

Program EVALPLOT  
(Version 2017-1)

by

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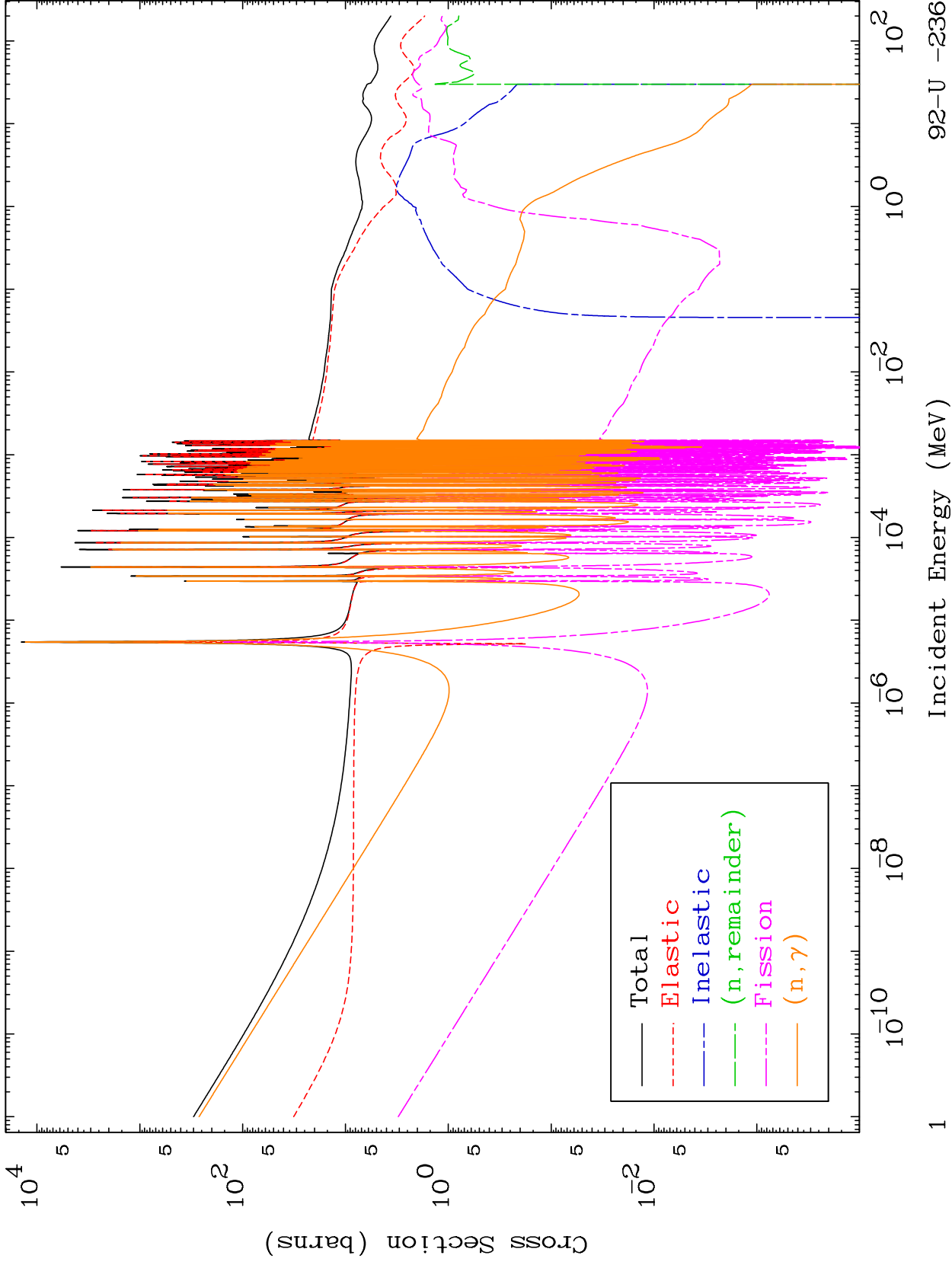
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Press Mouse Button to Start

MAT 9231

Major  
293 Kelvin Cross Sections

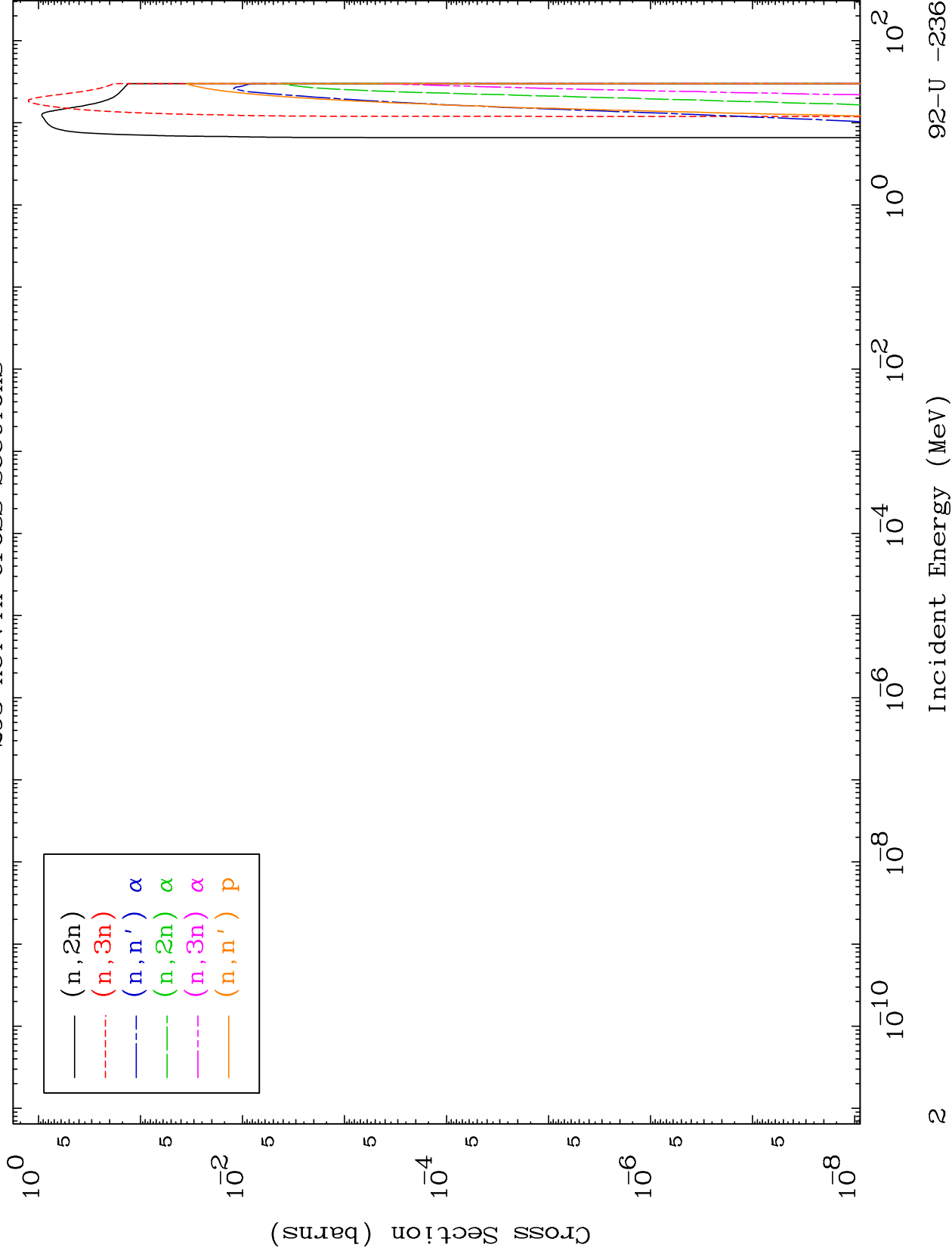
92-U -236

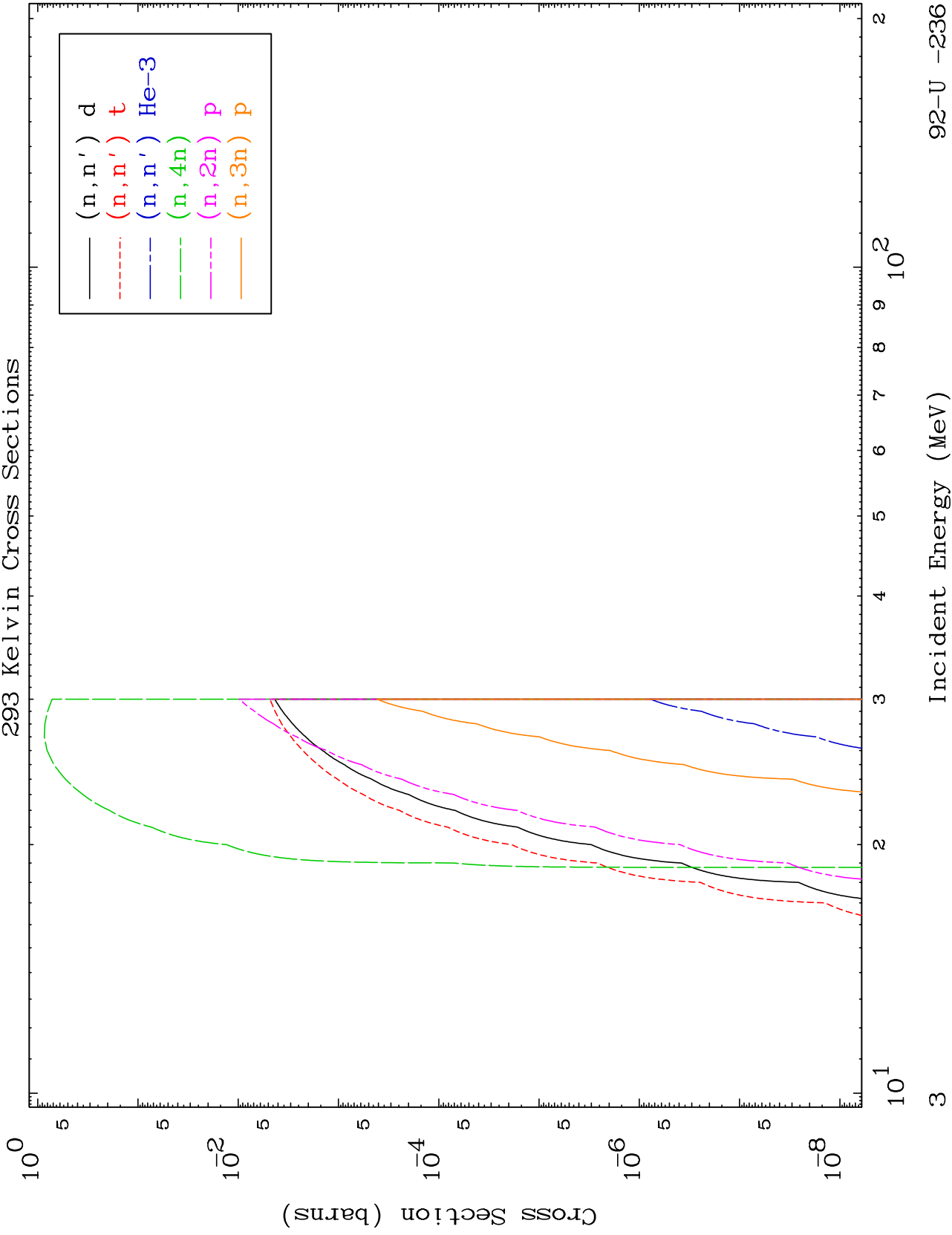


MAT 9231

Neutron Production  
293 Kelvin Cross Sections

92-U -236

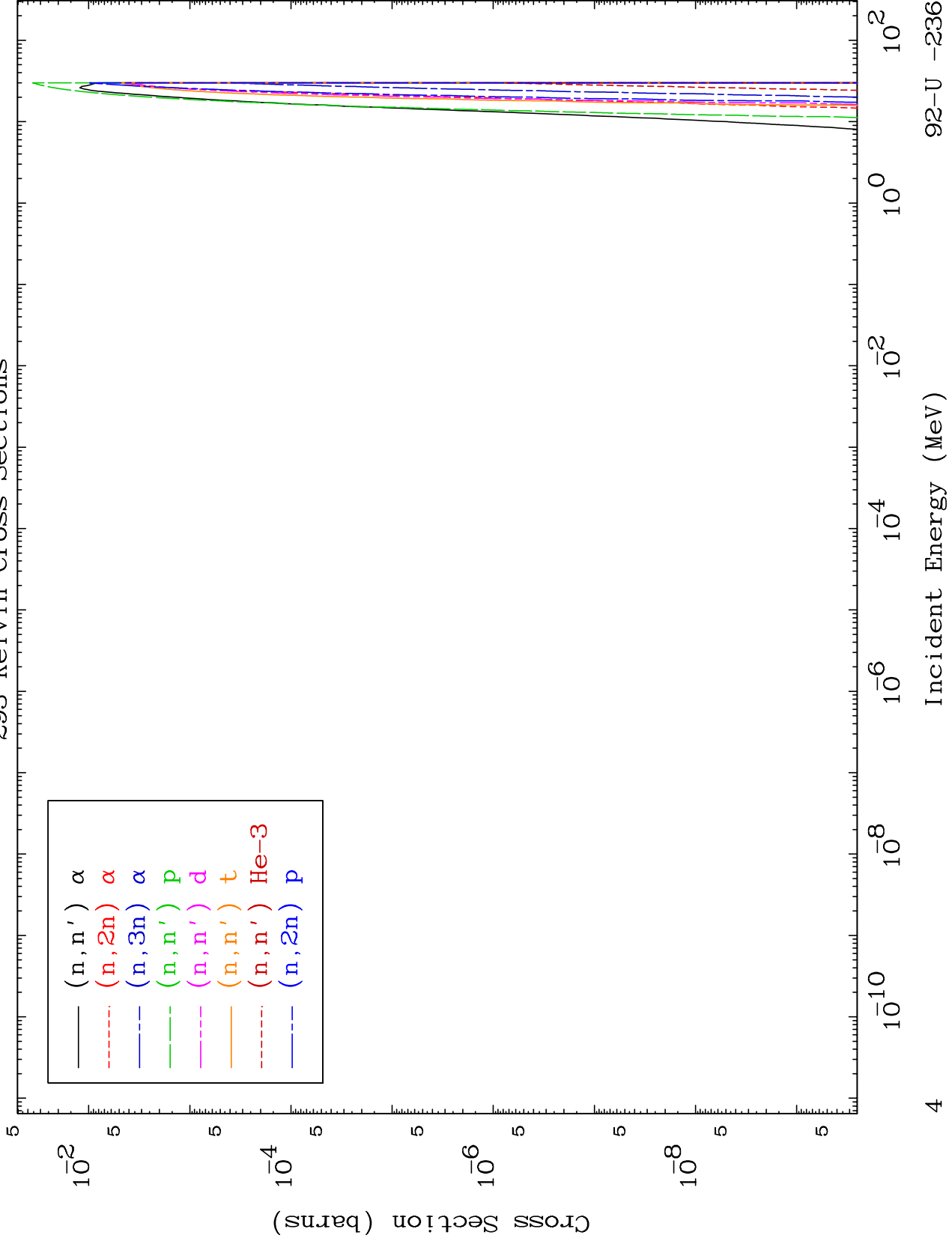




MAT 9231

Charged Particle  
293 Kelvin Cross Sections

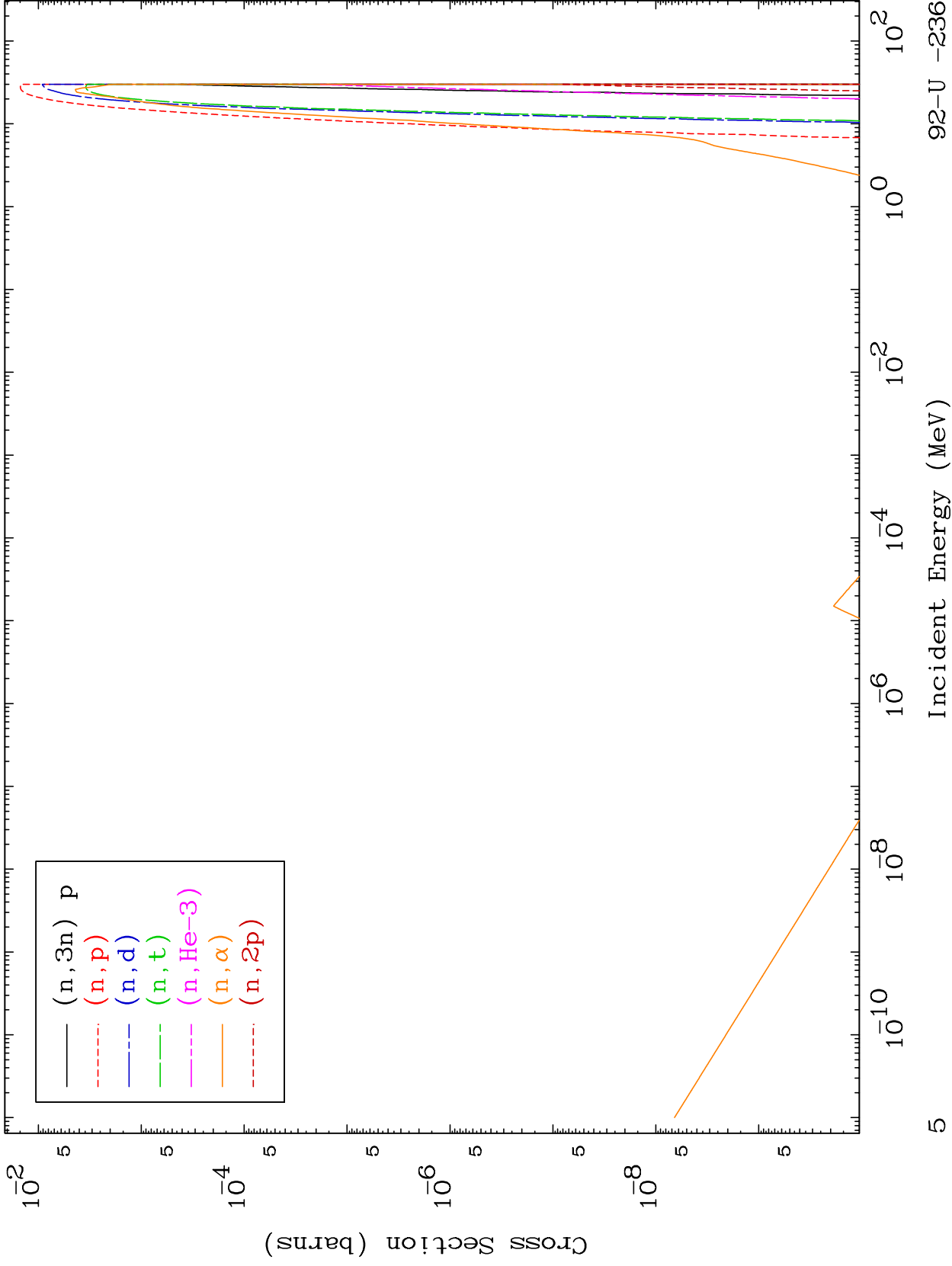
92-U -236



MAT 9231

Charged Particle  
293 Kelvin Cross Sections

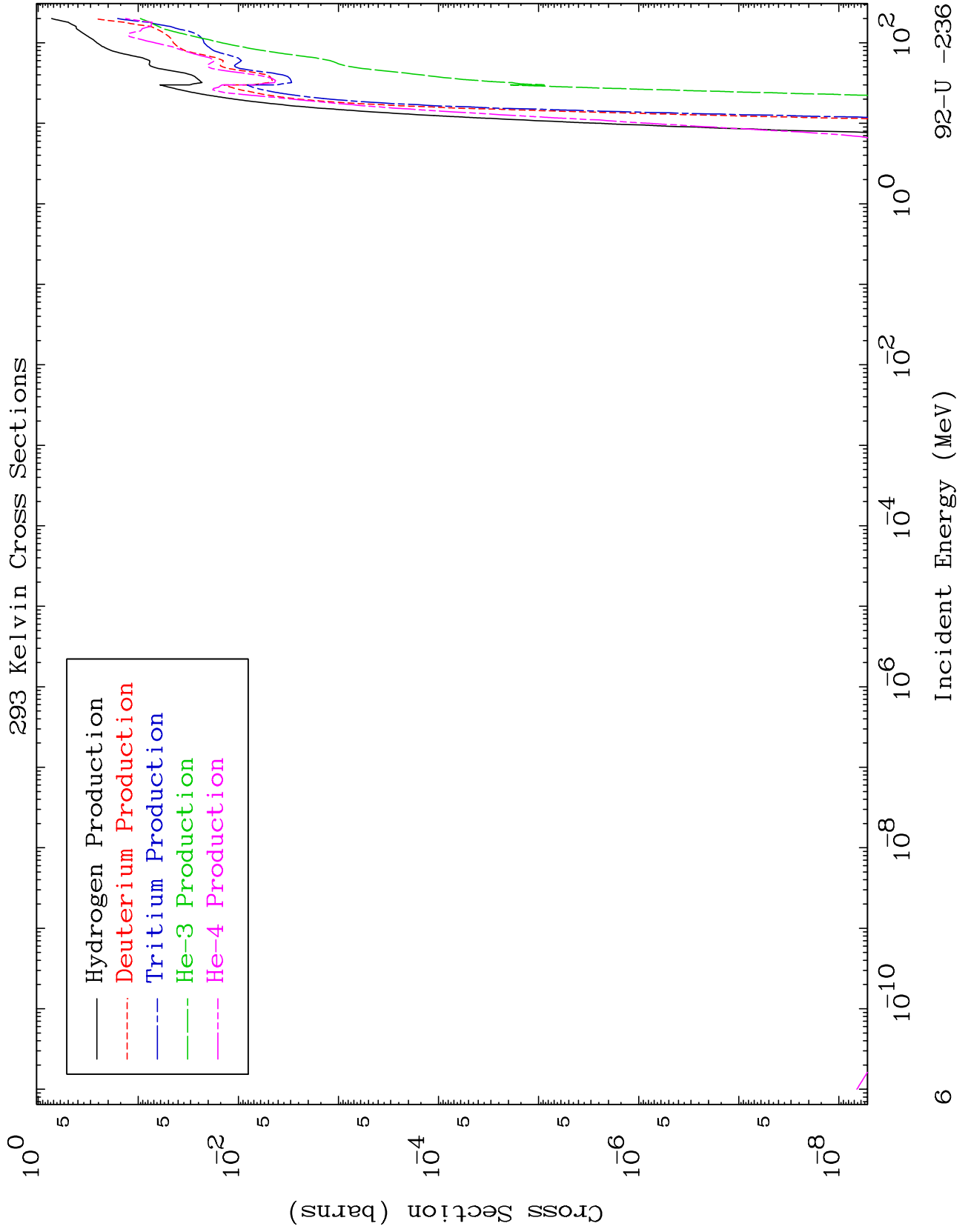
92-U -236



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Particle Production  
293 Kelvin Cross Sections

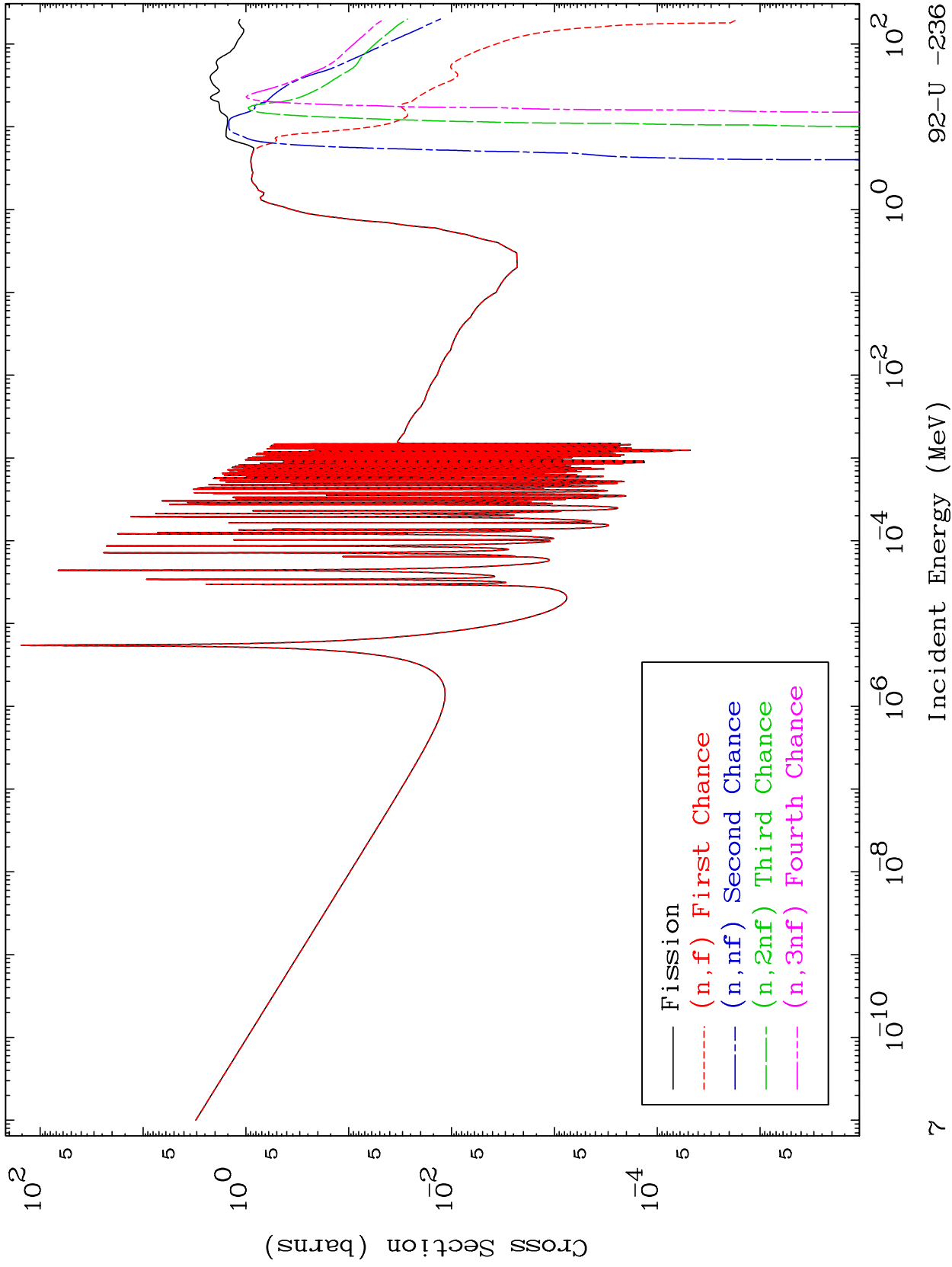
92-U -236



MAT 9231

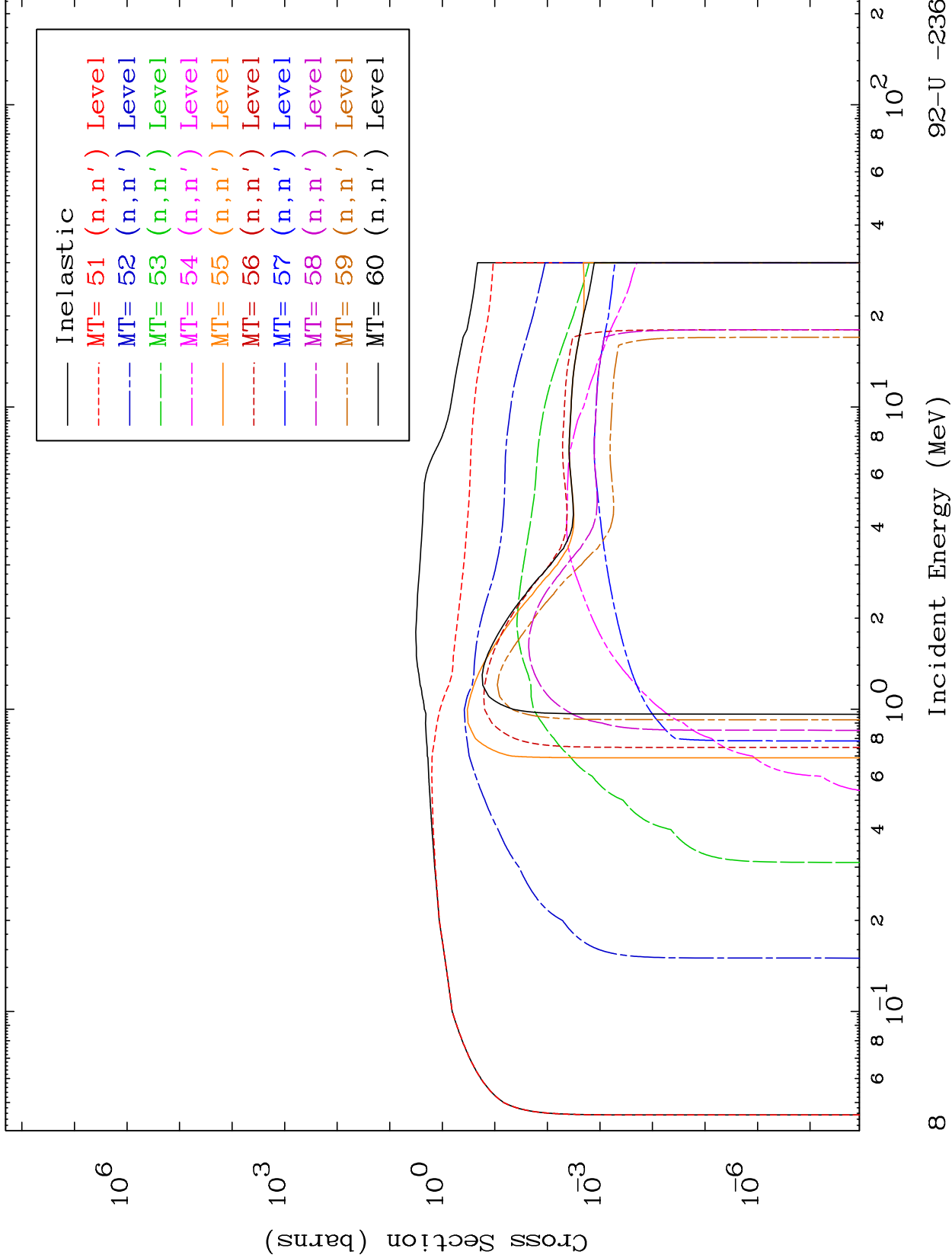
Fission  
293 Kelvin Cross Sections

92-U -236





293 Kelvin Cross Sections

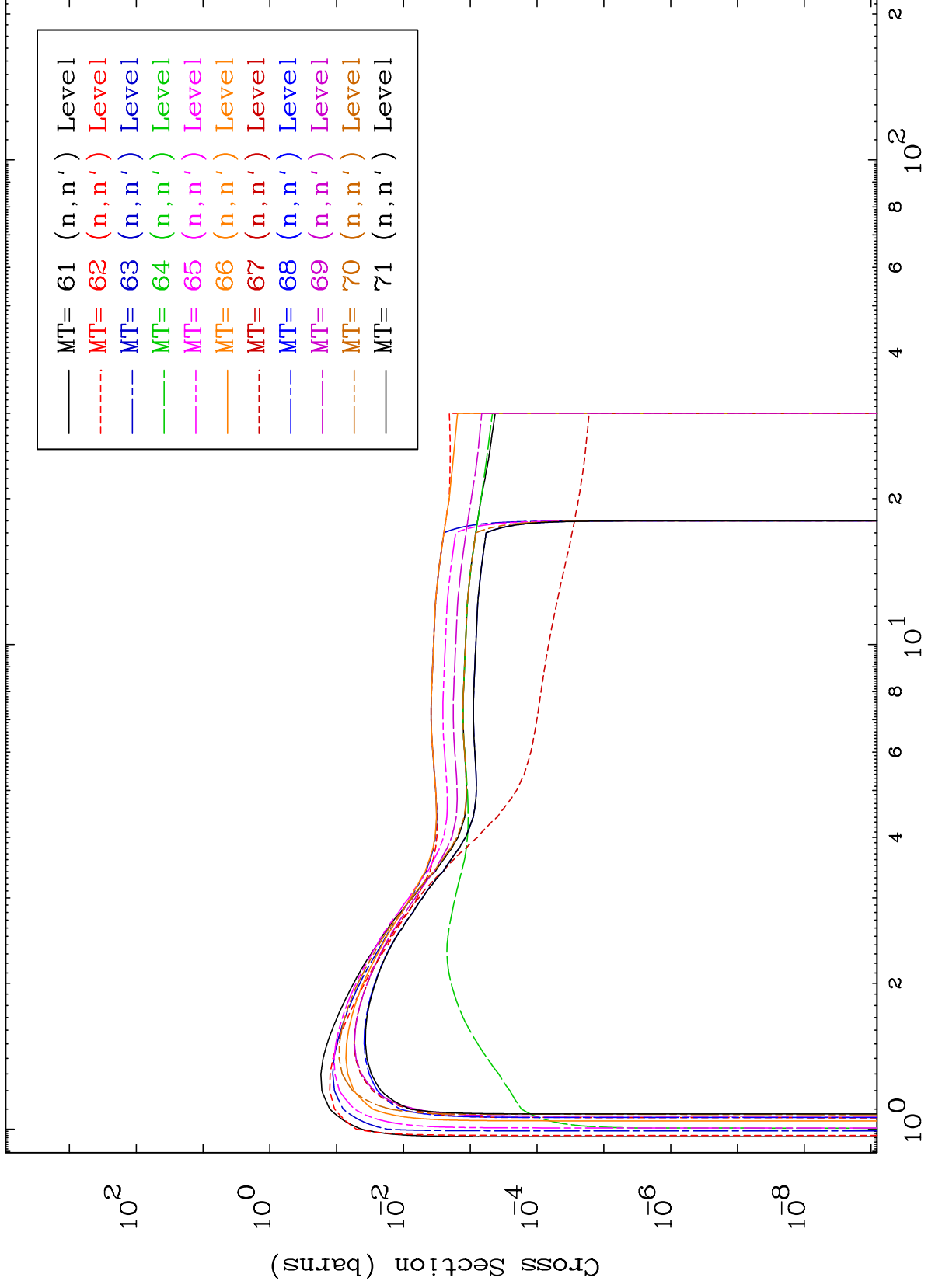


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(n,n') Level

92-U -236

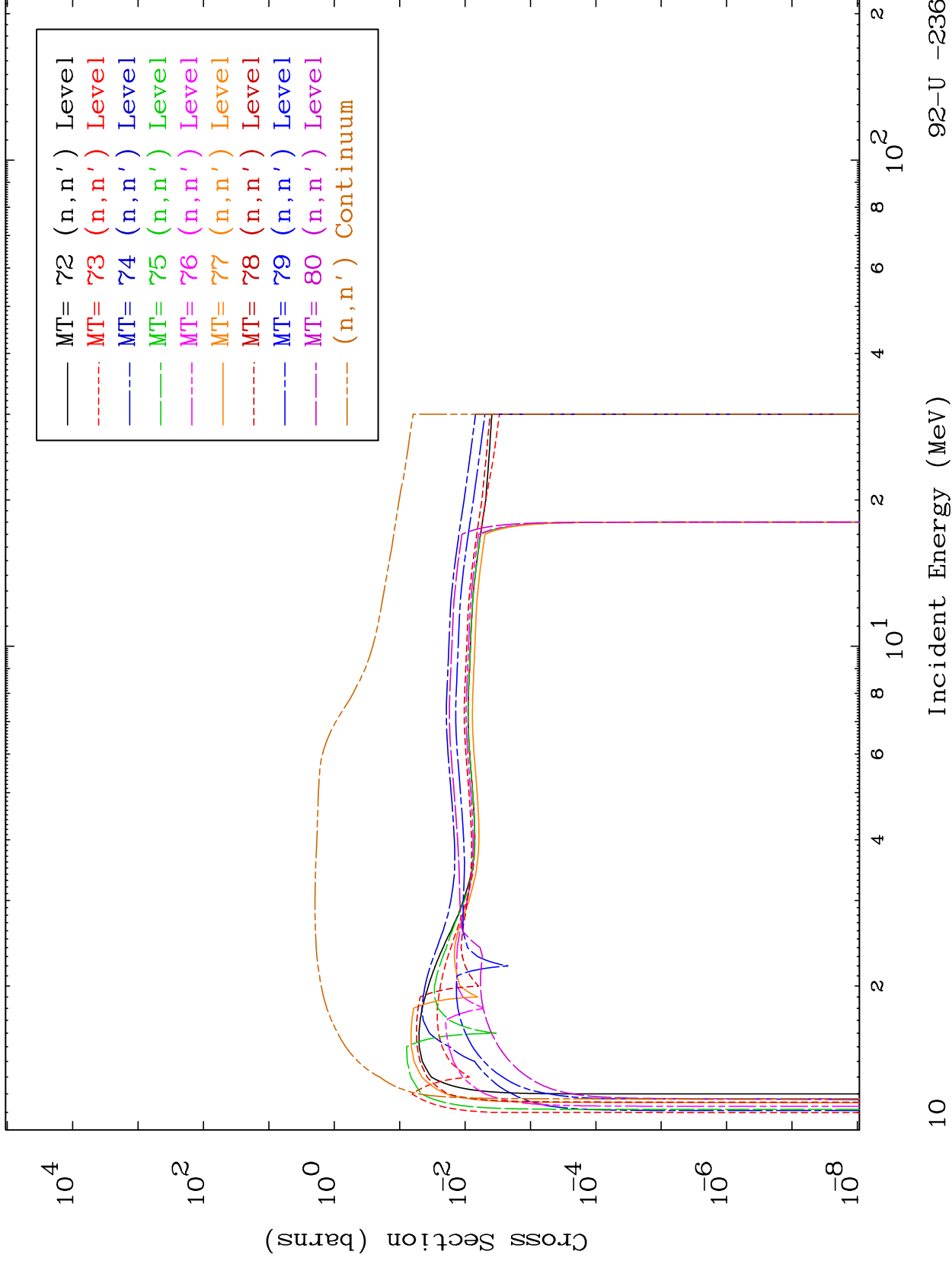
293 Kelvin Cross Sections



Incident Energy (MeV)

92-U -236

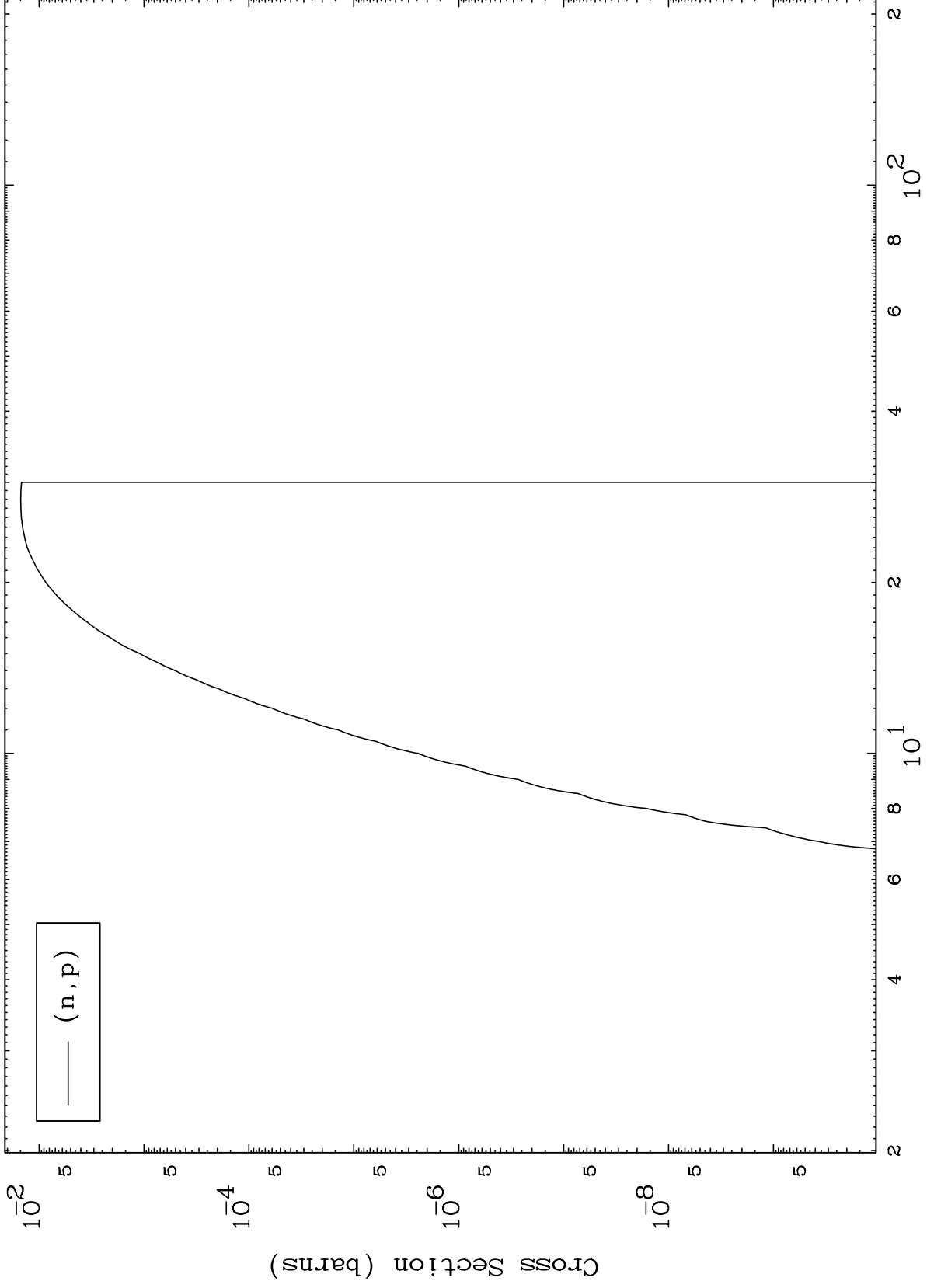
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

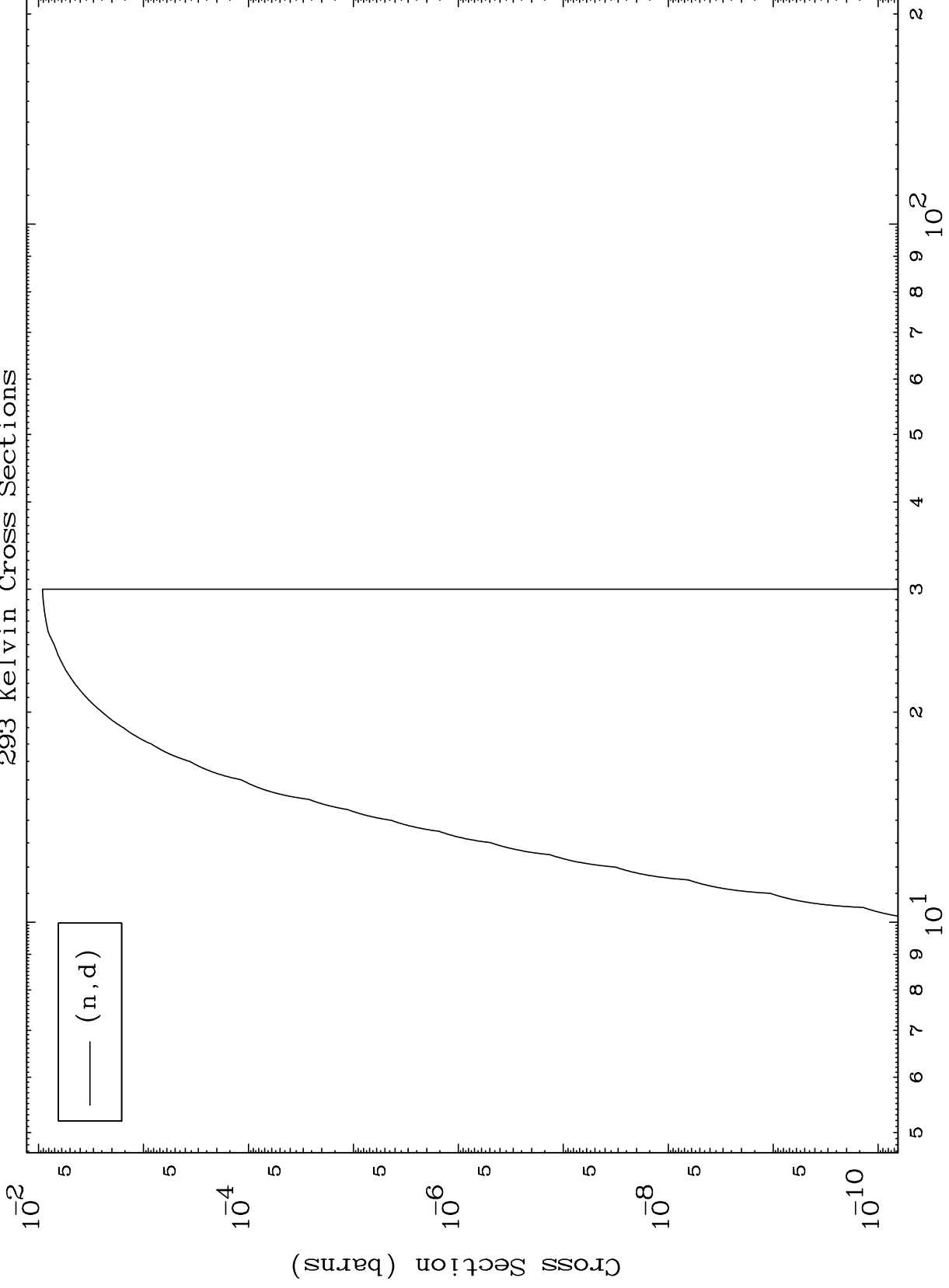
92-U -236



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(n,d) Levels  
293 Kelvin Cross Sections

92-U -236



12

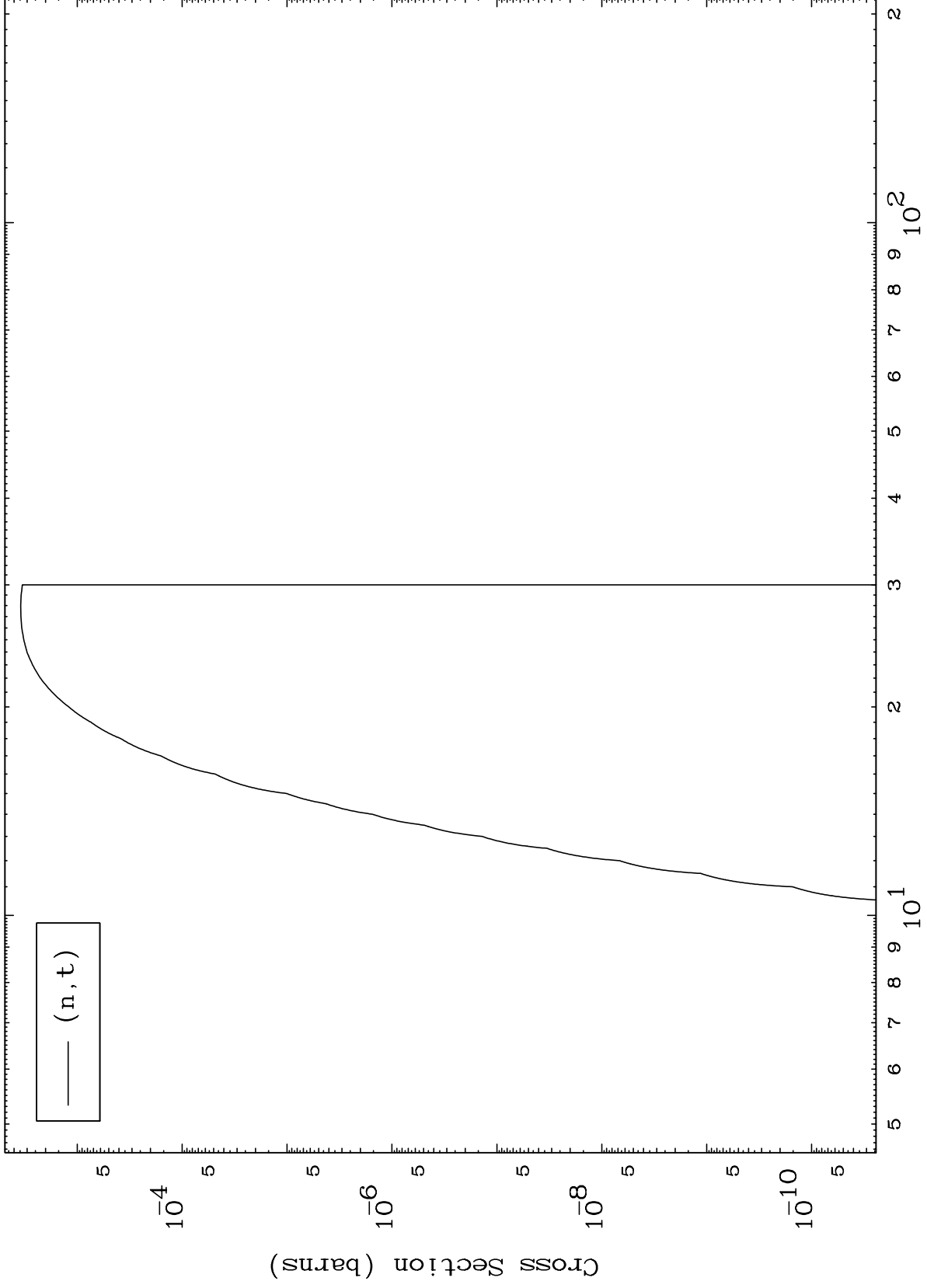
Incident Energy (MeV)

92-U -236

MAT 9231

(n,t) Levels  
293 Kelvin Cross Sections

92-U -236



13

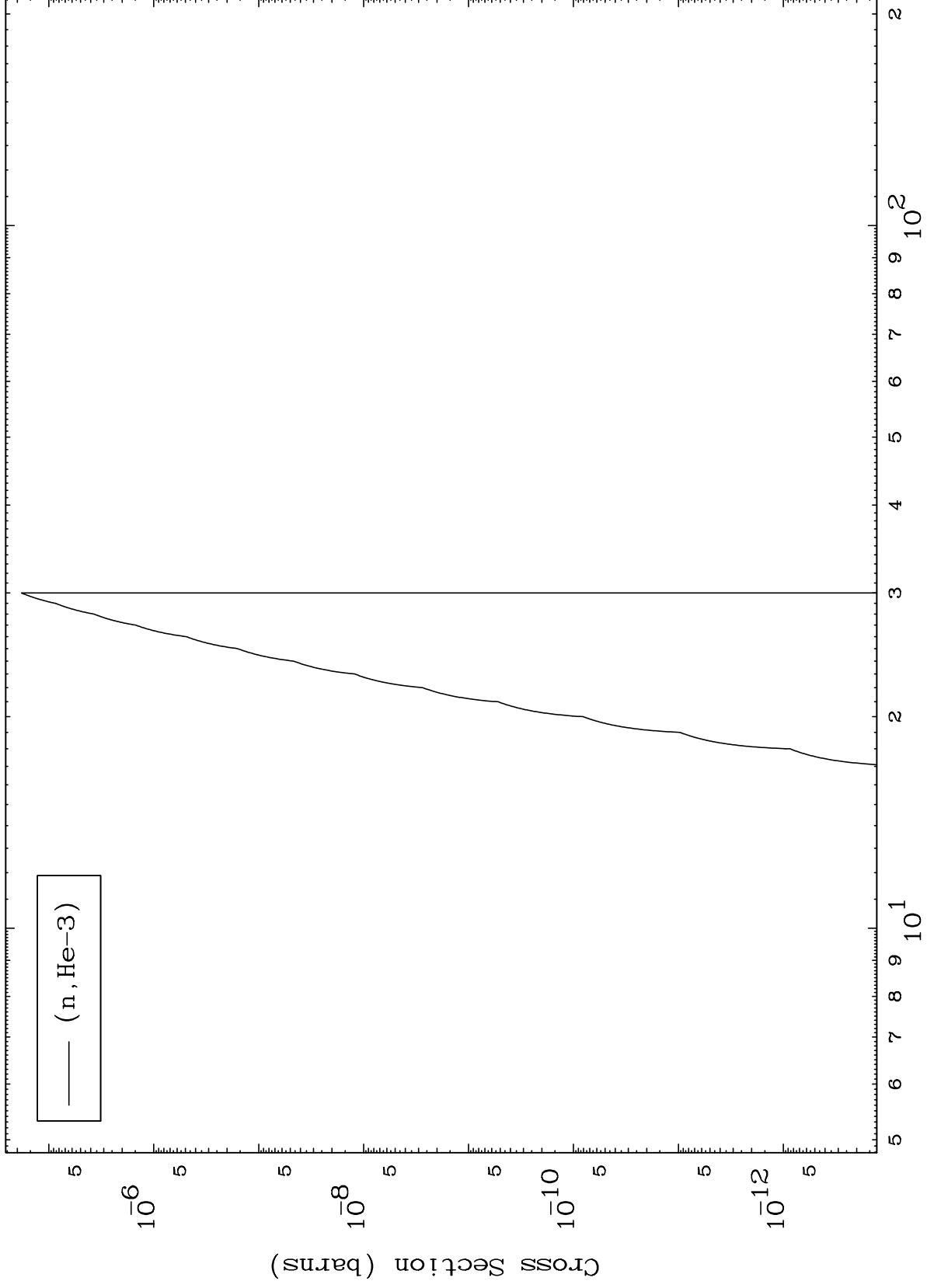
Incident Energy (MeV)

92-U -236

MAT 9231

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -236



14

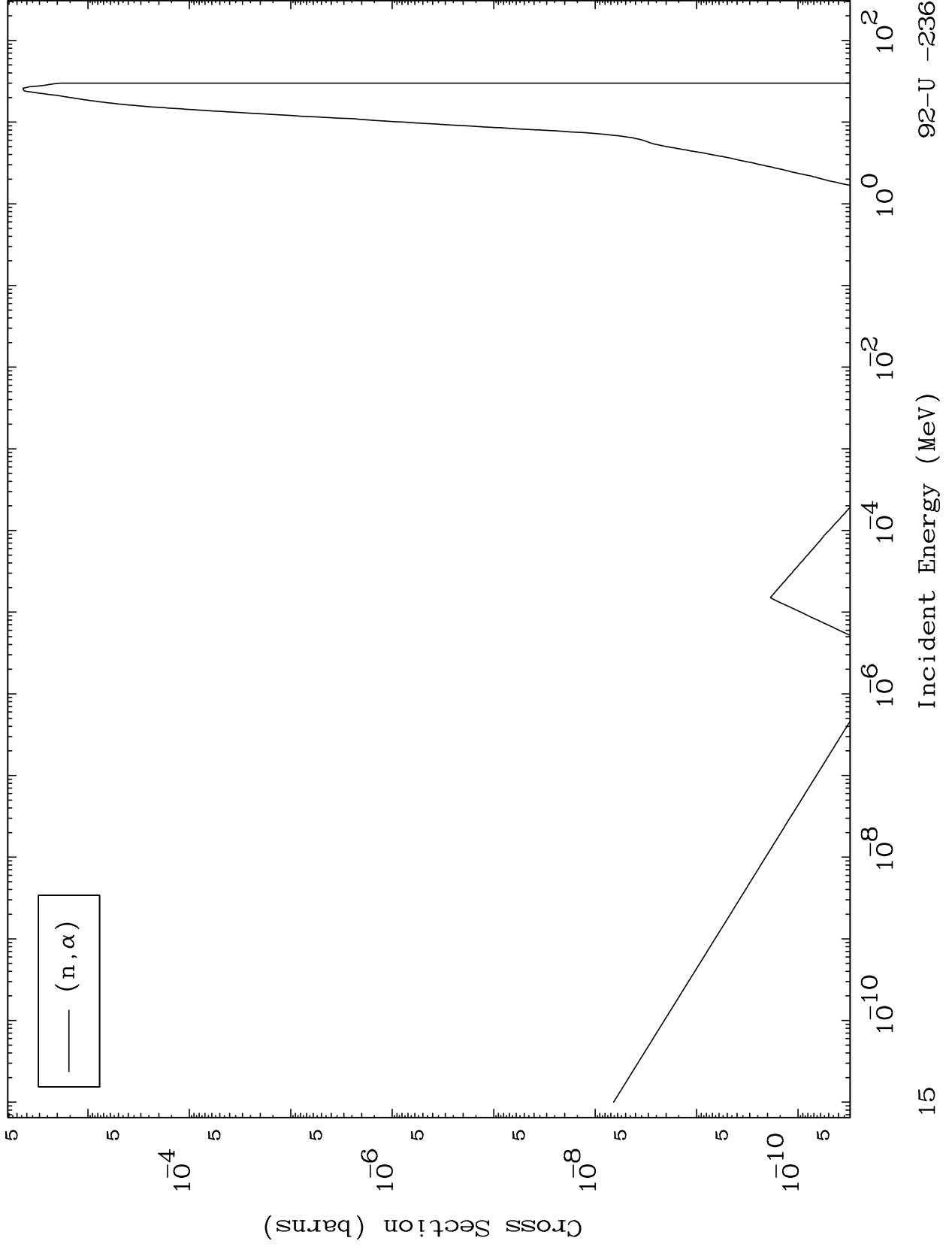
Incident Energy (MeV)

92-U -236

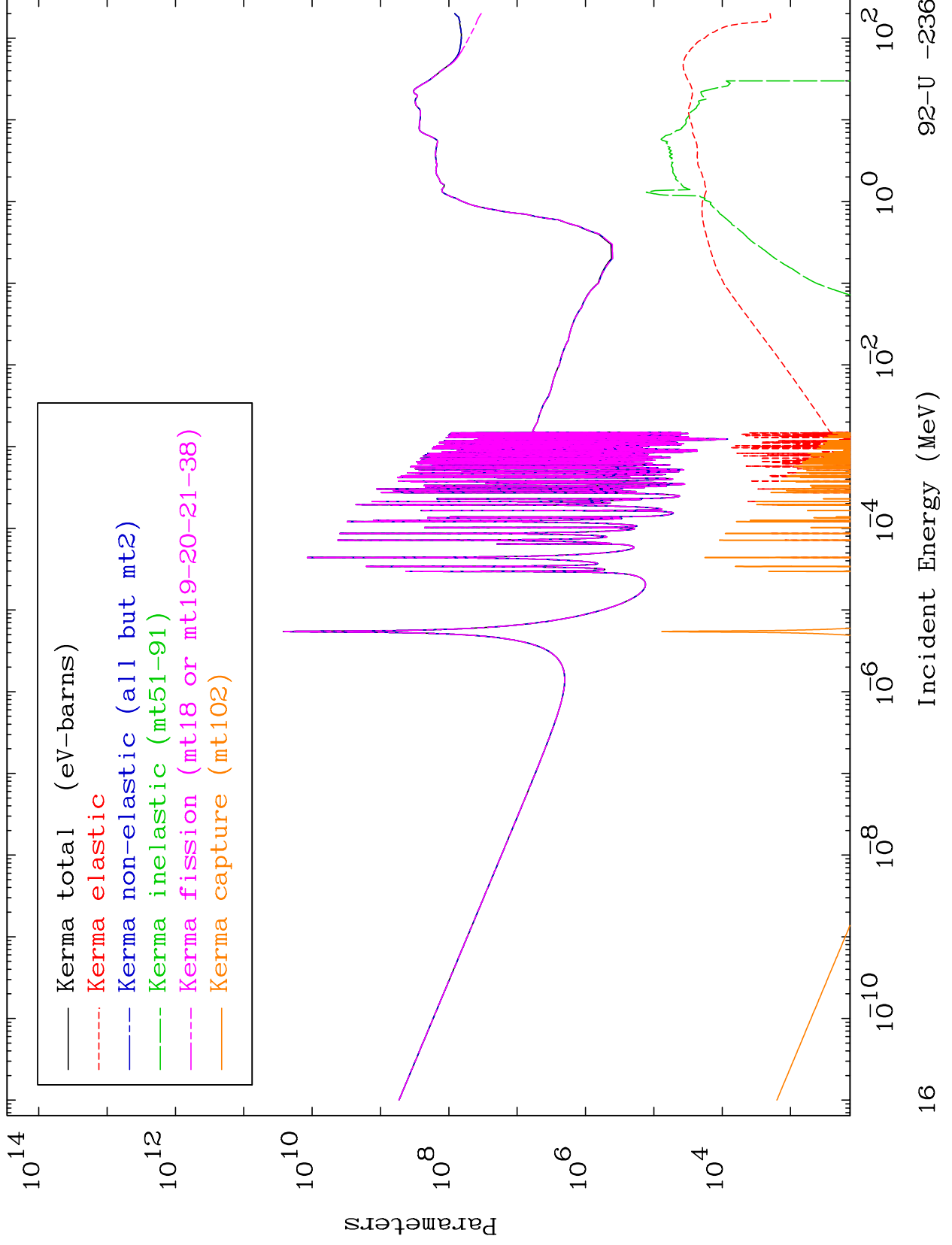
MAT 9231

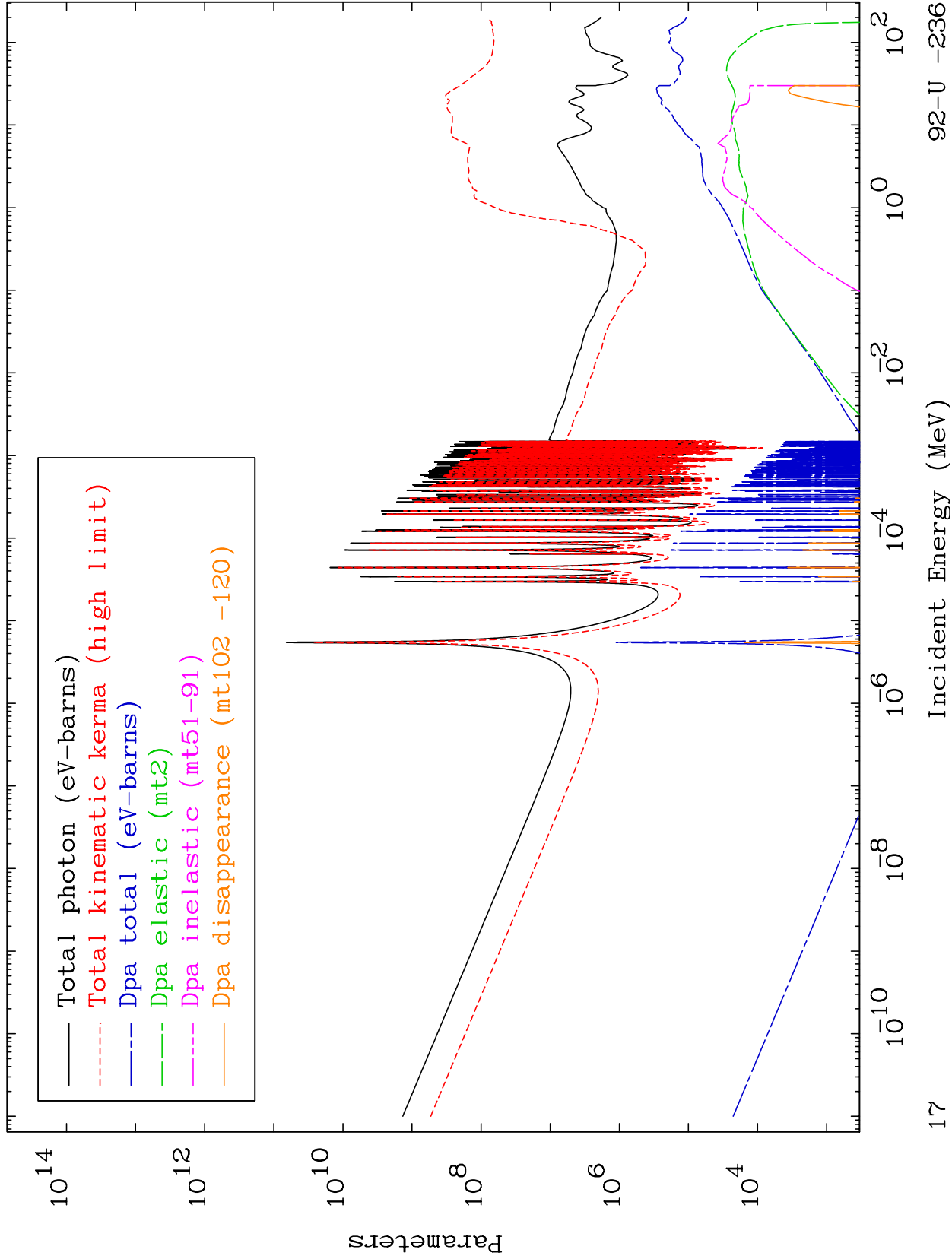
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

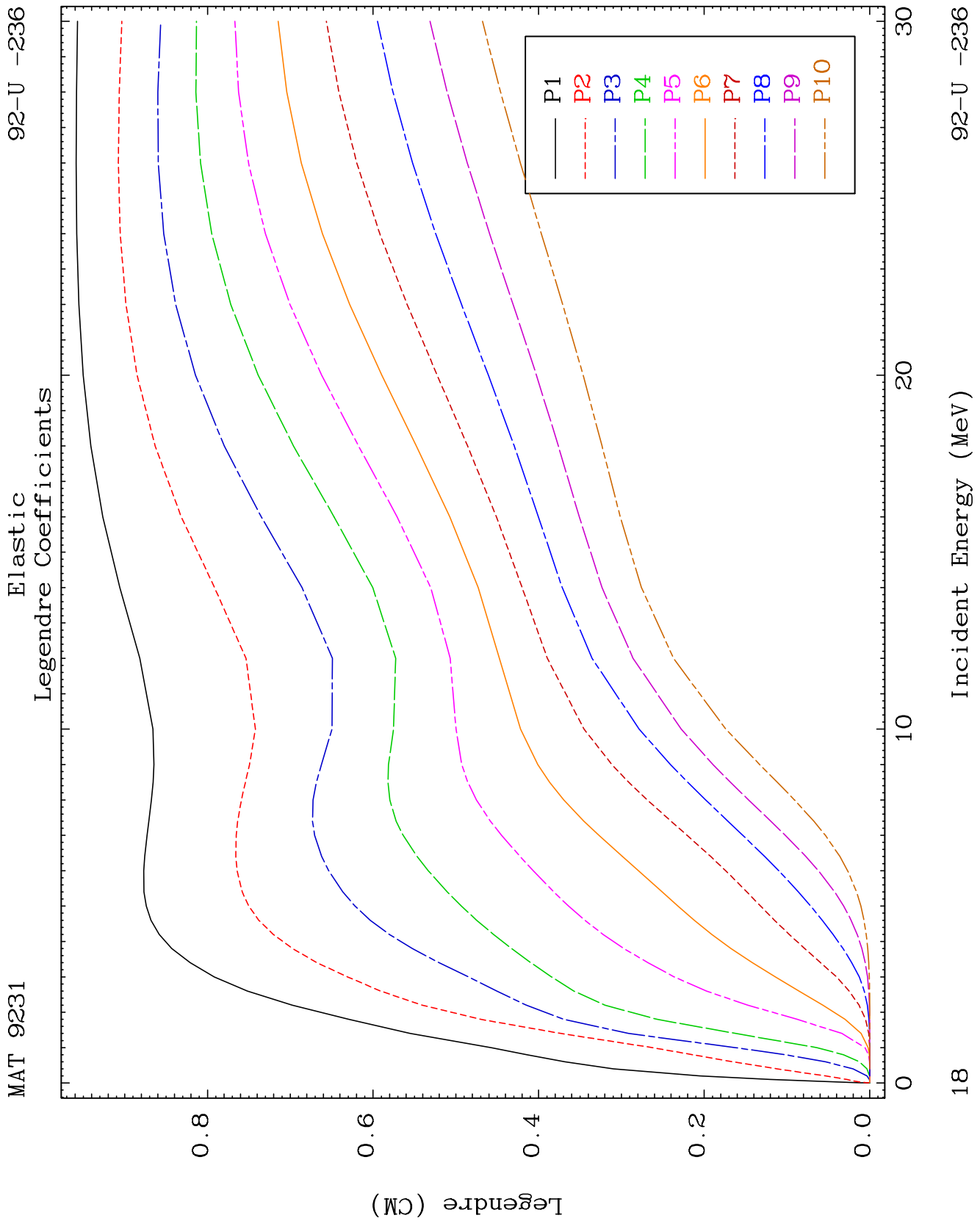
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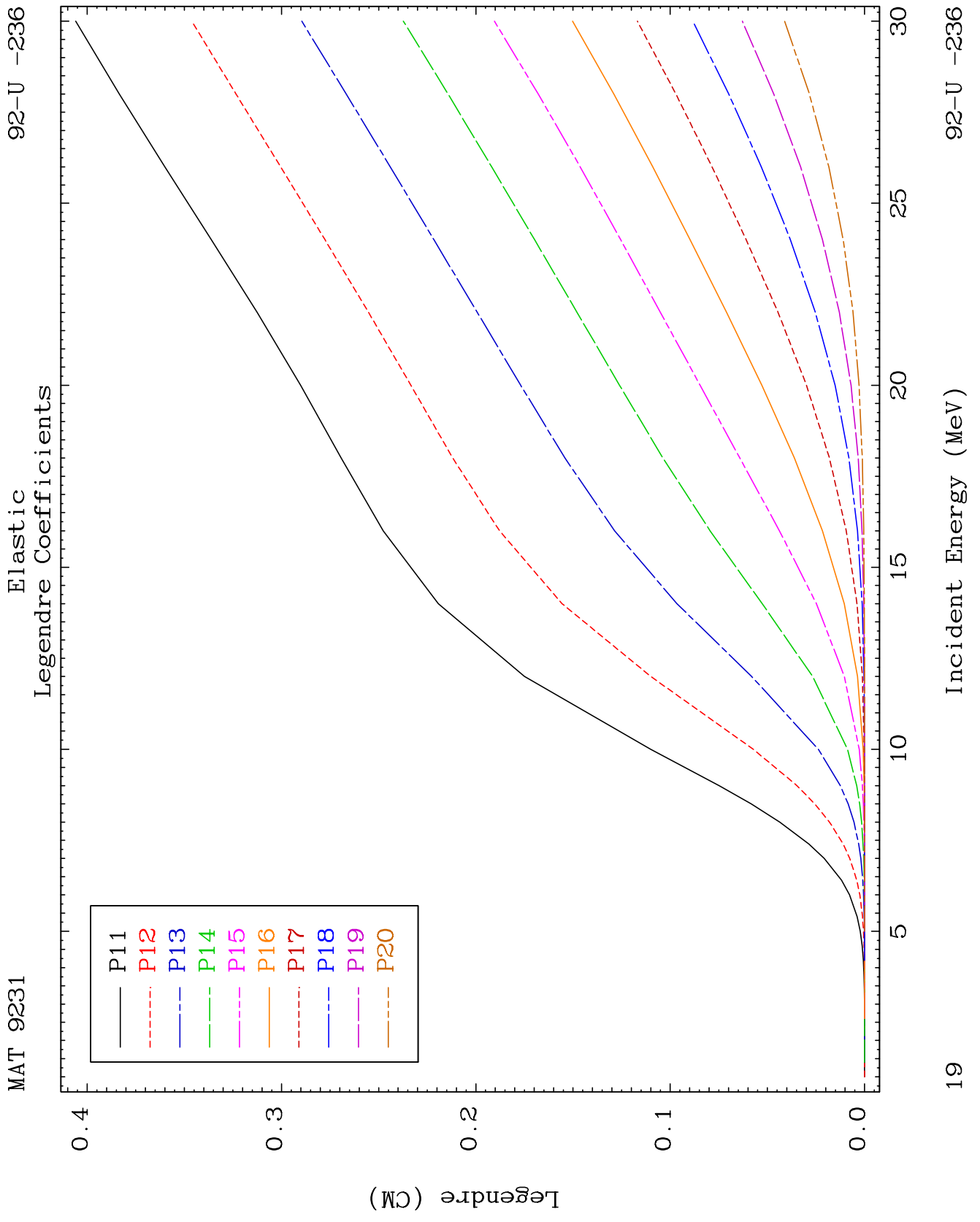








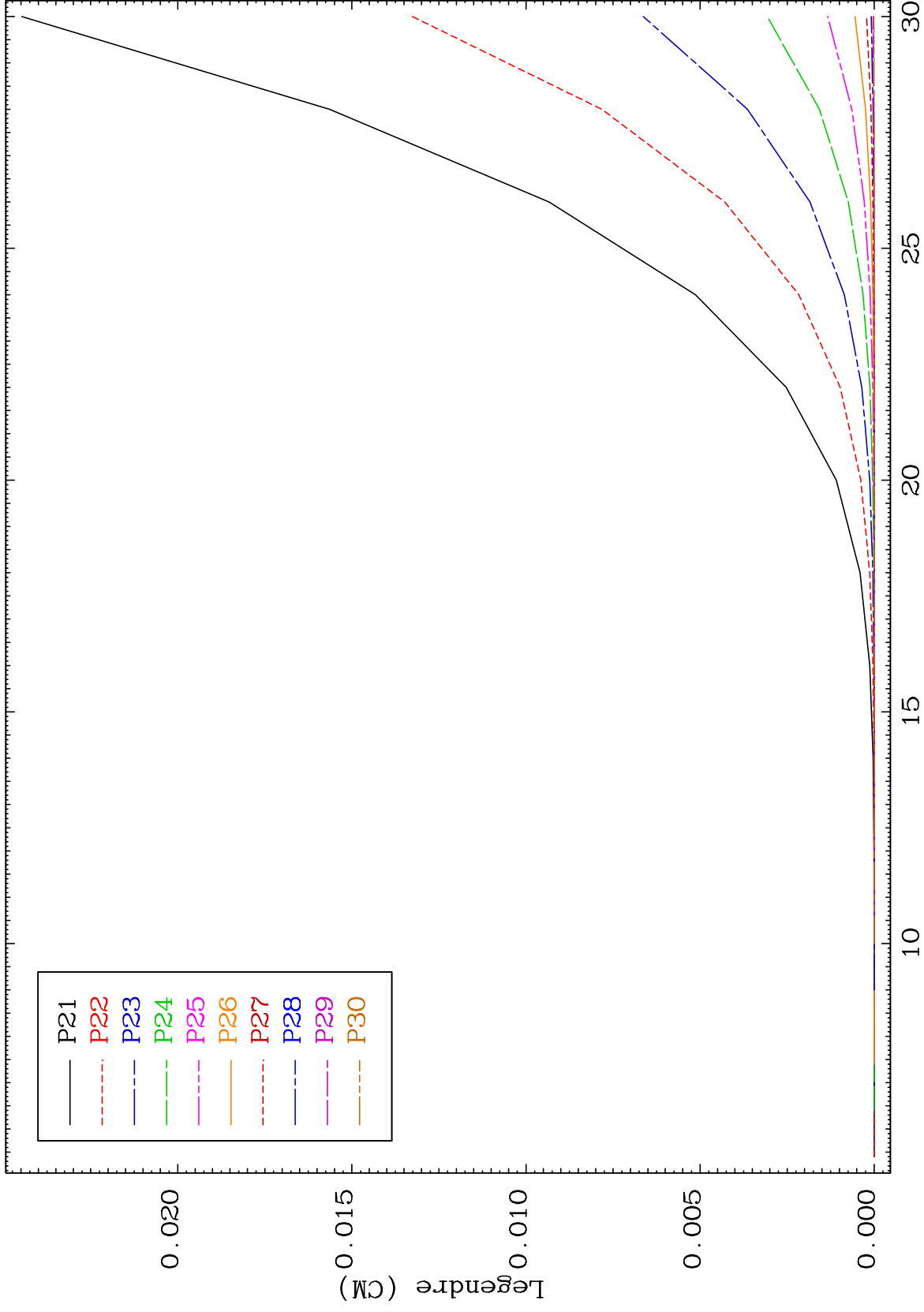




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### Elastic Legendre Coefficients

92-U -236

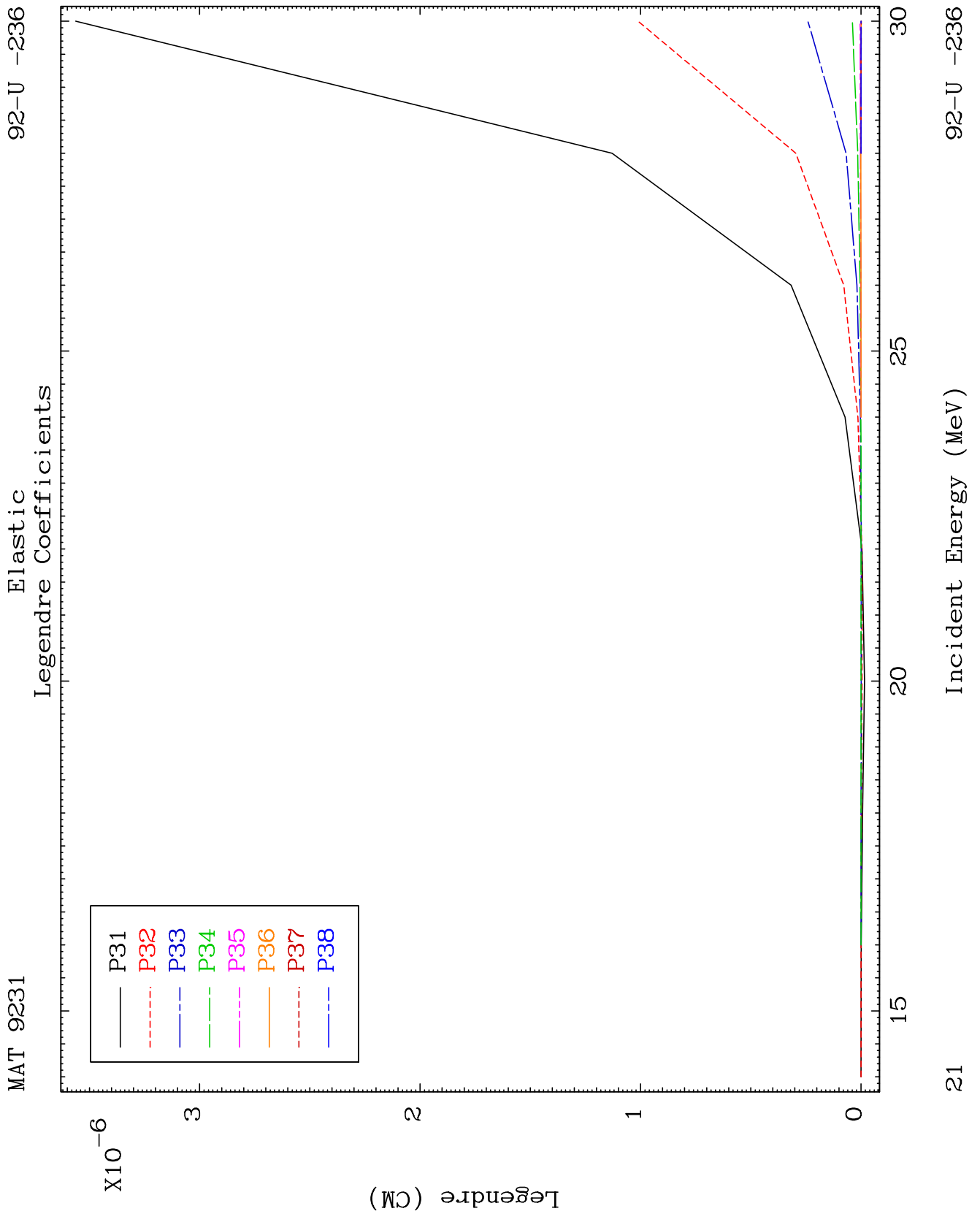


20

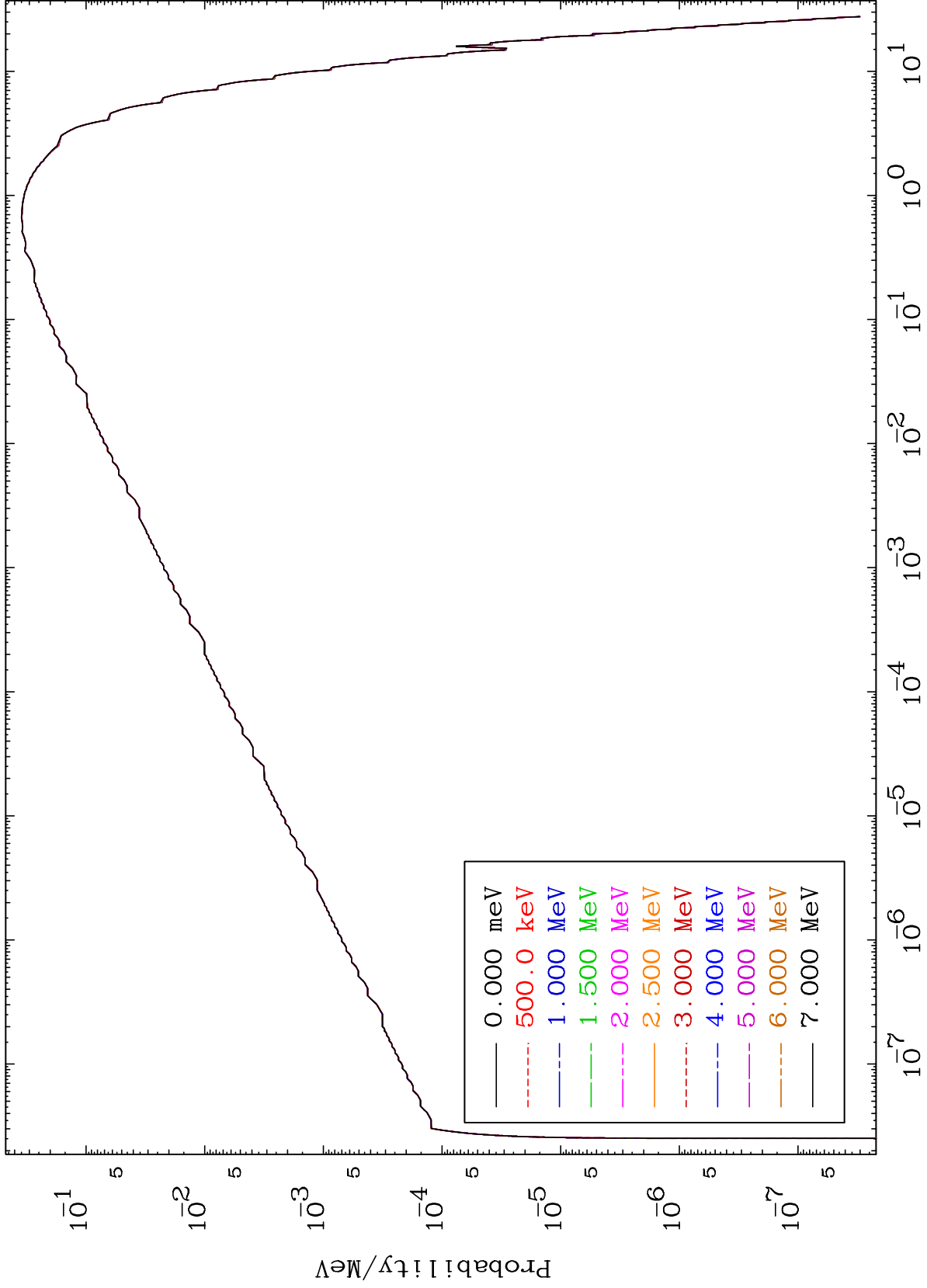
Incident Energy (MeV)

92-U -236

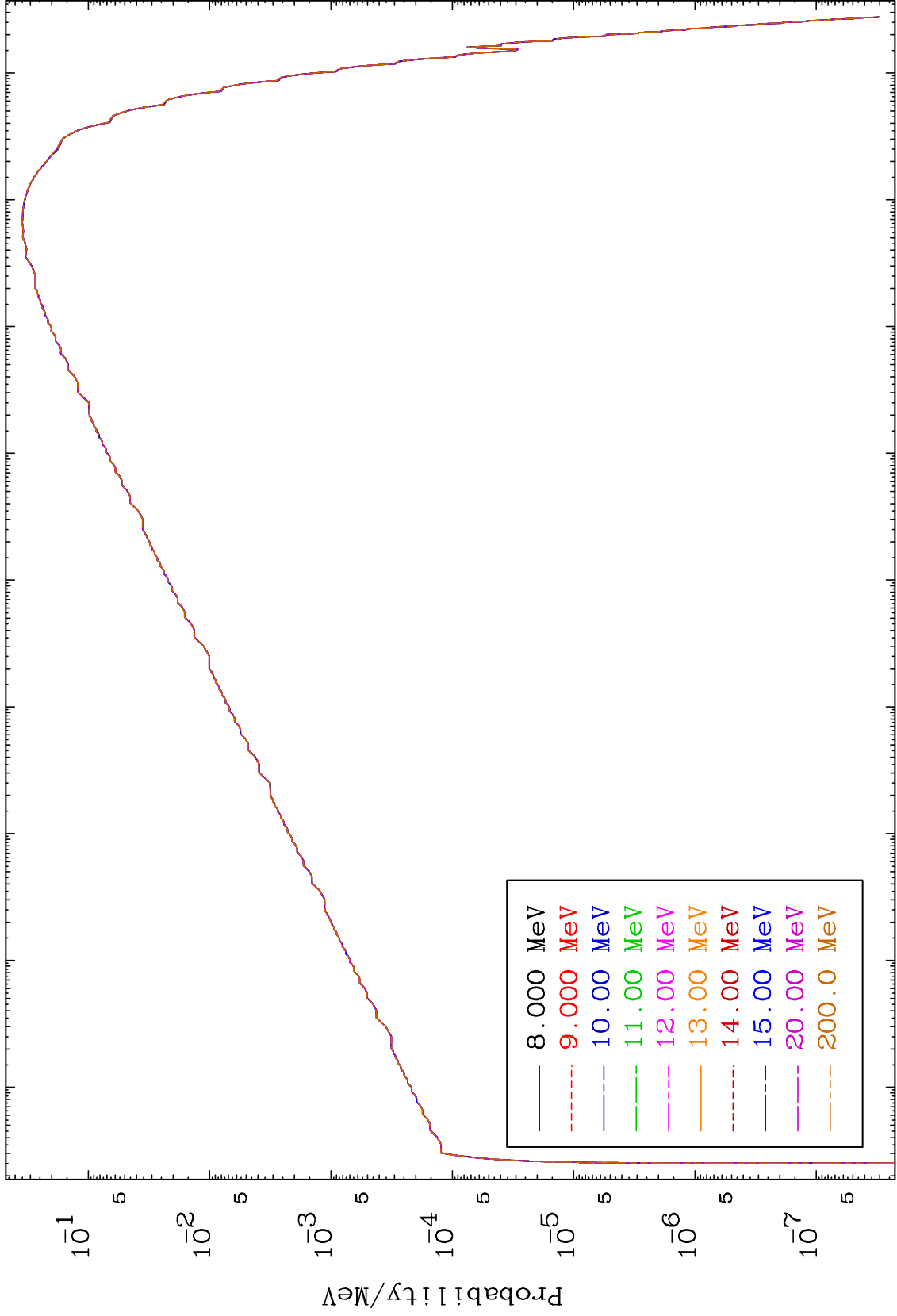
30



Fission  
Energy Distribution



Fission  
Energy Distribution

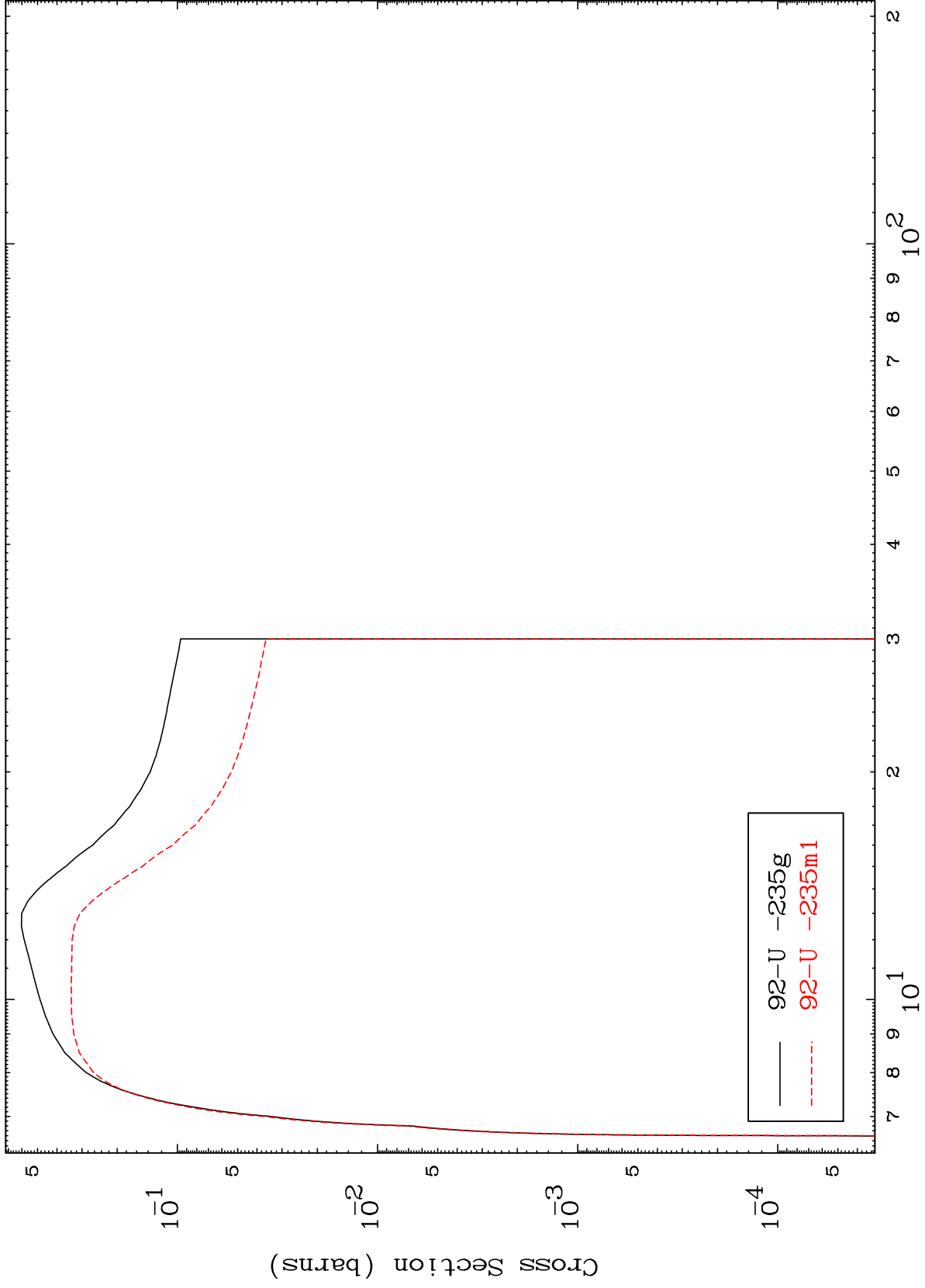




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92-U -236

(n,2n)  
Radionuclide Production Cross Section



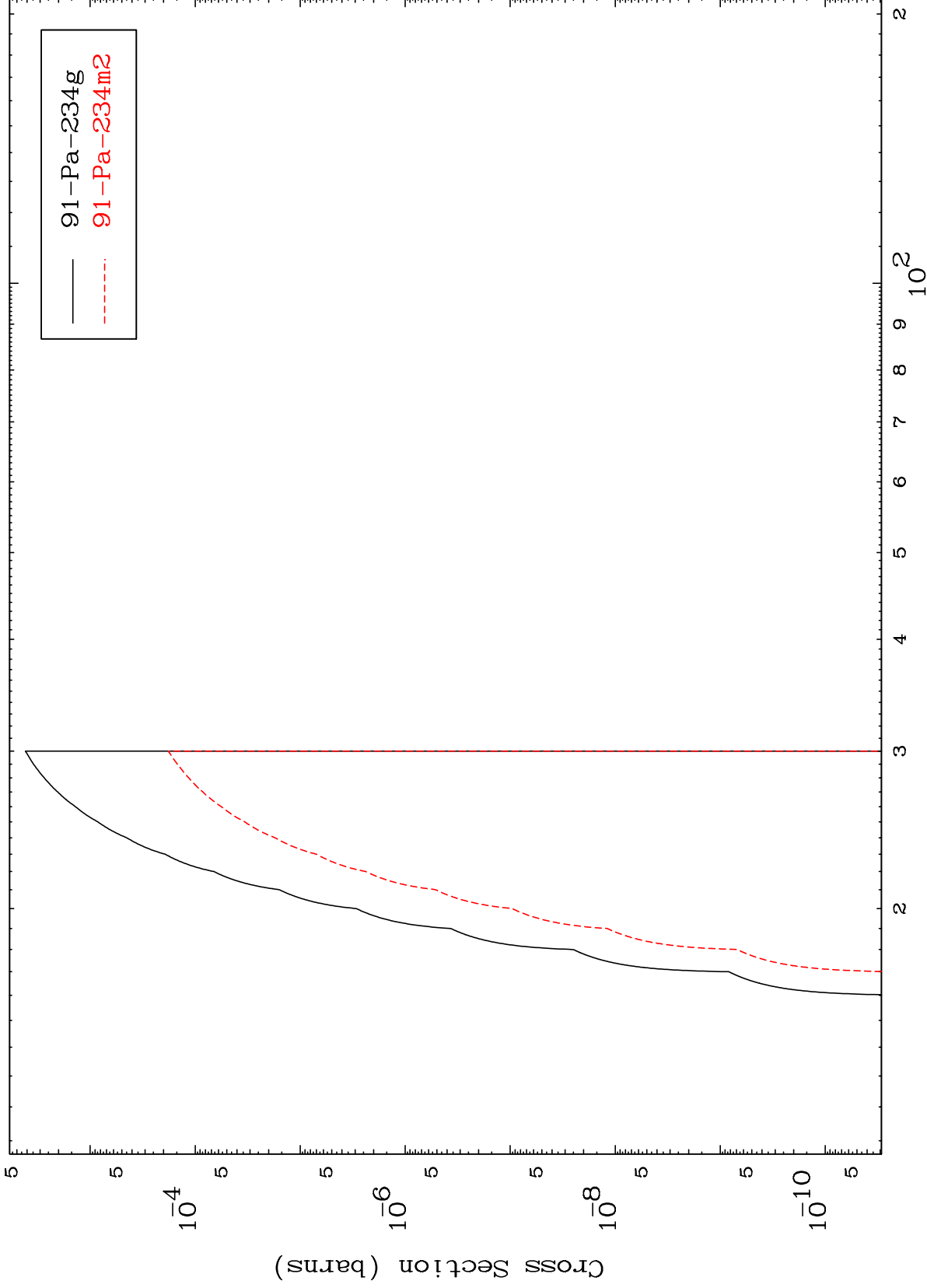
24

Incident Energy (MeV)

92-U -236

(n,n') d

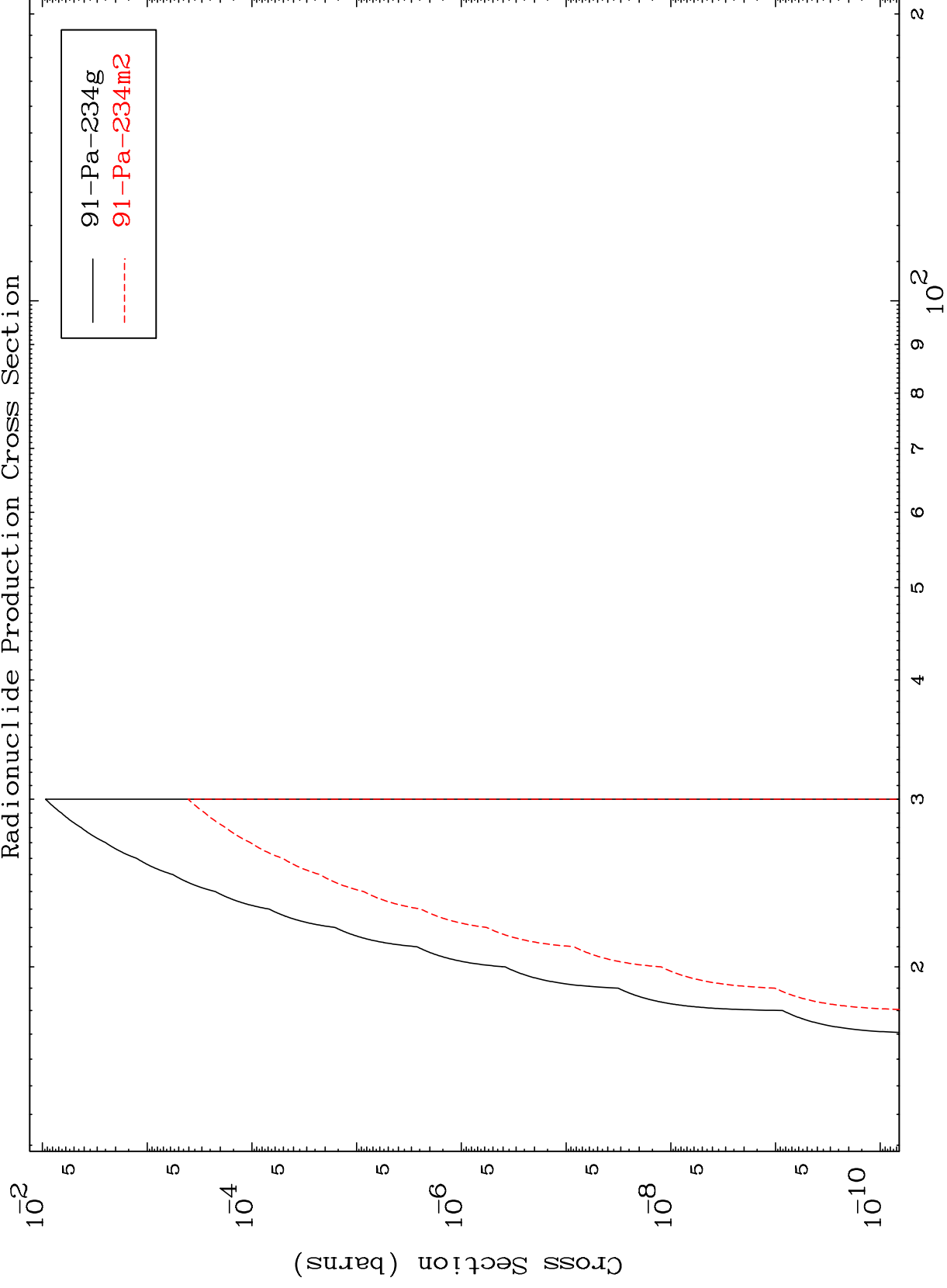
Radionuclide Production Cross Section



MAT 9231

92-U -236

(n,2n) p  
Radionuclide Production Cross Section



92-U -236

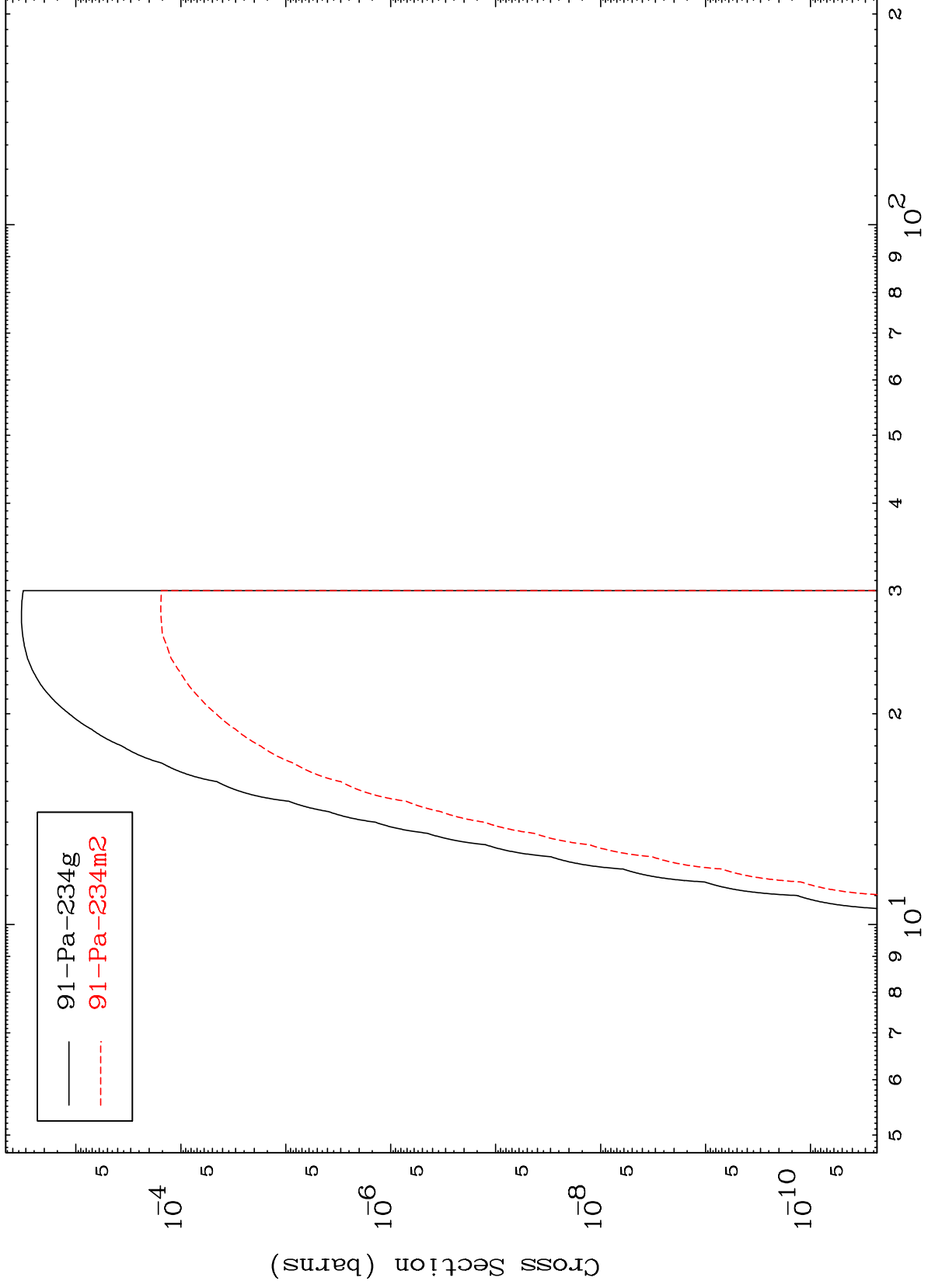
Incident Energy (MeV)

26

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92-U -236

(n, t)  
Radionuclide Production Cross Section



27

Incident Energy (MeV)

92-U -236