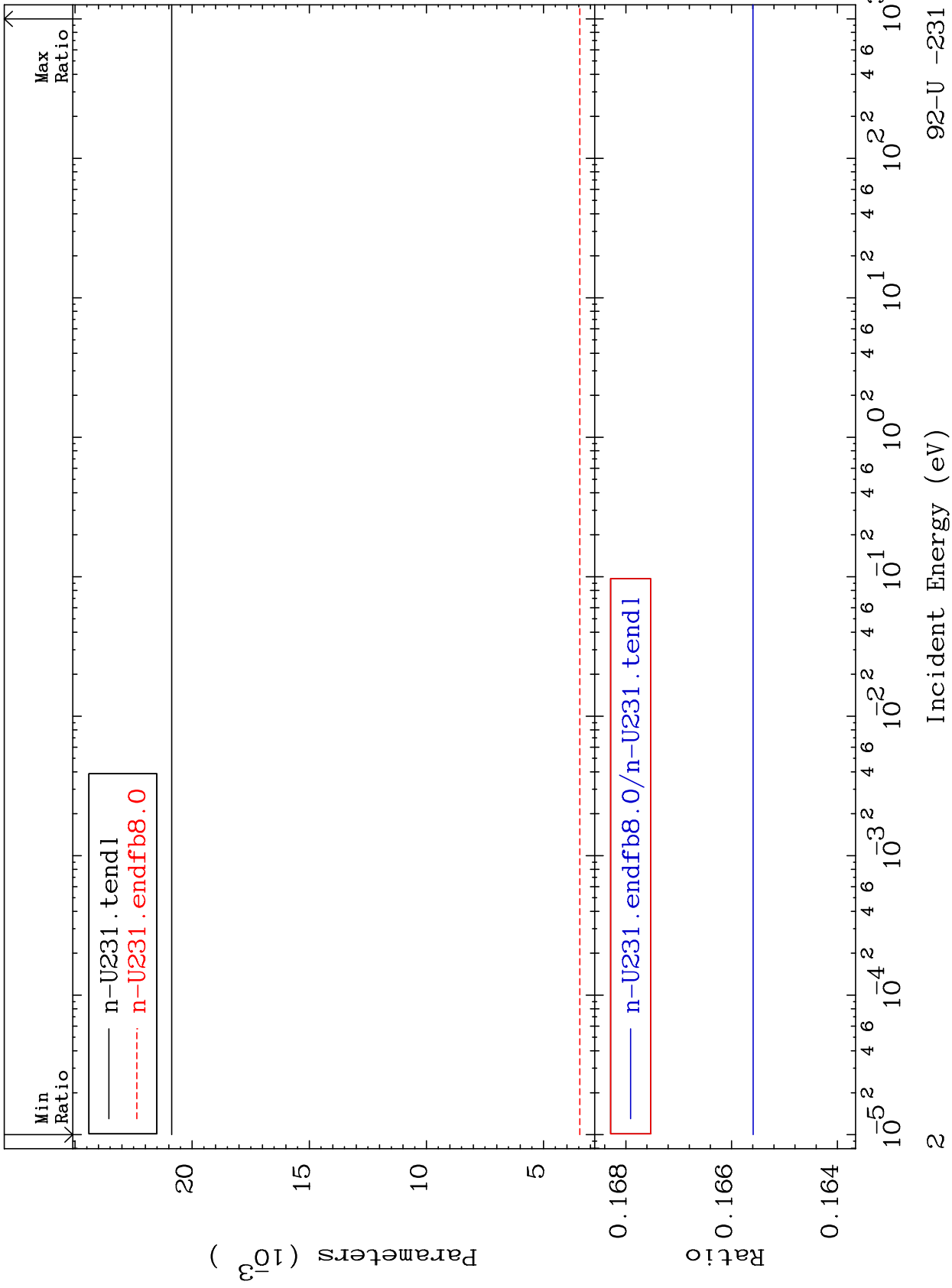
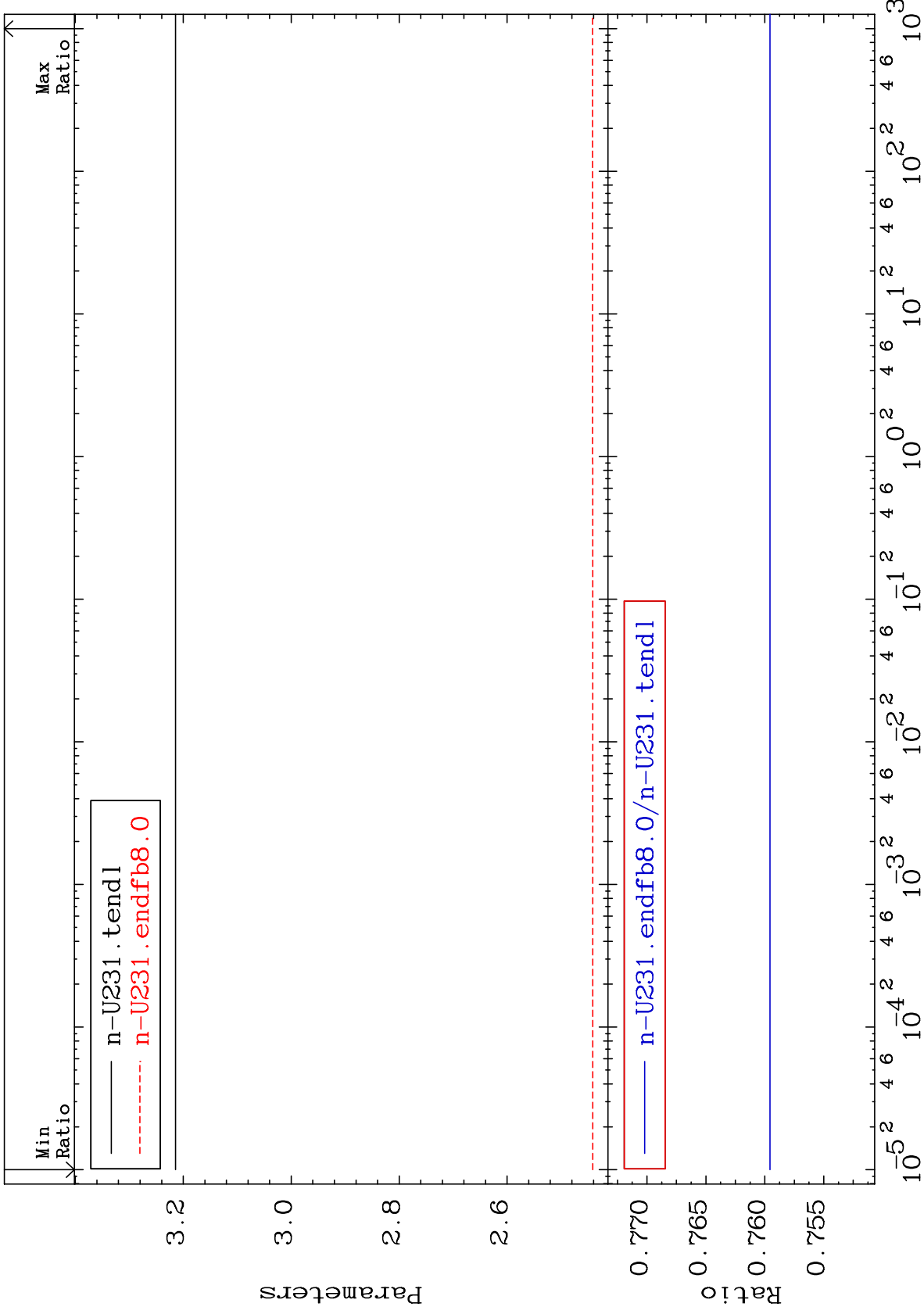


MAT 9216

Delayed $\bar{\nu}$
Parameters

92-U -231
-83.44 To -83.44%

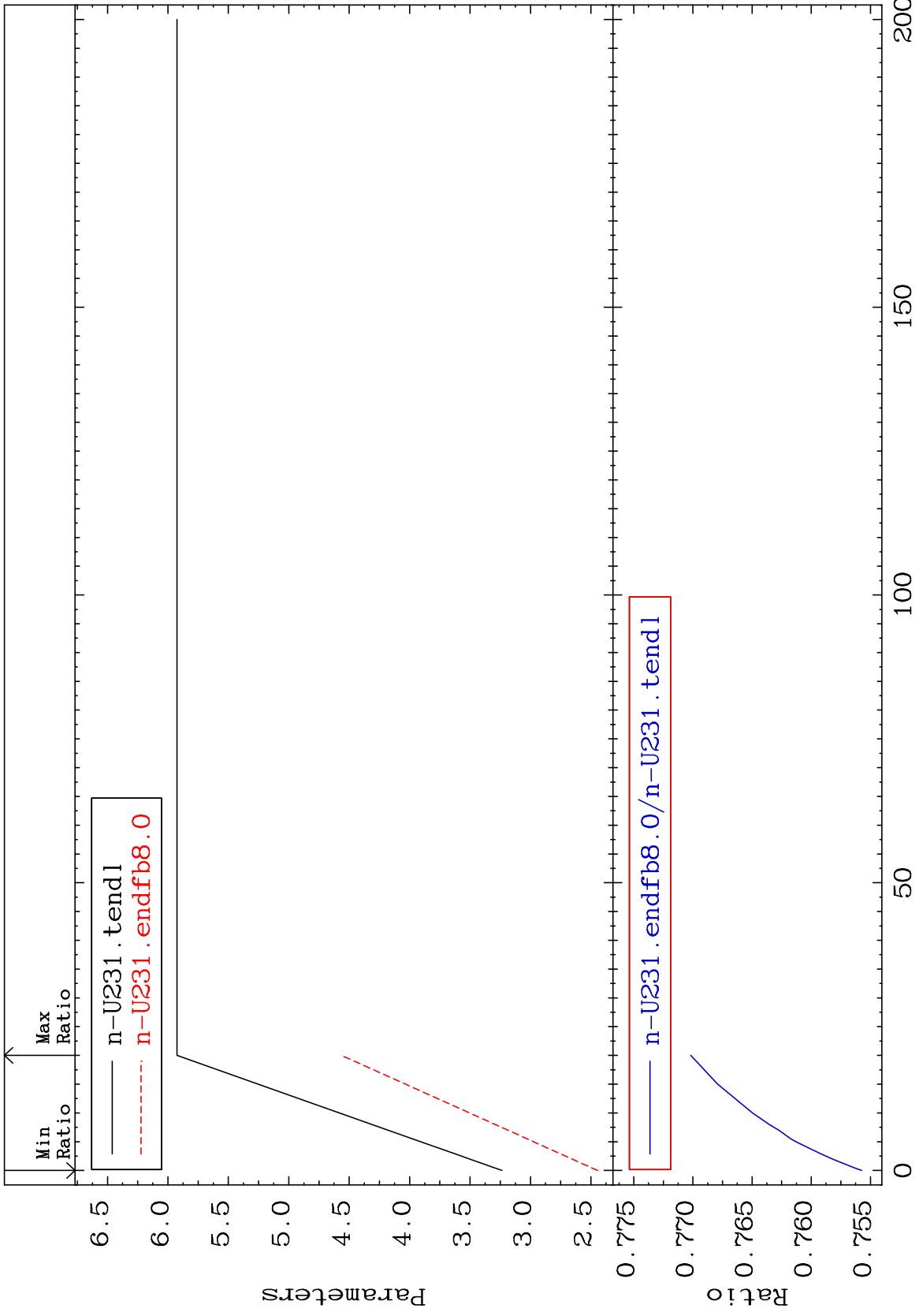




MAT 9216

Total $\bar{\nu}$
Parameters

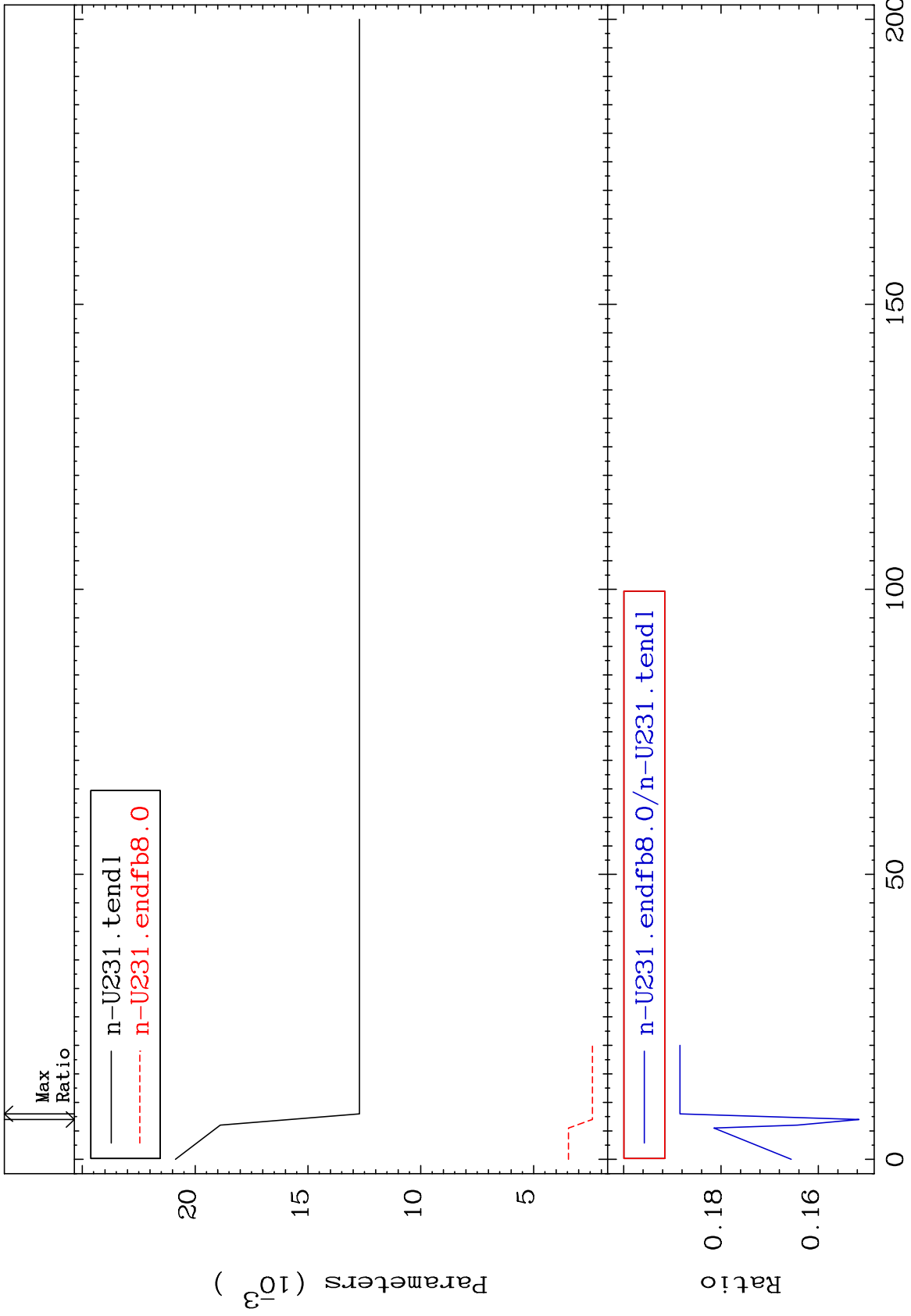
92-U -231
-24.43 To -22.98%



MAT 9216

Delayed $\bar{\nu}$
Parameters

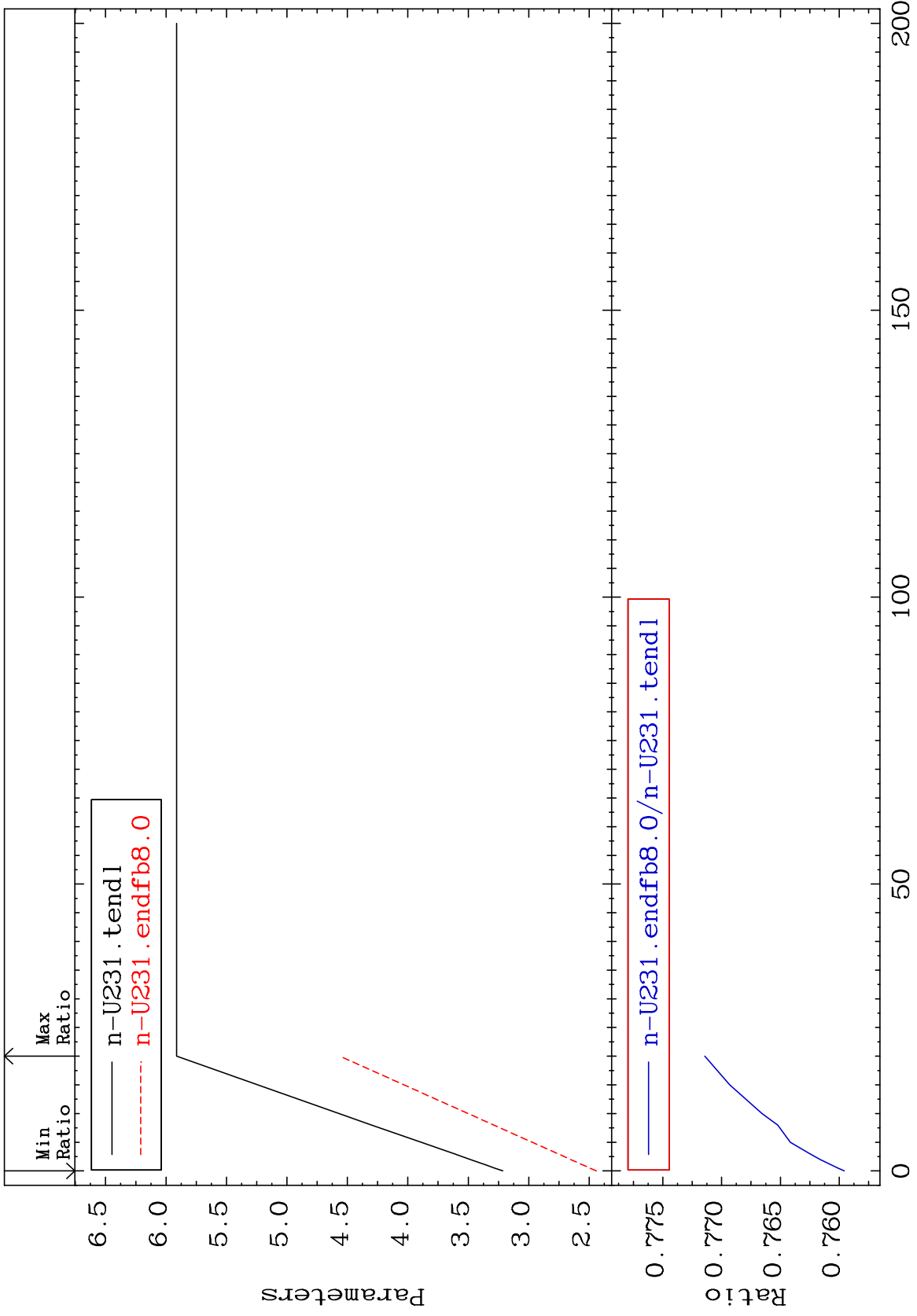
92-U -231
-84.83 To -81.16%



Incident Energy (MeV)

92-U -231

2



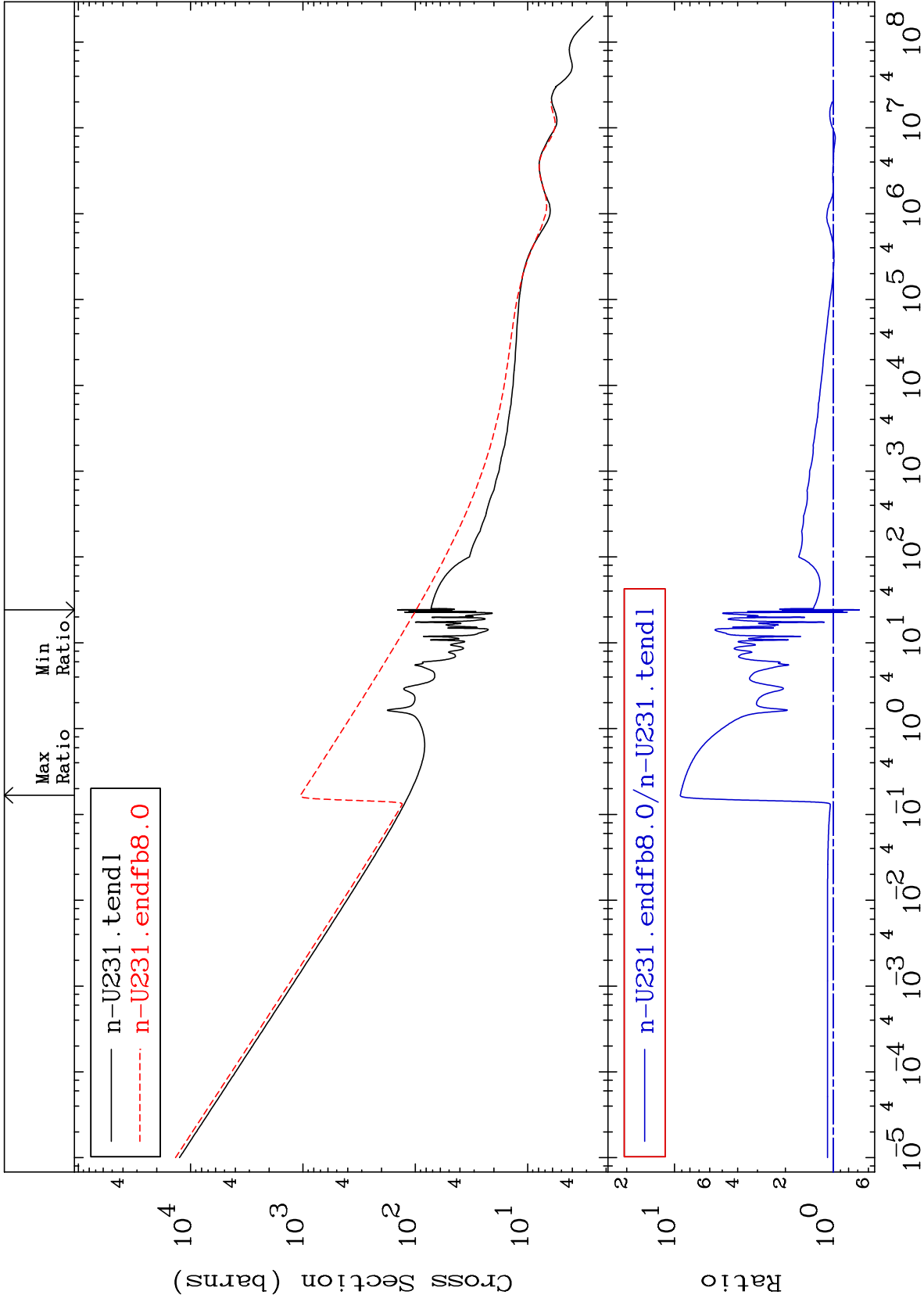
MAT 9216

Total

92-U -231

Cross Section

-31.76 To 818.8 %



92-U -231

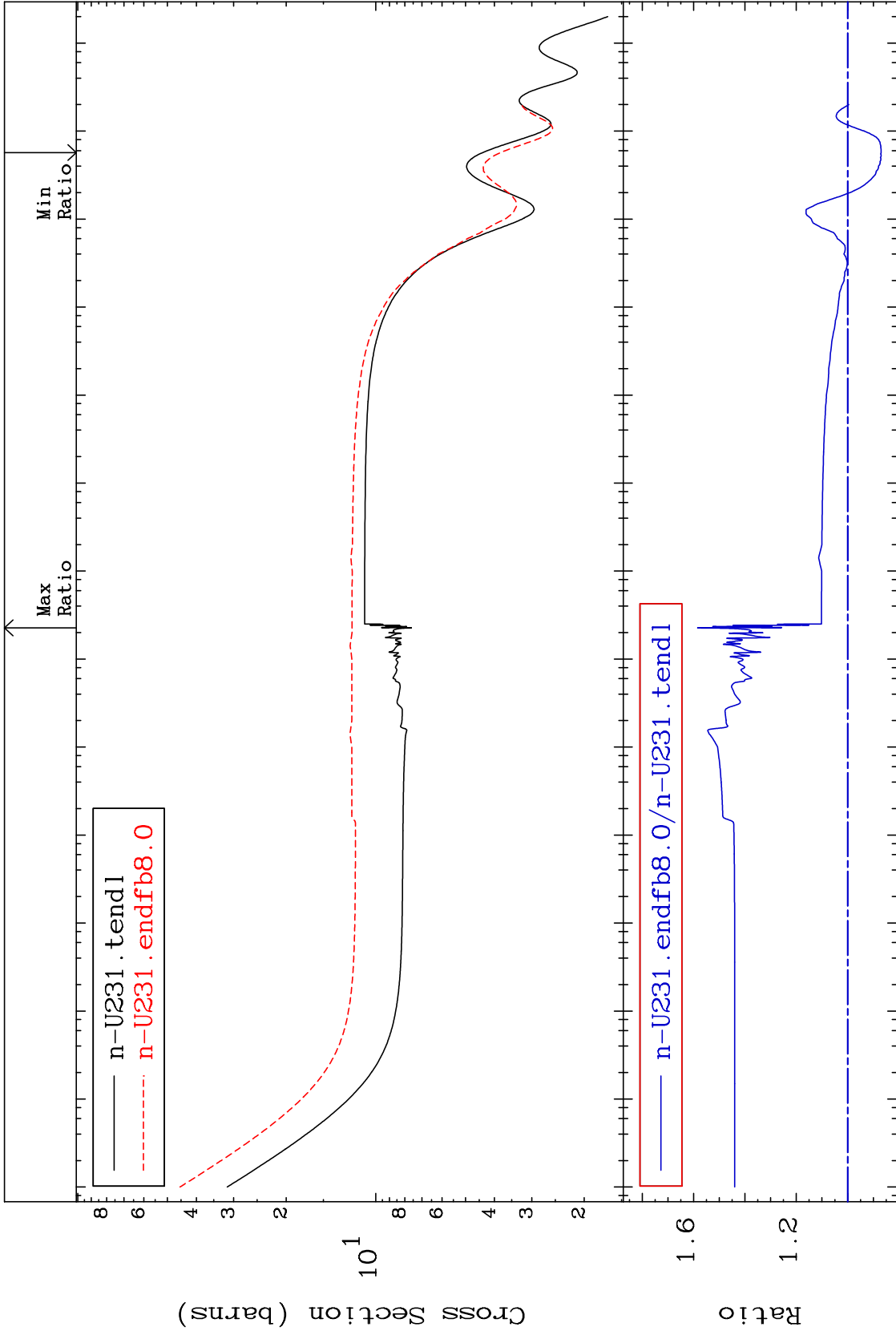
MAT 9216

Elastic

92-U -231

Cross Section

-13.02 To 58.48 %



92-U -231

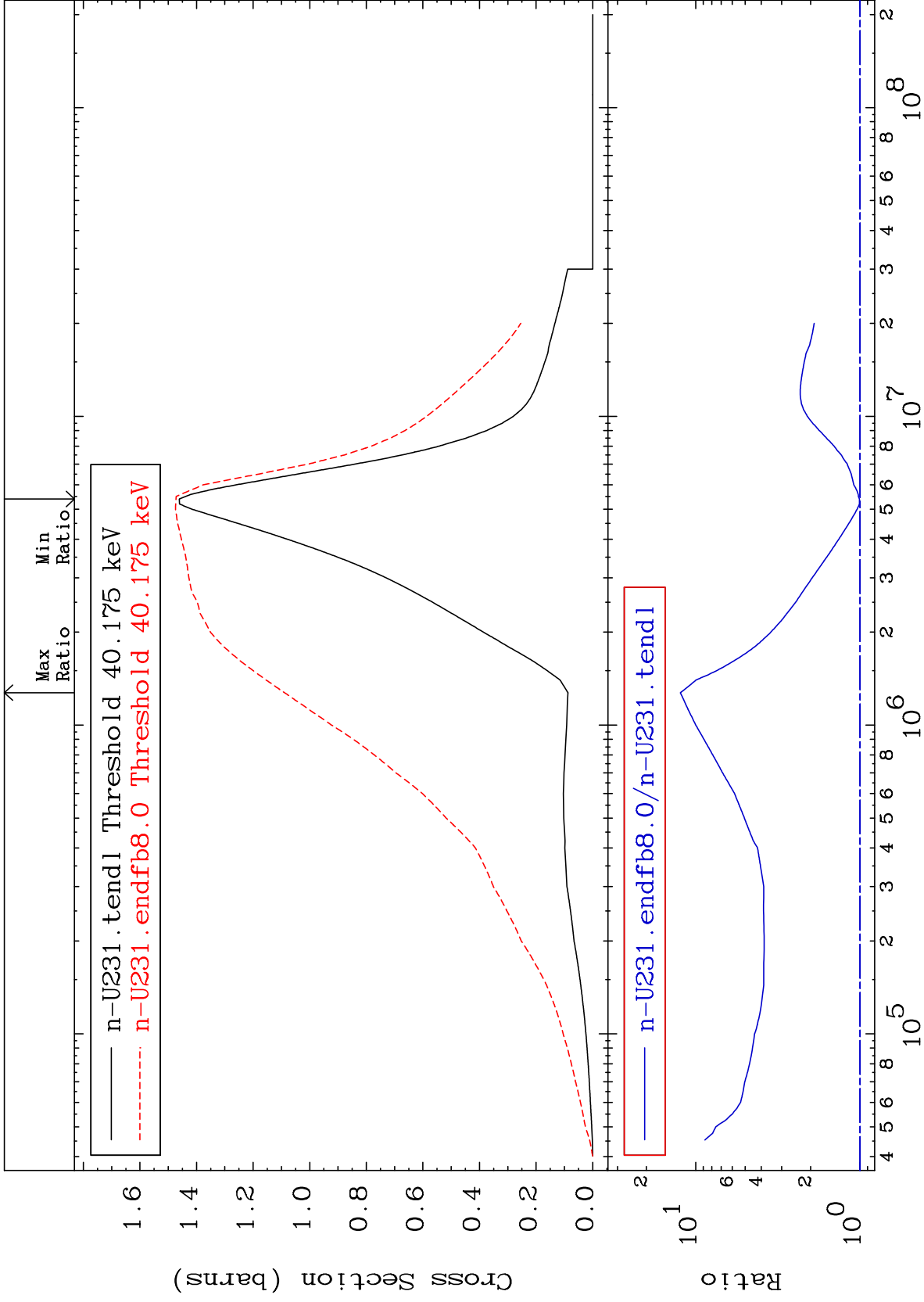
Incident Energy (eV)

2

MAT 9216

92-U -231
0.812 To 1142. %

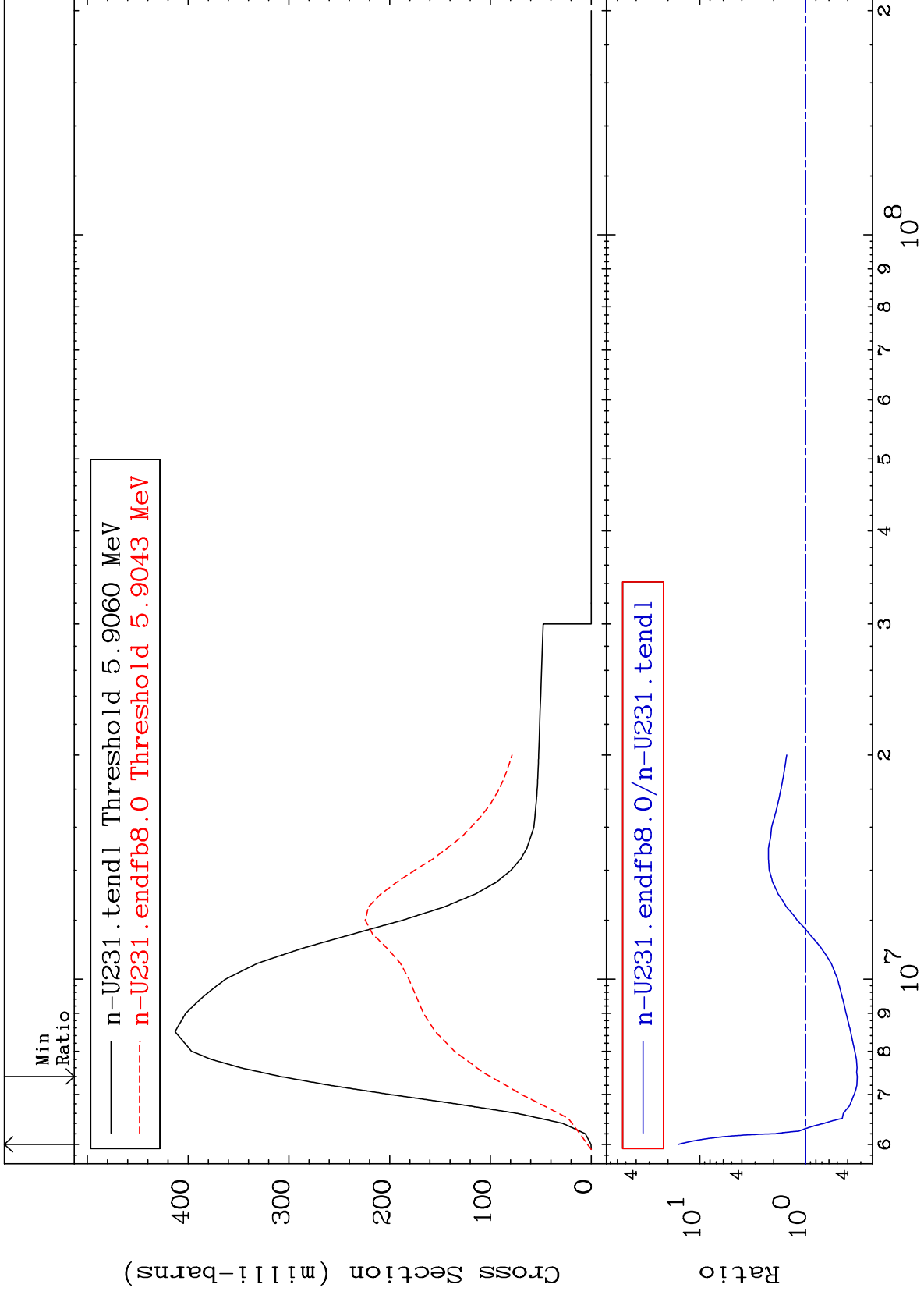
Inelastic
Cross Section



MAT 9216

(n,2n)
Cross Section

92-U -231
-67.56 To 1483. %



4

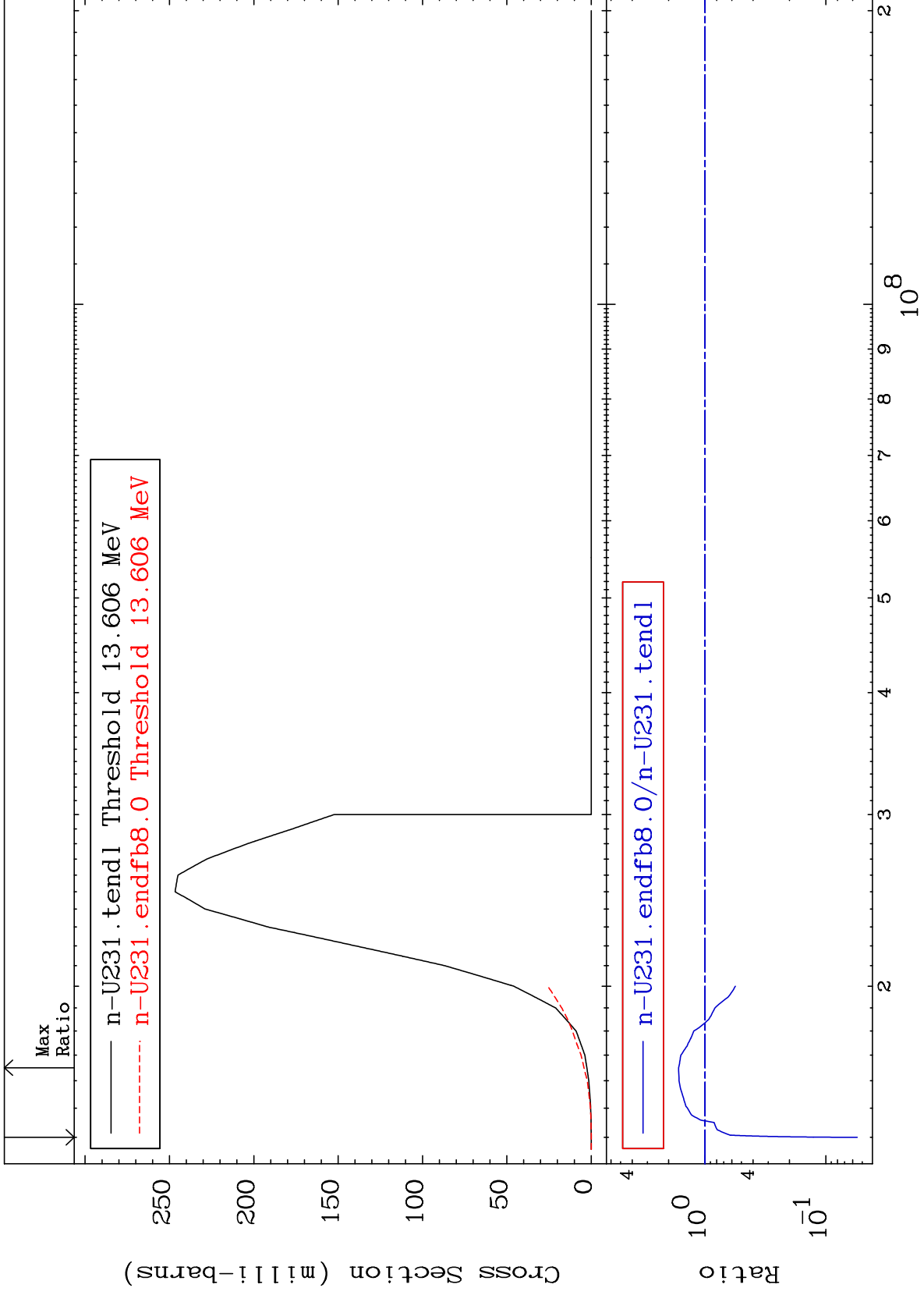
Incident Energy (eV)

92-U -231

MAT 9216

(n,3n)
Cross Section

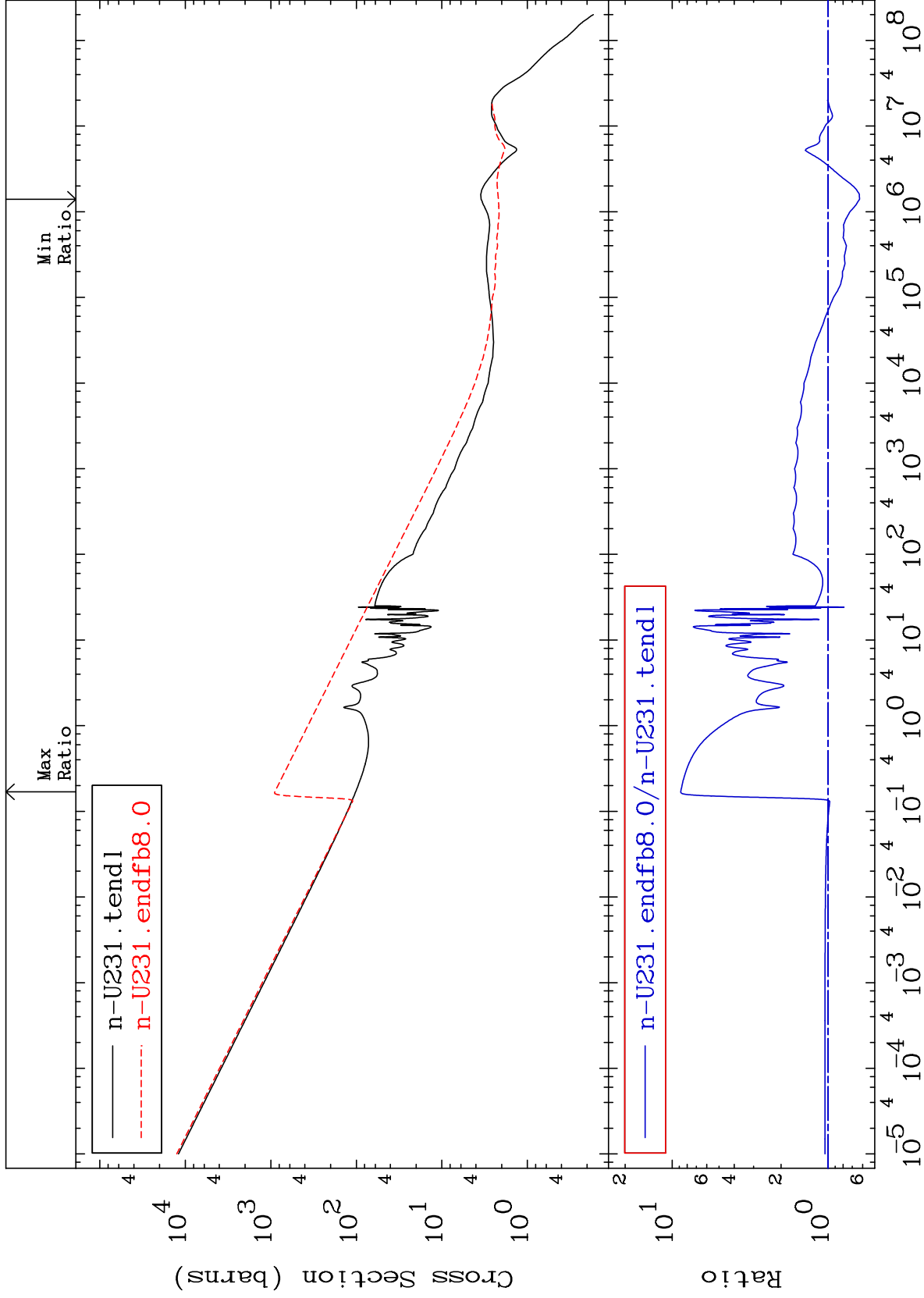
92-U -231
-94.51 To 65.20 %



MAT 9216

Fission
Cross Section

92-U -231
-37.21 To 779.7 %



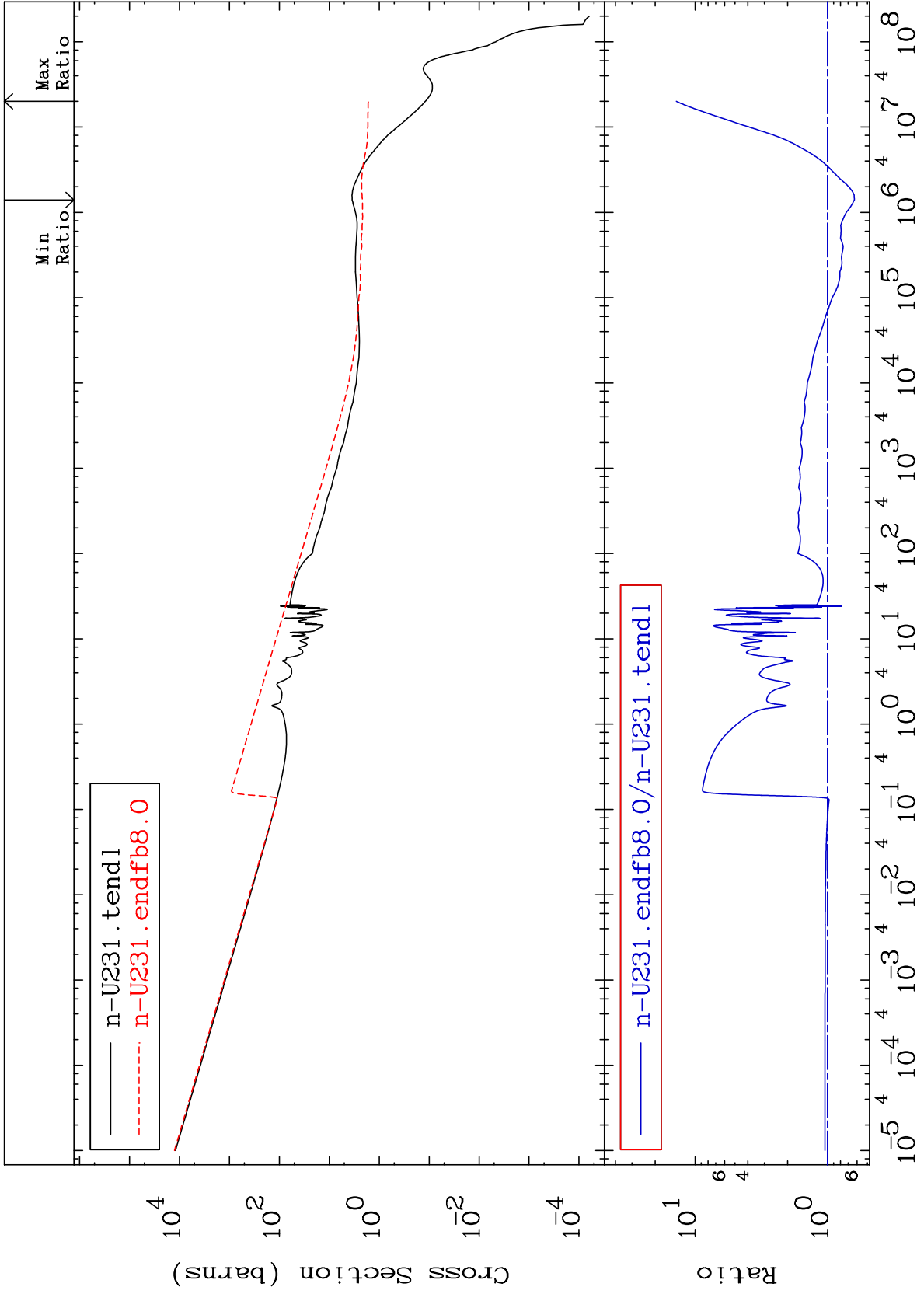
Incident Energy (eV)

92-U -231

MAT 9216

(n,f) First Chance
Cross Section

92-U -231
-37.21 To 1286. %



Incident Energy (eV)

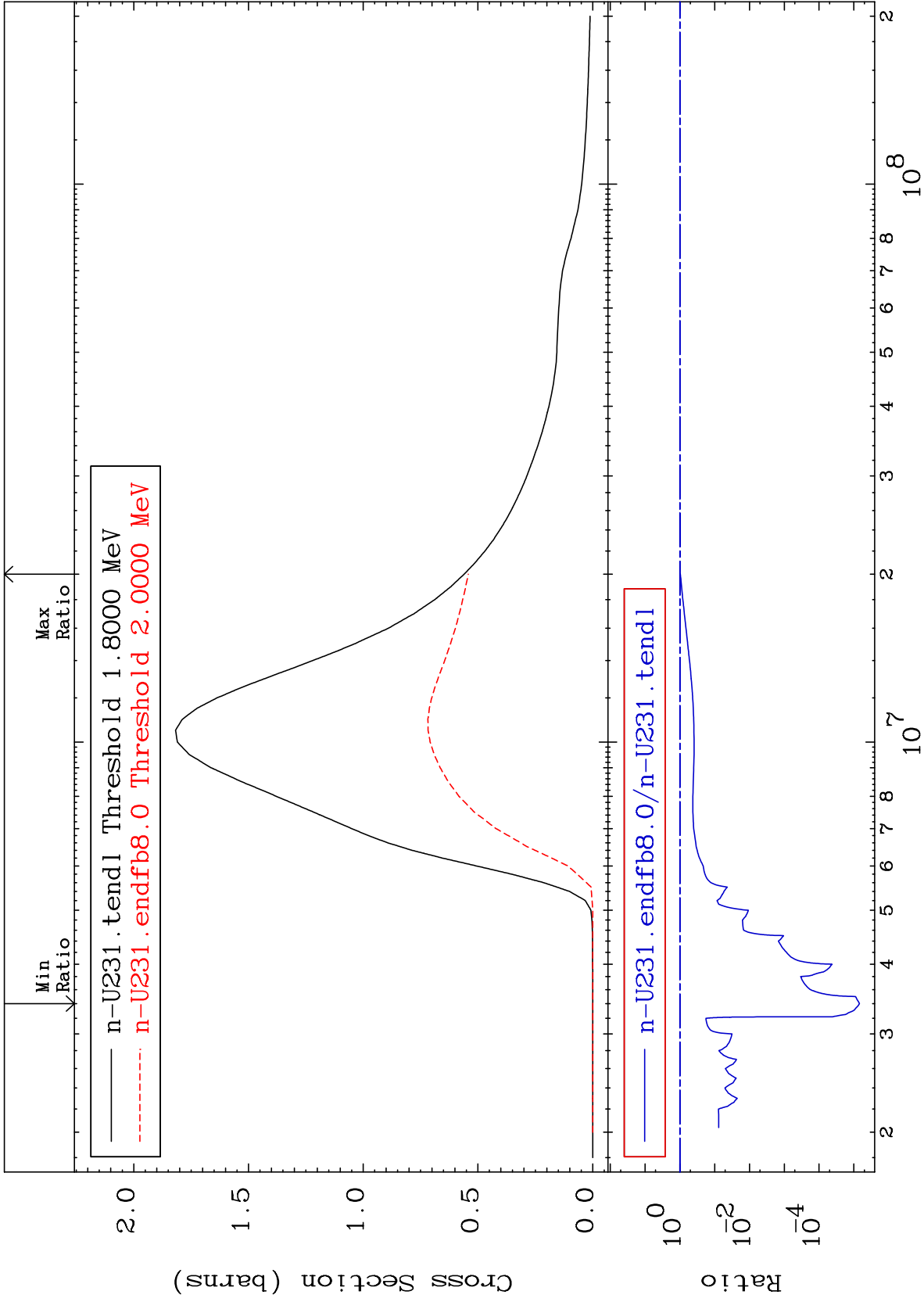
92-U -231

7

MAT 9216

(n, nf) Second Chance
Cross Section

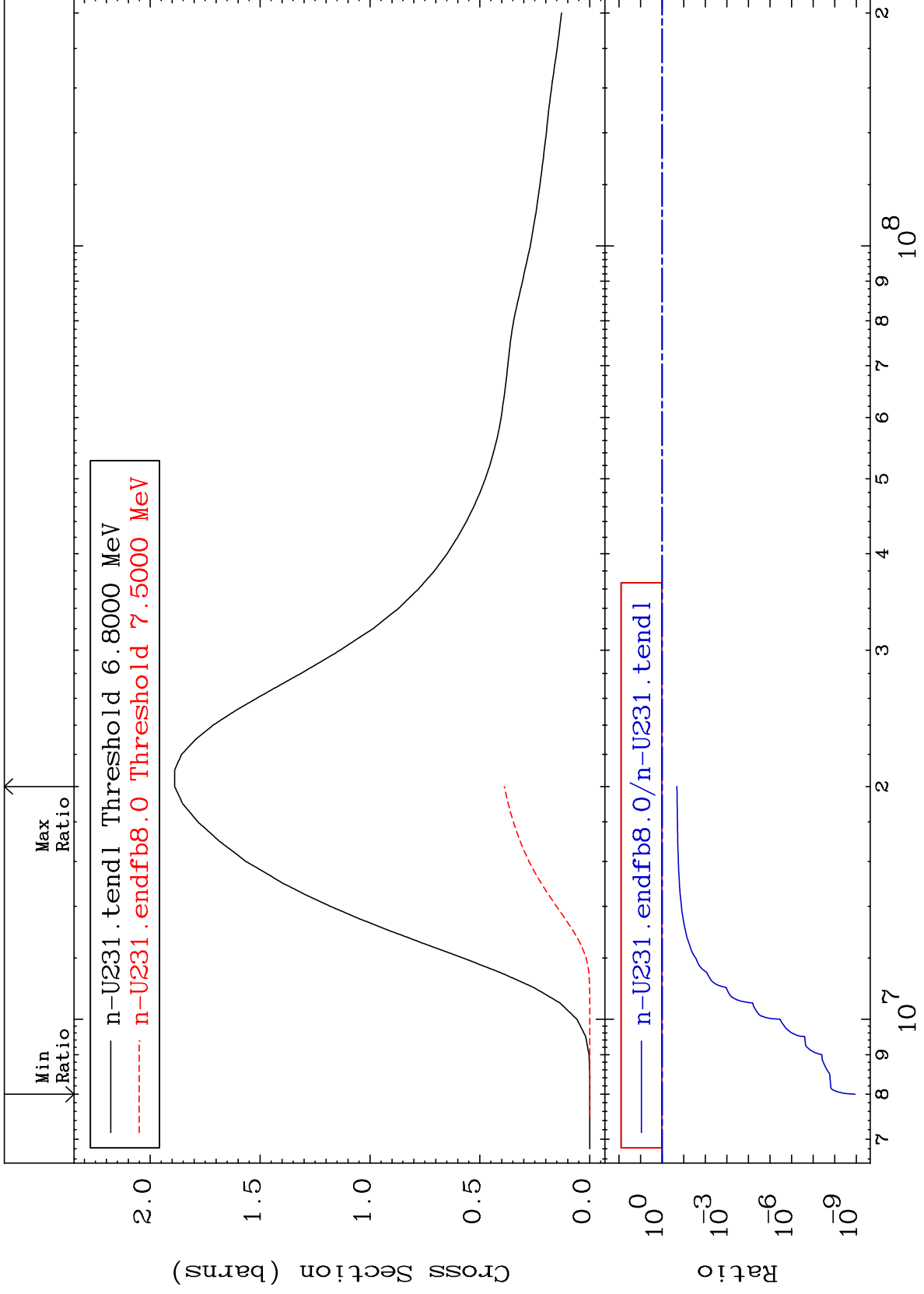
92-U -231
-100.0 To -3.212%



MAT 9216

(n,2nf) Third Chance
Cross Section

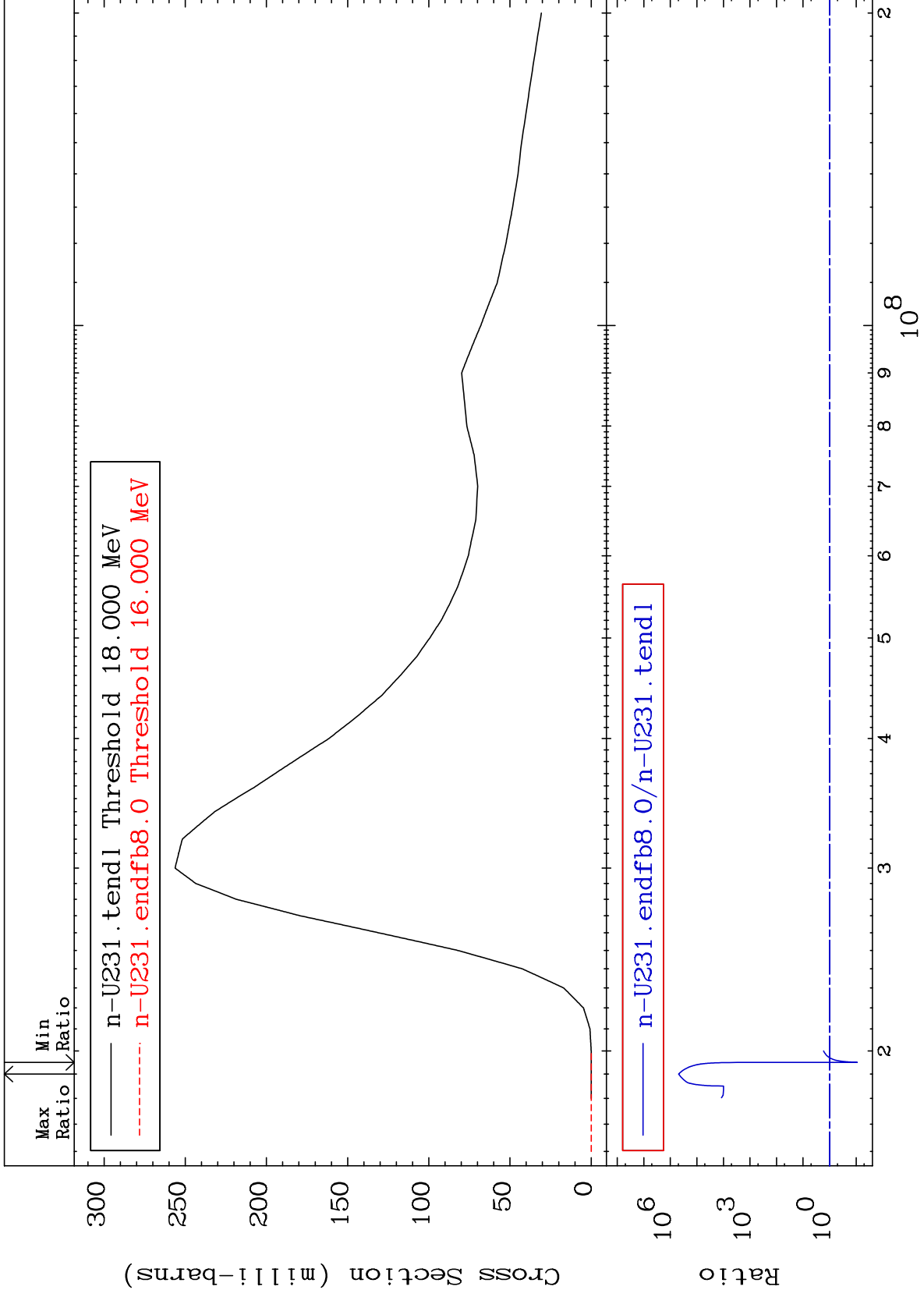
92-U -231
-100.0 To -79.44%



MAT 9216

(n,3nf) Fourth Chance
Cross Section

92-U -231
-90.94 To 9999. %



10

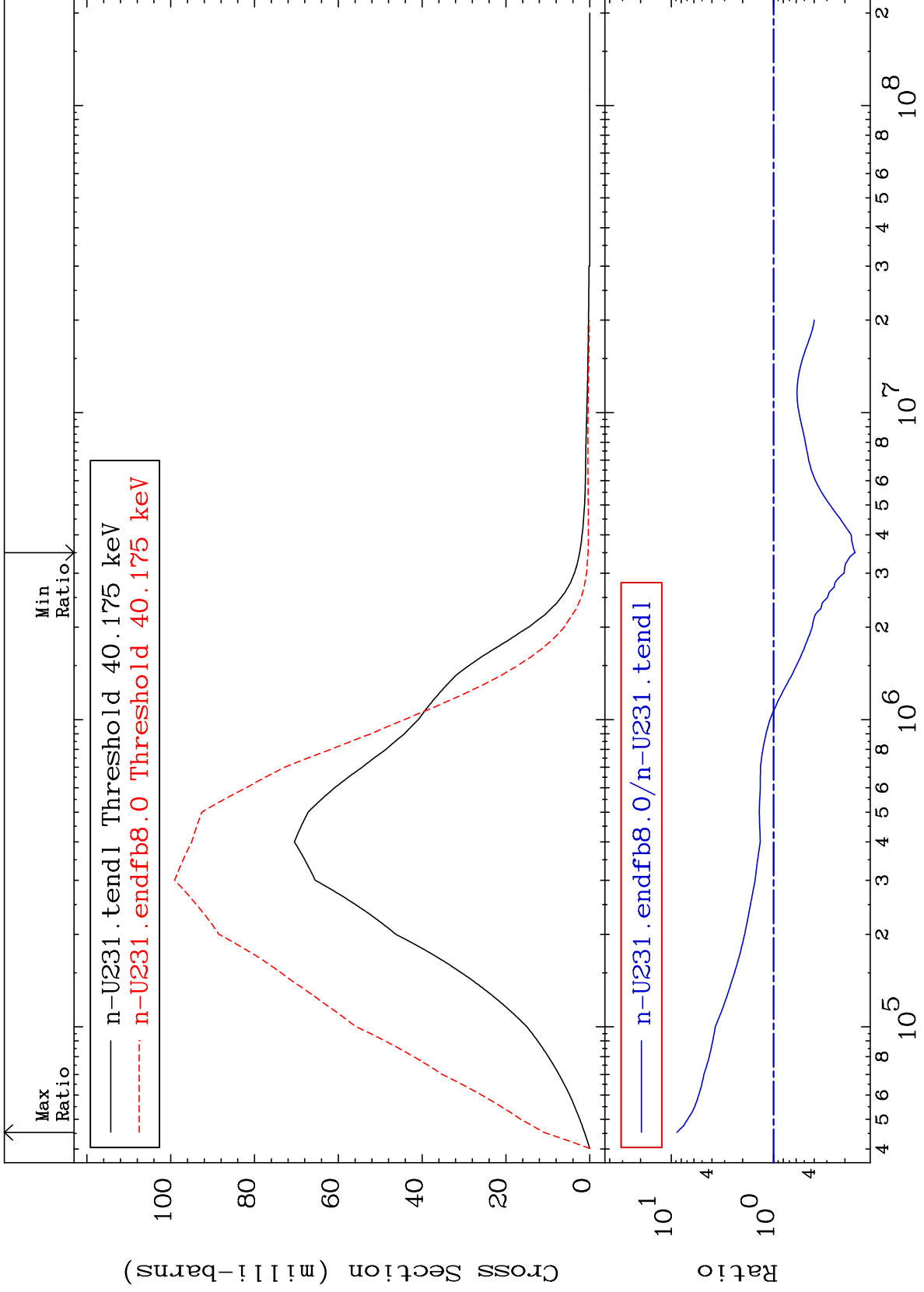
Incident Energy (eV)

92-U -231

MAT 9216

MT= 51 (n, n') Level
Cross Section

92-U -231
-84.10 To 779.9 %



11

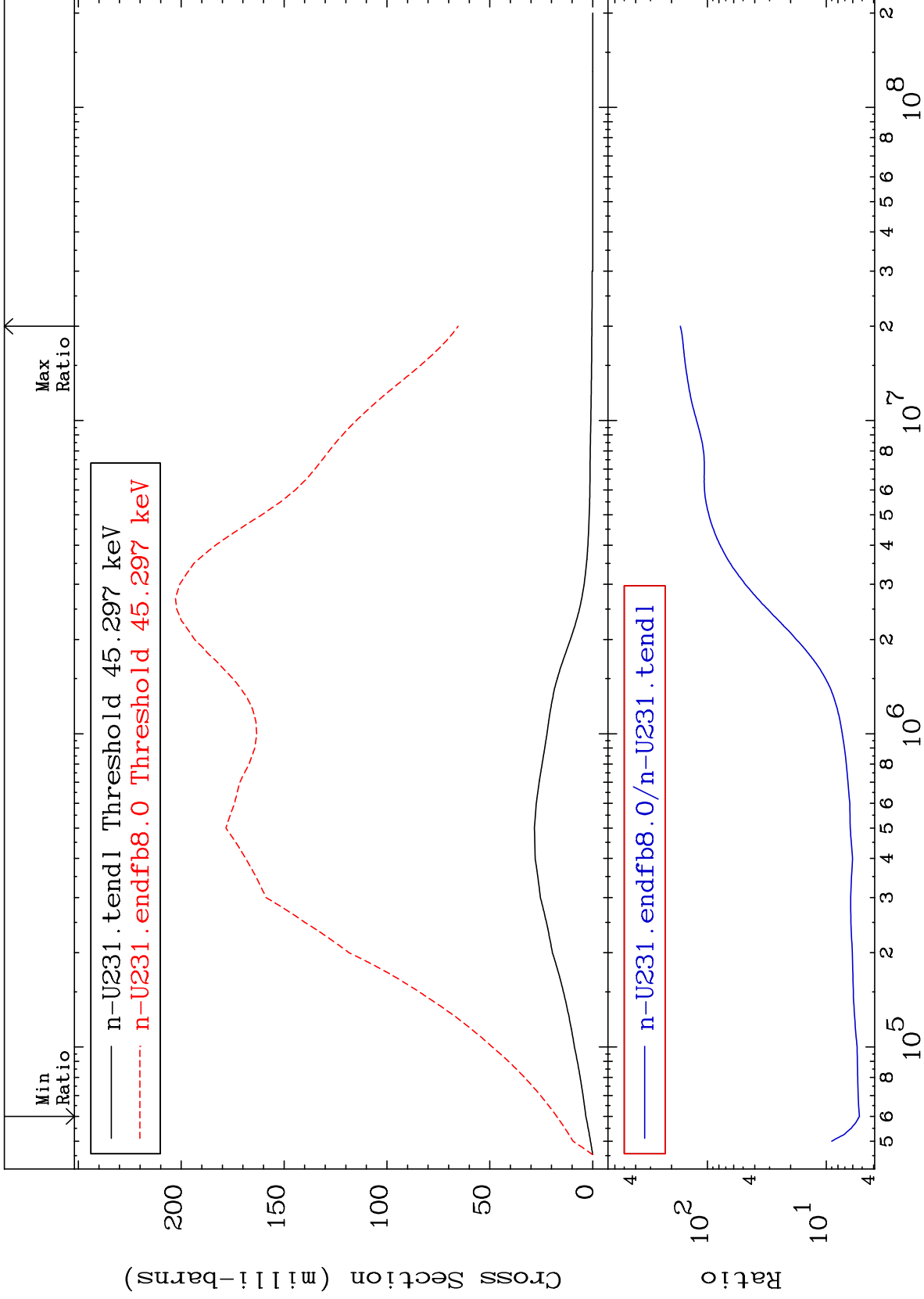
Incident Energy (eV)

92-U -231

MAT 9216

MT= 52 (n,n') Level
Cross Section

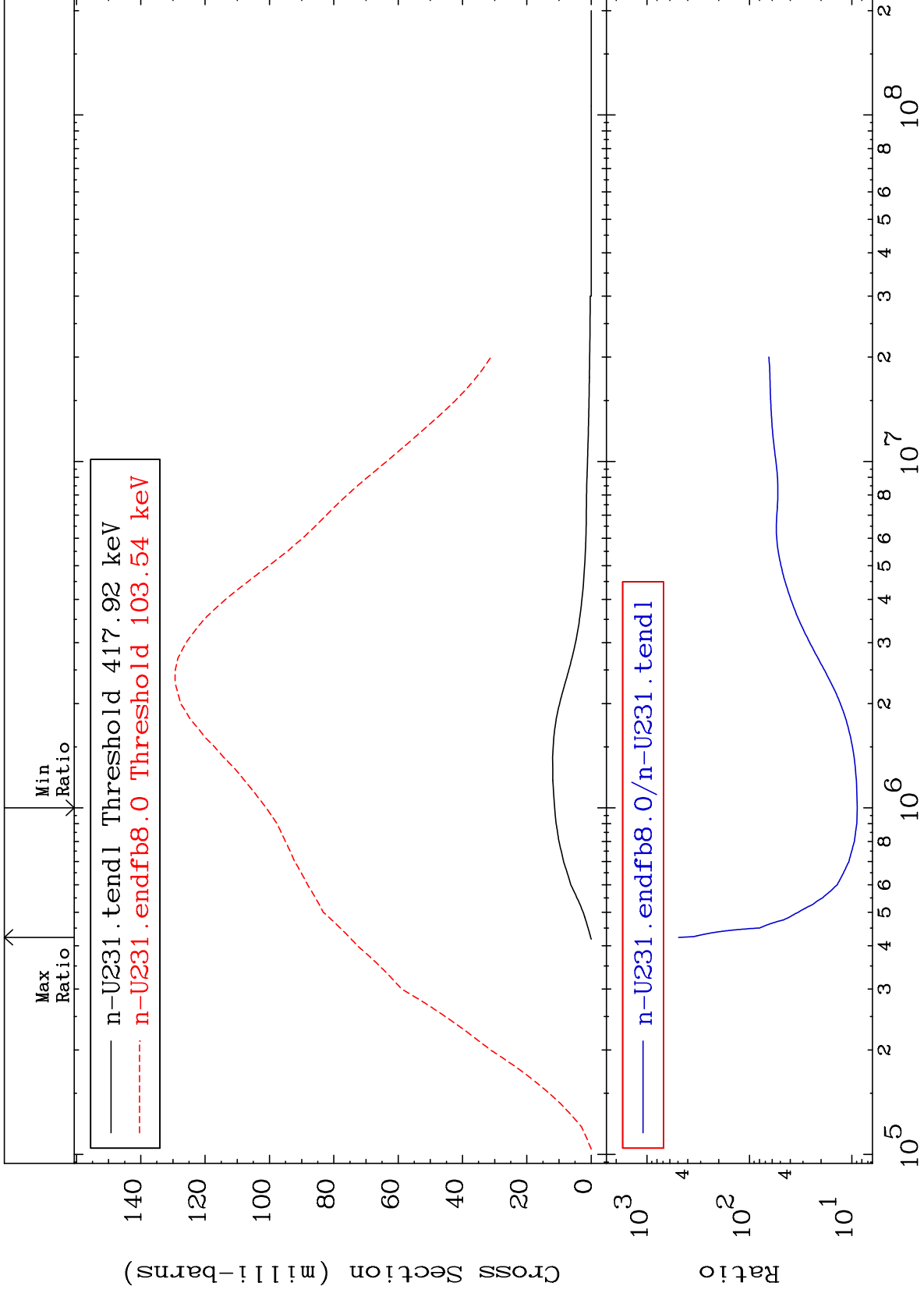
92-U -231
427.1 To 9999. %



MAT 9216

MT= 53 (n,n') Level
Cross Section

92-U -231
787.2 To 9999. %



13

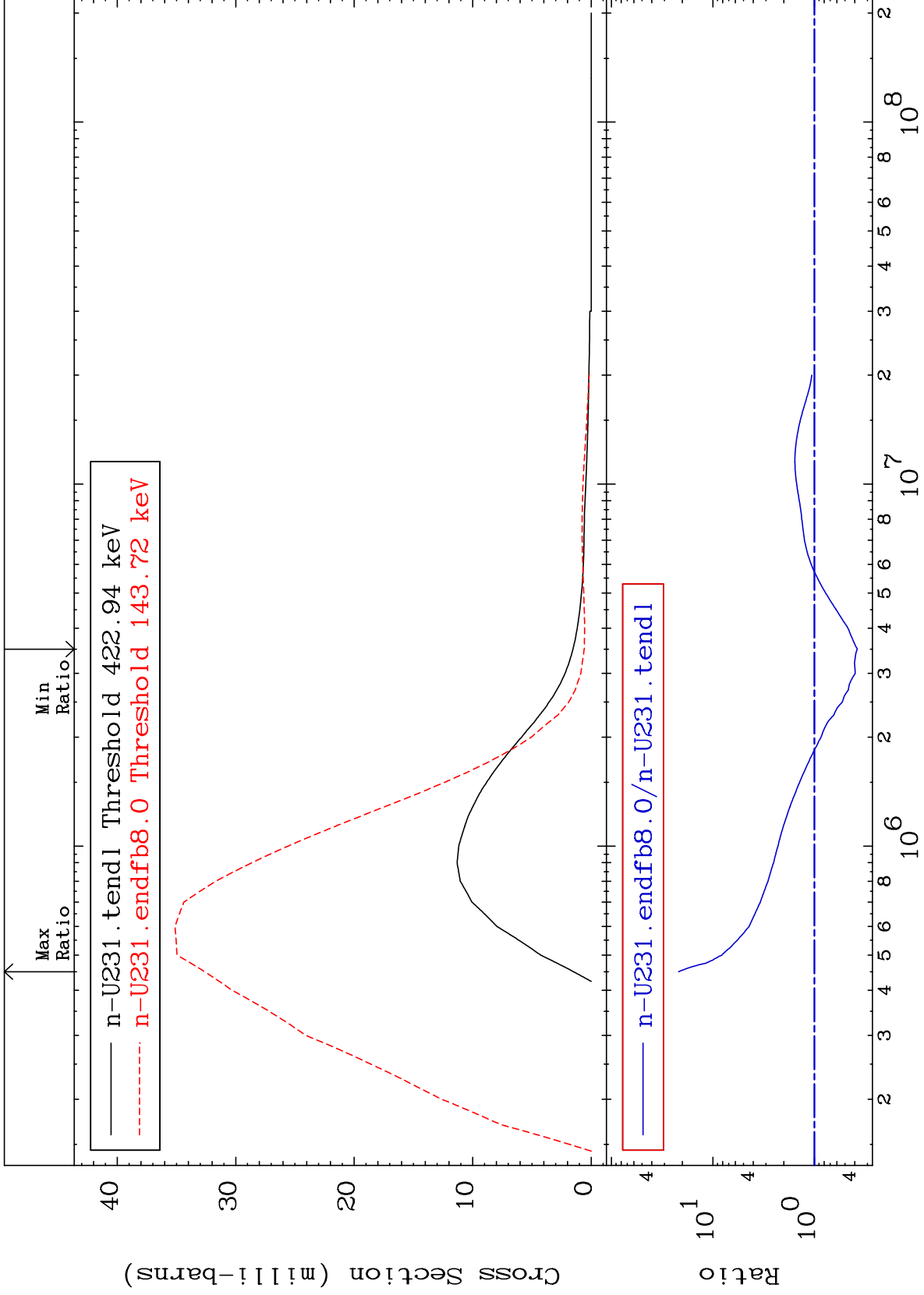
Incident Energy (eV)

92-U -231

MAT 9216

MT= 54 (n,n') Level
Cross Section

92-U -231
-62.15 To 2077. %



14

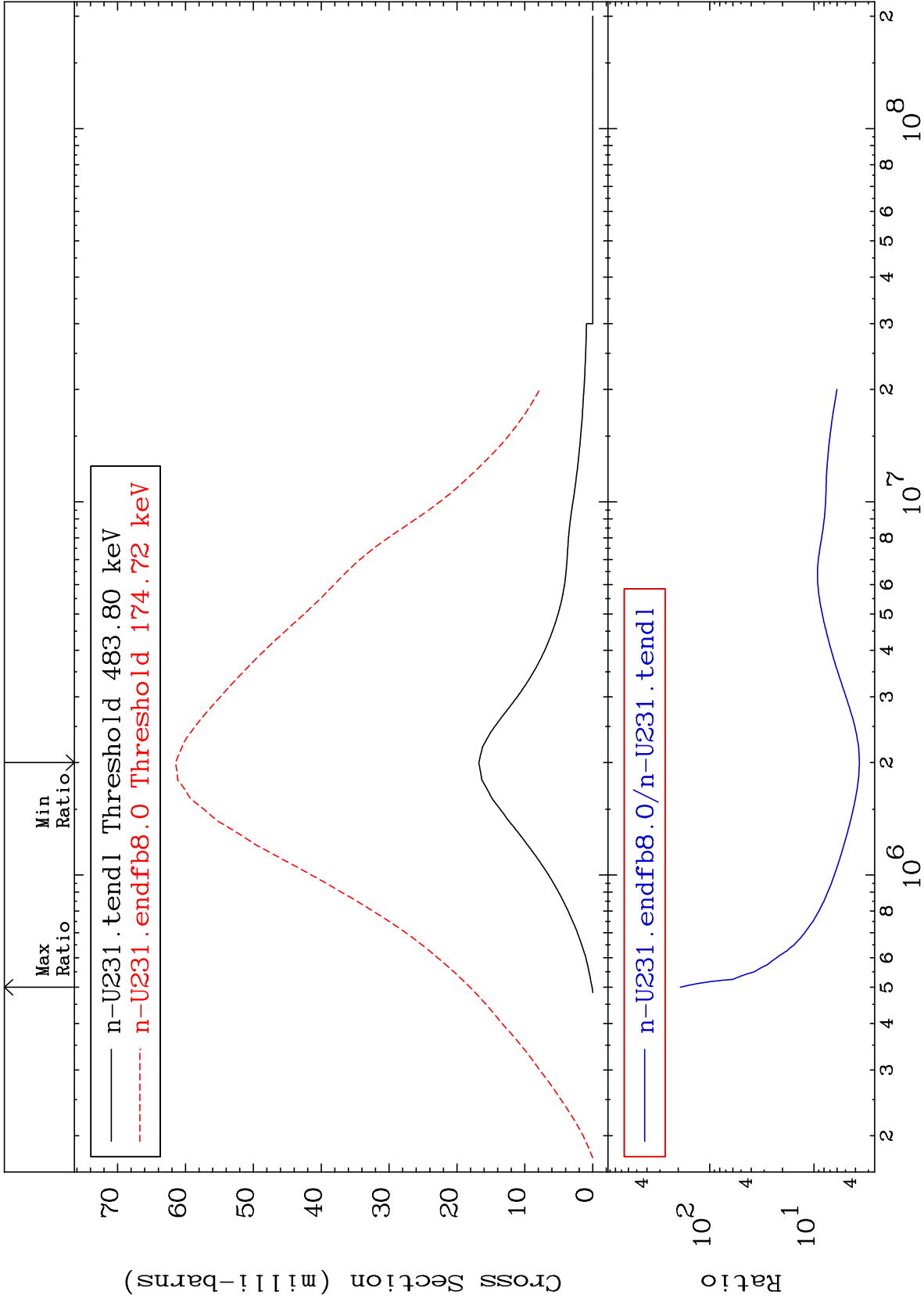
Incident Energy (eV)

92-U -231

MAT 9216

MT= 55 (n,n') Level
Cross Section

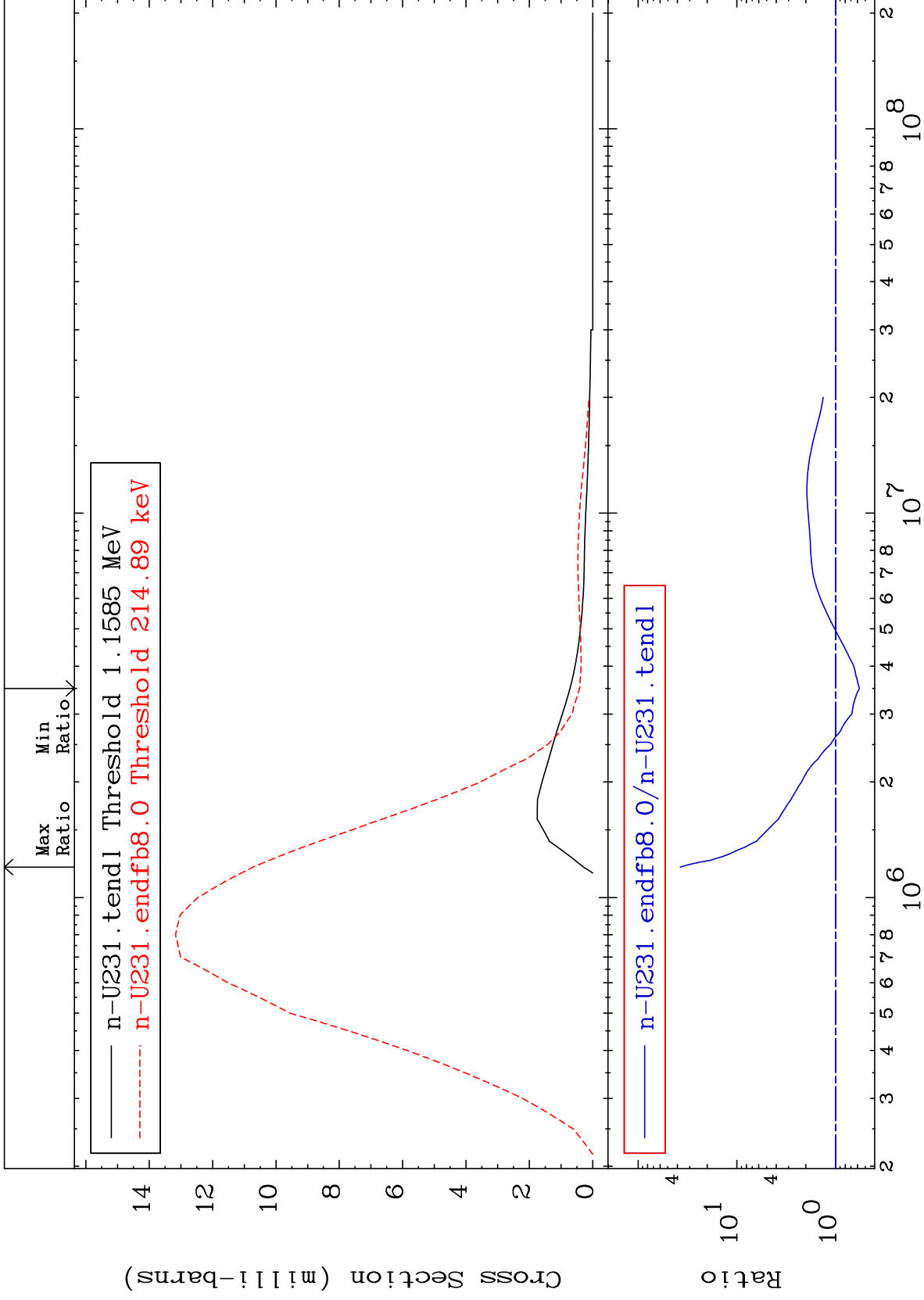
92-U -231
266.4 To 9999. %



15

Incident Energy (eV)

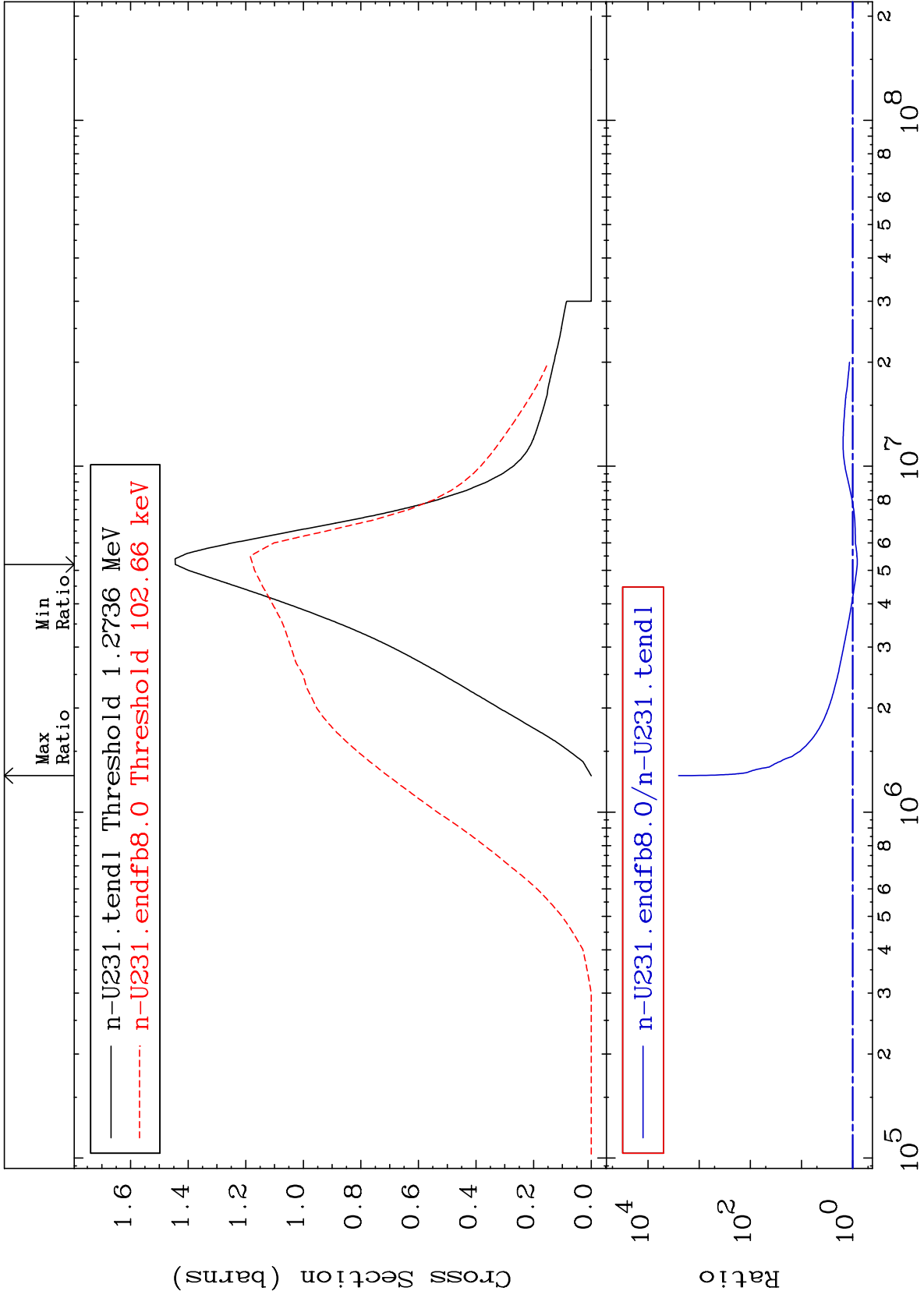
92-U -231



MAT 9216

(n, n') Continuum
Cross Section

92-U -231
-18.61 To 9999. %



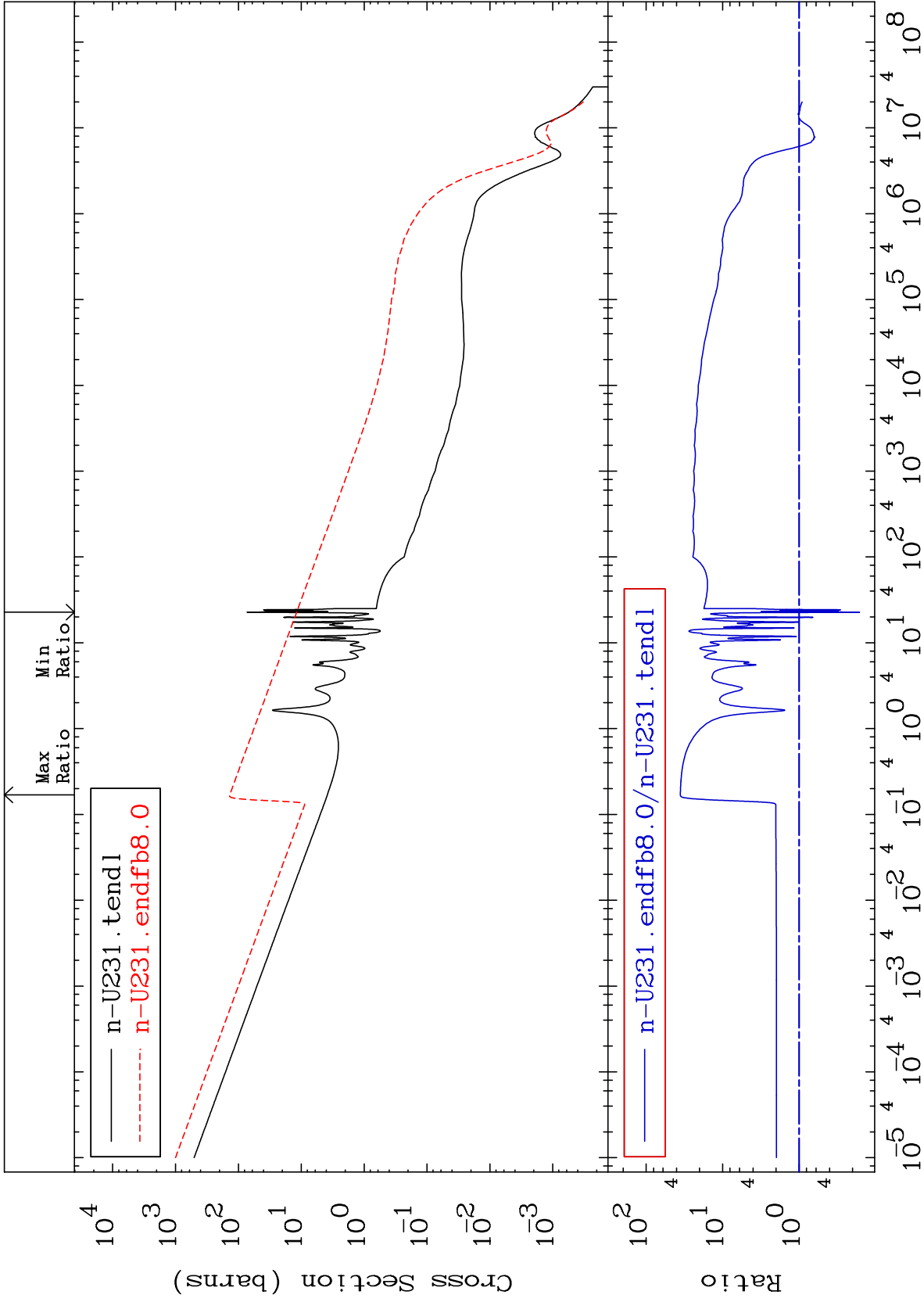
17

Incident Energy (eV)

92-U -231

MAT 9216

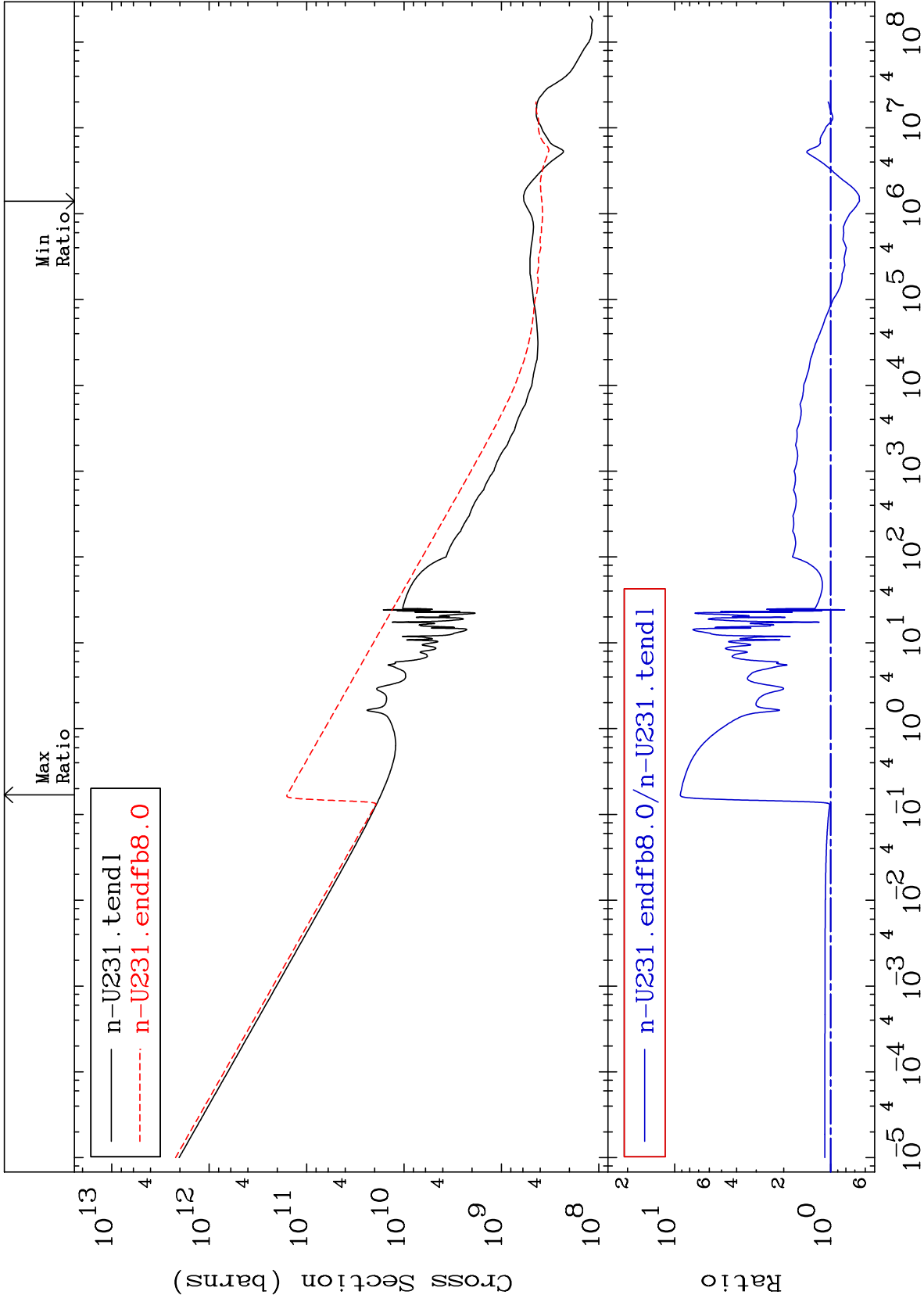
(n, γ)
Cross Section
92-U -231
-83.74 To 3469. %



18

Incident Energy (eV)

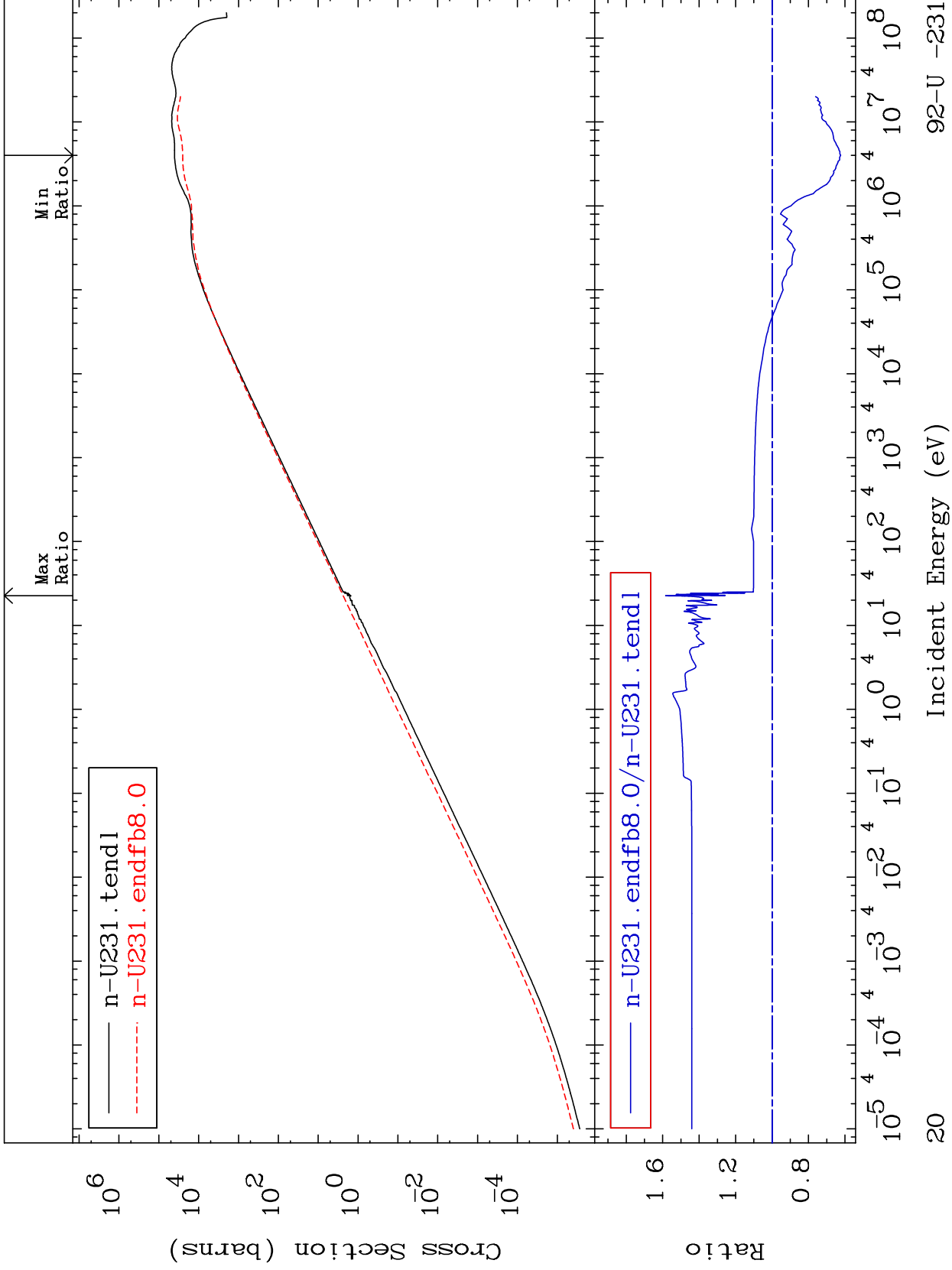
92-U -231



MAT 9216

Kerma elastic
Cross Section

92-U -231
-37.64 To 58.45 %



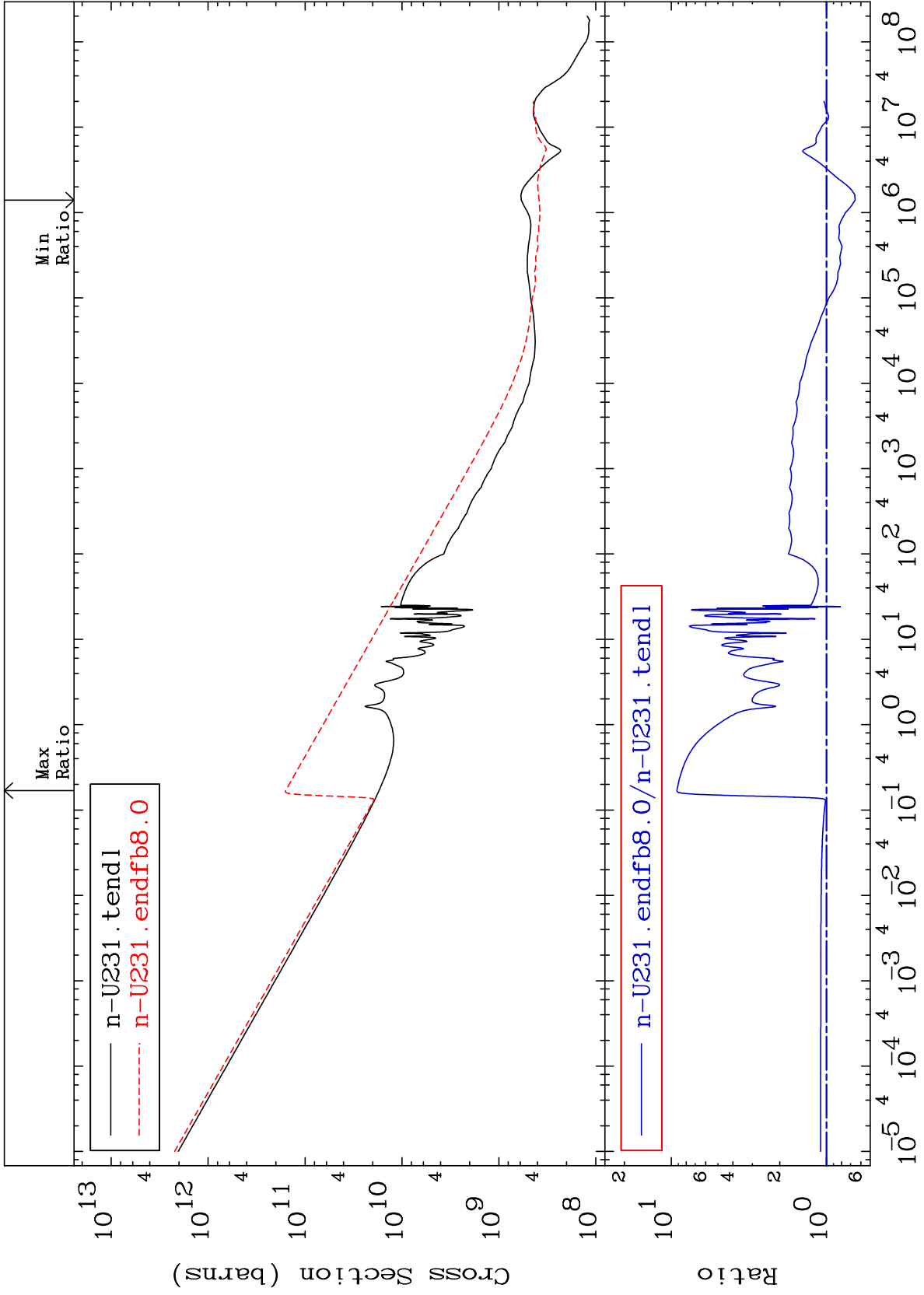
20

92-U -231

MAT 9216

Kerma non-elastic (all but mt2)
Cross Section

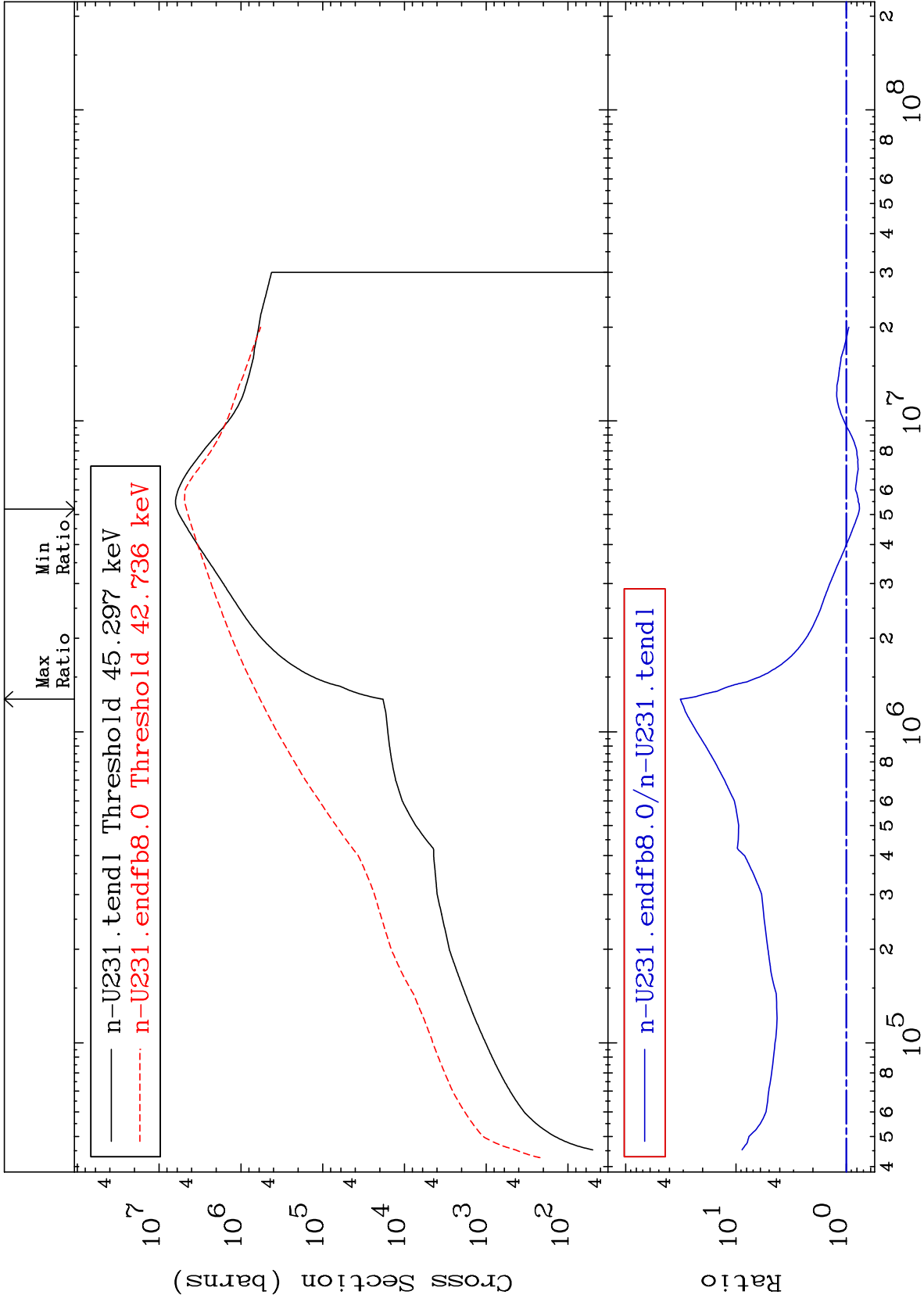
92-U -231
-34.58 To 818.4 %



21

Incident Energy (eV)

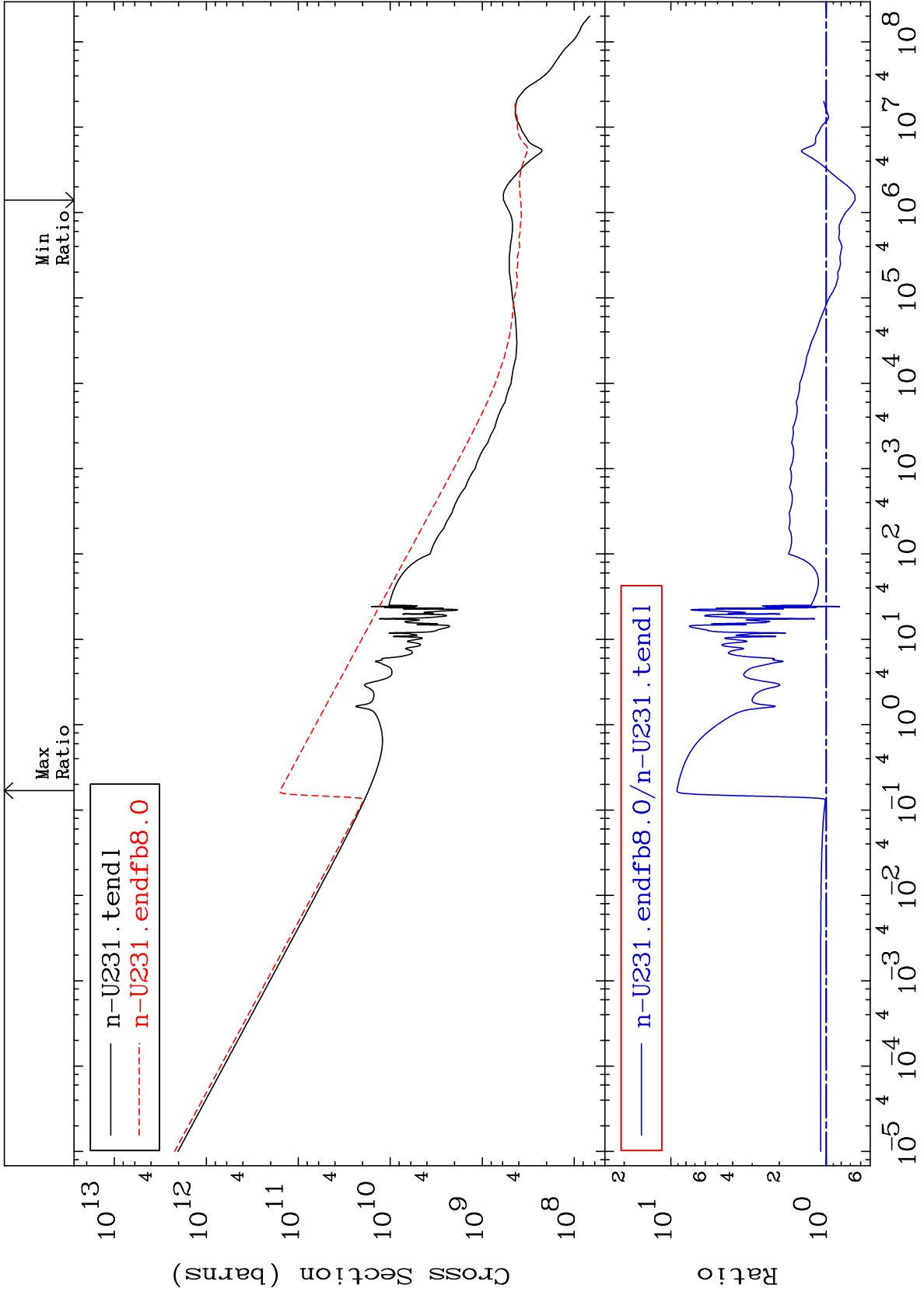
92-U -231



MAT 9216

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

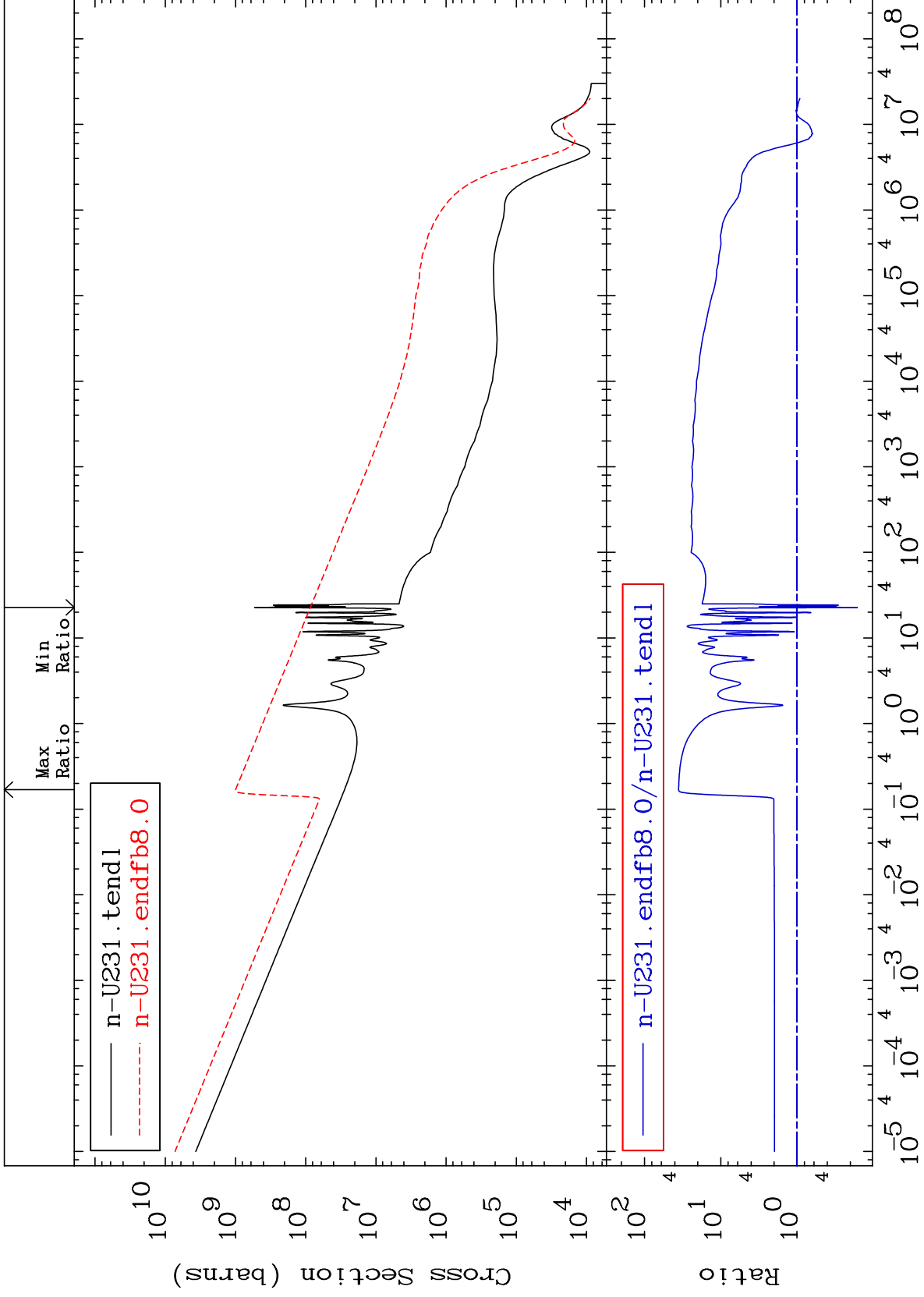
92-U -231
-34.82 To 814.1 %

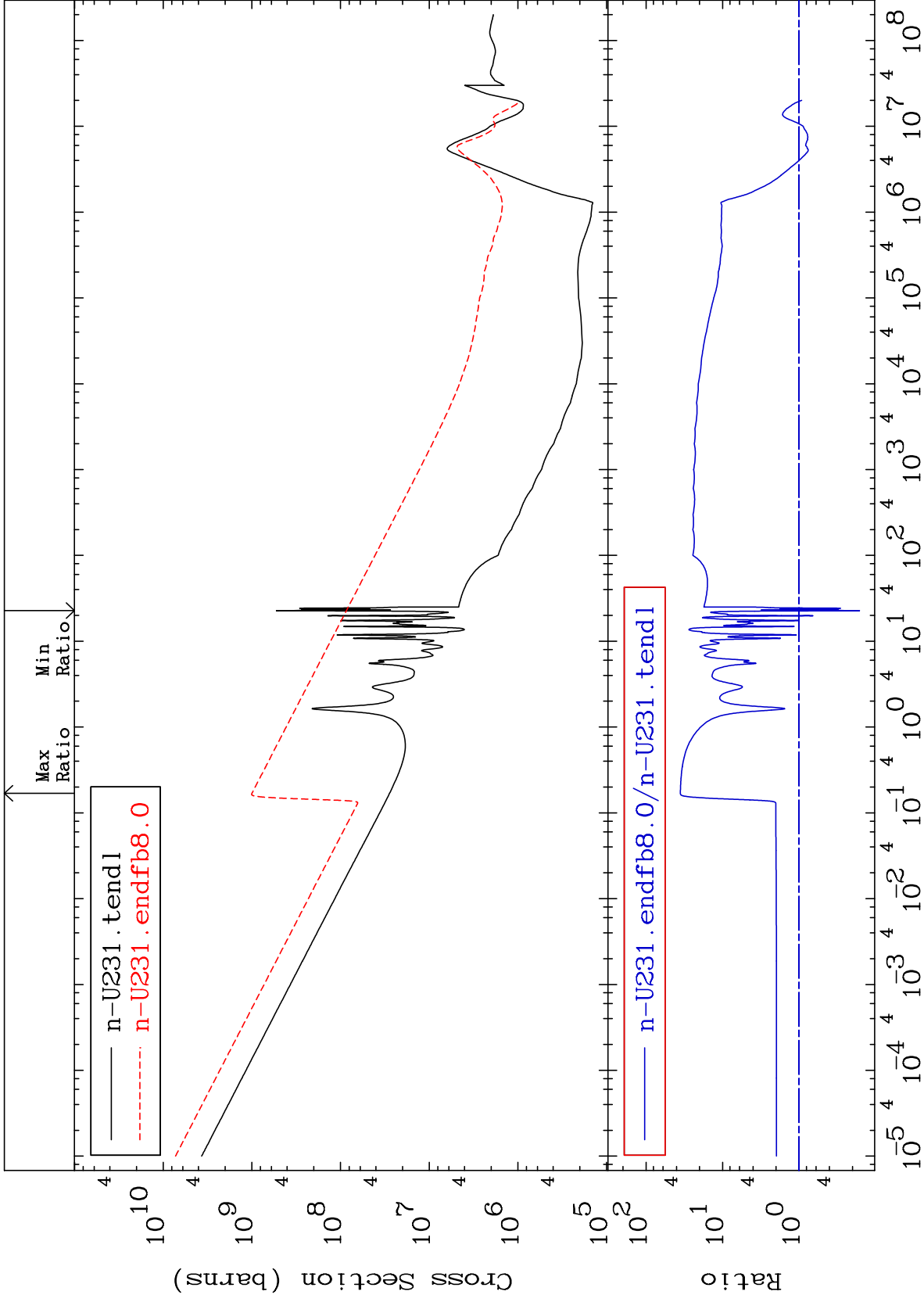


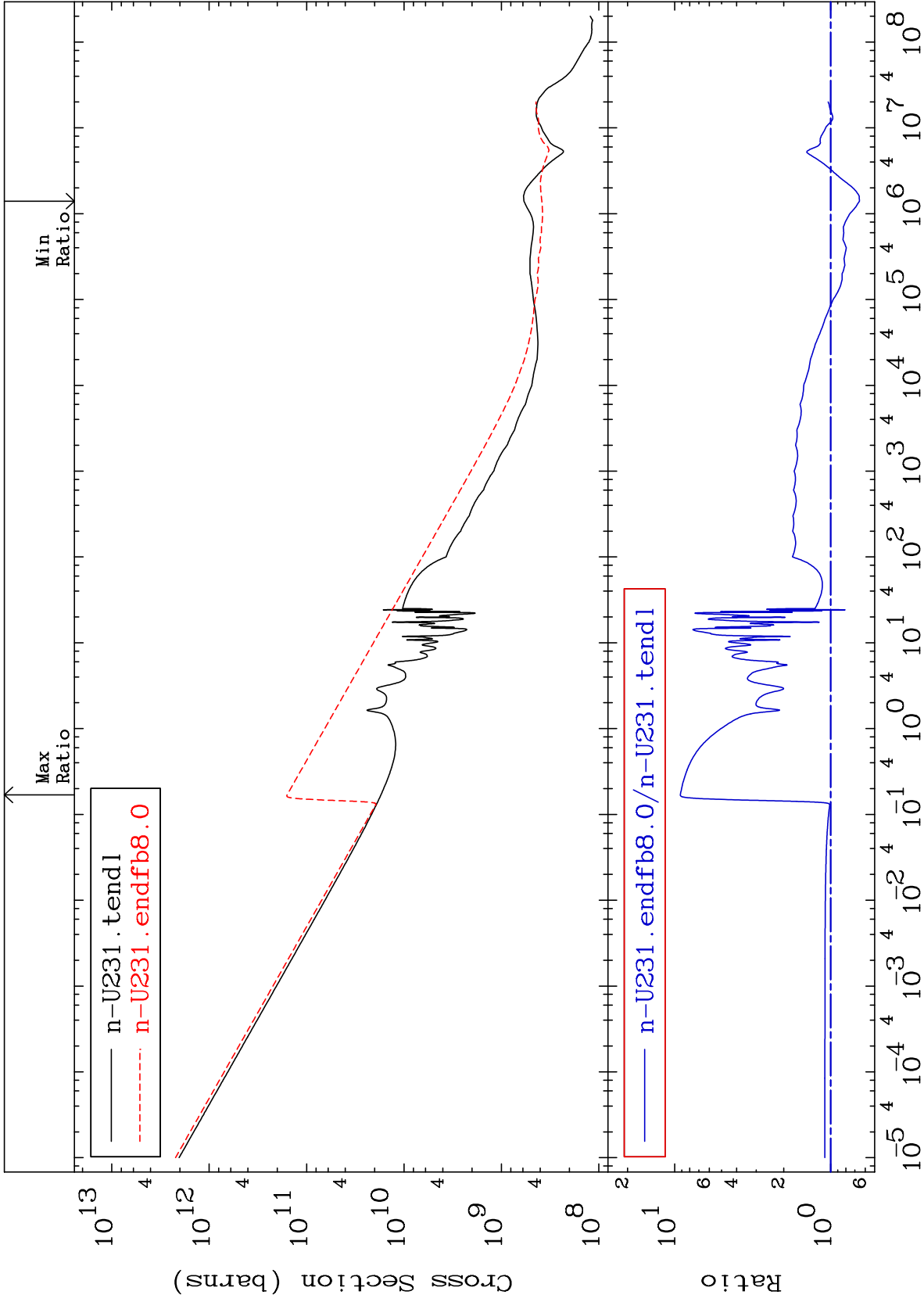
23

Incident Energy (eV)

92-U -231



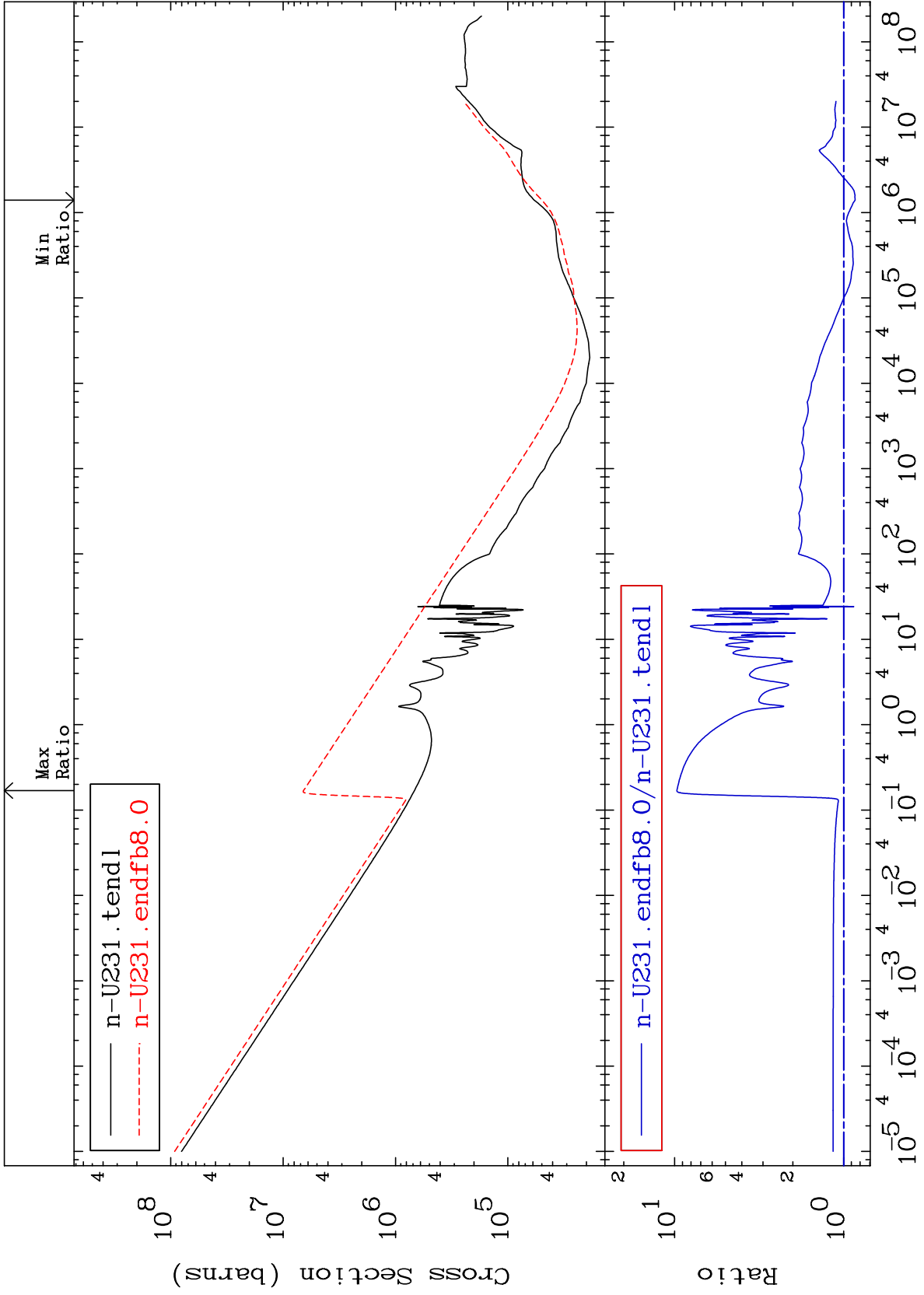




MAT 9216

Dpa total (eV-barns)
Cross Section

92-U -231
-14.47 To 869.7 %



27

Incident Energy (eV)

92-U -231

MAT 9216

Dpa elastic (mt2)
Cross Section

92-U -231
-36.82 To 9.633 %

