

Program EVALPLOT
(Version 2017-1)

by

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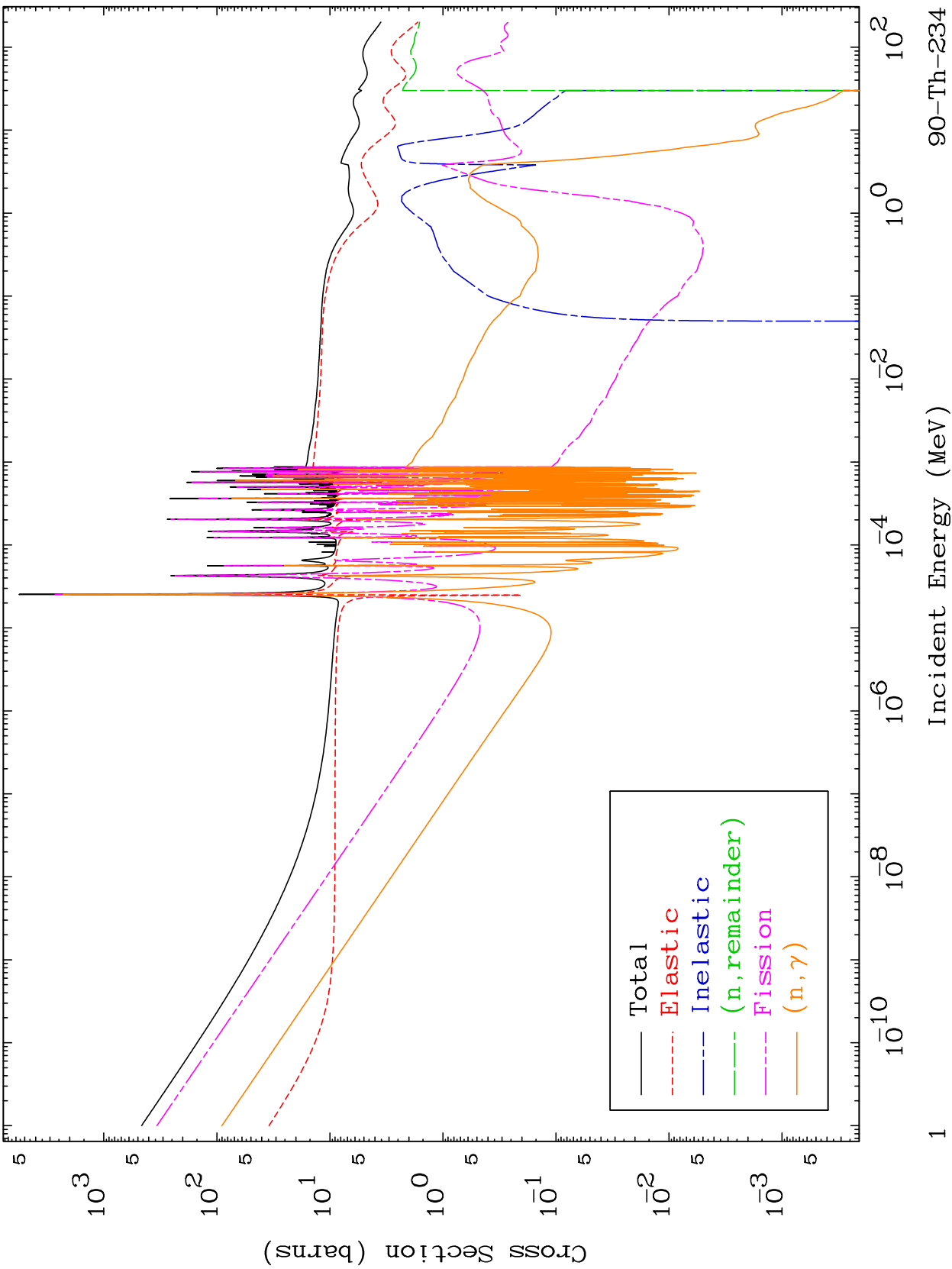
Press Mouse Button to Start

MAT 9046

Major

293 Kelvin Cross Sections

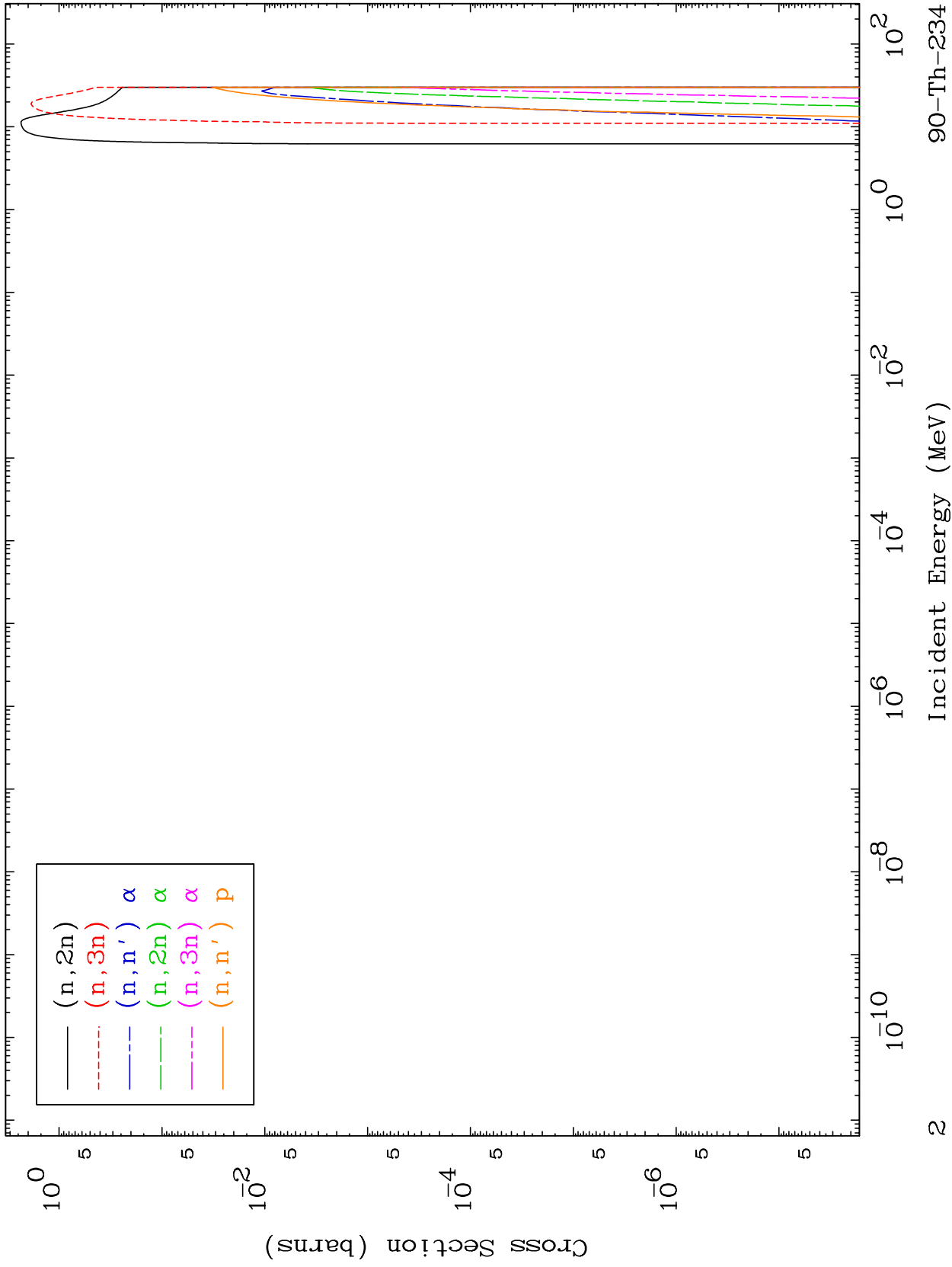
90-Th-234



MAT 9046

Neutron Production
293 Kelvin Cross Sections

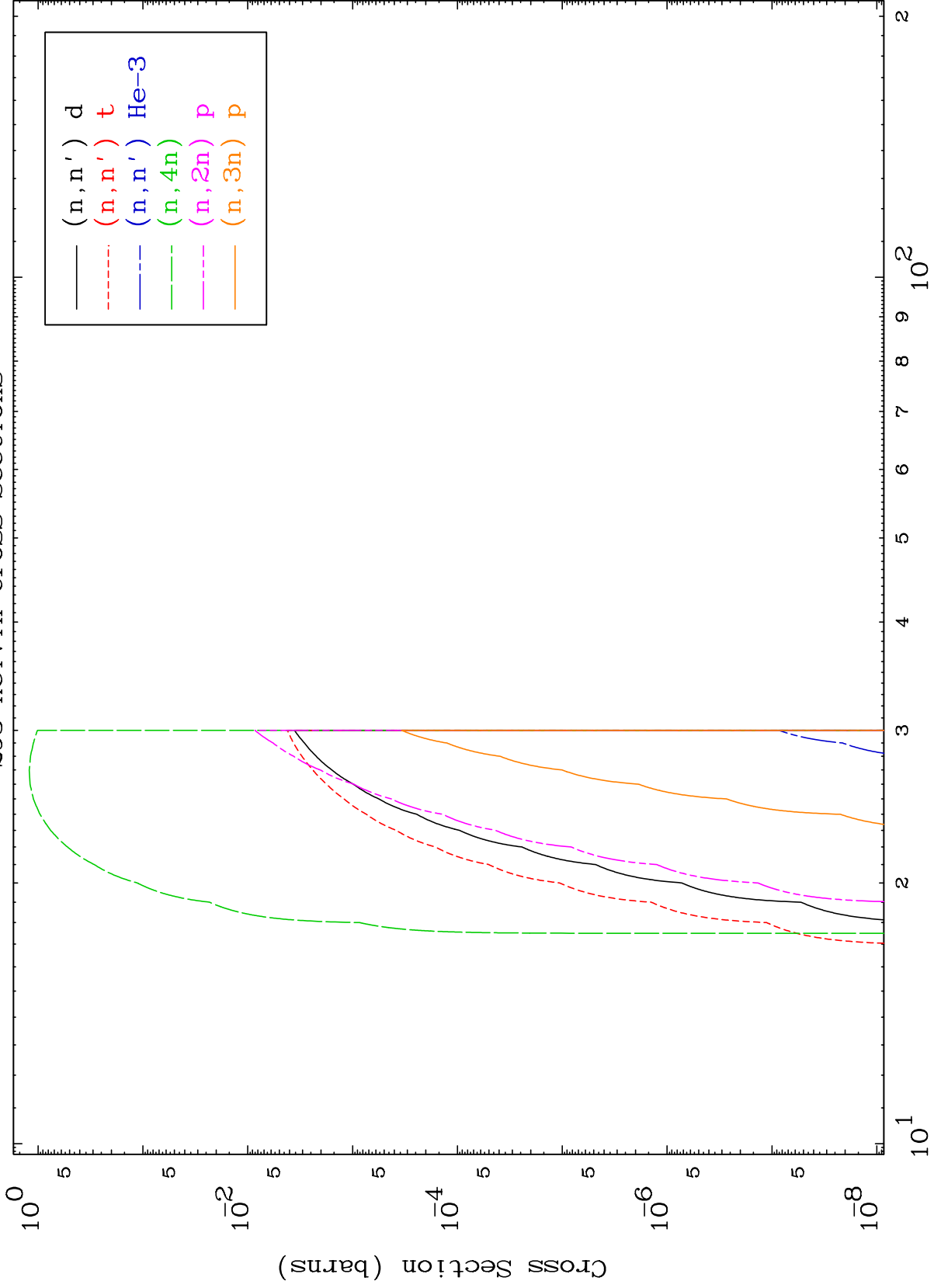
90-Th-234



MAT 9046

Neutron Production
293 Kelvin Cross Sections

90-Th-234



90-Th-234

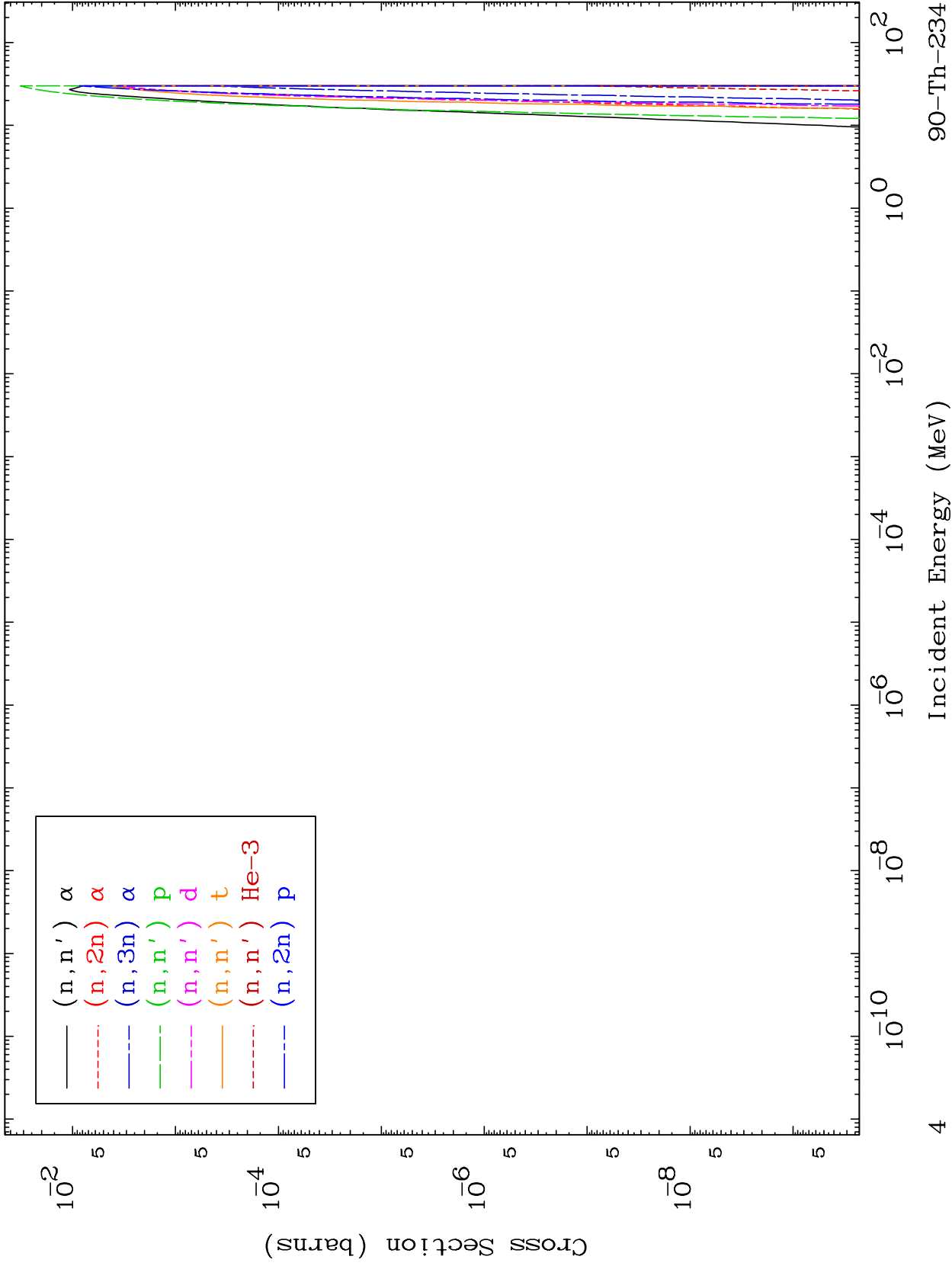
Incident Energy (MeV)

3

MAT 9046

Charged Particle
293 Kelvin Cross Sections

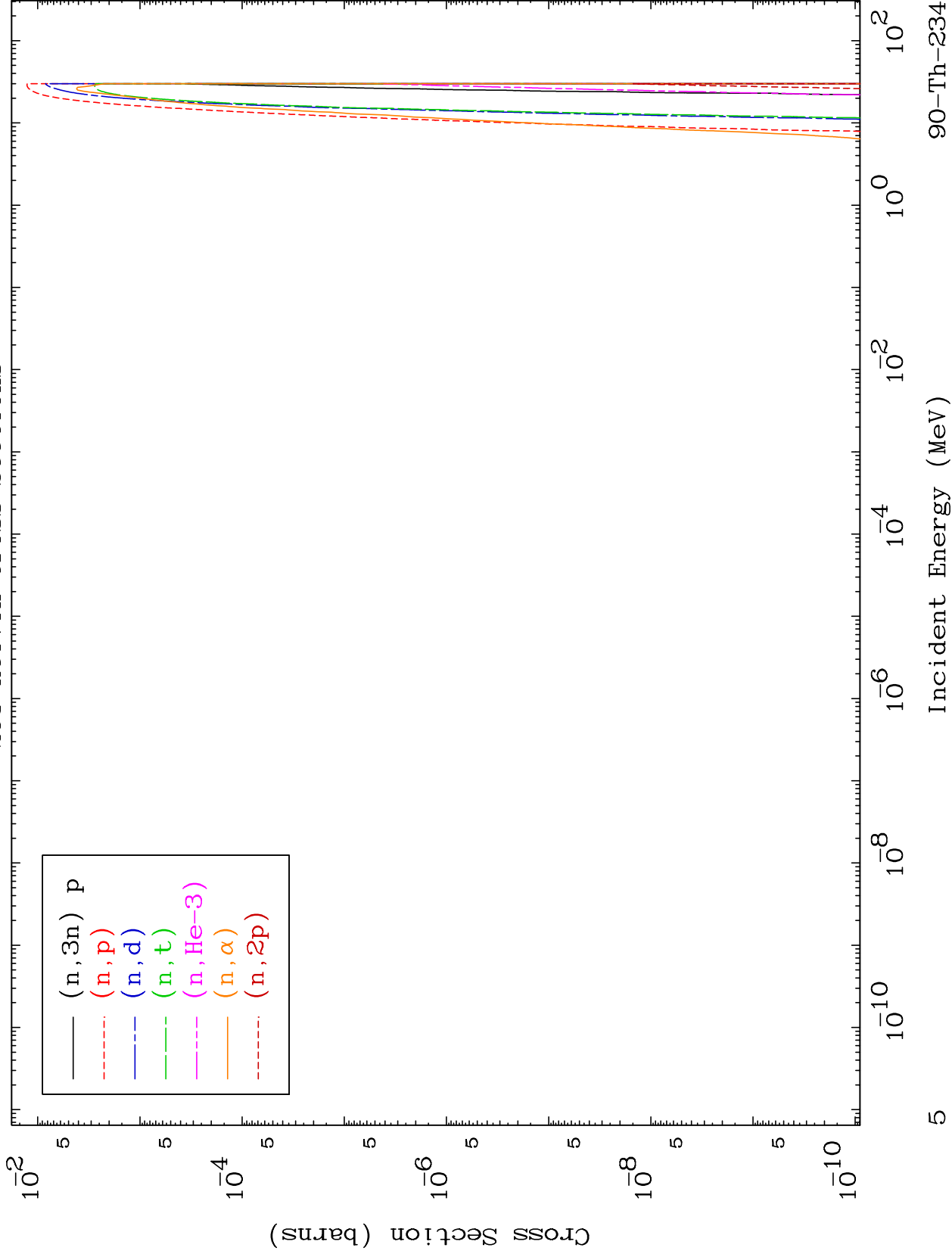
90-Th-234



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Charged Particle
293 Kelvin Cross Sections

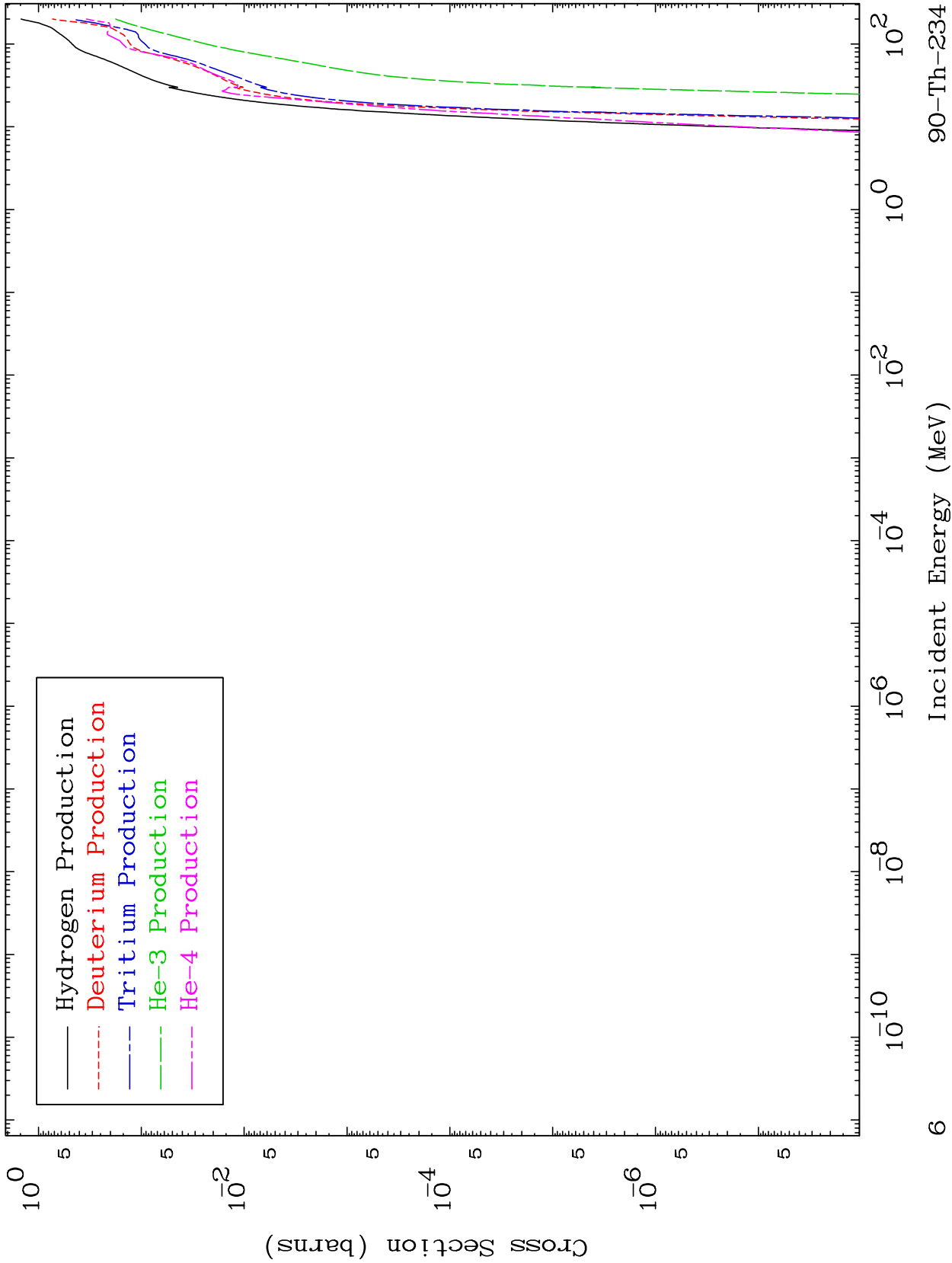
90-Th-234



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Particle Production
293 Kelvin Cross Sections

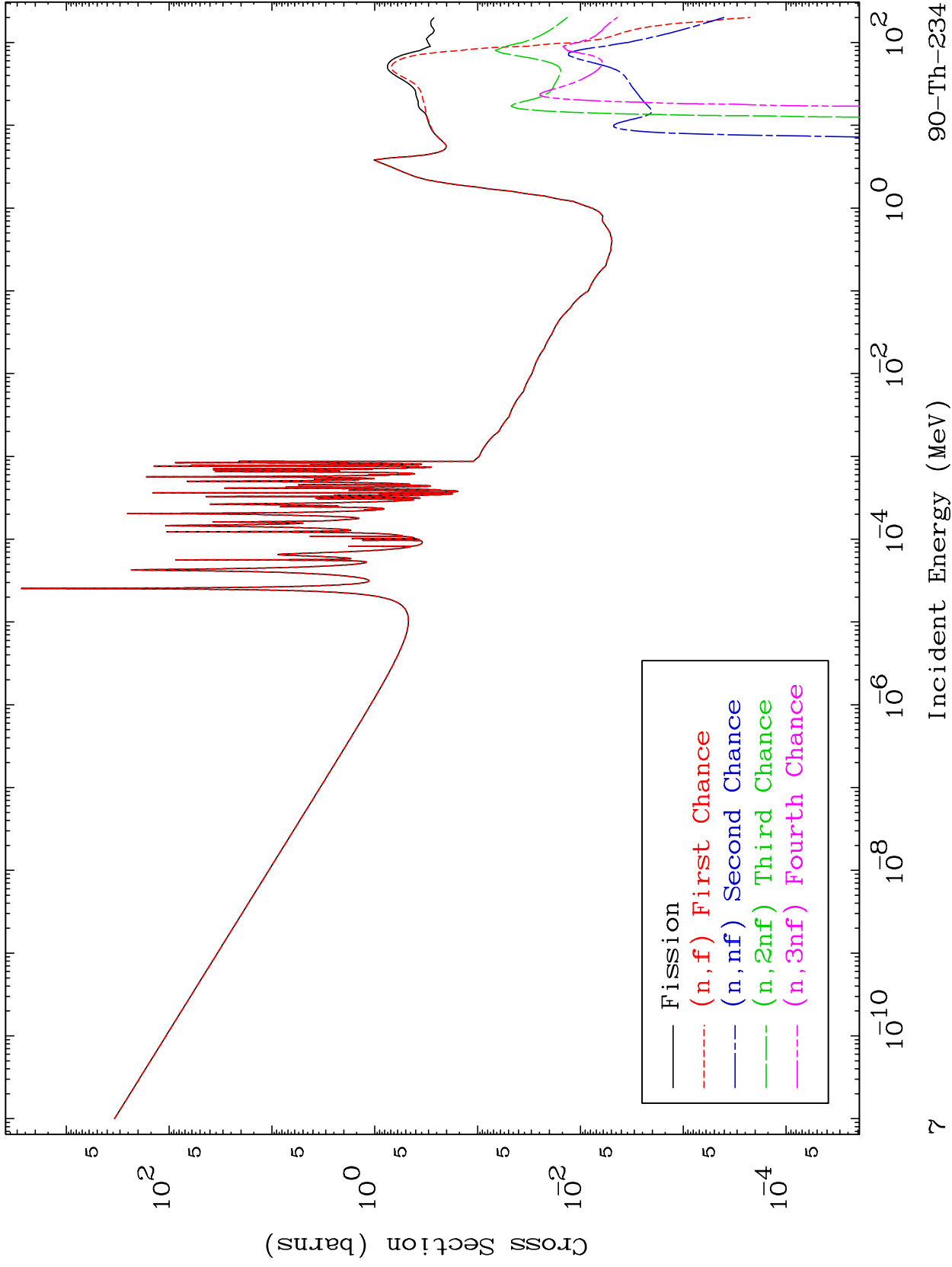
90-Th-234



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Fission
293 Kelvin Cross Sections

90-Th-234

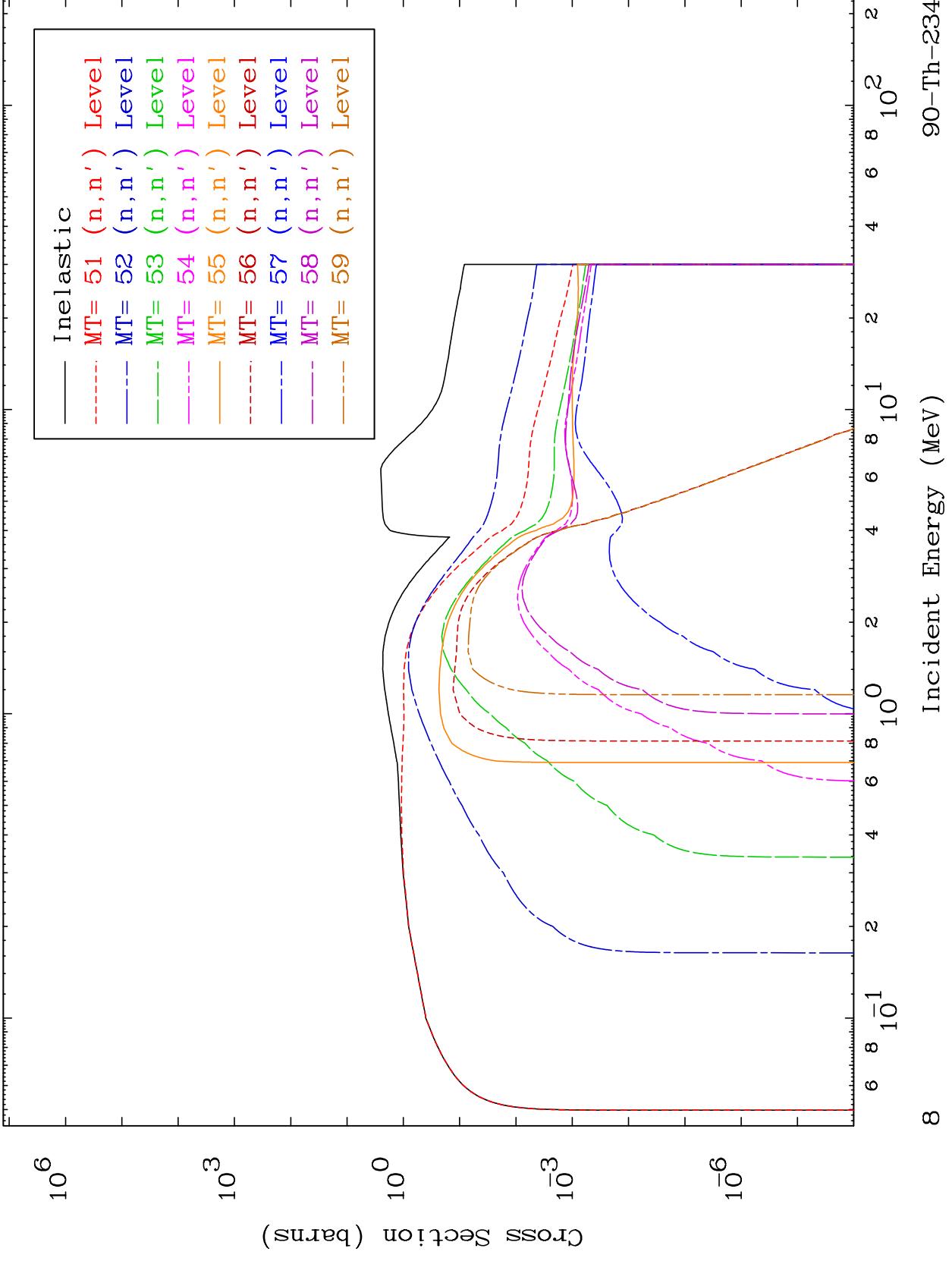


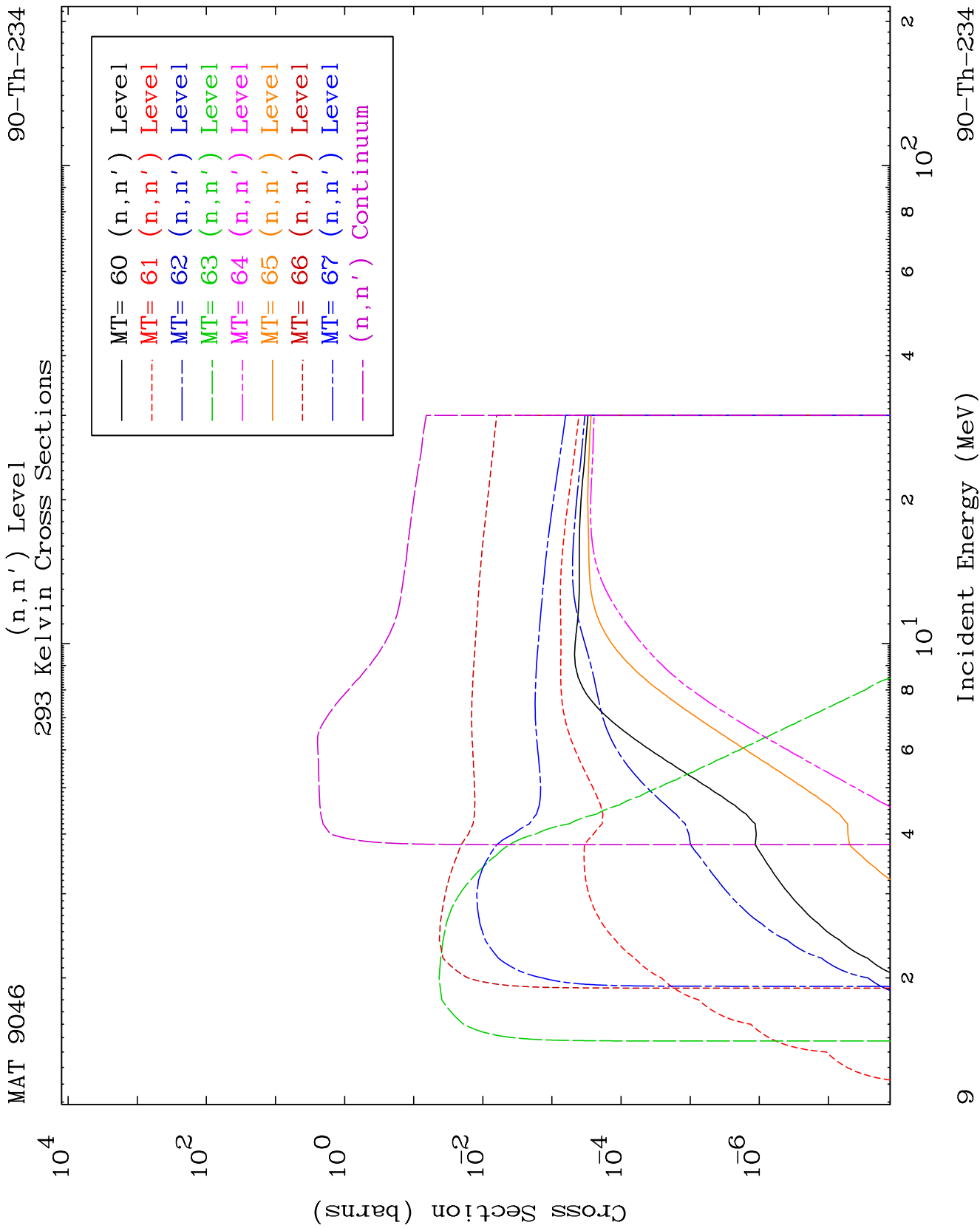
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(n,n') Level

90-Th-234

293 Kelvin Cross Sections

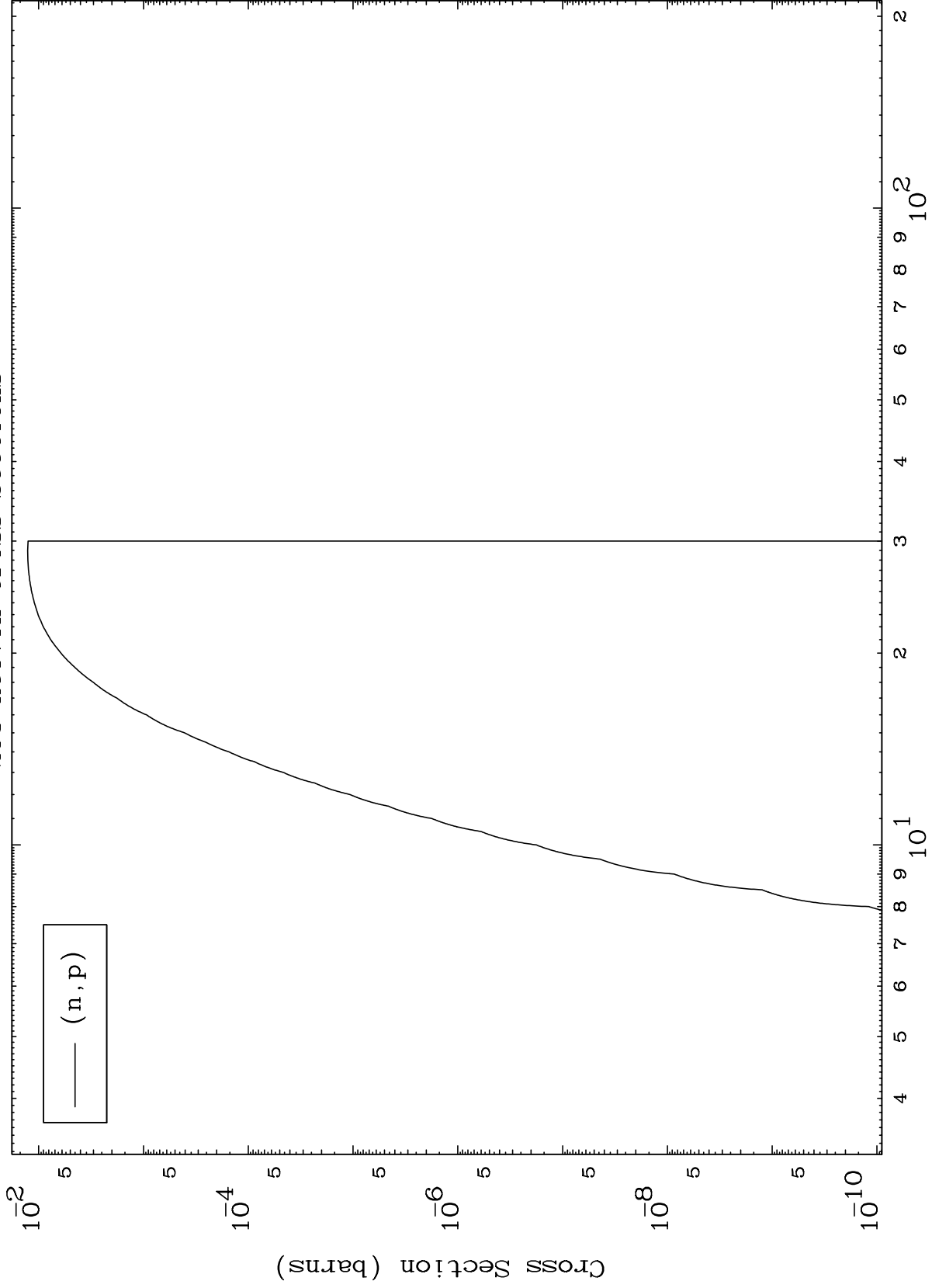




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(n,p) Levels
293 Kelvin Cross Sections

90-Th-234



10

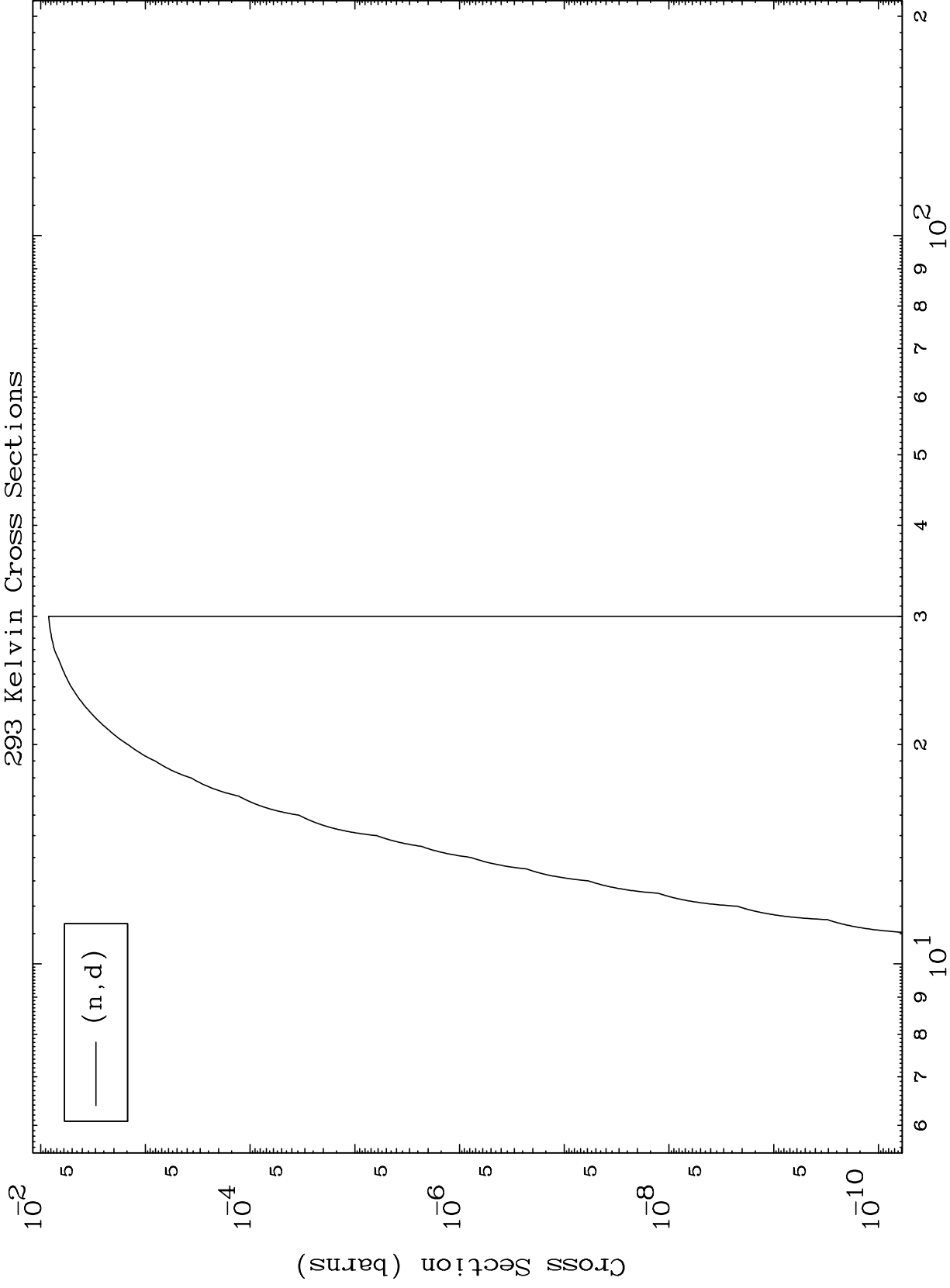
Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

90-Th-234



11

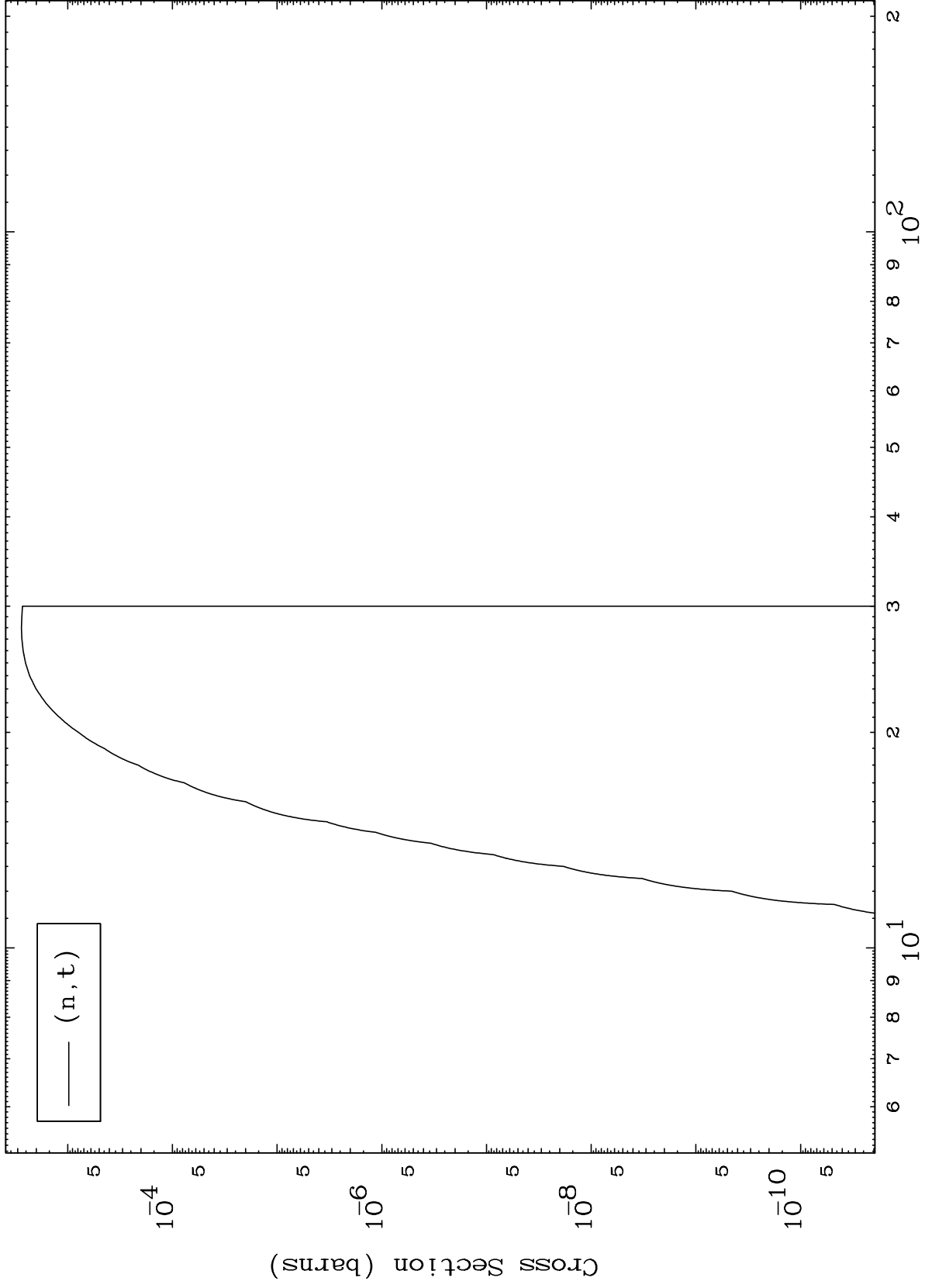
Incident Energy (MeV)

90-Th-234

MAT 9046

(n,t) Levels
293 Kelvin Cross Sections

90-Th-234



12

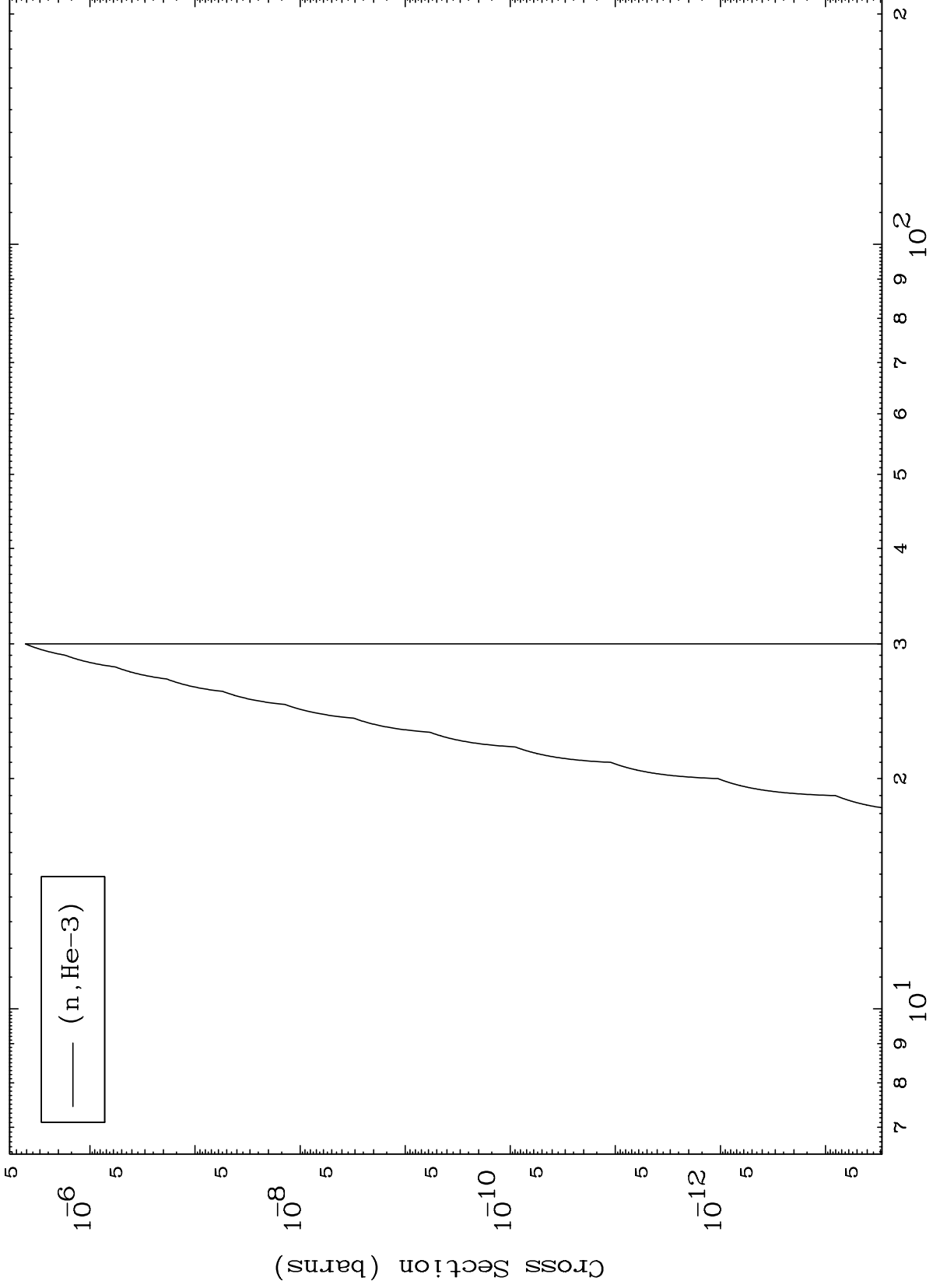
Incident Energy (MeV)

90-Th-234

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(n,He3) Levels
293 Kelvin Cross Sections

90-Th-234



13

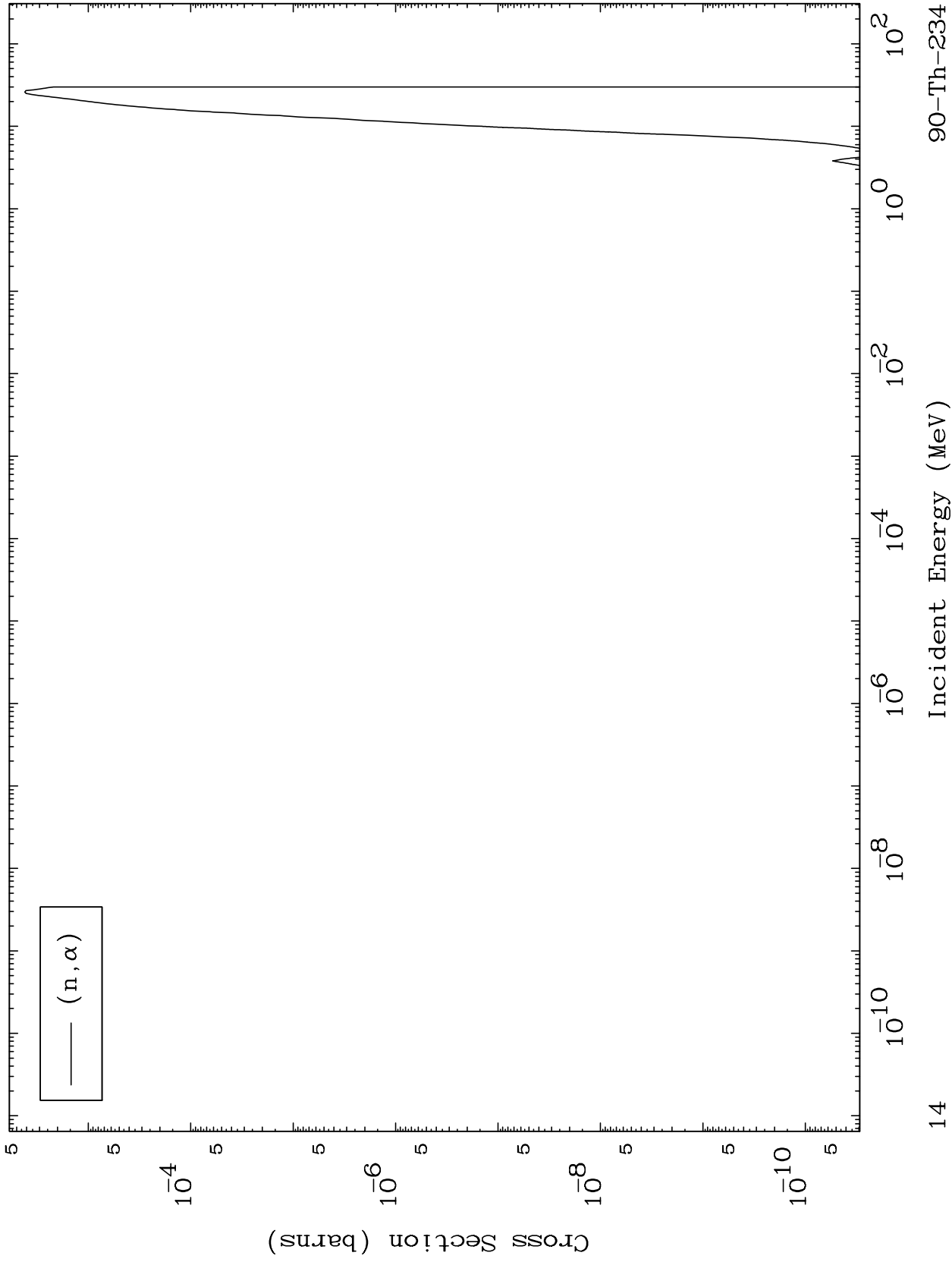
Incident Energy (MeV)

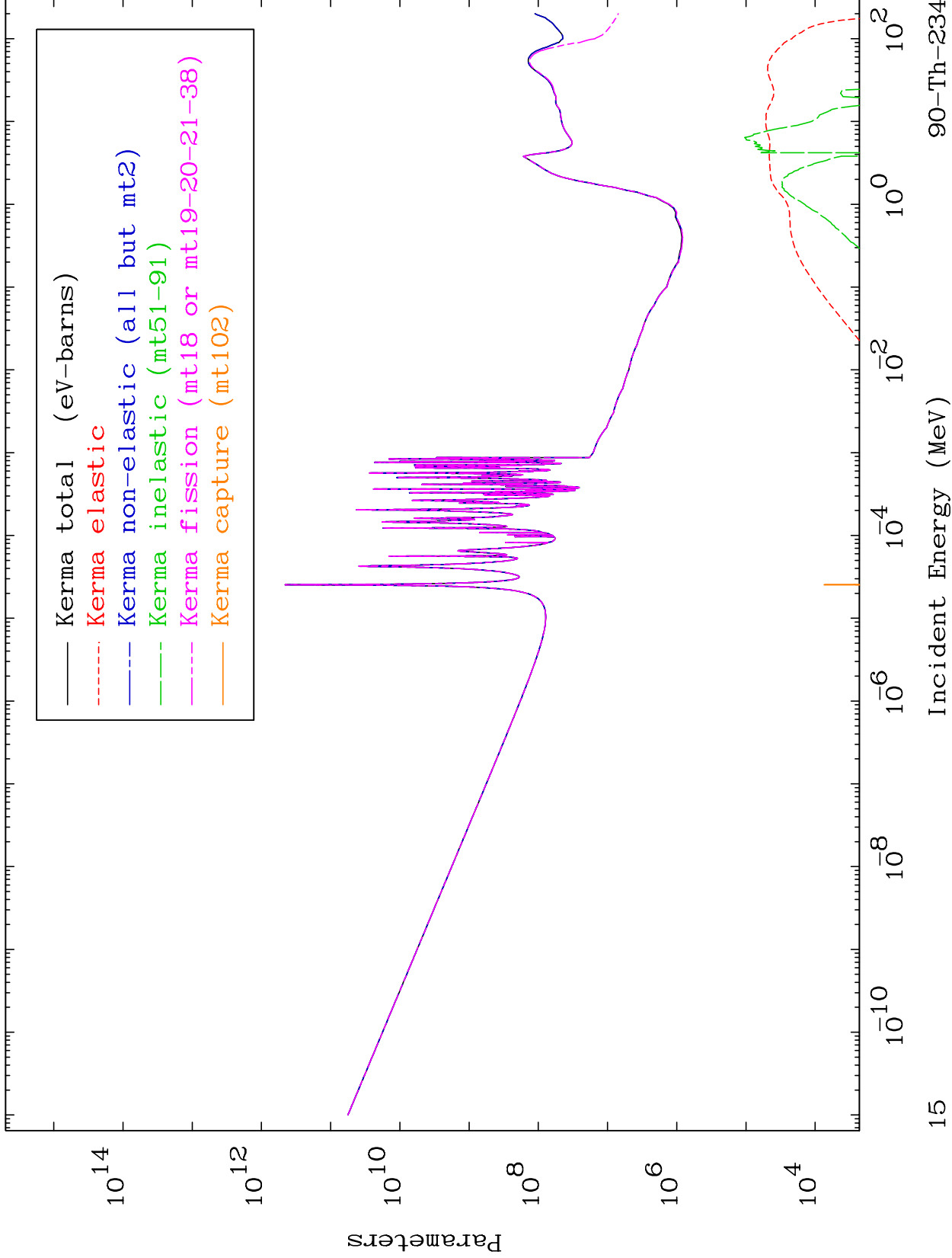
90-Th-234

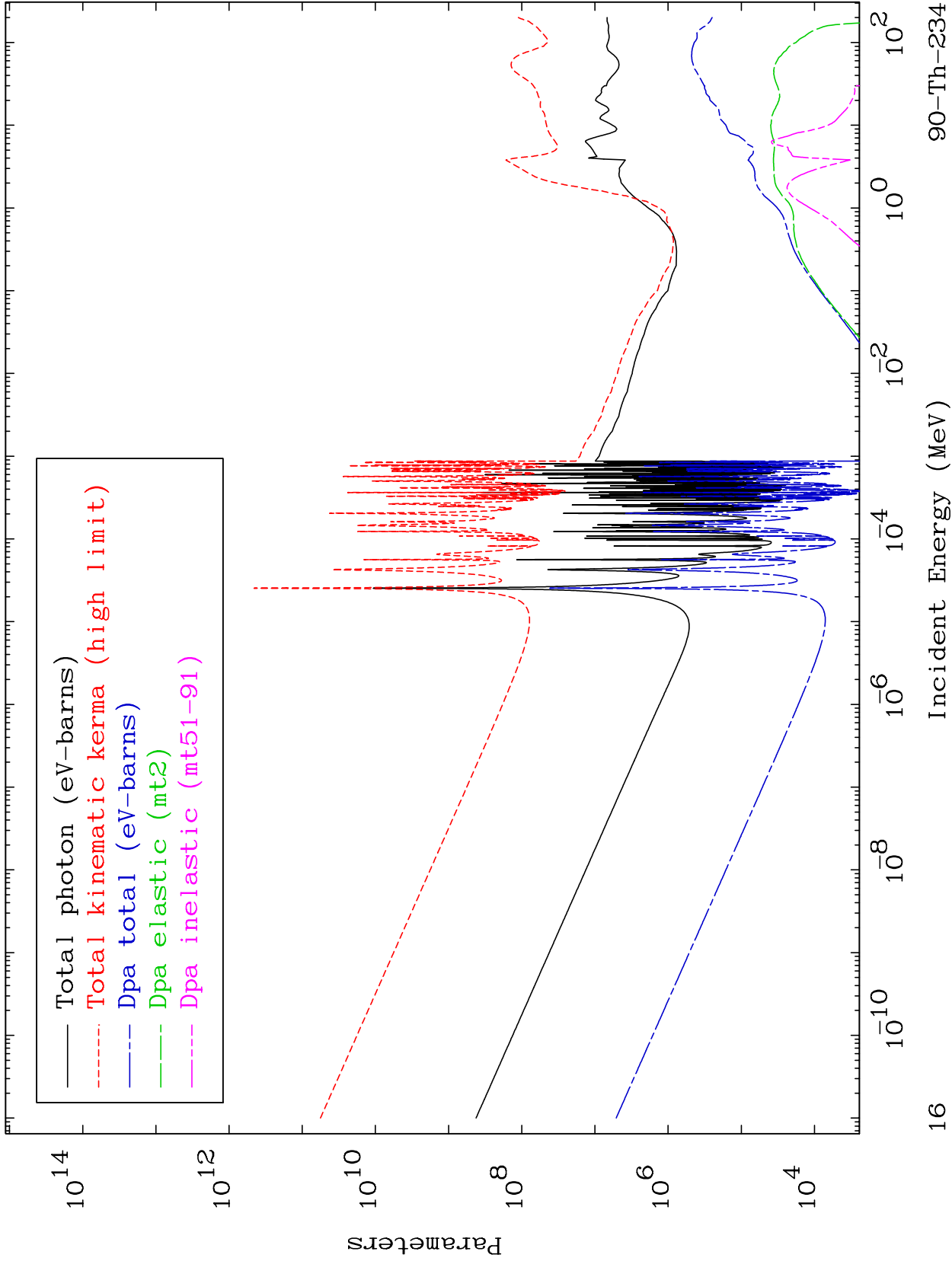
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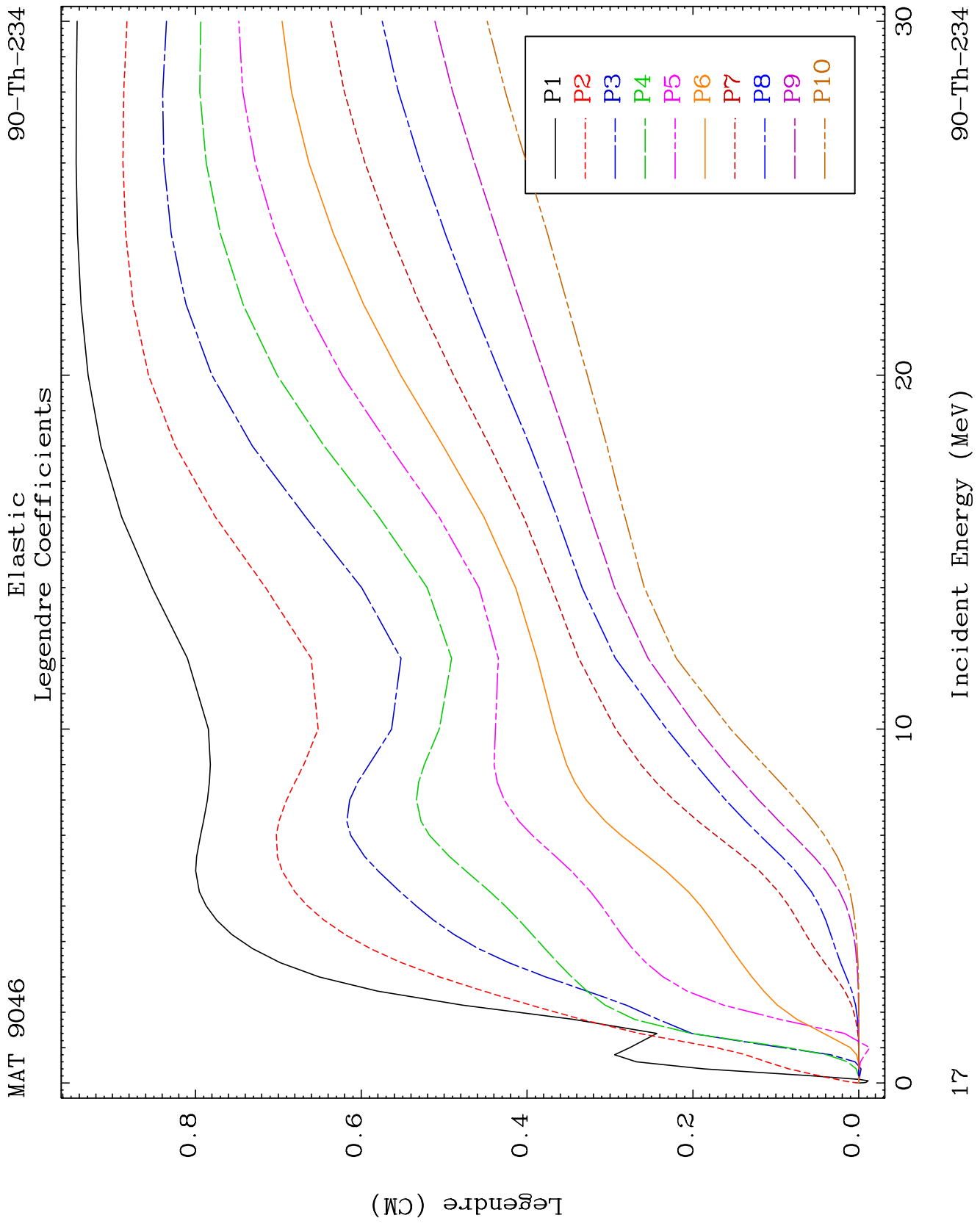
(n, α) Levels
293 Kelvin Cross Sections

90-Th-234





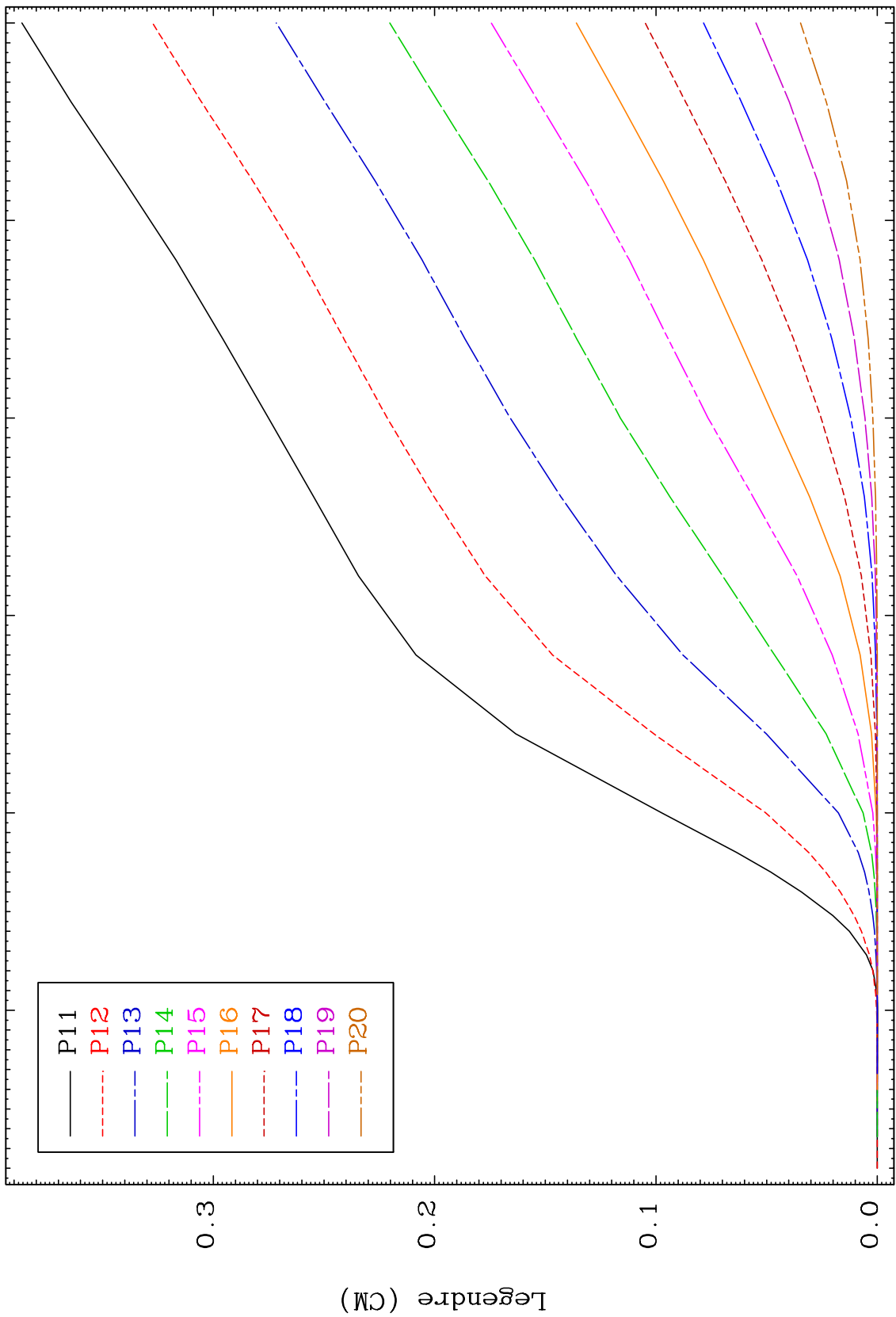




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Elastic Legendre Coefficients

90-Th-234



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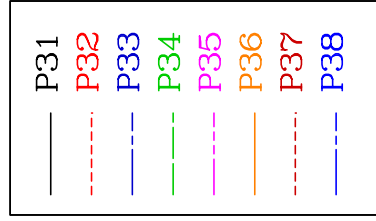
Incident Energy (MeV)

90-Th-234

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Elastic Legendre Coefficients

90-Th-234



$\times 10^{-6}$
1.5

Legendre (CM)

1.0

0.5

0.0

15

20

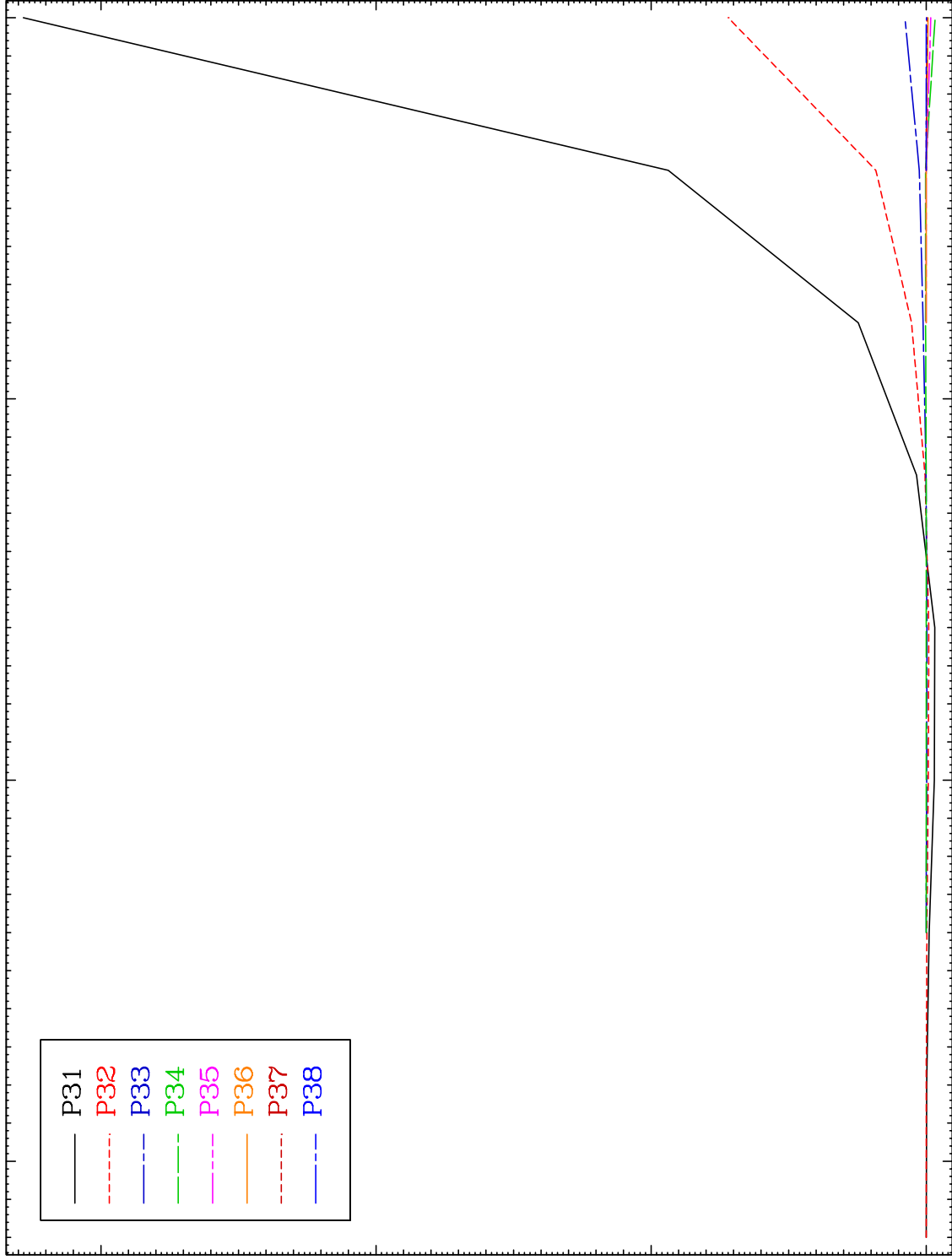
25

30

20

Incident Energy (MeV)

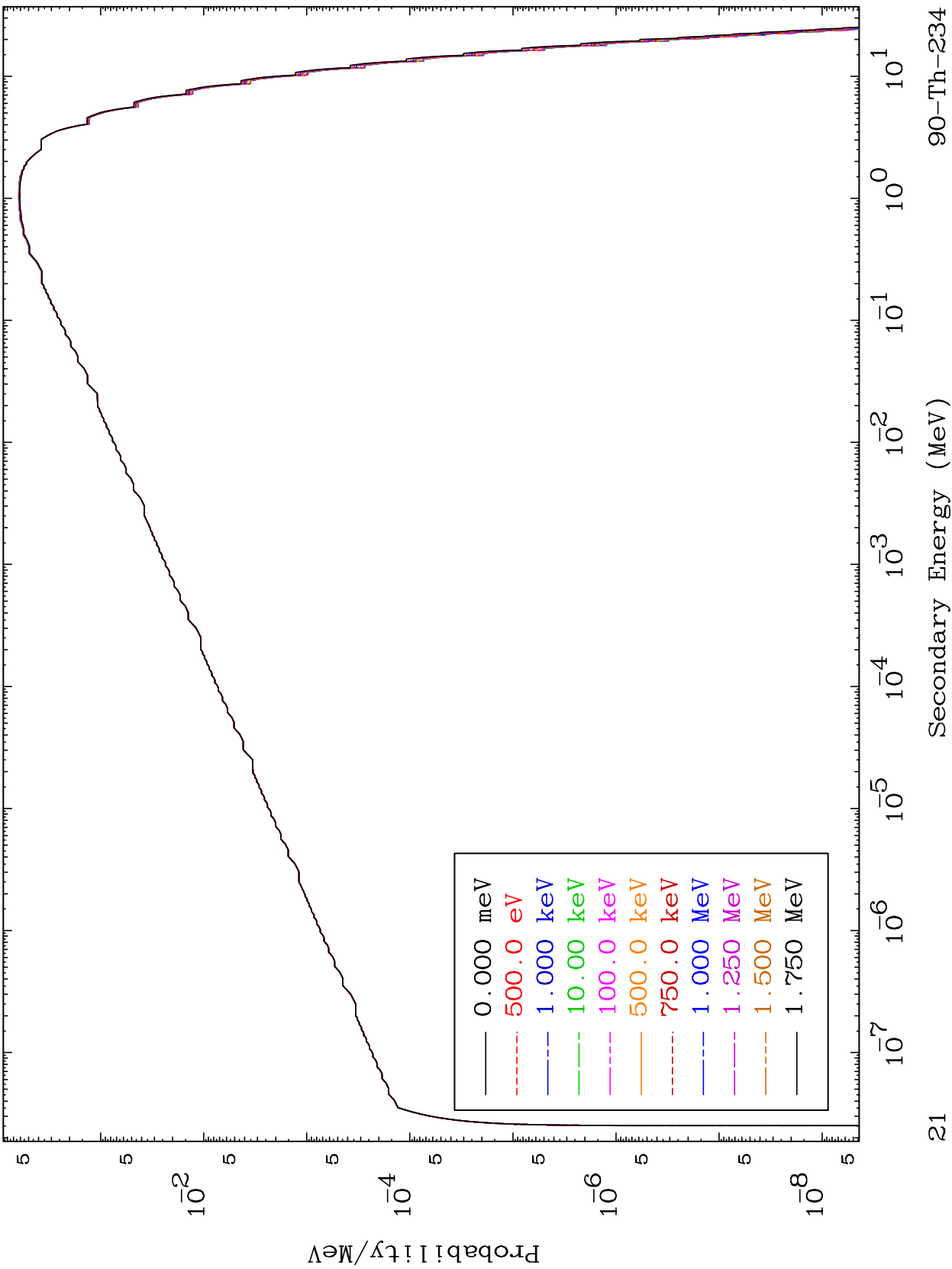
90-Th-234



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Fission Energy Distribution

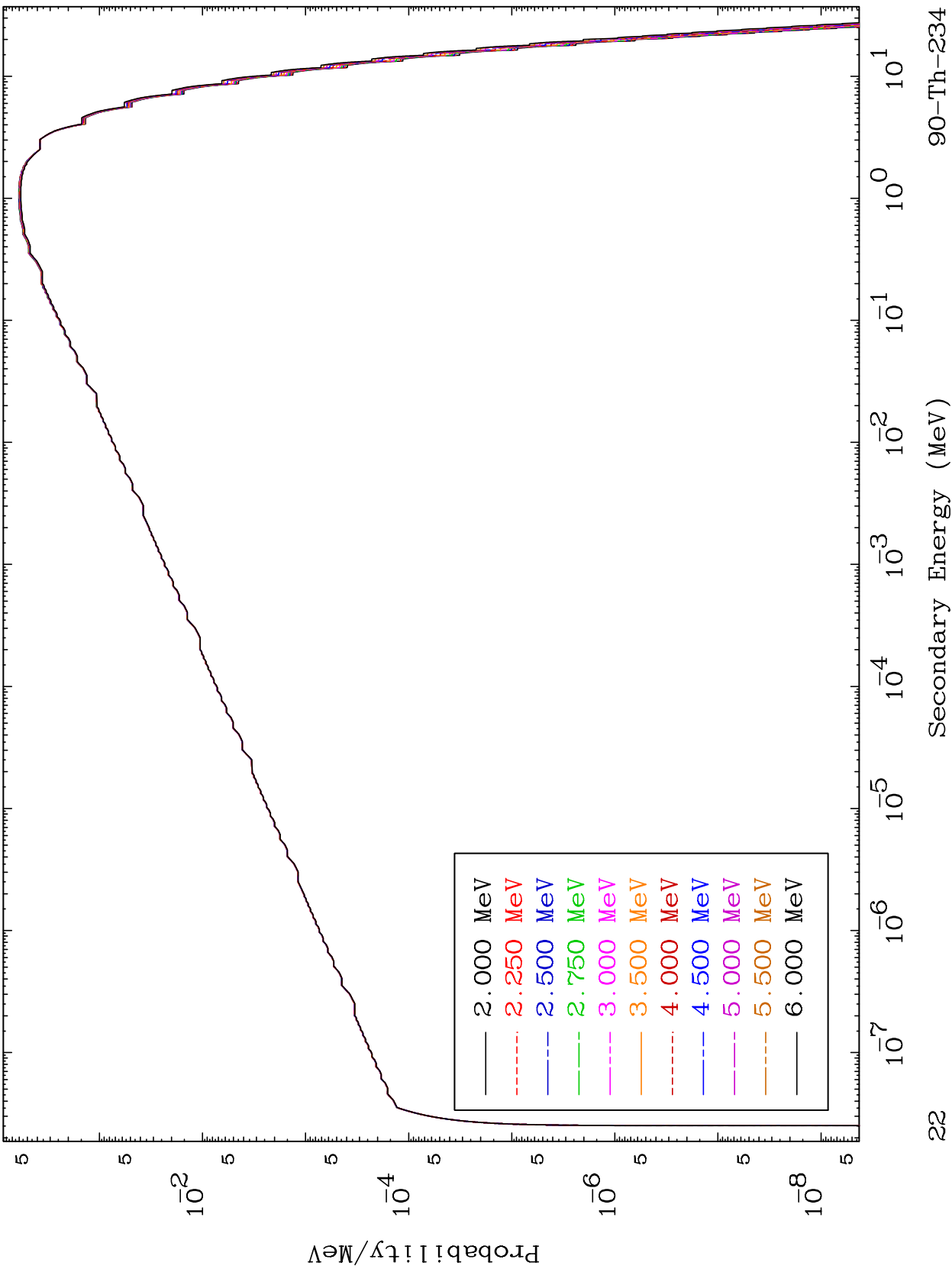
90-Th-234



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Fission Energy Distribution

90-Th-234



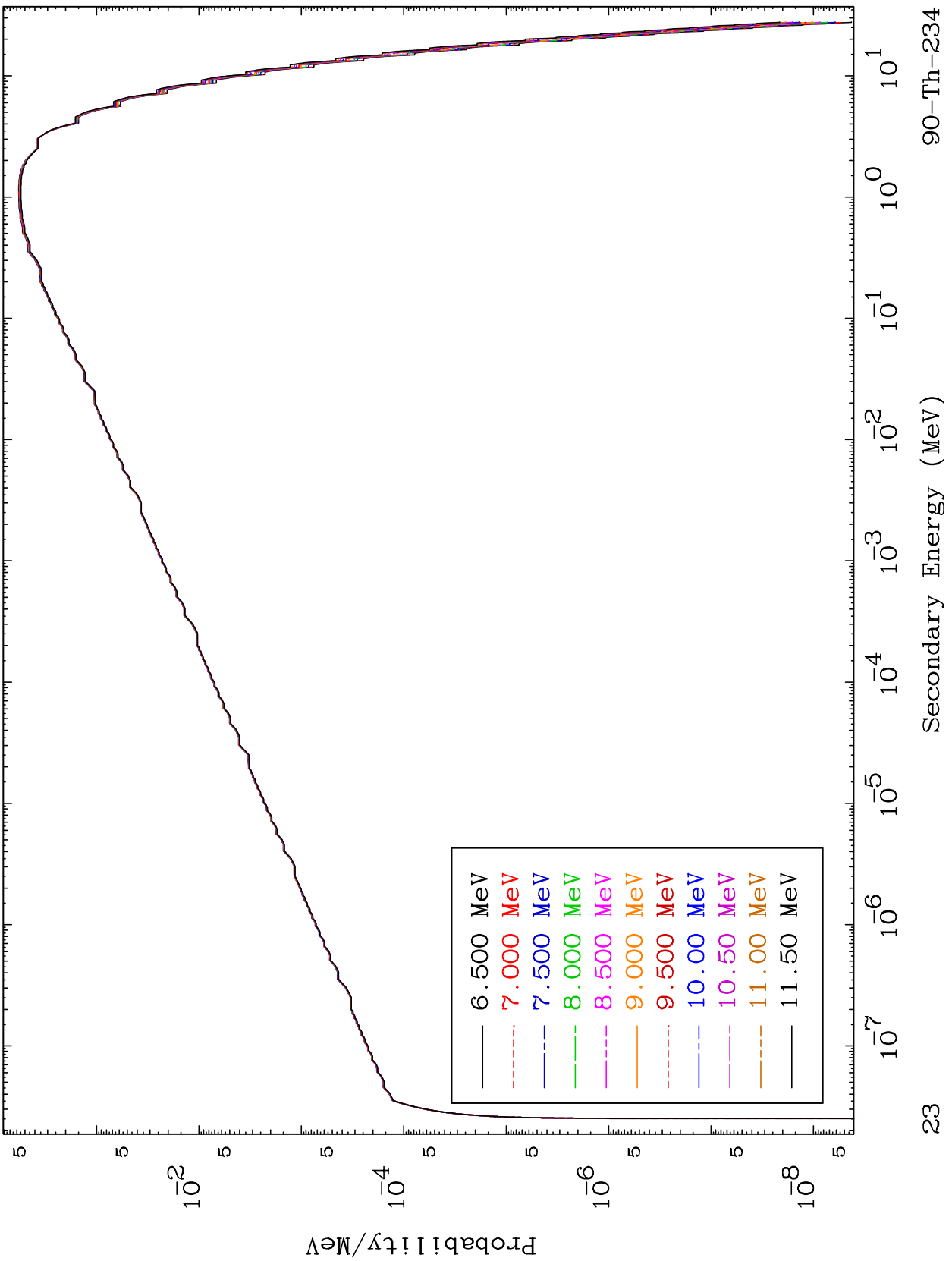
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Secondary Energy (MeV)

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Fission Energy Distribution

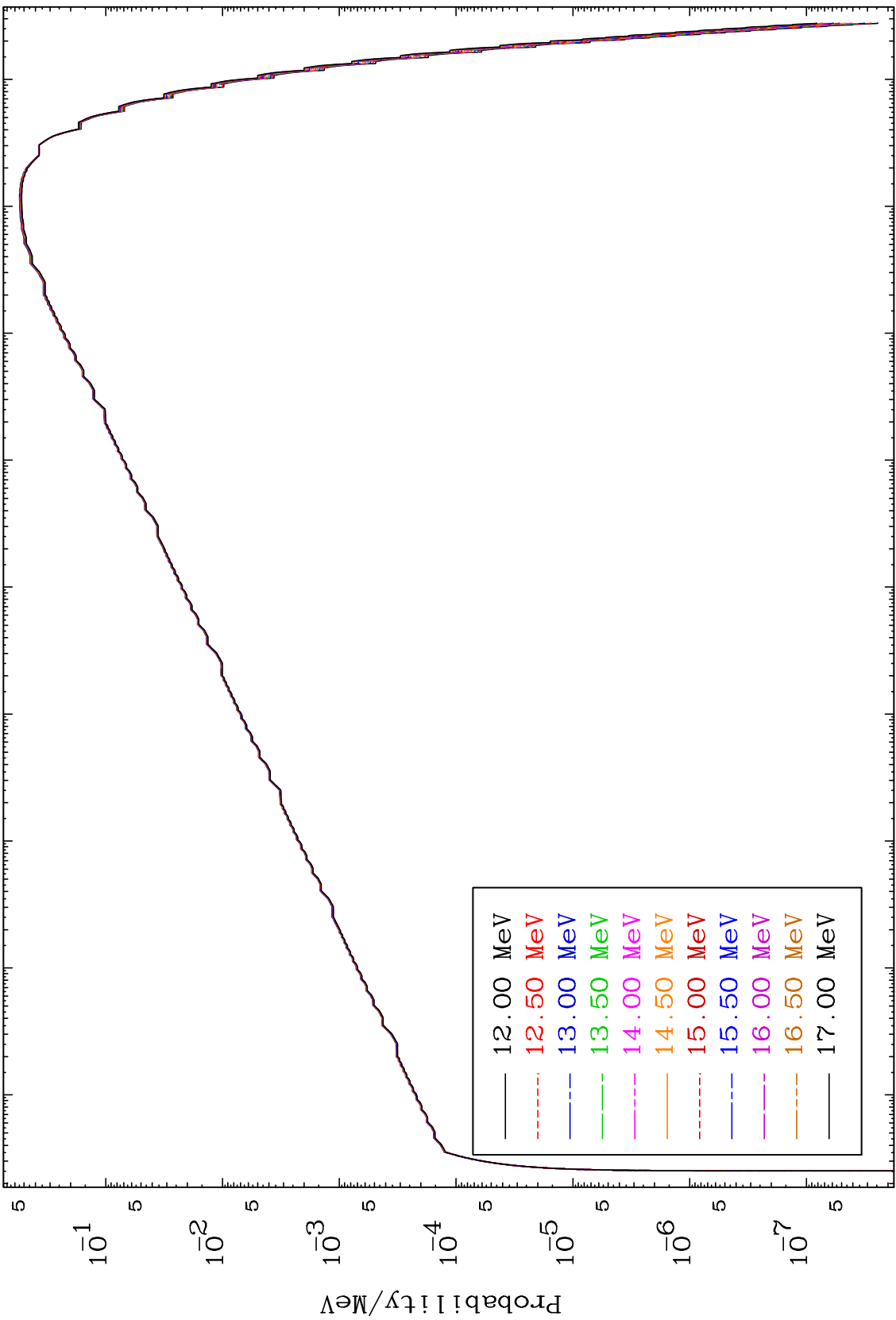
90-Th-234



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Fission Energy Distribution

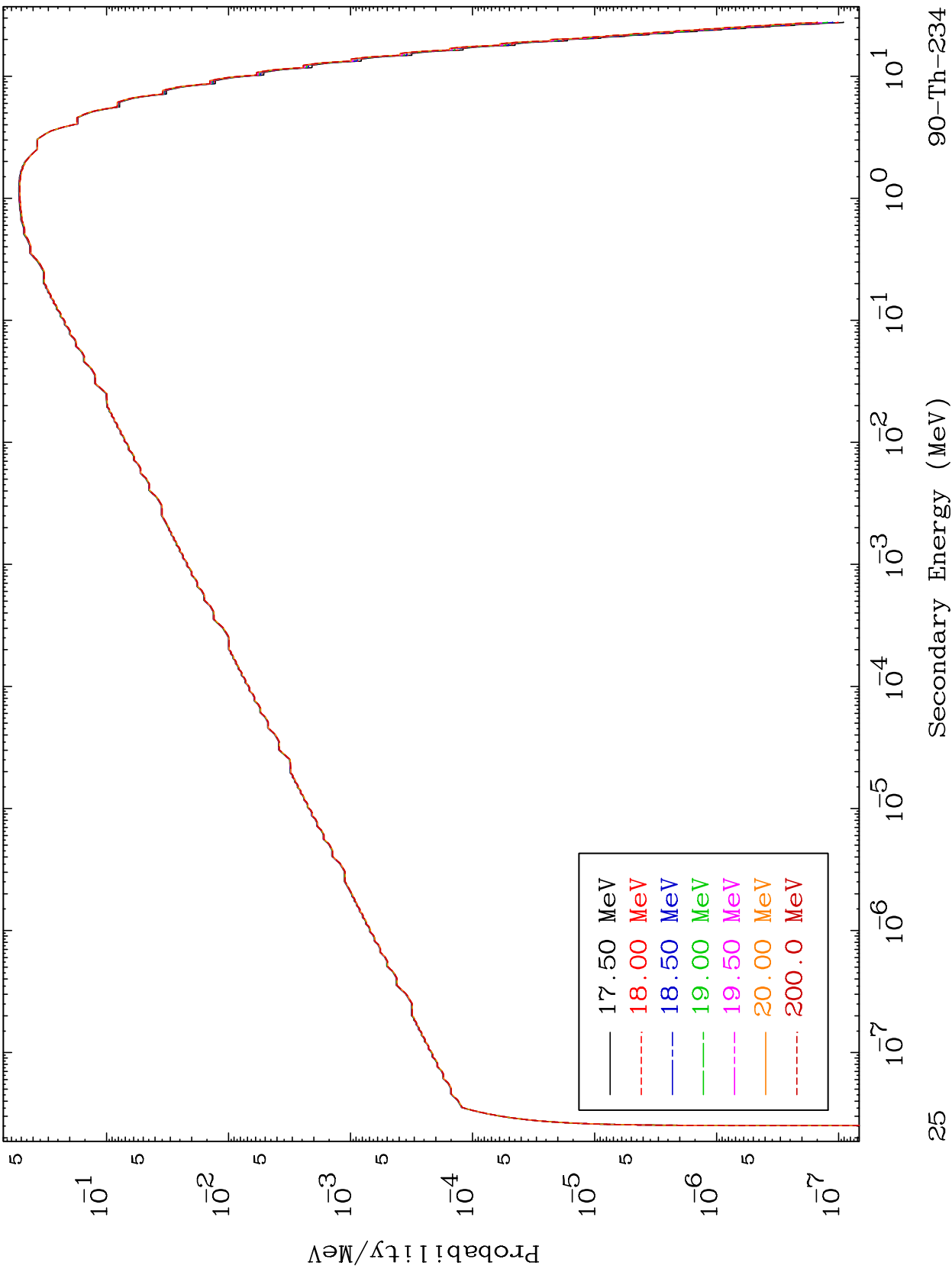
⁹⁰Th-234



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Fission Energy Distribution

90-Th-234



90-Th-234