

Program EVALPLOT  
(Version 2017-1)

by

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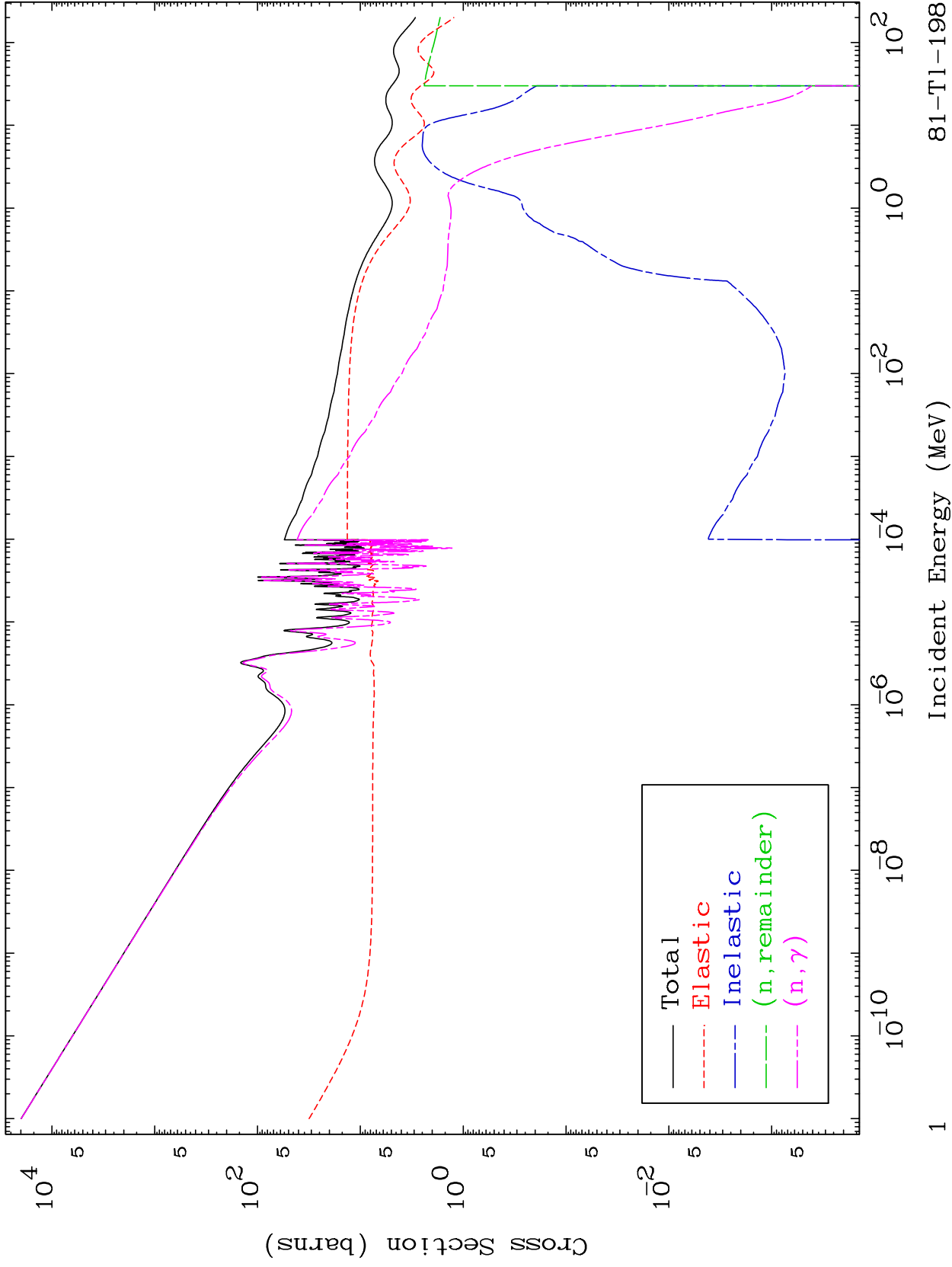
Press Mouse Button to Start

MAT 8111

Major

293 Kelvin Cross Sections

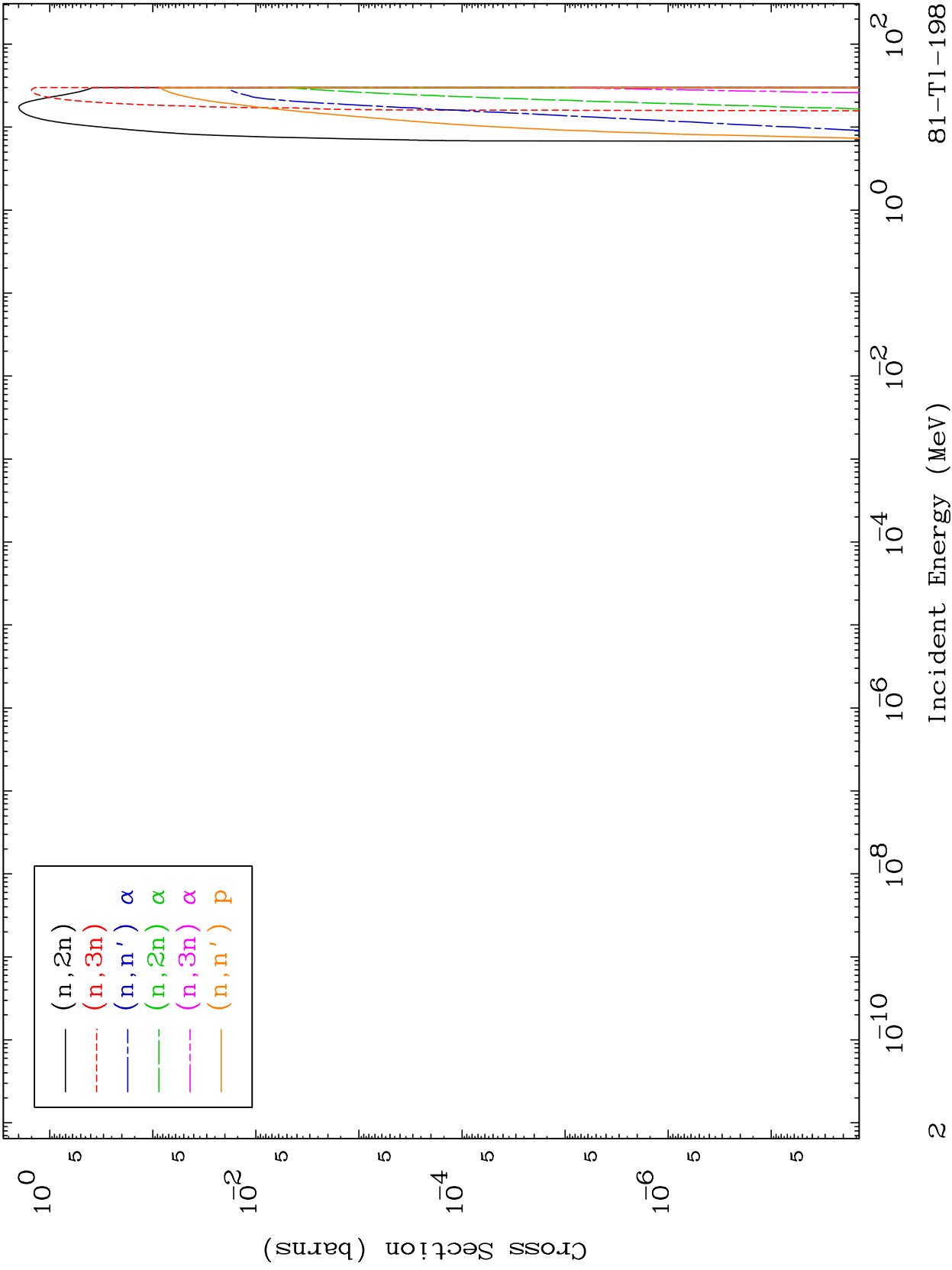
81-Tl-198

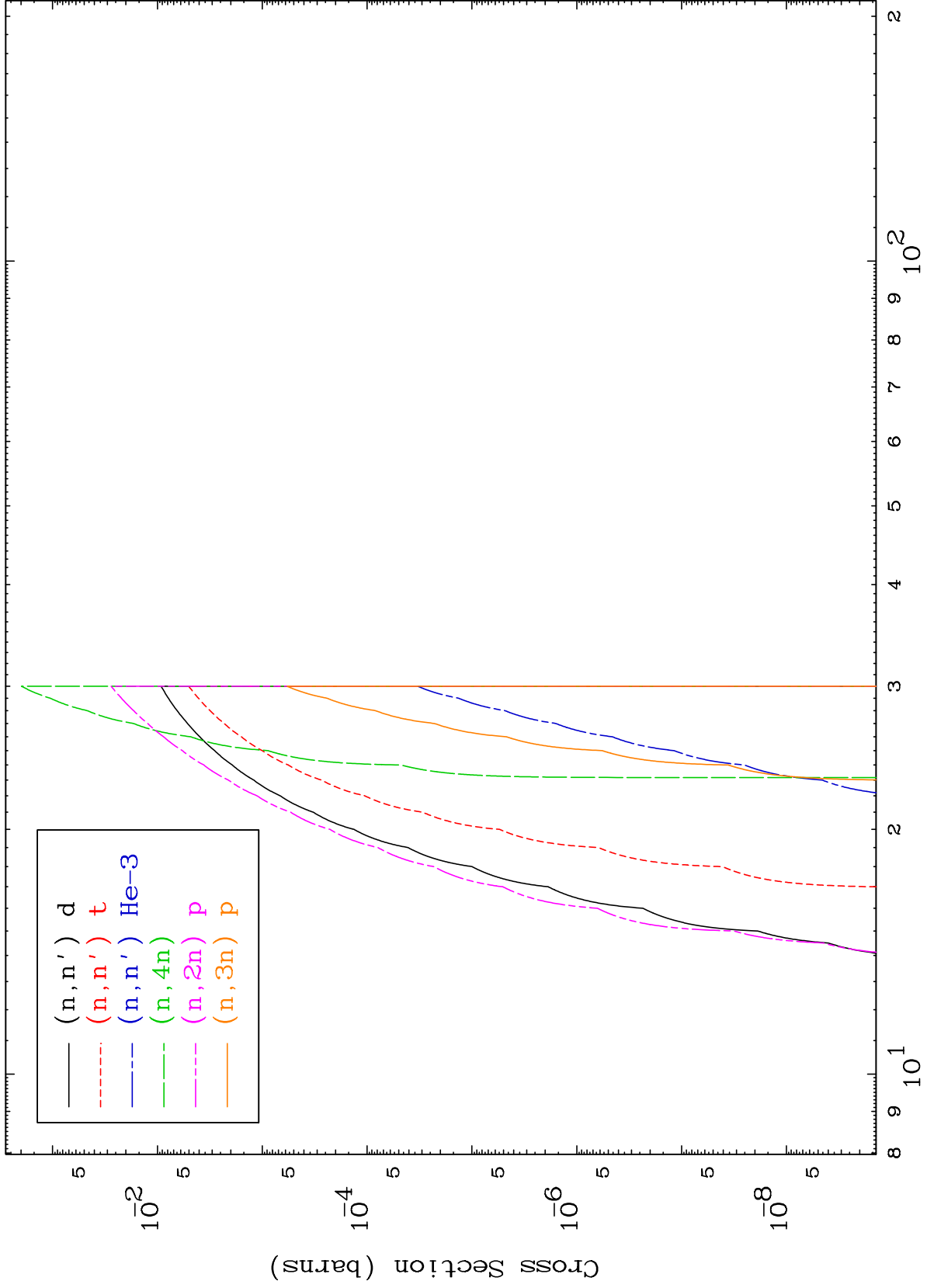


MAT 81111

Neutron Production  
293 Kelvin Cross Sections

81-Tl-198

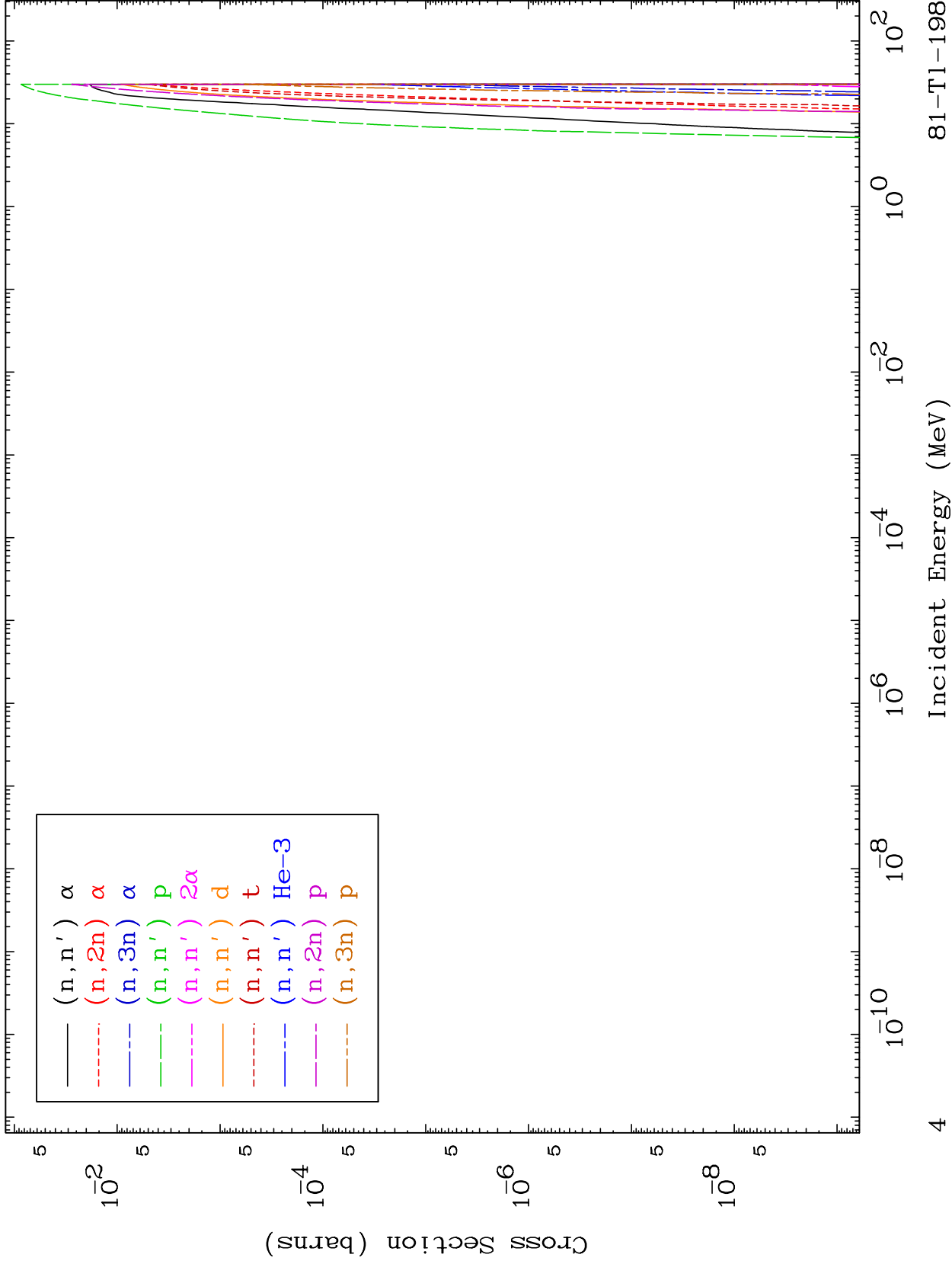




MAT 8111

Charged Particle  
293 Kelvin Cross Sections

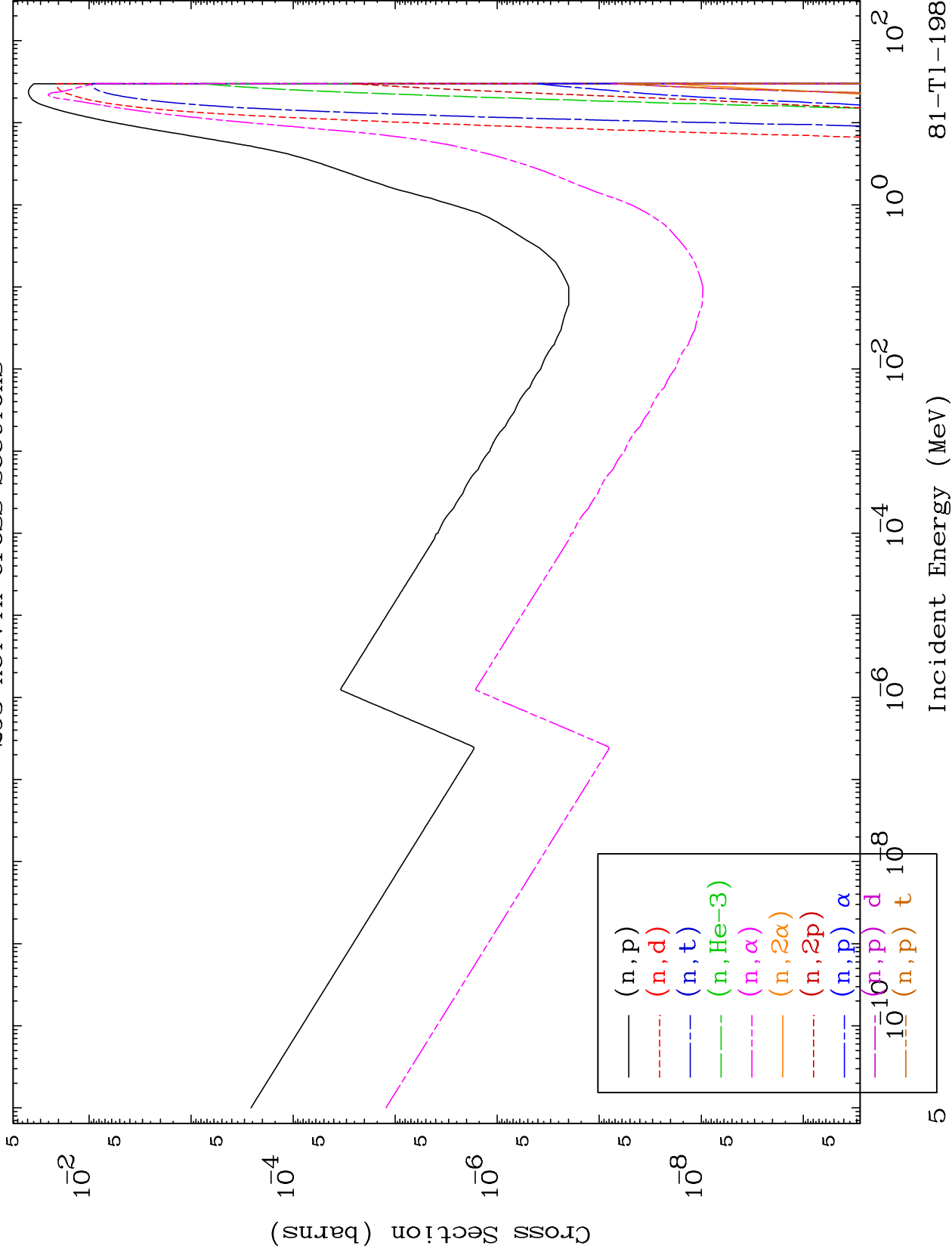
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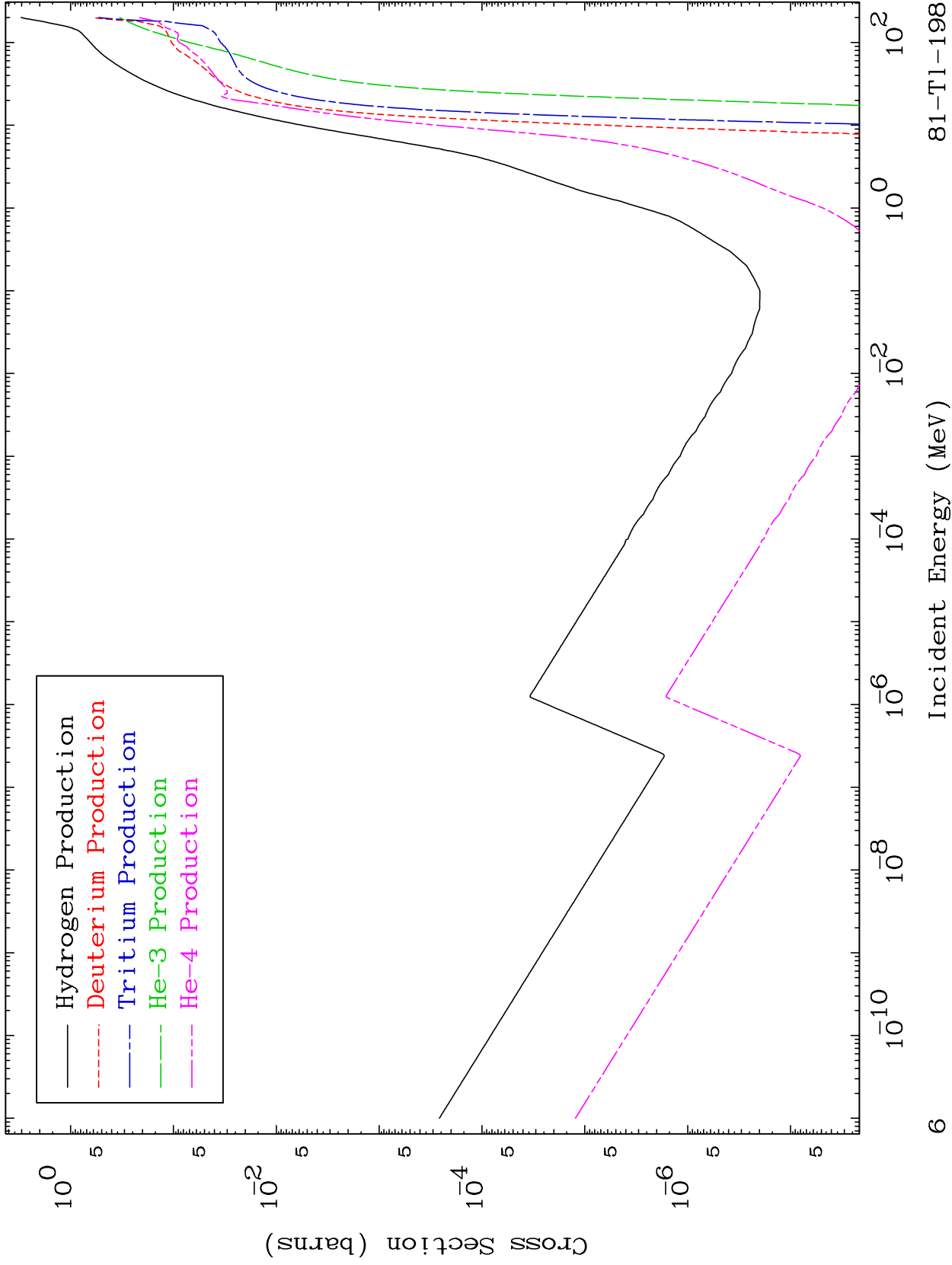


MAT 81111

Charged Particle  
293 Kelvin Cross Sections

81-Tl-198

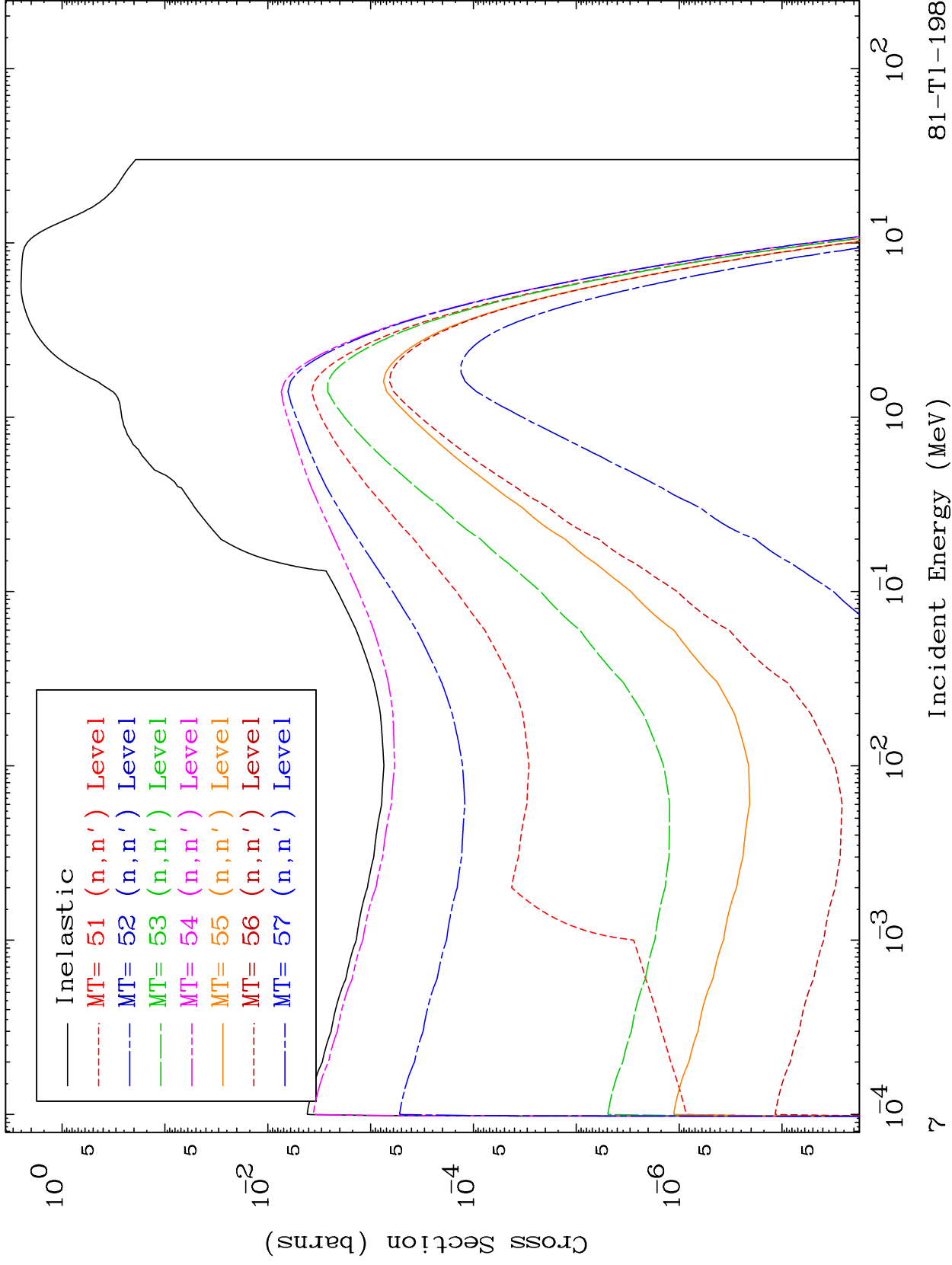




MAT 8111

(n,n') Level  
293 Kelvin Cross Sections

81-T1-198



Incident Energy (MeV)

81-T1-198

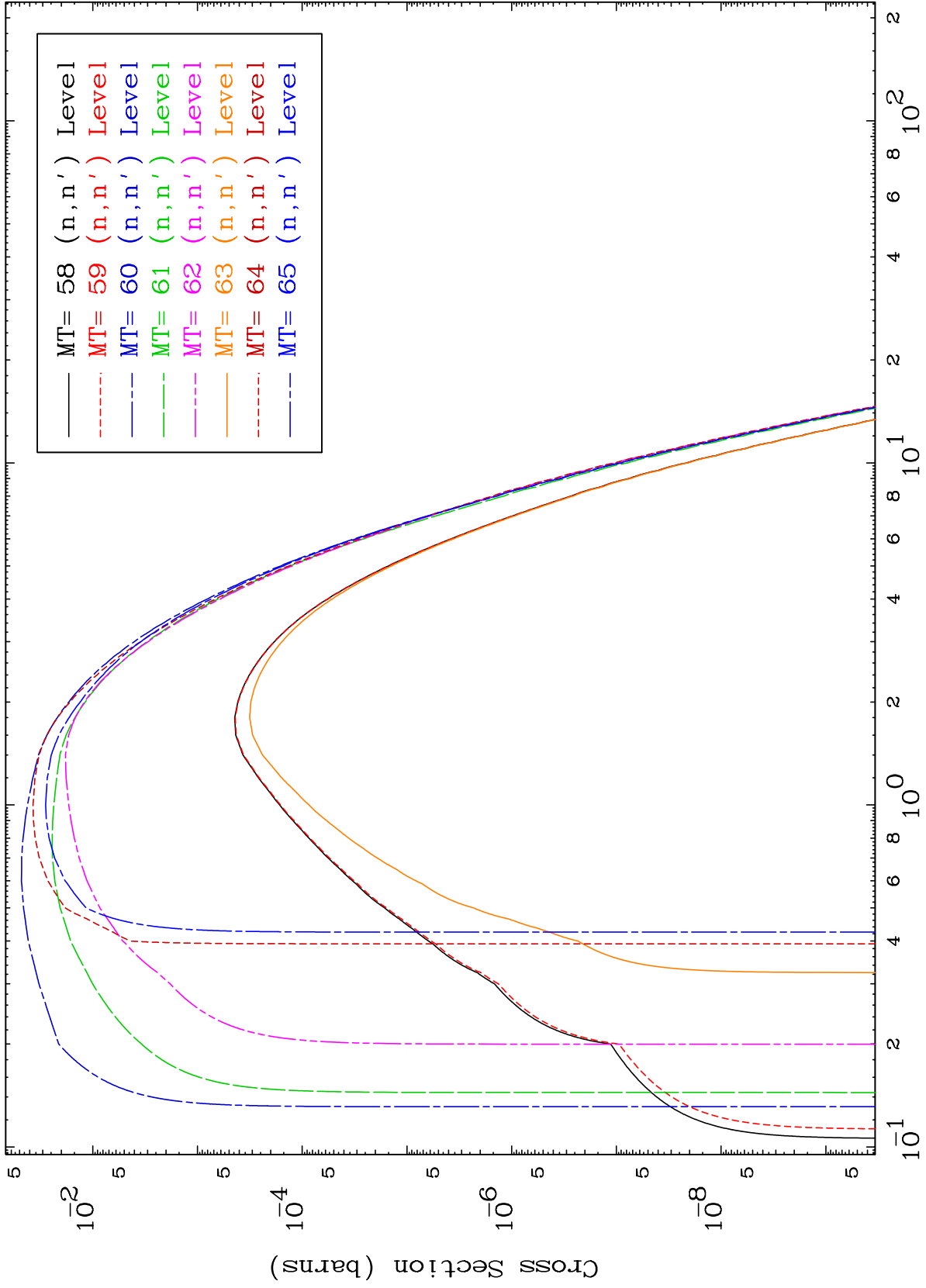


MAT 8111

(n,n') Level

81-Tl-198

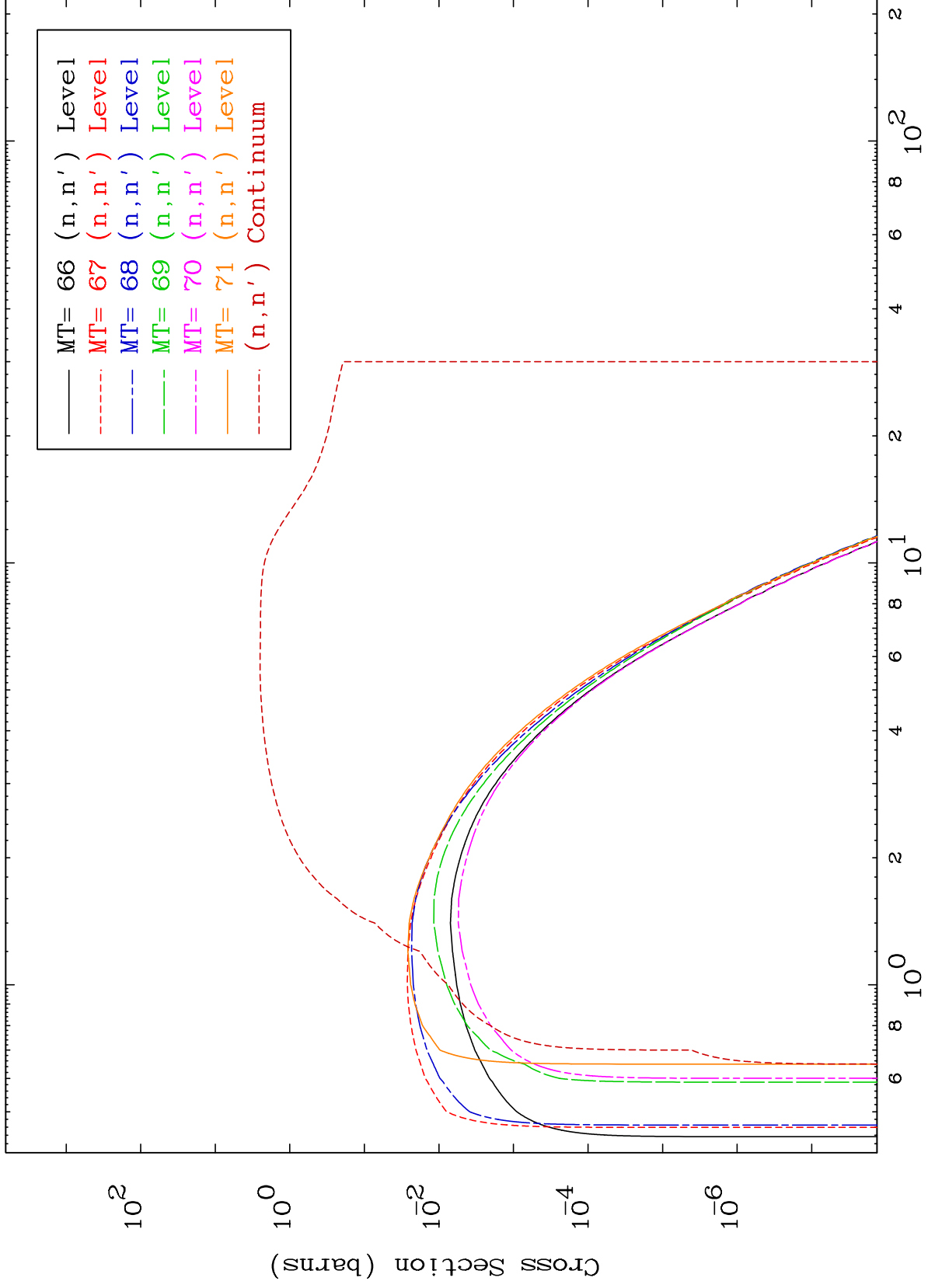
293 Kelvin Cross Sections



Incident Energy (MeV)

81-Tl-198

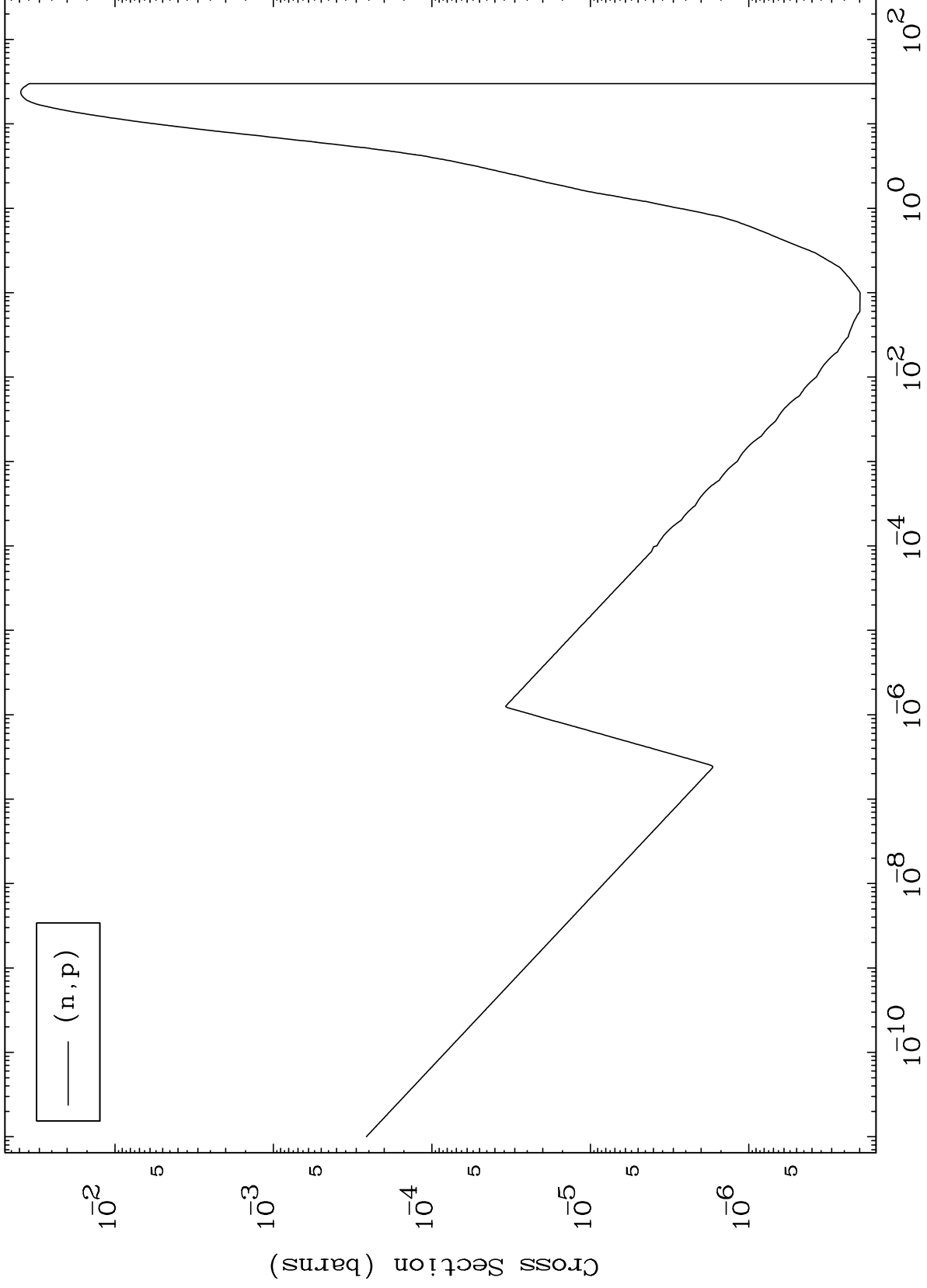
293 Kelvin Cross Sections



MAT 81111

(n,p) Levels  
293 Kelvin Cross Sections

81-Tl-198



10

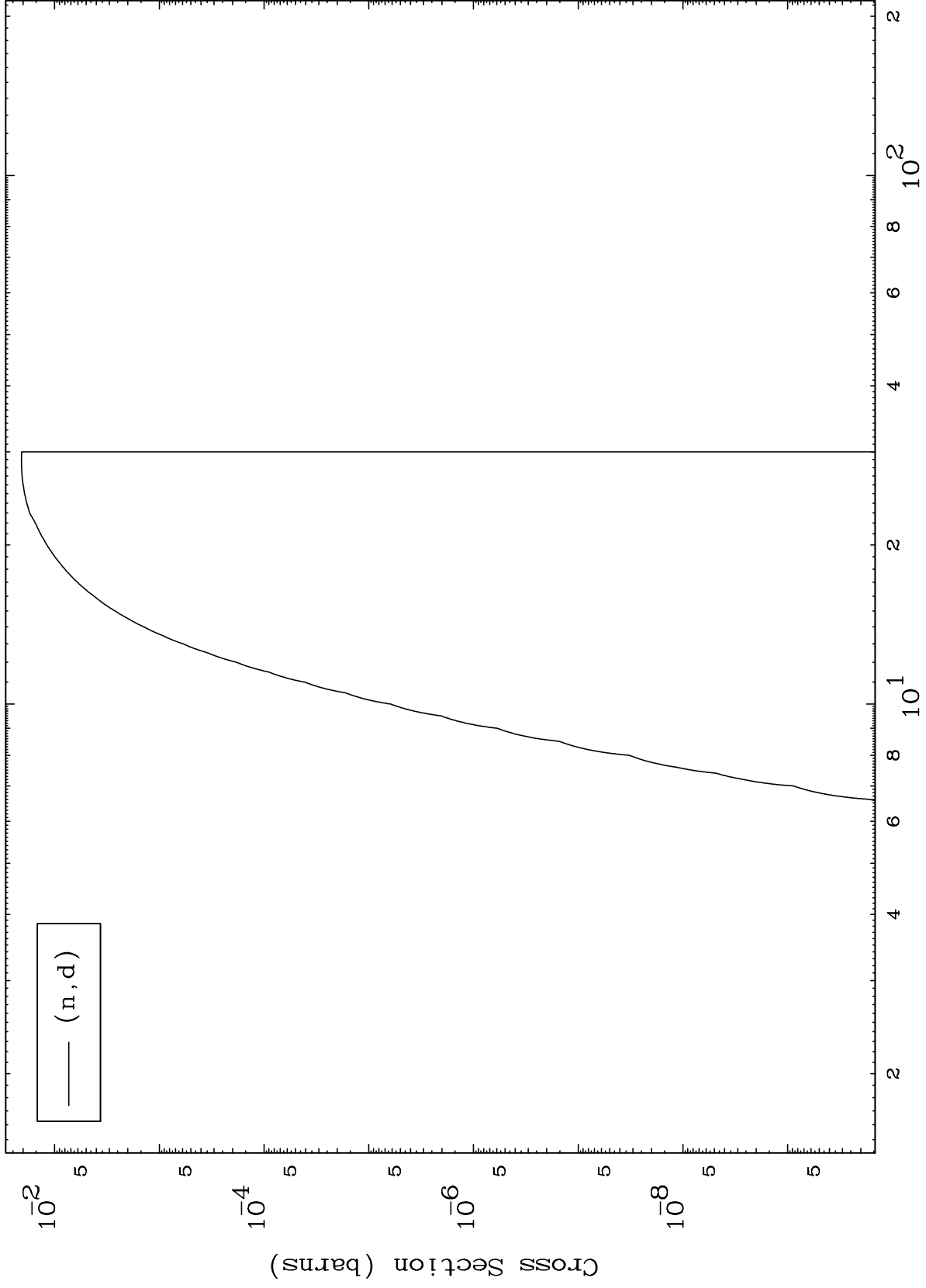
Incident Energy (MeV)

81-Tl-198

MAT 81111

(n,d) Levels  
293 Kelvin Cross Sections

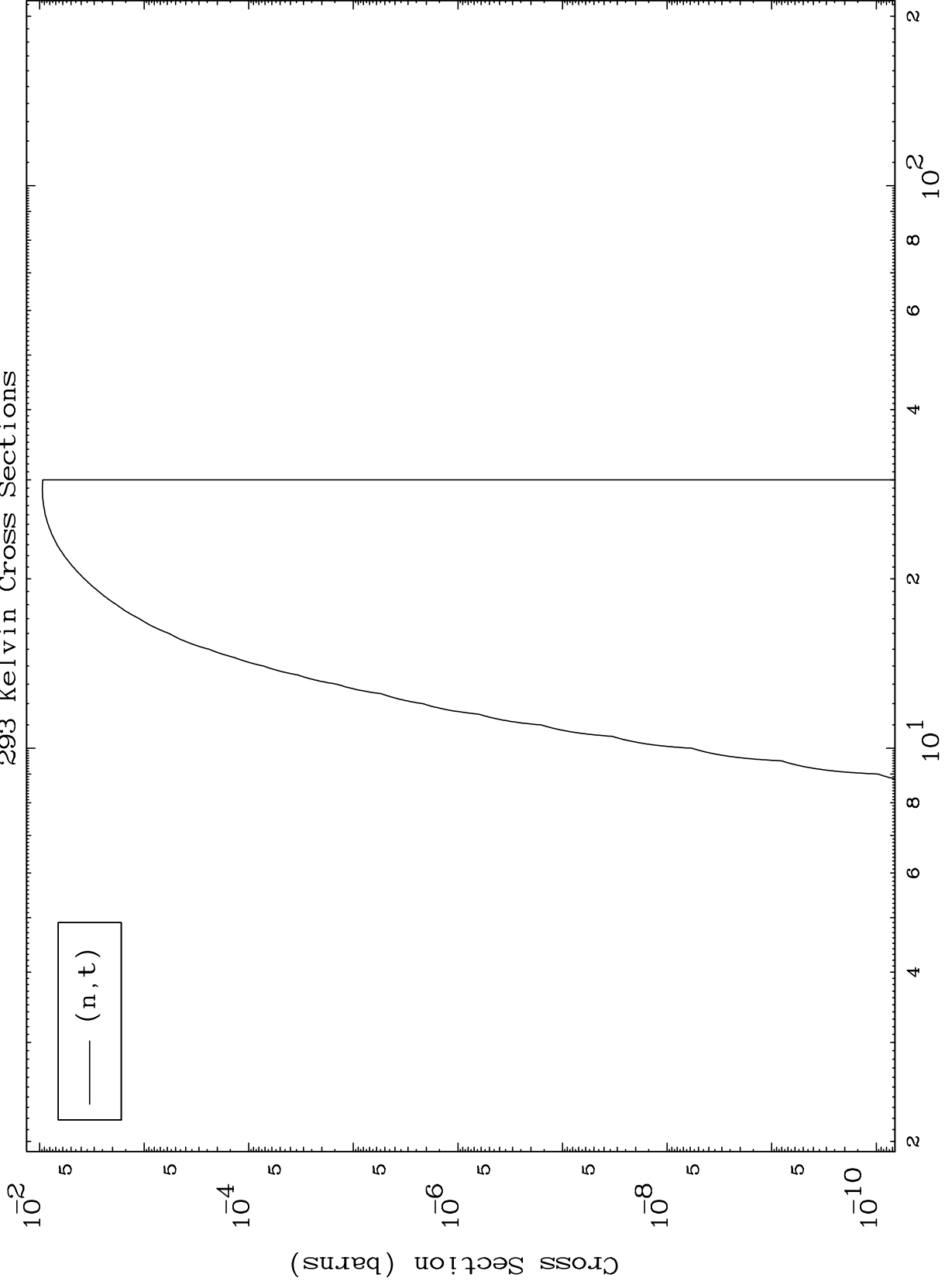
81-Tl-198



MAT 81111

(n,t) Levels  
293 Kelvin Cross Sections

81-Tl-198



12

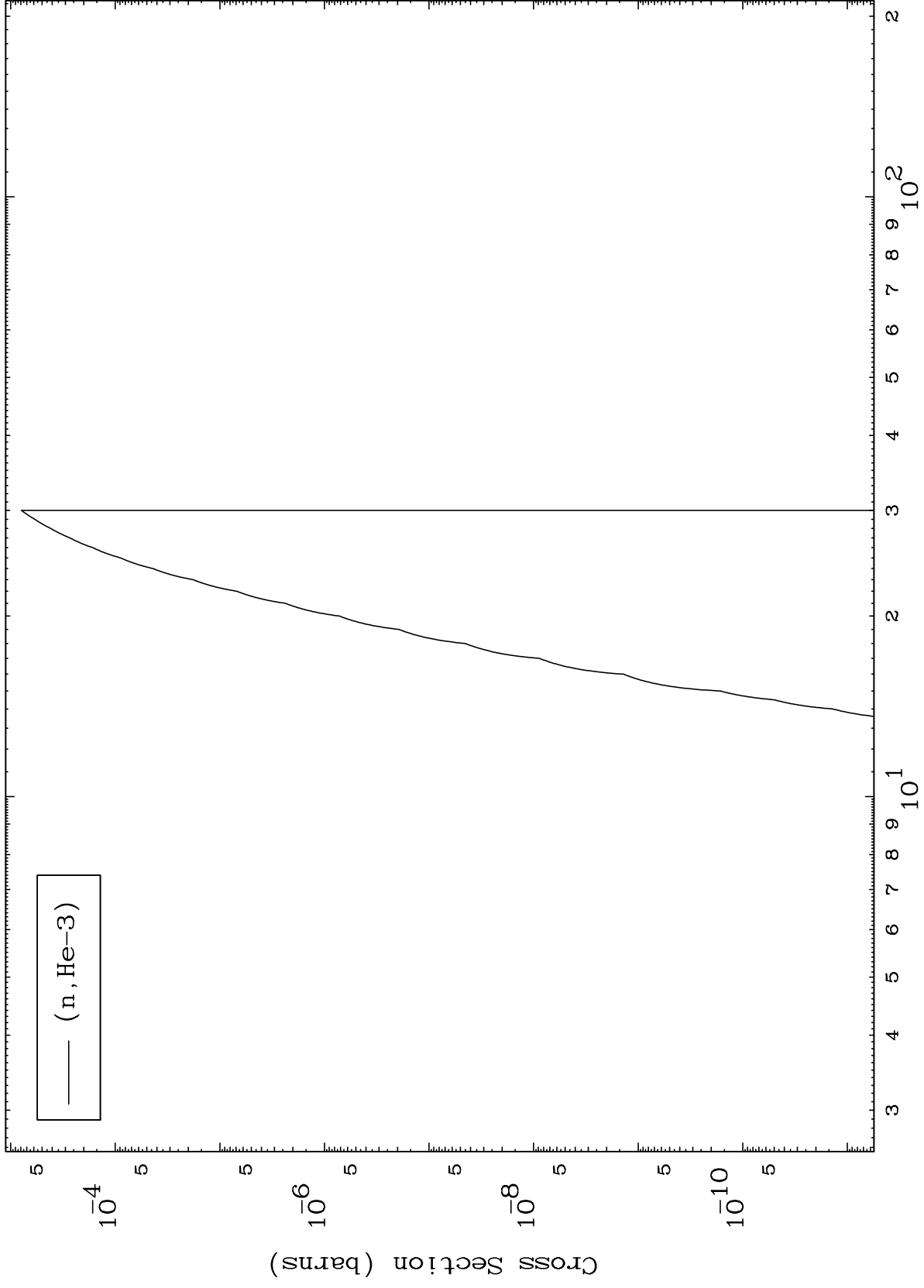
Incident Energy (MeV)

81-Tl-198

MAT 81111

(n,He3) Levels  
293 Kelvin Cross Sections

81-T1-198



13

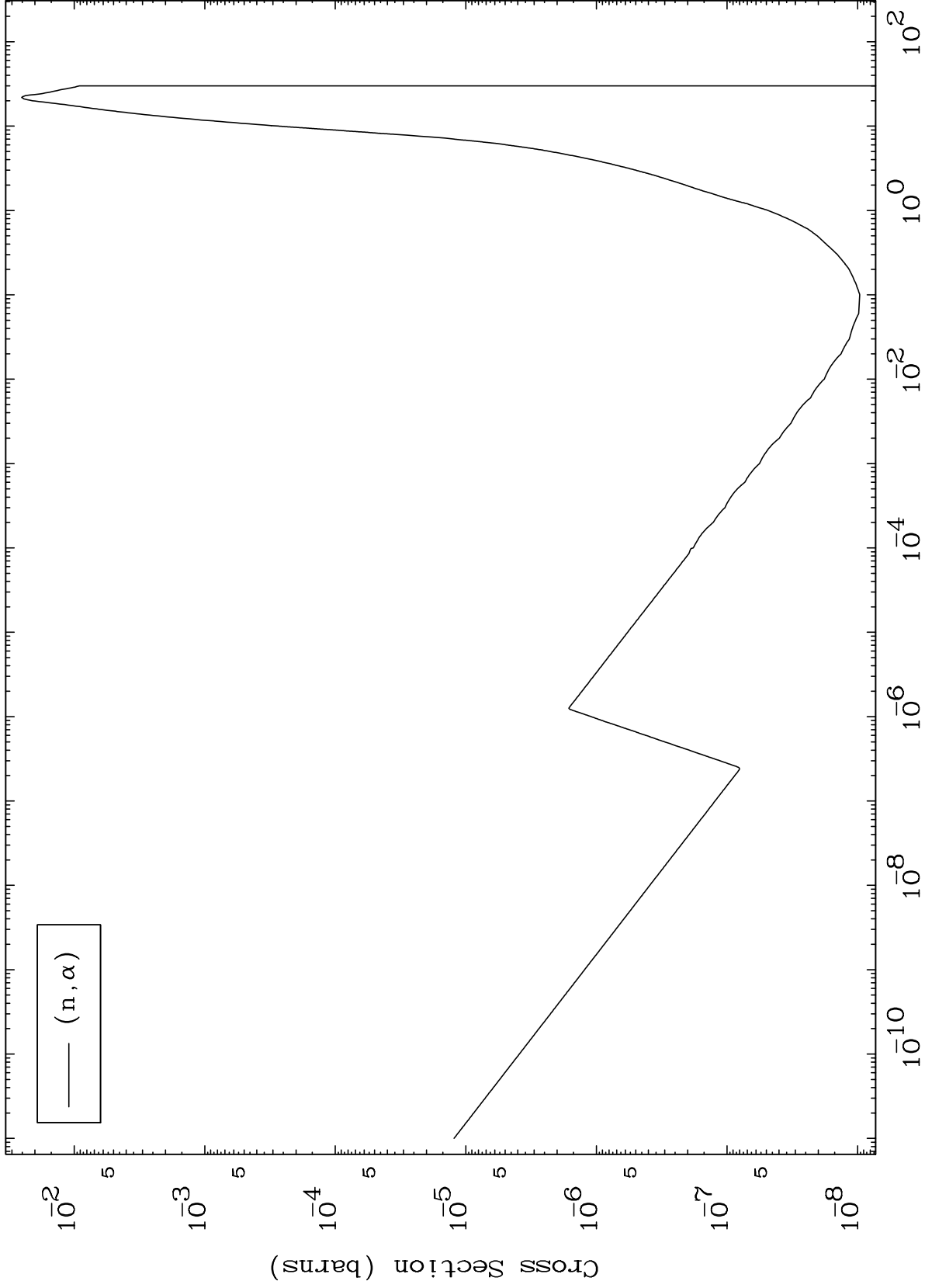
Incident Energy (MeV)

81-T1-198

MAT 81111

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

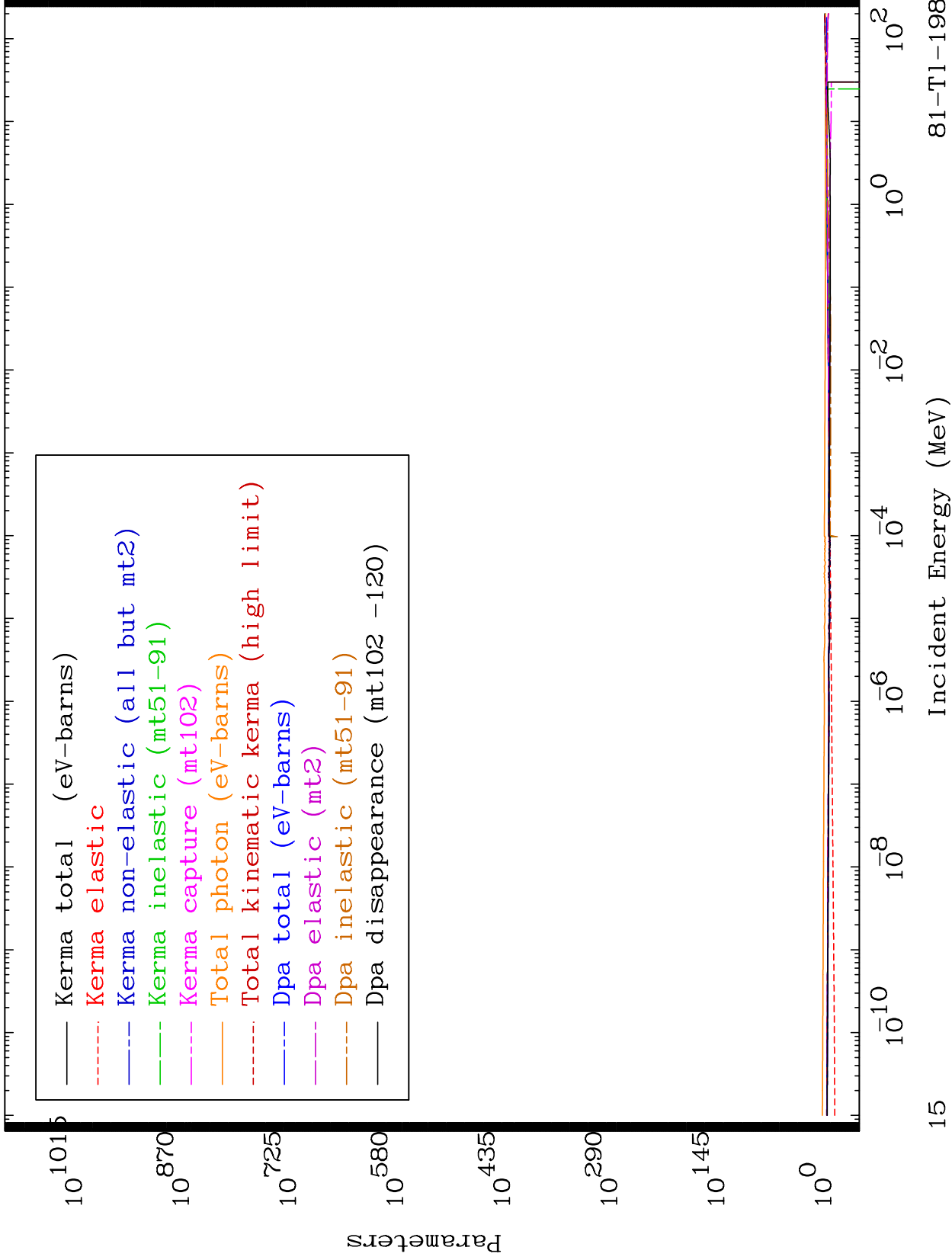
81-Tl-198



14

Incident Energy (MeV)

81-Tl-198



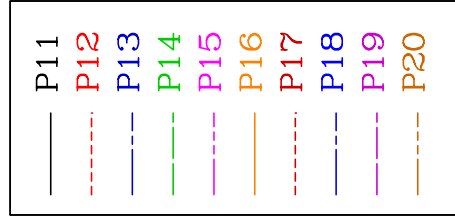




MAT 8111

Elastic Legendre Coefficients

81-T1-198



$\times 10^{-4}$

2.0

Legendre (CM)

1.5

1.0

0.5

0.0

5

10

15

20

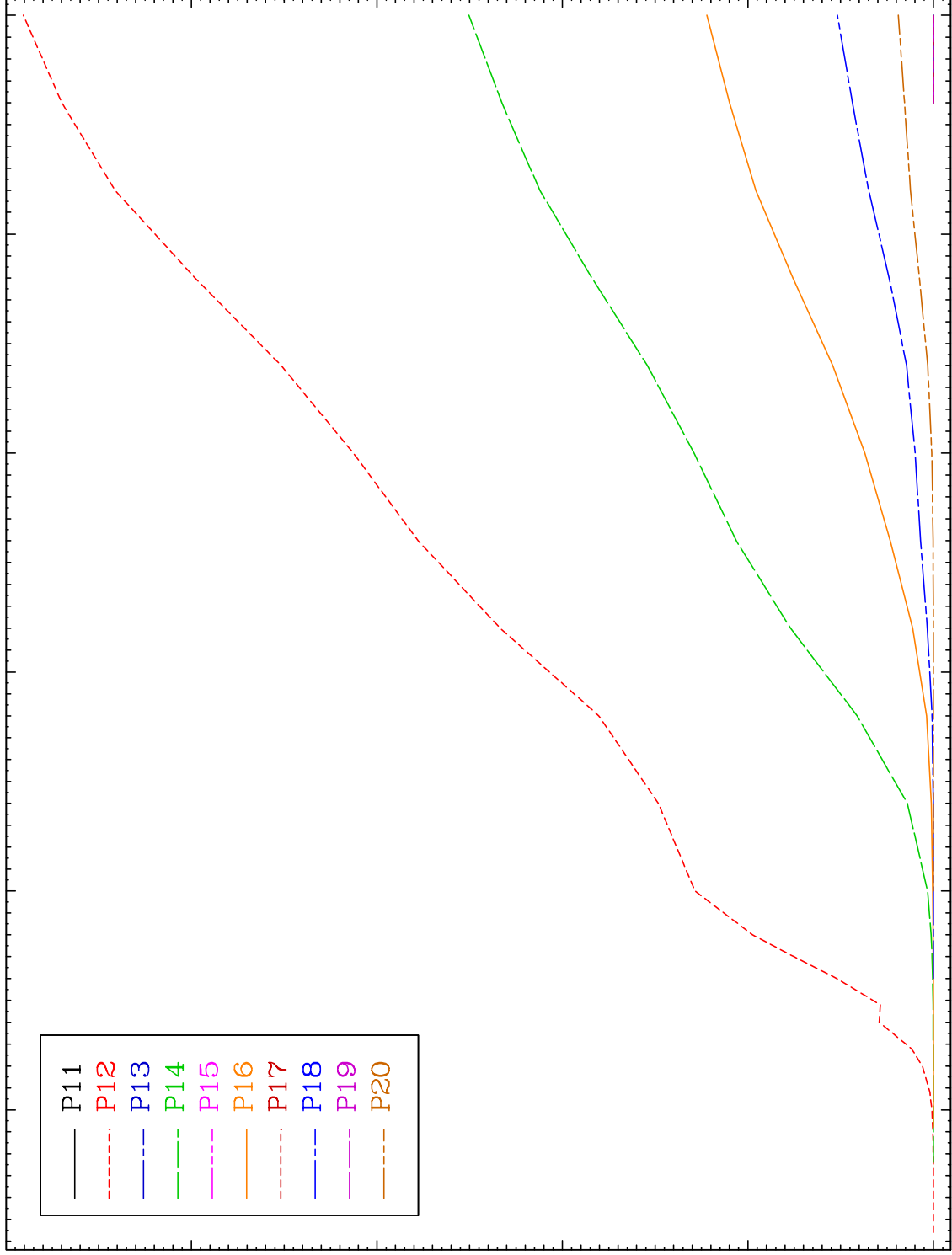
25

30

17

Incident Energy (MeV)

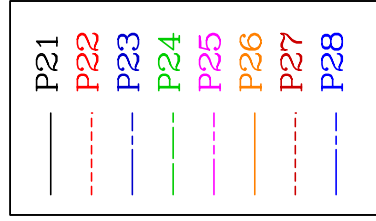
81-T1-198



MAT 8111

Elastic  
Legendre Coefficients

81-T1-198



$\times 10^{-6}$

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

25

30

18

Incident Energy (MeV)

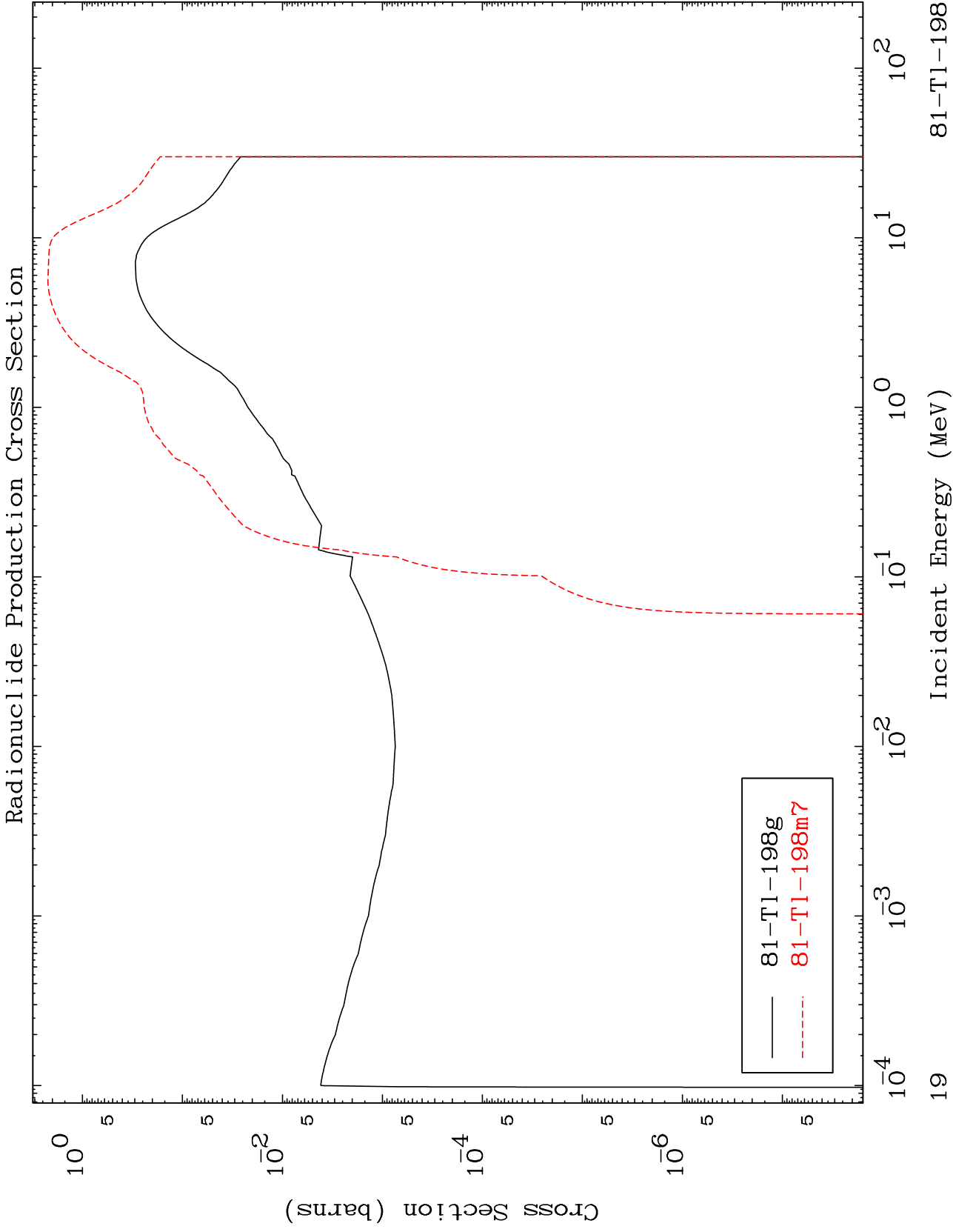
81-T1-198



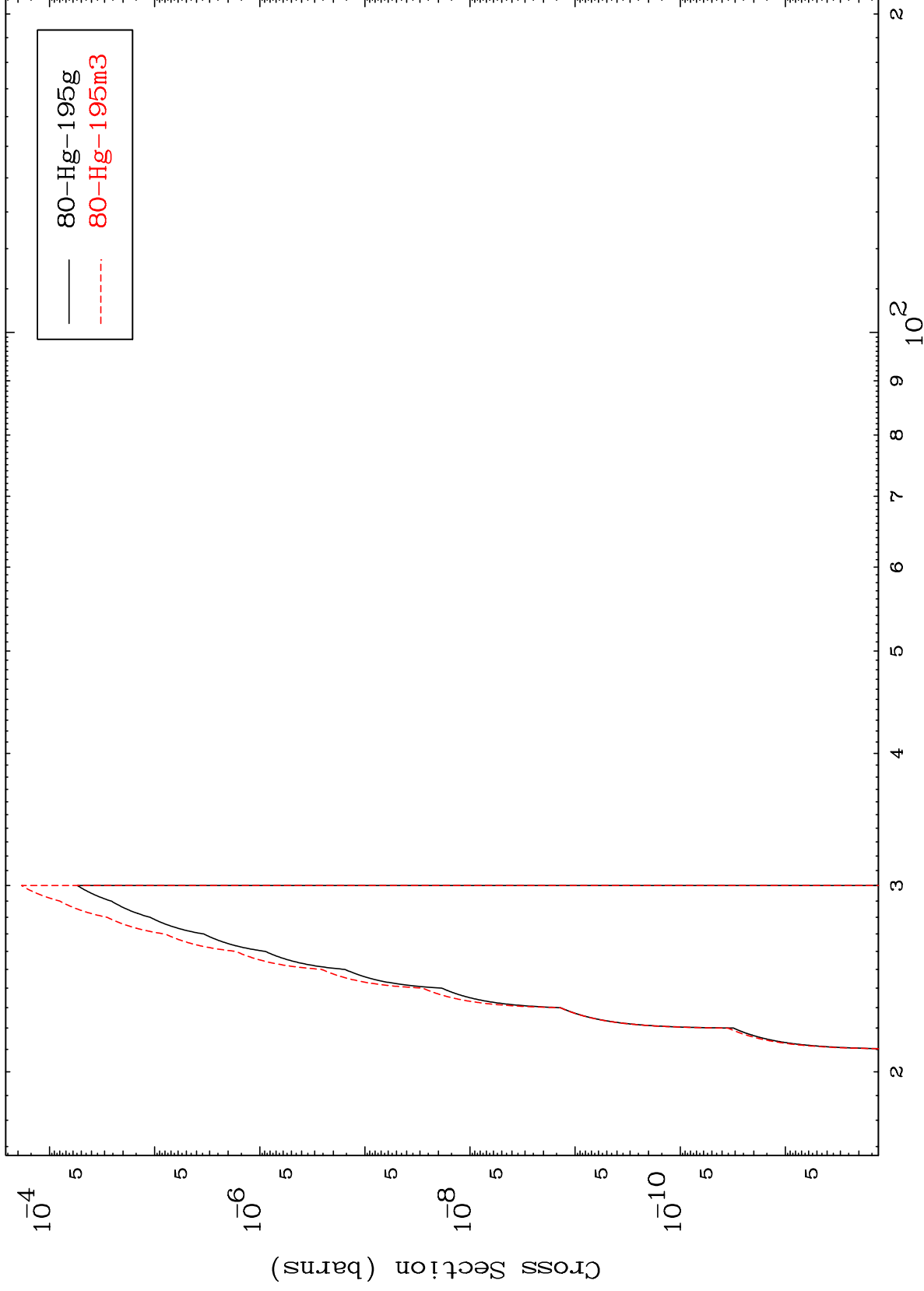
MAT 81111

81-Tl-198

Inelastic  
Radionuclide Production Cross Section



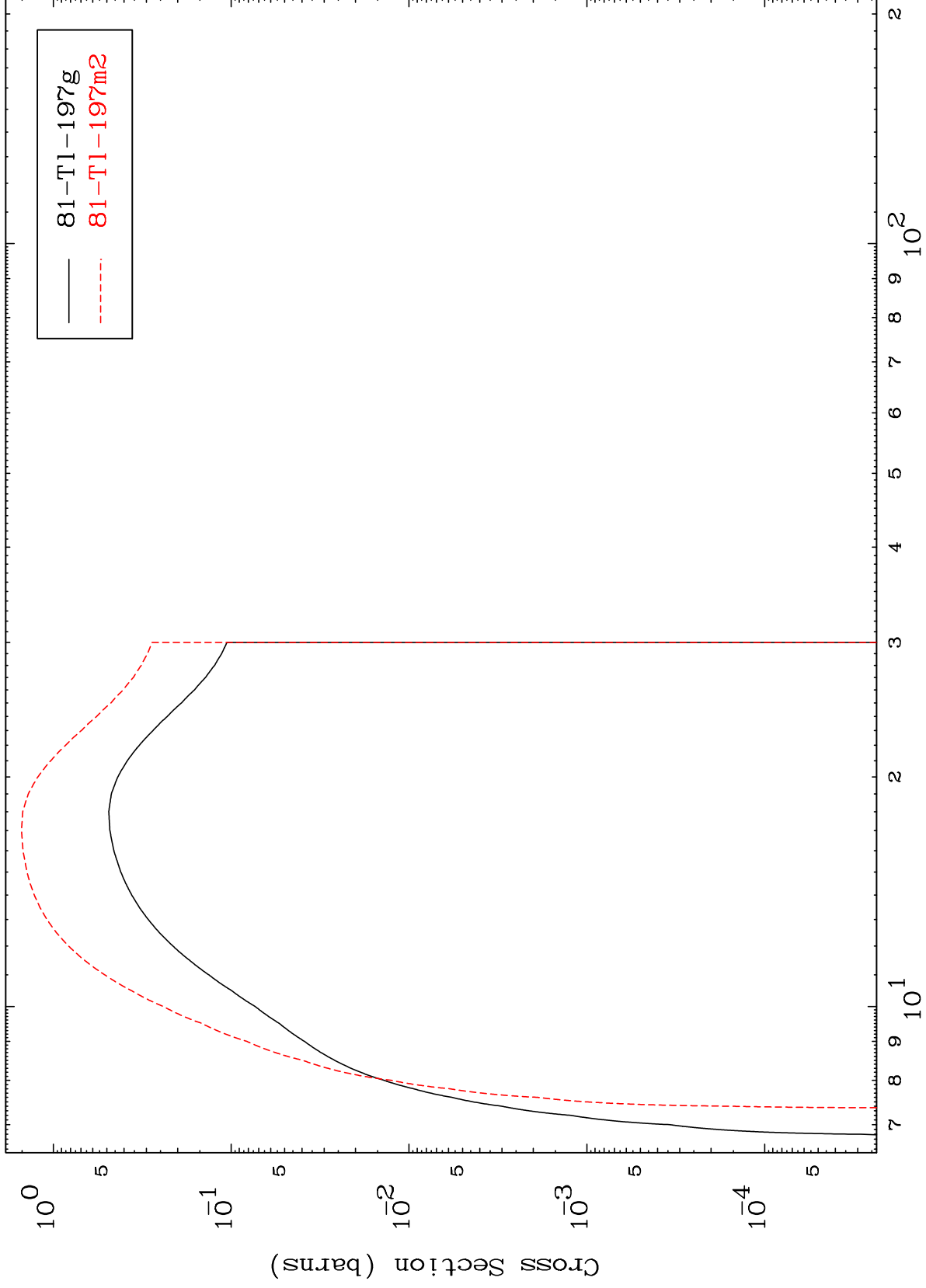
Radionuclide Production Cross Section



MAT 81111

81-Tl-198

(n,2n)  
Radionuclide Production Cross Section



21

Incident Energy (MeV)

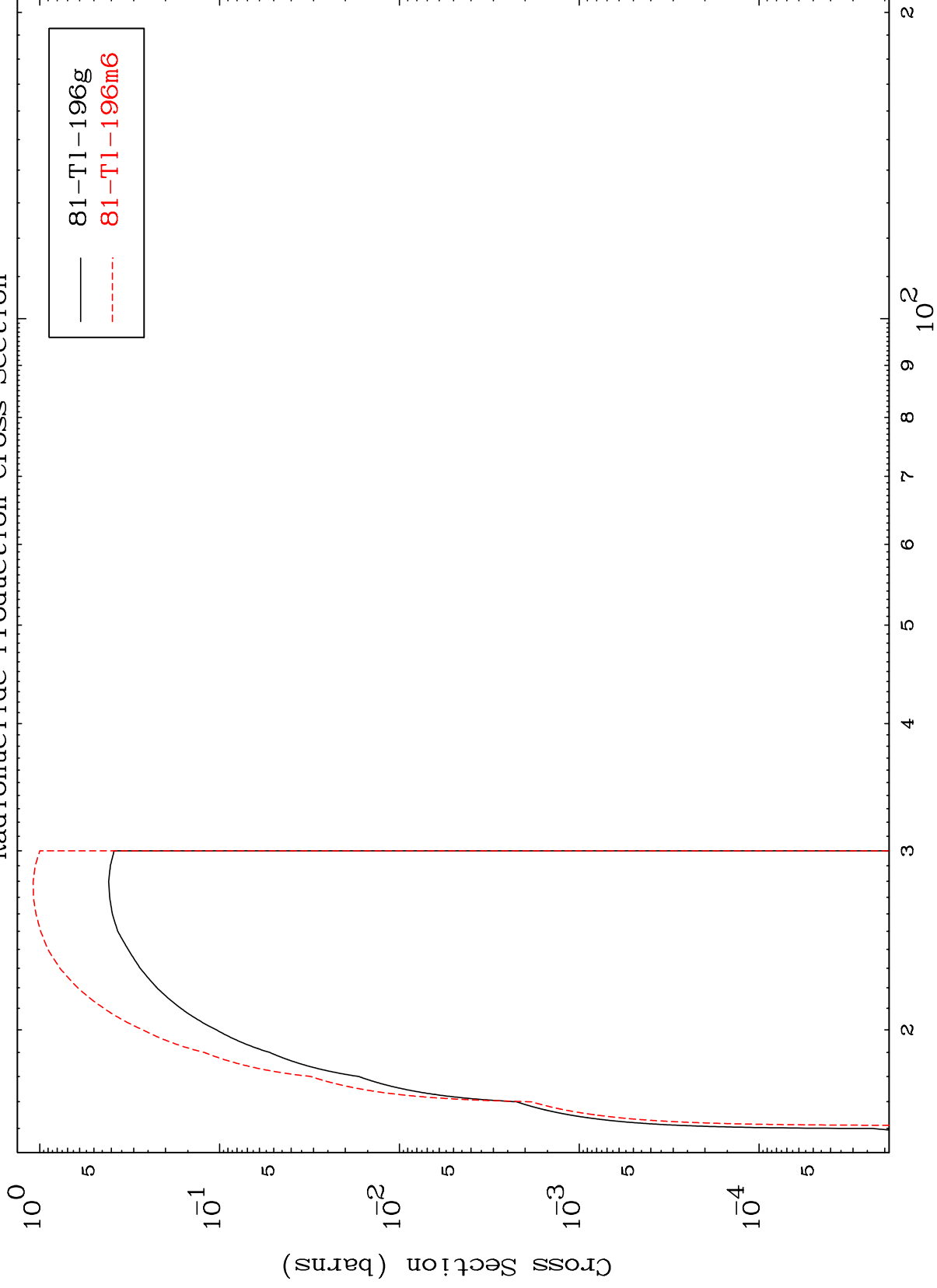
81-Tl-198

MAT 81111

(n,3n)

81-Tl-198

Radionuclide Production Cross Section



22

Incident Energy (MeV)

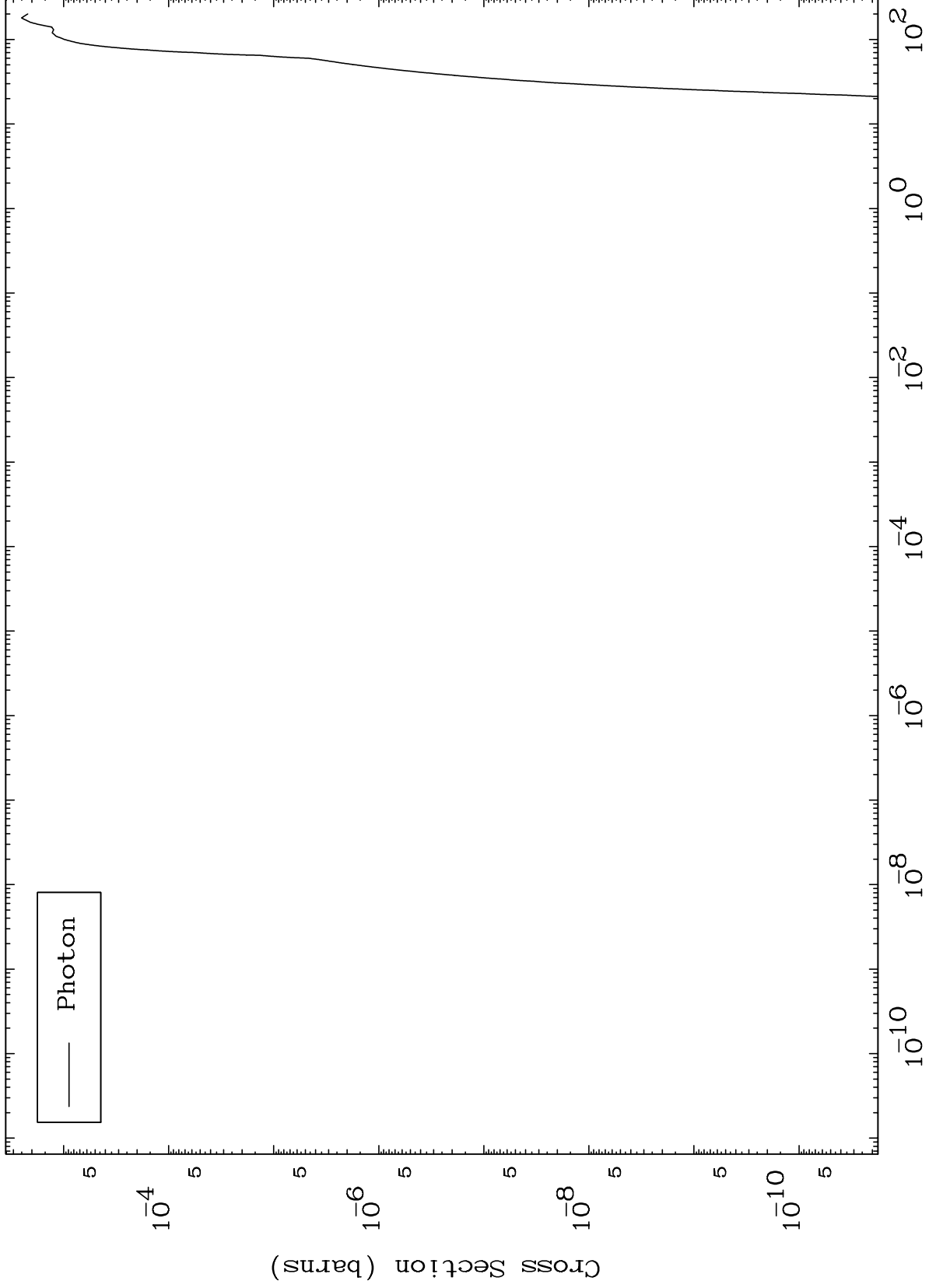
81-Tl-198

MAT 81111

Fission

81-Tl-198

Radionuclide Production Cross Section



23

Incident Energy (MeV)

81-Tl-198

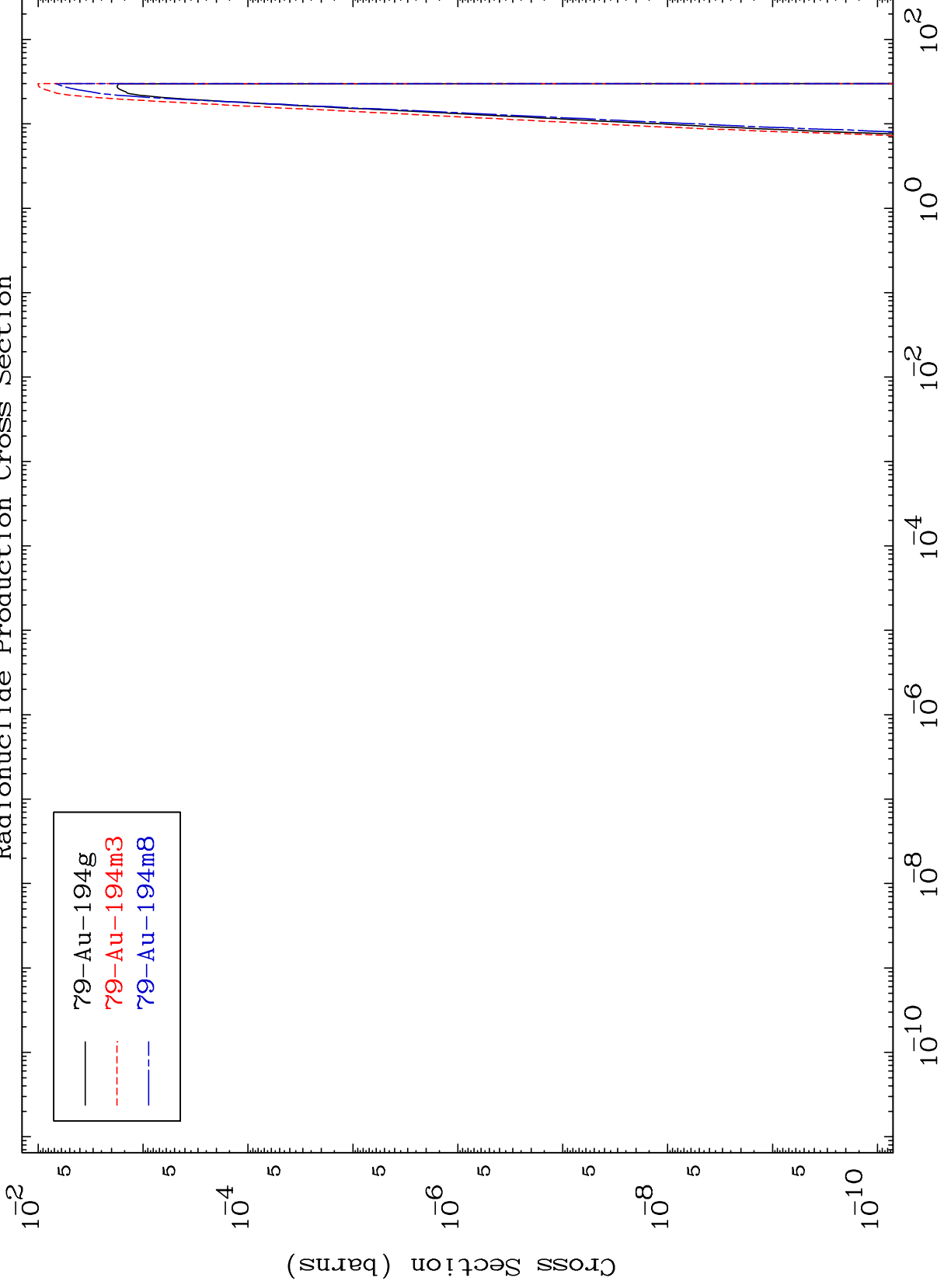


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$(n, n') \alpha$

81-Tl-198

Radionuclide Production Cross Section

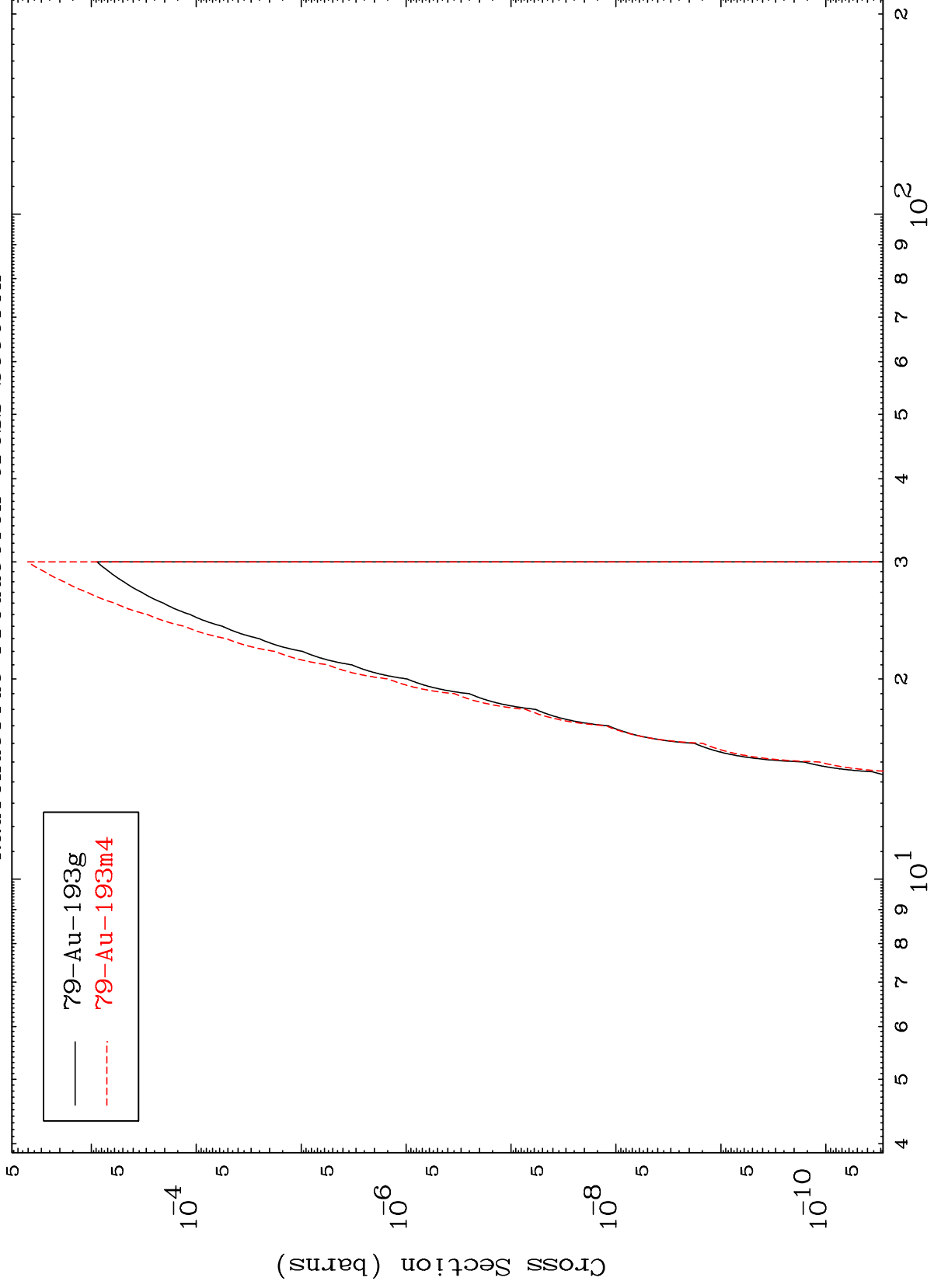


MAT 81111

(n,2n)  $\alpha$

81-Tl-198

Radionuclide Production Cross Section

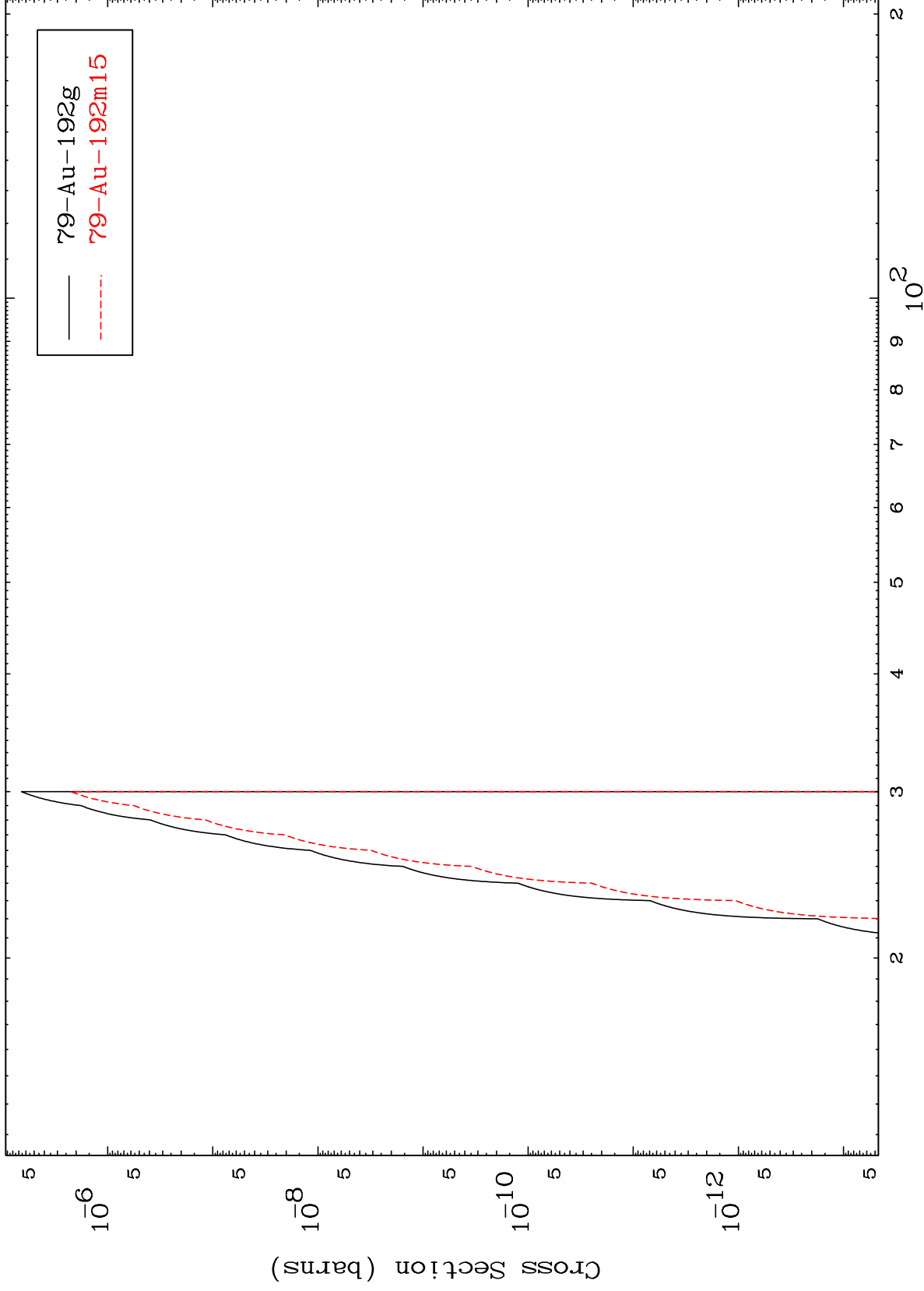


25

Incident Energy (MeV)

81-Tl-198

Radionuclide Production Cross Section

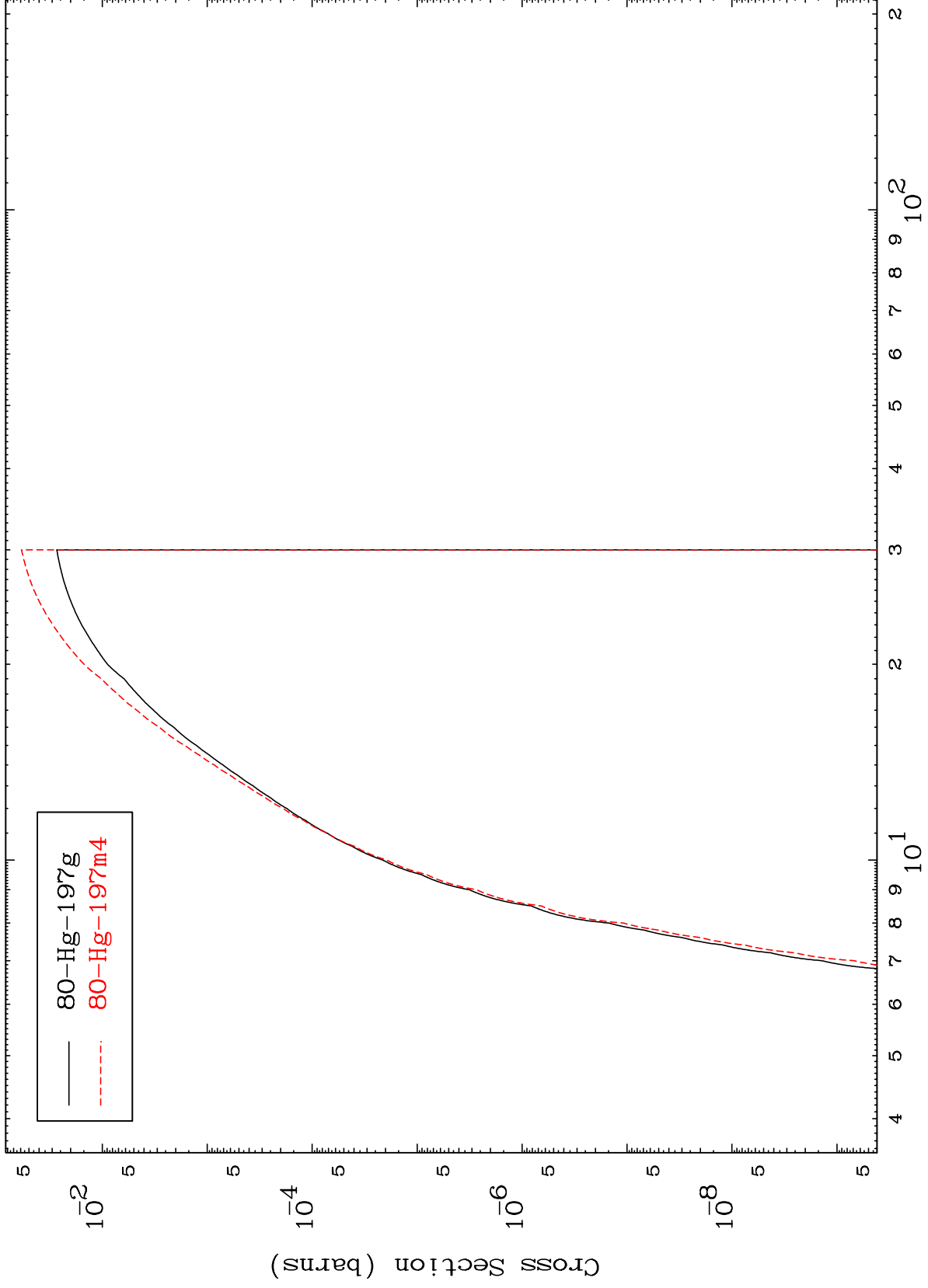


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(n,n') p

81-Tl-198

Radionuclide Production Cross Section



27

Incident Energy (MeV)

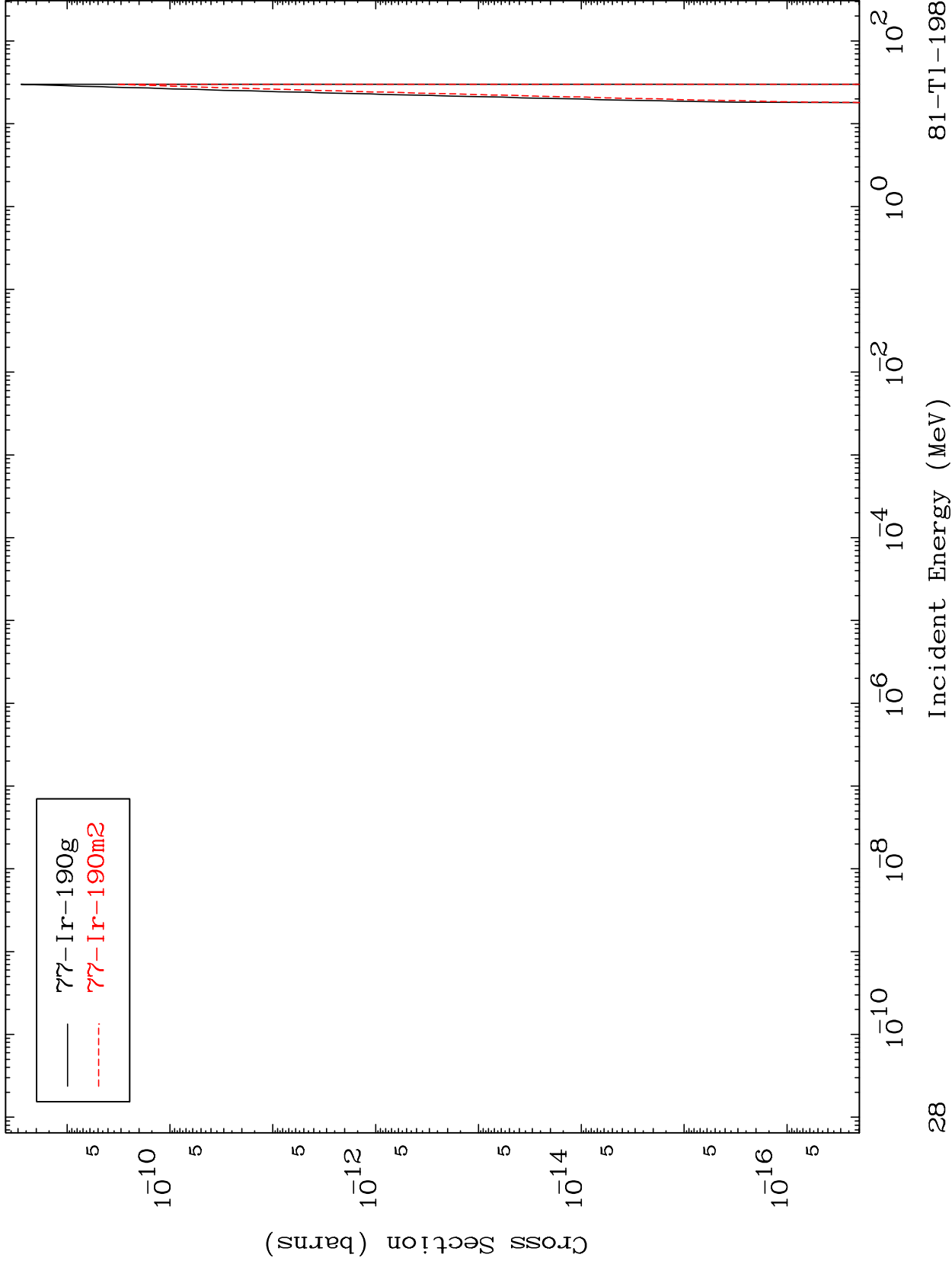
81-Tl-198

MAT 81111

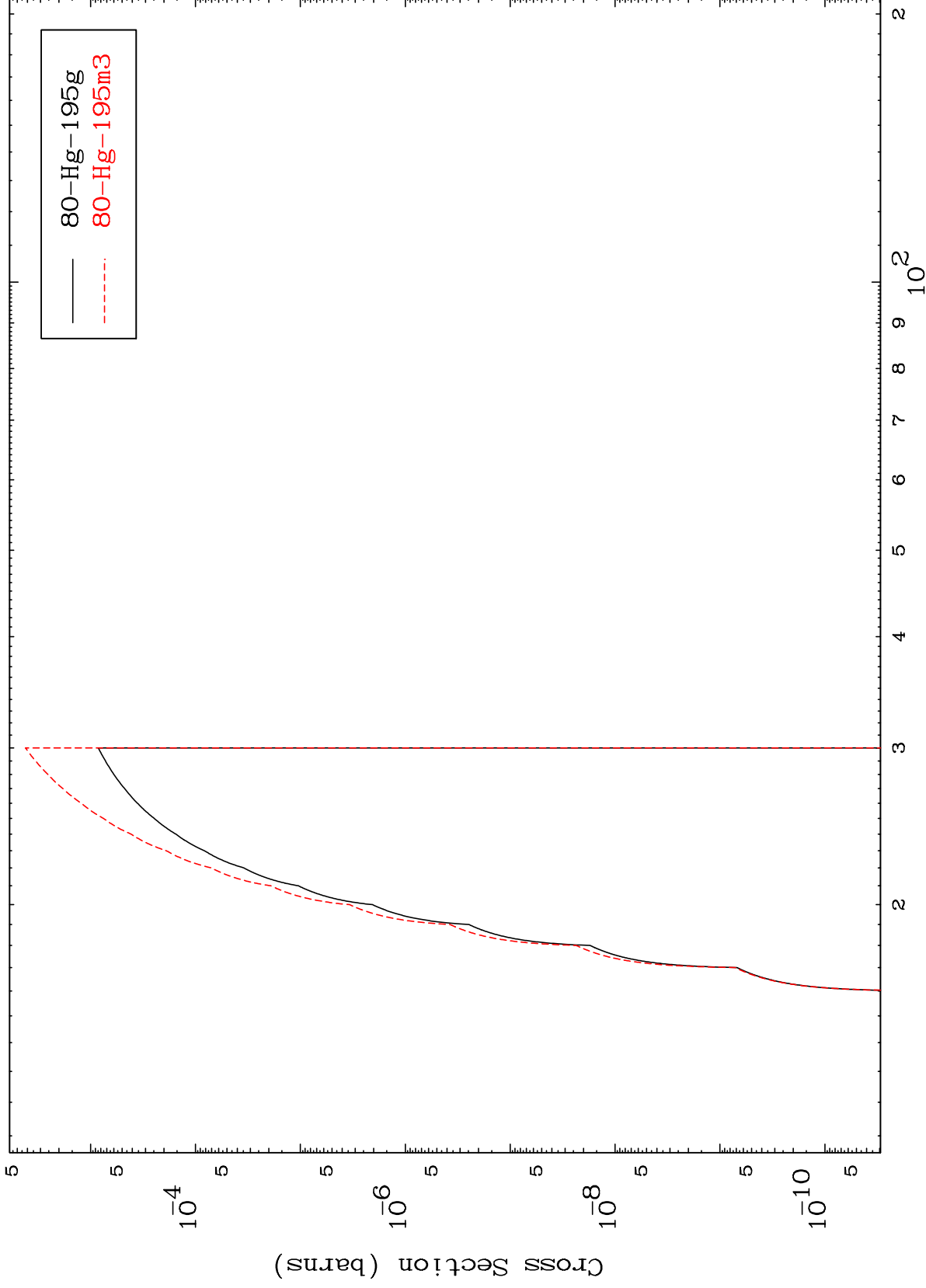
(n,n') 2α

81-Tl-198

Radionuclide Production Cross Section



Radionuclide Production Cross Section

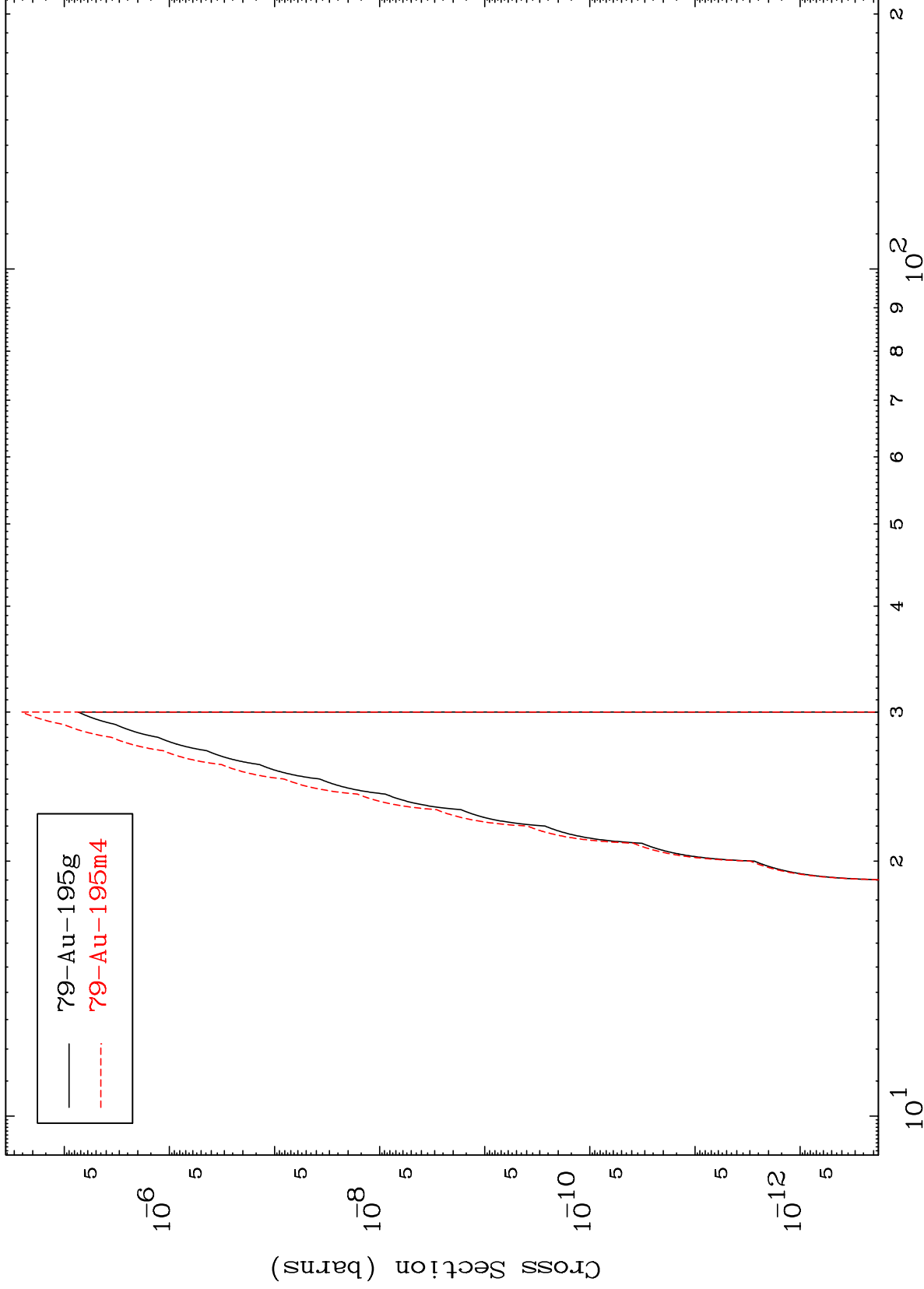


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(n,n') He-3

81-Tl-198

Radionuclide Production Cross Section

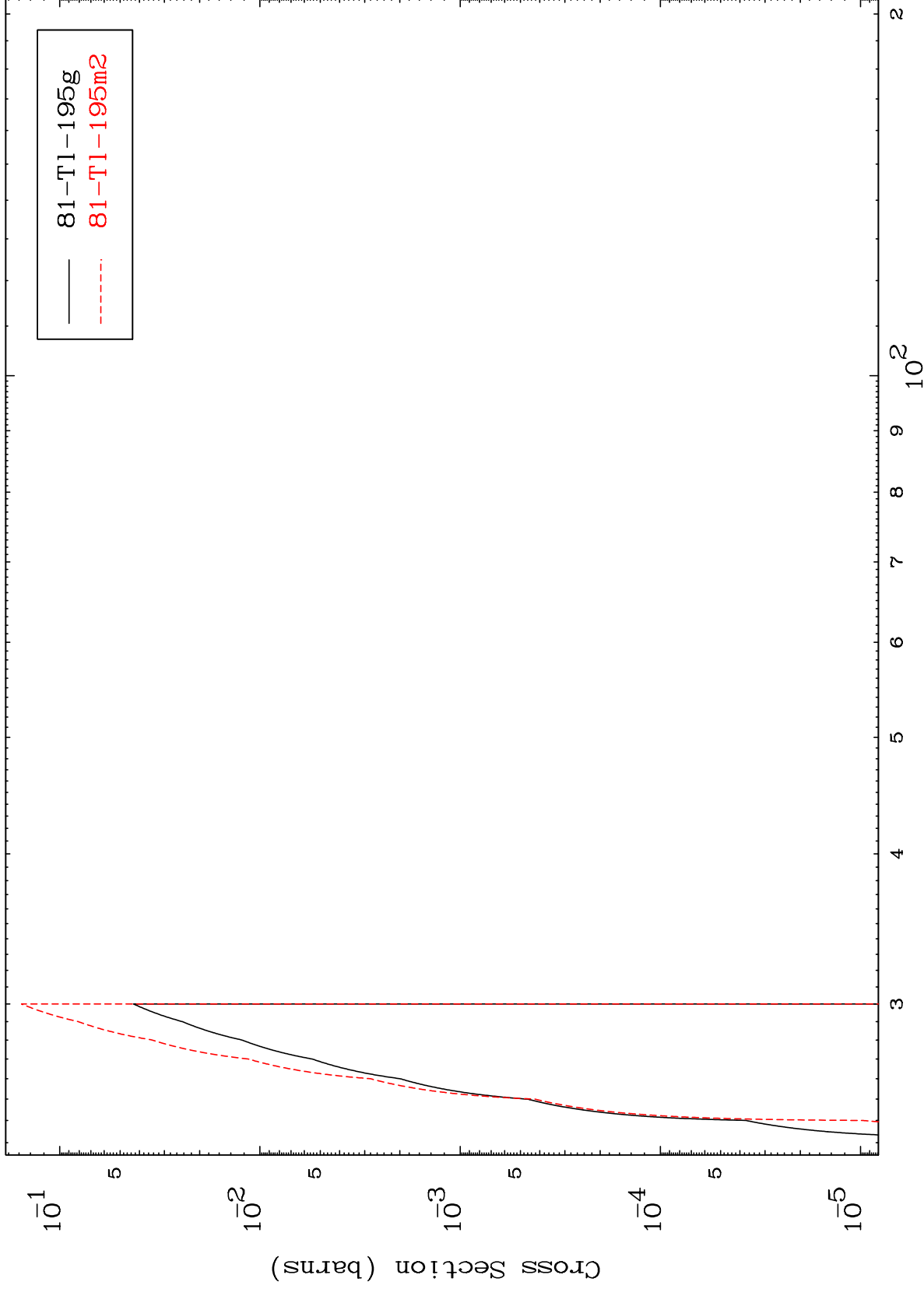


30

Incident Energy (MeV)

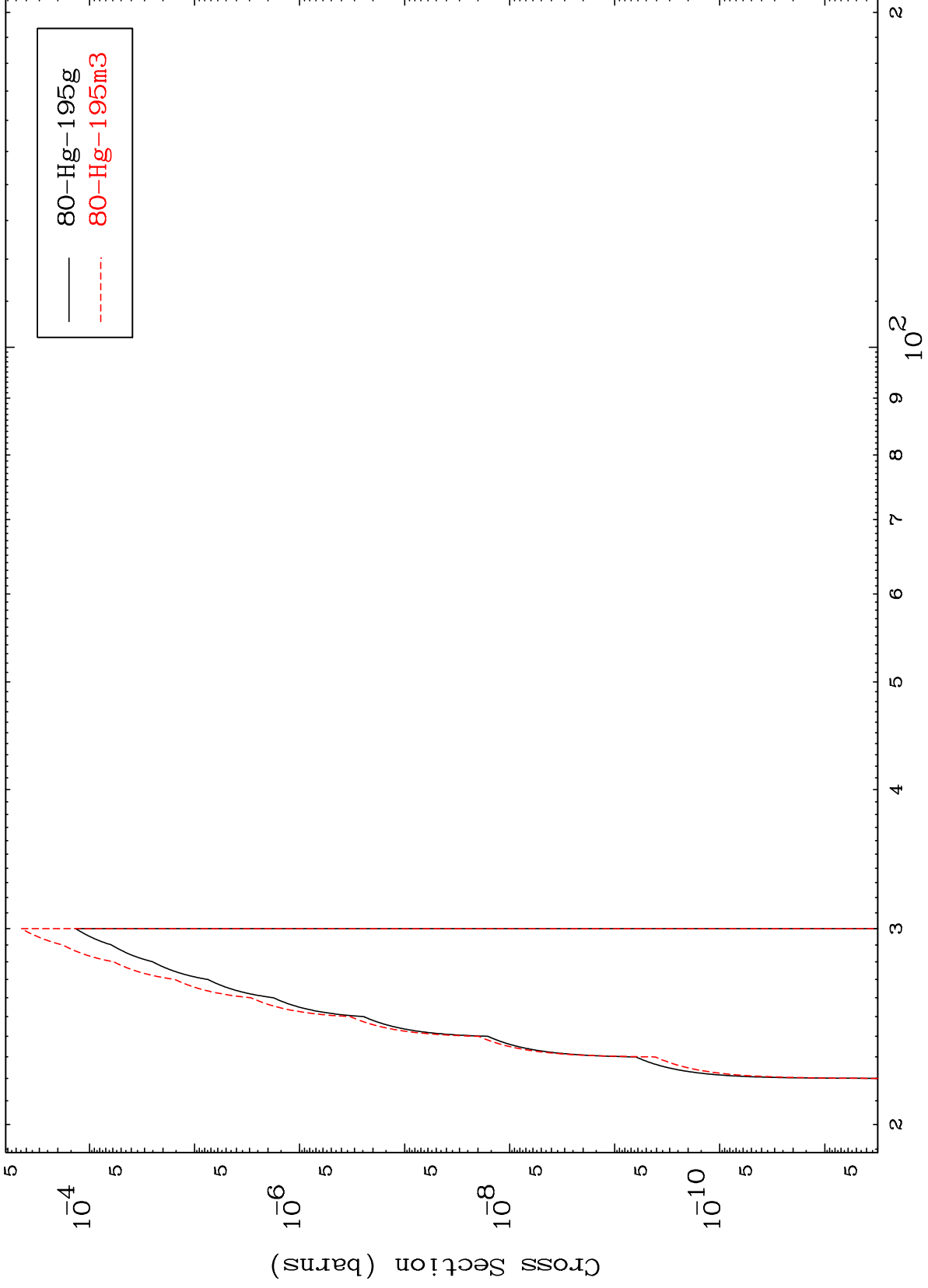
81-Tl-198

Radionuclide Production Cross Section

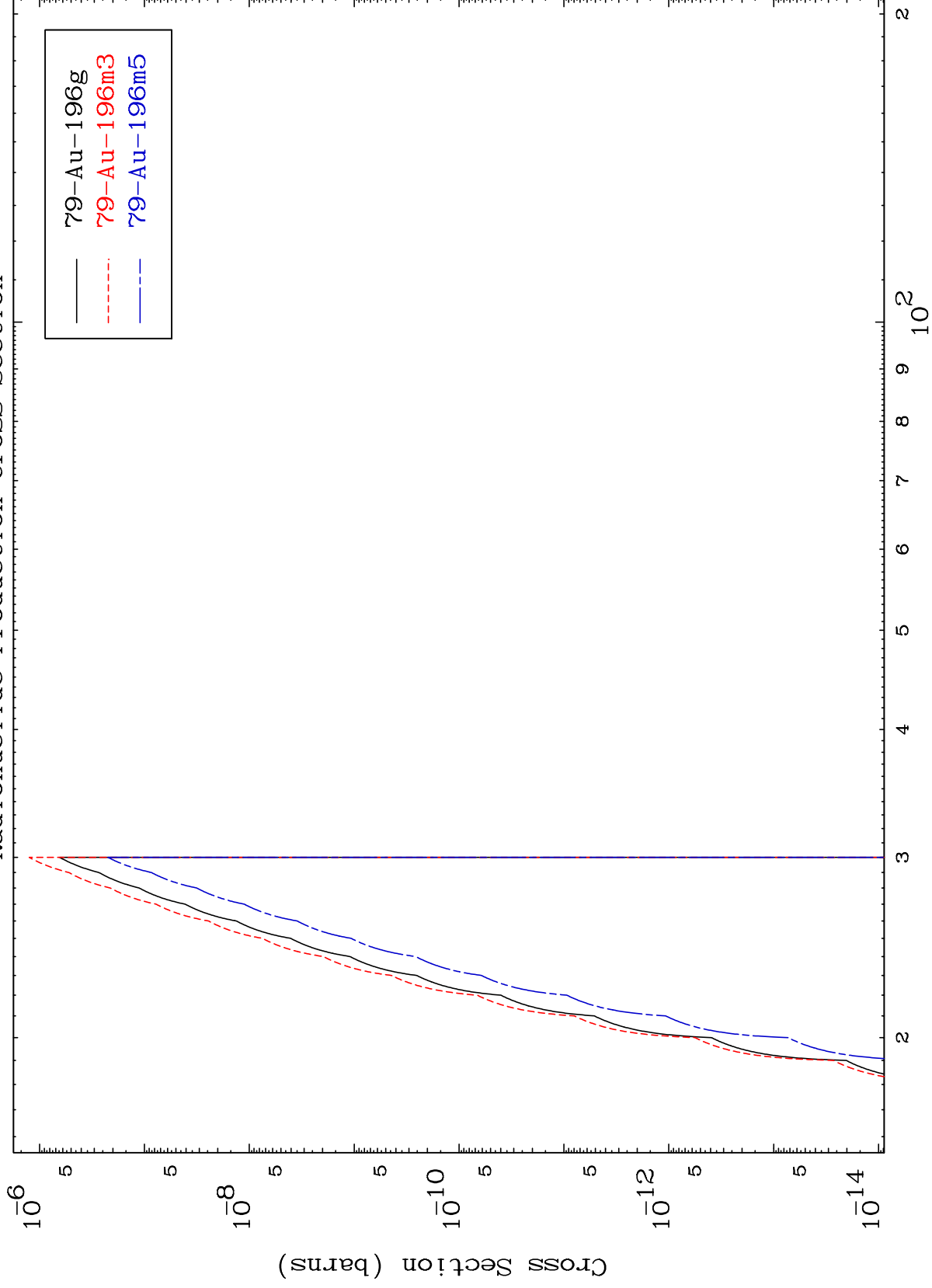




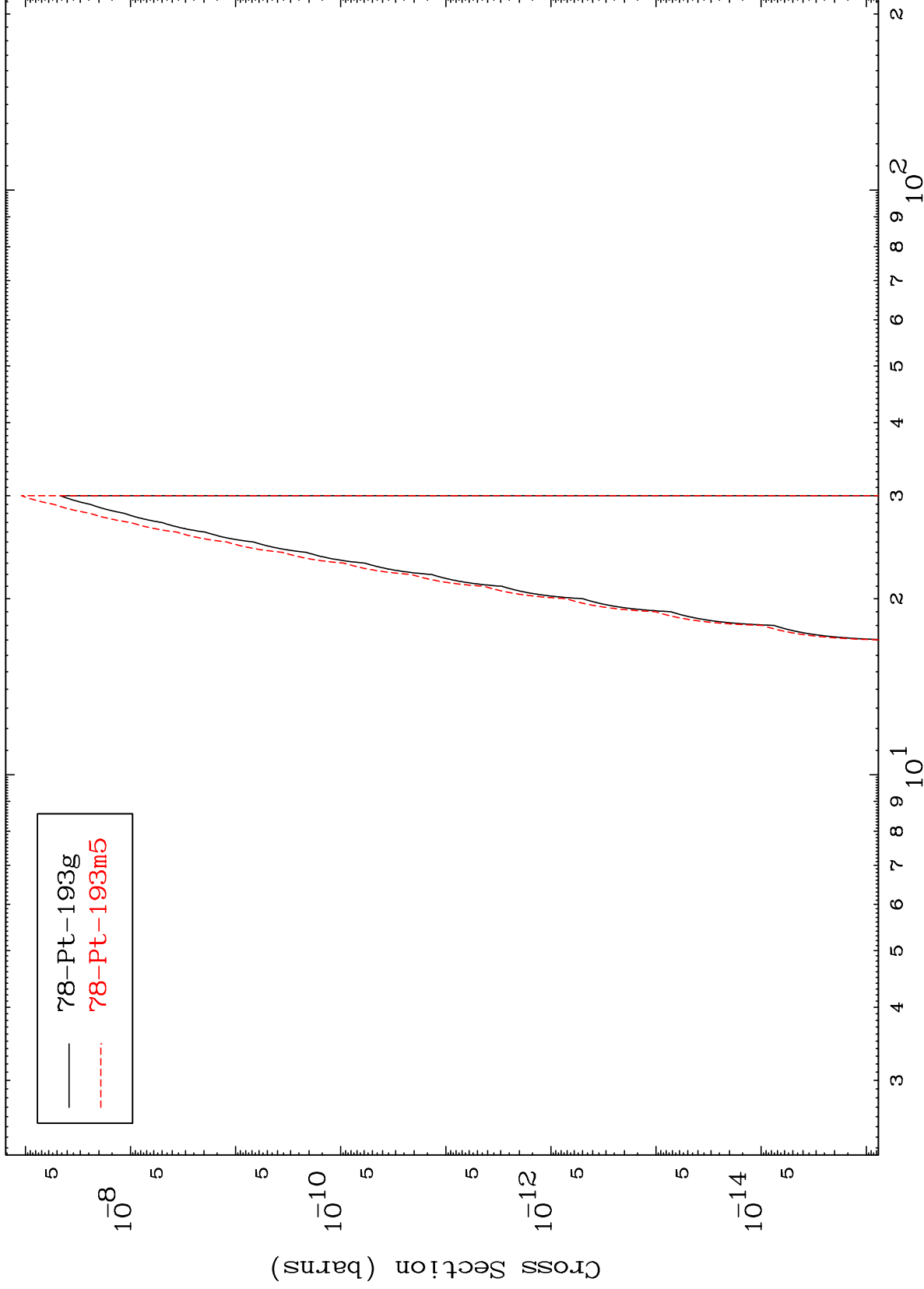
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

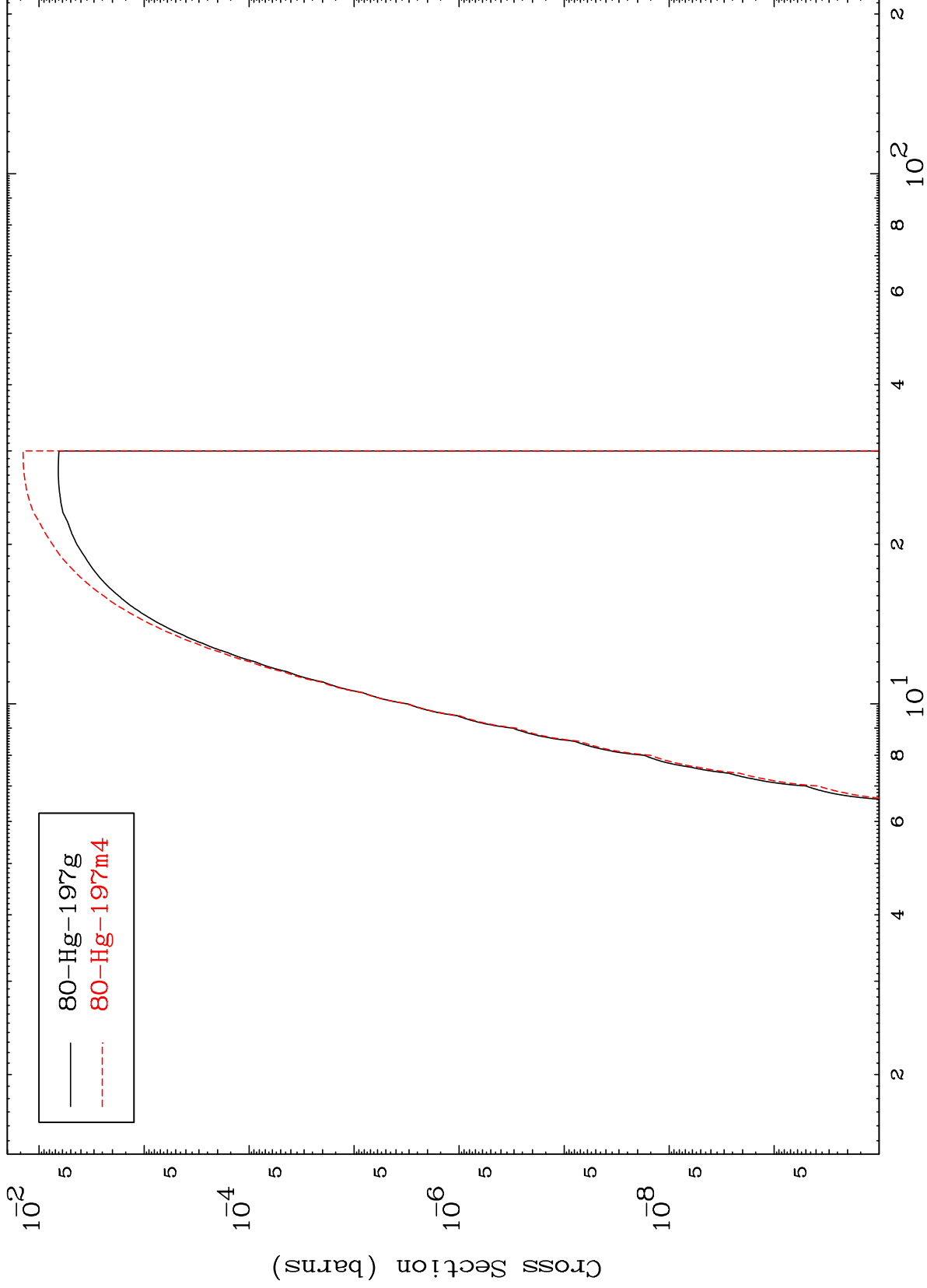


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(n,d)

81-Tl-198

Radionuclide Production Cross Section

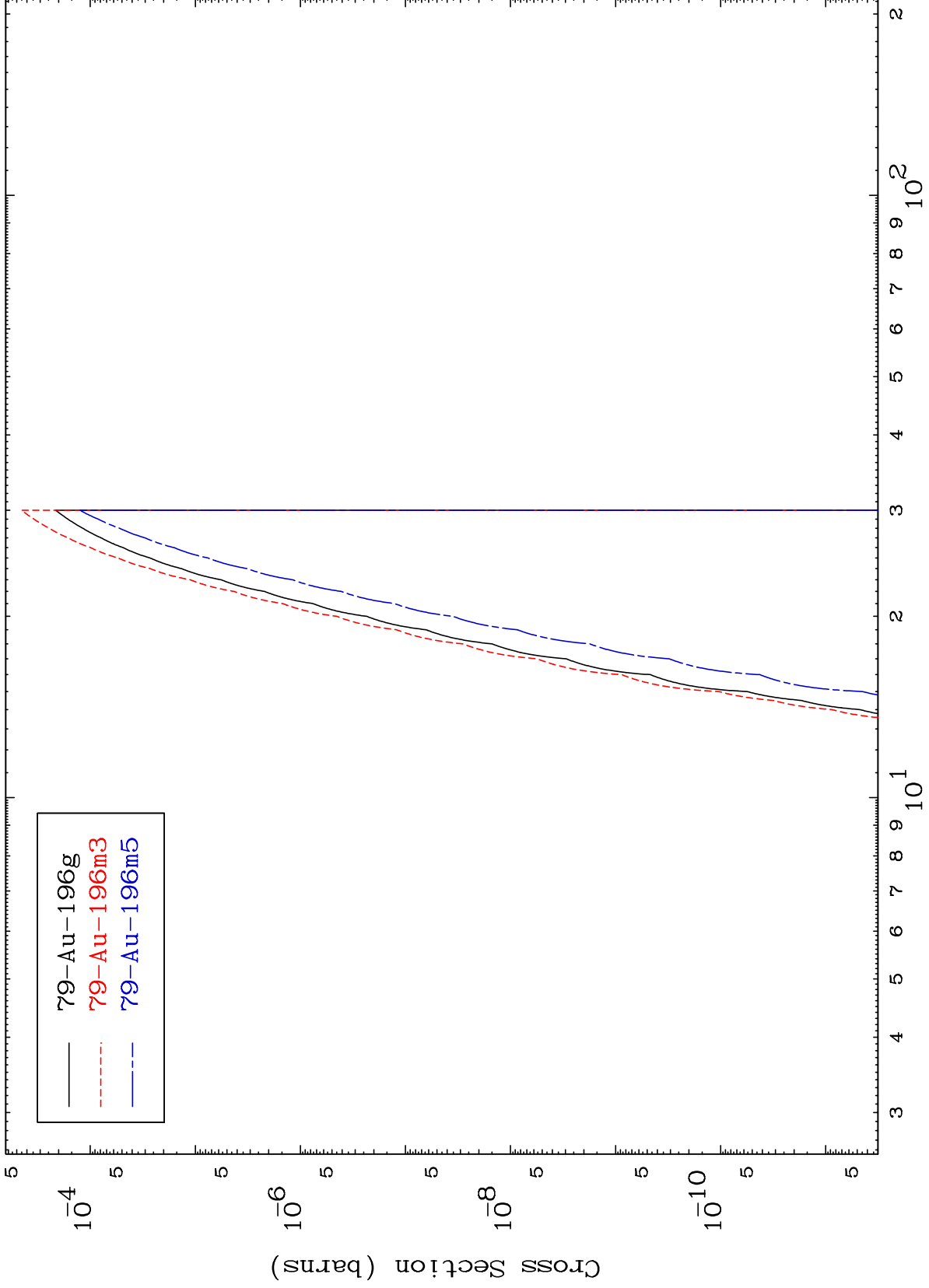


35

Incident Energy (MeV)

81-Tl-198

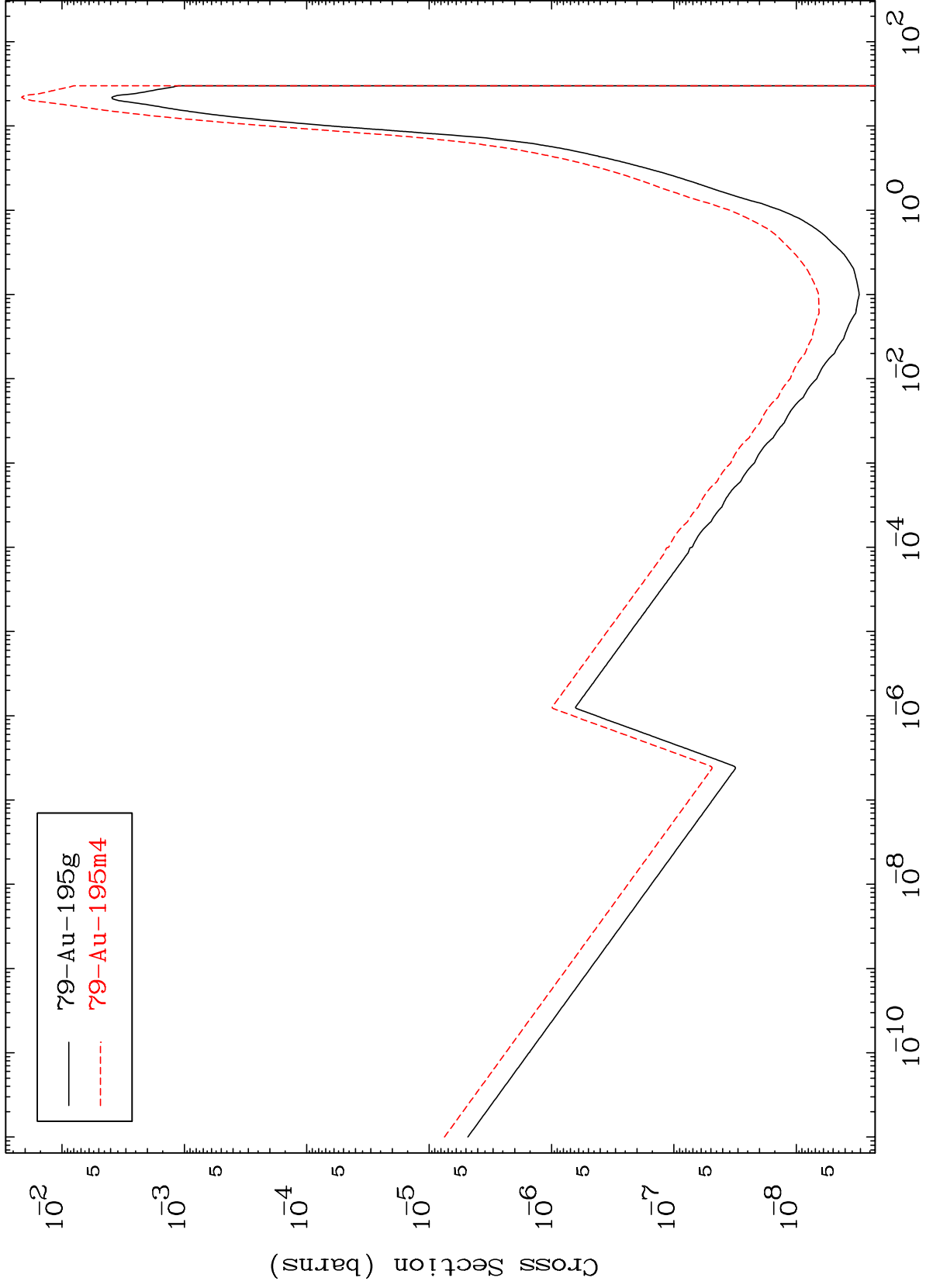
Radionuclide Production Cross Section



MAT 81111

81-Tl-198

Radionuclide Production Cross Section  
(n,  $\alpha$ )



37

Incident Energy (MeV)

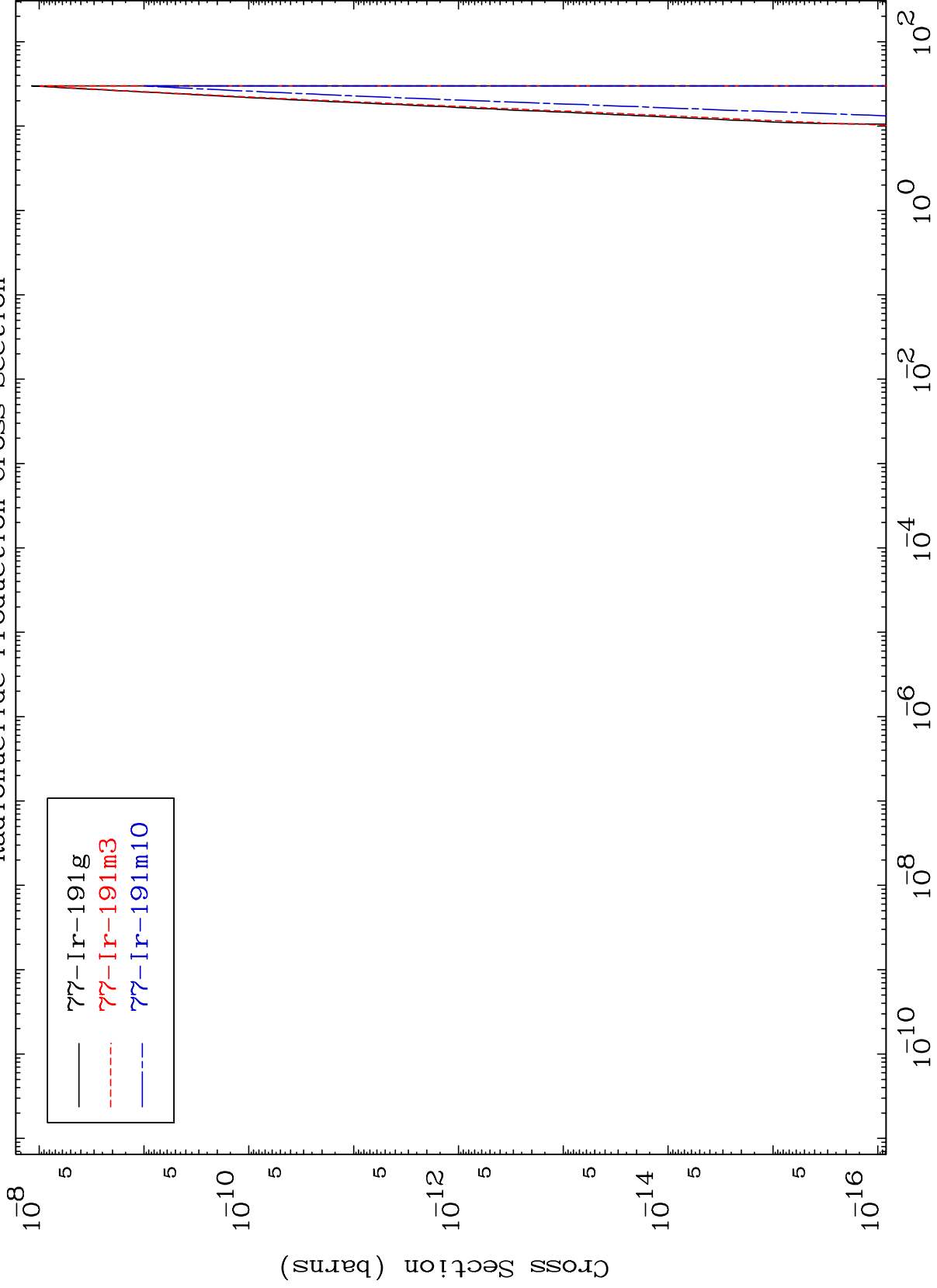
81-Tl-198

MAT 81111

(n,2α)

81-Tl-198

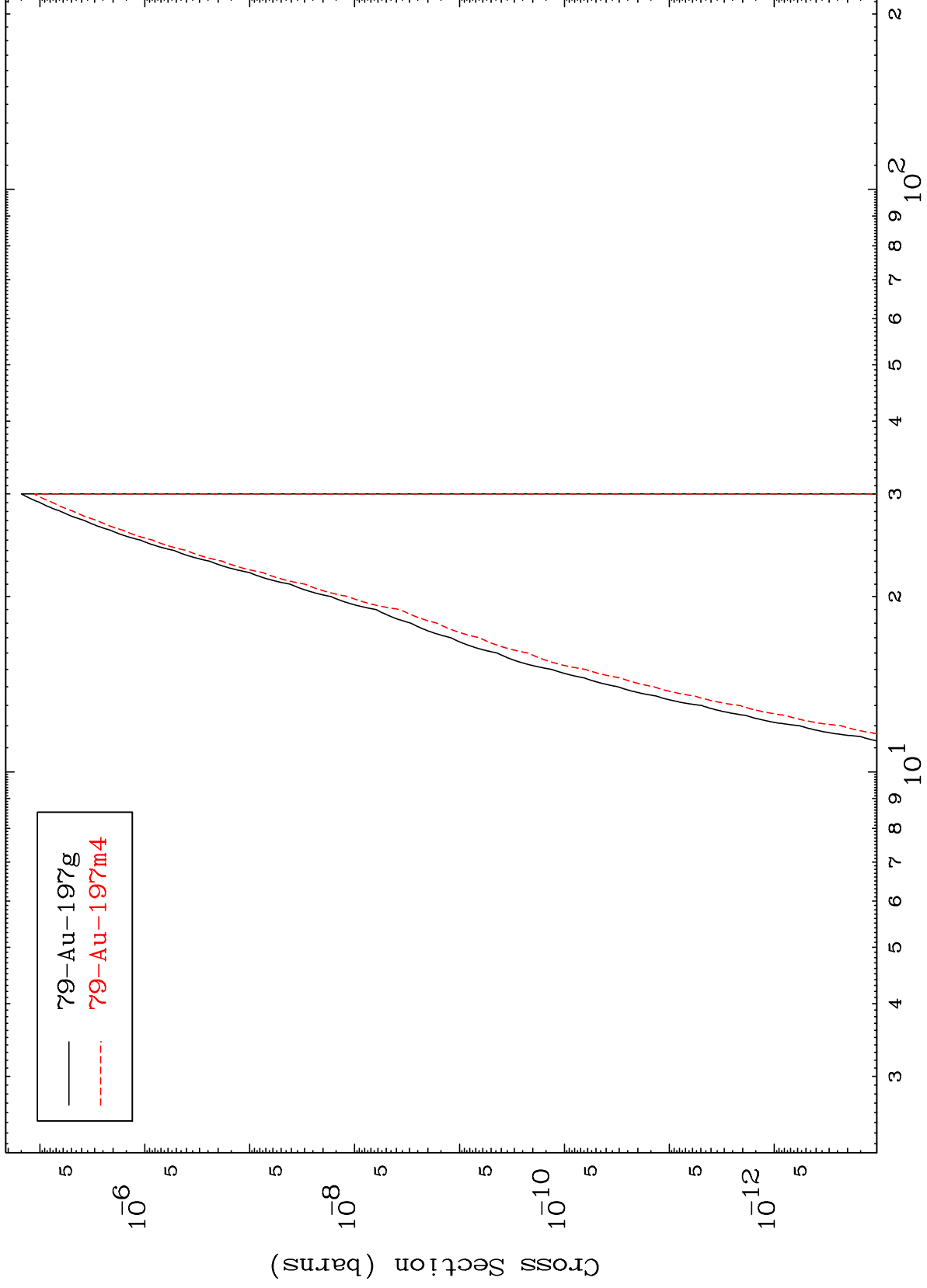
Radionuclide Production Cross Section



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81-Tl-198

(n,2p)  
Radionuclide Production Cross Section



39

Incident Energy (MeV)

81-Tl-198

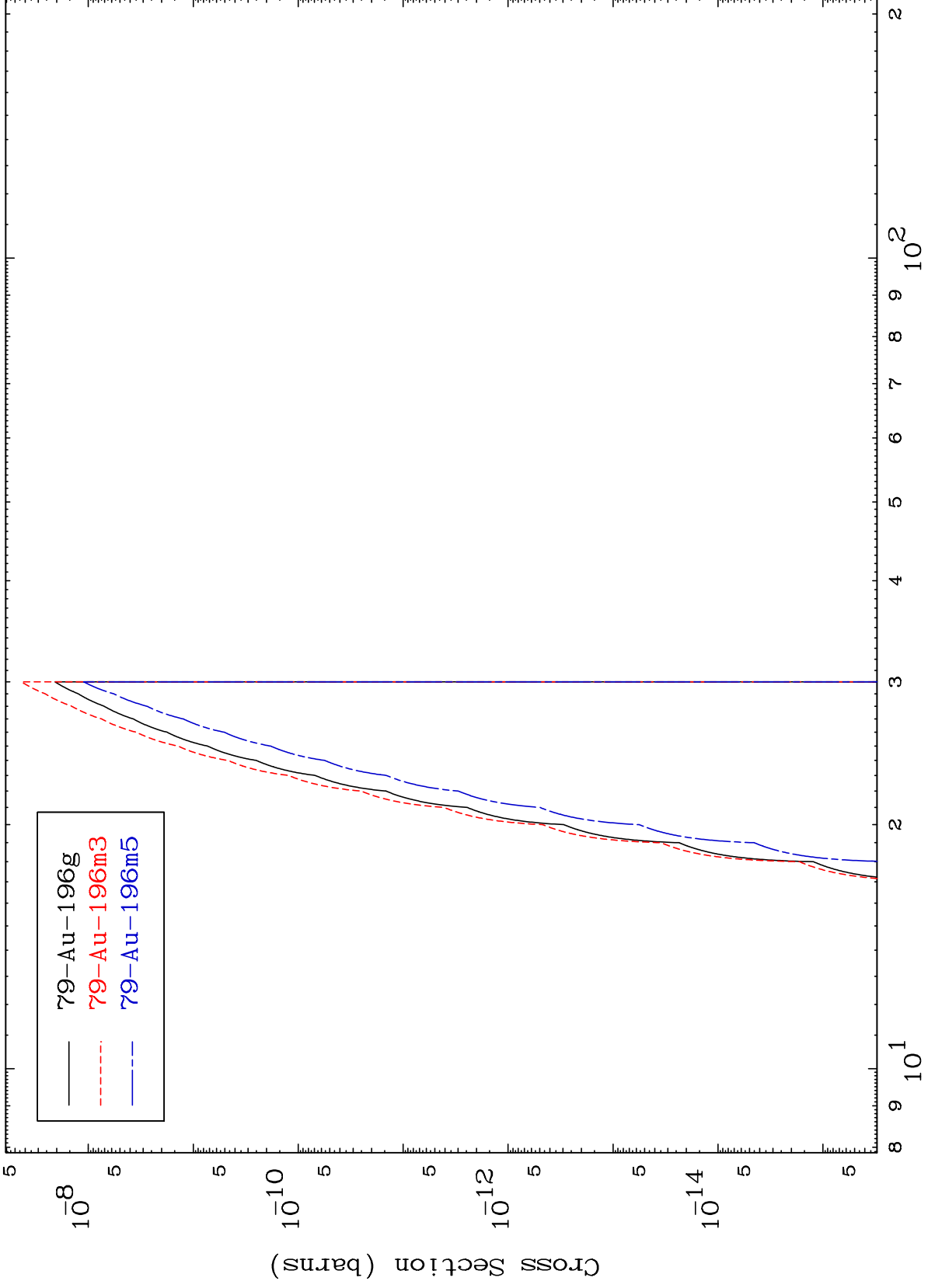


MAT 81111

(n,p) d

81-Tl-198

Radionuclide Production Cross Section



40

Incident Energy (MeV)

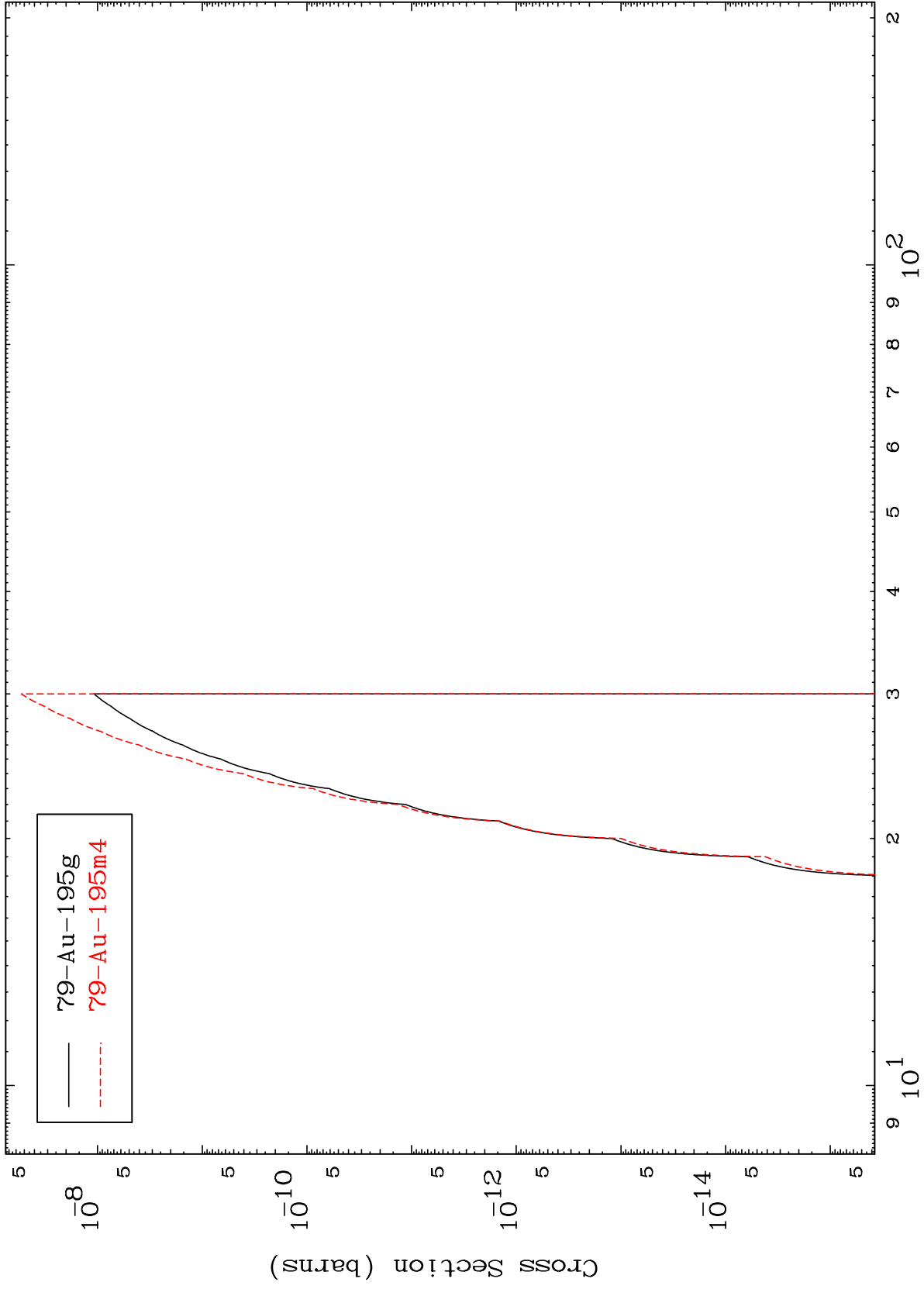
81-Tl-198

MAT 81111

(n,p) t

81-Tl-198

Radionuclide Production Cross Section



41

Incident Energy (MeV)

81-Tl-198

MAT 81111

(n,d)  $\alpha$

81-Tl-198

Radionuclide Production Cross Section

