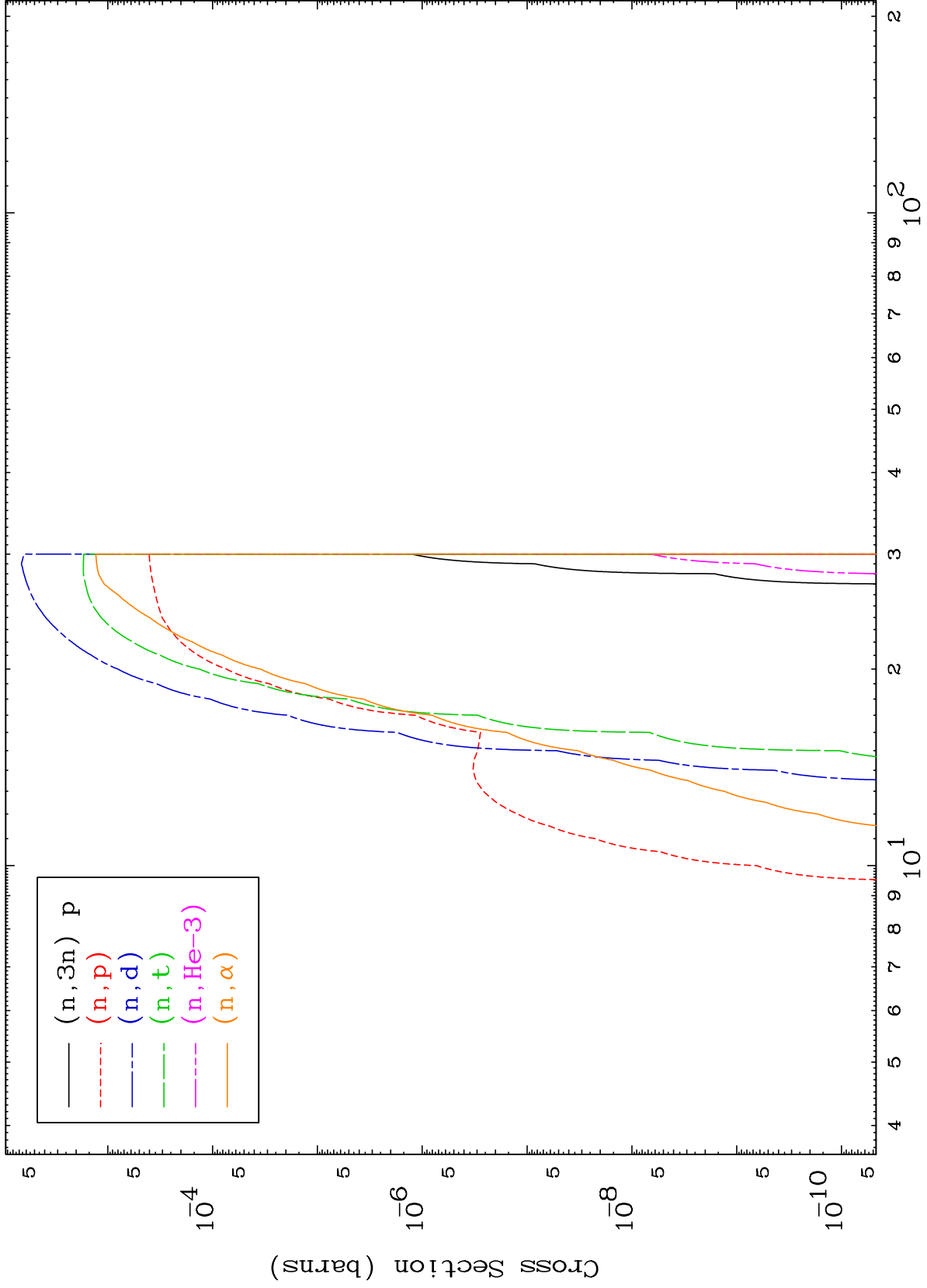
50-Sn-130

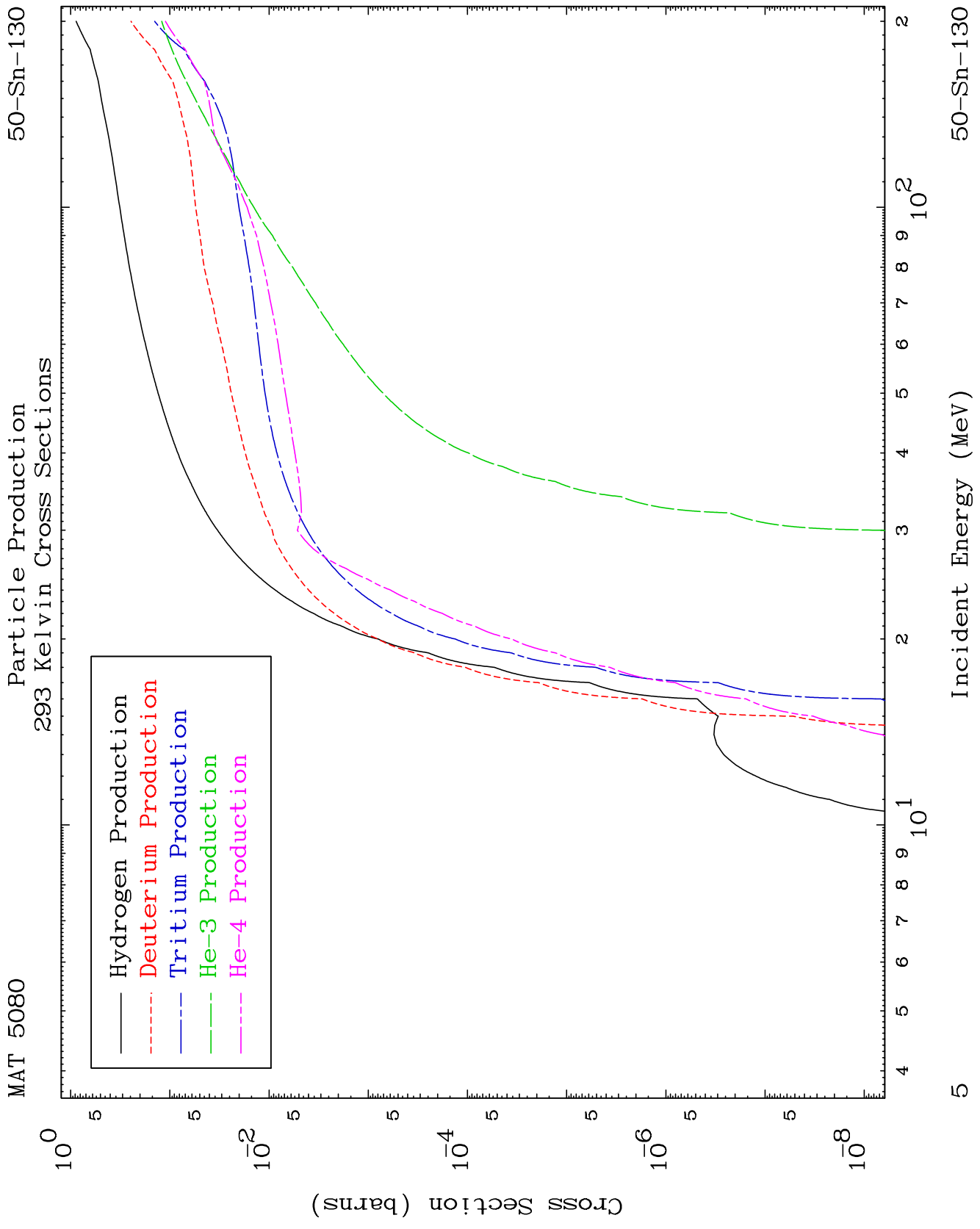


MAT 5080

Charged Particle  
293 Kelvin Cross Sections

50-Sn-130



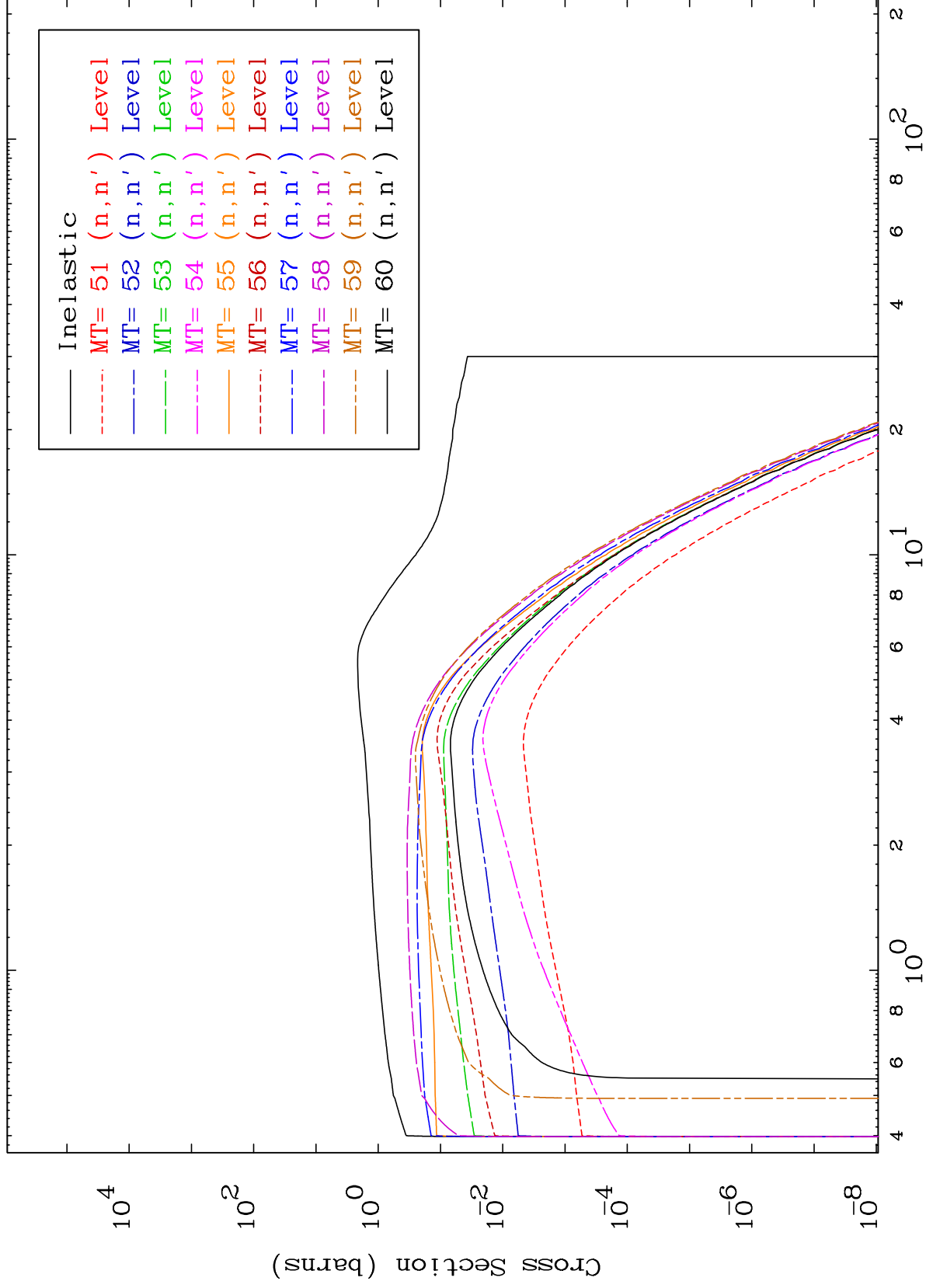


MAT 5080

(n,n') Level

50-Sn-130

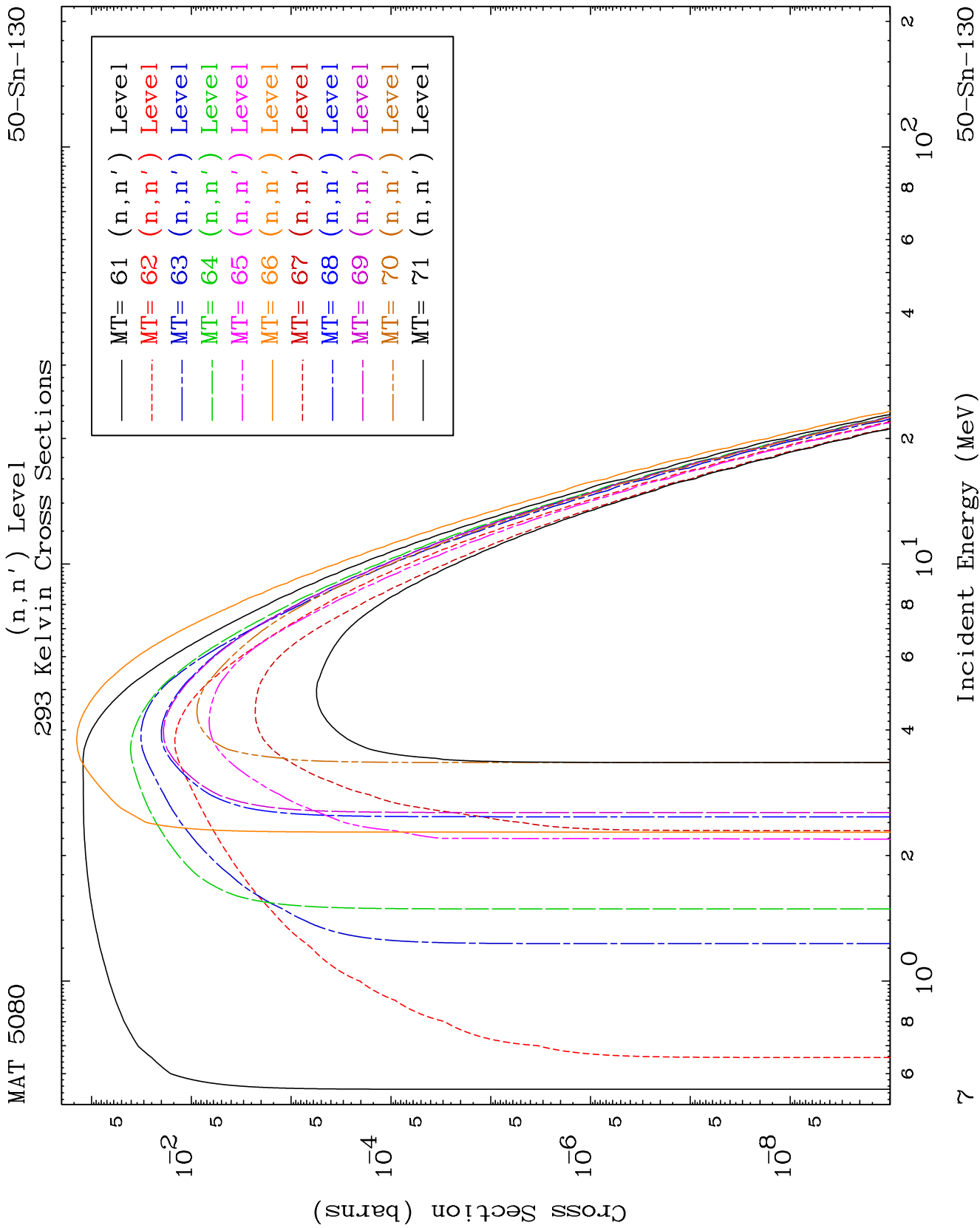
293 Kelvin Cross Sections



Incident Energy (MeV)

50-Sn-130

6



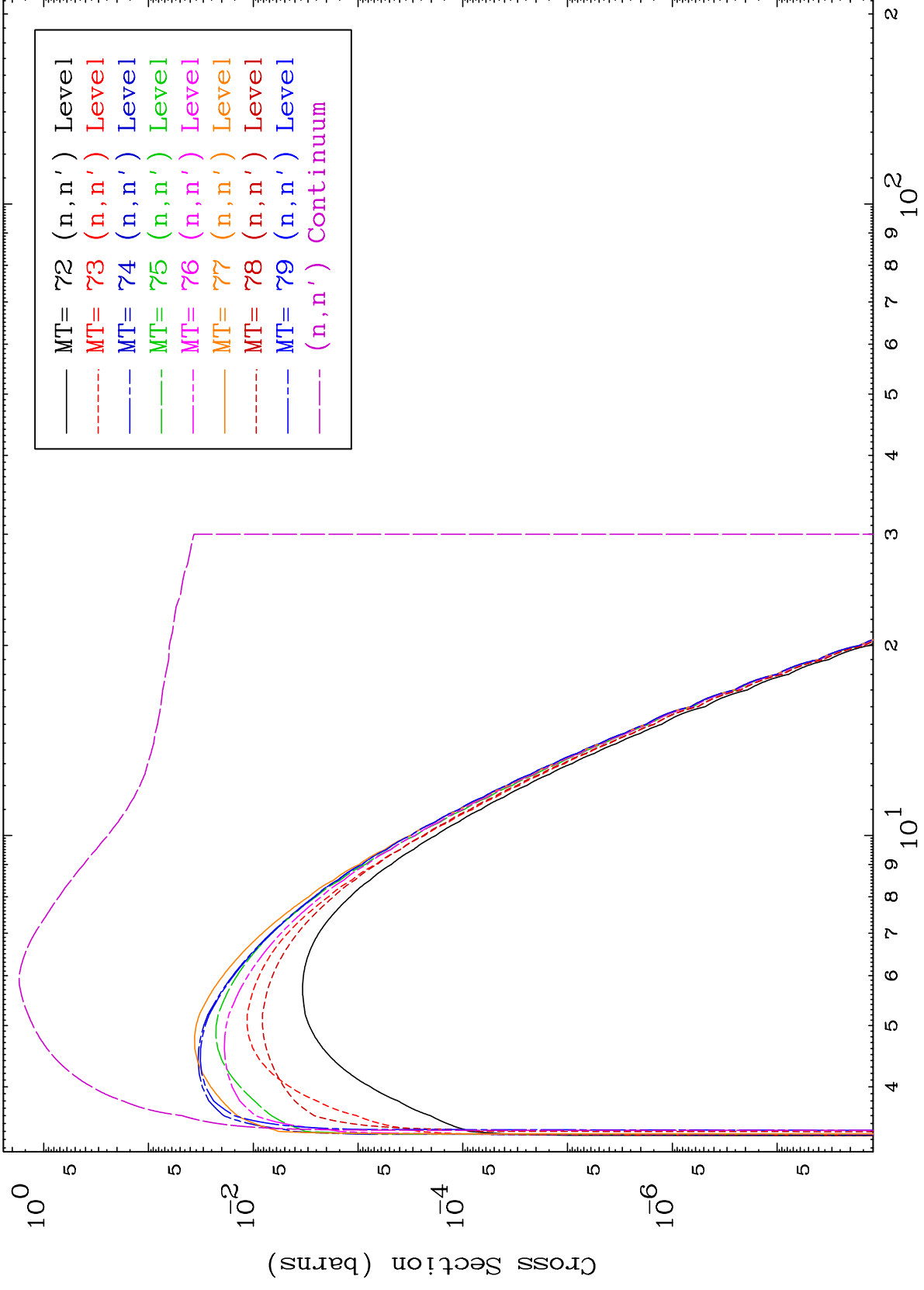


MAT 5080

(n,n') Level

50-Sn-130

293 Kelvin Cross Sections



8

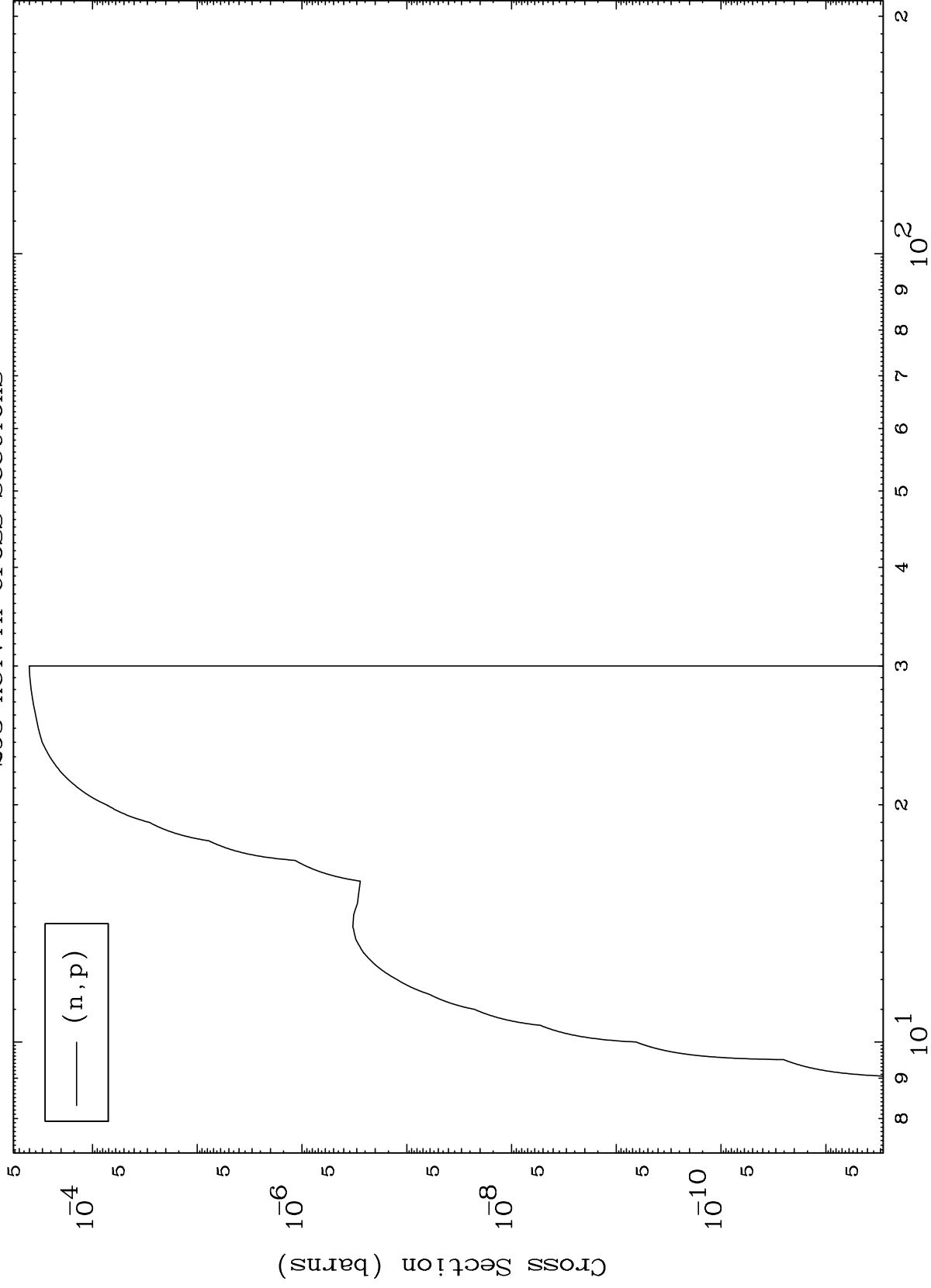
Incident Energy (MeV)

50-Sn-130

MAT 5080

(n,p) Levels  
293 Kelvin Cross Sections

50-Sn-130



9

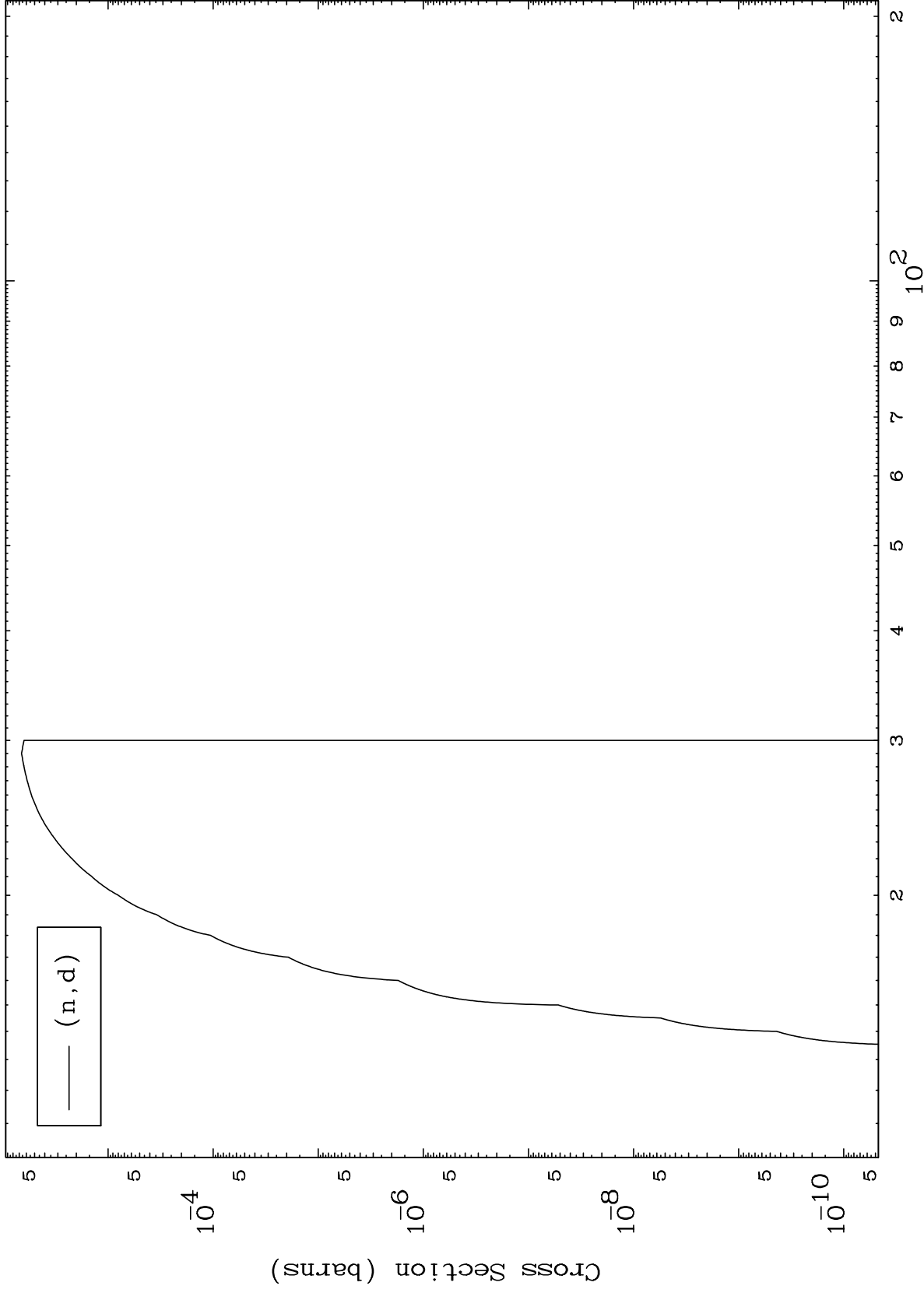
Incident Energy (MeV)

50-Sn-130

MAT 5080

(n,d) Levels  
293 Kelvin Cross Sections

50-Sn-130



10

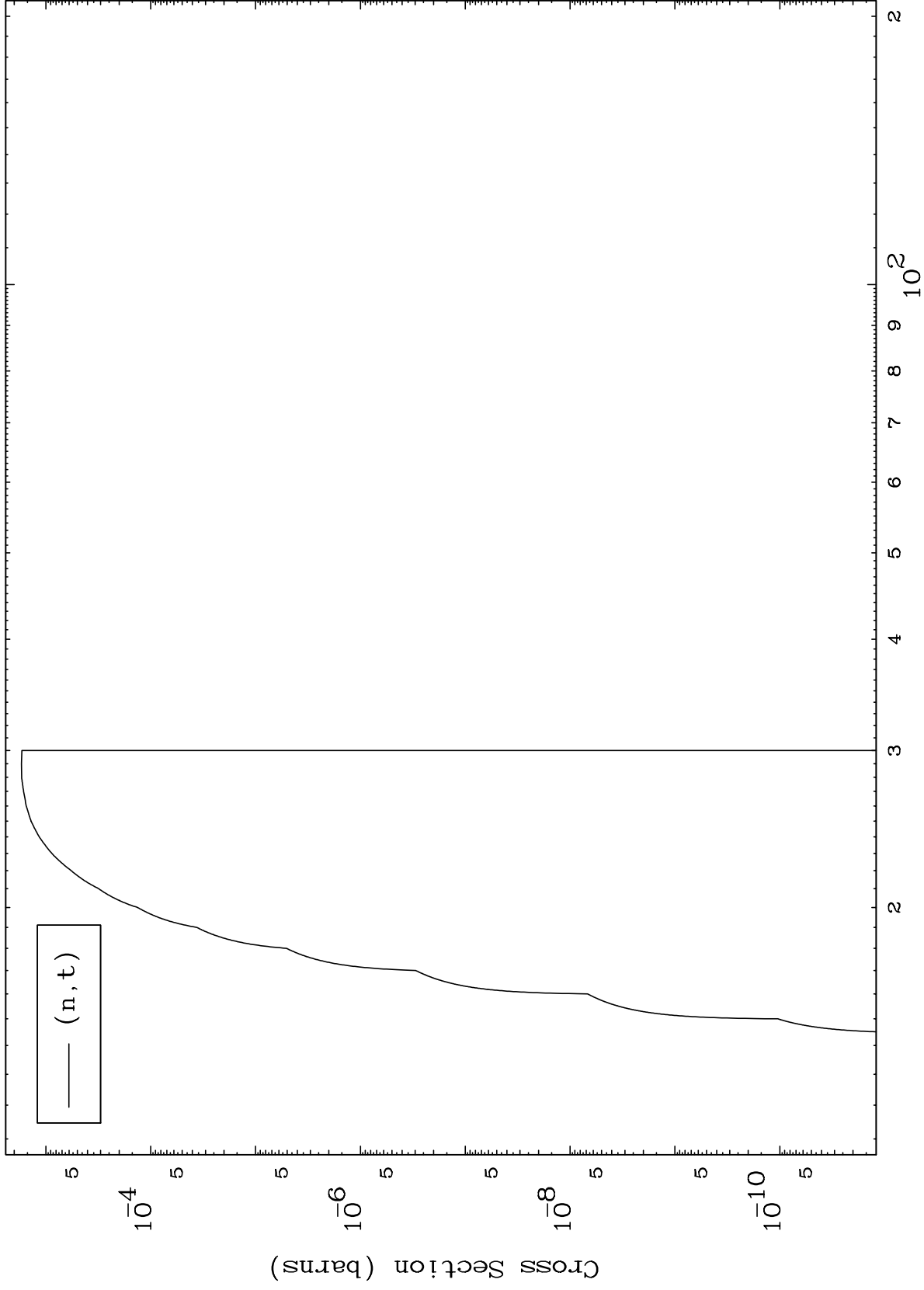
Incident Energy (MeV)

50-Sn-130

MAT 5080

(n,t) Levels  
293 Kelvin Cross Sections

50-Sn-130



11

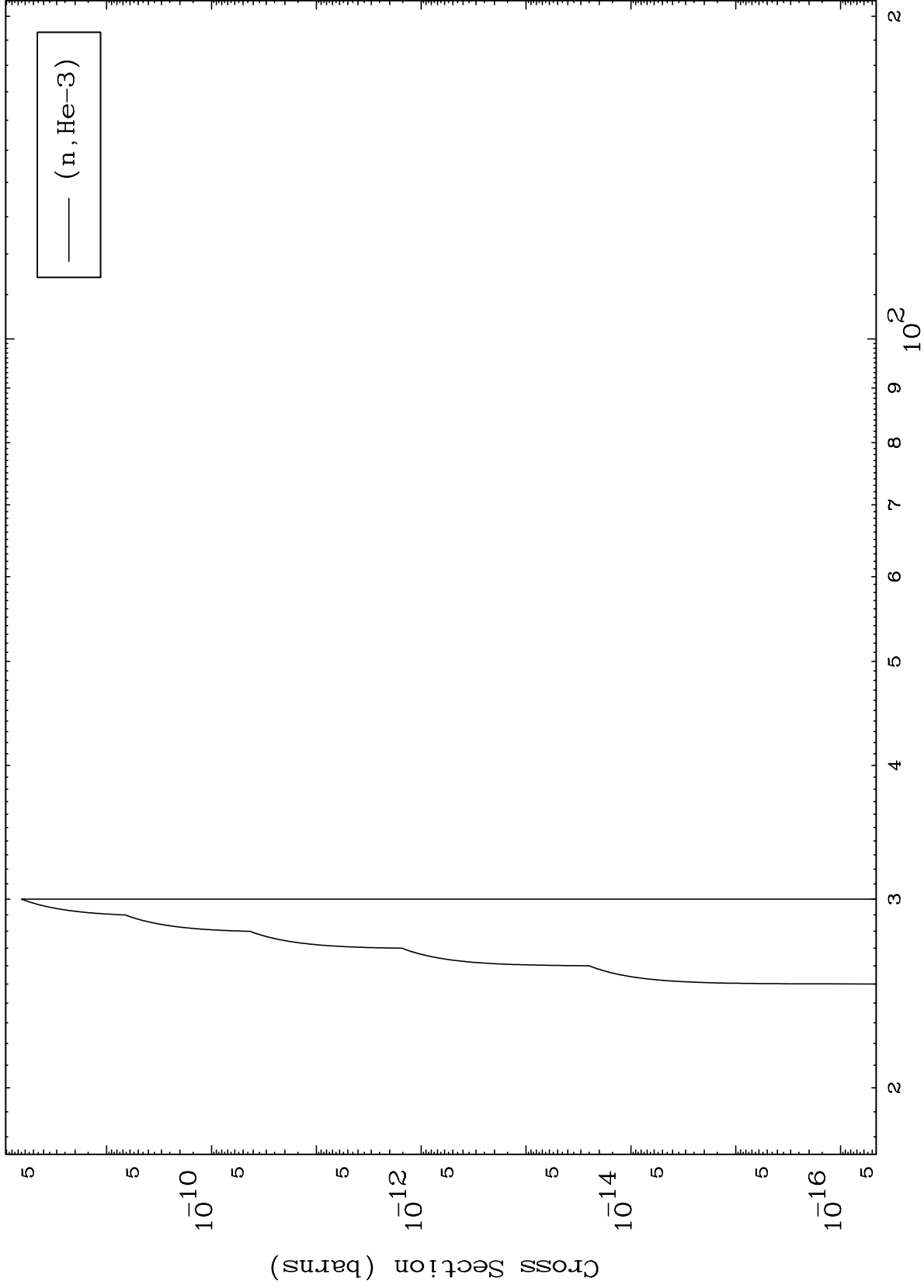
Incident Energy (MeV)

50-Sn-130

MAT 5080

(n,He3) Levels  
293 Kelvin Cross Sections

50-Sn-130



12

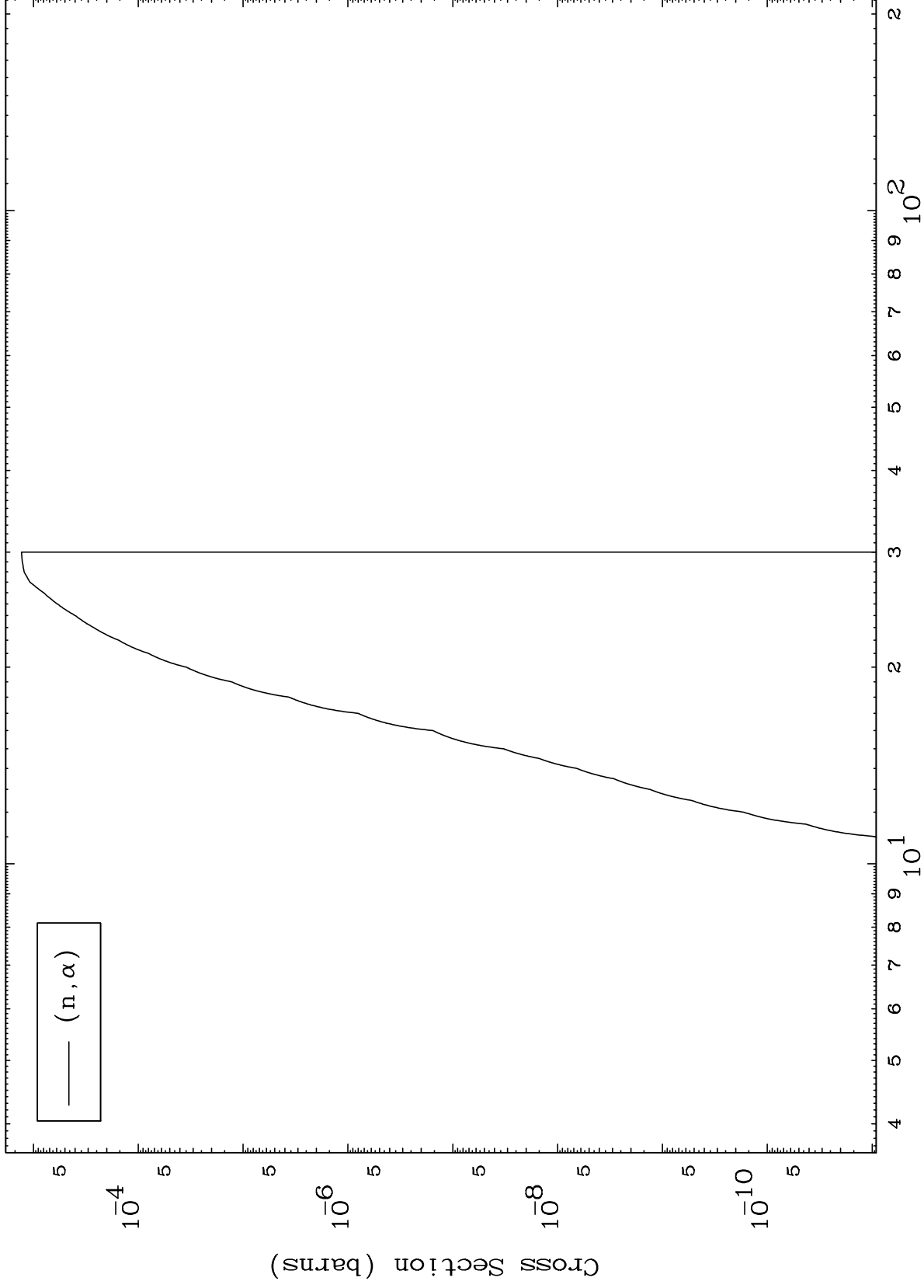
Incident Energy (MeV)

50-Sn-130

MAT 5080

(n,α) Levels  
293 Kelvin Cross Sections

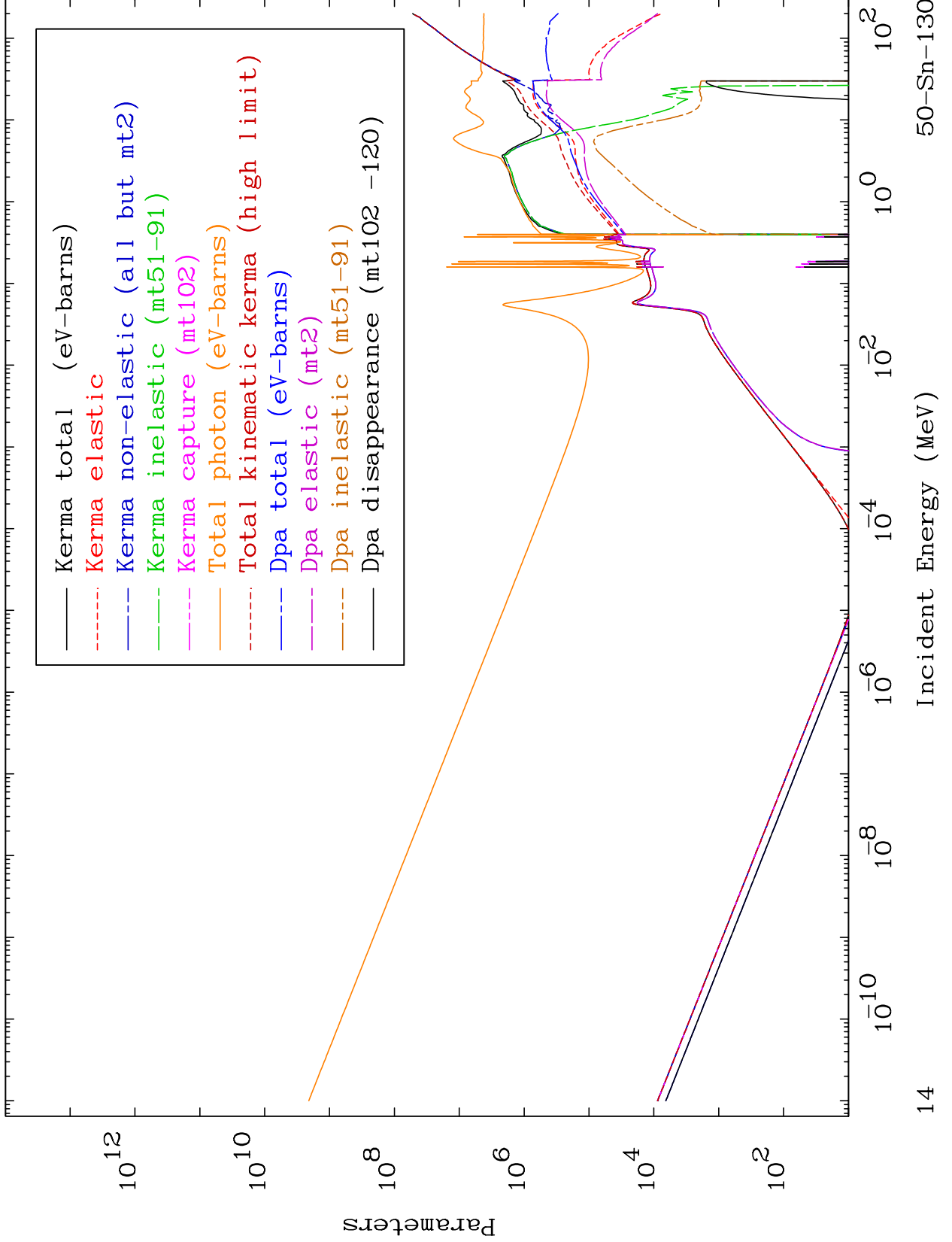
50-Sn-130

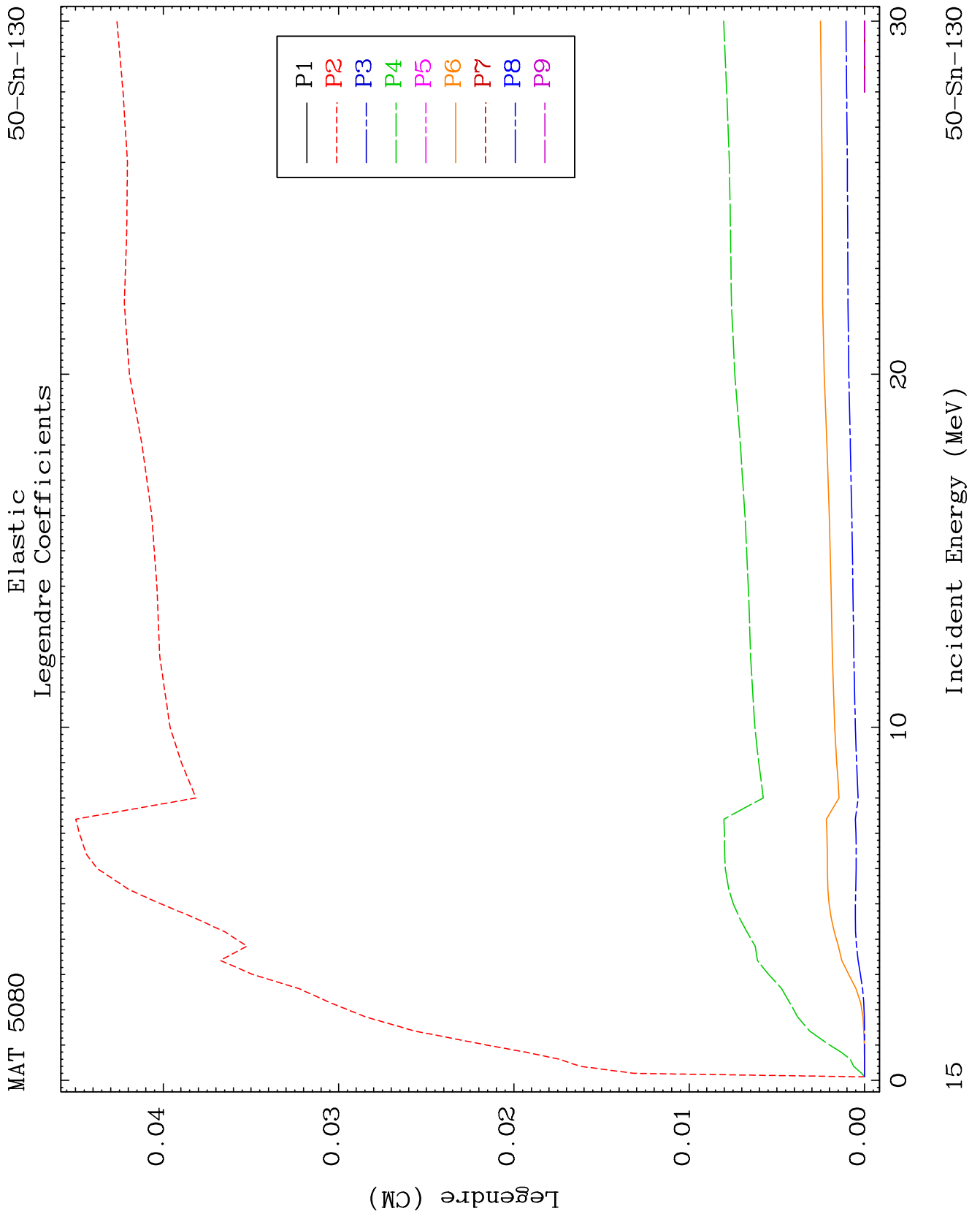


13

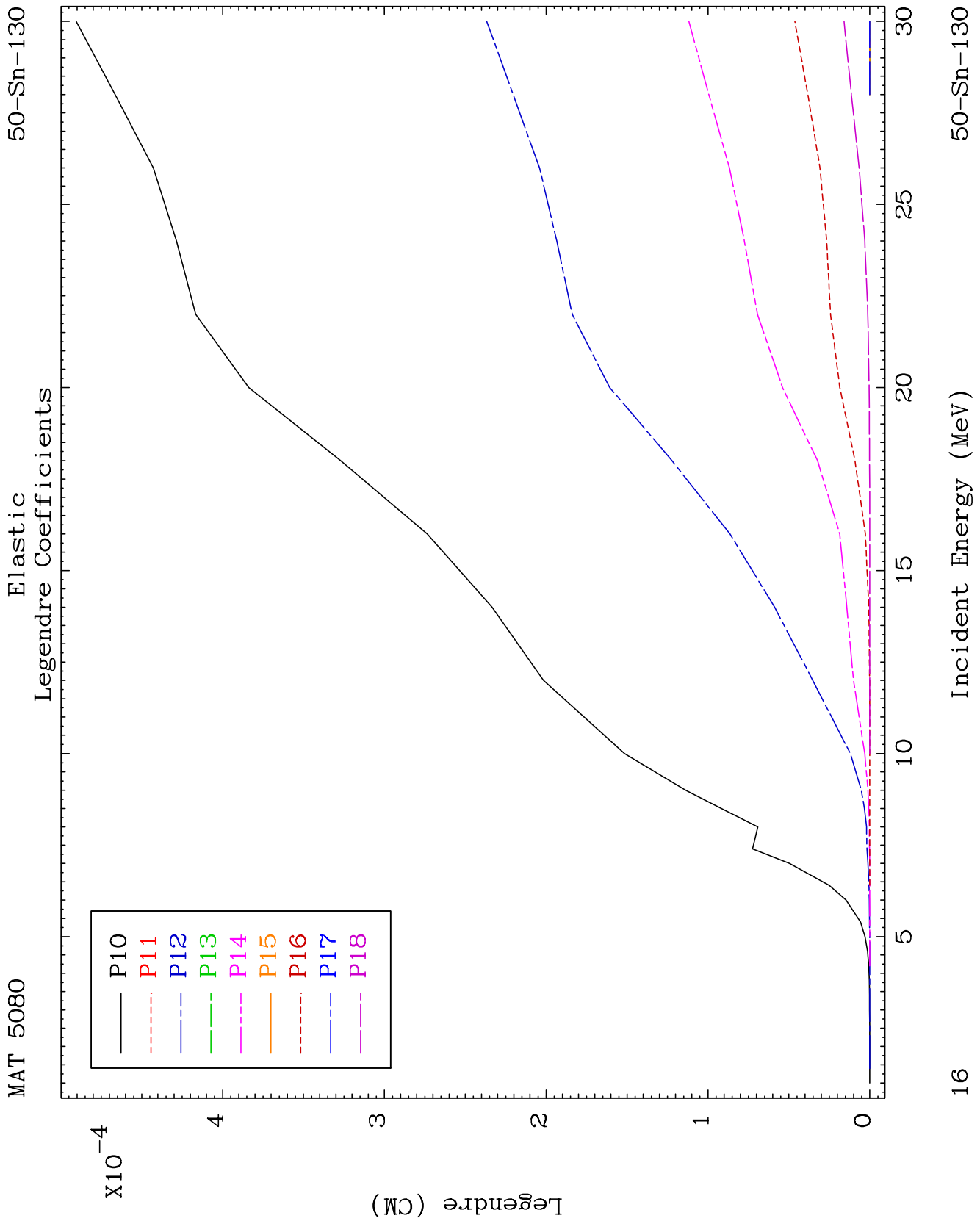
Incident Energy (MeV)

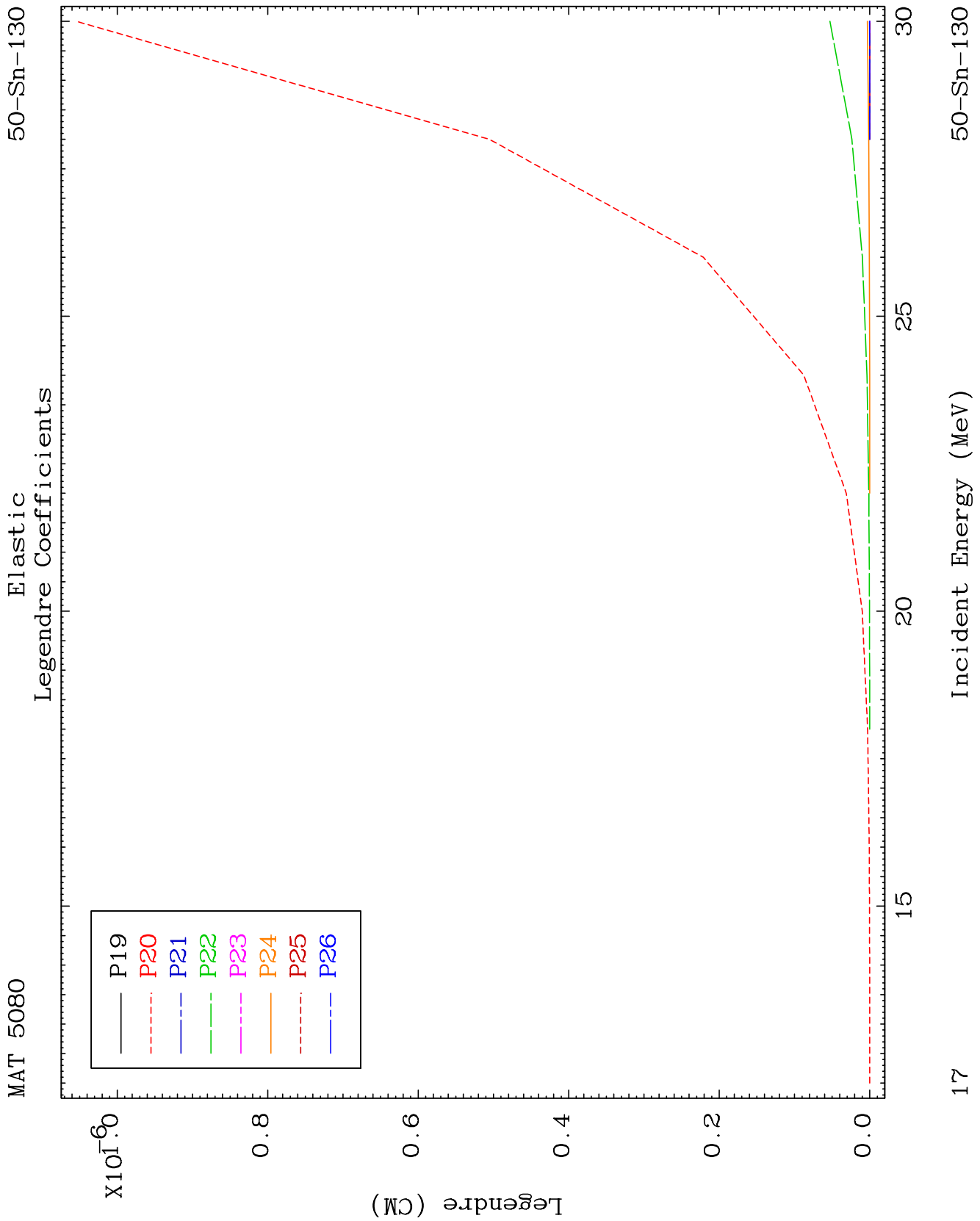
50-Sn-130

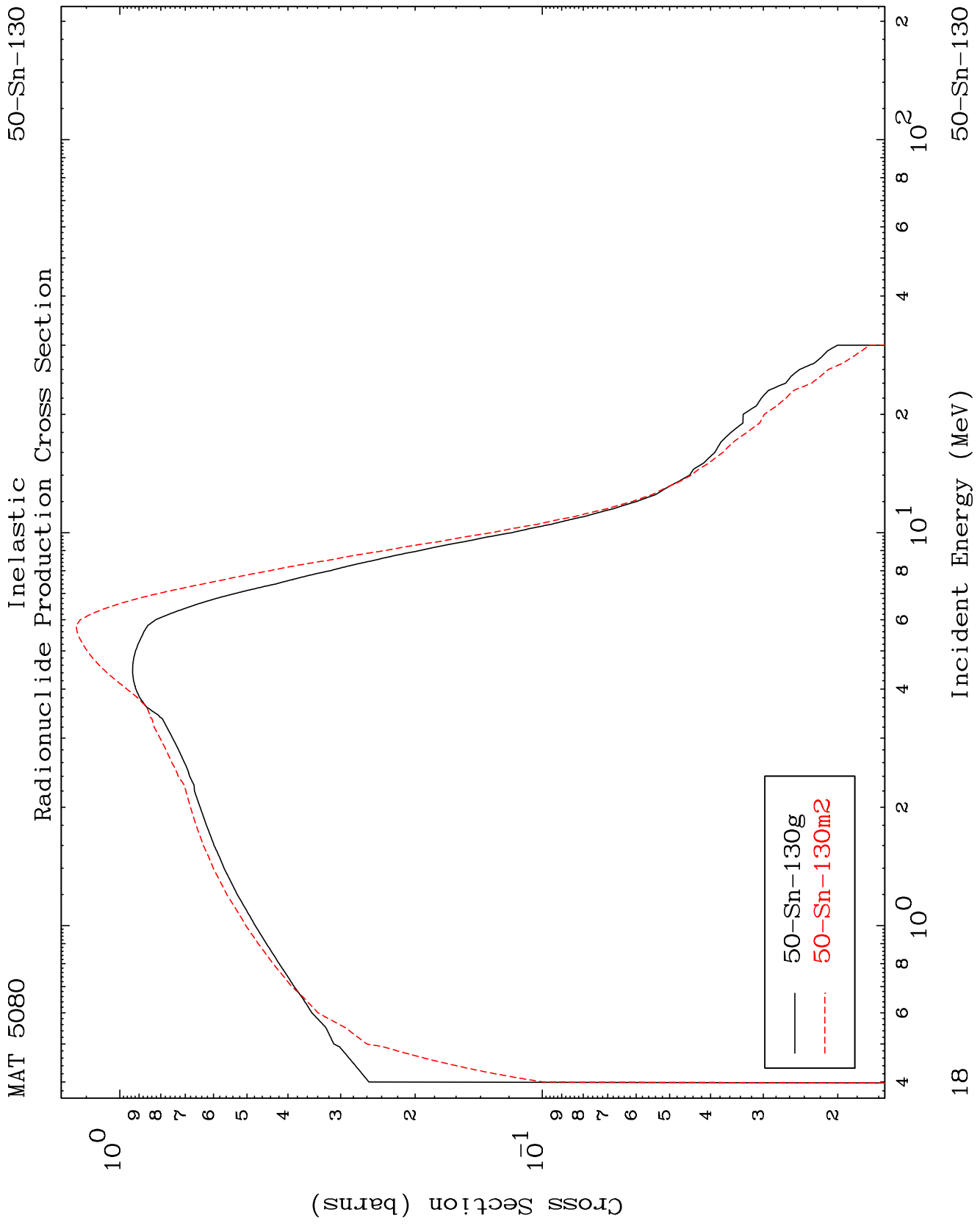










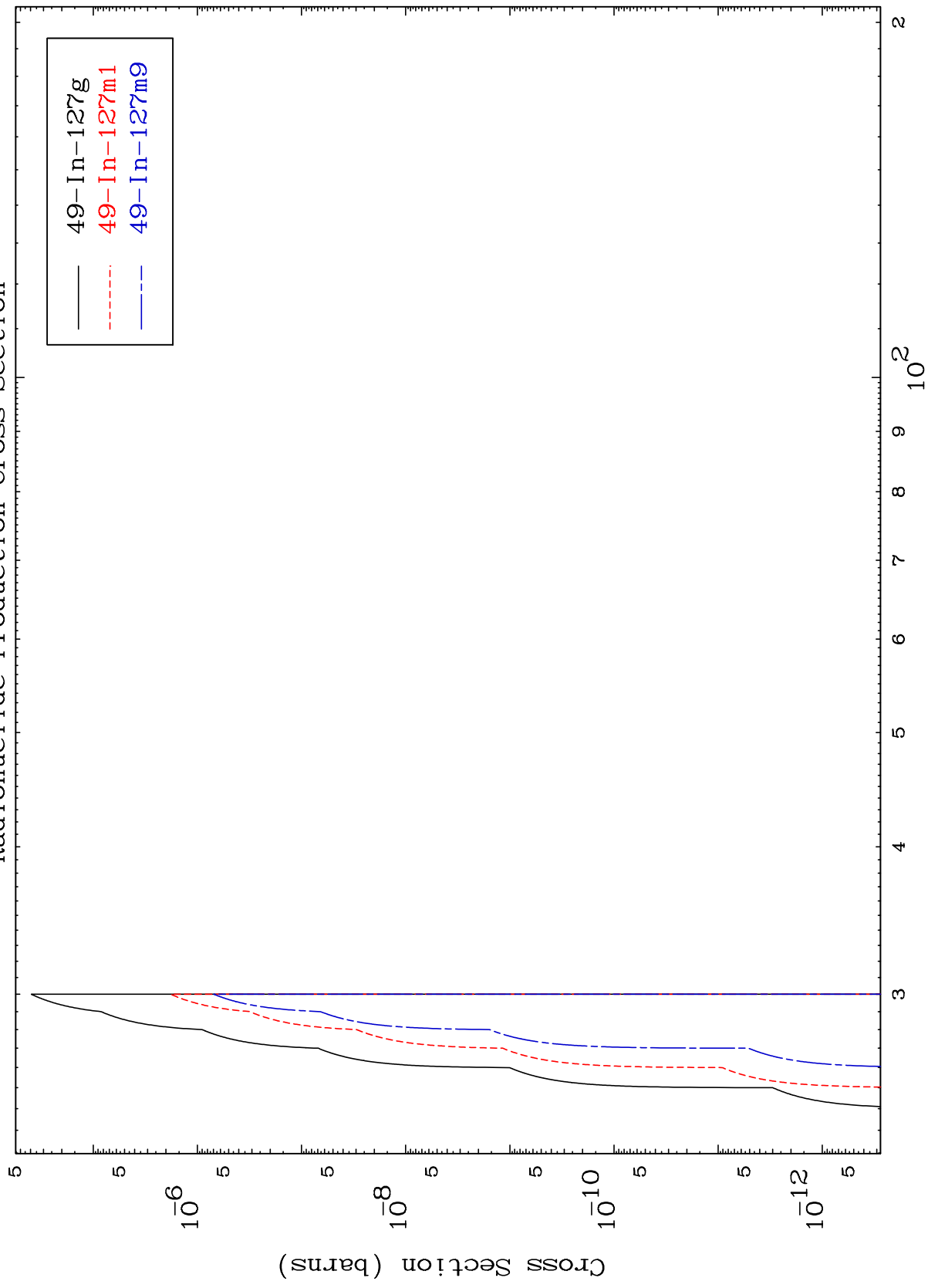


MAT 5080

(n,2n) d

50-Sn-130

Radionuclide Production Cross Section



19

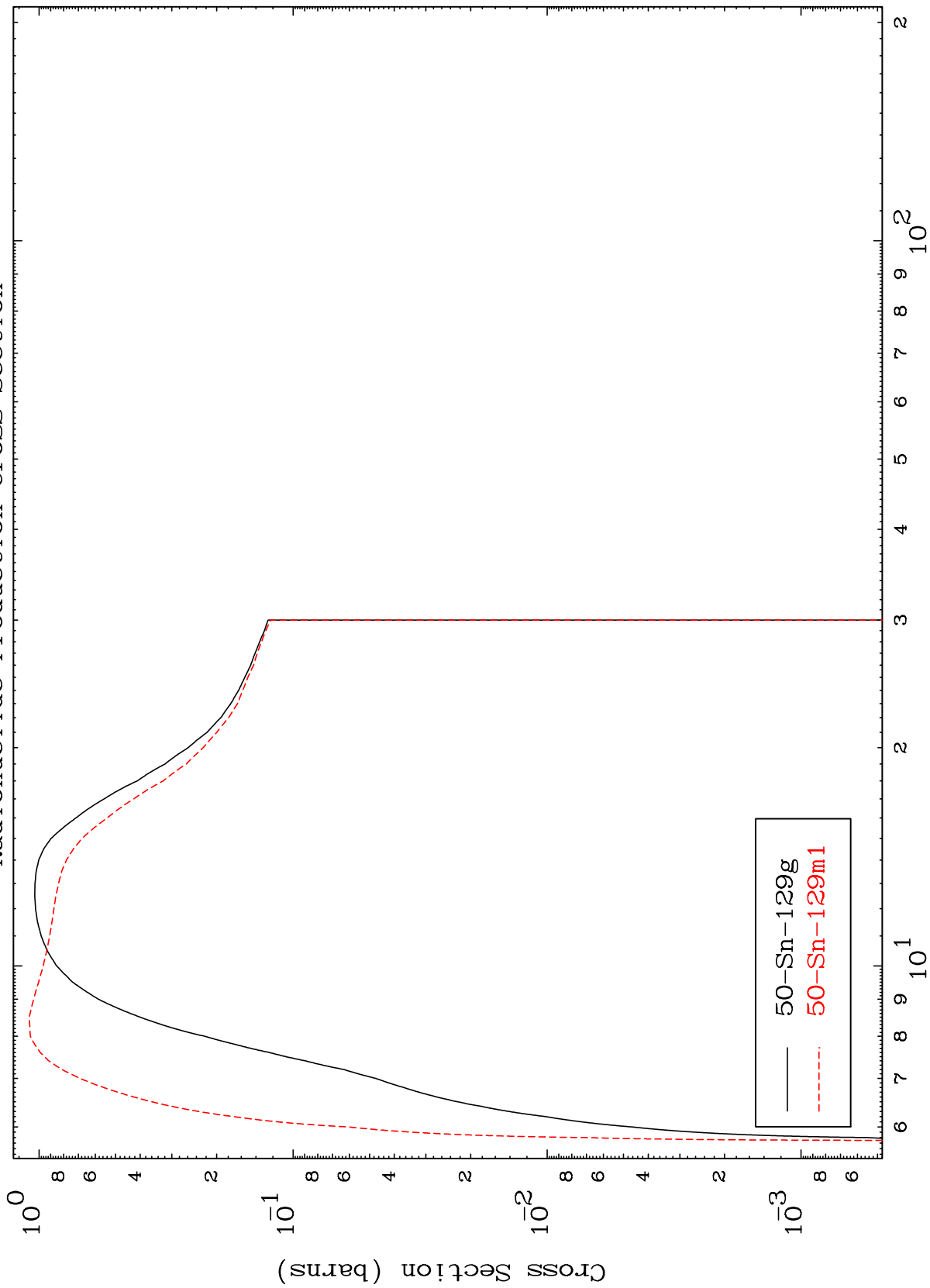
Incident Energy (MeV)

50-Sn-130

MAT 5080

50-Sn-130

Radionuclide Production Cross Section  
(n,2n)



50-Sn-130

Incident Energy (MeV)

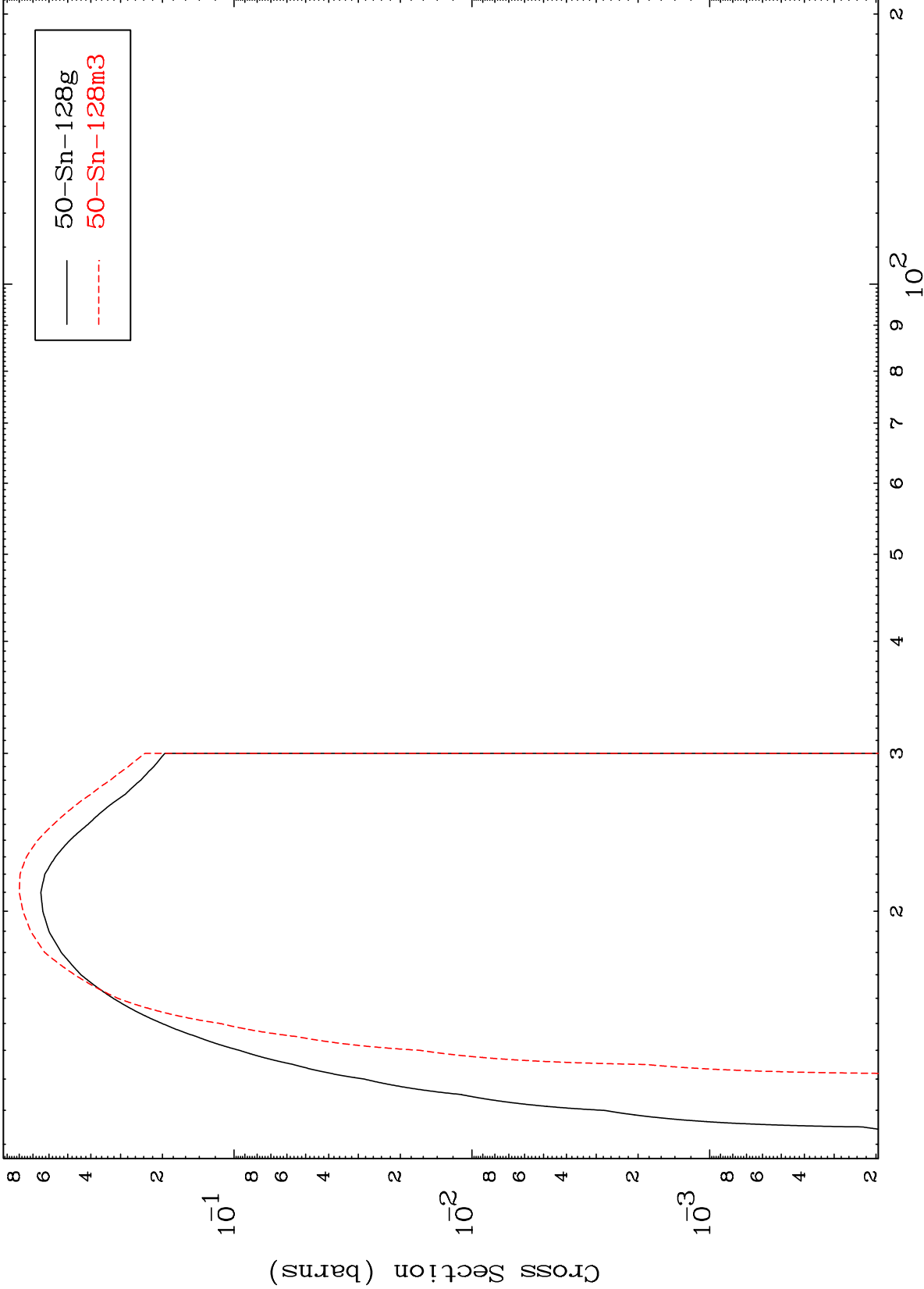
20

MAT 5080

(n,3n)

50-Sn-130

Radionuclide Production Cross Section

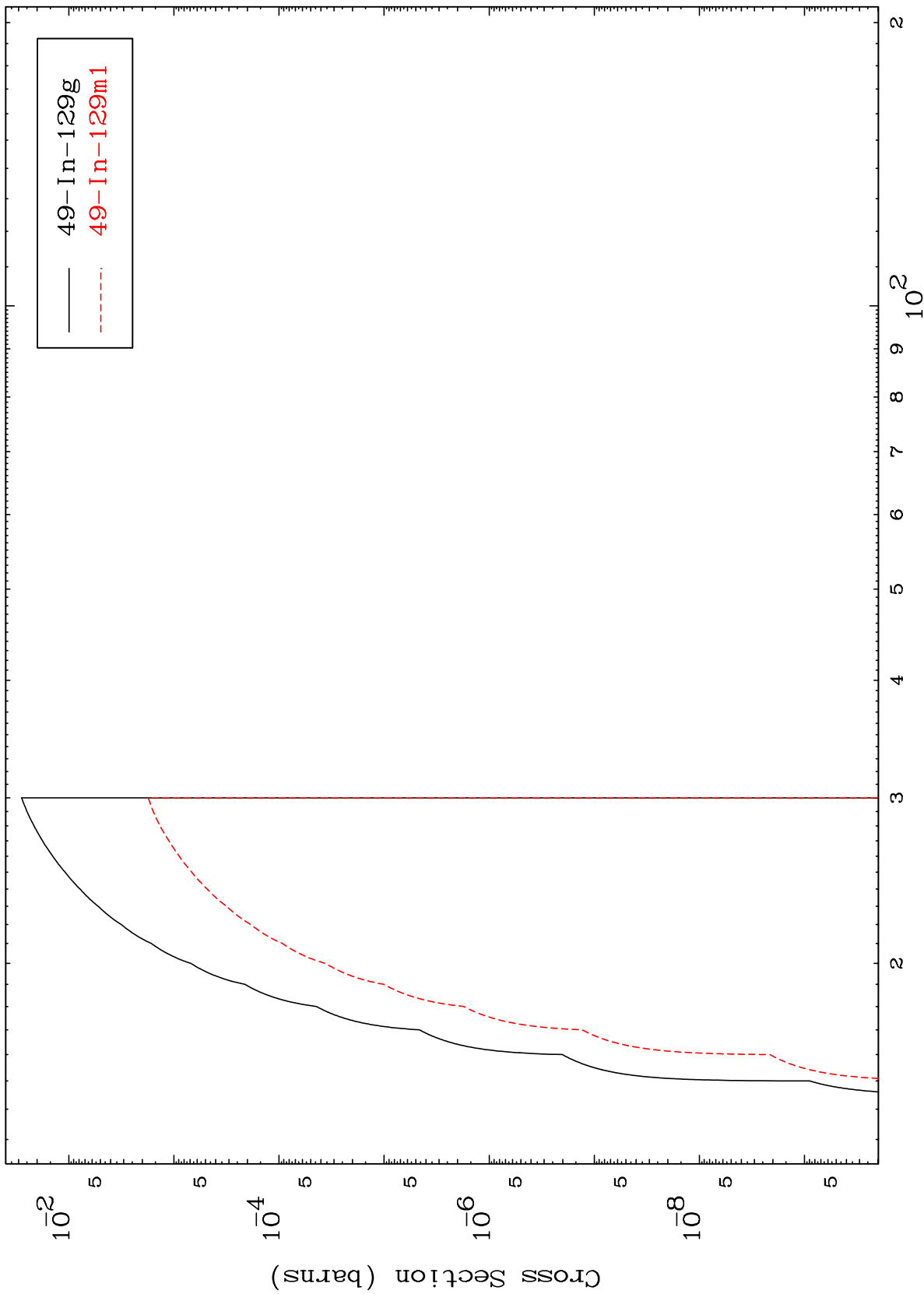


50-Sn-128g  
50-Sn-128m3

MAT 5080

50-Sn-130

$(n, n')$  p  
Radionuclide Production Cross Section

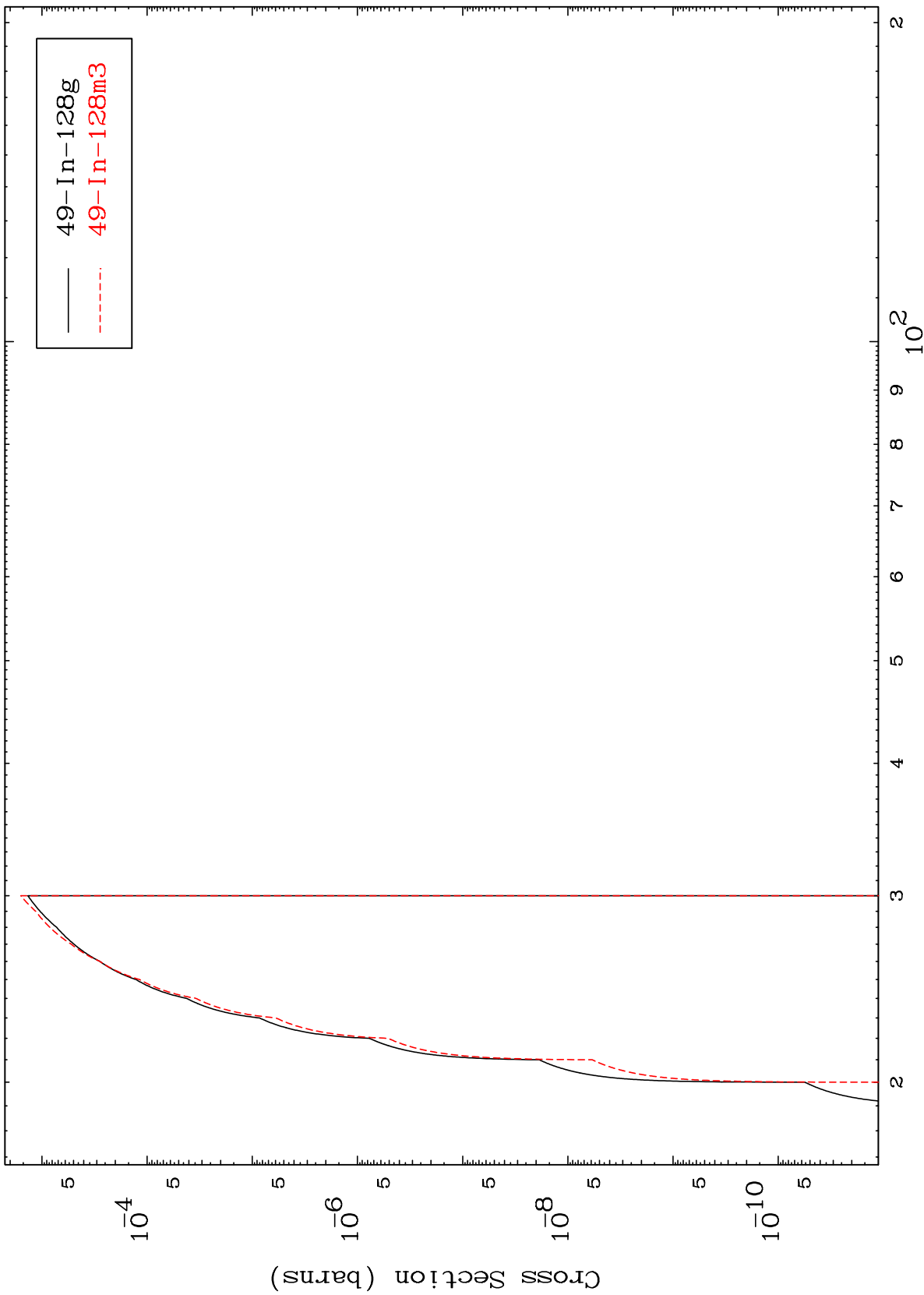


50-Sn-130

Incident Energy (MeV)

22

Radionuclide Production Cross Section



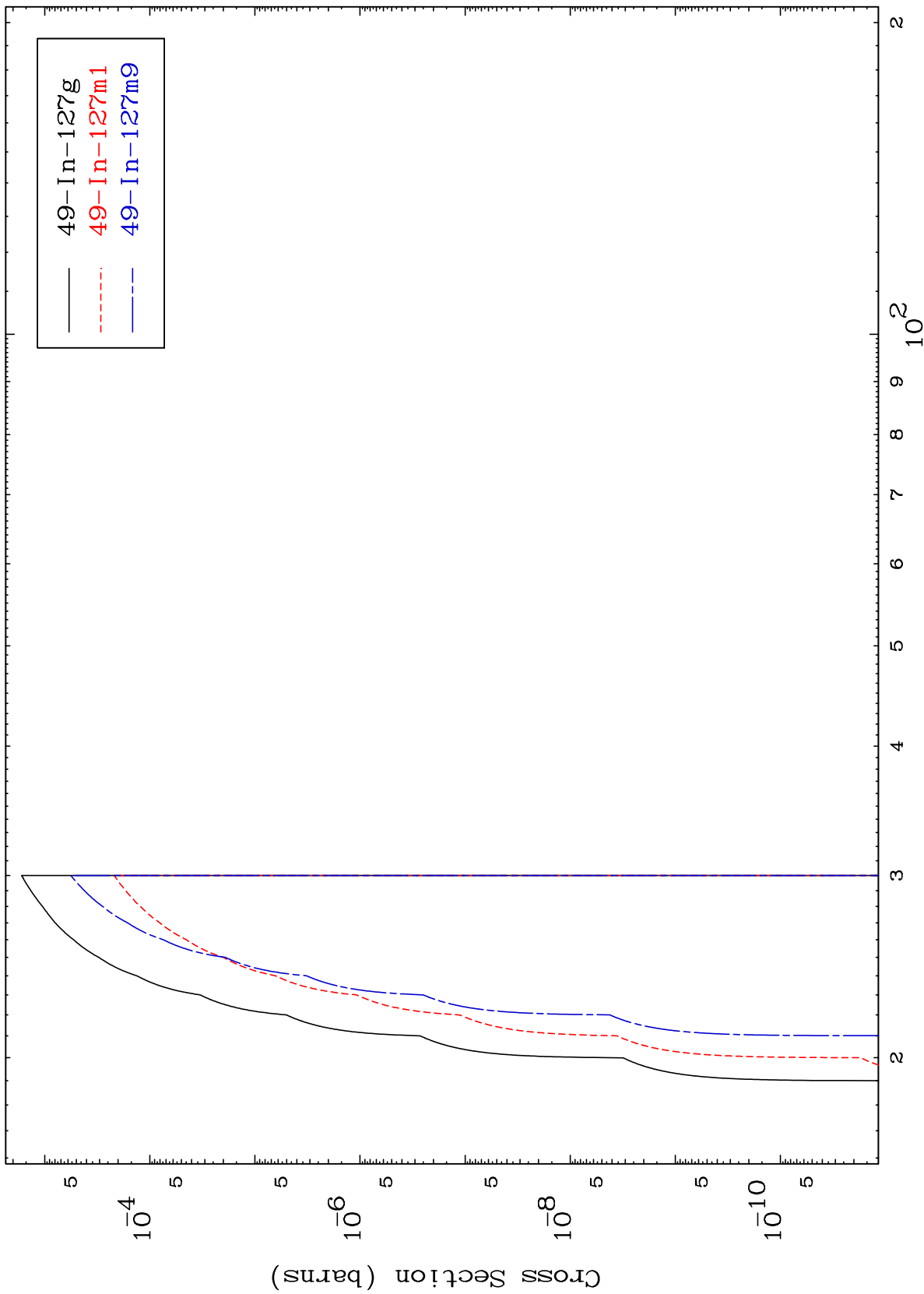


MAT 5080

(n,n') t

50-Sn-130

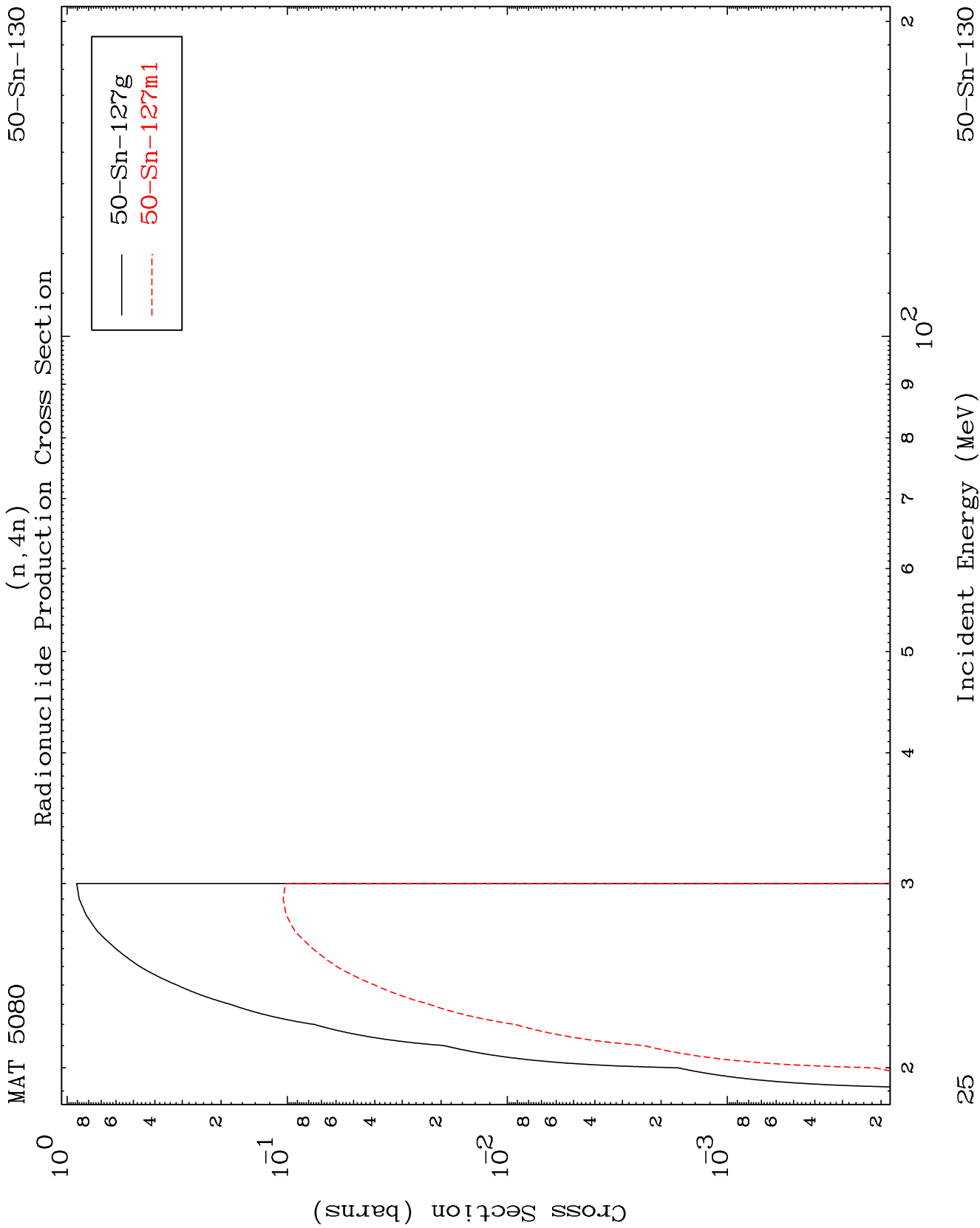
Radionuclide Production Cross Section



24

Incident Energy (MeV)

50-Sn-130

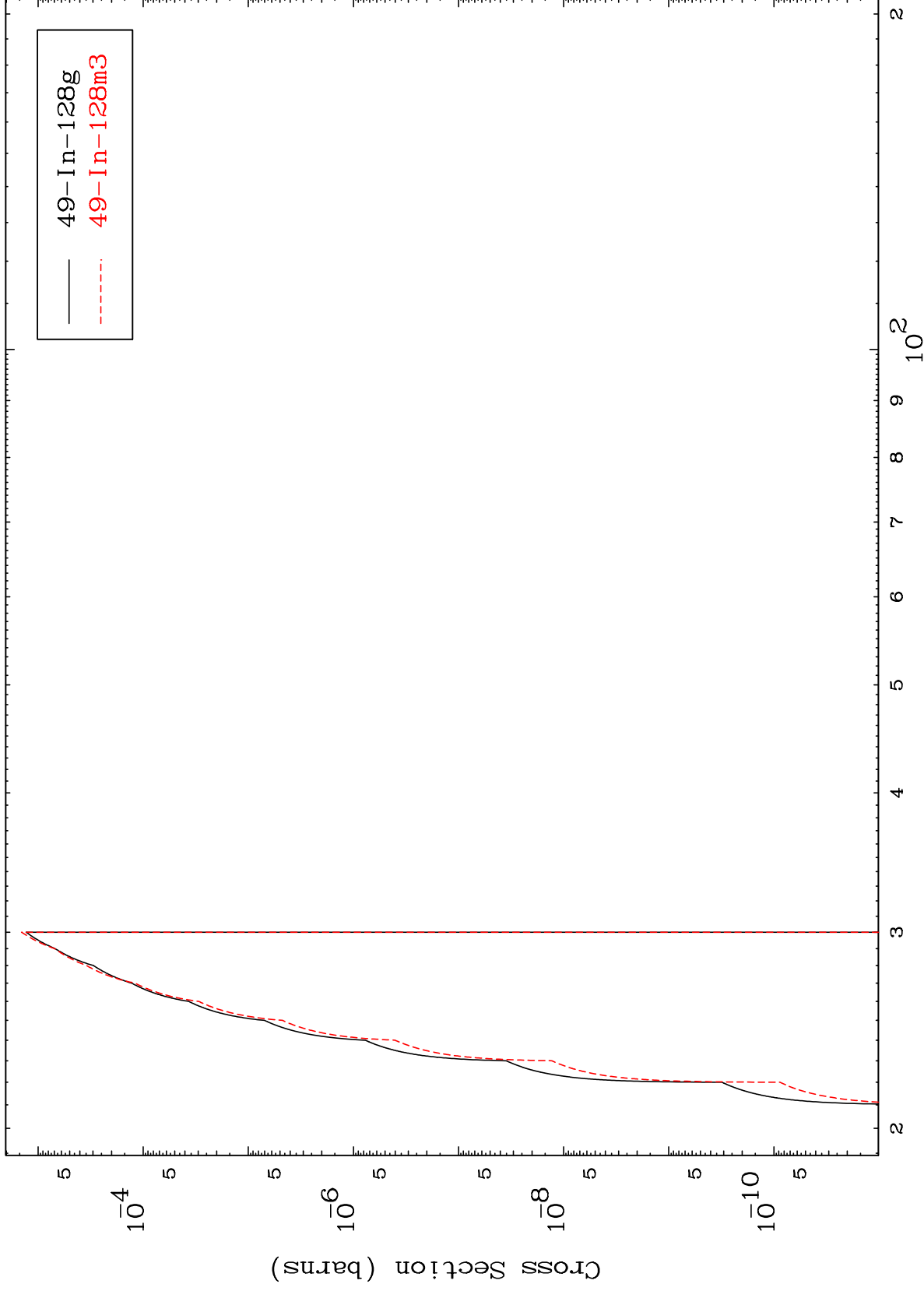


MAT 5080

(n,2n) p

50-Sn-130

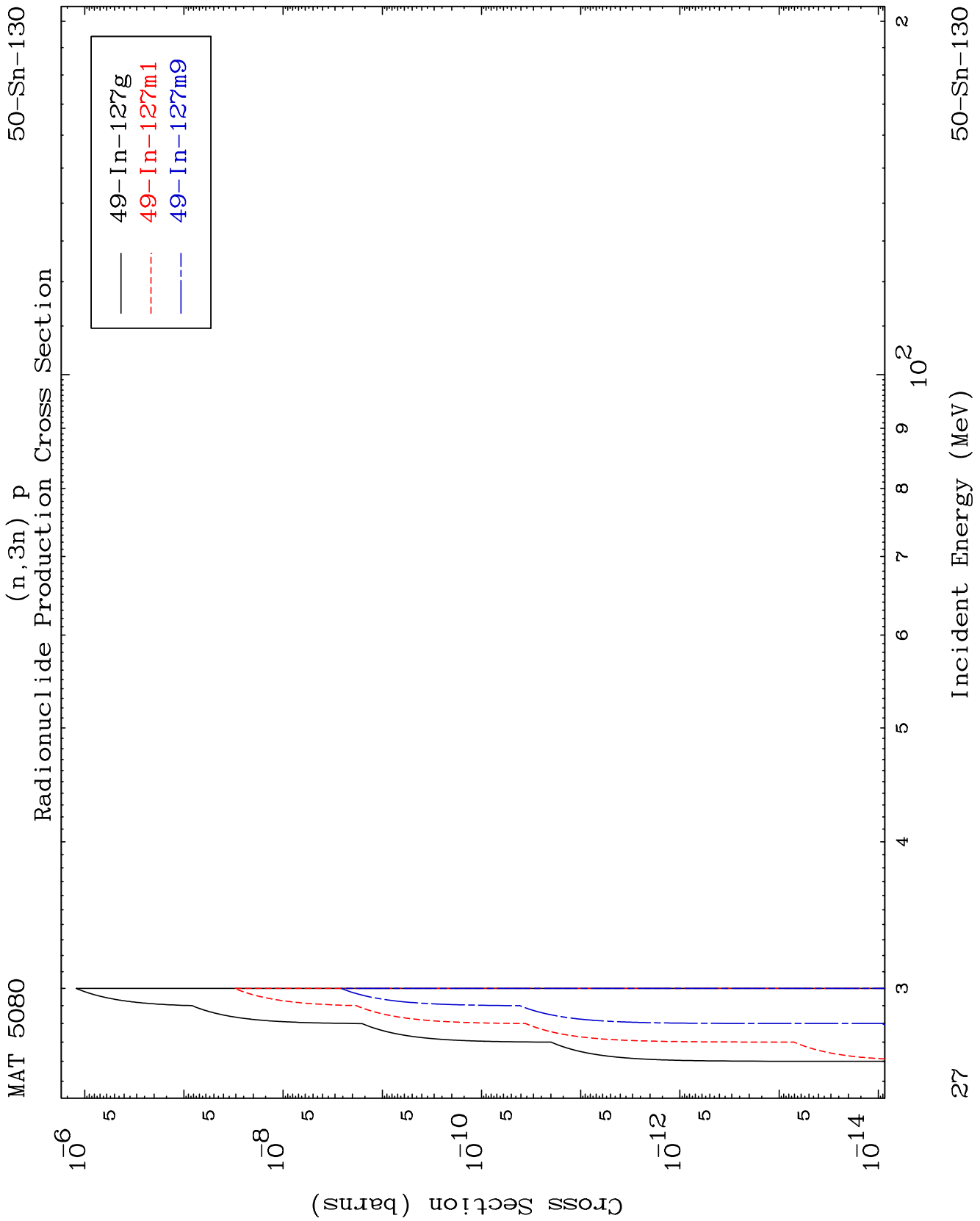
Radionuclide Production Cross Section



26

Incident Energy (MeV)

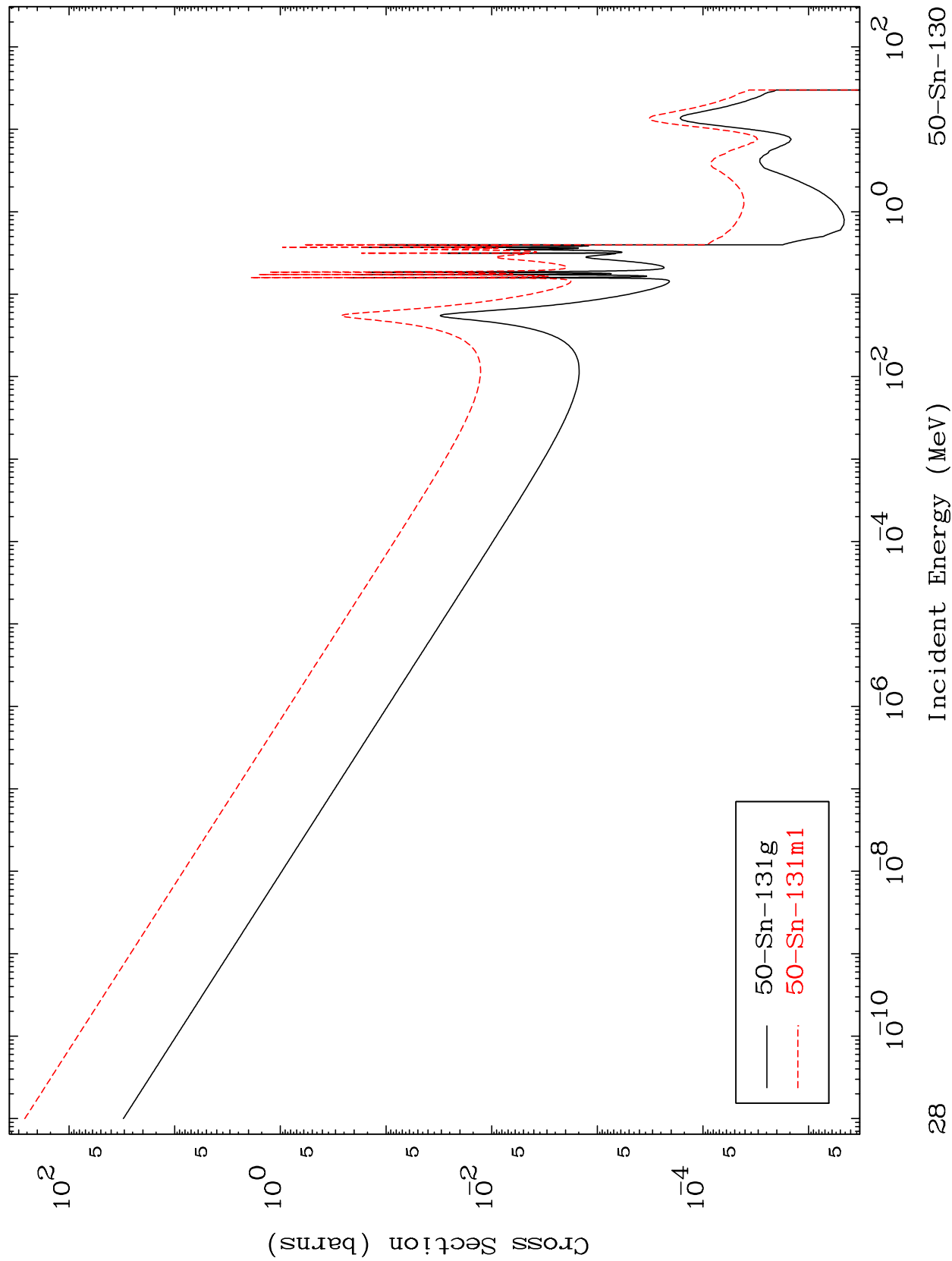
50-Sn-130



MAT 5080

50-Sn-130

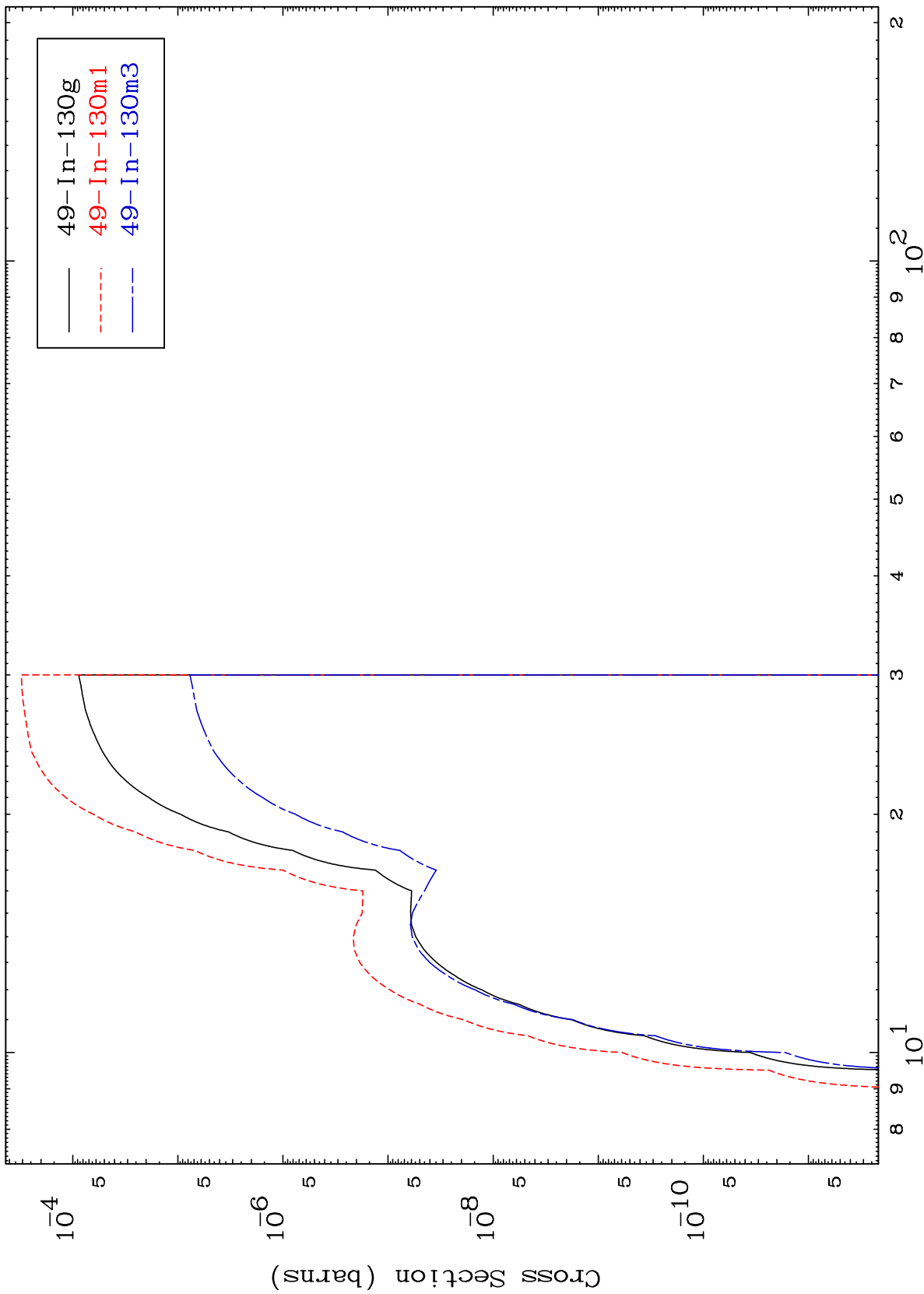
$(n, \gamma)$   
Radionuclide Production Cross Section



MAT 5080

50-Sn-130

(n,p)  
Radionuclide Production Cross Section



29

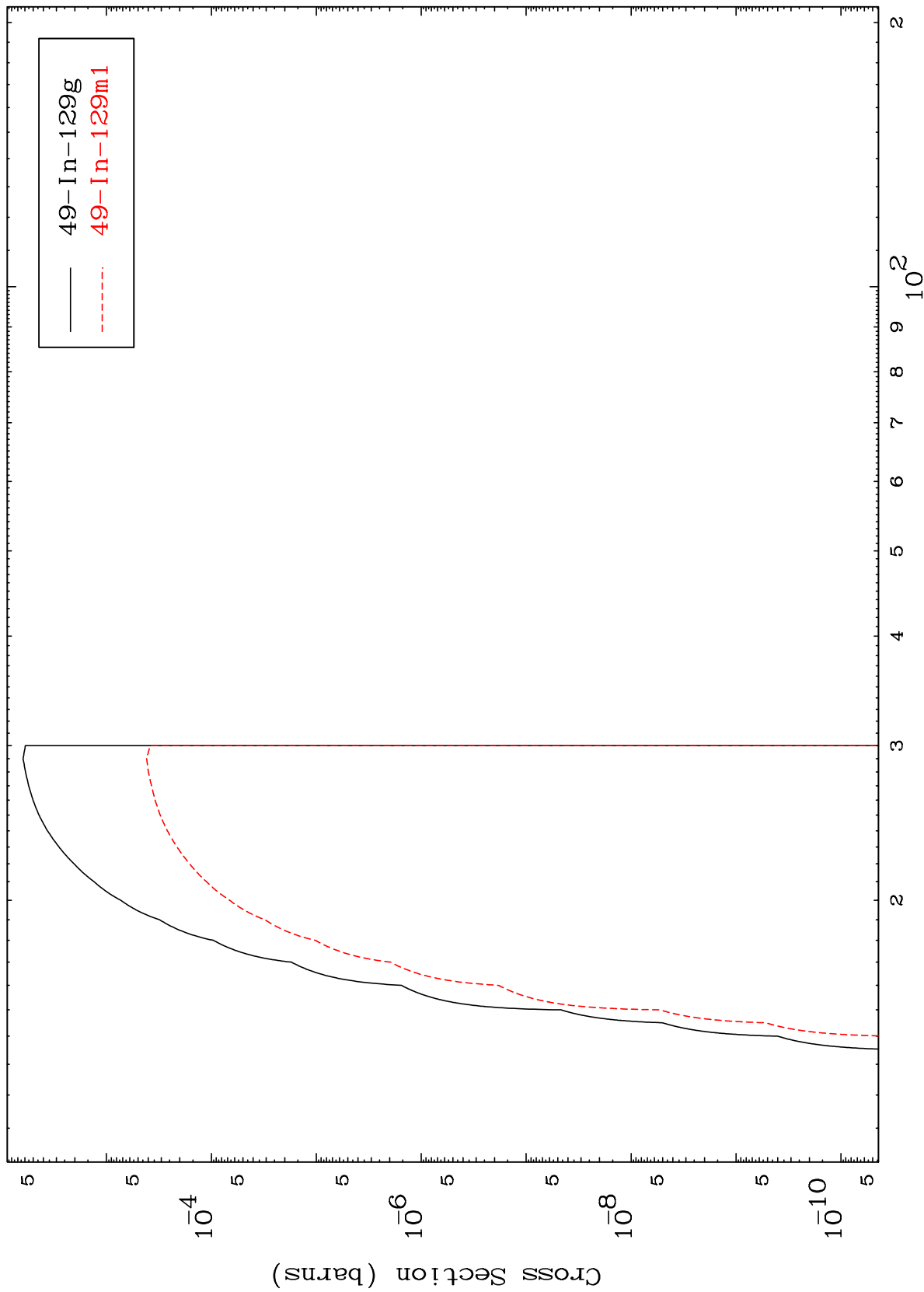
Incident Energy (MeV)

50-Sn-130

MAT 5080

50-Sn-130

(n,d)  
Radionuclide Production Cross Section



50-Sn-130

Incident Energy (MeV)

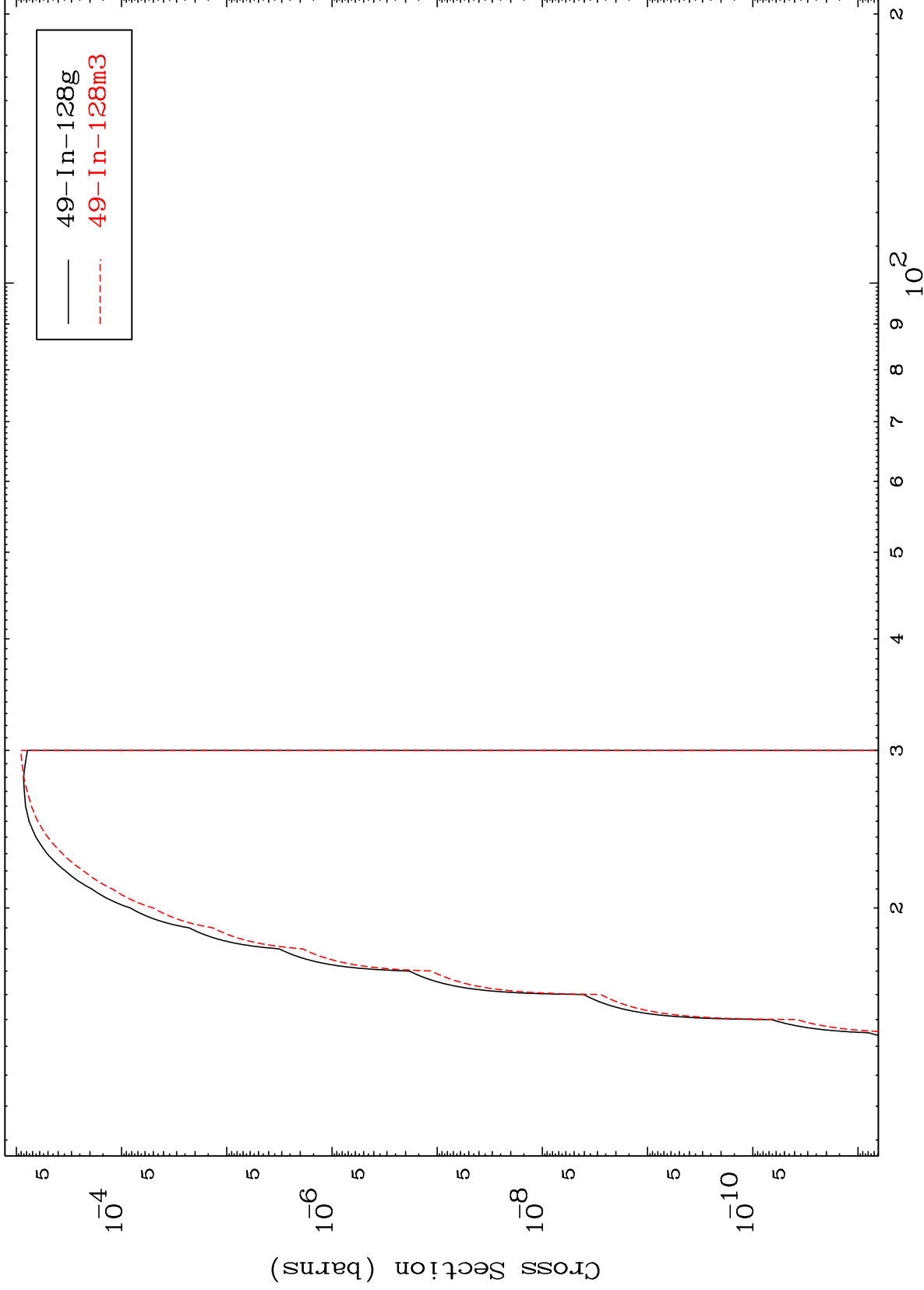
30

MAT 5080

(n, t)

50-Sn-130

Radionuclide Production Cross Section



31

Incident Energy (MeV)

50-Sn-130