

Program EVALPLOT
(Version 2017-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

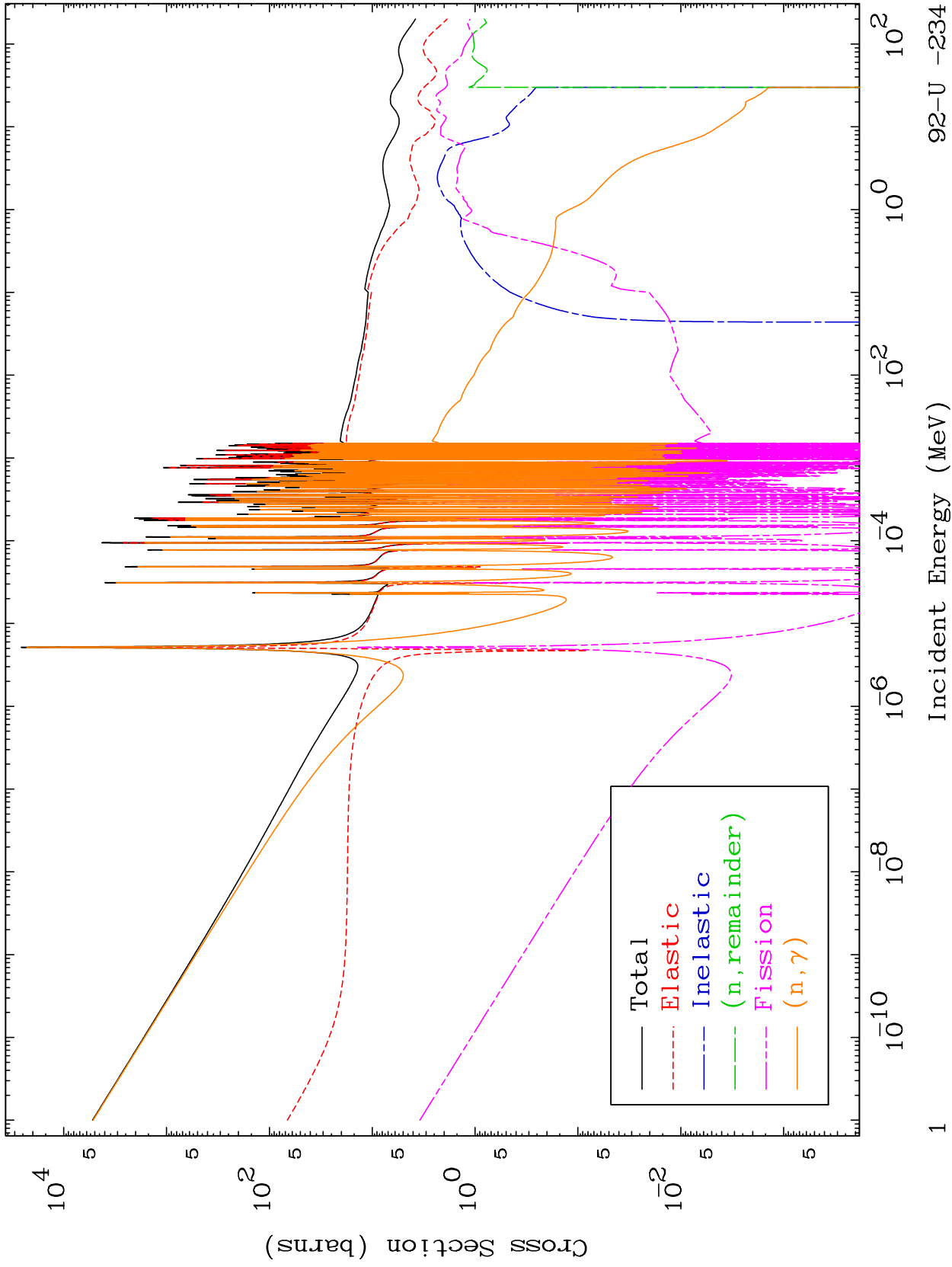
Press Mouse Button to Start

MAT 9225

Major

293 Kelvin Cross Sections

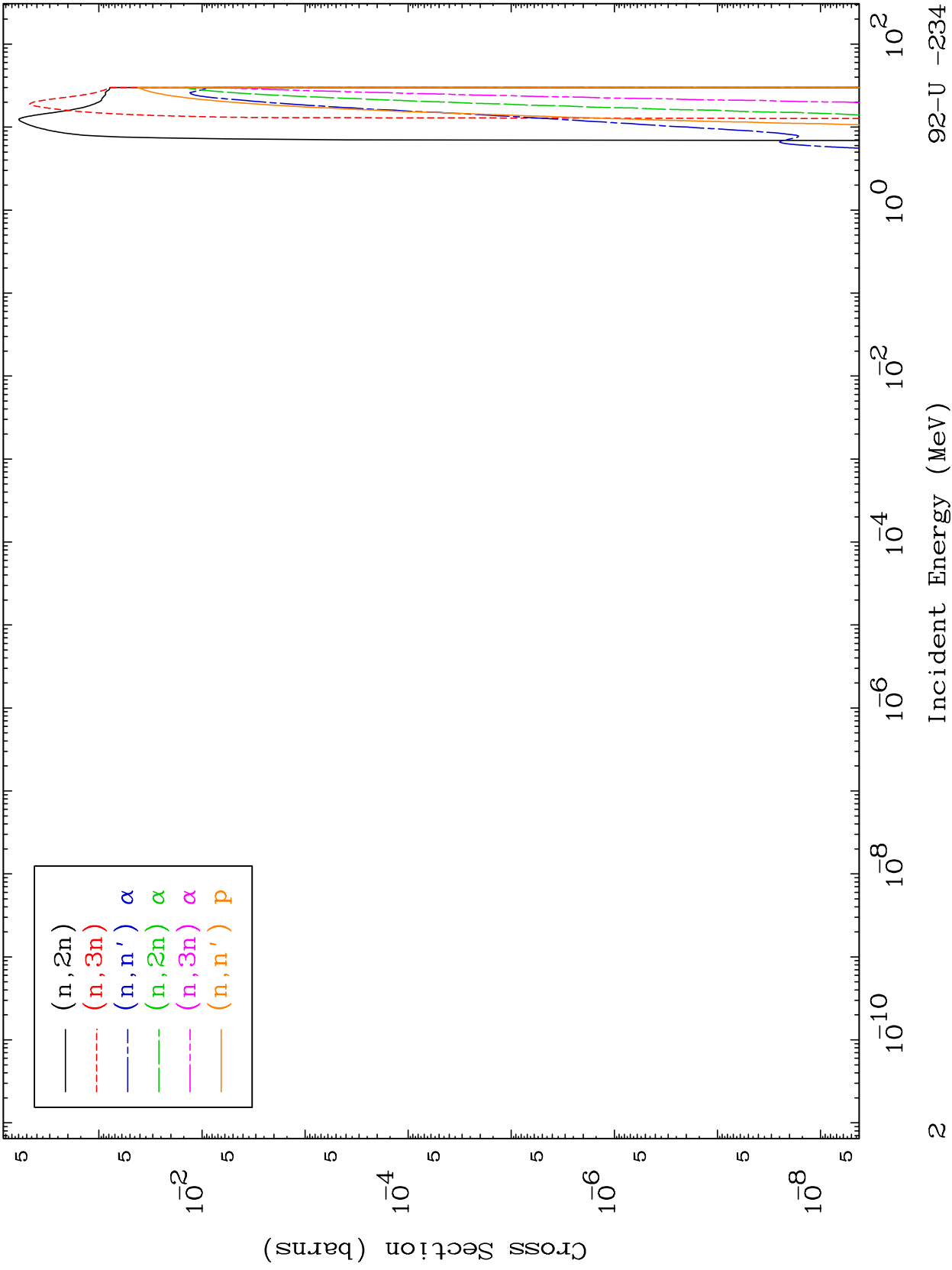
92-U -234



MAT 9225

Neutron Production
293 Kelvin Cross Sections

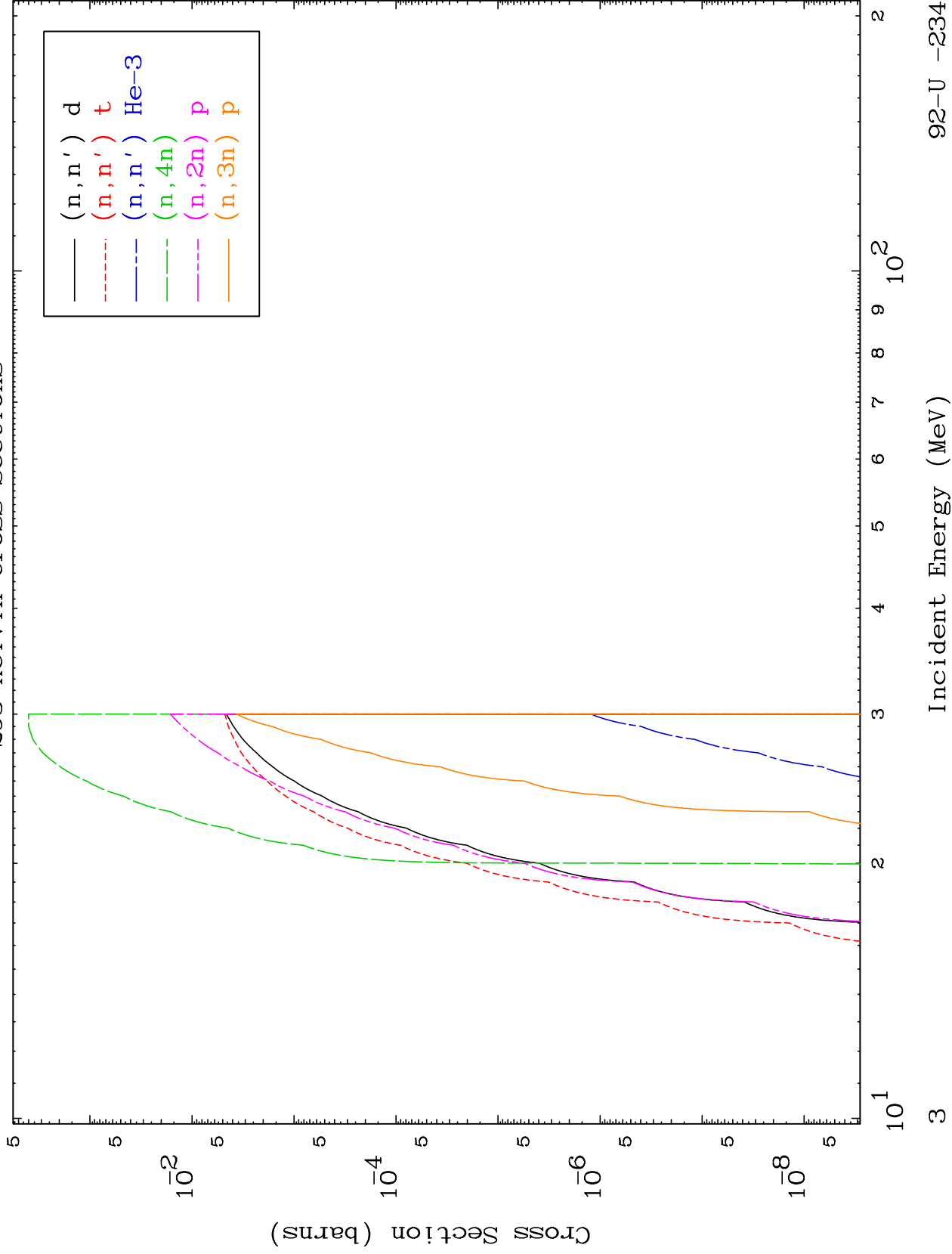
92-U -234



MAT 9225

Neutron Production
293 Kelvin Cross Sections

92-U -234



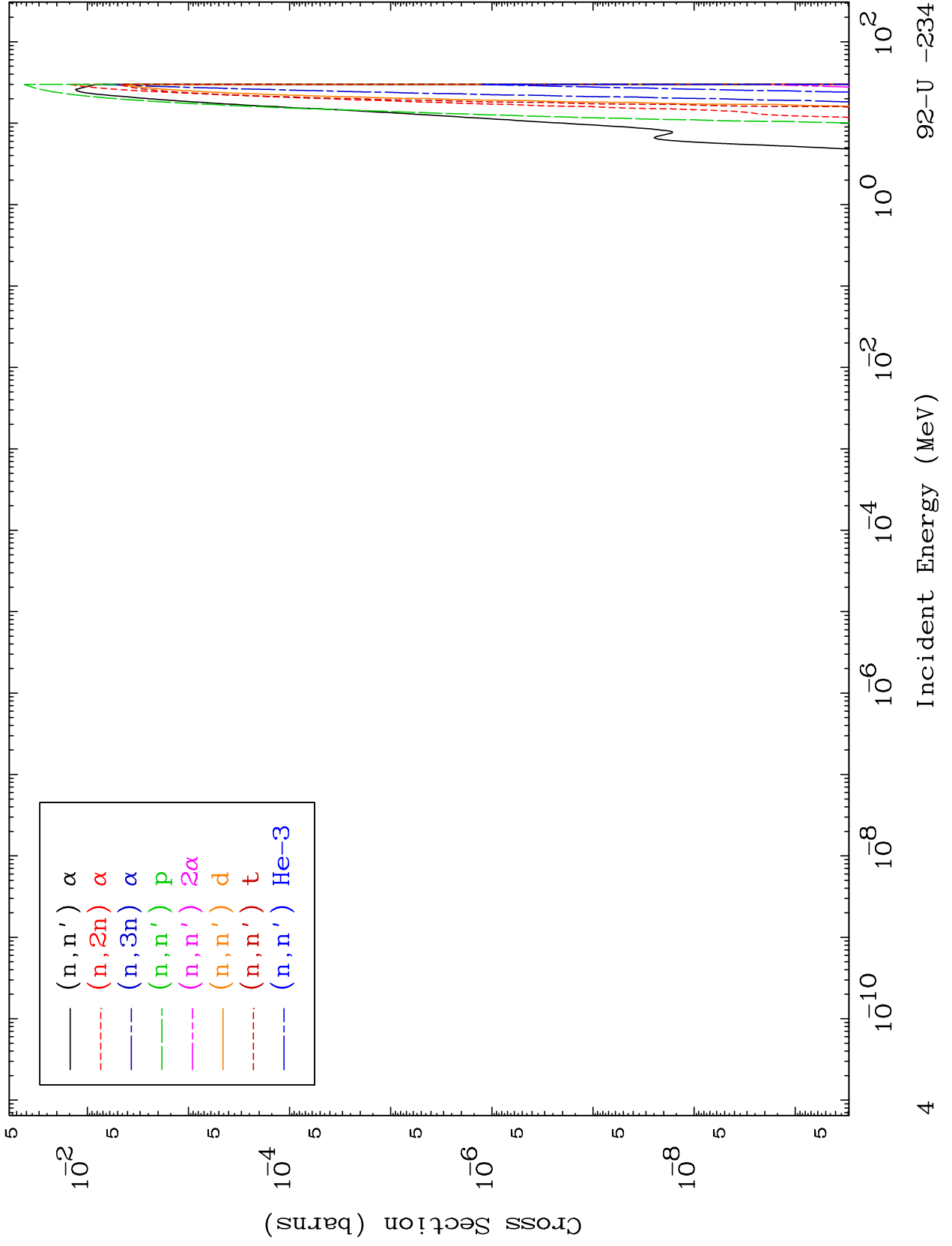
92-U -234

Incident Energy (MeV)

MAT 9225

Charged Particle
293 Kelvin Cross Sections

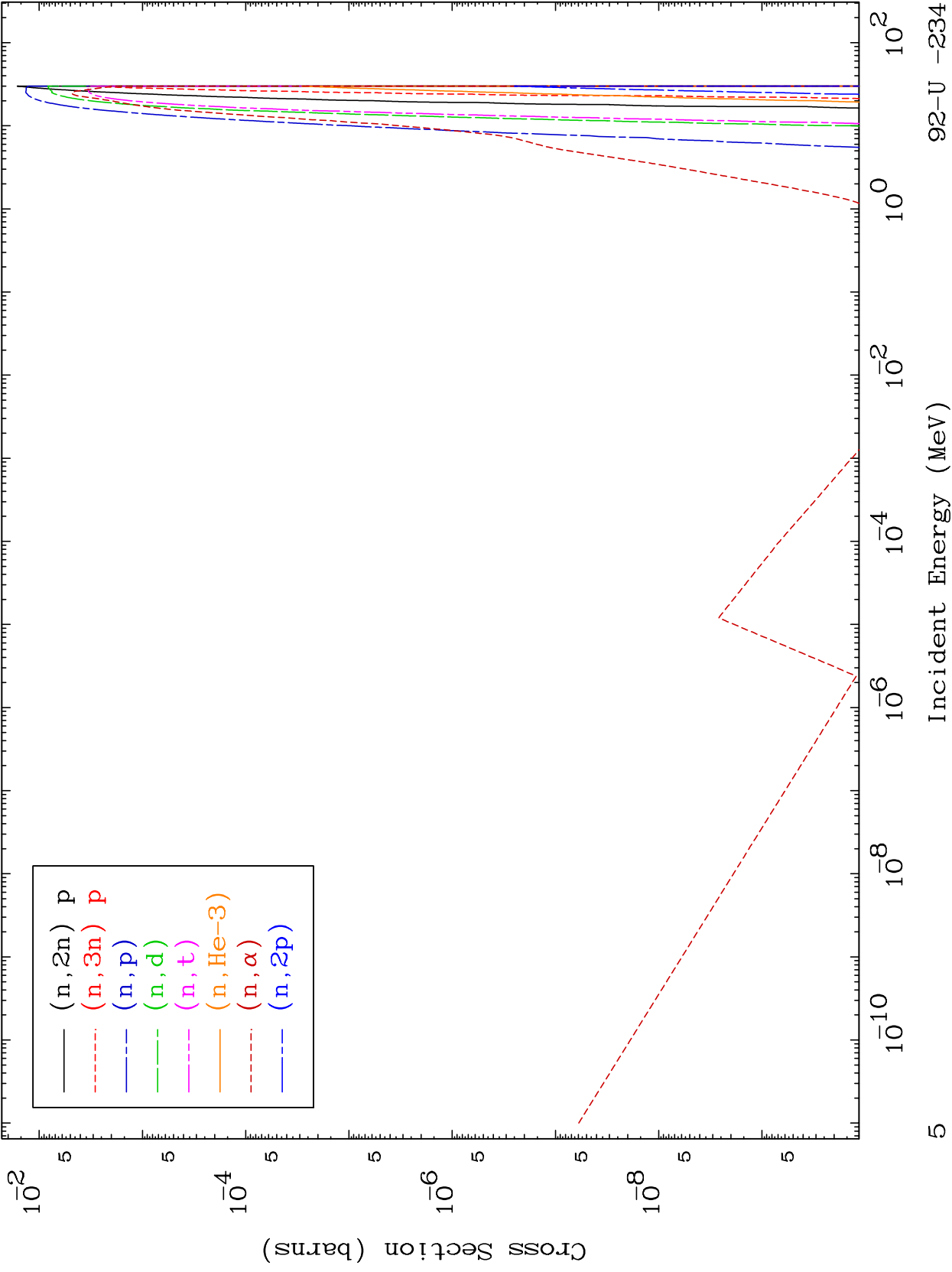
92-U -234



MAT 9225

Charged Particle
293 Kelvin Cross Sections

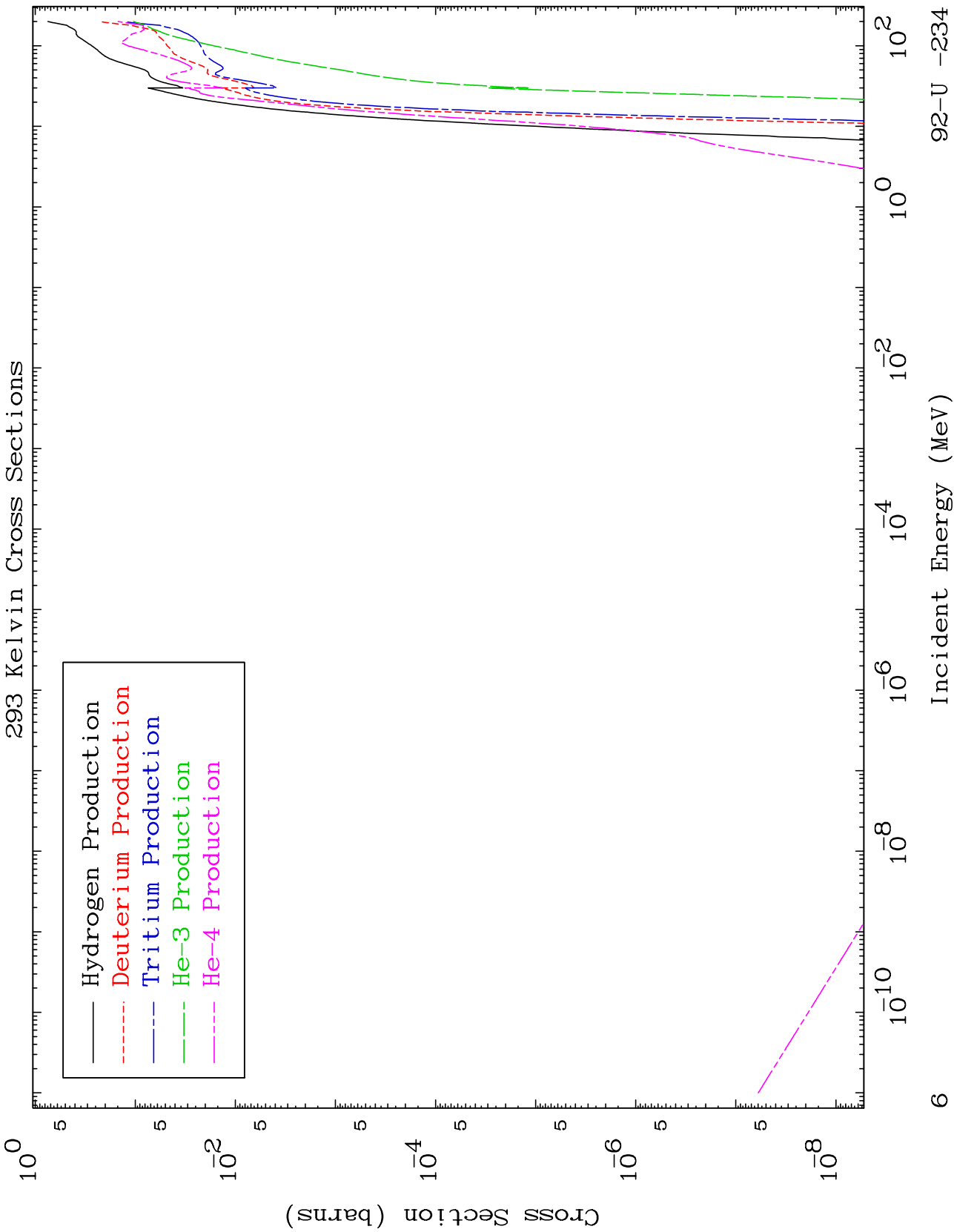
92-U -234



MAT 9225

Particle Production
293 Kelvin Cross Sections

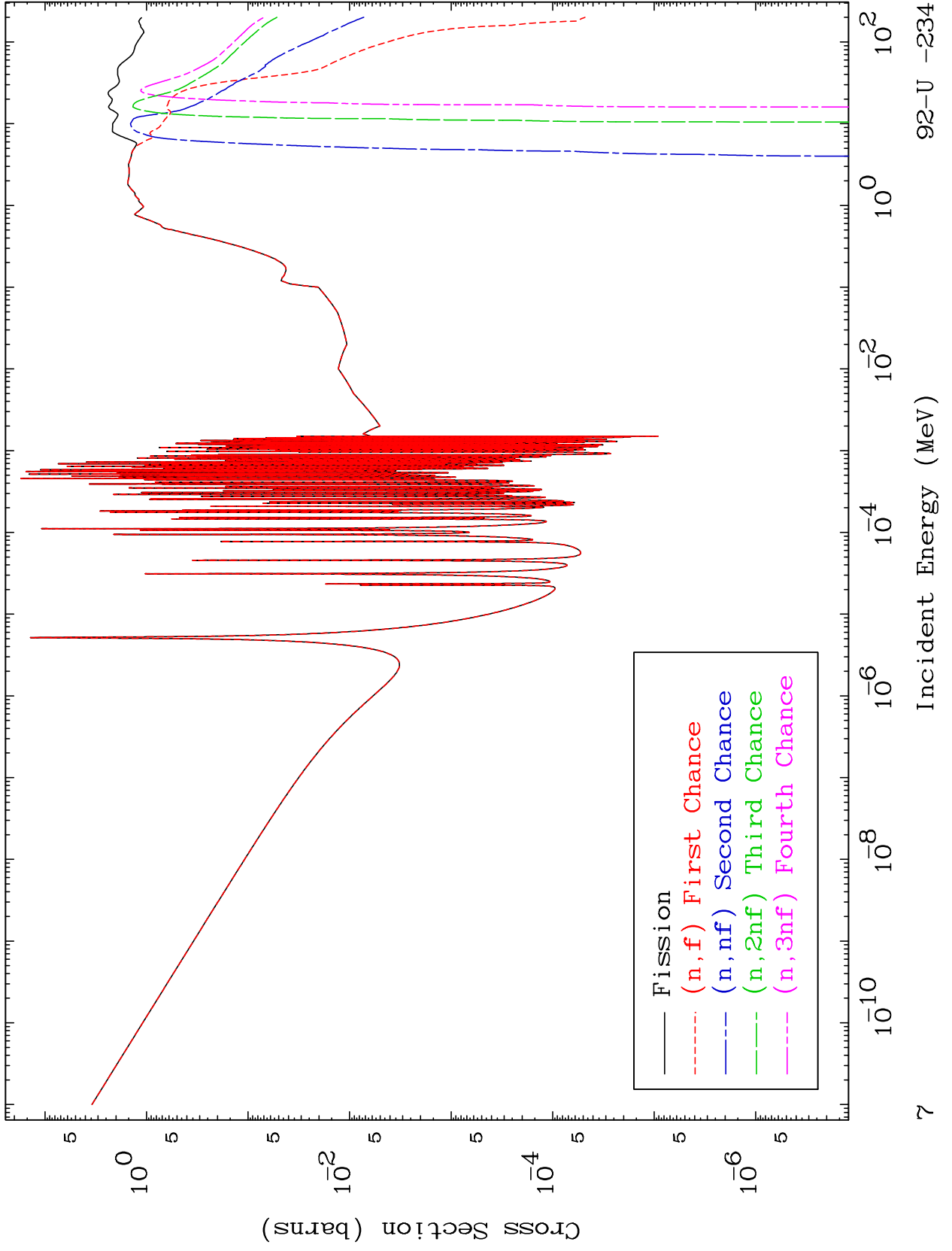
92-U -234



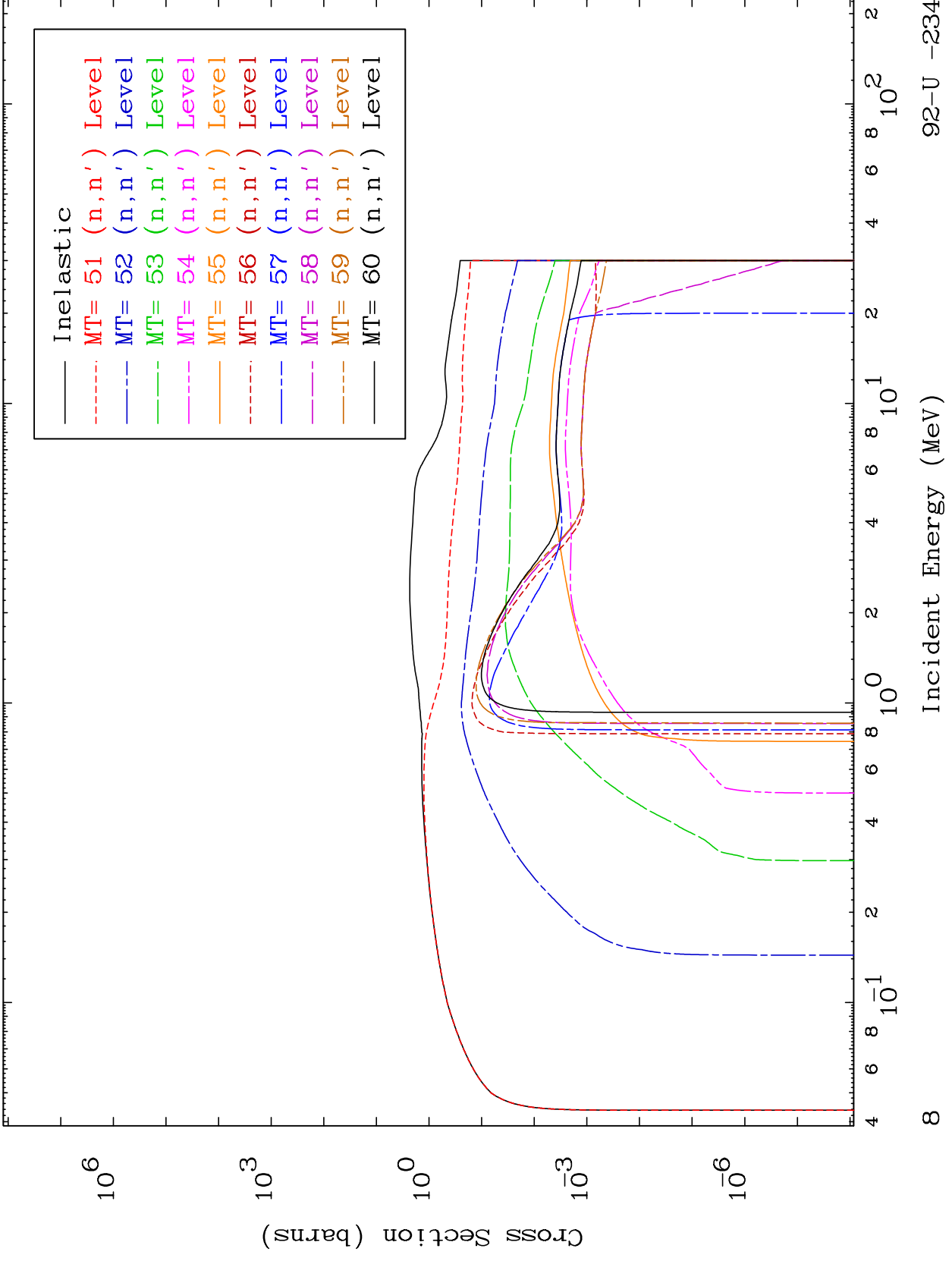
MAT 9225

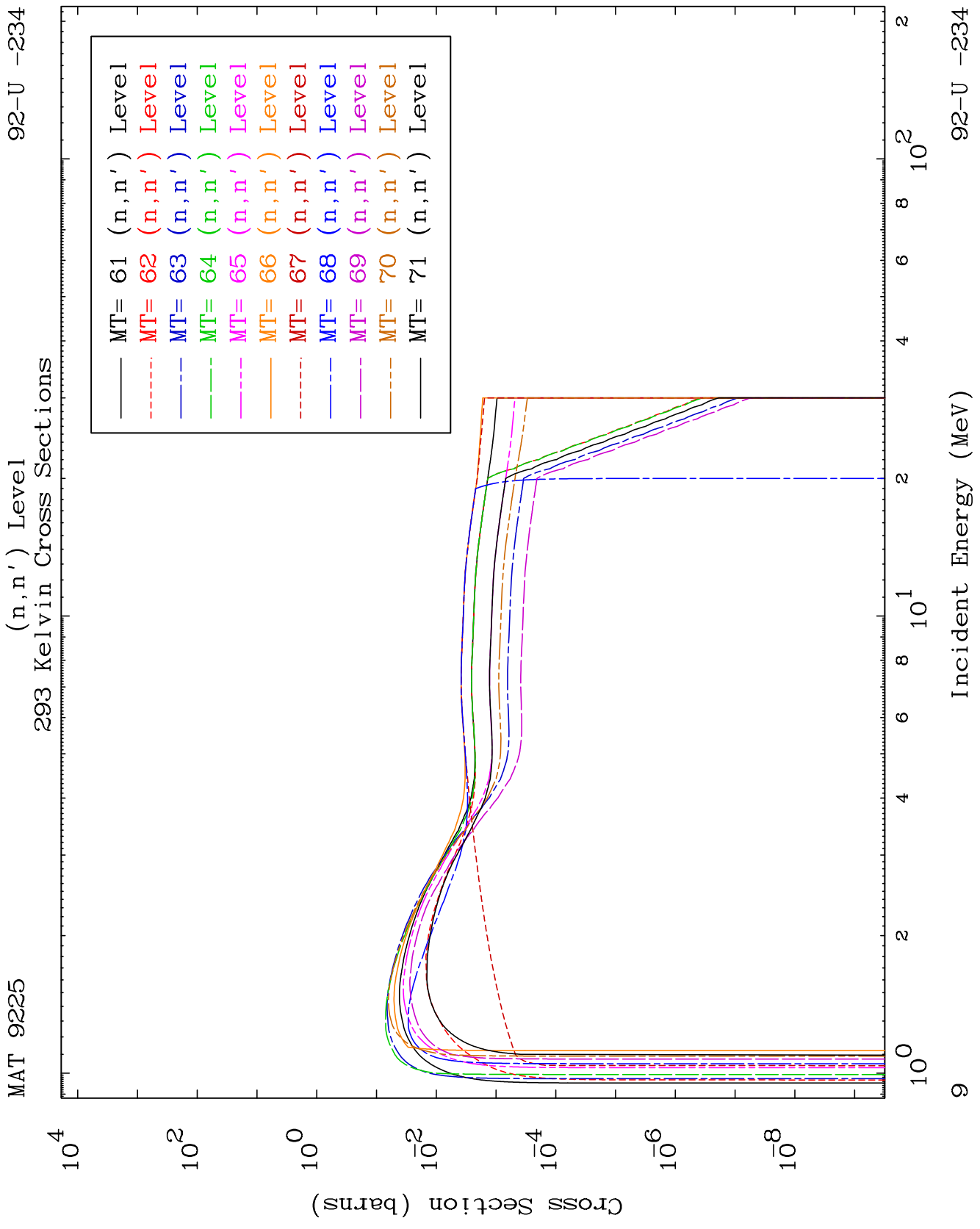
Fission
293 Kelvin Cross Sections

92-U -234



293 Kelvin Cross Sections



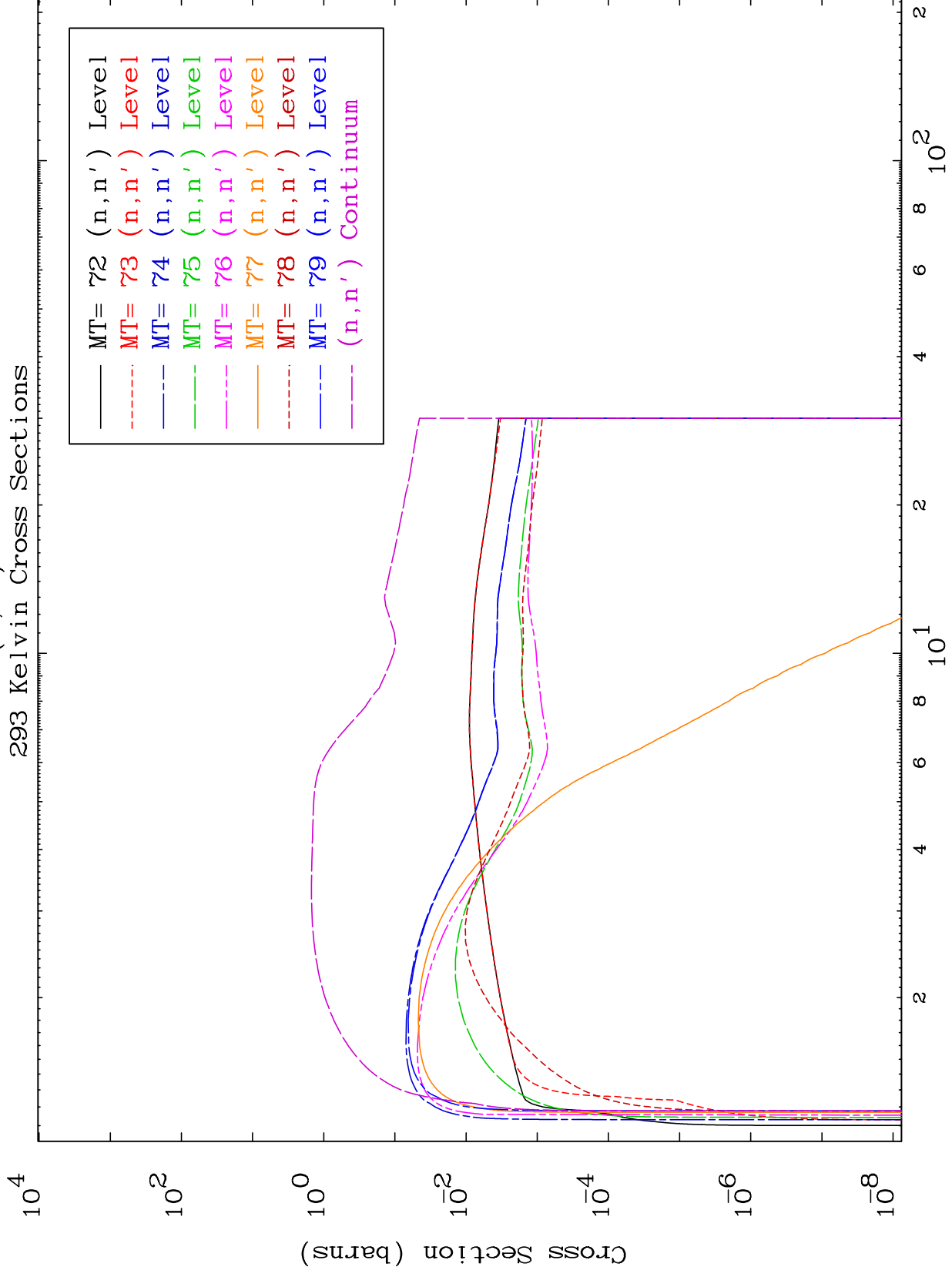


MAT 9225

(n,n') Level

293 Kelvin Cross Sections

92-U -234



10

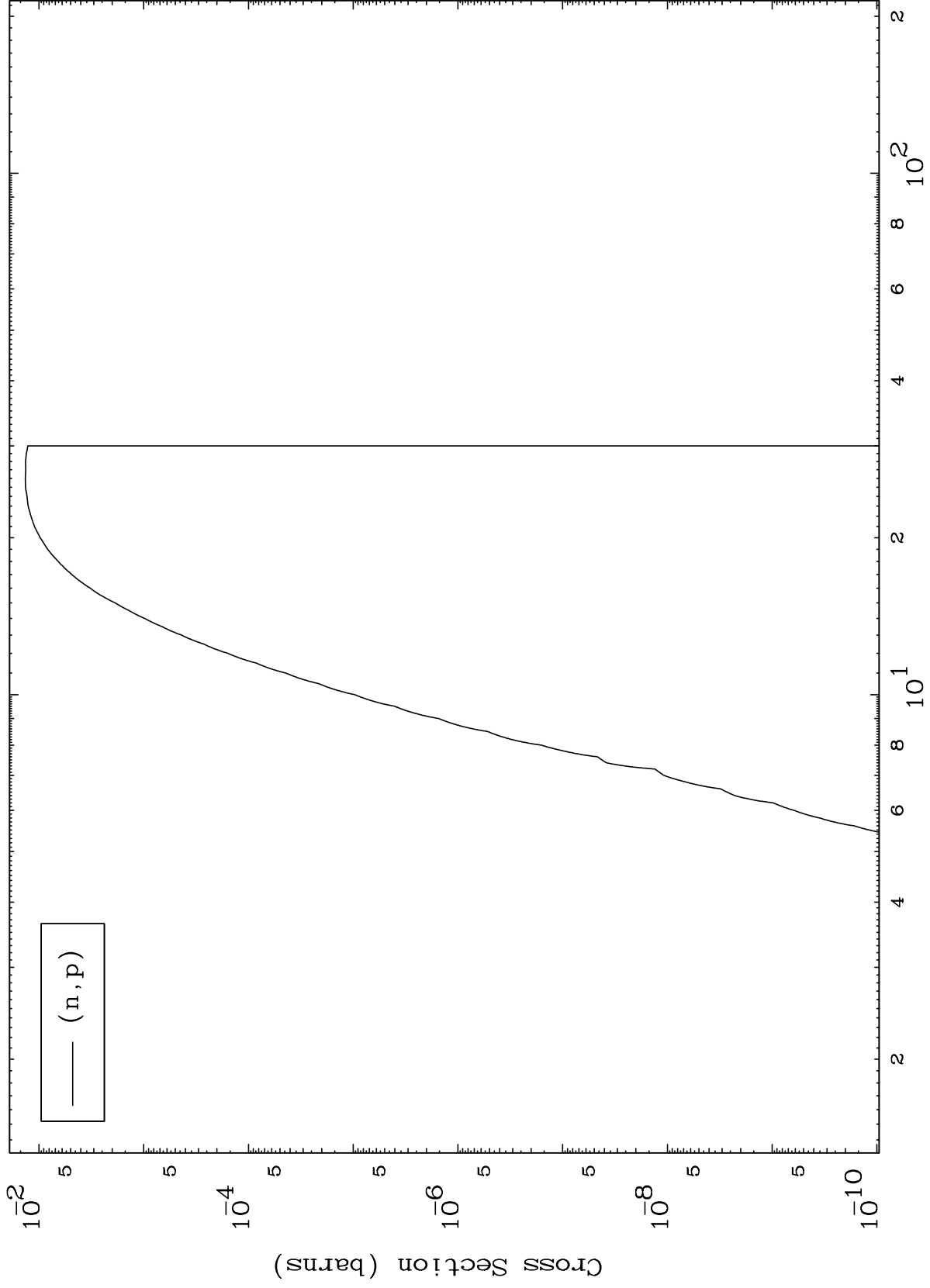
Incident Energy (MeV)

92-U -234

MAT 9225

(n,p) Levels
293 Kelvin Cross Sections

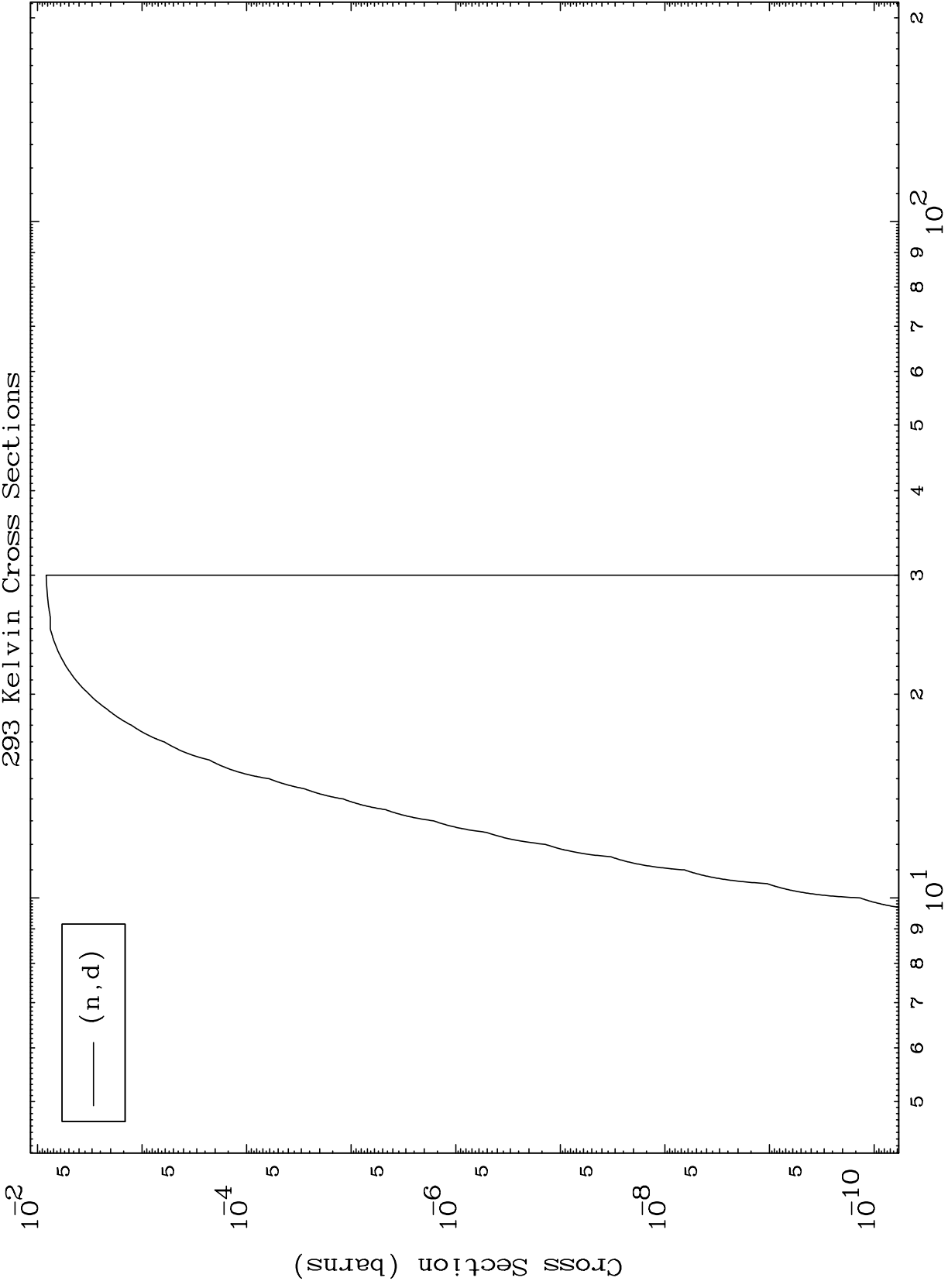
92-U -234



MAT 9225

(n,d) Levels
293 Kelvin Cross Sections

92-U -234



12

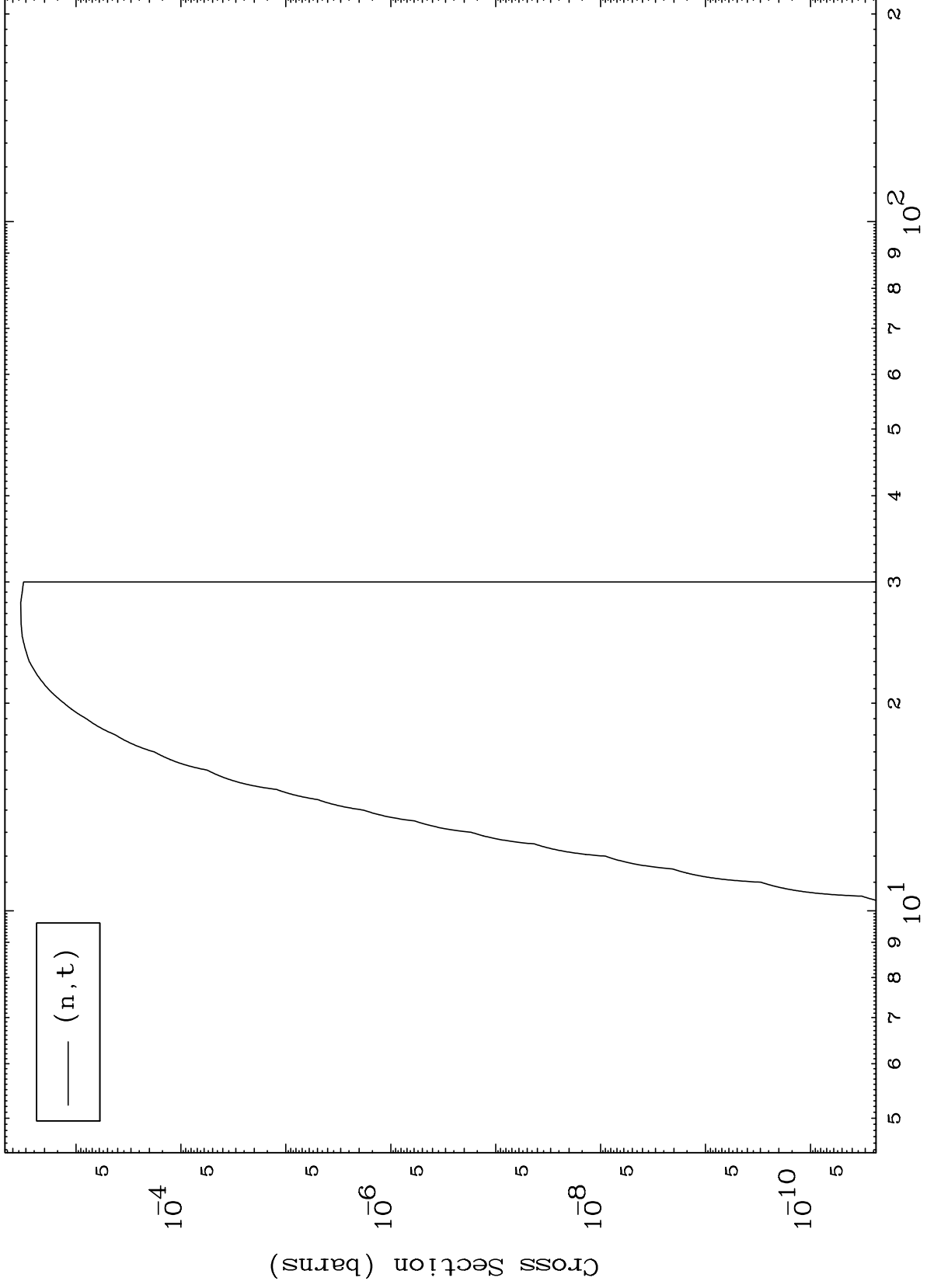
Incident Energy (MeV)

92-U -234

MAT 9225

(n,t) Levels
293 Kelvin Cross Sections

92-U -234



13

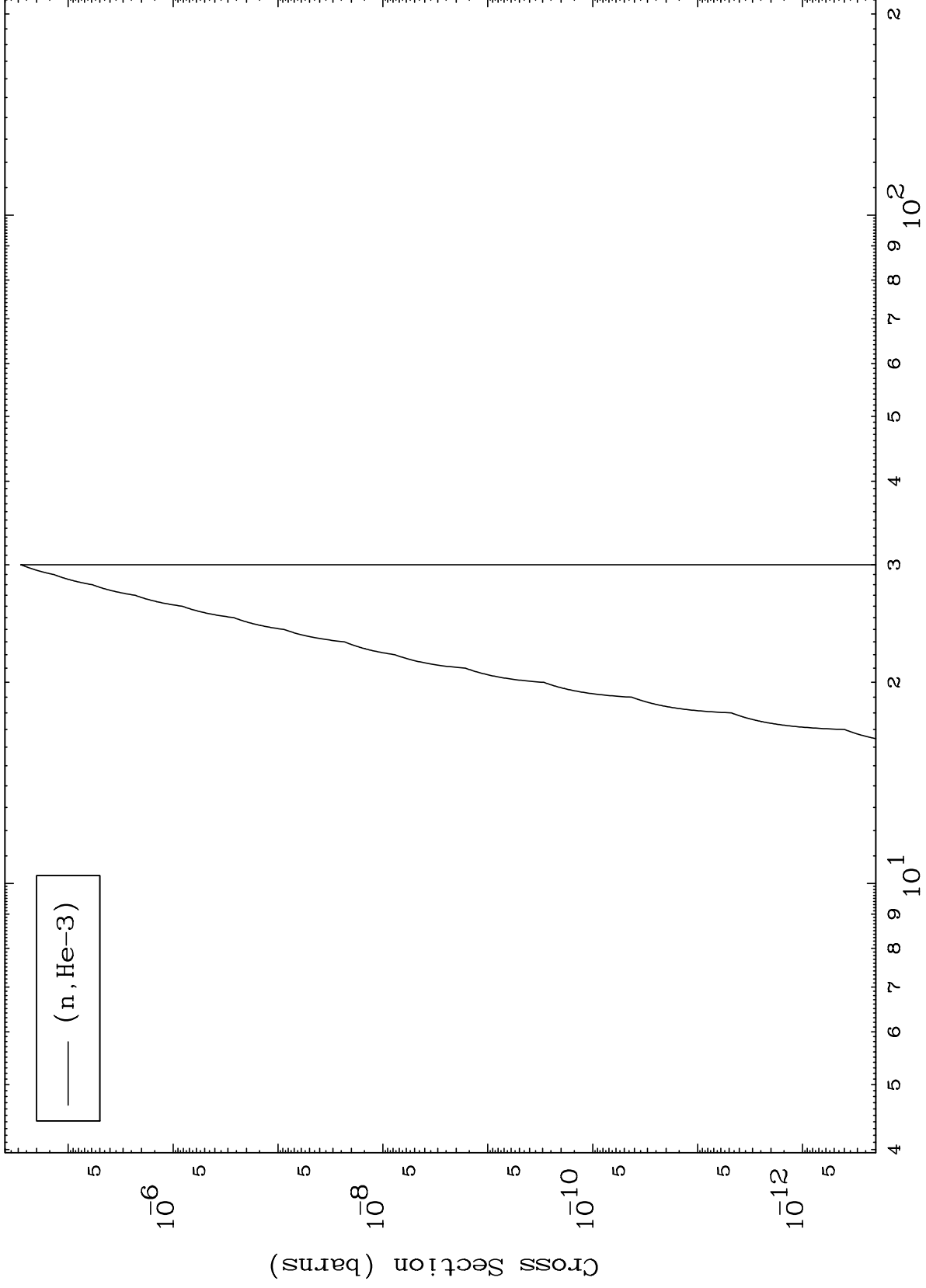
Incident Energy (MeV)

92-U -234

MAT 9225

(n,He3) Levels
293 Kelvin Cross Sections

92-U -234



14

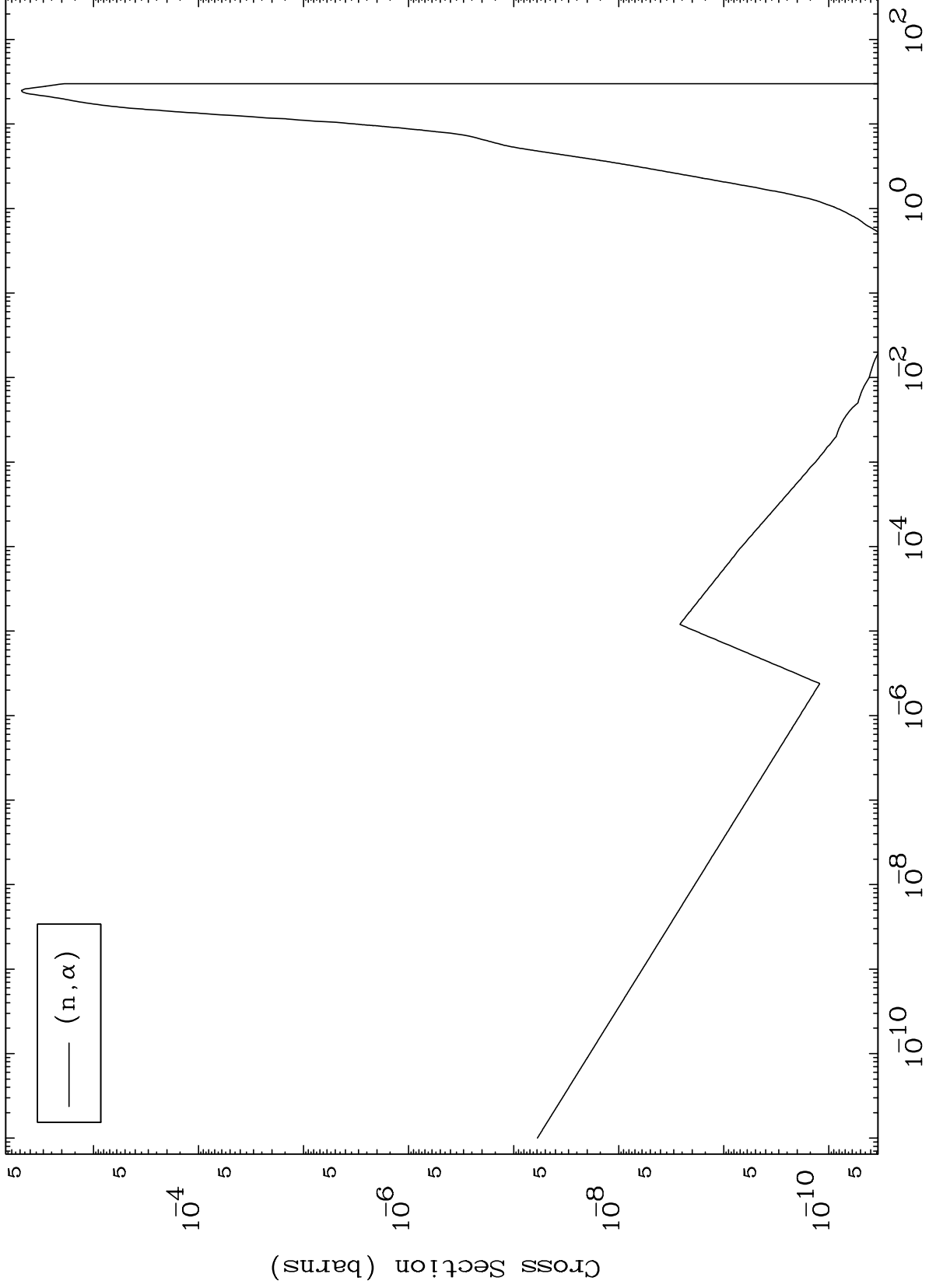
Incident Energy (MeV)

92-U -234

MAT 9225

(n, α) Levels
293 Kelvin Cross Sections

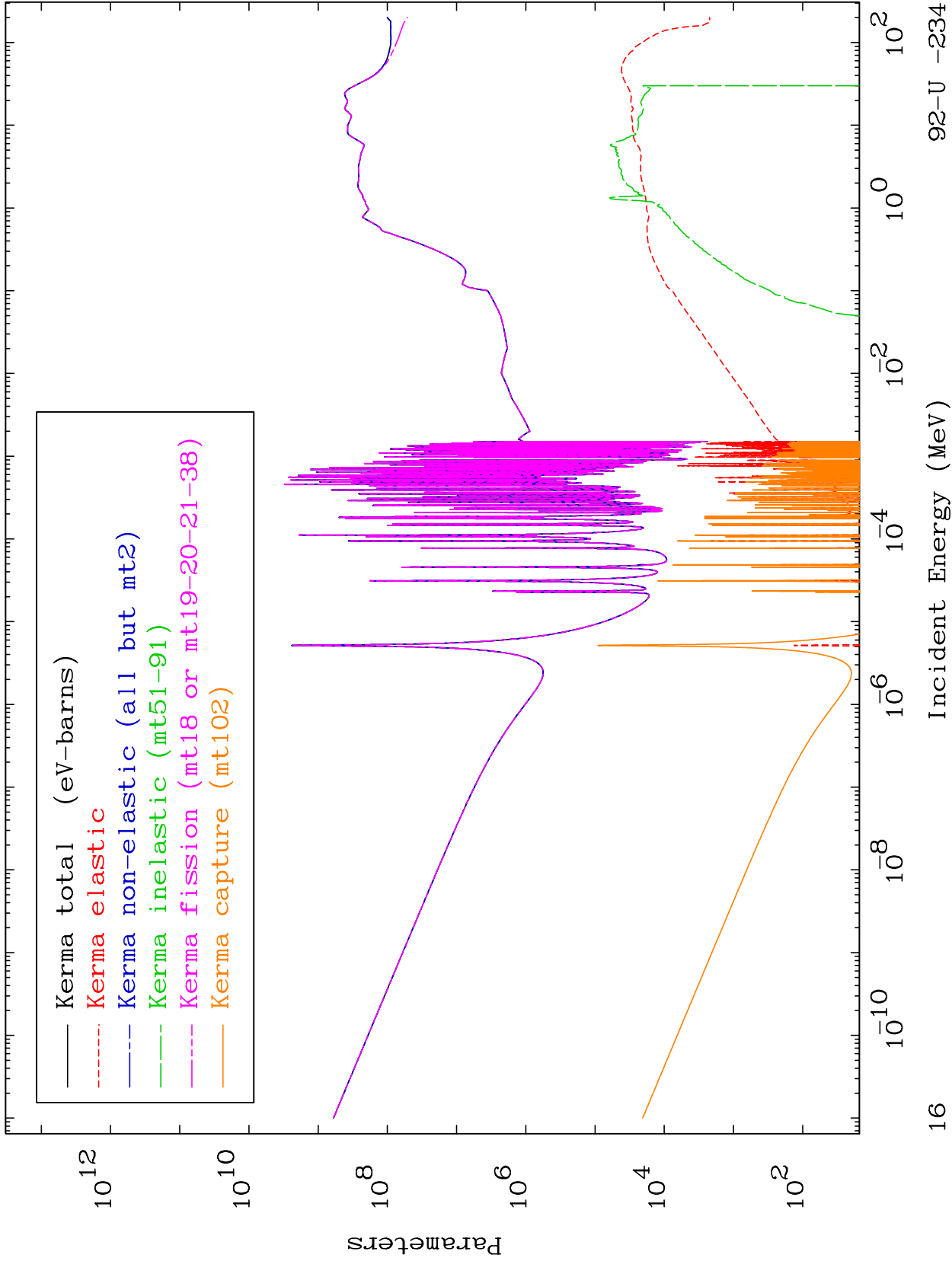
92-U -234

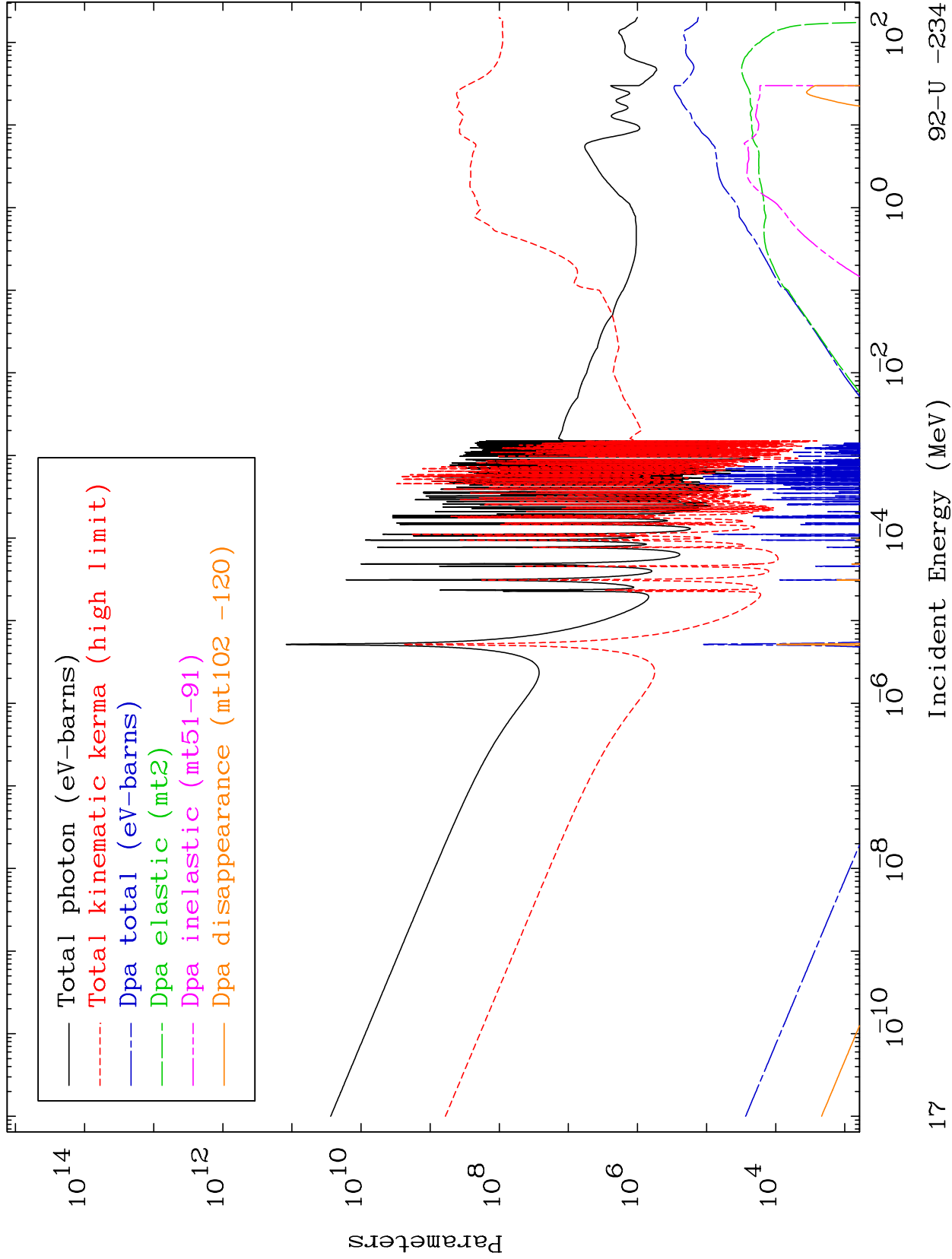


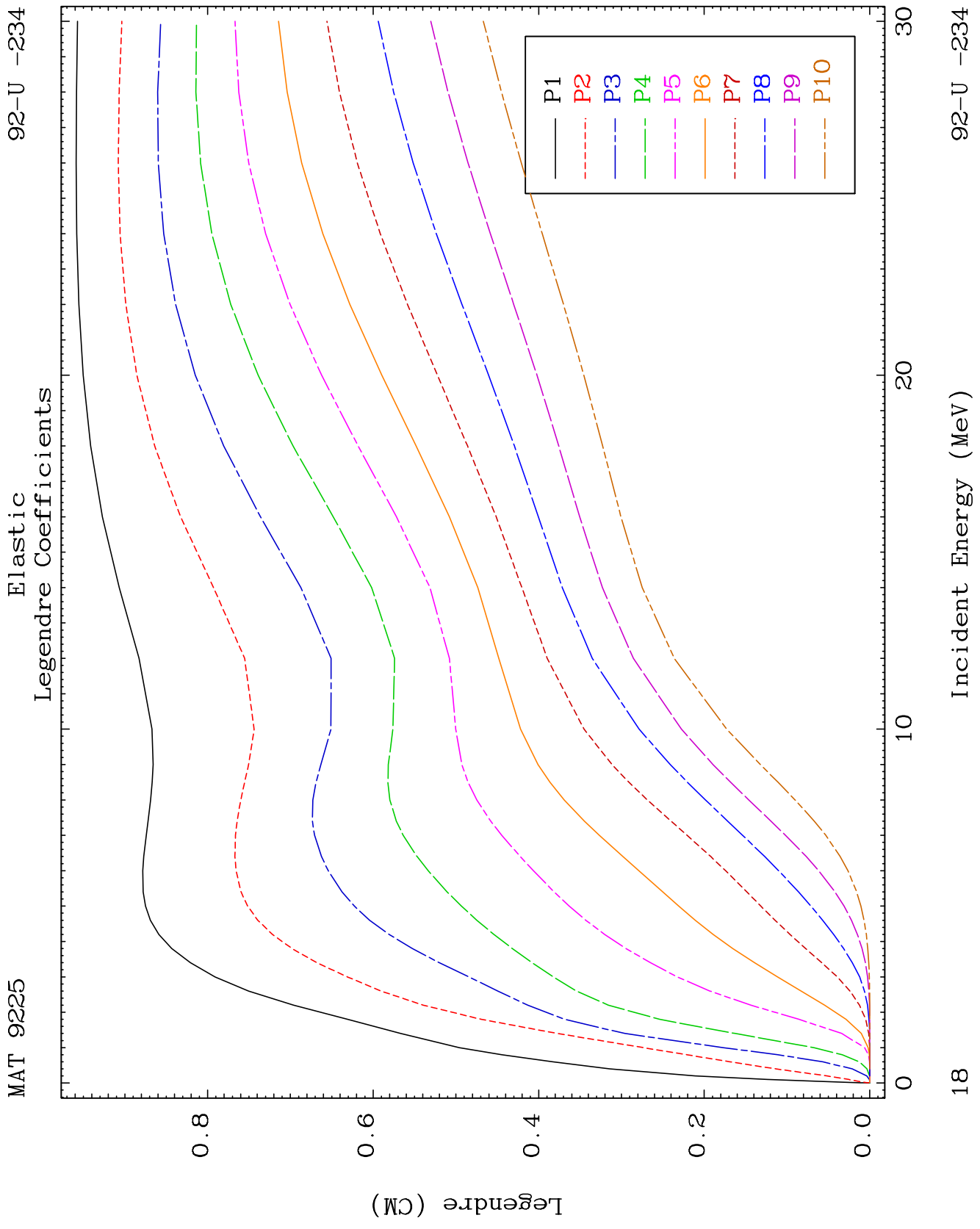
15

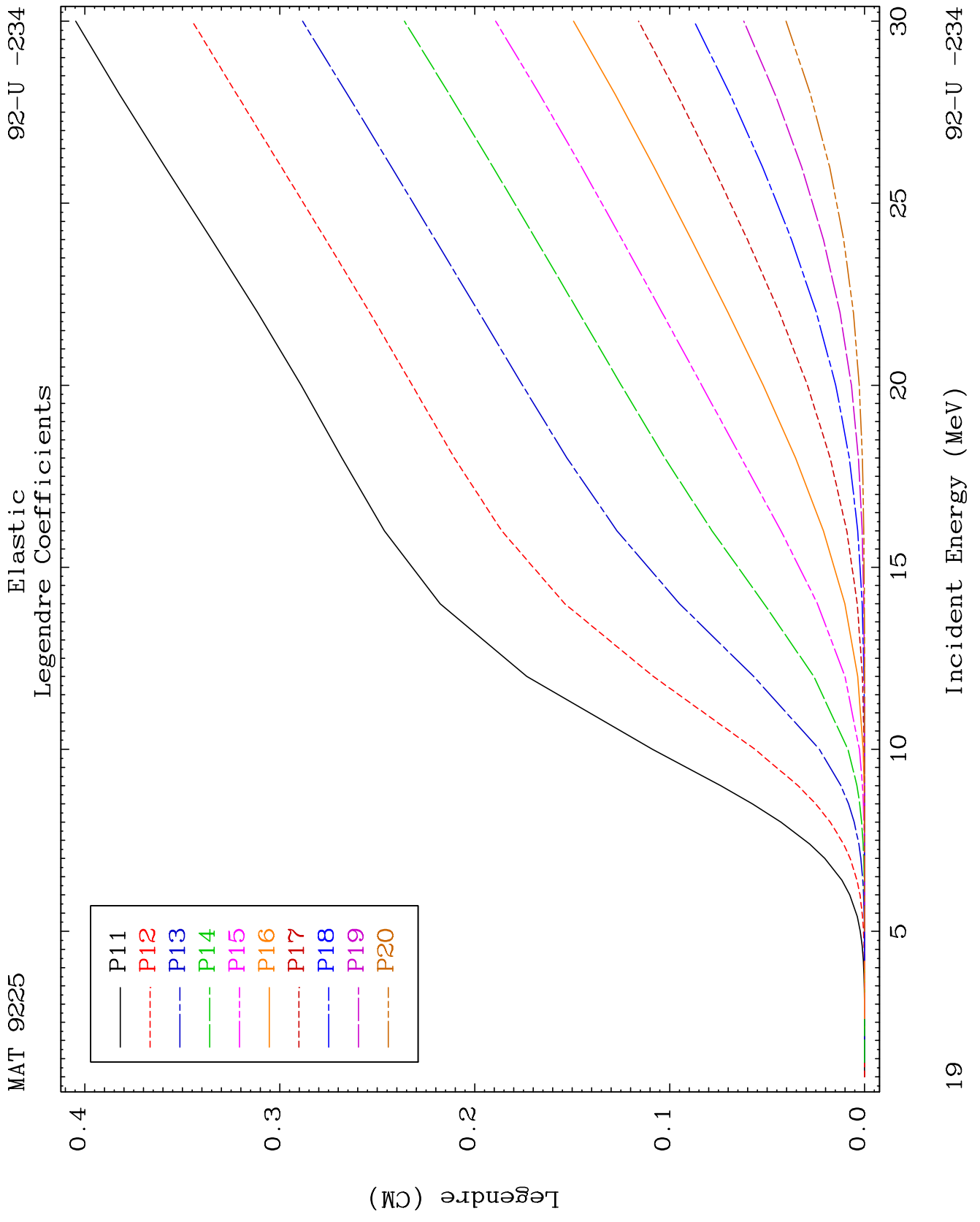
Incident Energy (MeV)

92-U -234









MAT 9225

Elastic Legendre Coefficients

92-U -234

19

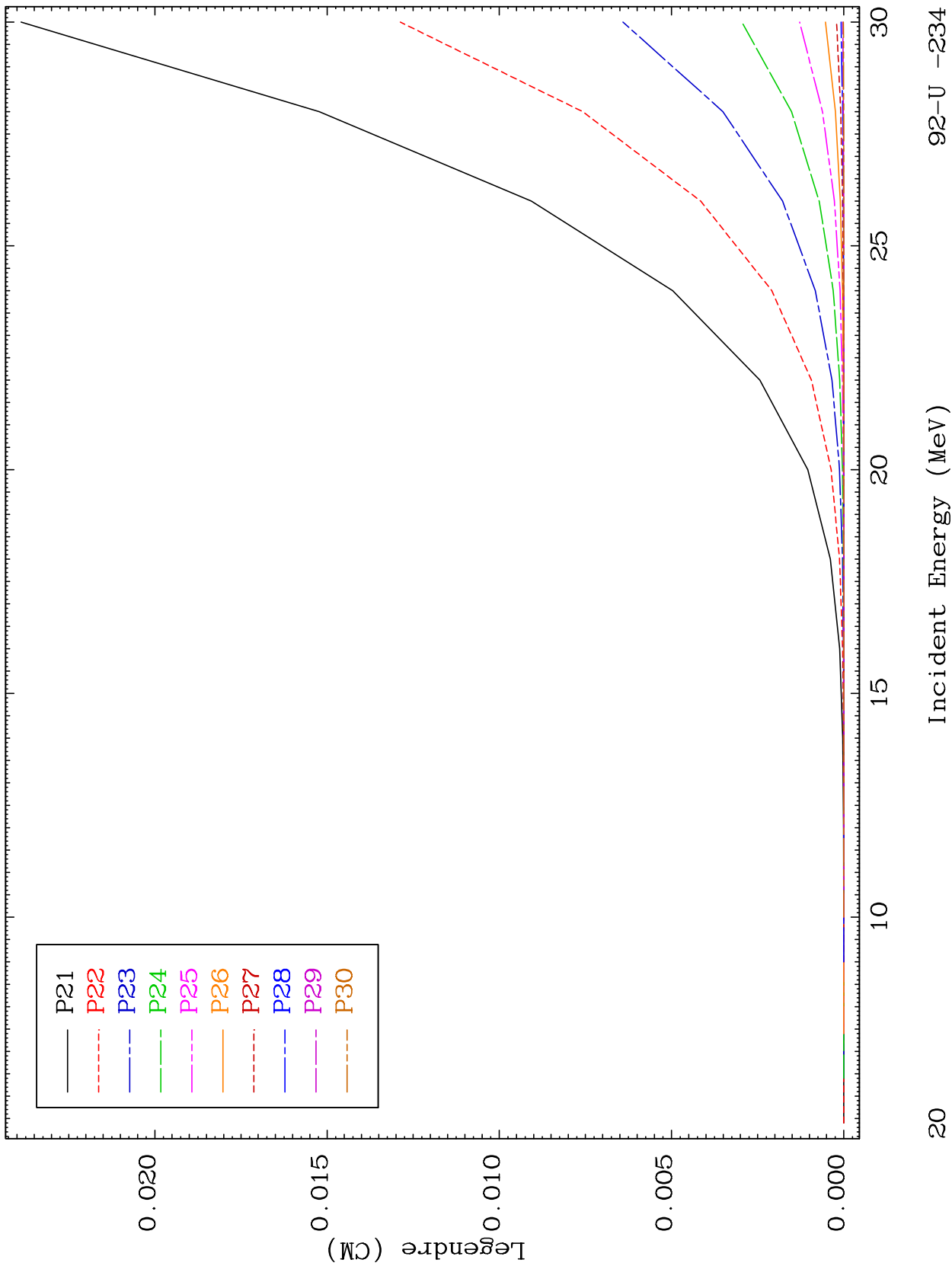
Incident Energy (MeV)

92-U -234

MAT 9225

Elastic Legendre Coefficients

92-U -234



20

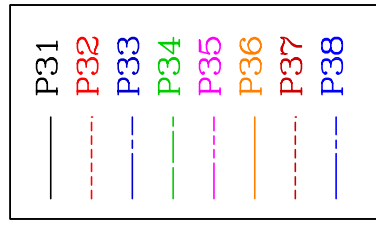
Incident Energy (MeV)

92-U -234

MAT 9225

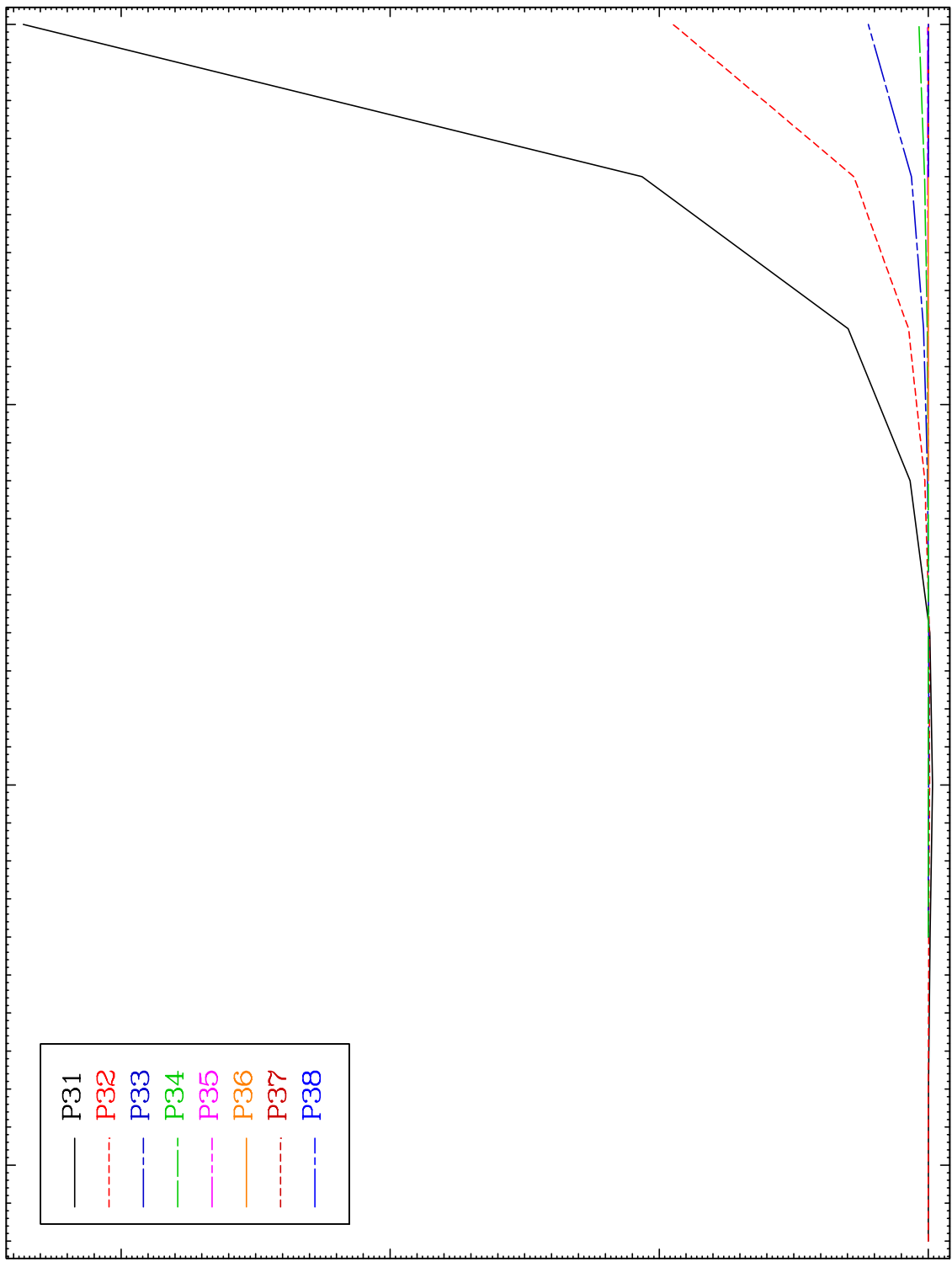
Elastic Legendre Coefficients

92-U -234



$\times 10^{-6}$

Legendre (CM)

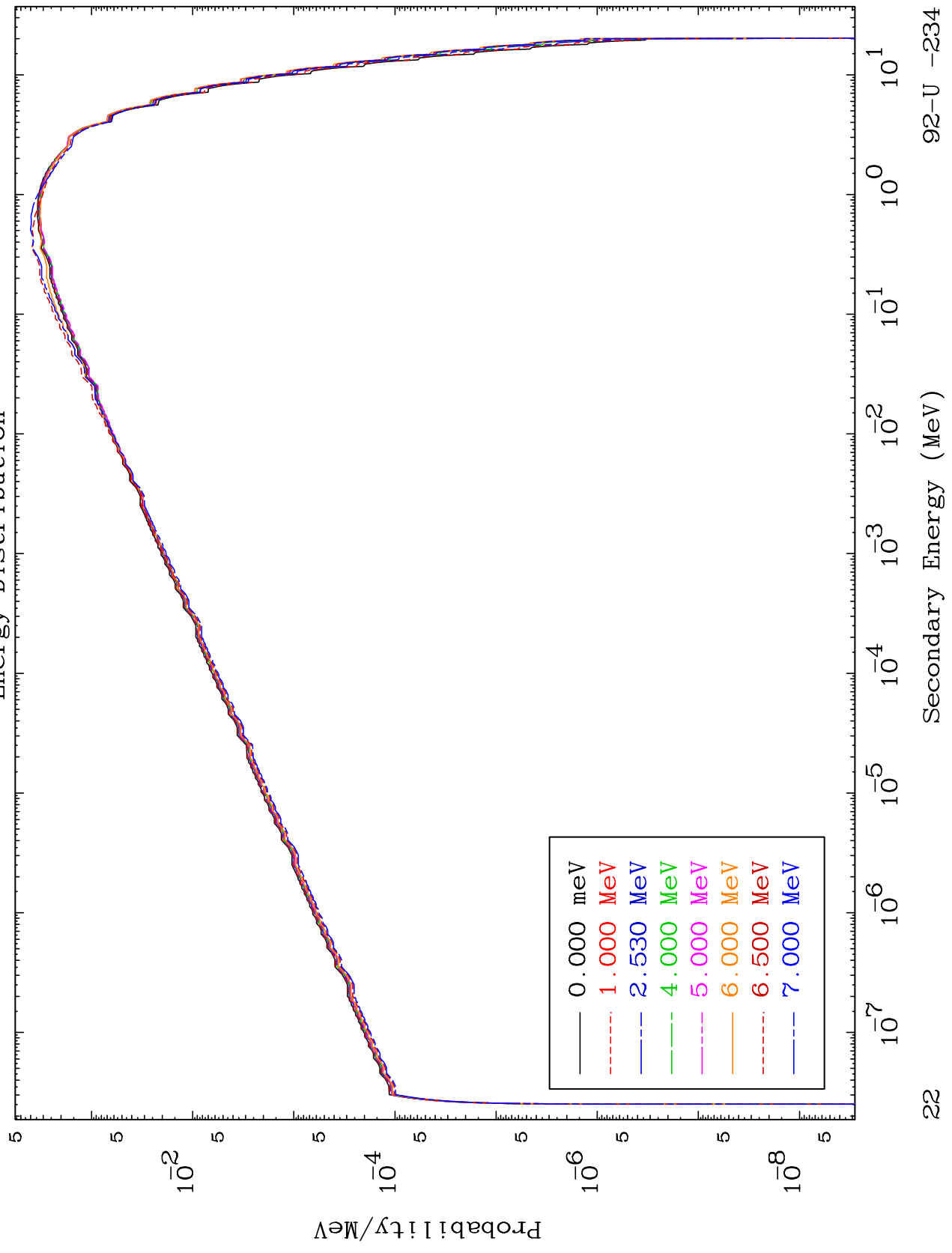


21

Incident Energy (MeV)

92-U -234

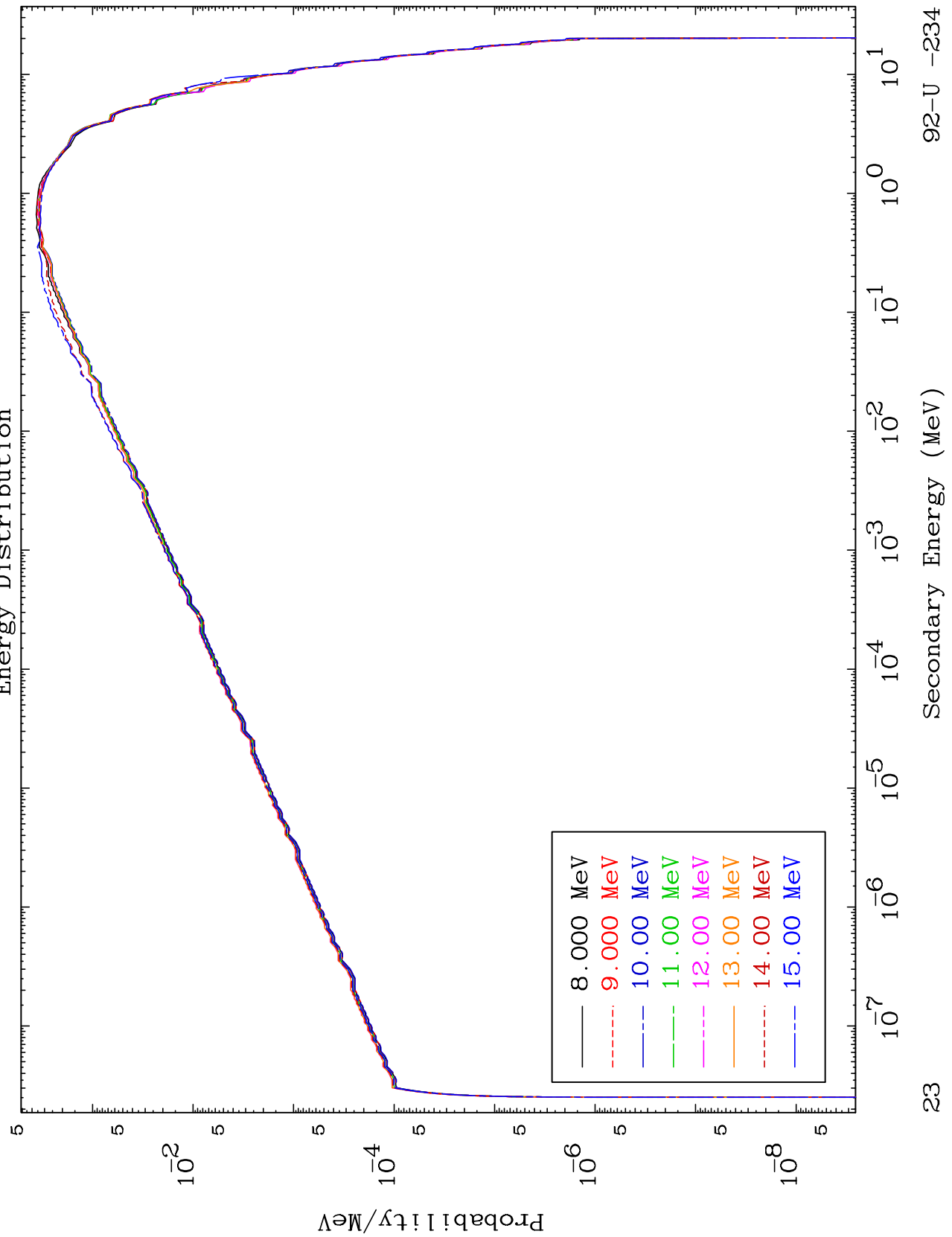
Fission Energy Distribution



MAT 9225

Fission Energy Distribution

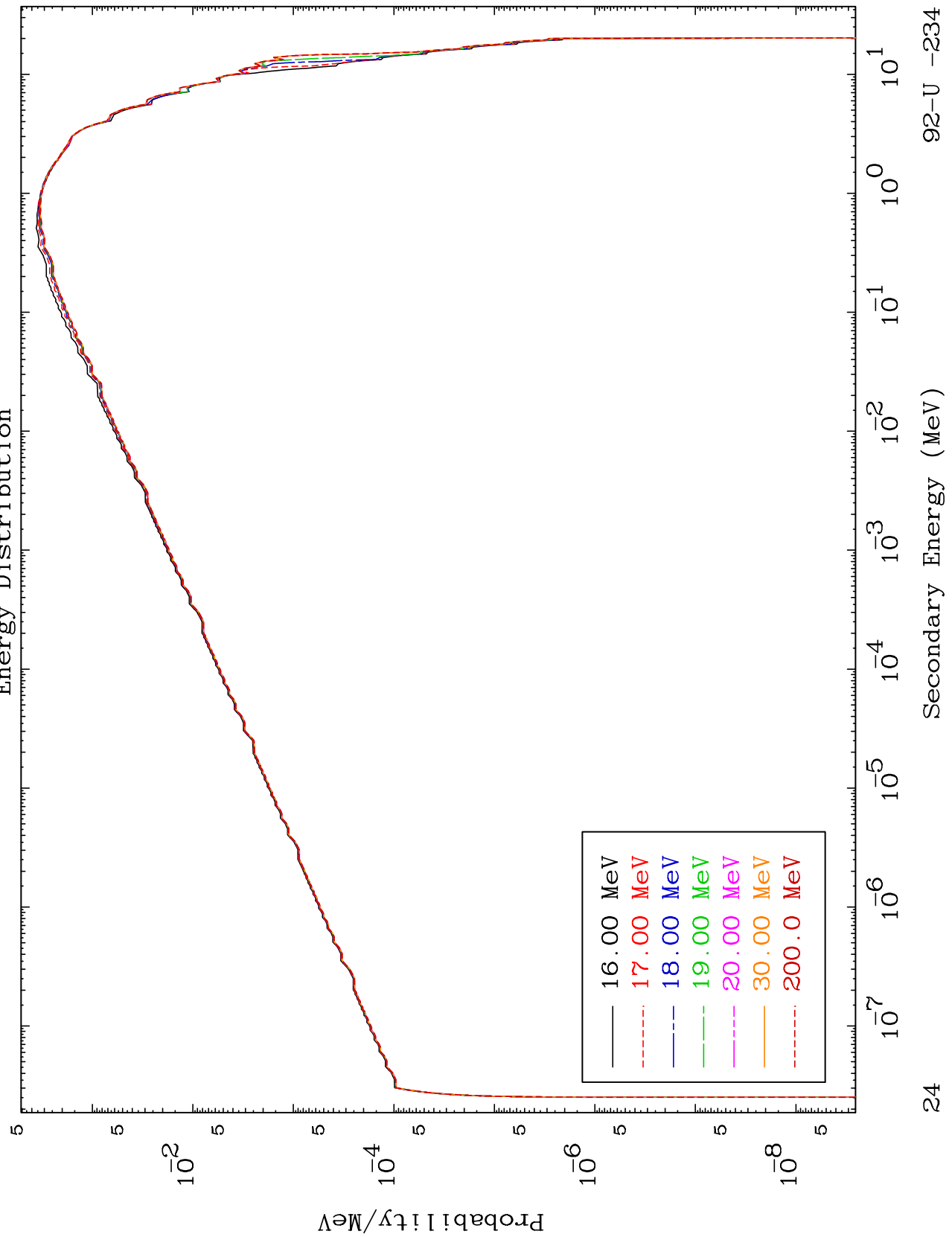
92-U -234



MAT 9225

Fission Energy Distribution

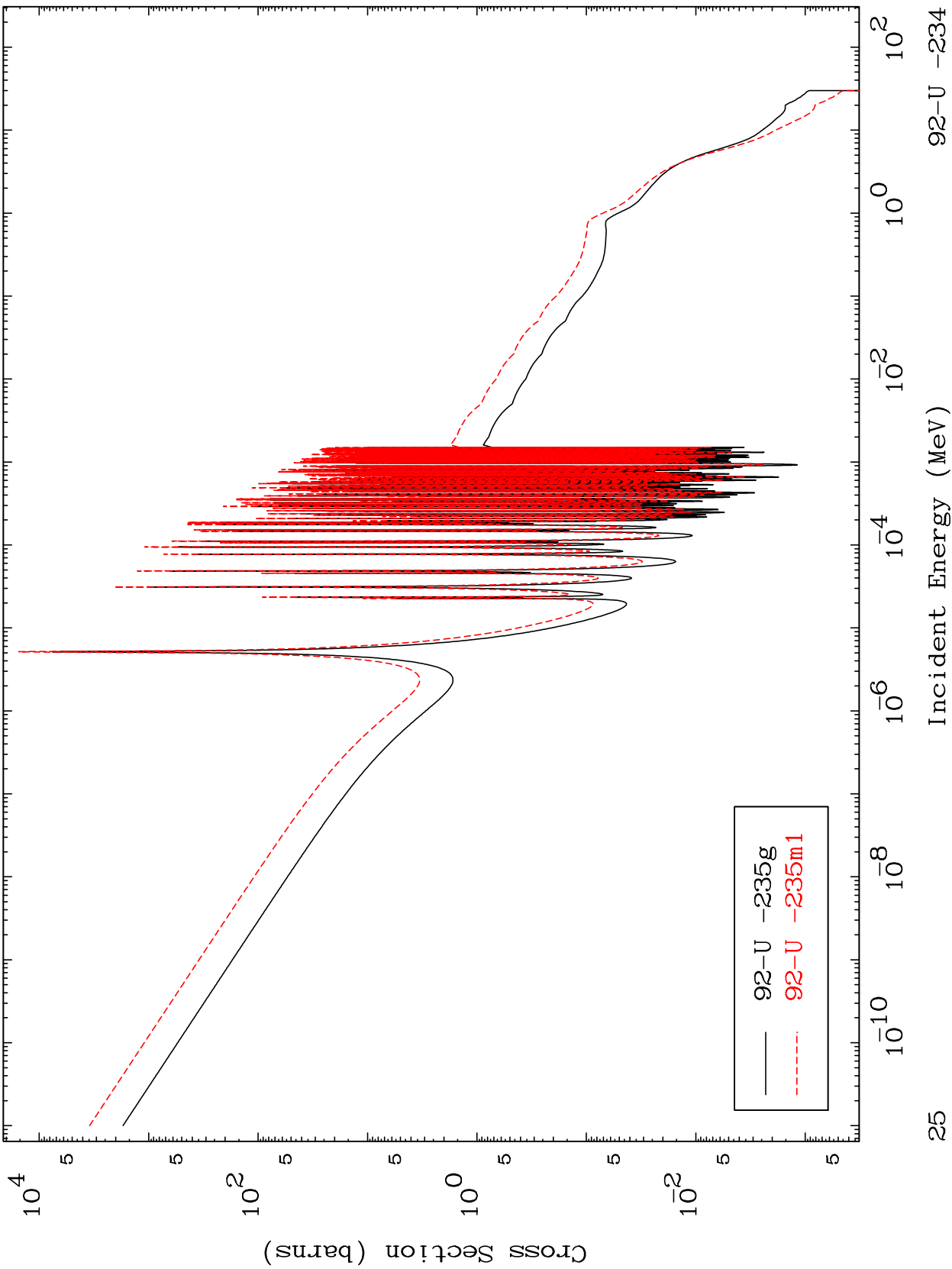
92-U -234



MAT 9225

92-U -234

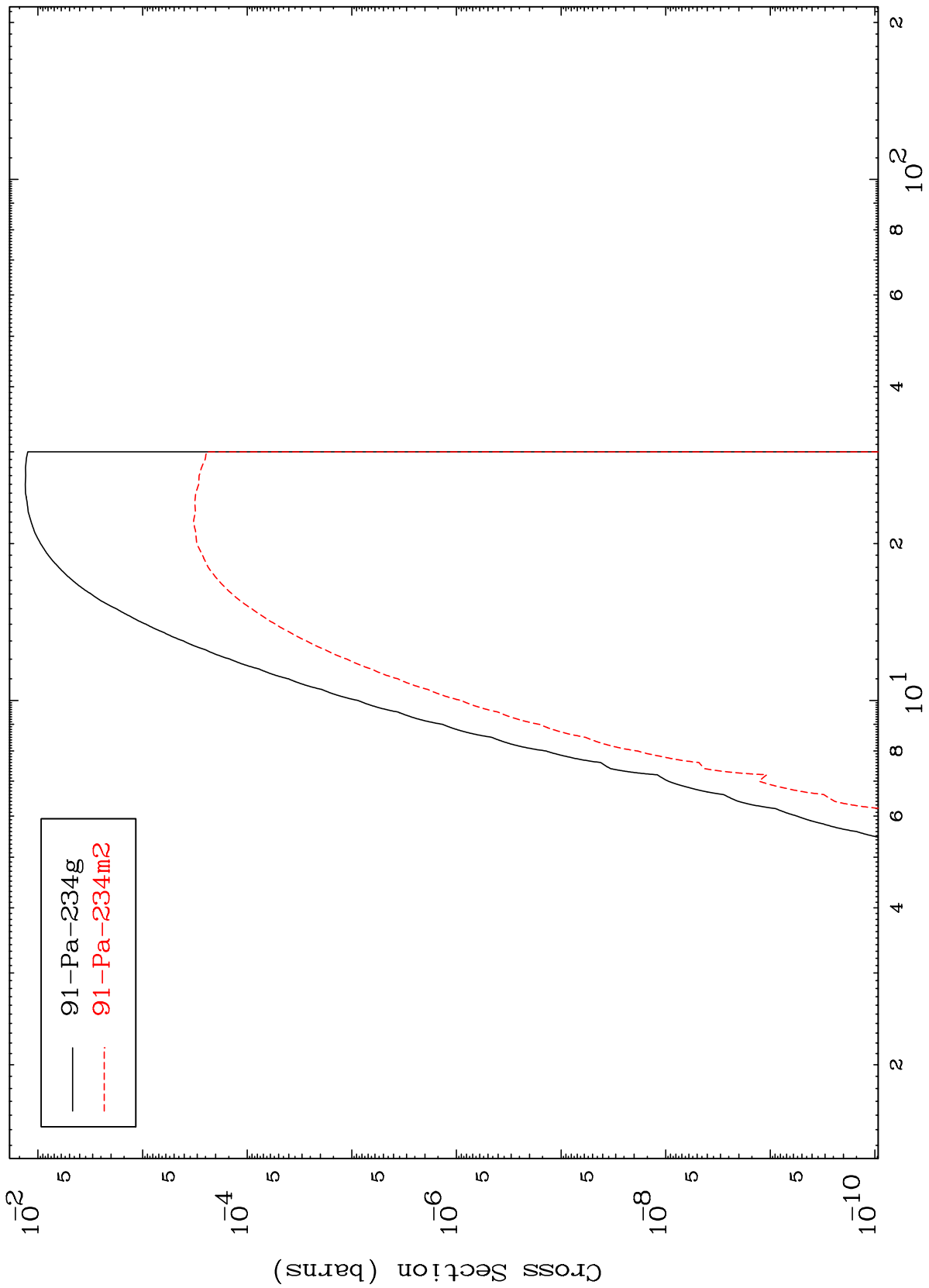
(n,γ)
Radionuclide Production Cross Section



MAT 9225

92-U -234

Radionuclide Production Cross Section (n,p)



— 91-Pa-234g
- - - 91-Pa-234m2

26

92-U -234

Incident Energy (MeV)