

Program EVALPLOT  
(Version 2017-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

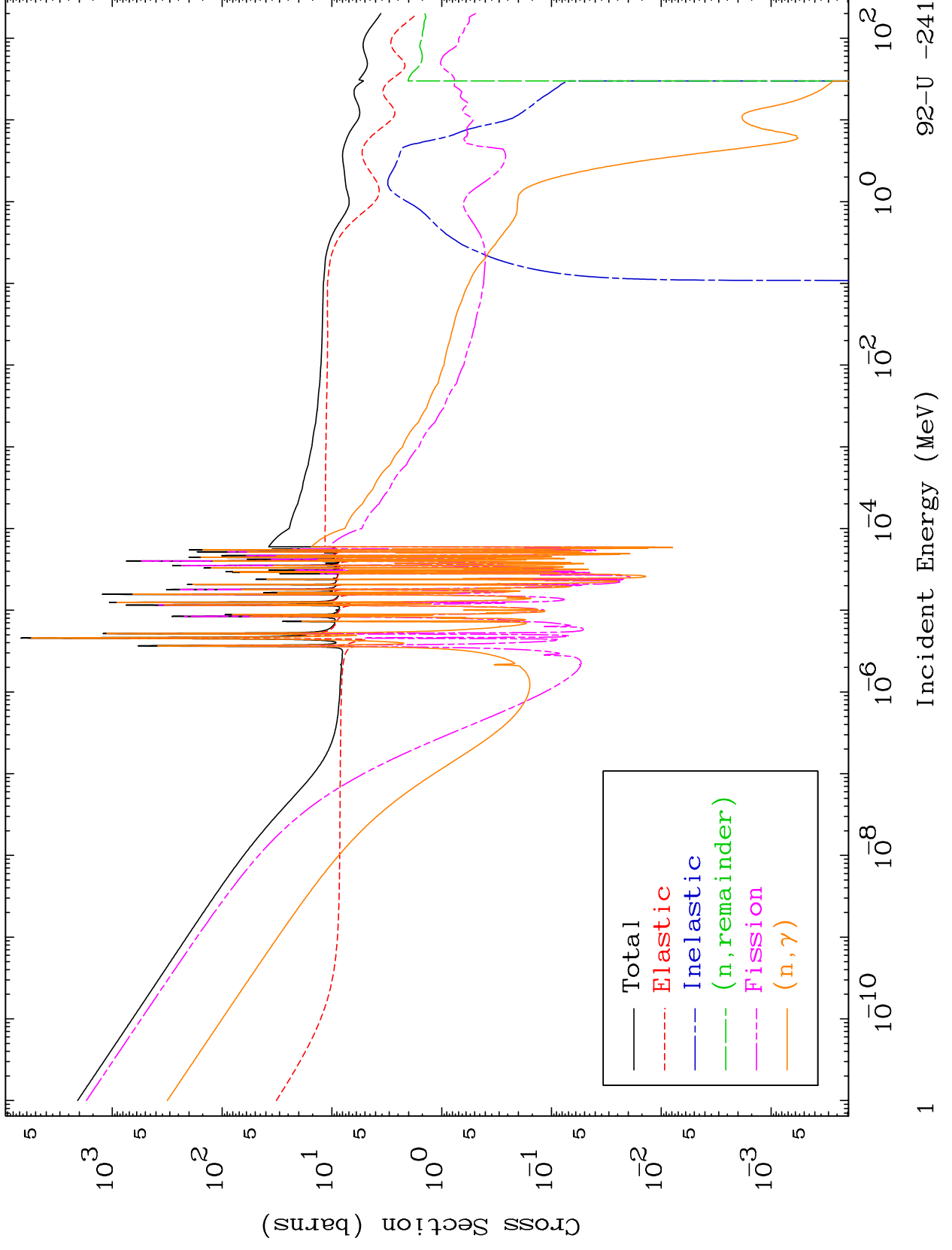
Press Mouse Button to Start

MAT 9246

Major

293 Kelvin Cross Sections

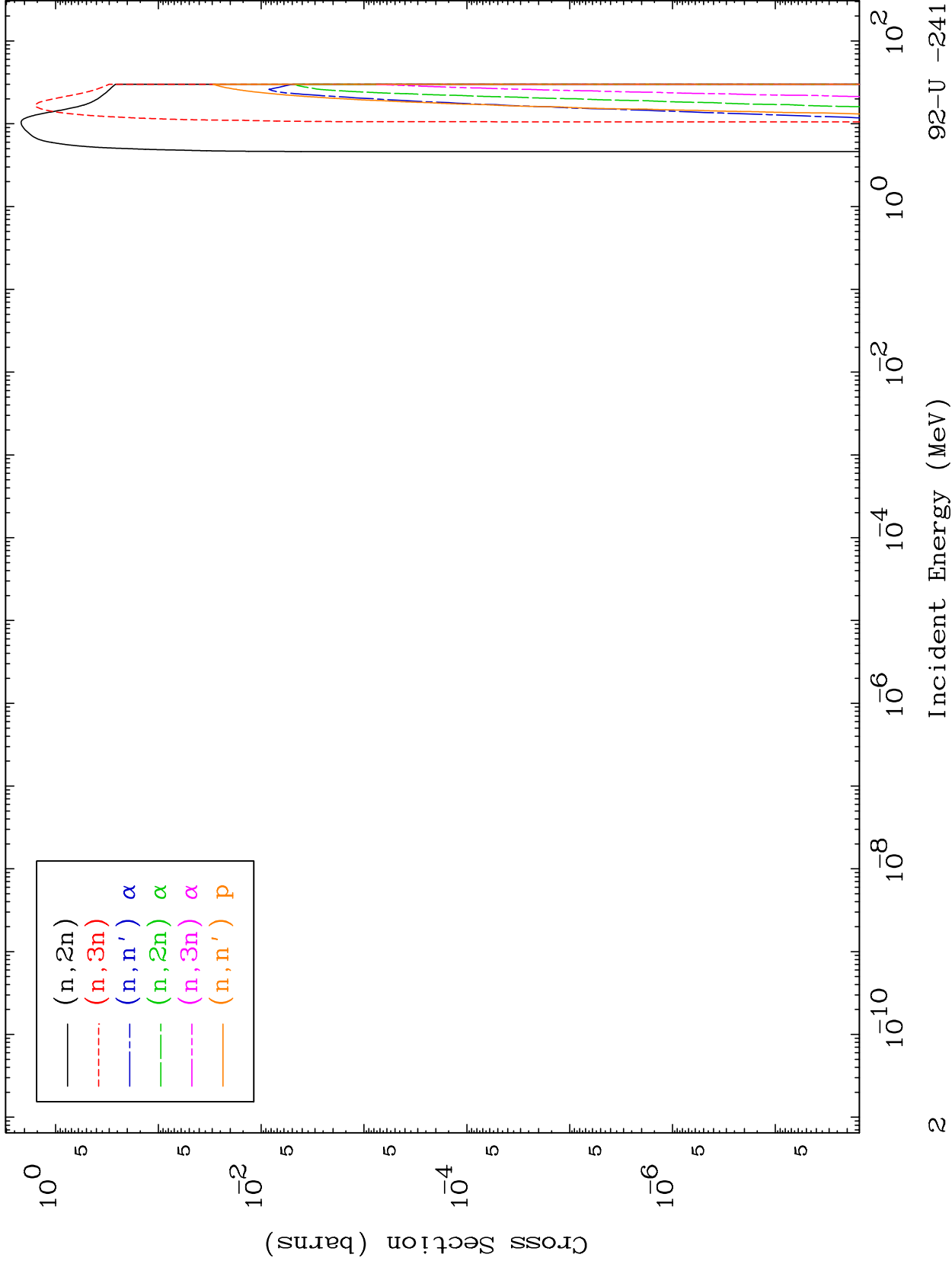
92-U -241



MAT 9246

Neutron Production  
293 Kelvin Cross Sections

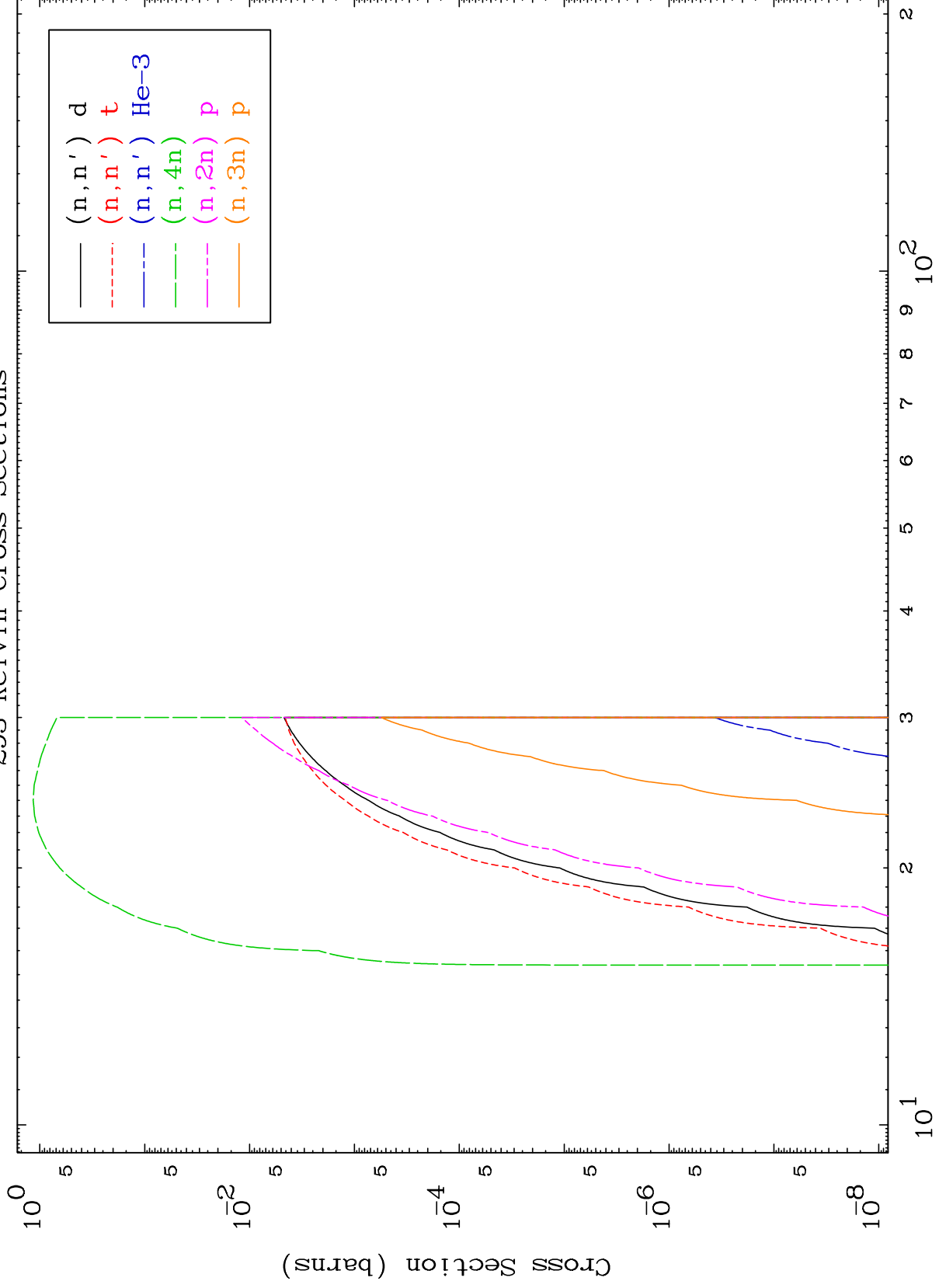
92-U -241



MAT 9246

Neutron Production  
293 Kelvin Cross Sections

92-U -241



92-U -241

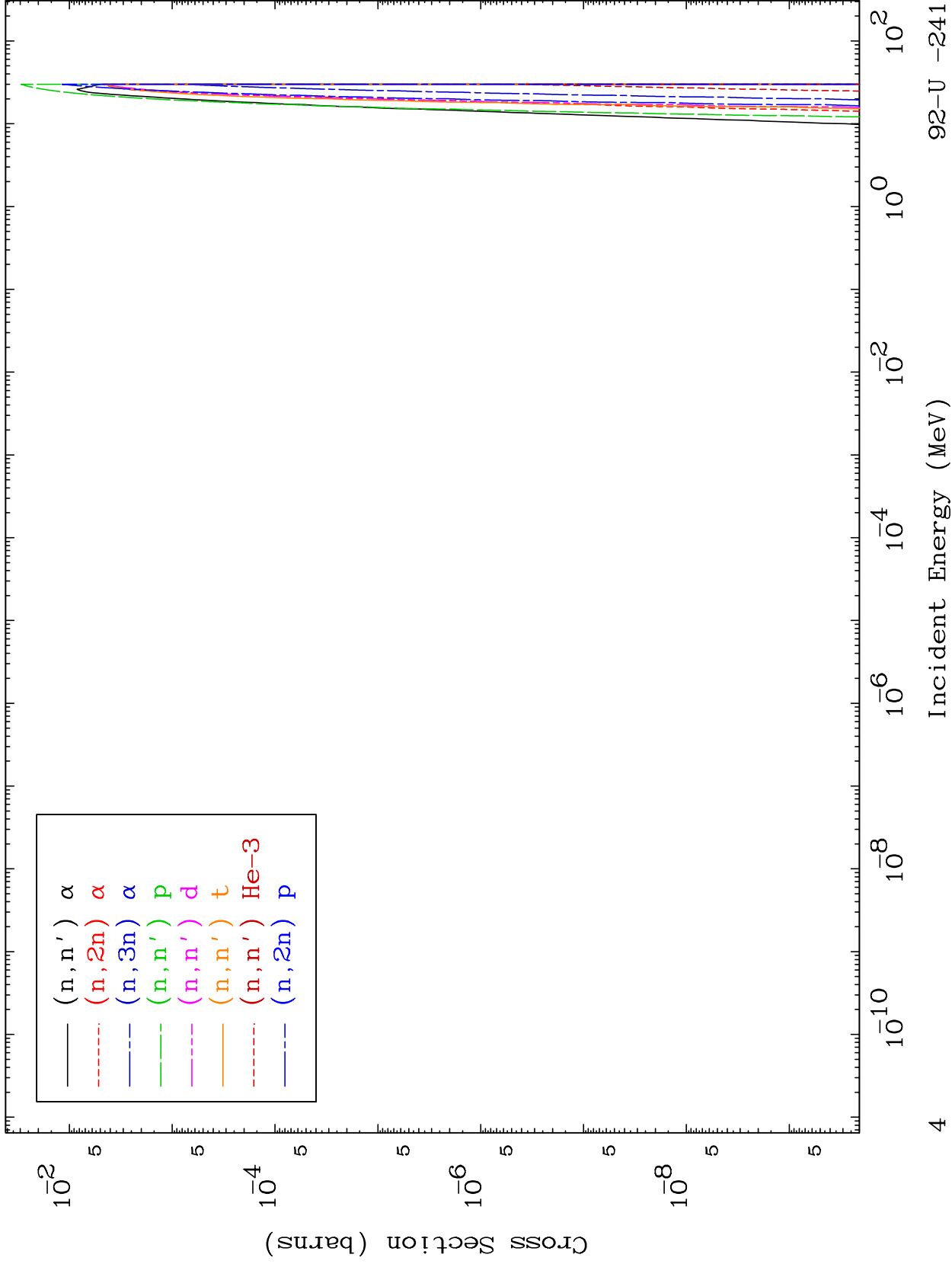
Incident Energy (MeV)

3

MAT 9246

Charged Particle  
293 Kelvin Cross Sections

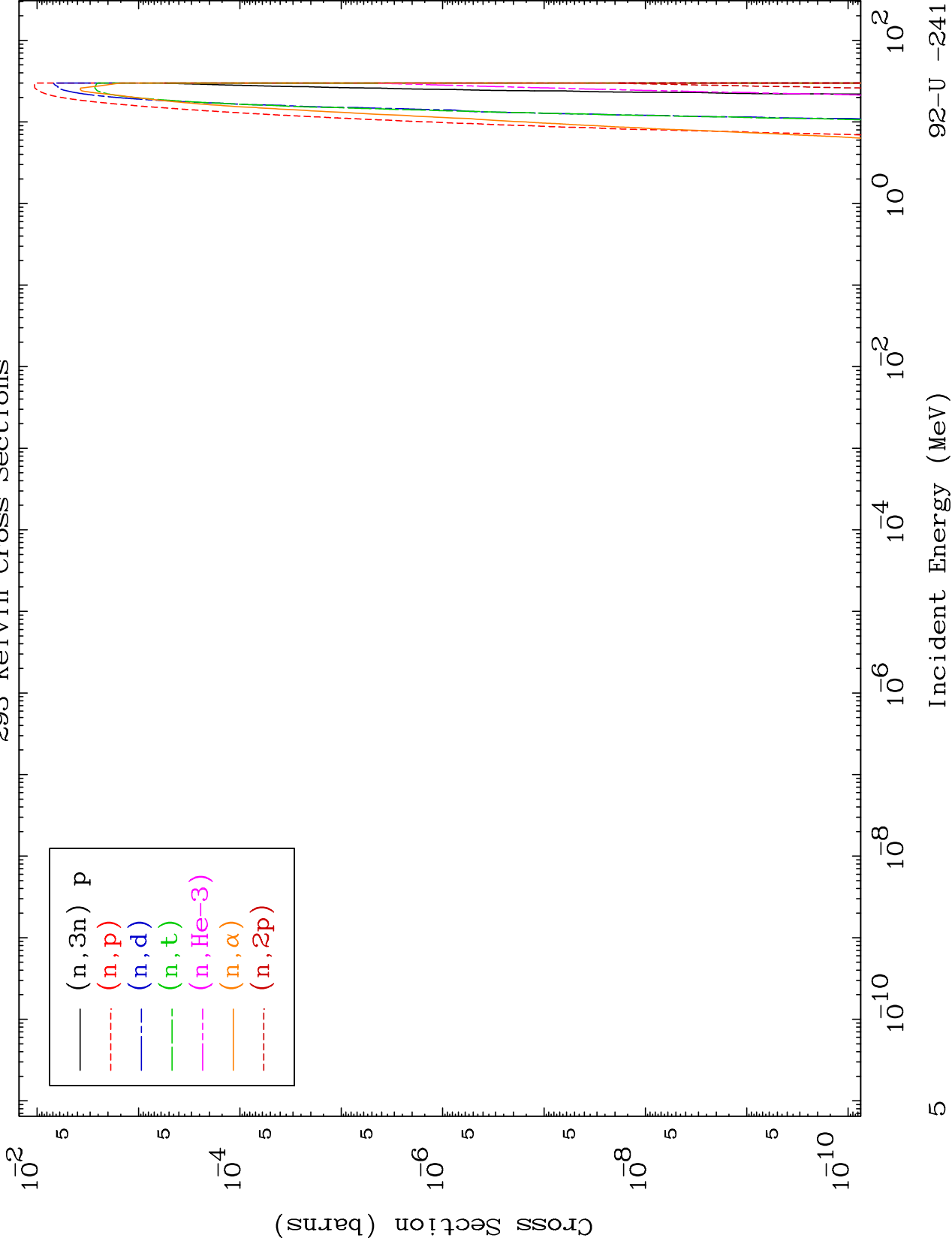
92-U -241



MAT 9246

Charged Particle  
293 Kelvin Cross Sections

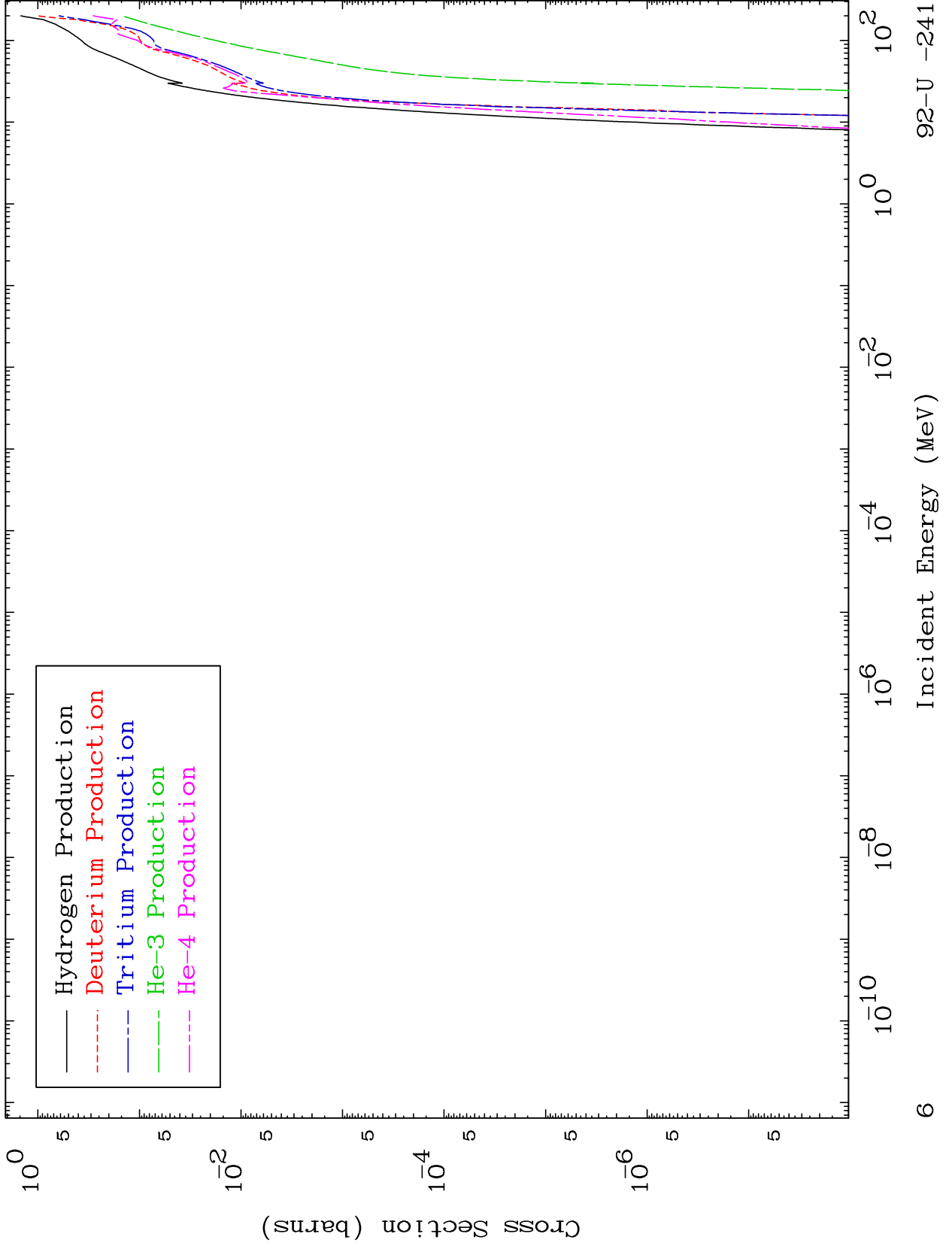
92-U -241



MAT 9246

Particle Production  
293 Kelvin Cross Sections

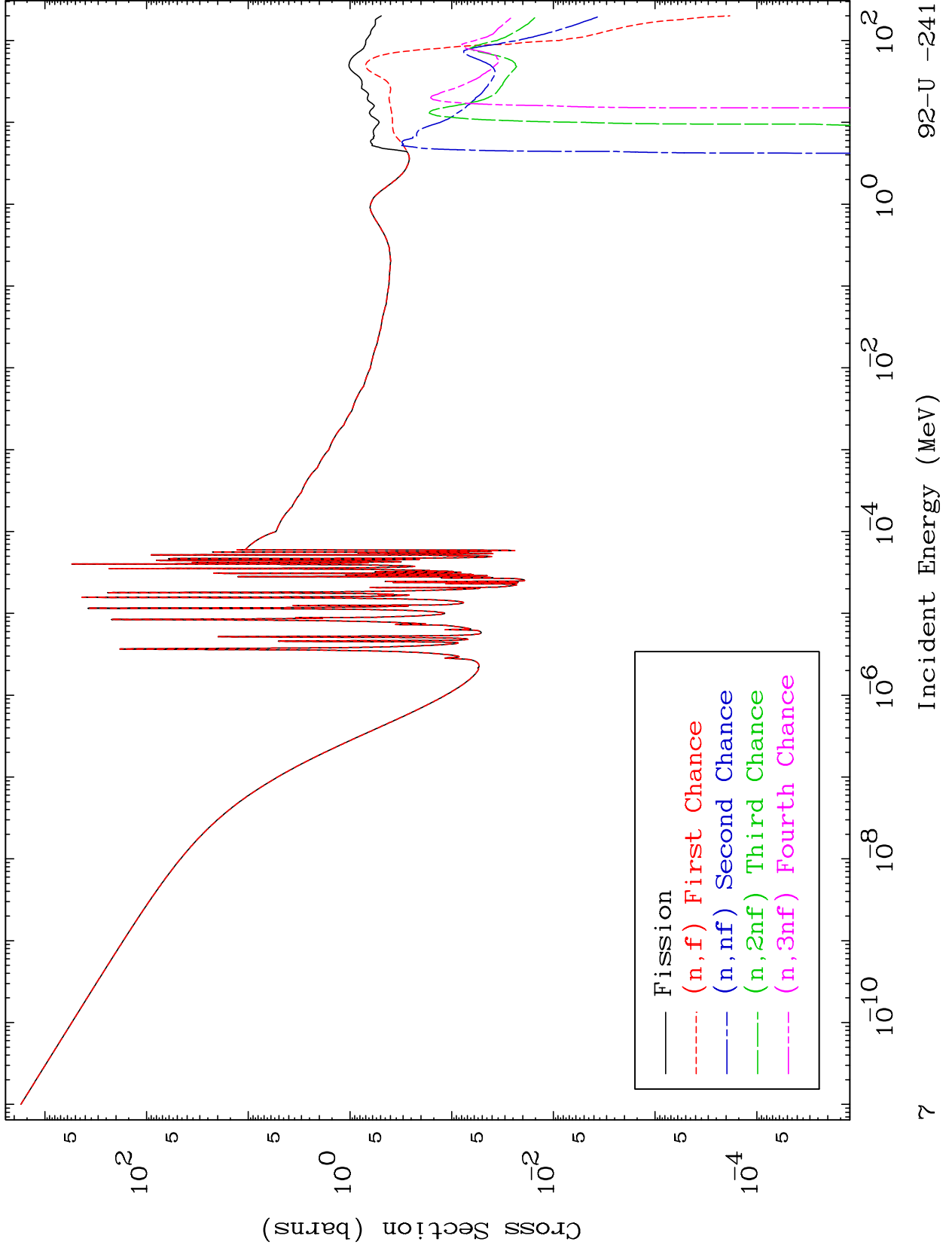
92-U -241



MAT 9246

Fission  
293 Kelvin Cross Sections

92-U -241



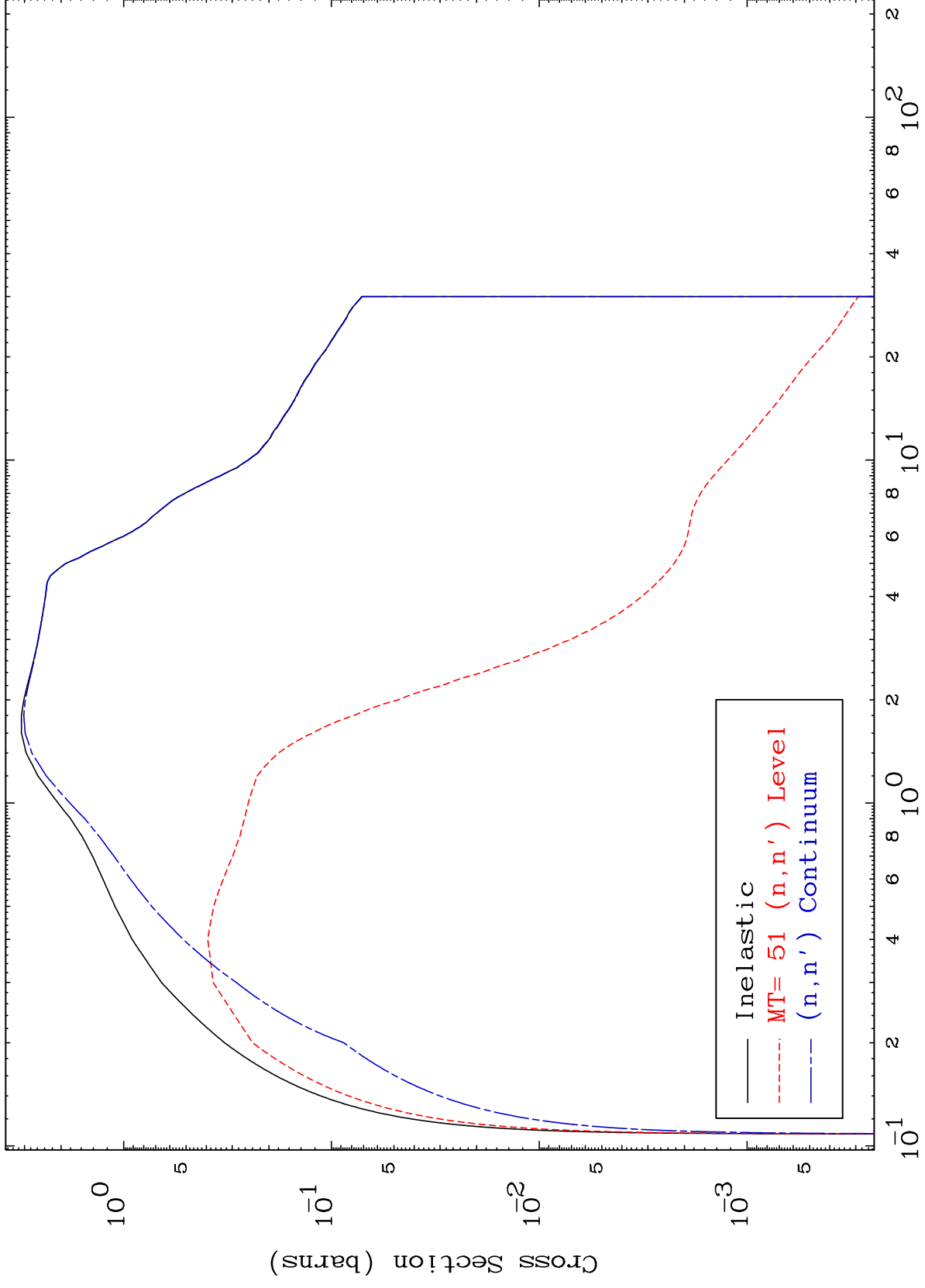


MAT 9246

(n,n') Level

92-U -241

293 Kelvin Cross Sections



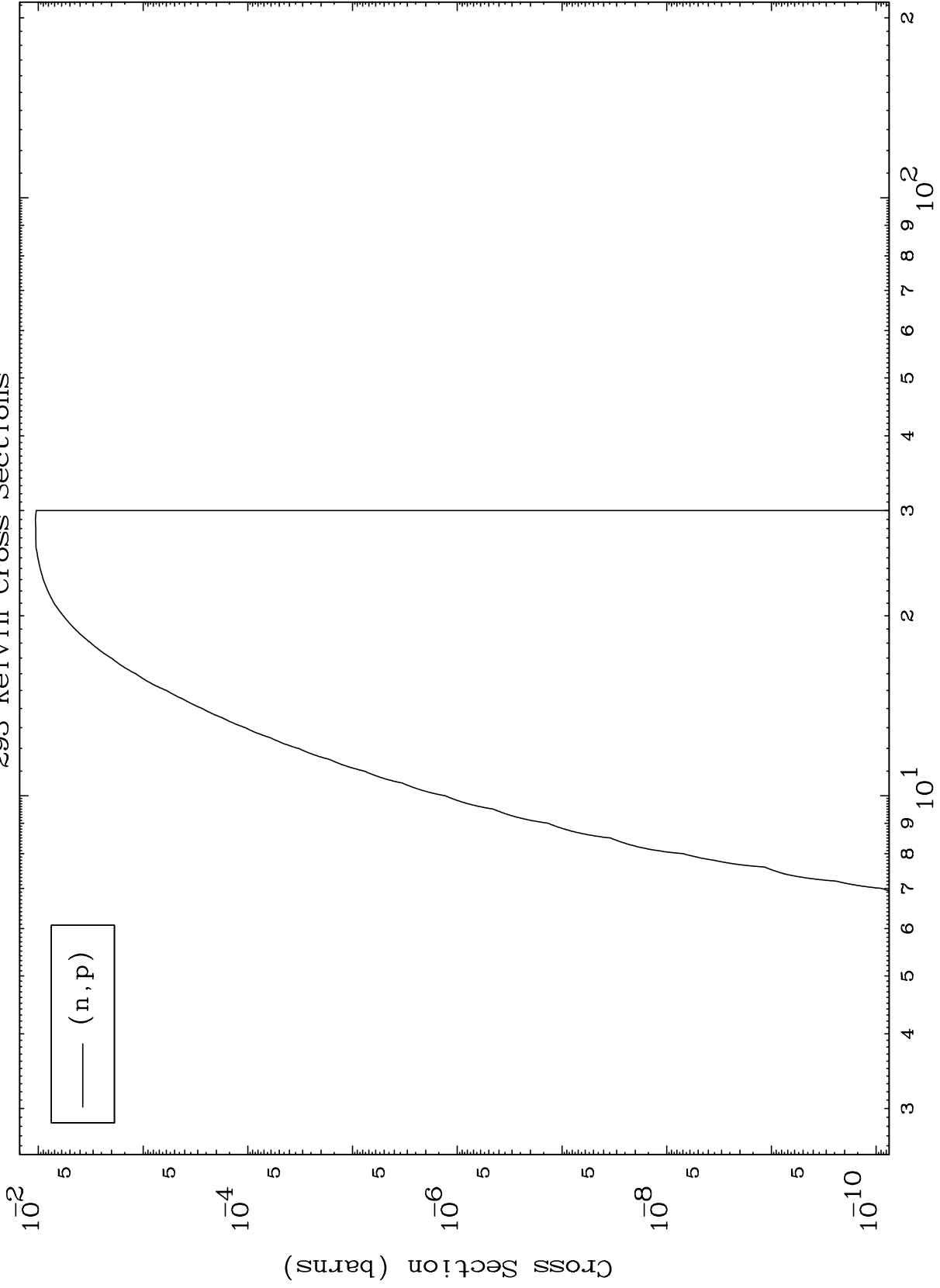
Incident Energy (MeV)

92-U -241

MAT 9246

(n,p) Levels  
293 Kelvin Cross Sections

92-U -241



9

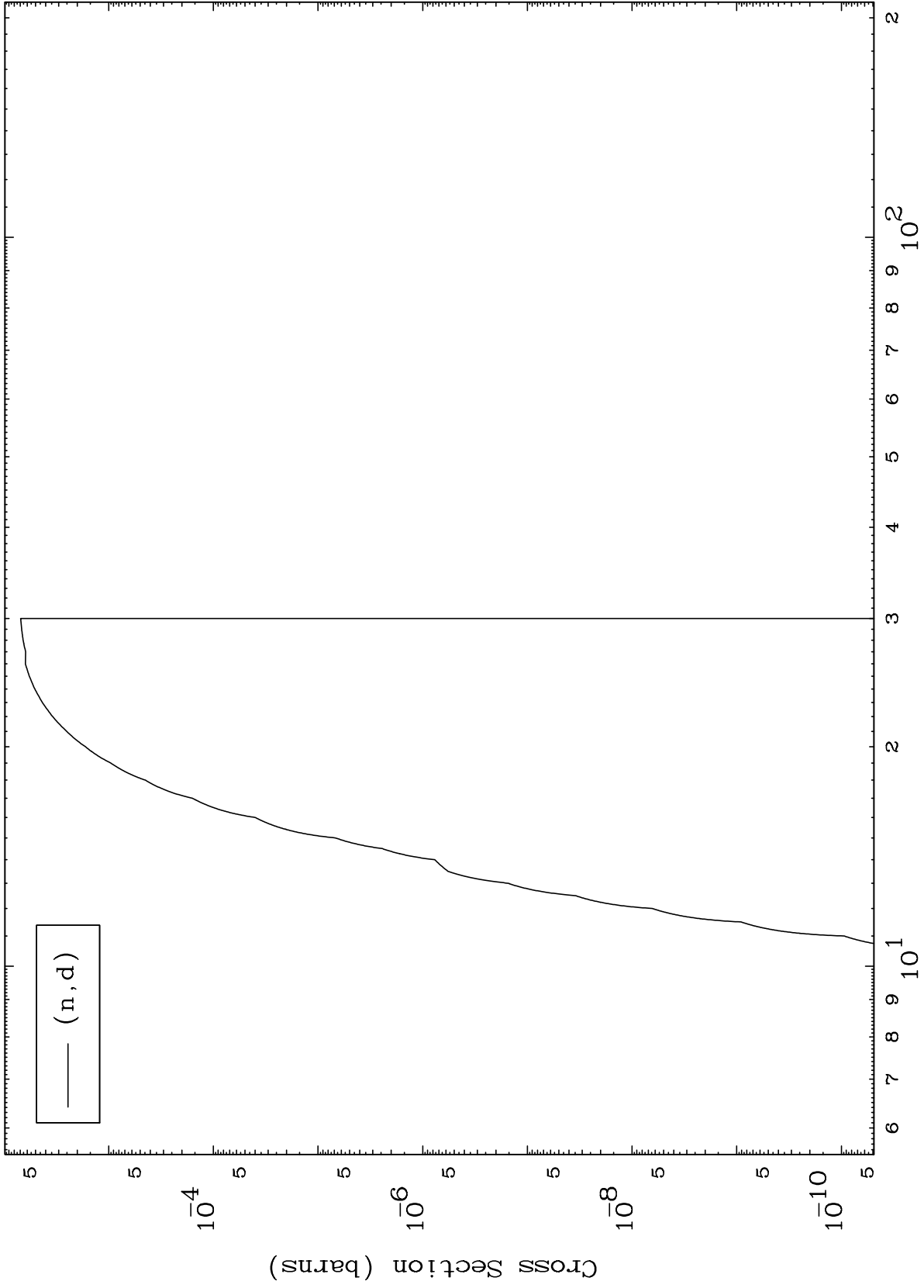
Incident Energy (MeV)

92-U -241

MAT 9246

(n,d) Levels  
293 Kelvin Cross Sections

92-U -241



10

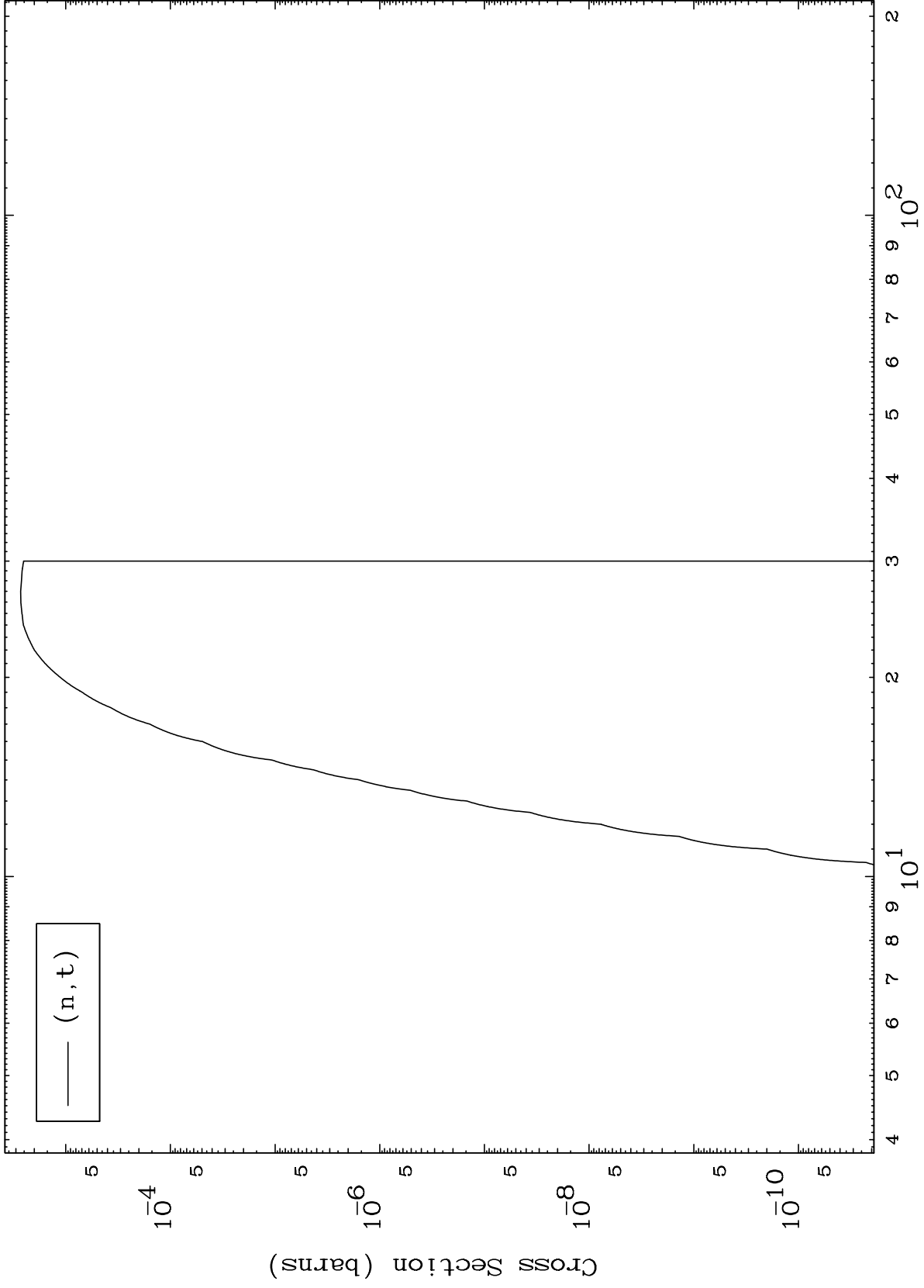
Incident Energy (MeV)

92-U -241

MAT 9246

(n,t) Levels  
293 Kelvin Cross Sections

92-U -241



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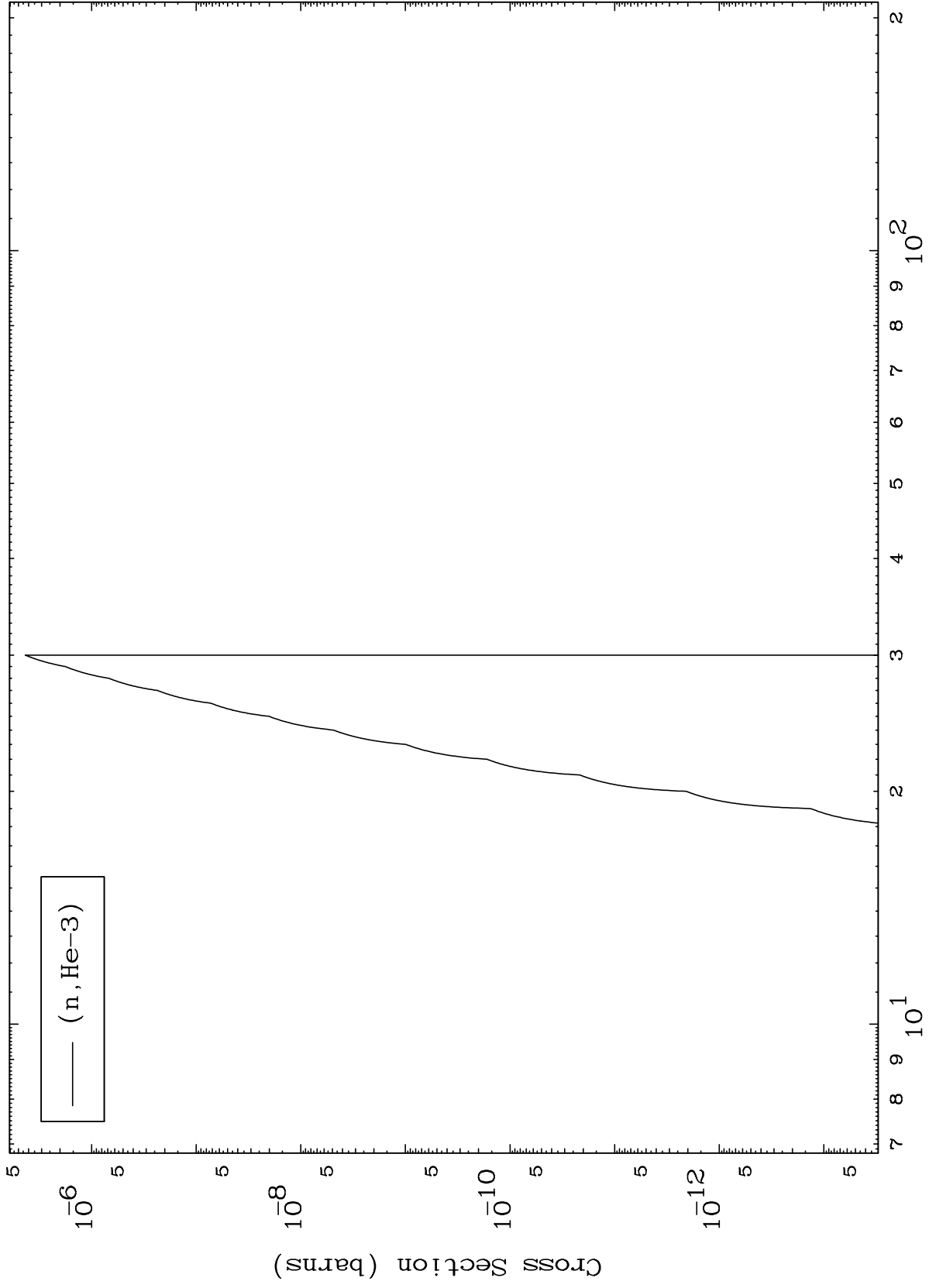
Incident Energy (MeV)

92-U -241

MAT 9246

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -241



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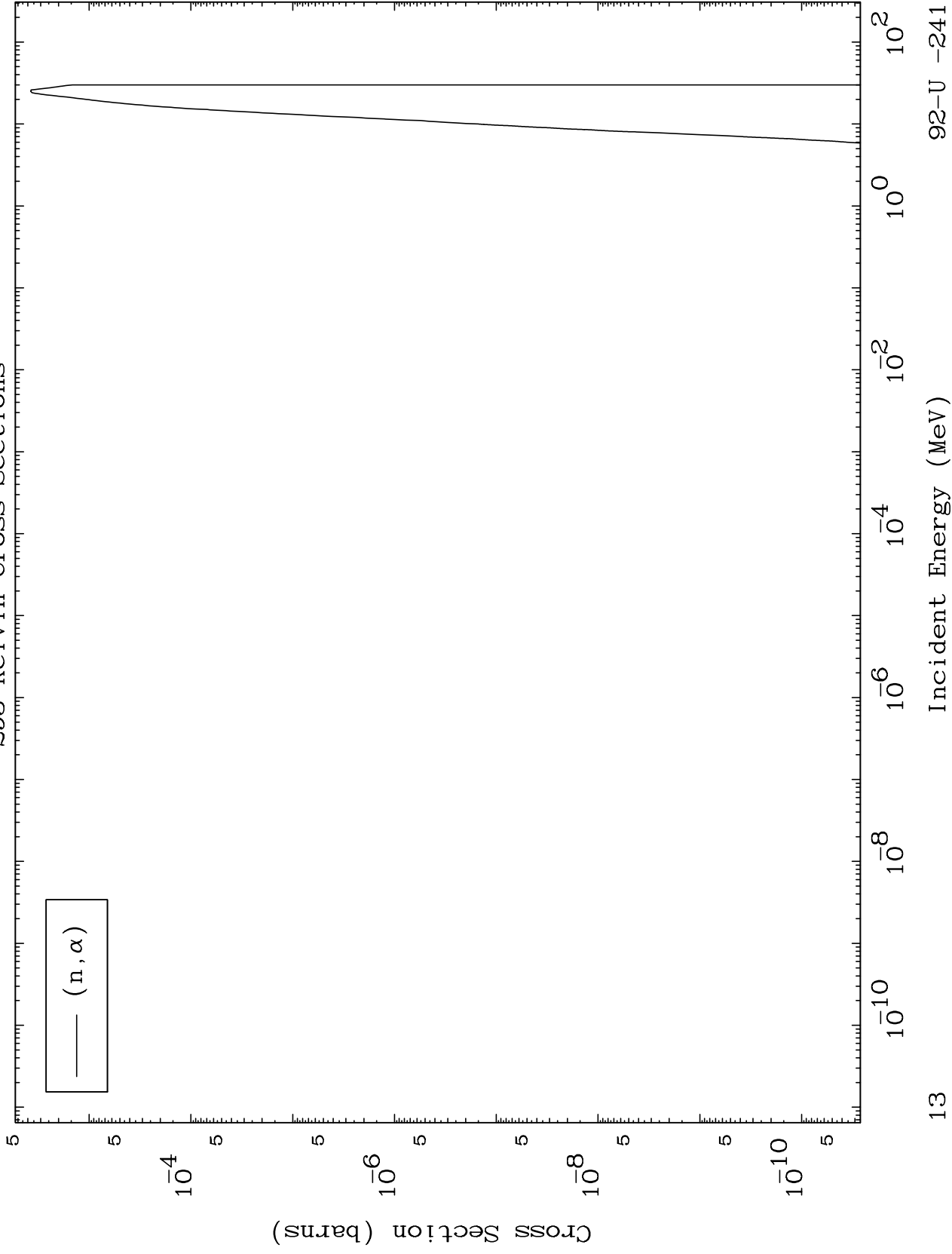
Incident Energy (MeV)

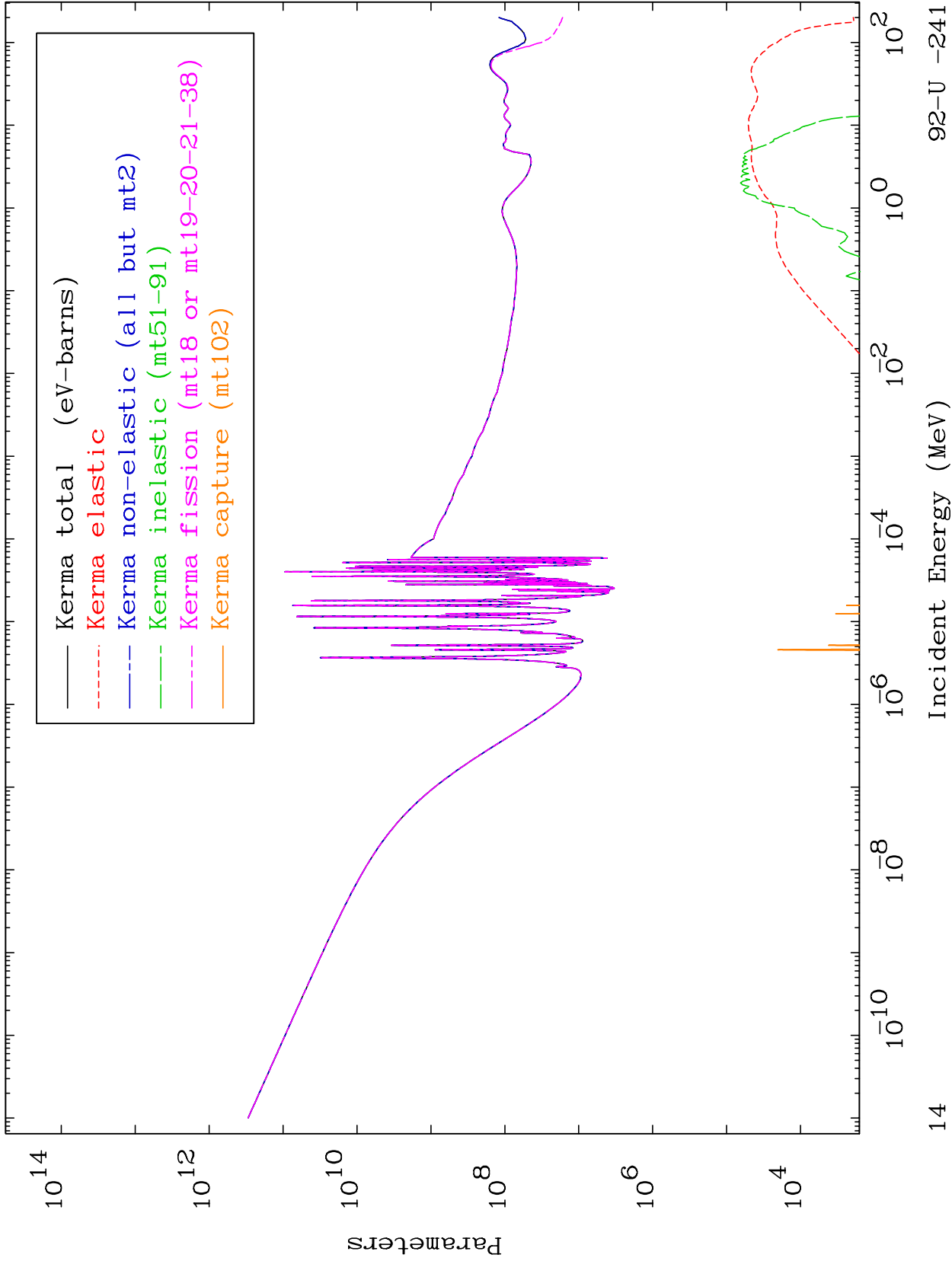
92-U -241

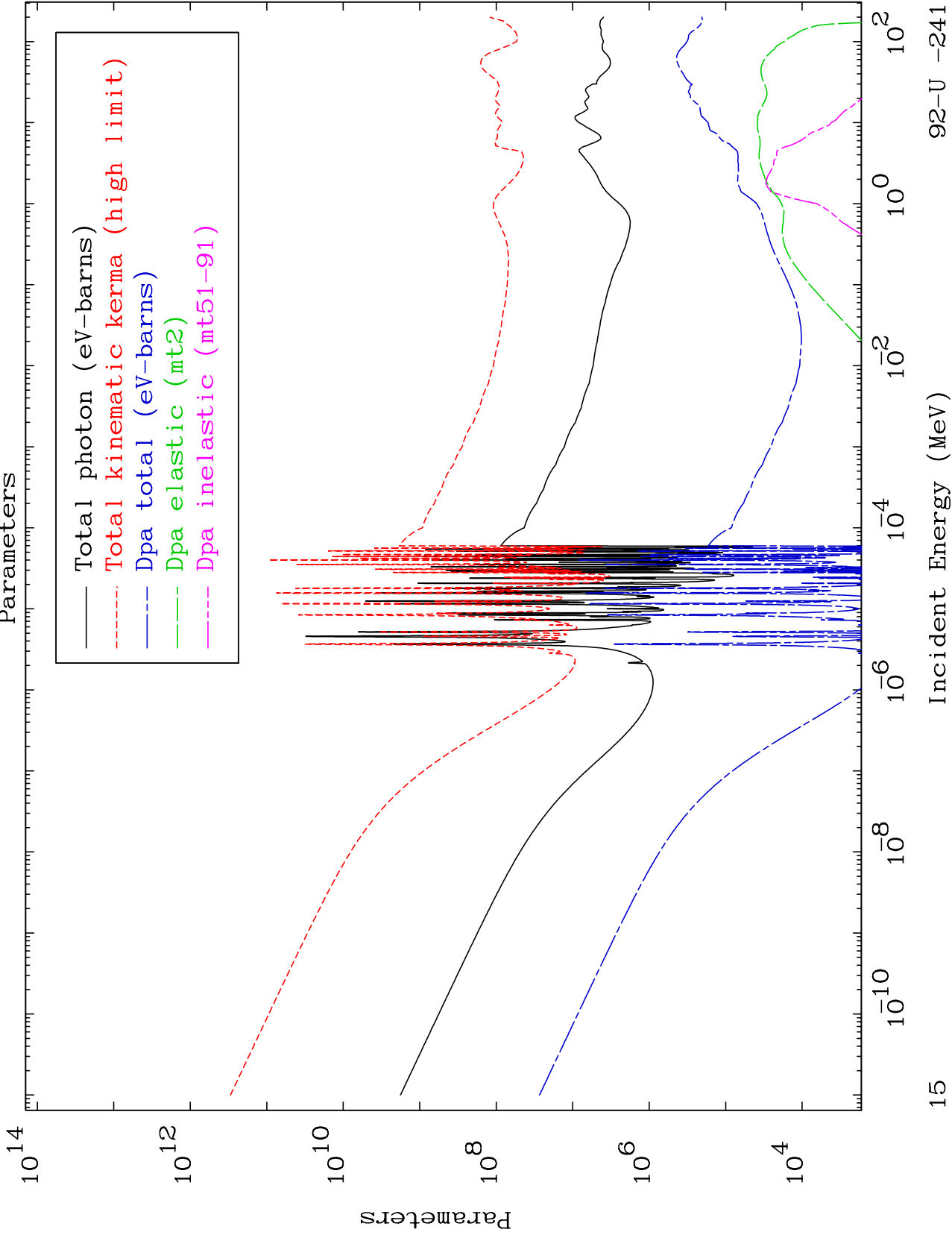
MAT 9246

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

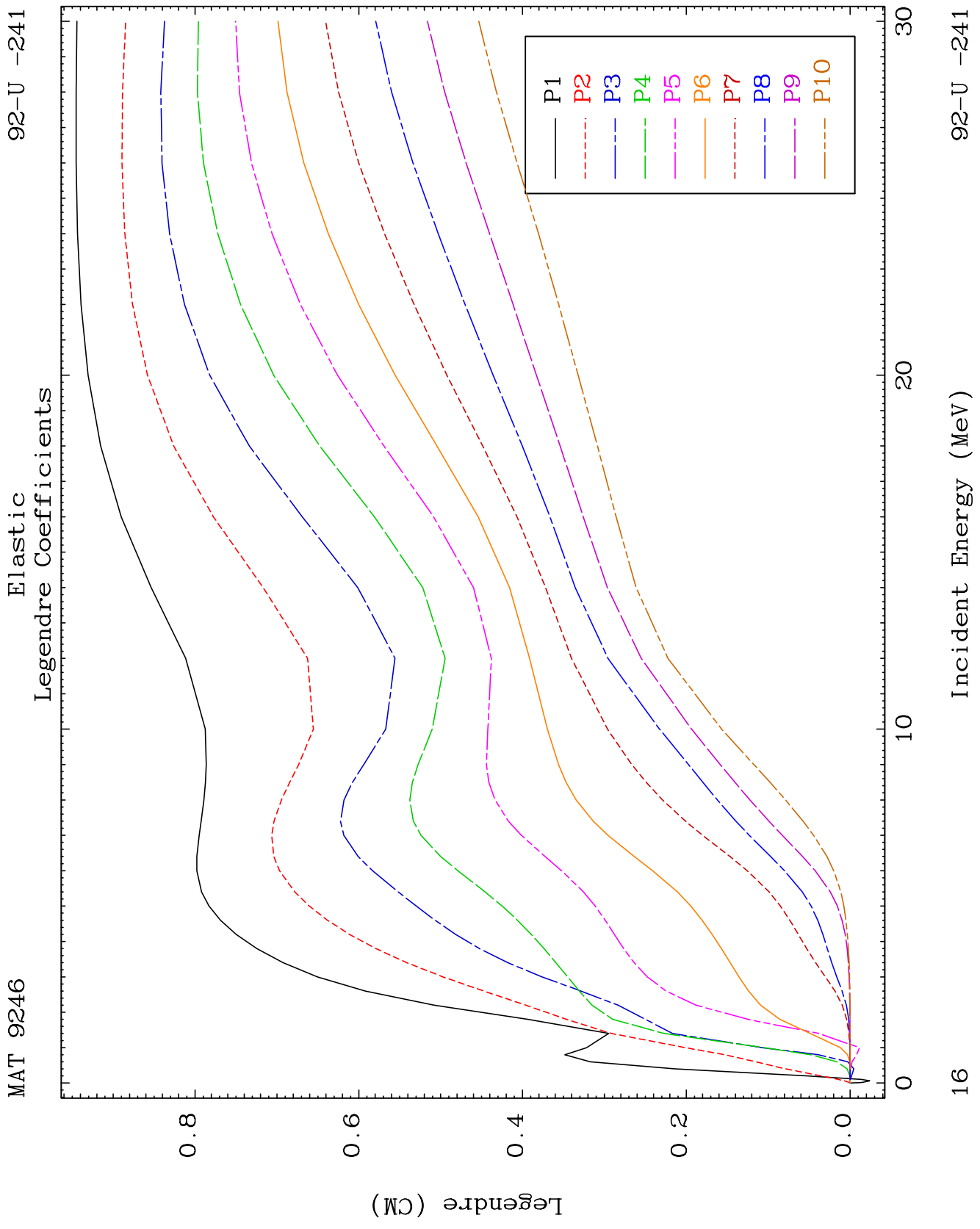
92-U -241







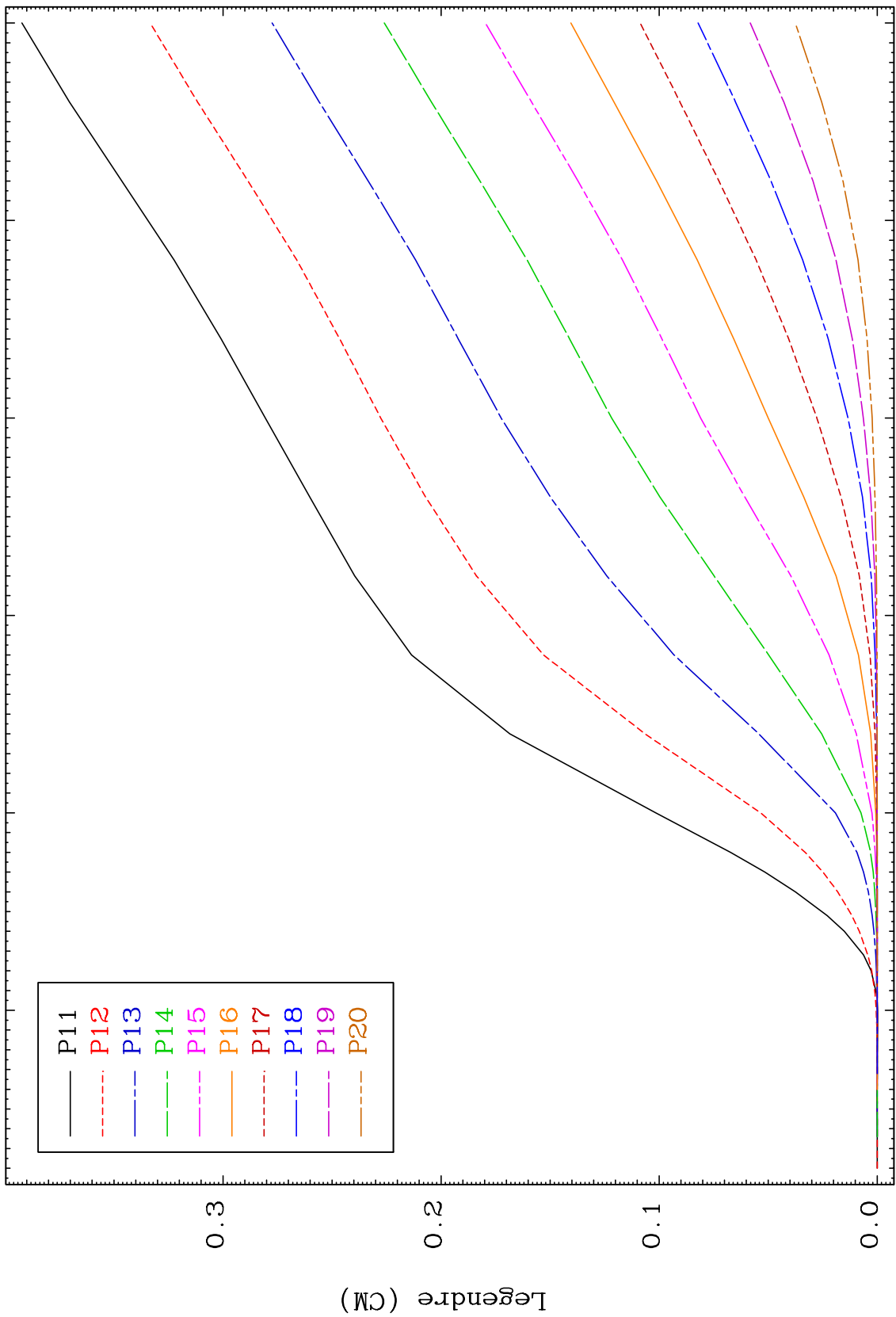




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Elastic Legendre Coefficients

92-U -241



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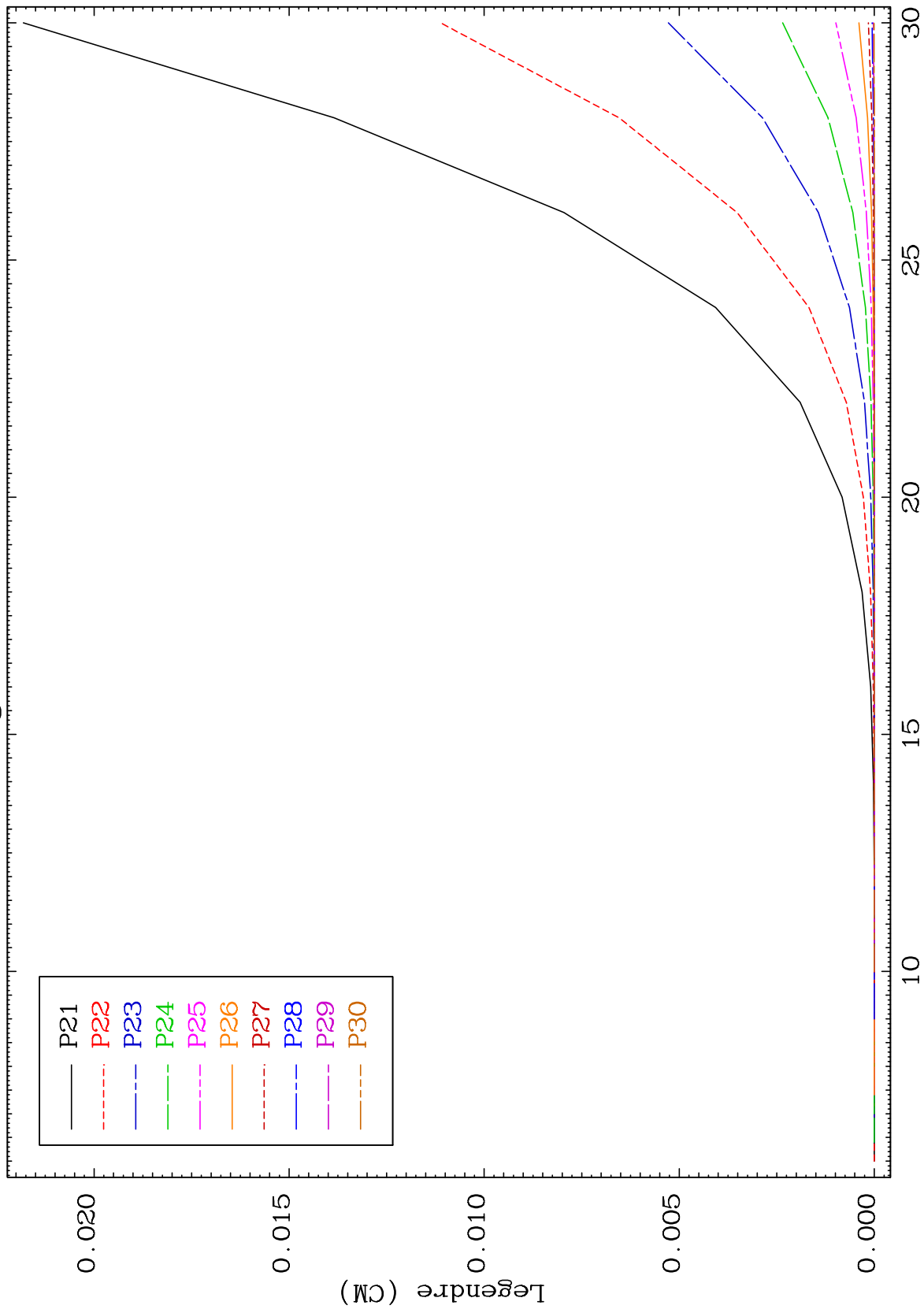
Incident Energy (MeV)

92-U -241

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Elastic  
Legendre Coefficients

92-U -241



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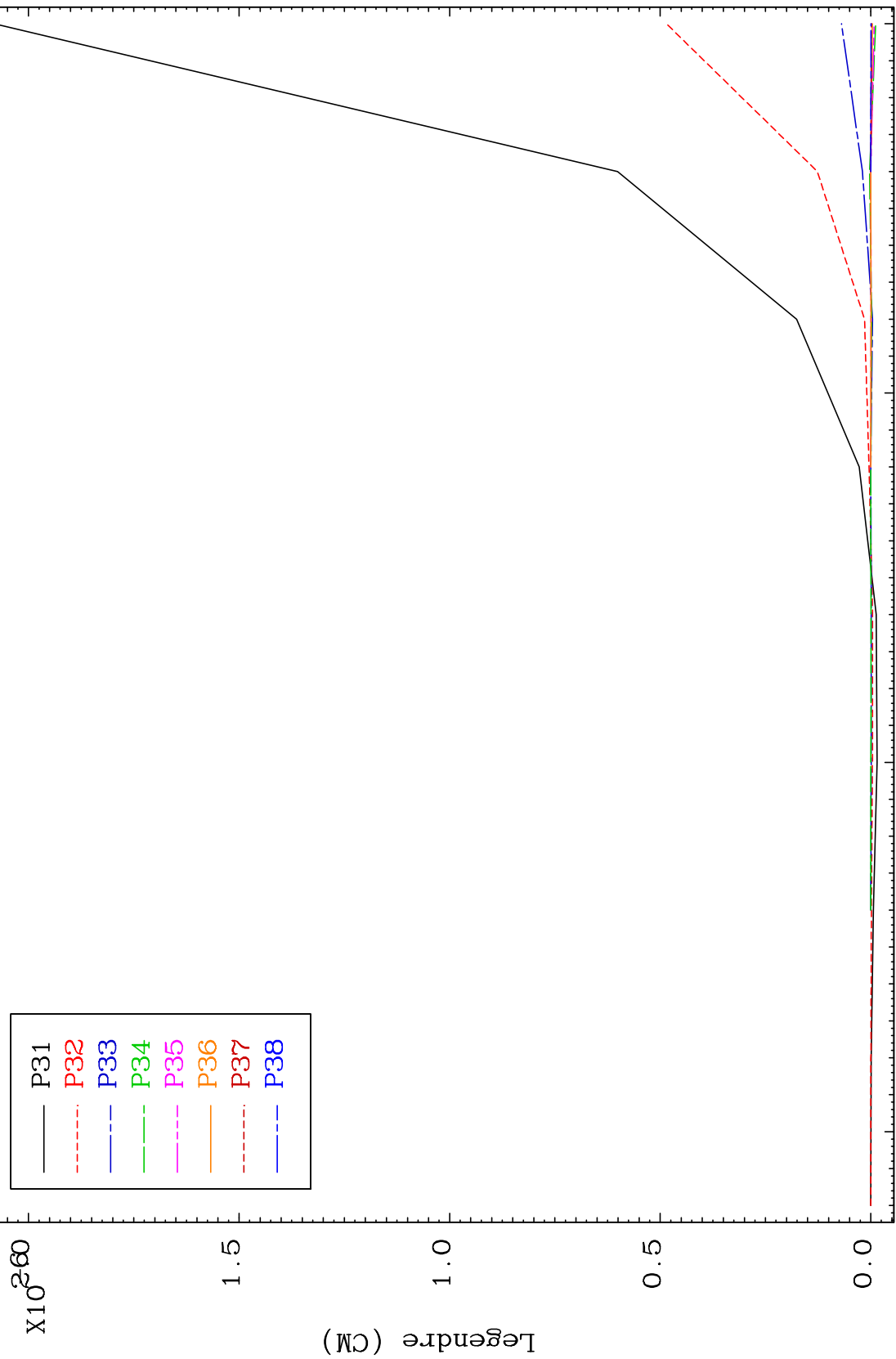
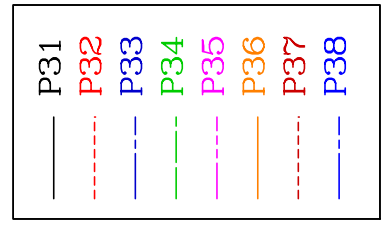
Incident Energy (MeV)

92-U -241

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Elastic Legendre Coefficients

92-U -241



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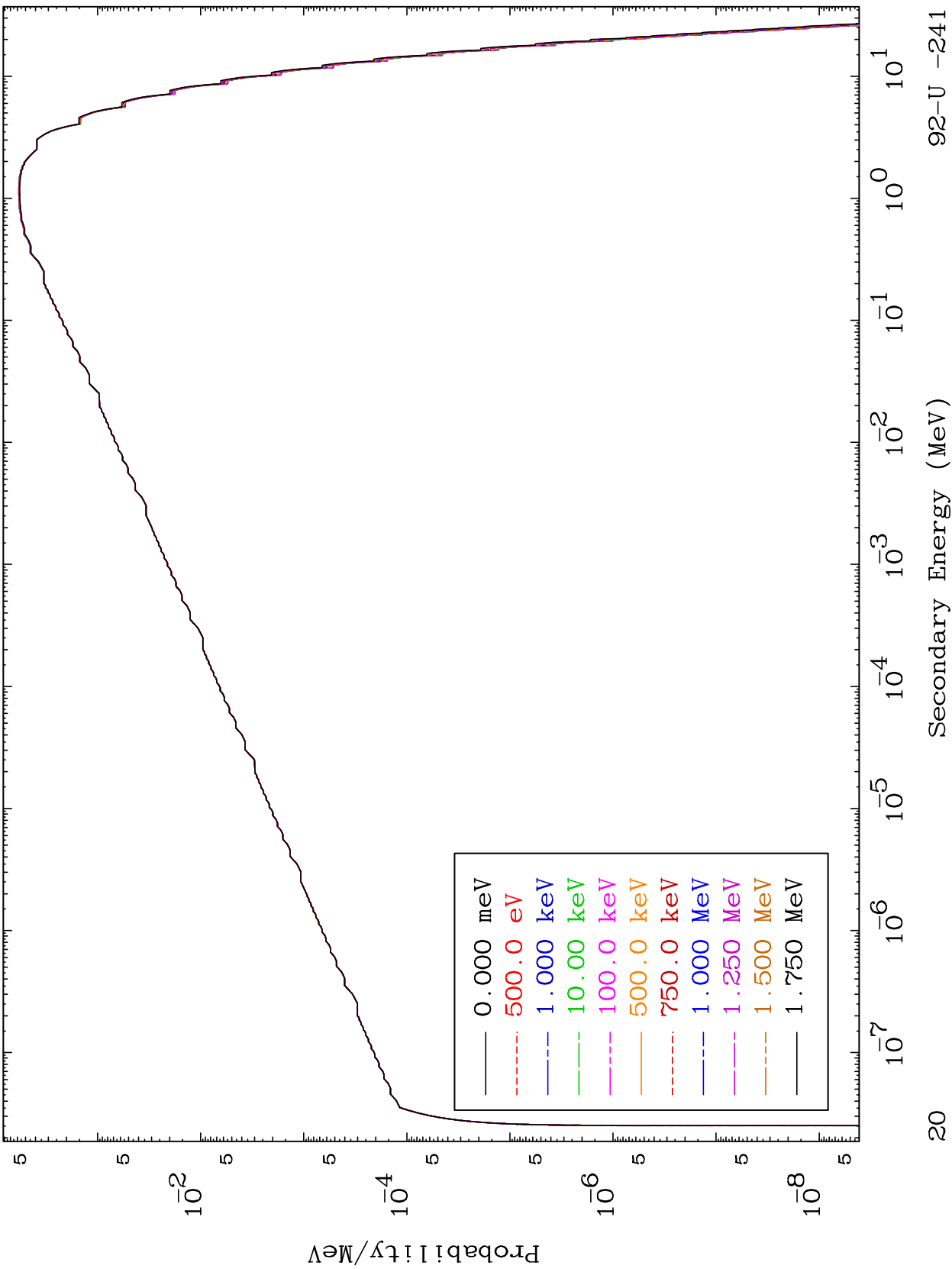
Incident Energy (MeV)

92-U -241

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### Fission Energy Distribution

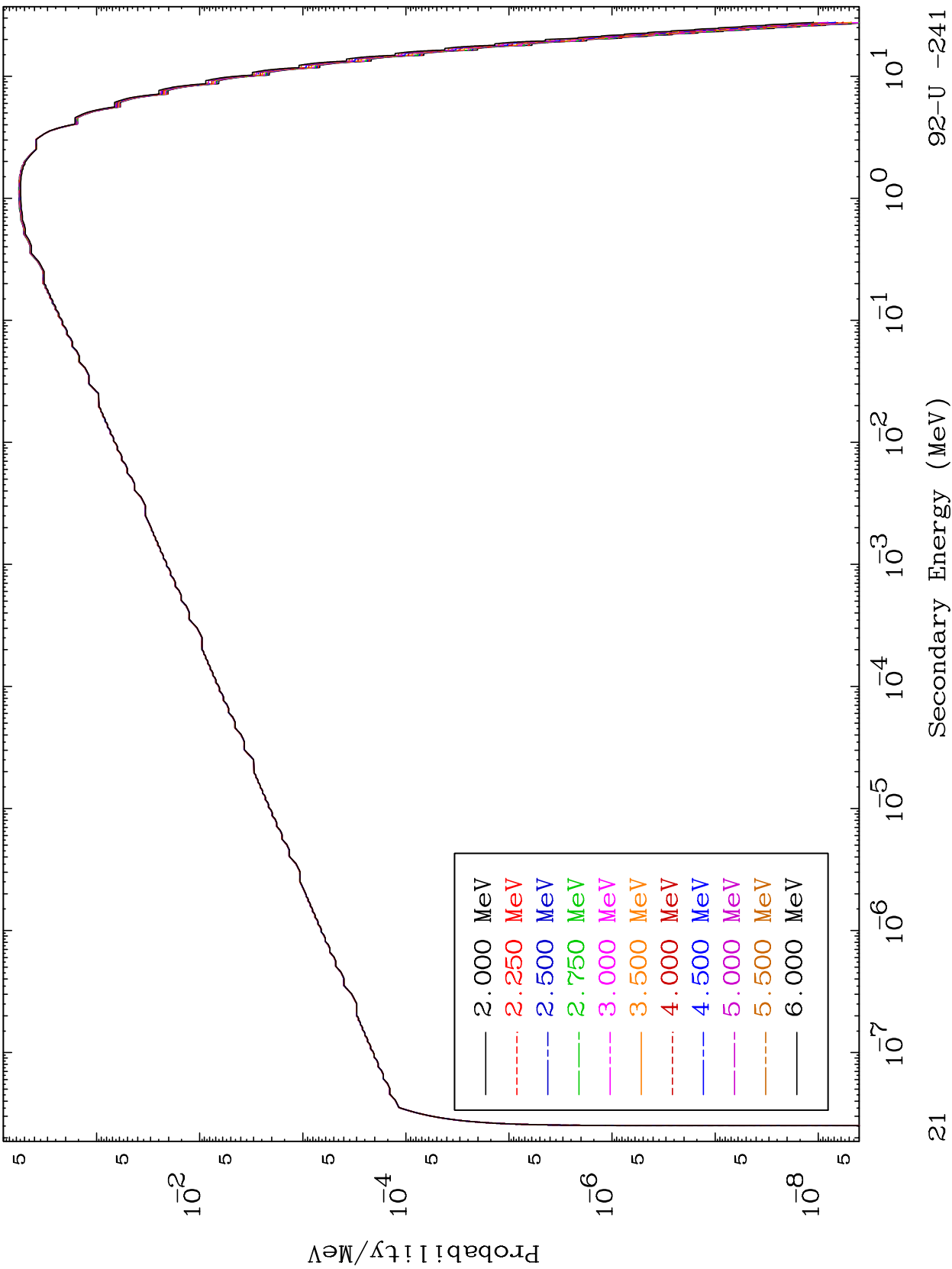
92-U -241



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### Fission Energy Distribution

92-U -241



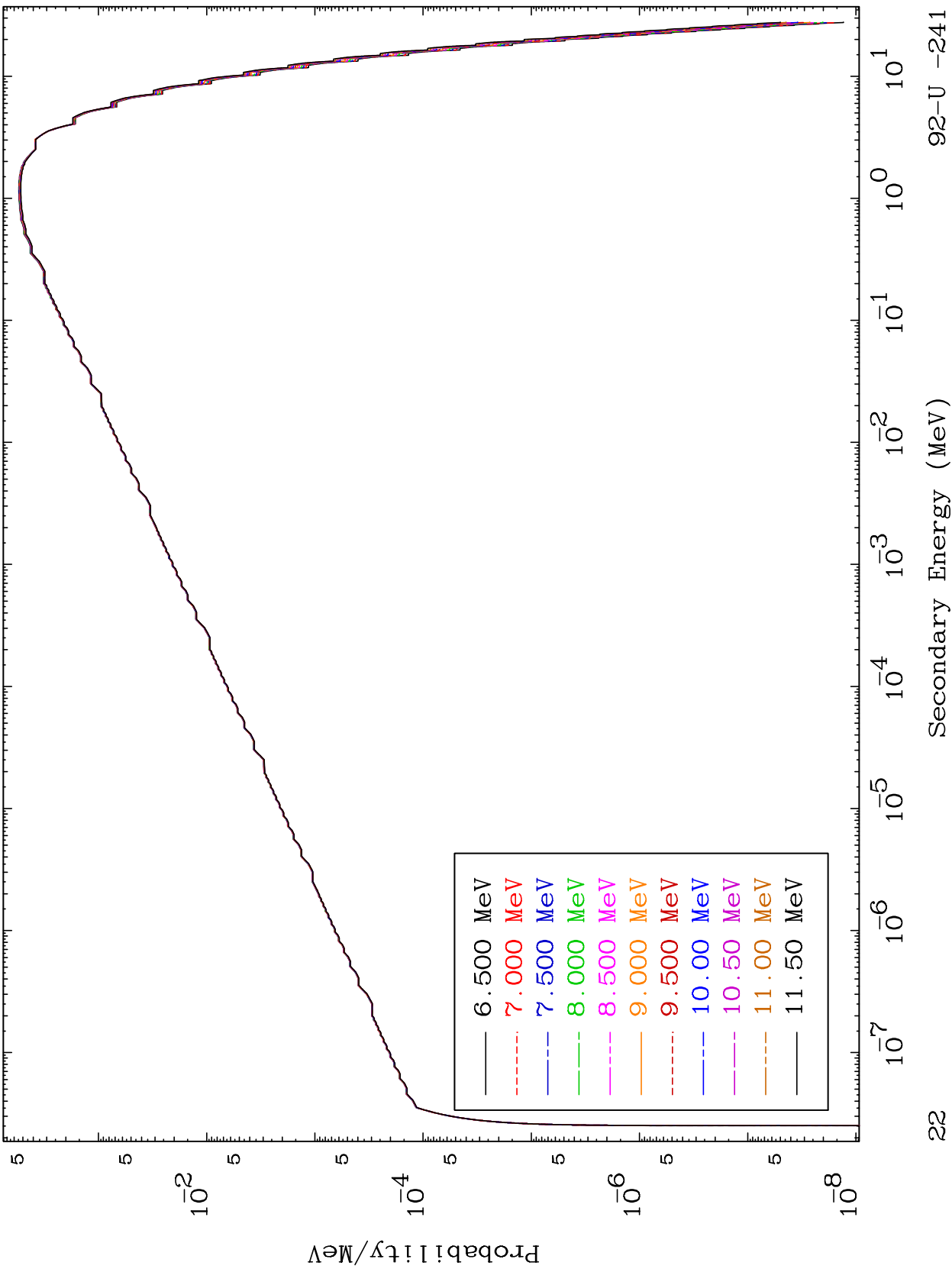
92-U -241

Secondary Energy (MeV)

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### Fission Energy Distribution

92-U -241

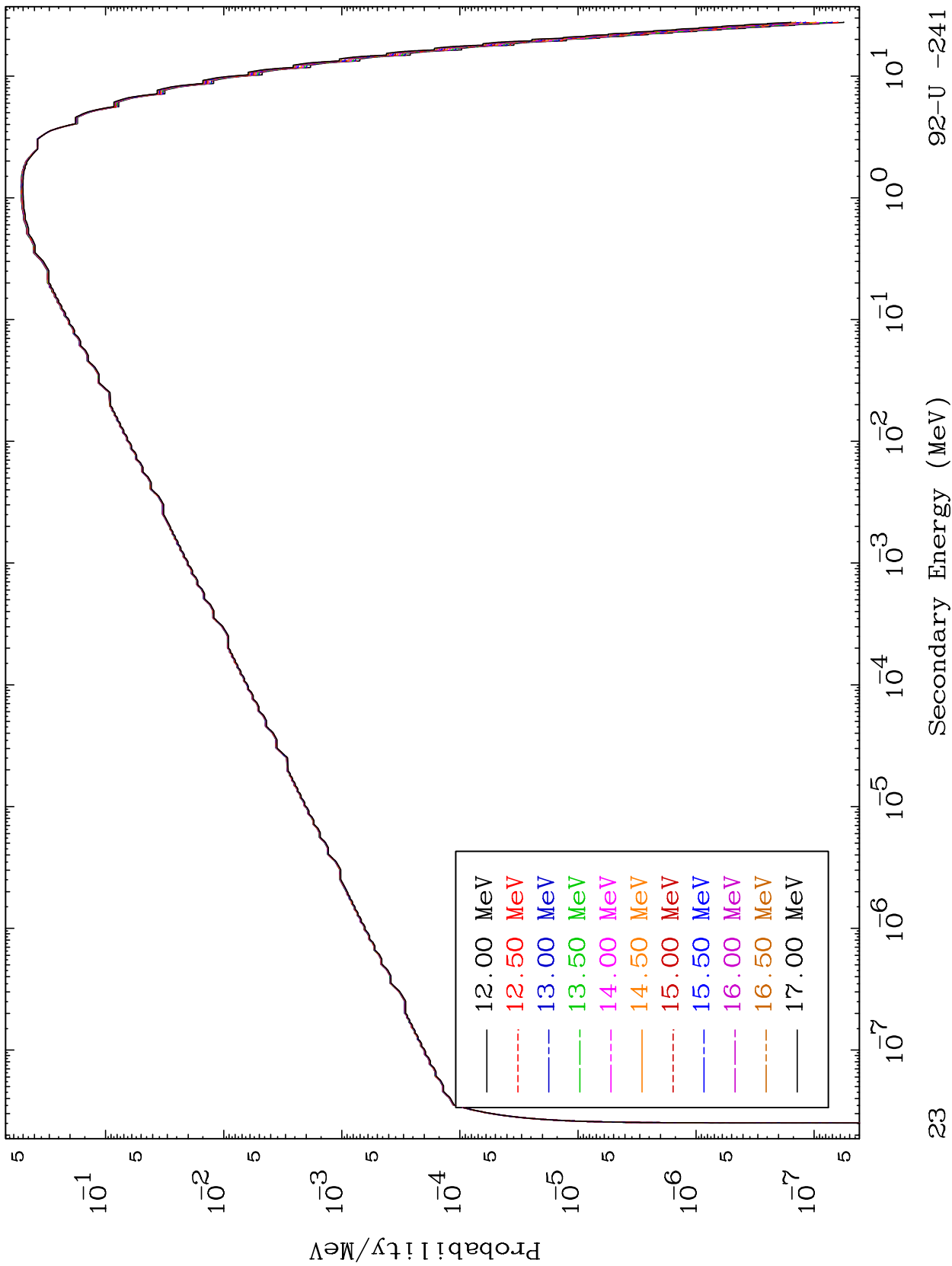


92-U -241

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### Fission Energy Distribution

92-U -241





MAT 9246

### Fission Energy Distribution

92-U -241

