

Program EVALPLOT
(Version 2017-1)

by

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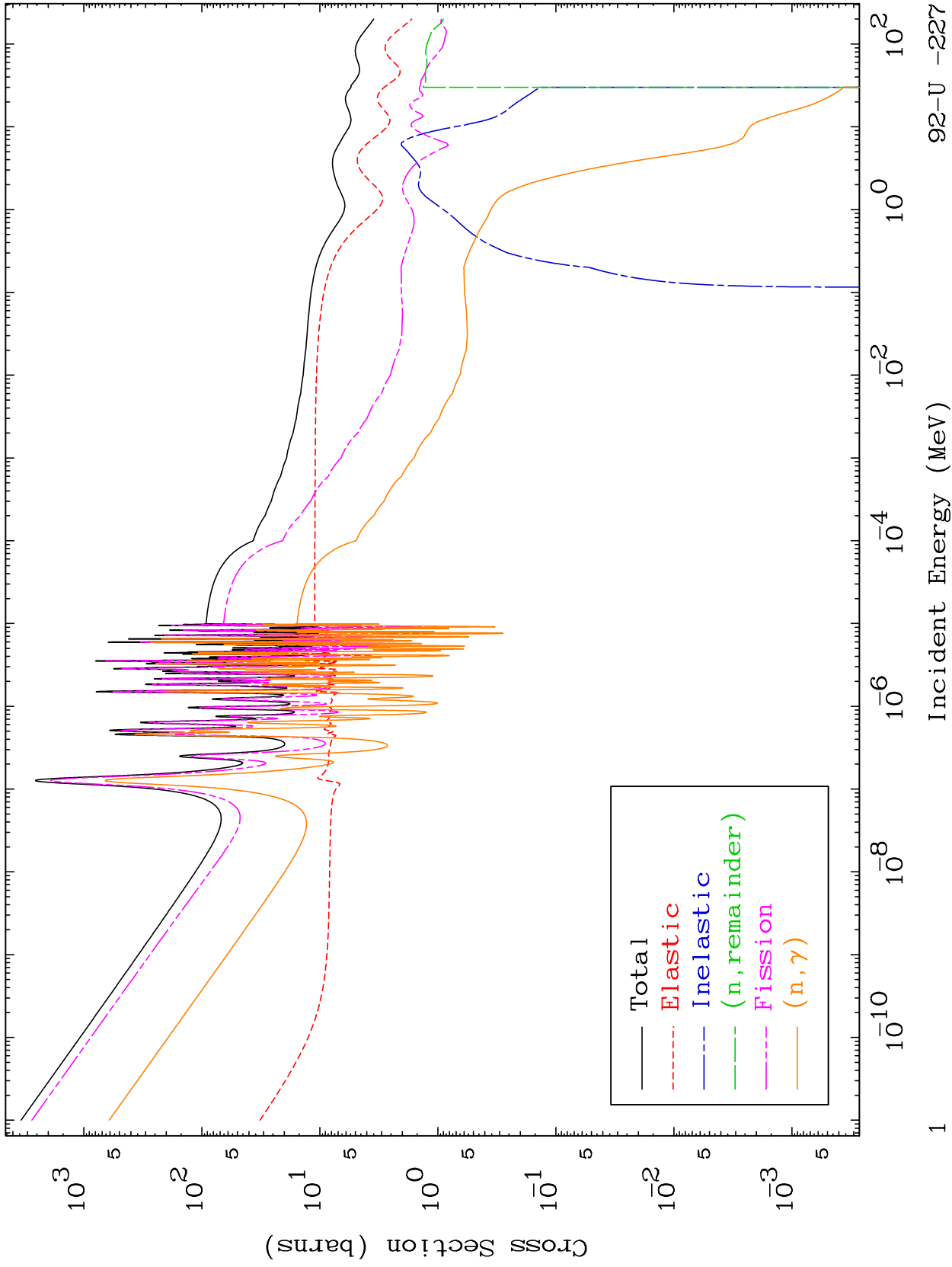
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9204

Major
293 Kelvin Cross Sections

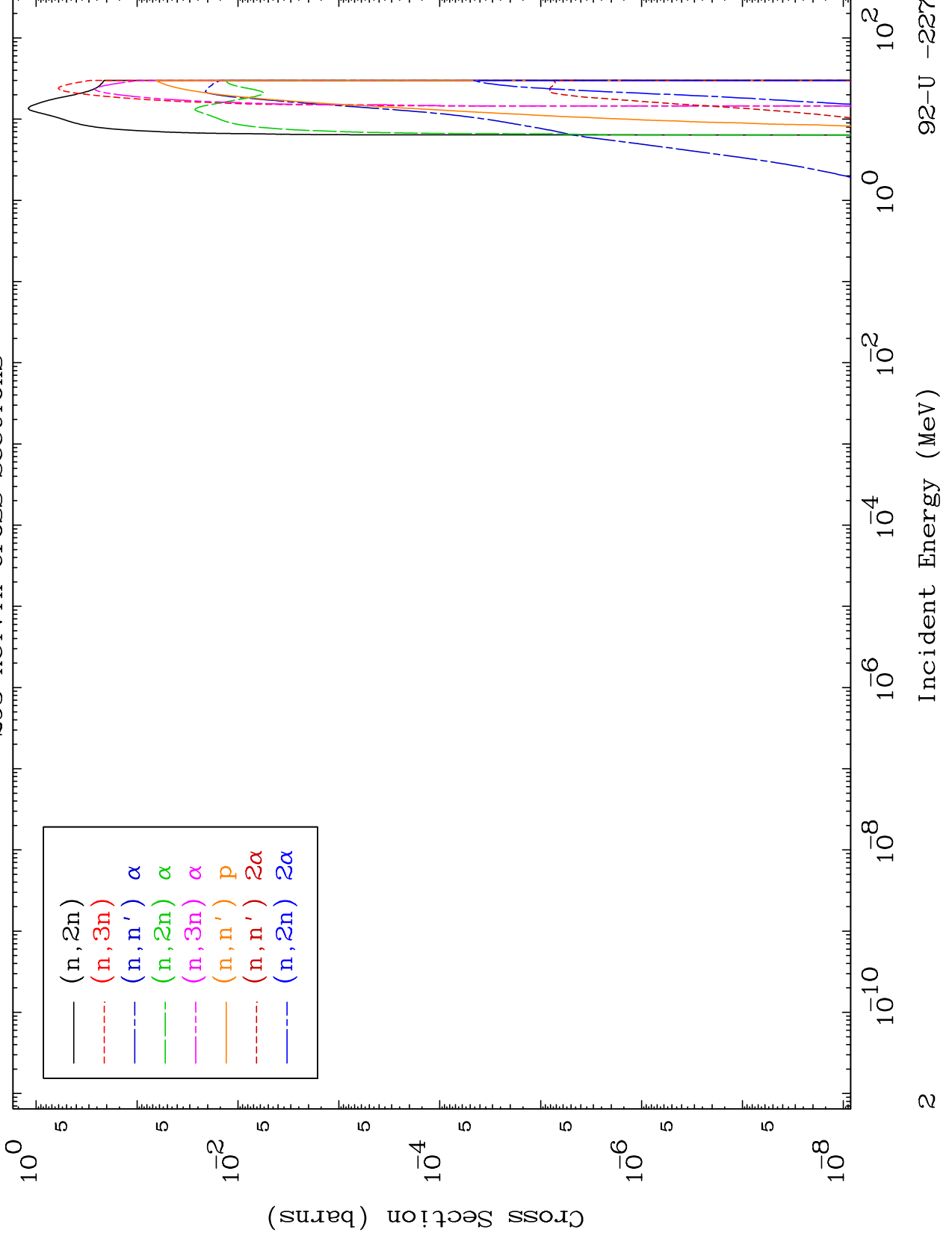
92-U -227



MAT 9204

Neutron Production
293 Kelvin Cross Sections

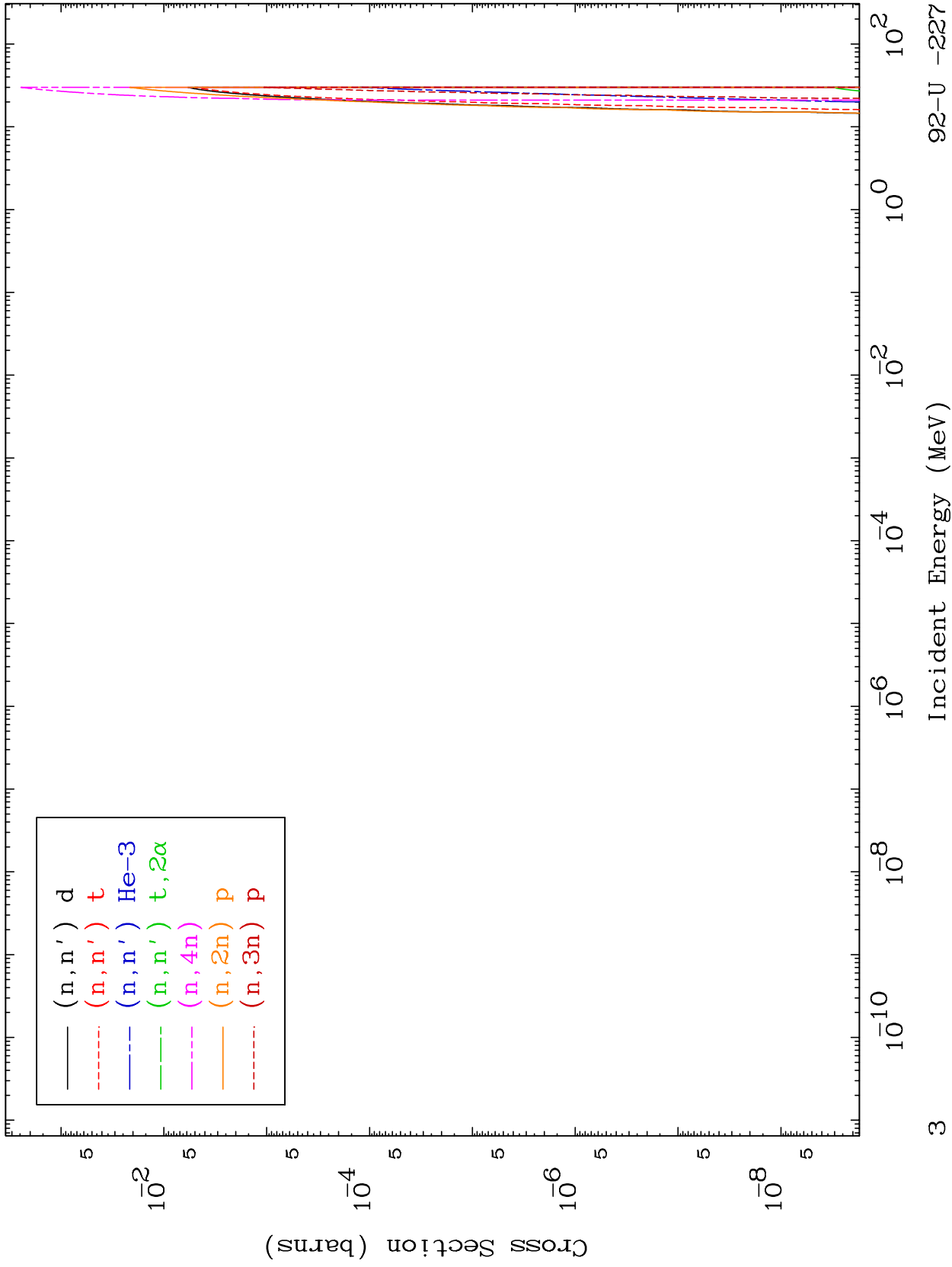
92-U -227



MAT 9204

Neutron Production
293 Kelvin Cross Sections

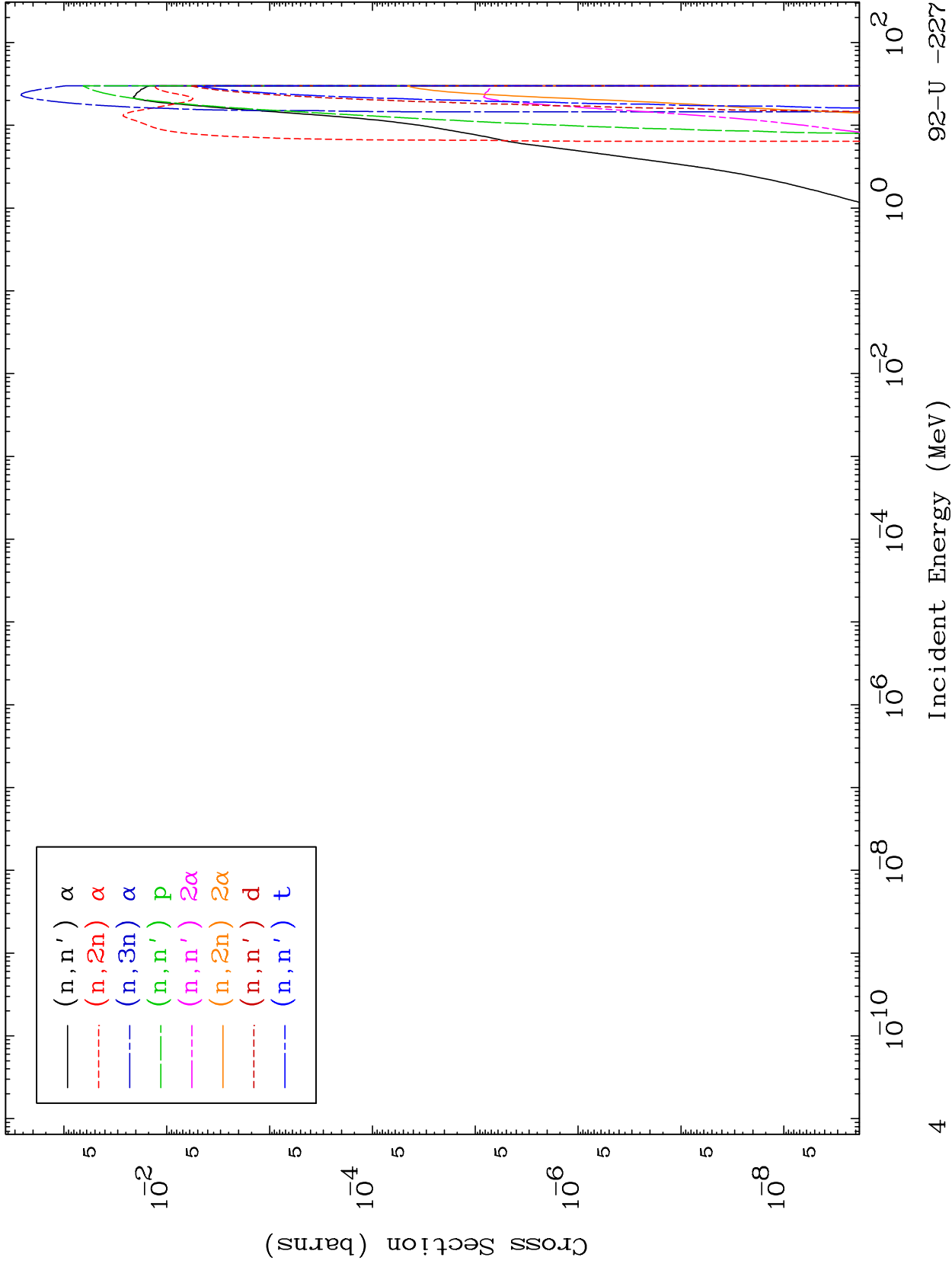
92-U -227



MAT 9204

Charged Particle
293 Kelvin Cross Sections

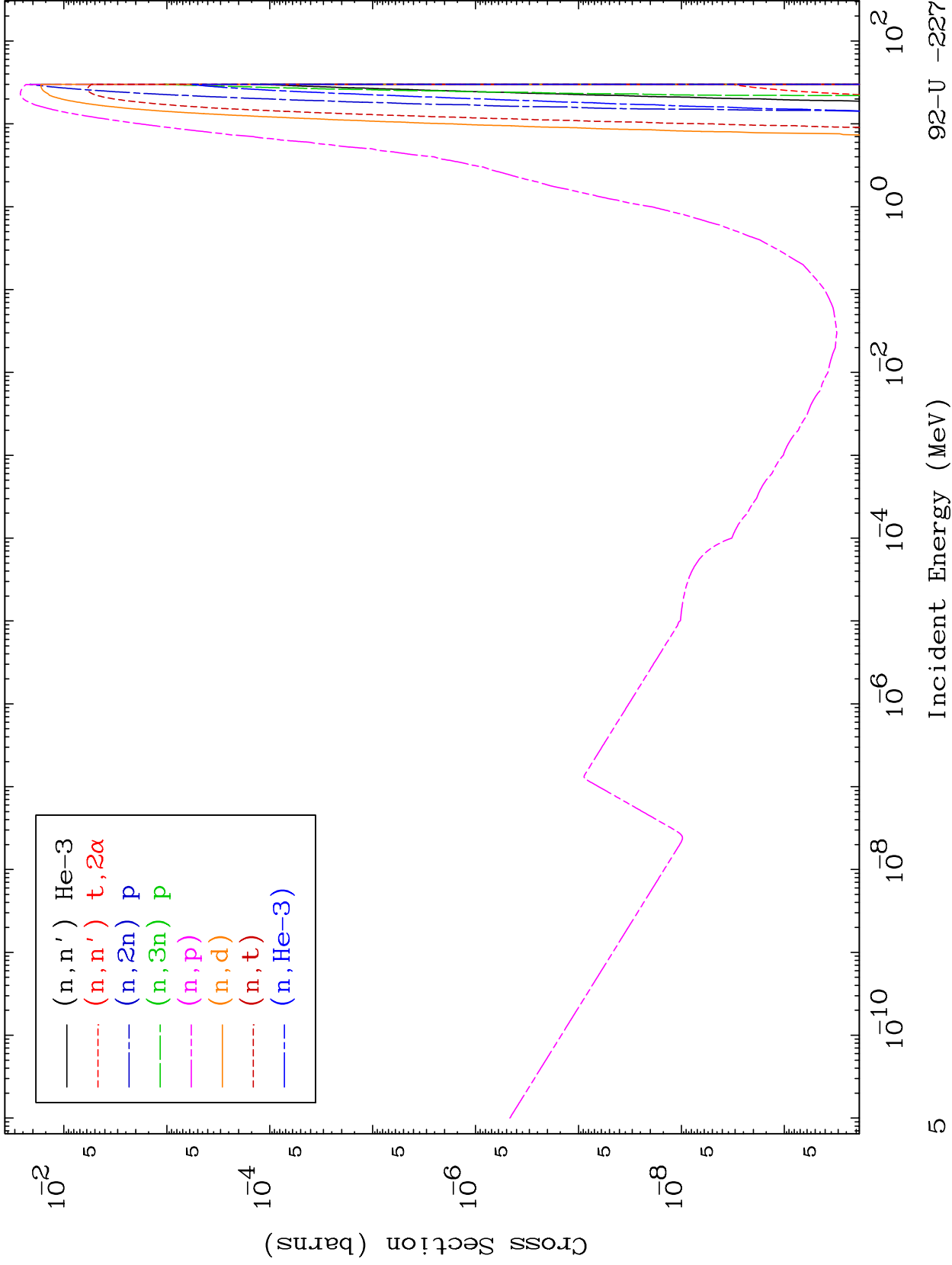
92-U -227



MAT 9204

Charged Particle
293 Kelvin Cross Sections

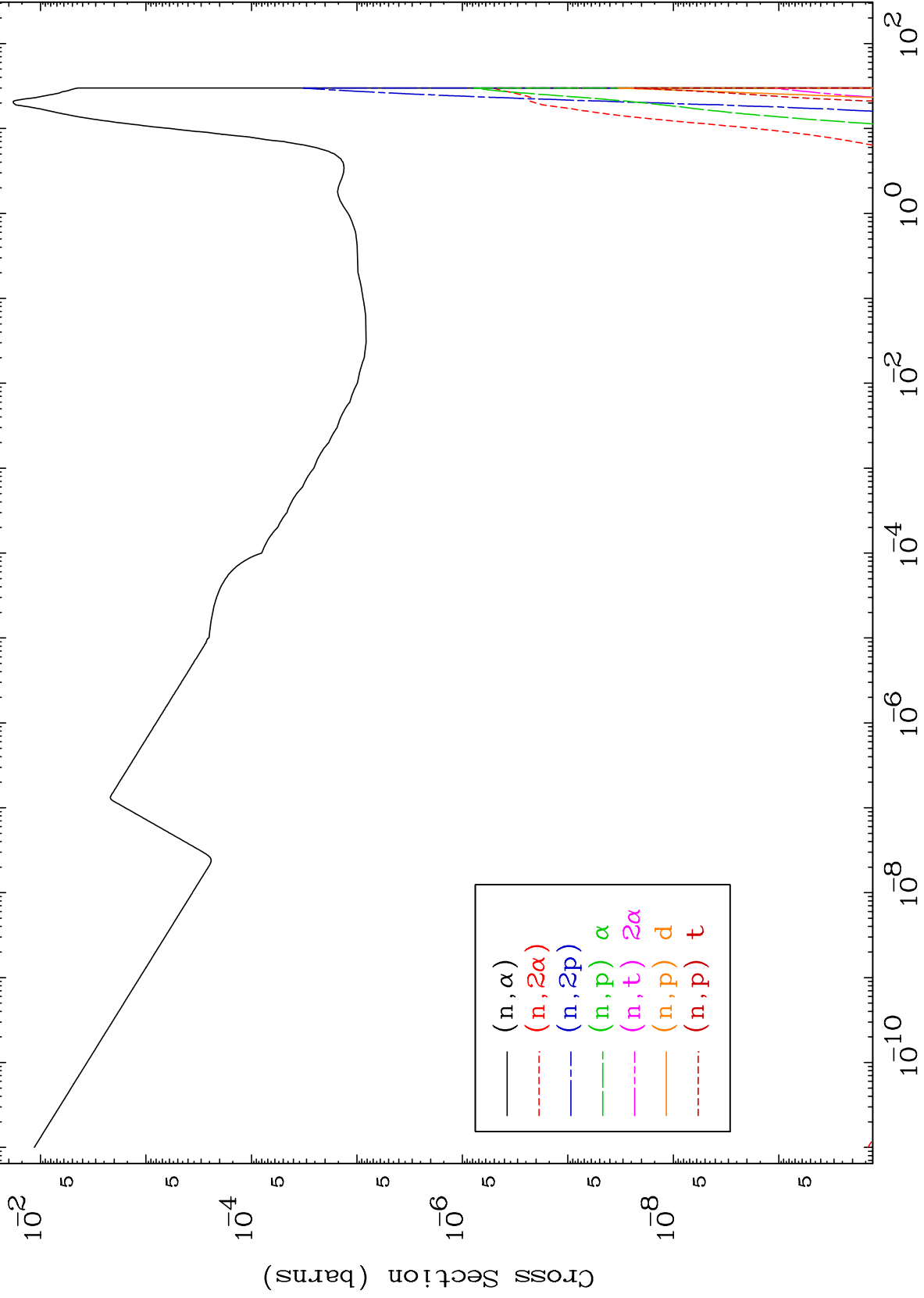
92-U -227



MAT 9204

Charged Particle
293 Kelvin Cross Sections

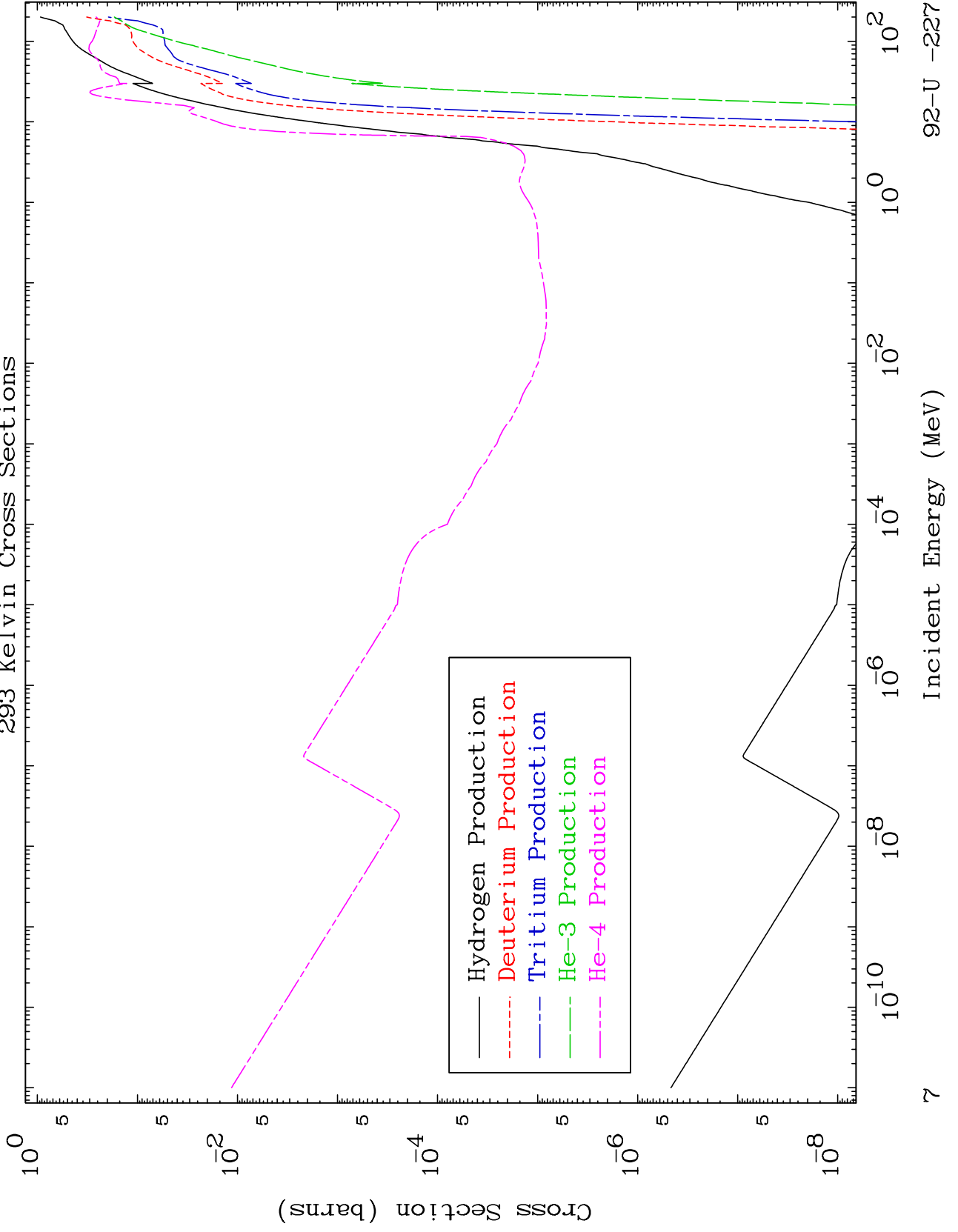
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Particle Production
293 Kelvin Cross Sections

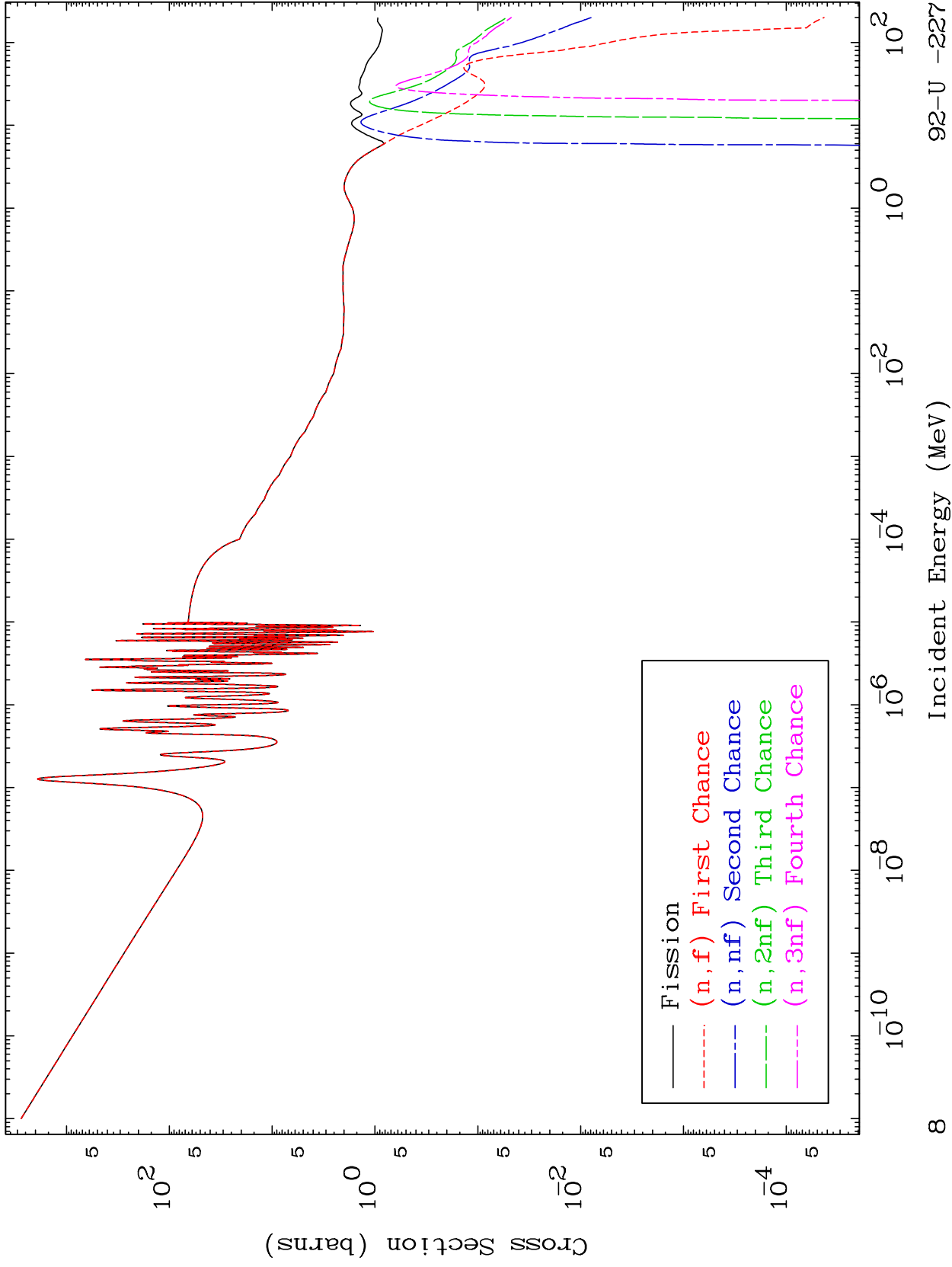
92-U -227



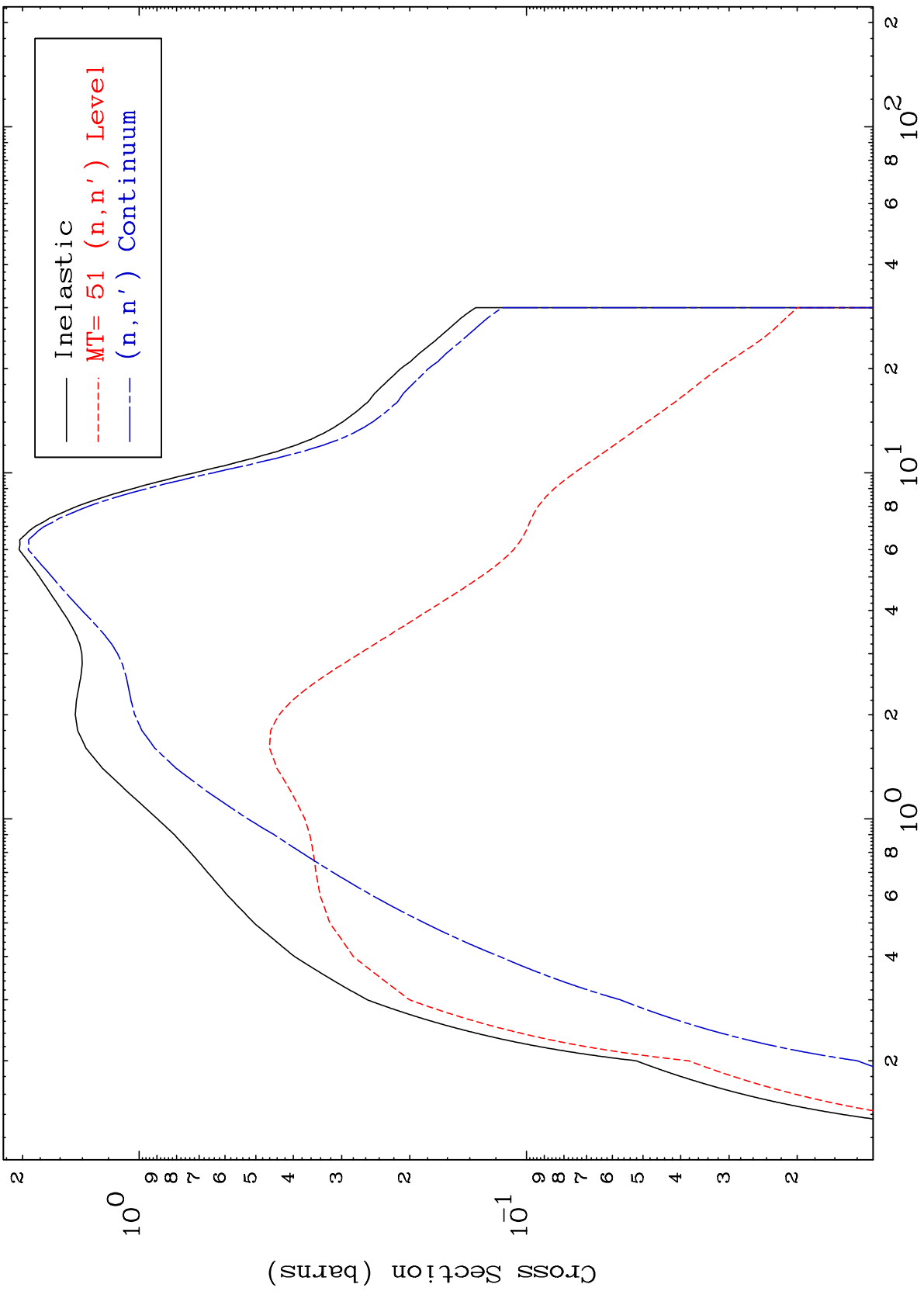
MAT 9204

Fission
293 Kelvin Cross Sections

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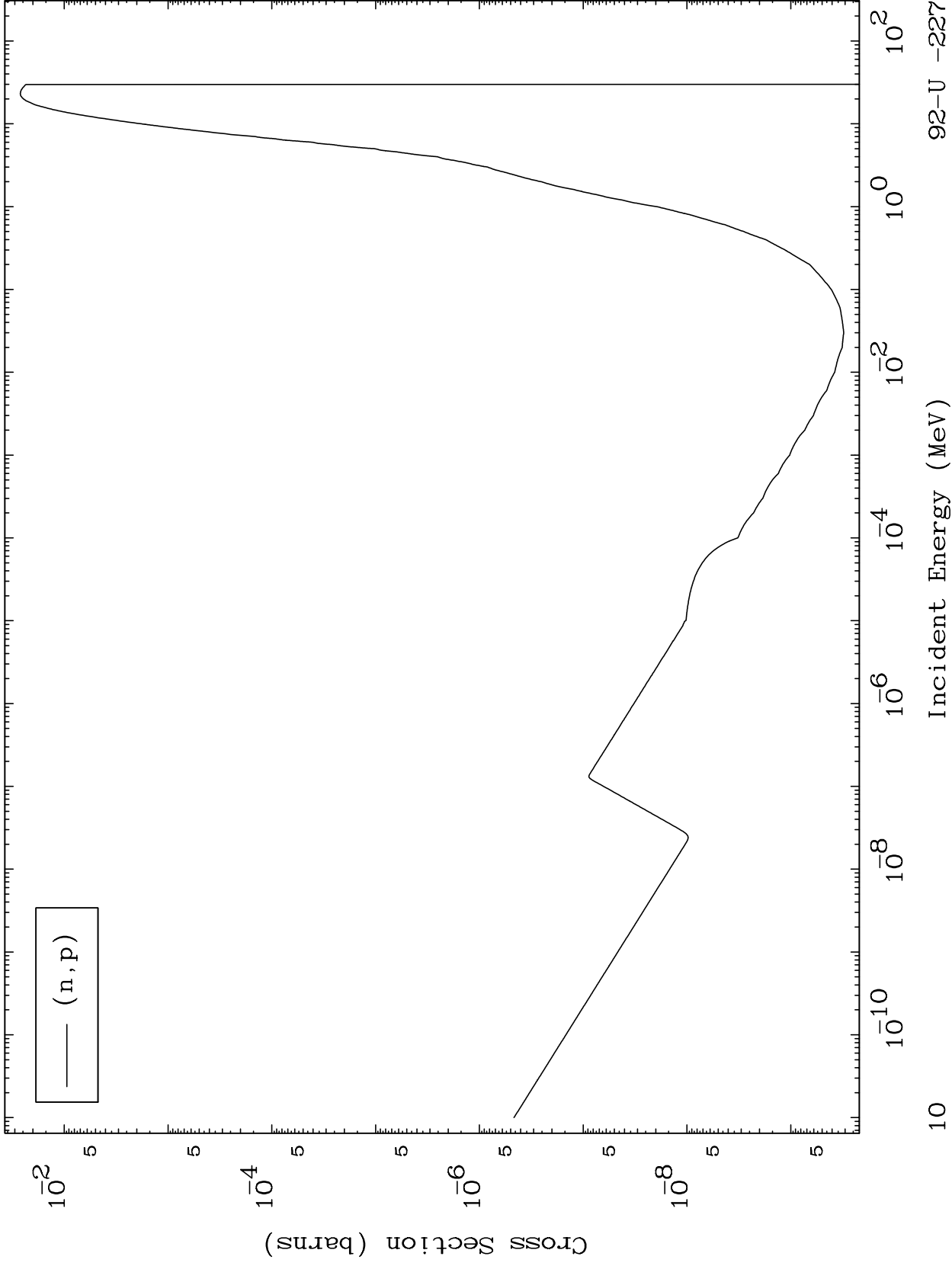
(n,n') Level
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

92-U -227



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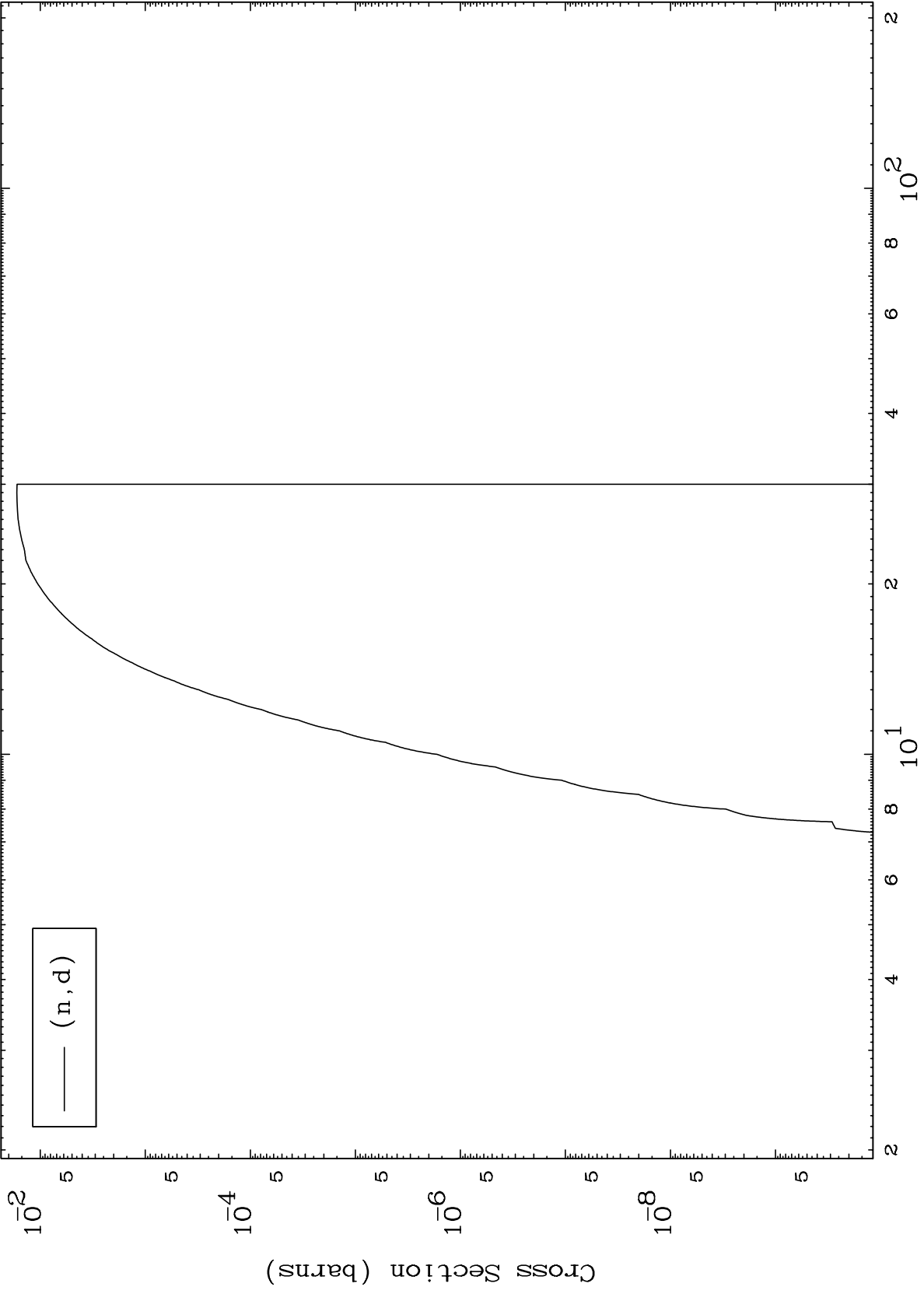
Incident Energy (MeV)

92-U -227

MAT 9204

(n,d) Levels
293 Kelvin Cross Sections

92-U -227



11

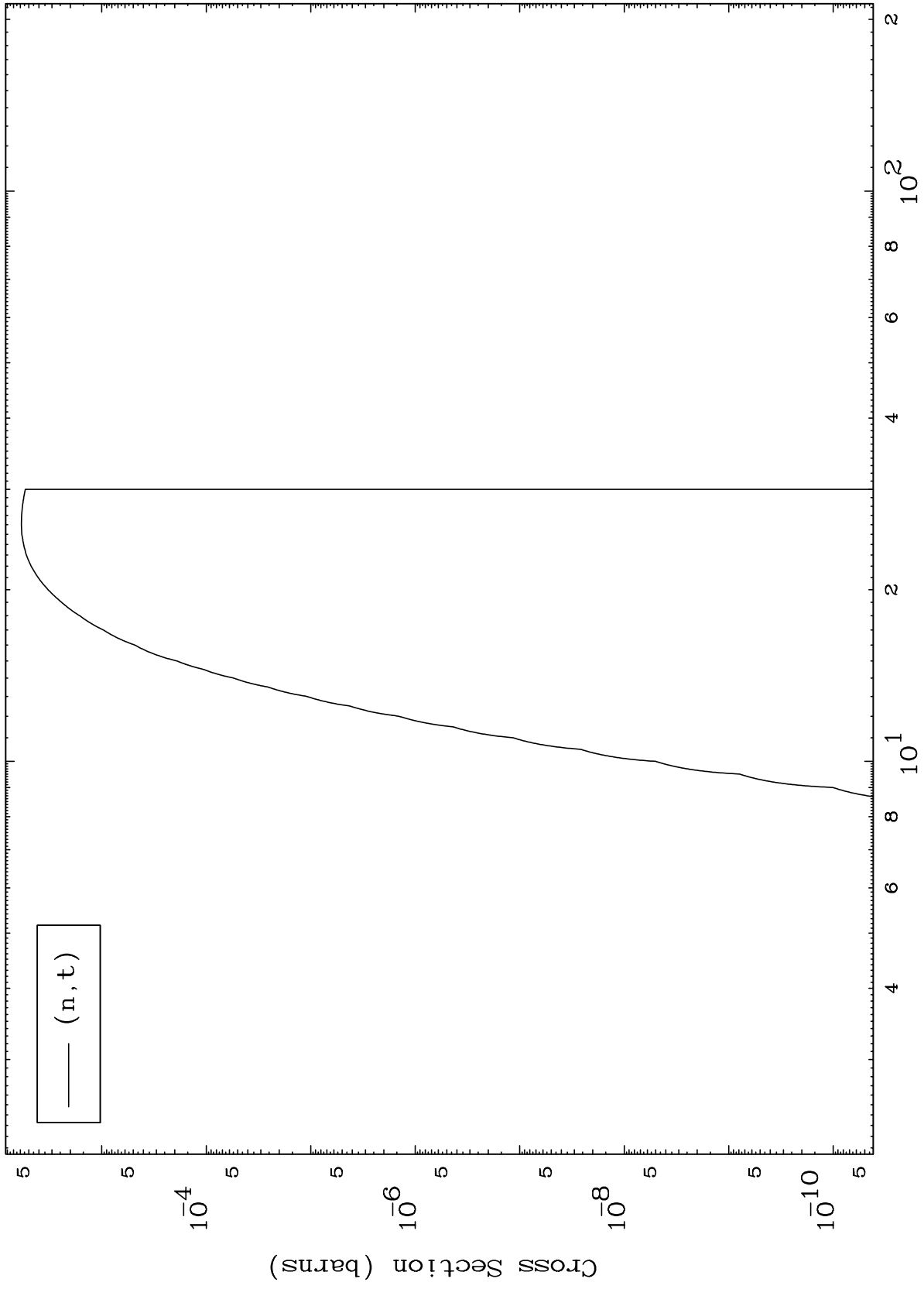
Incident Energy (MeV)

92-U -227

MAT 9204

(n,t) Levels
293 Kelvin Cross Sections

92-U -227



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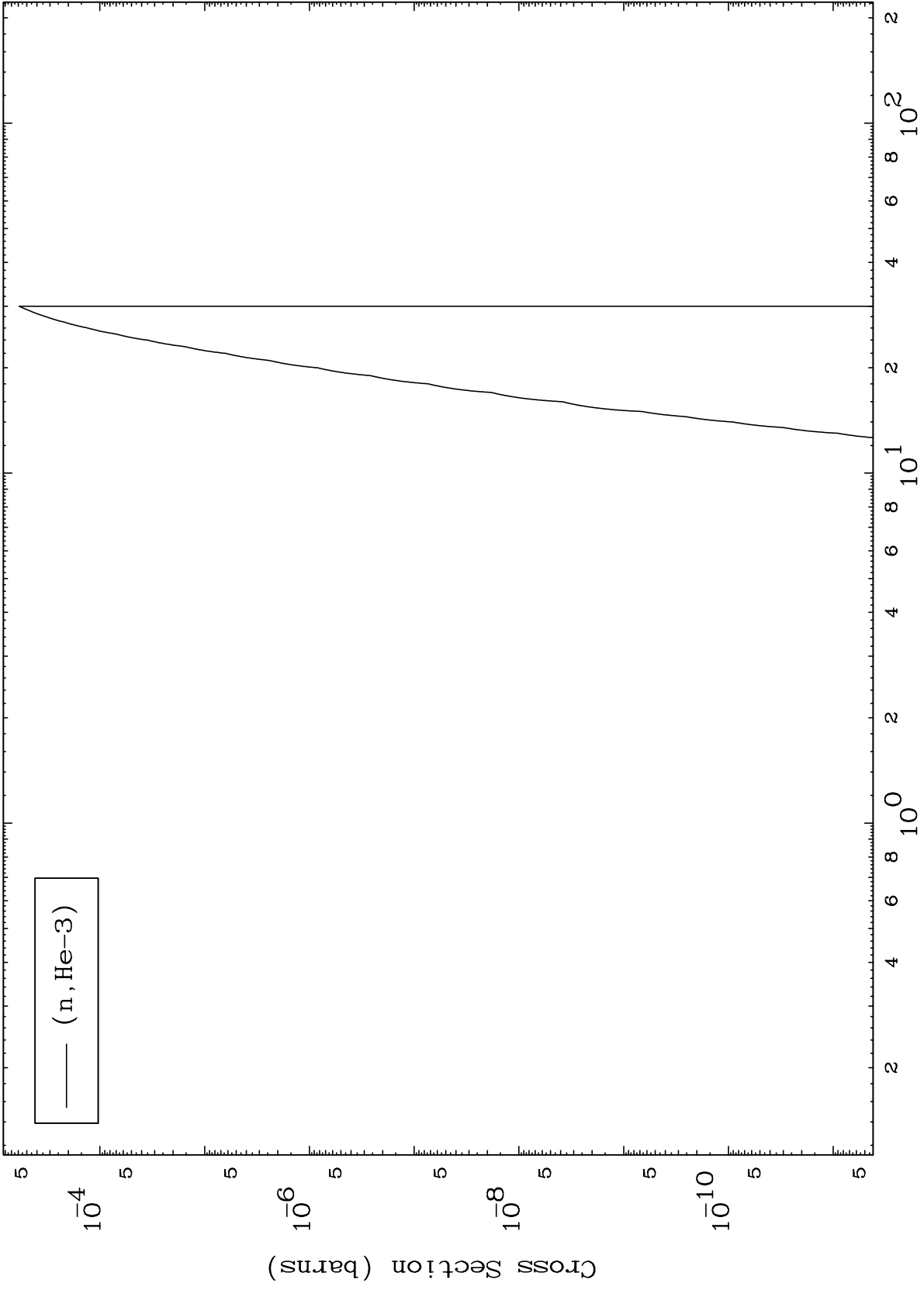
Incident Energy (MeV)

92-U -227

MAT 9204

(n,He3) Levels
293 Kelvin Cross Sections

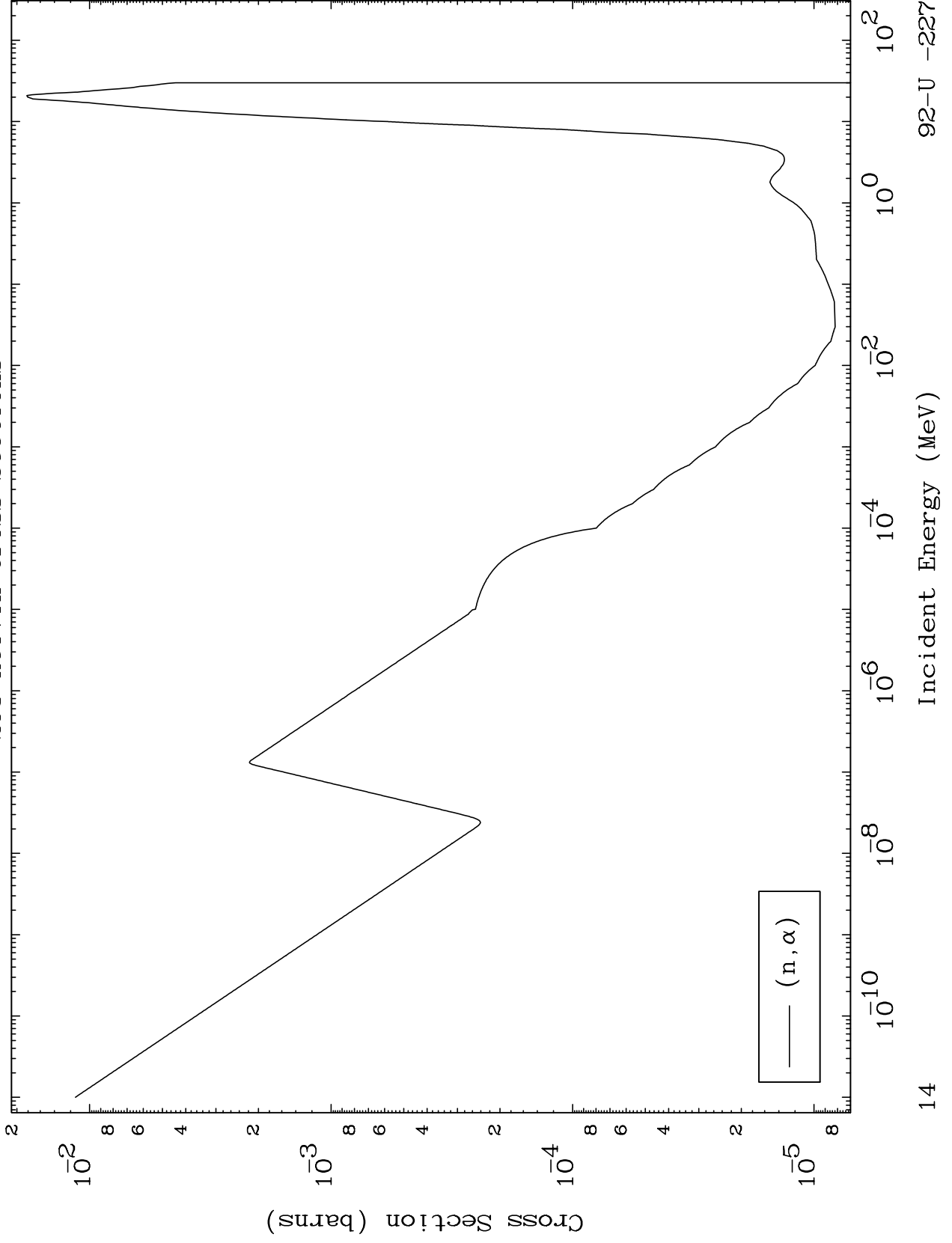
92-U -227

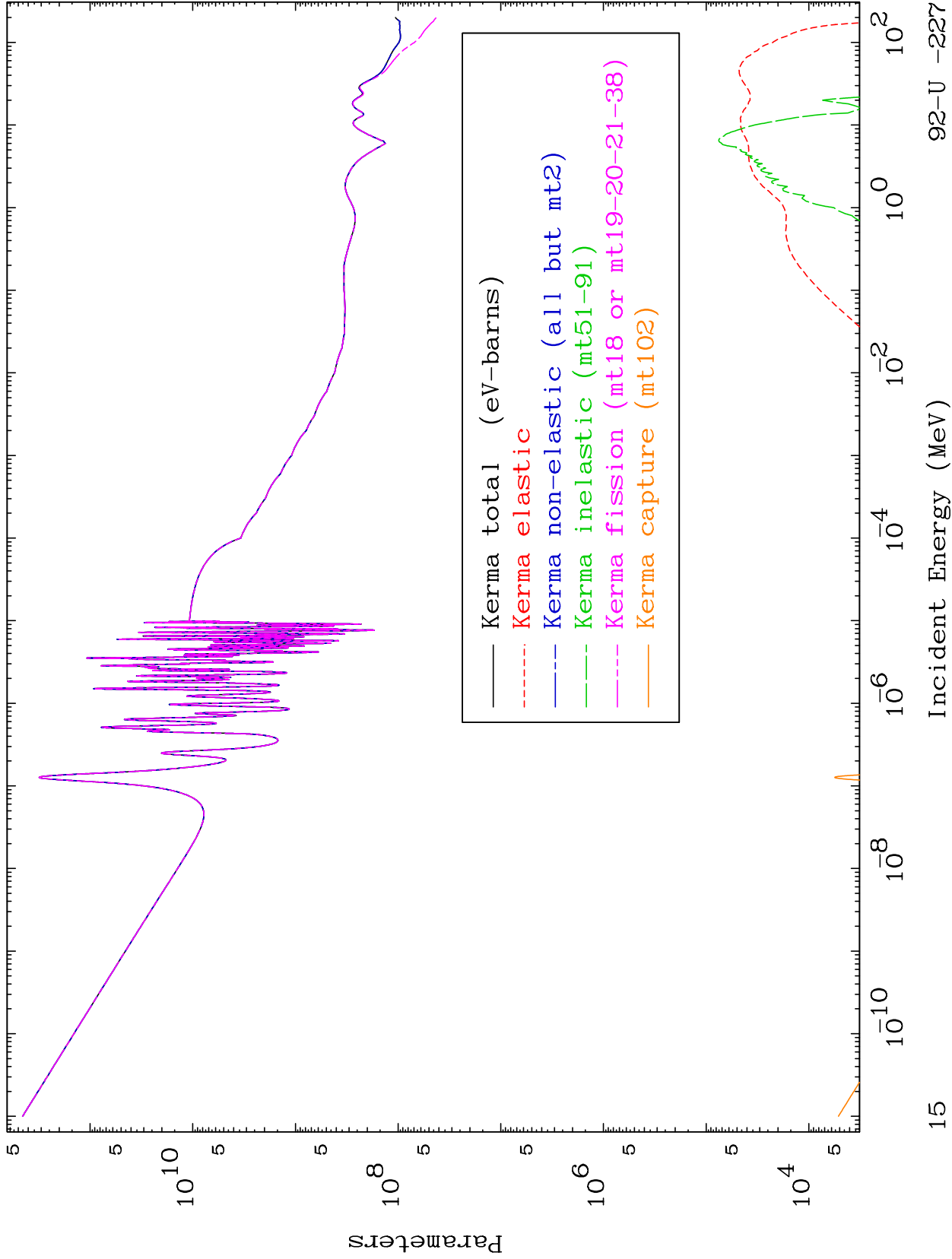


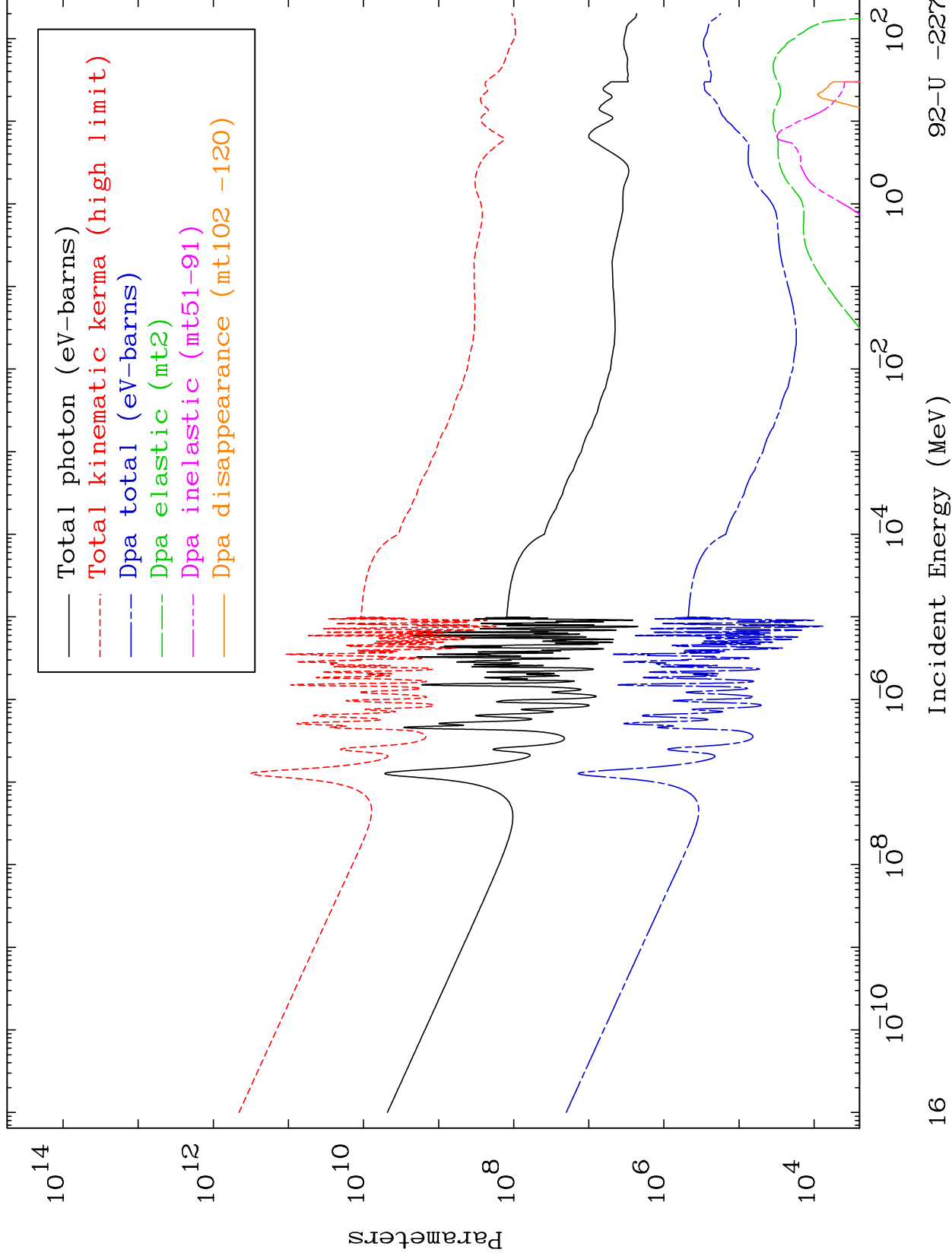
MAT 9204

(n, α) Levels
293 Kelvin Cross Sections

92-U -227



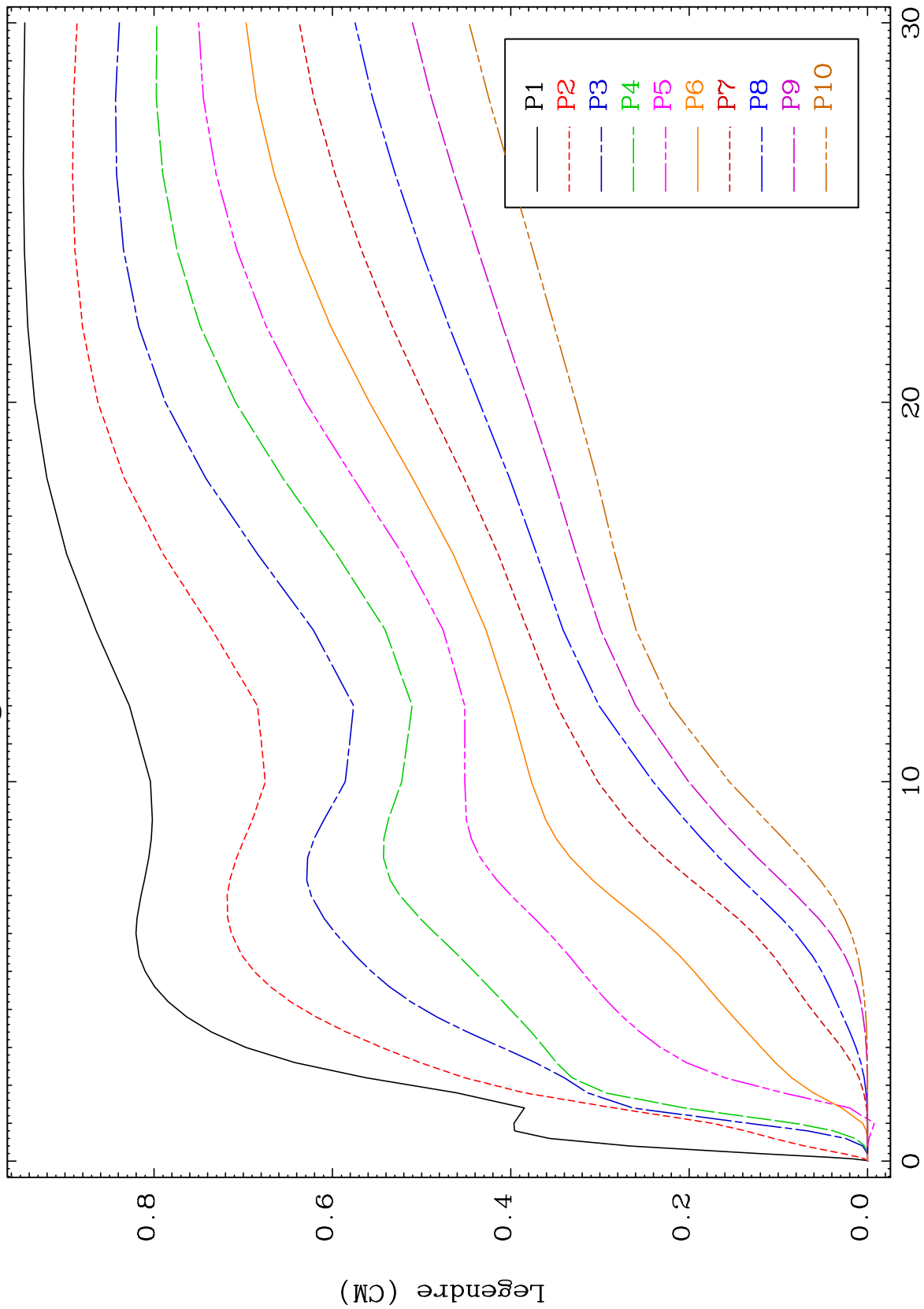




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Elastic Legendre Coefficients

92-U -227



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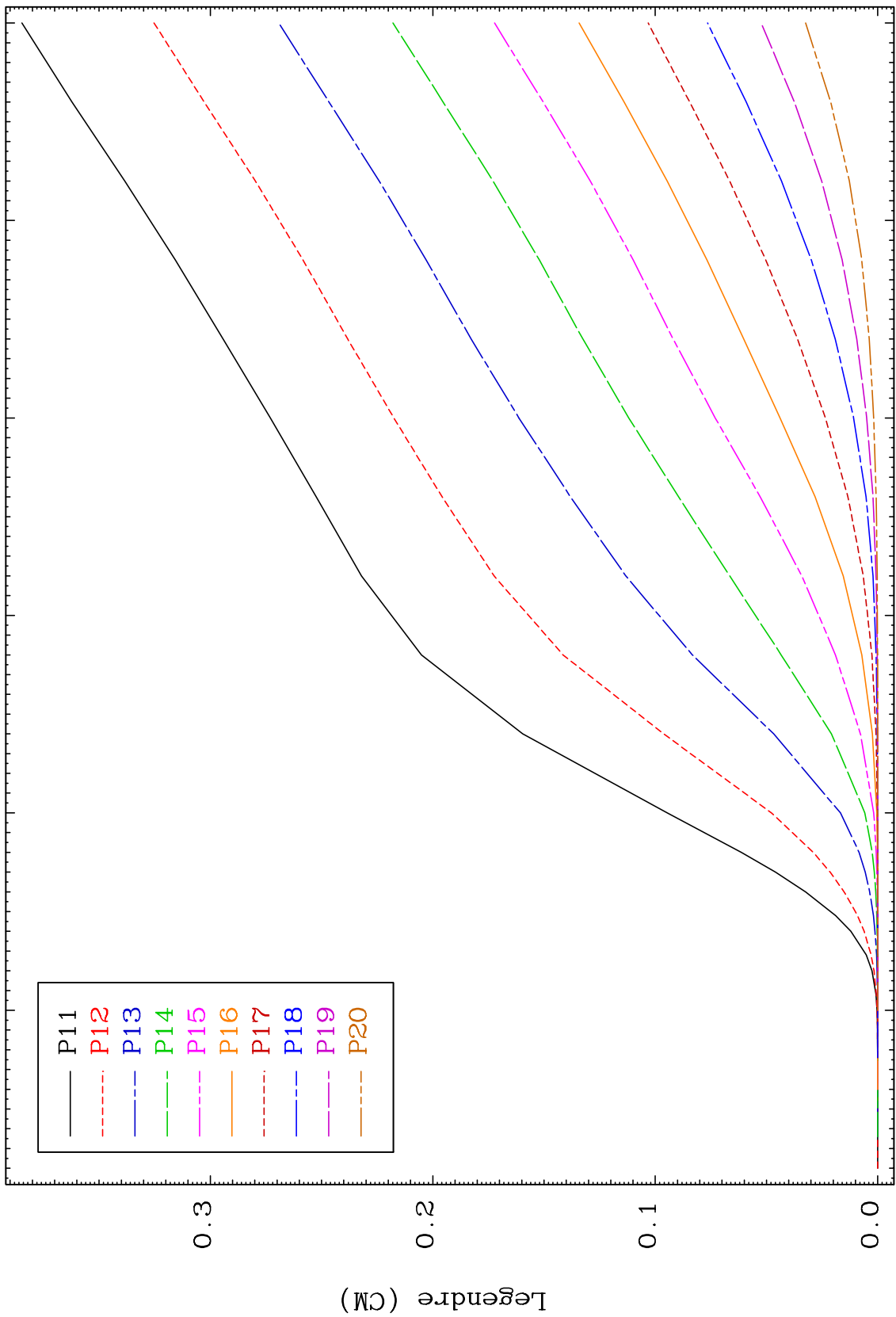
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



18

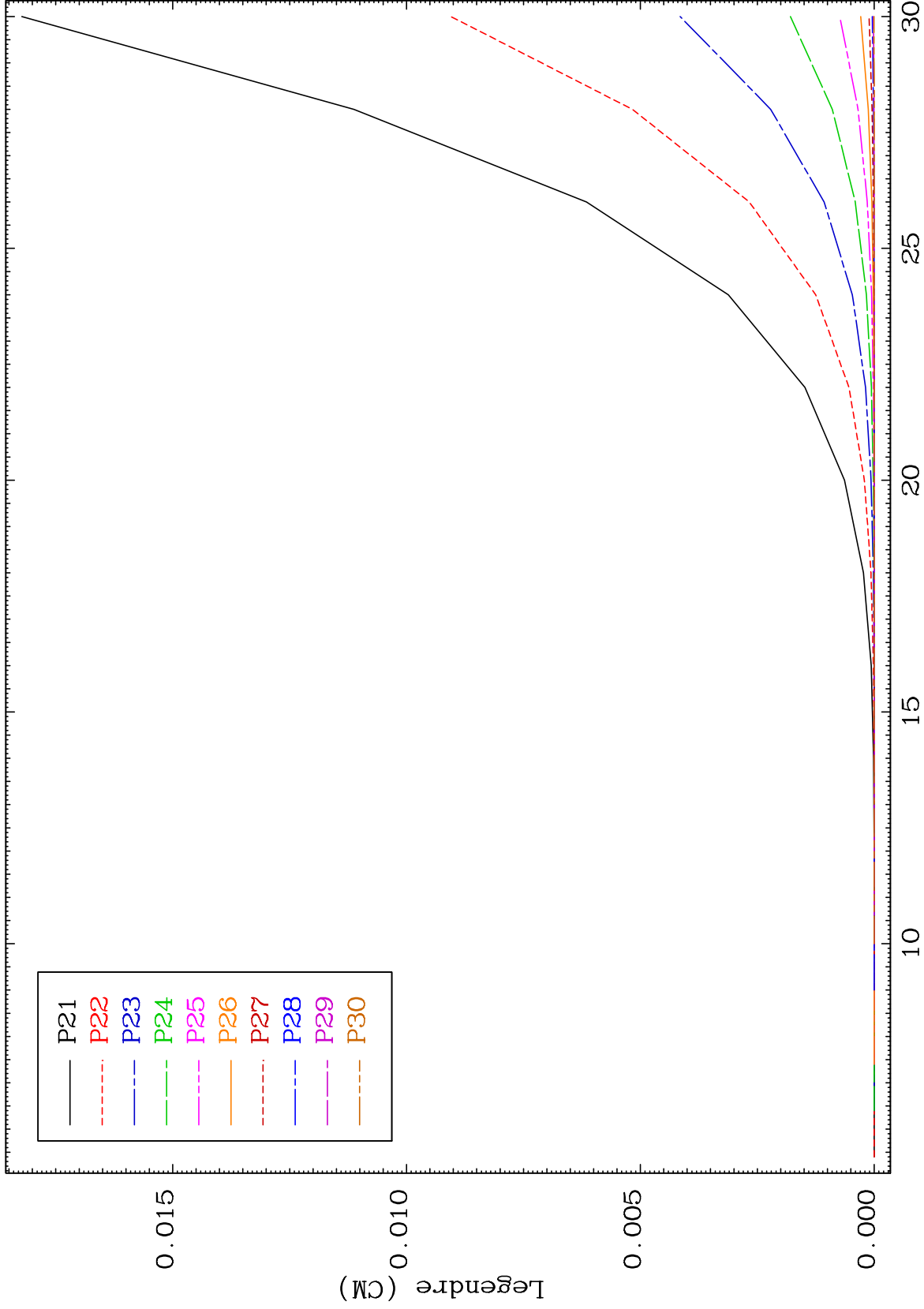
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



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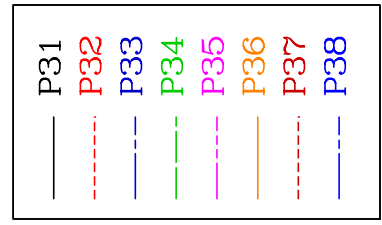
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



$\times 10^{-6}$
1.2
1.0
0.8

Legendre (CM)

0.6

0.4

0.2

0.0

15

20

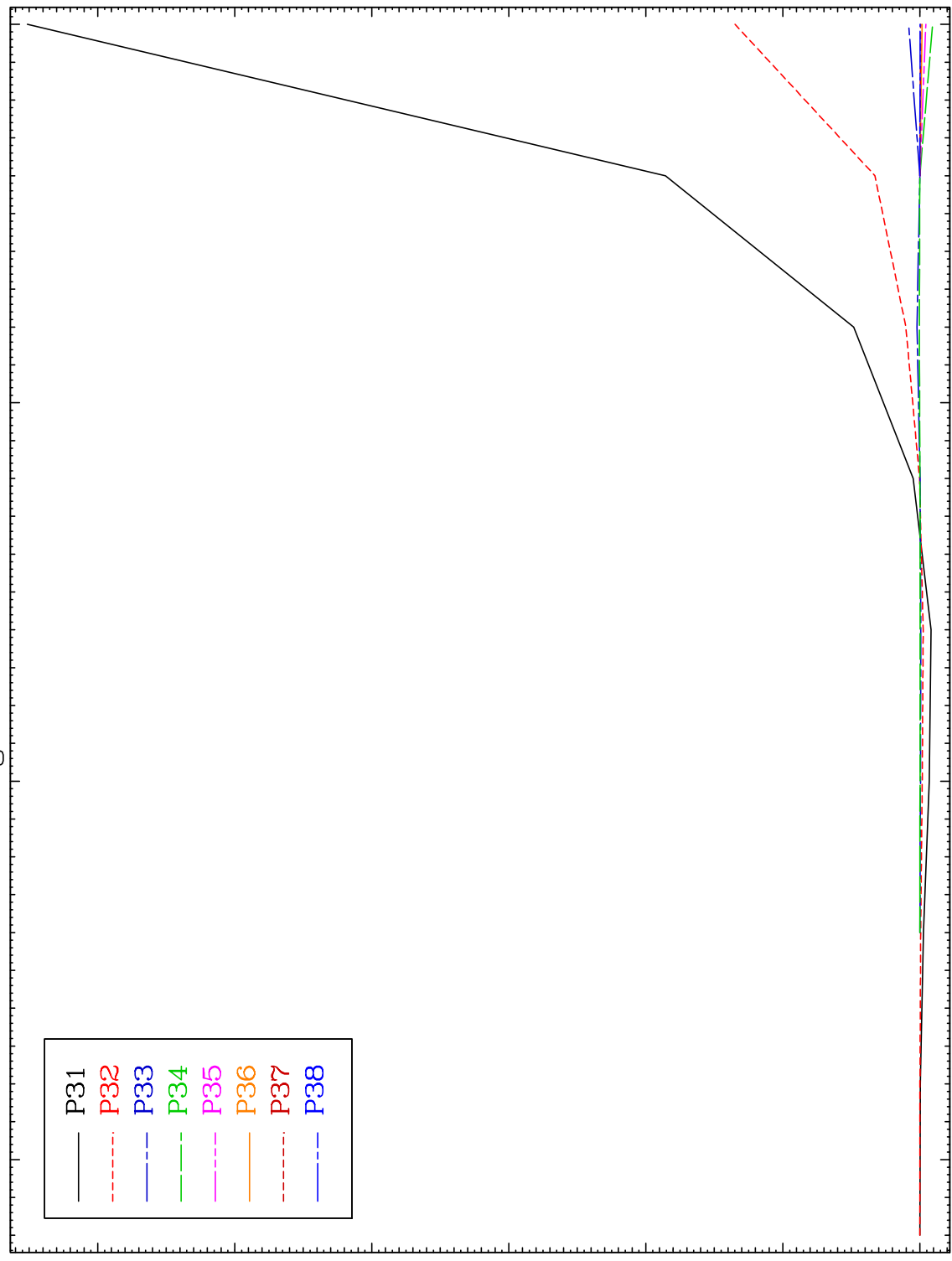
25

30

20

Incident Energy (MeV)

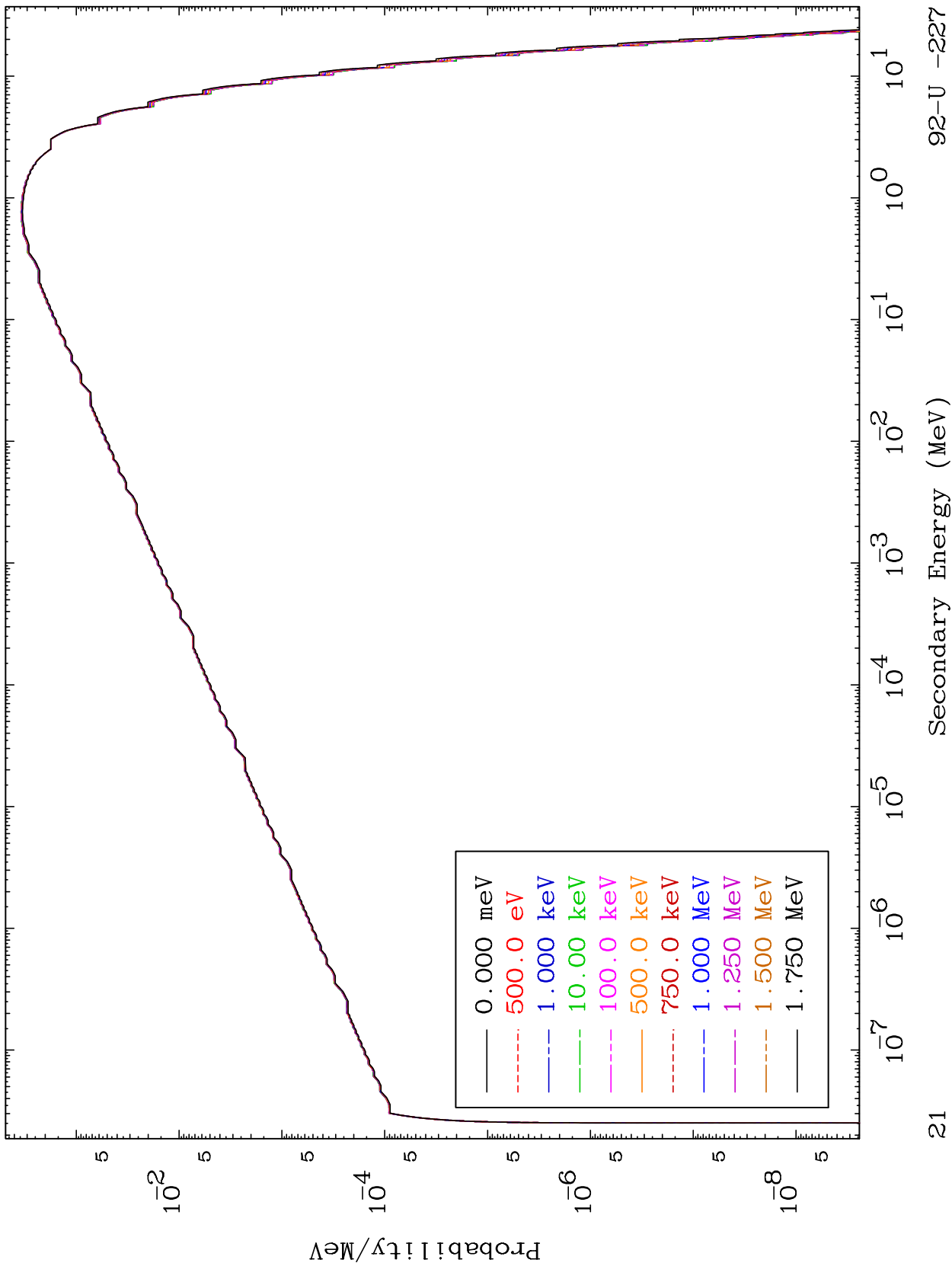
92-U -227



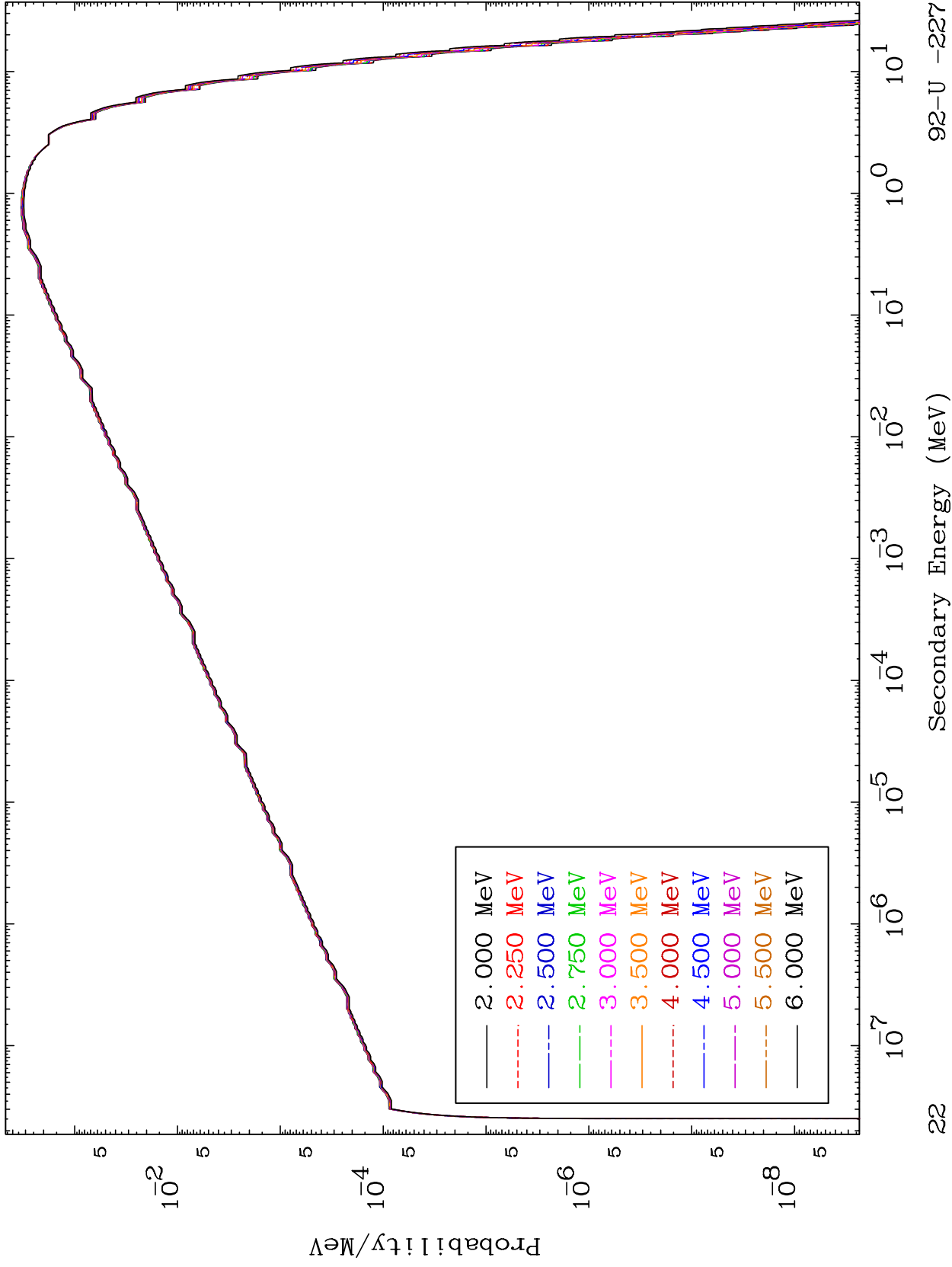
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Fission Energy Distribution

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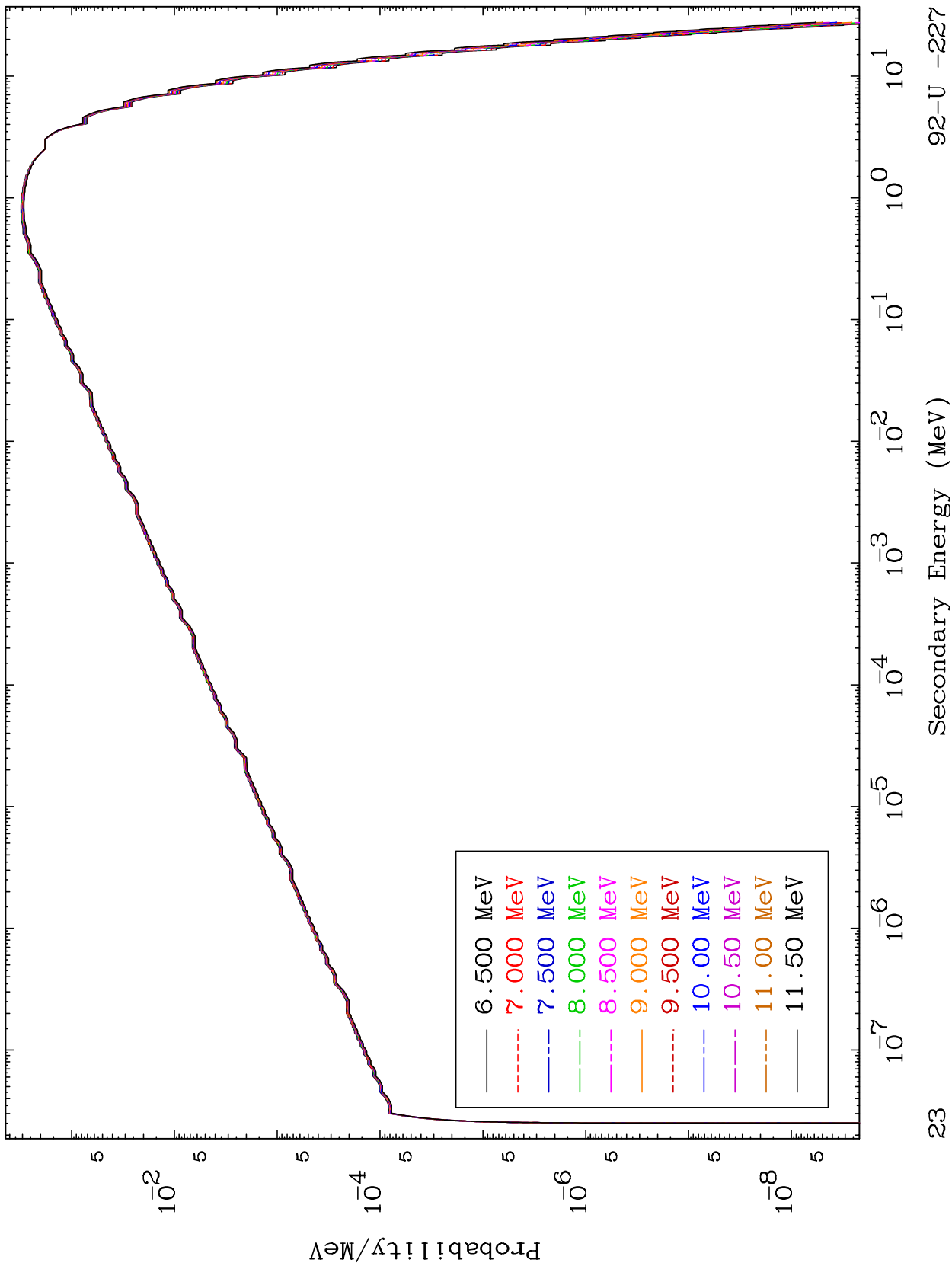
Fission Energy Distribution



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Fission Energy Distribution

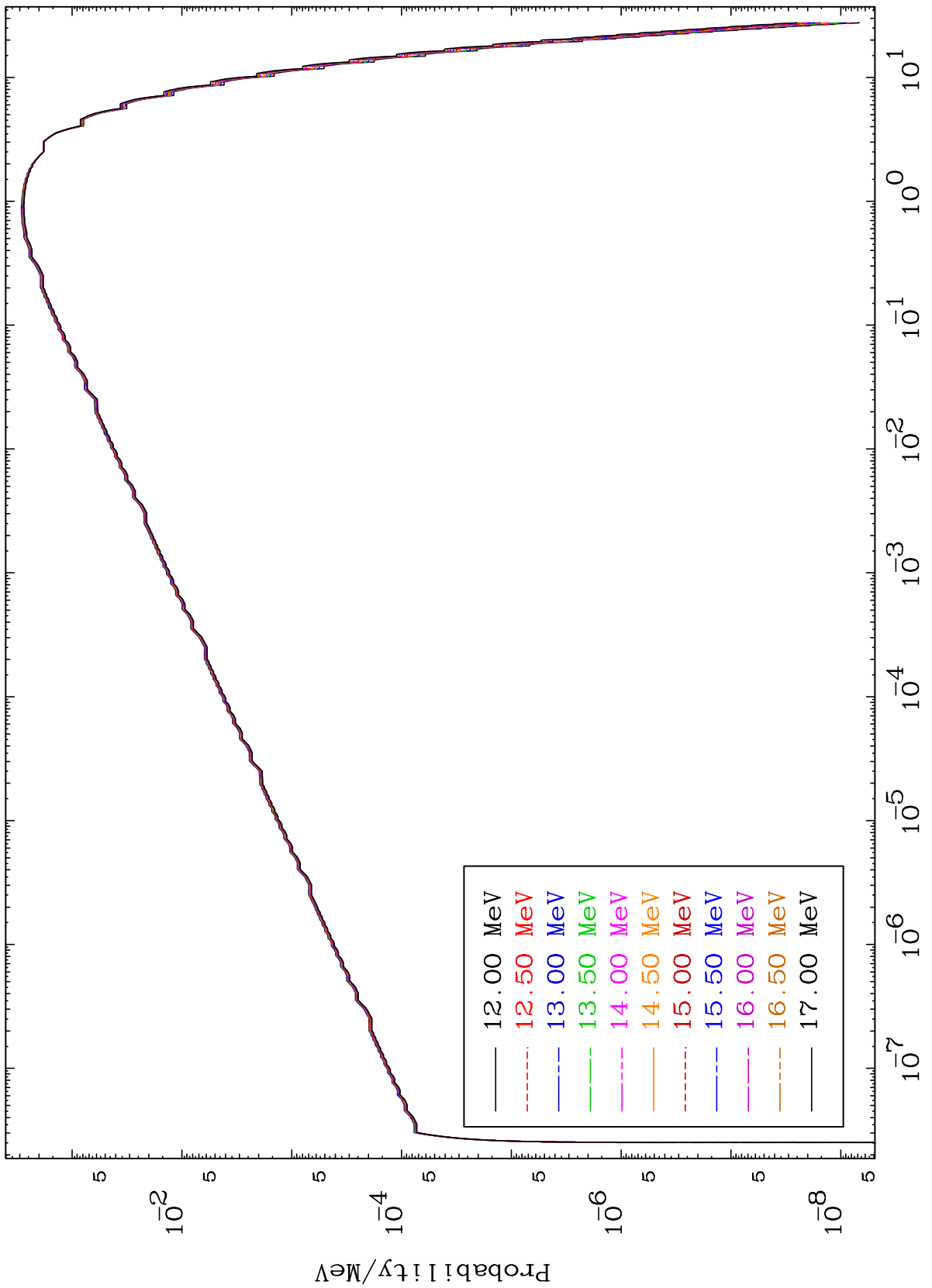
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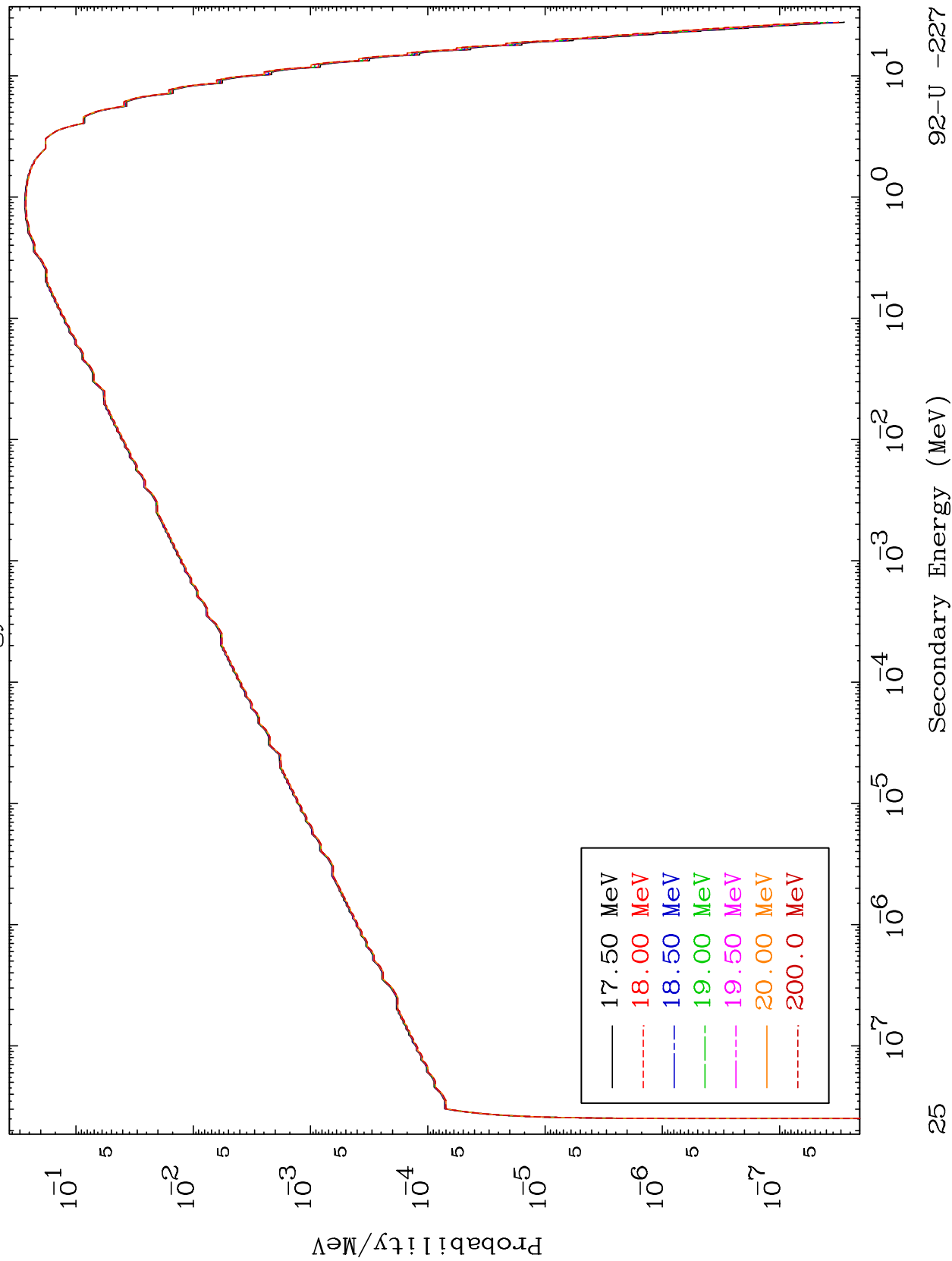
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Fission
Energy Distribution

92-U -227



Fission Energy Distribution

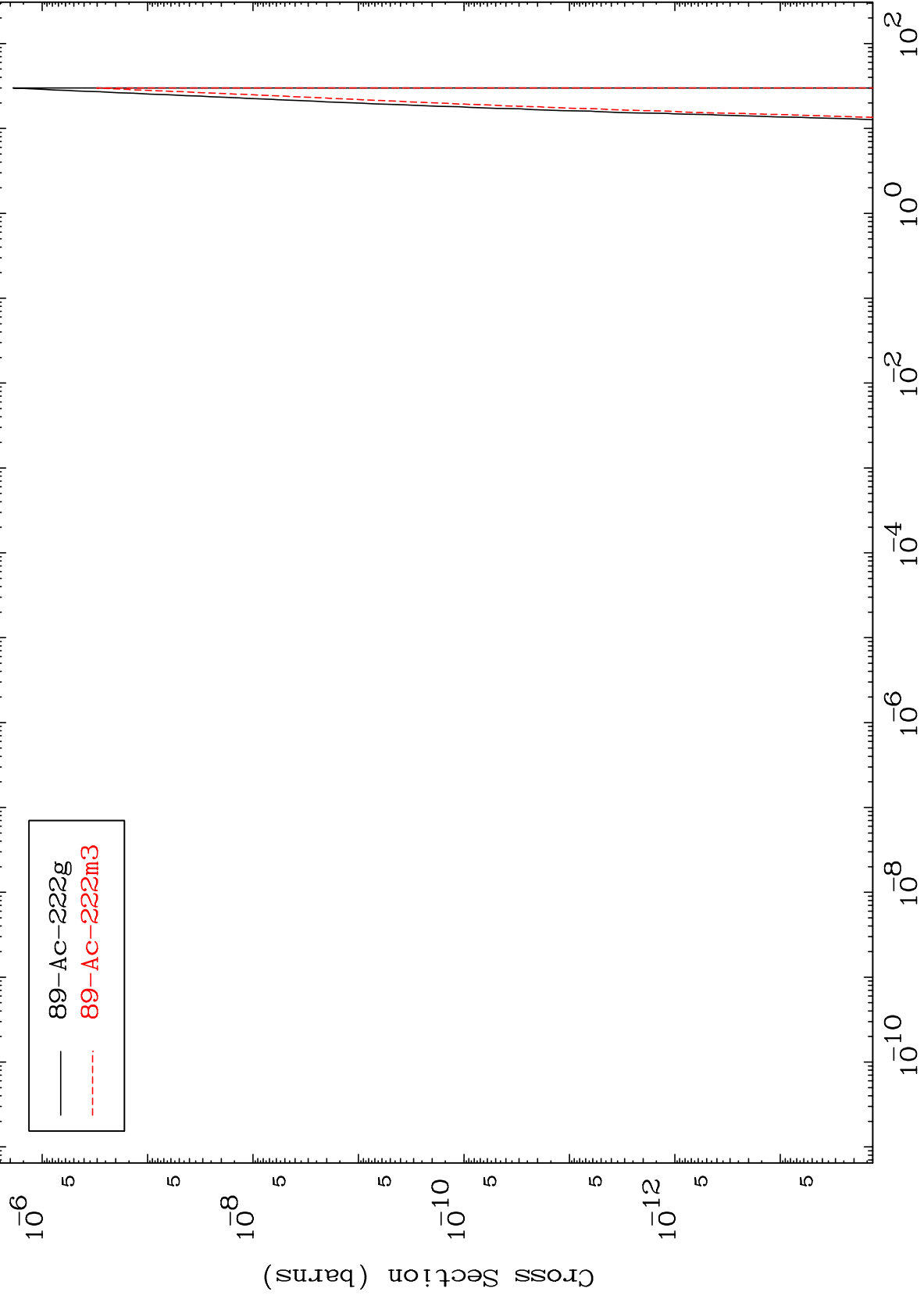


MAT 9204

(n,n') p α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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MAT 9204

(n,d) α

92-U -227

Radionuclide Production Cross Section

