

Program EVALPLOT  
(Version 2017-1)

by

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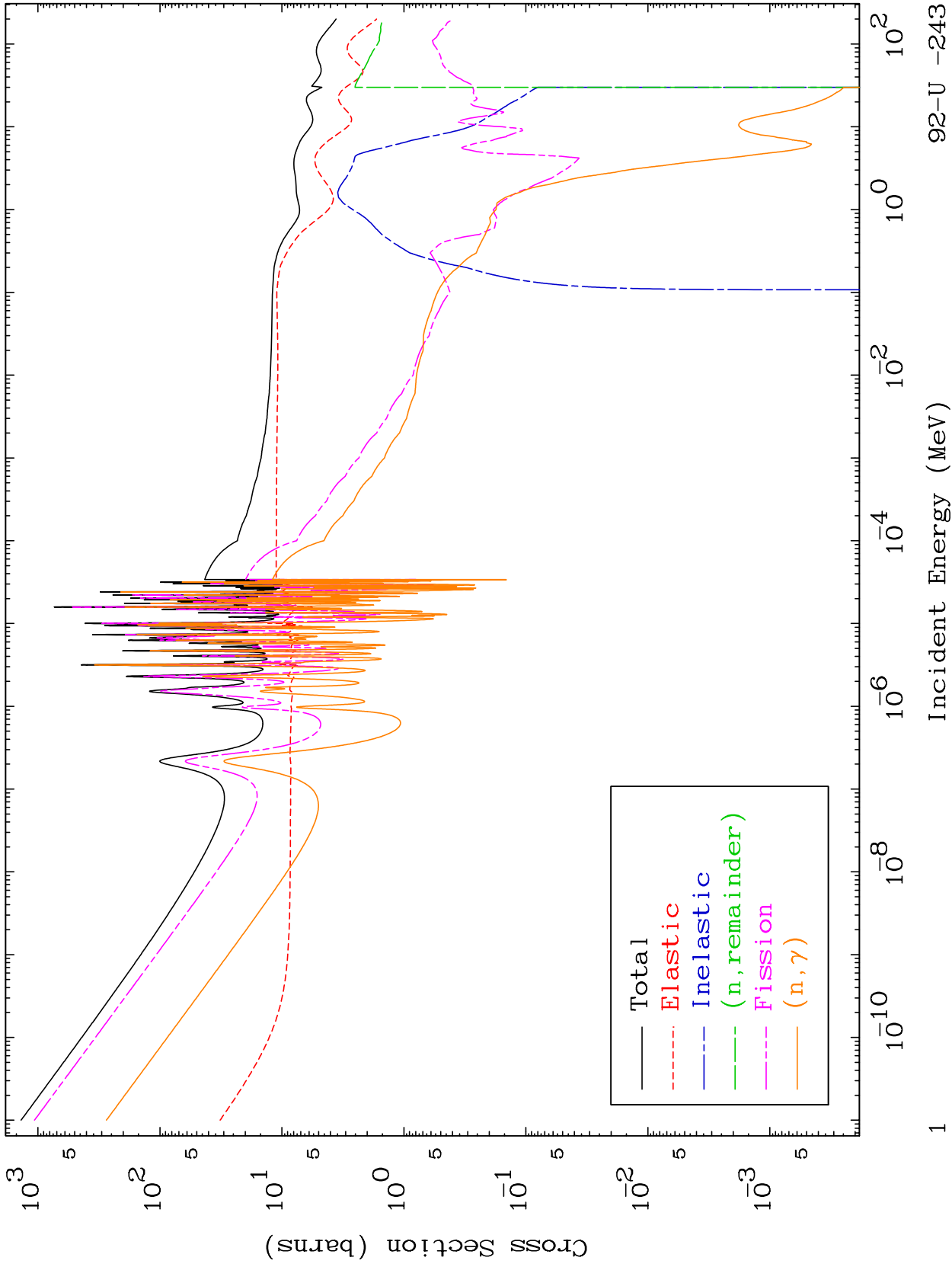
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9252

Major  
293 Kelvin Cross Sections

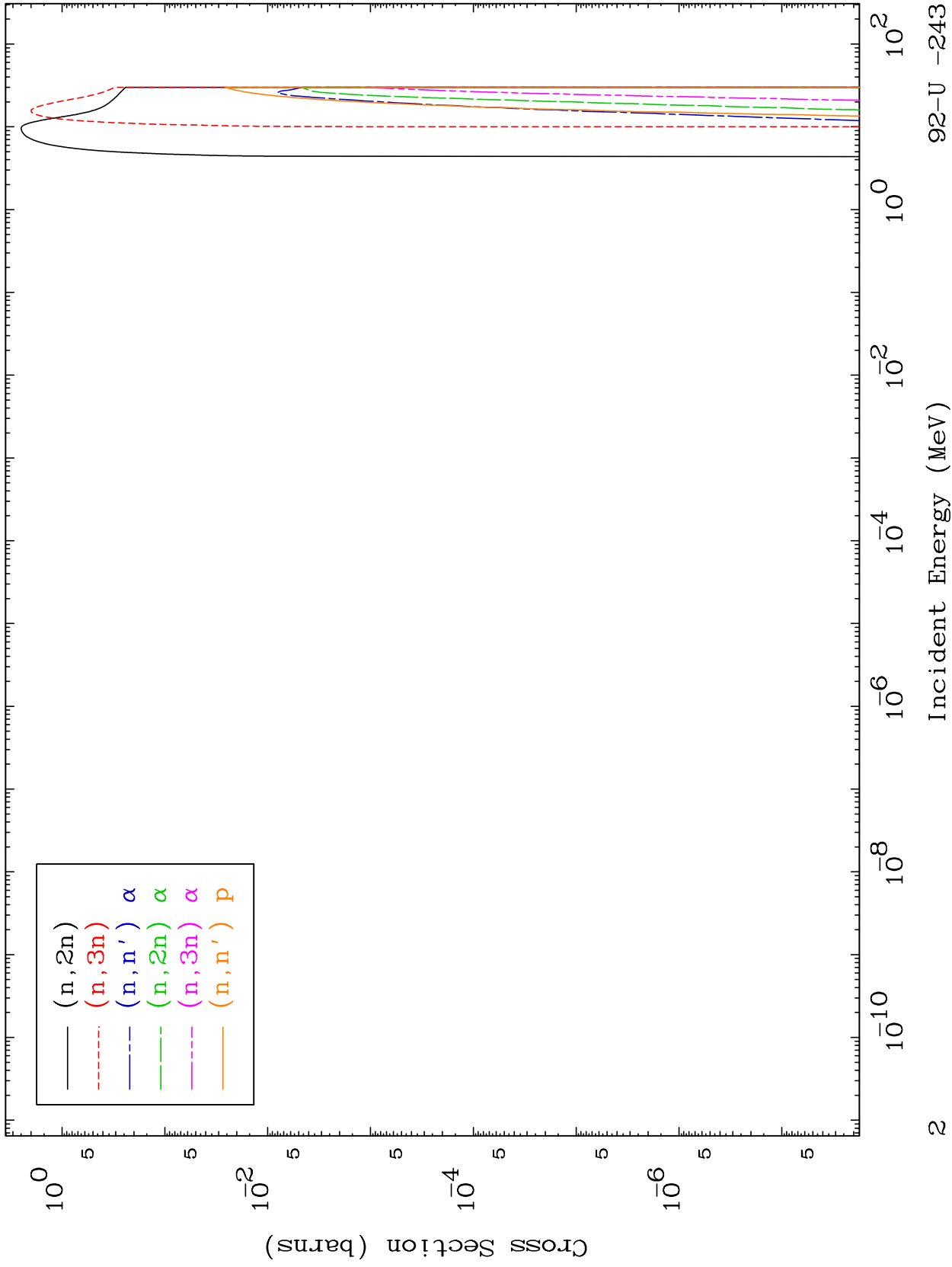
92-U -243



MAT 9252

Neutron Production  
293 Kelvin Cross Sections

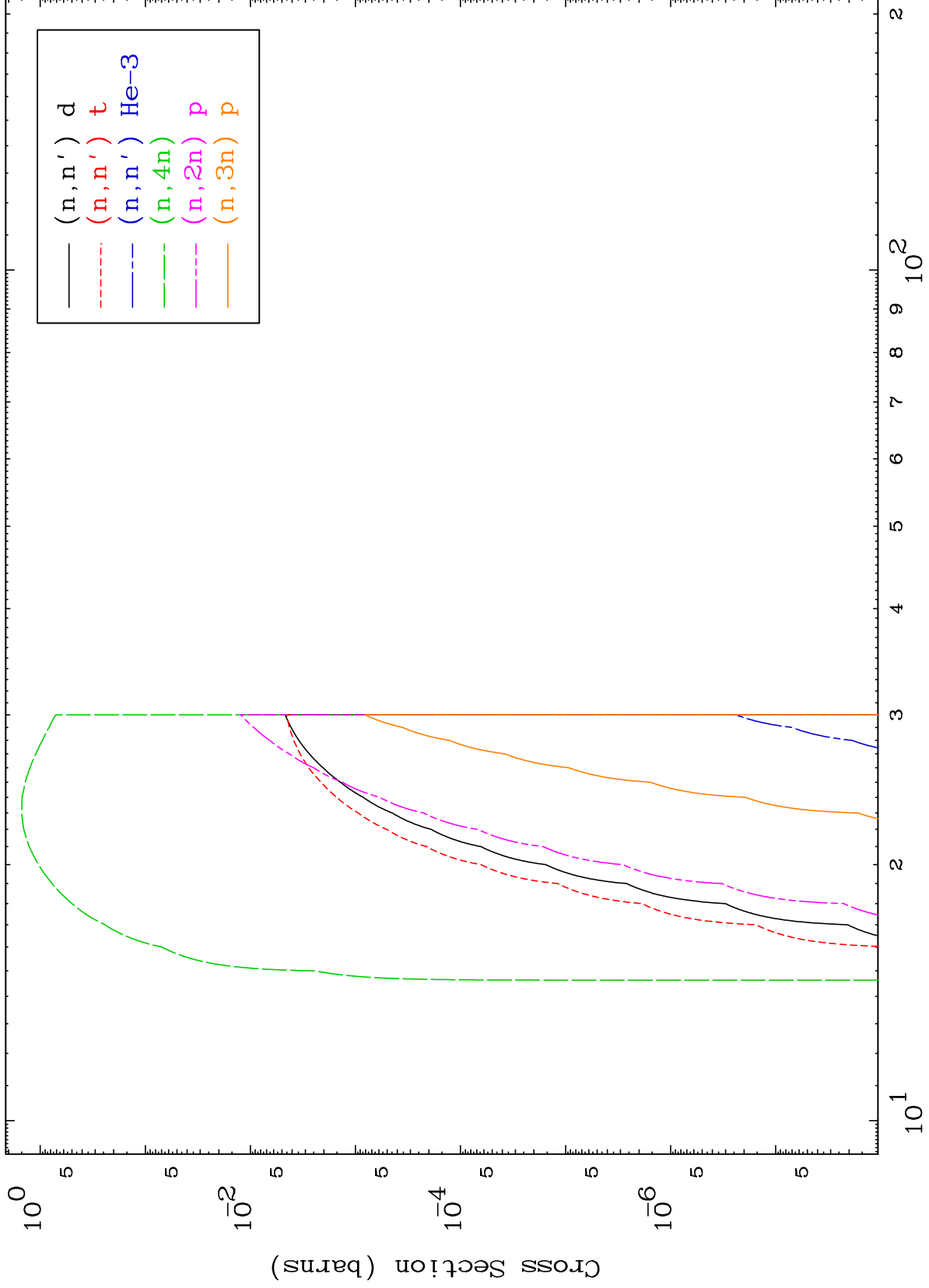
92-U -243



MAT 9252

Neutron Production  
293 Kelvin Cross Sections

92-U -243



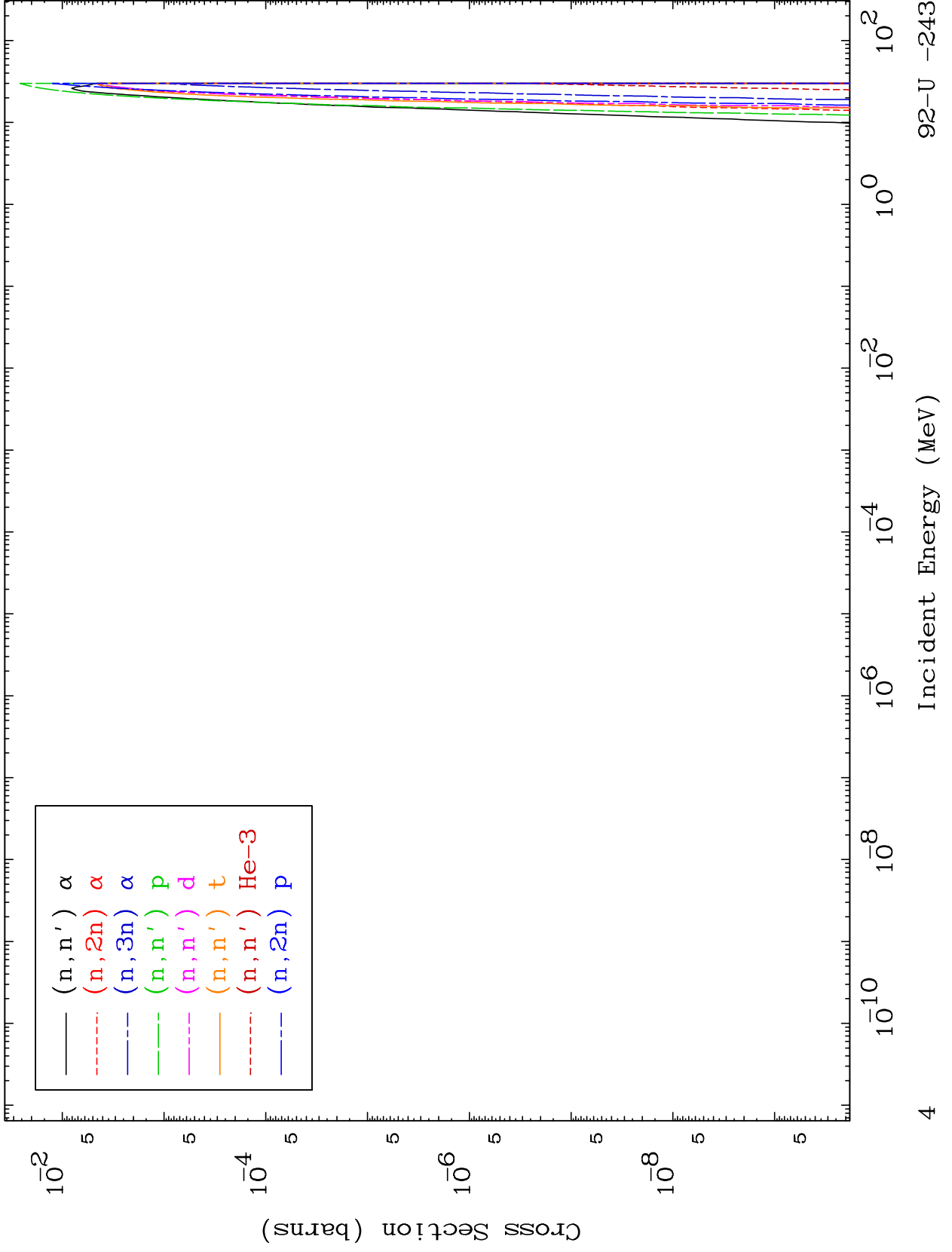
Incident Energy (MeV)

92-U -243

MAT 9252

Charged Particle  
293 Kelvin Cross Sections

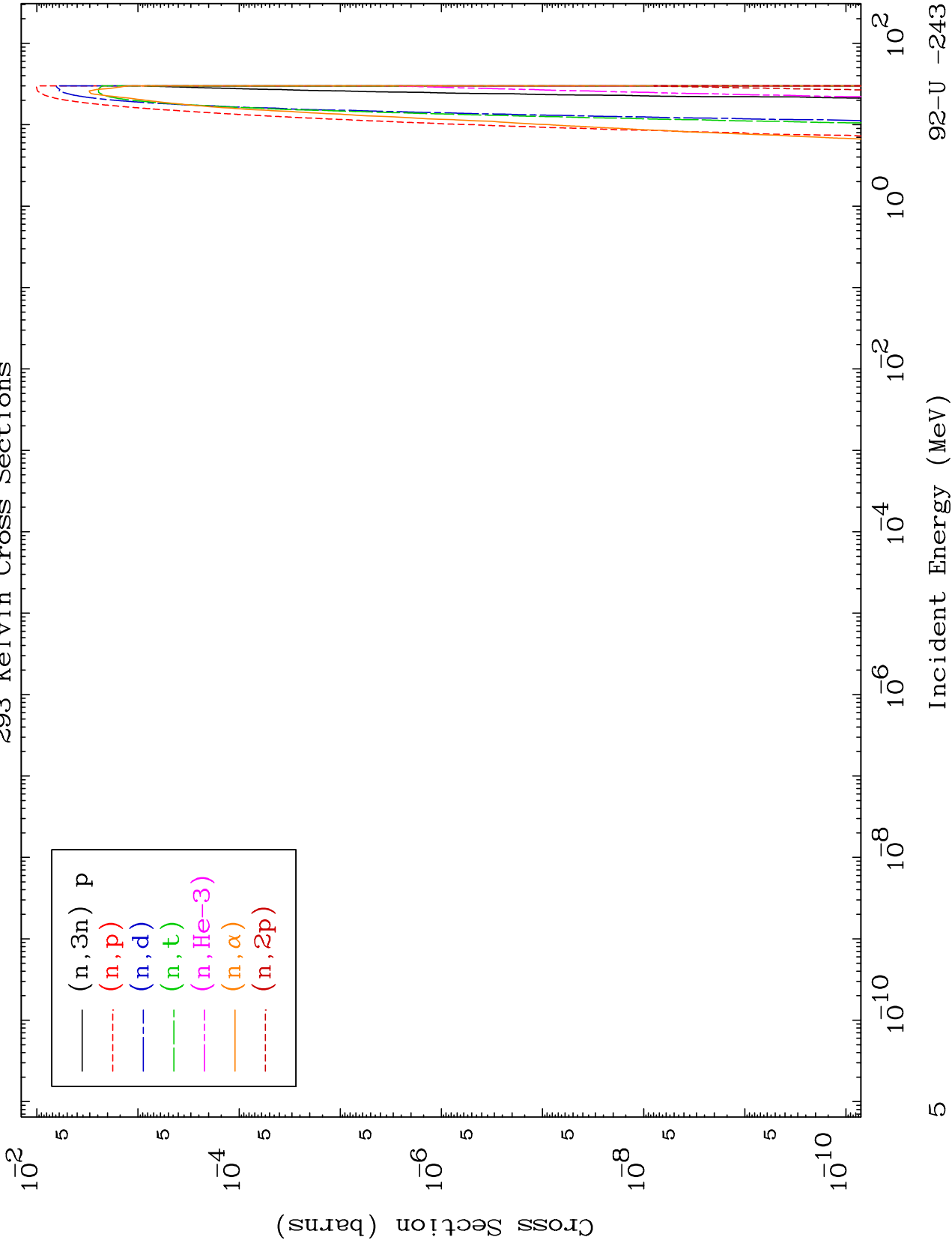
92-U -243



MAT 9252

Charged Particle  
293 Kelvin Cross Sections

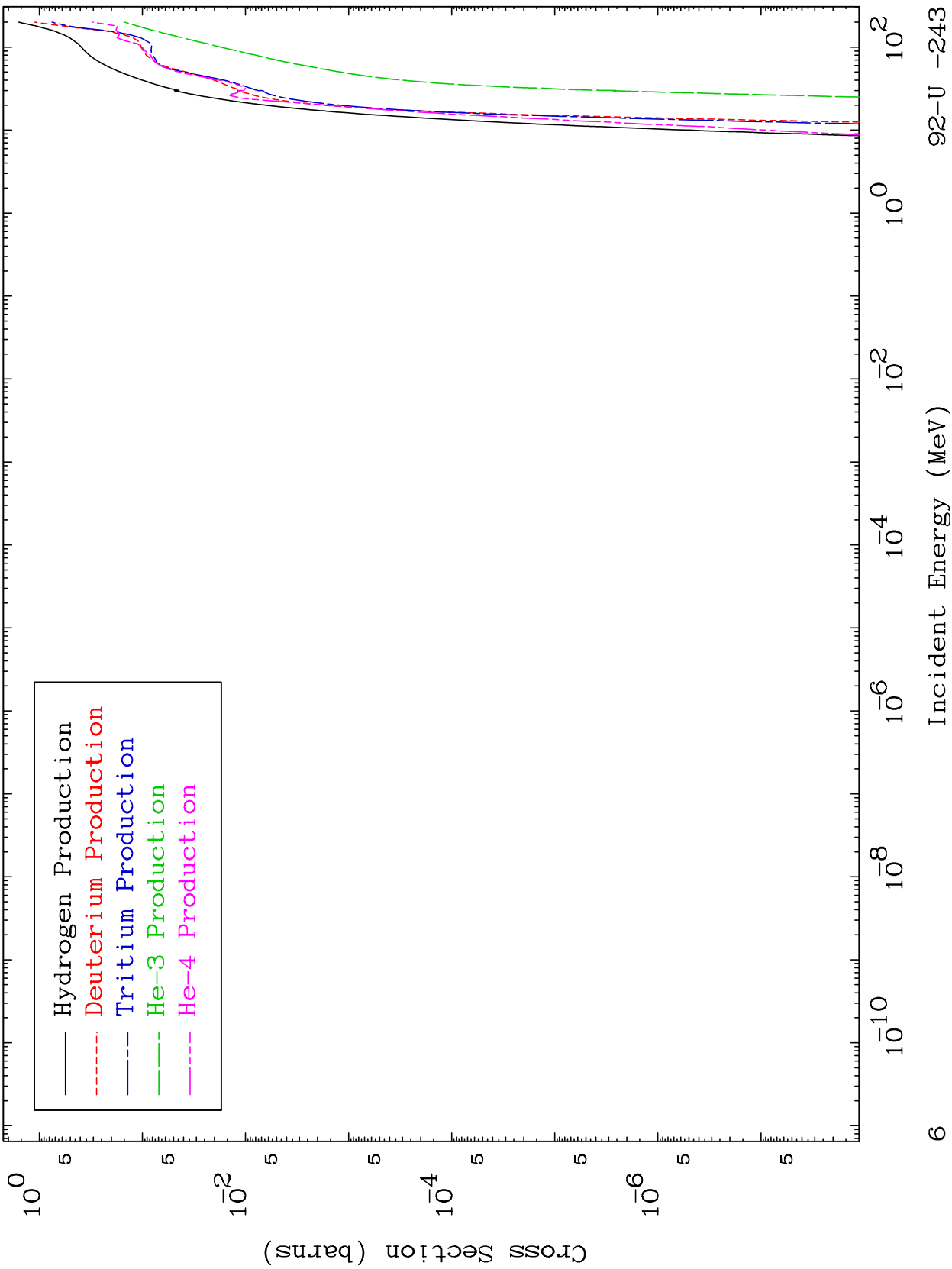
92-U -243



MAT 9252

Particle Production  
293 Kelvin Cross Sections

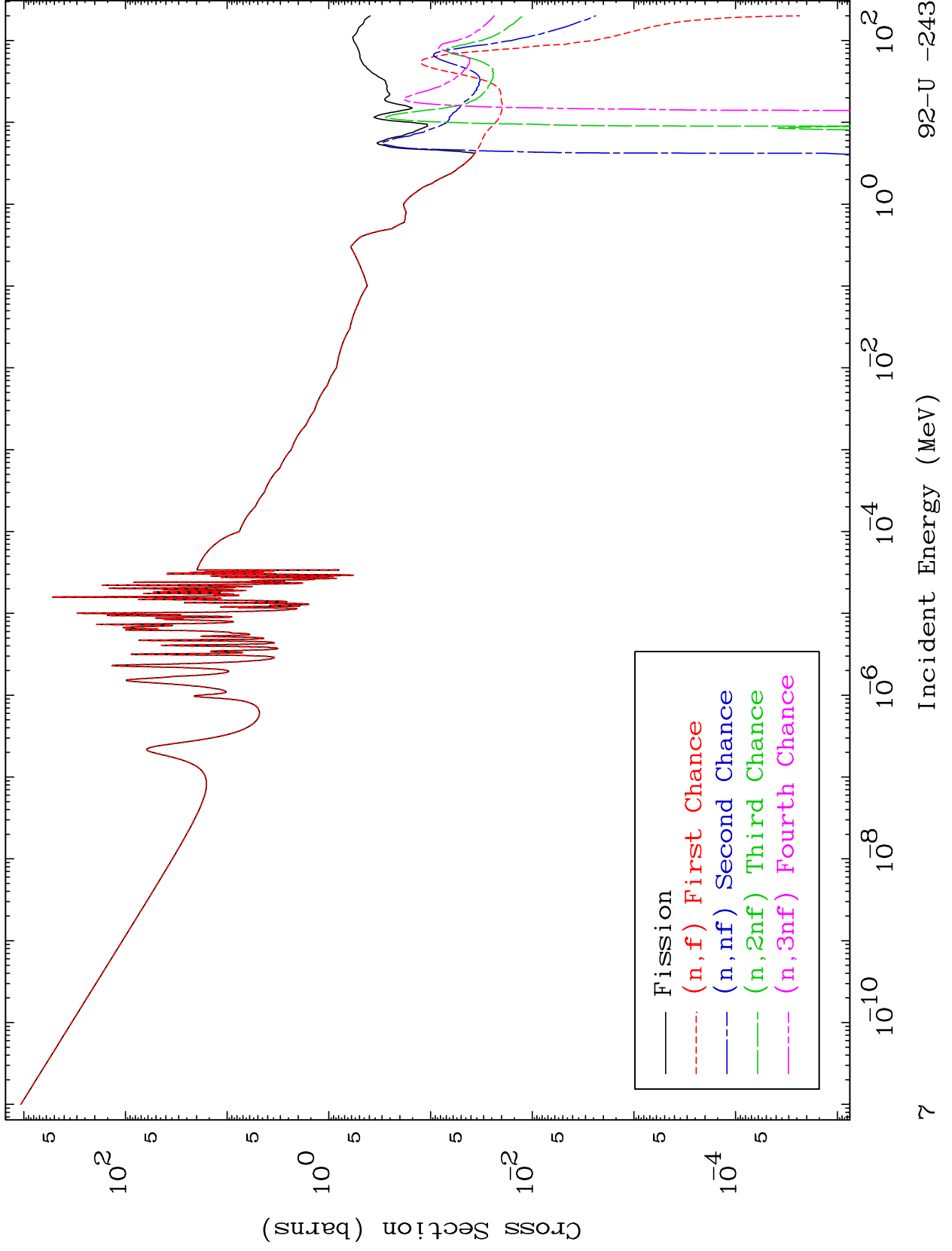
92-U -243



MAT 9252

Fission  
293 Kelvin Cross Sections

92-U -243



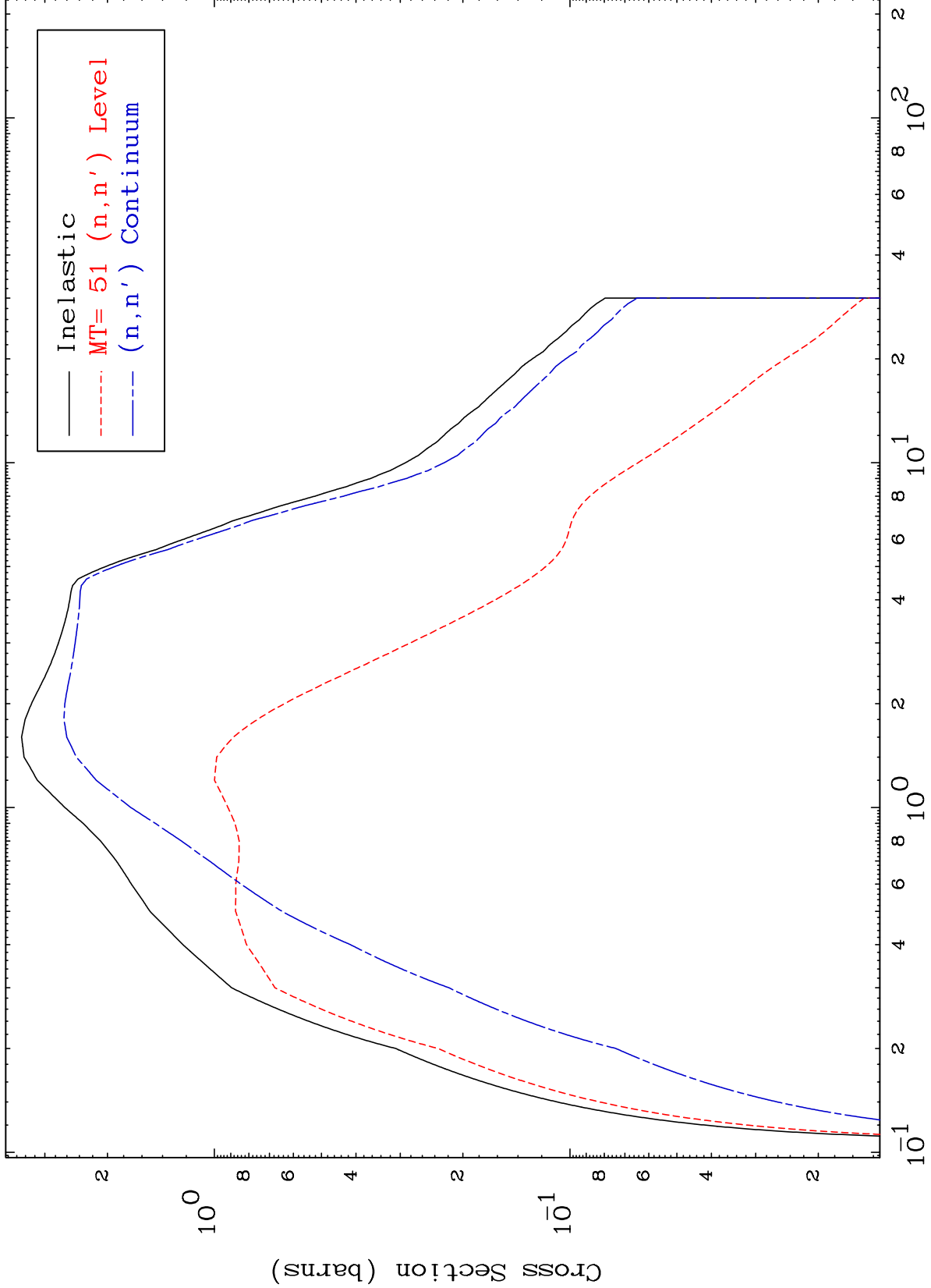


MAT 9252

(n,n') Level

92-U -243

293 Kelvin Cross Sections



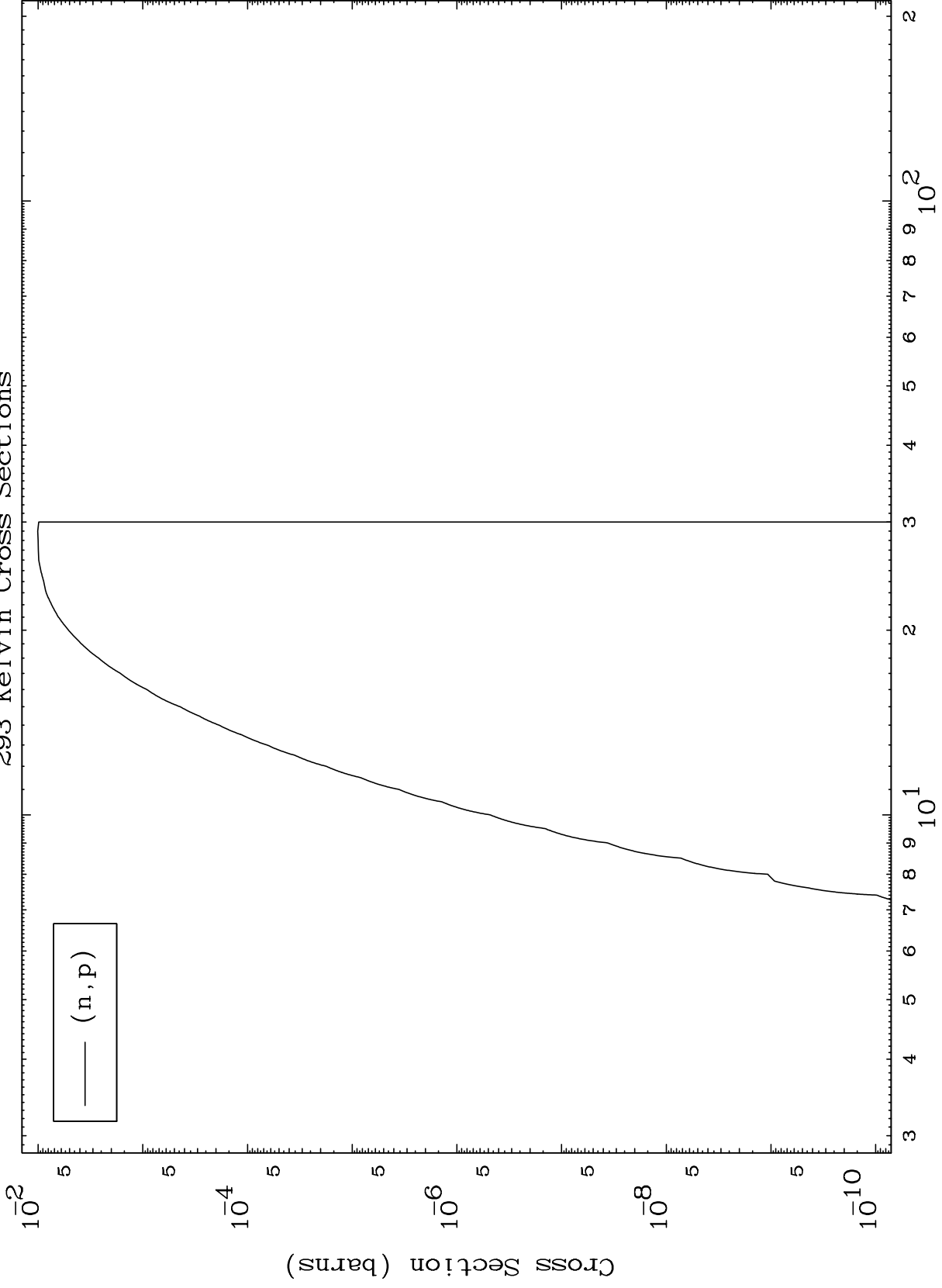
Incident Energy (MeV)

92-U -243

MAT 9252

(n,p) Levels  
293 Kelvin Cross Sections

92-U -243



9

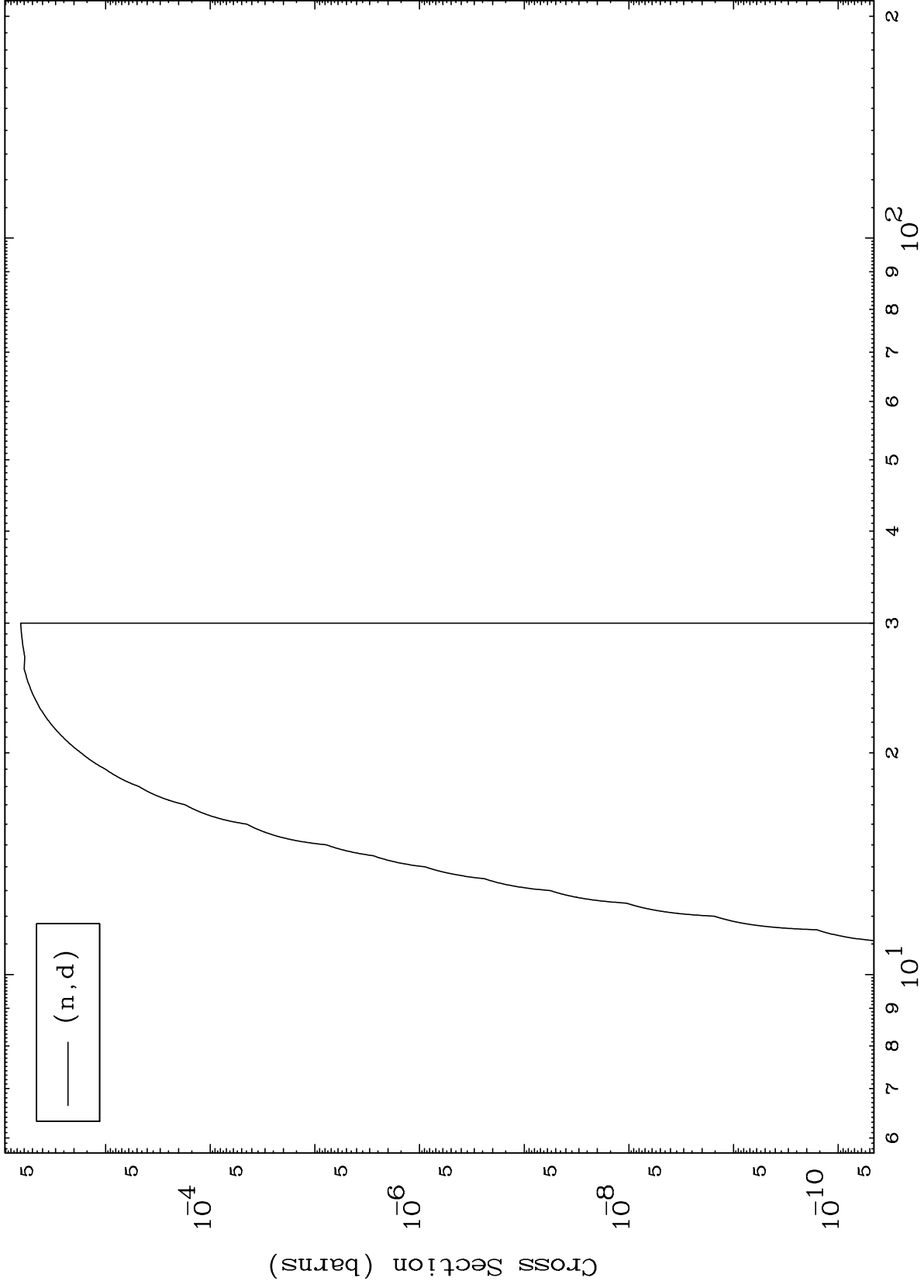
Incident Energy (MeV)

92-U -243

MAT 9252

(n,d) Levels  
293 Kelvin Cross Sections

92-U -243



10

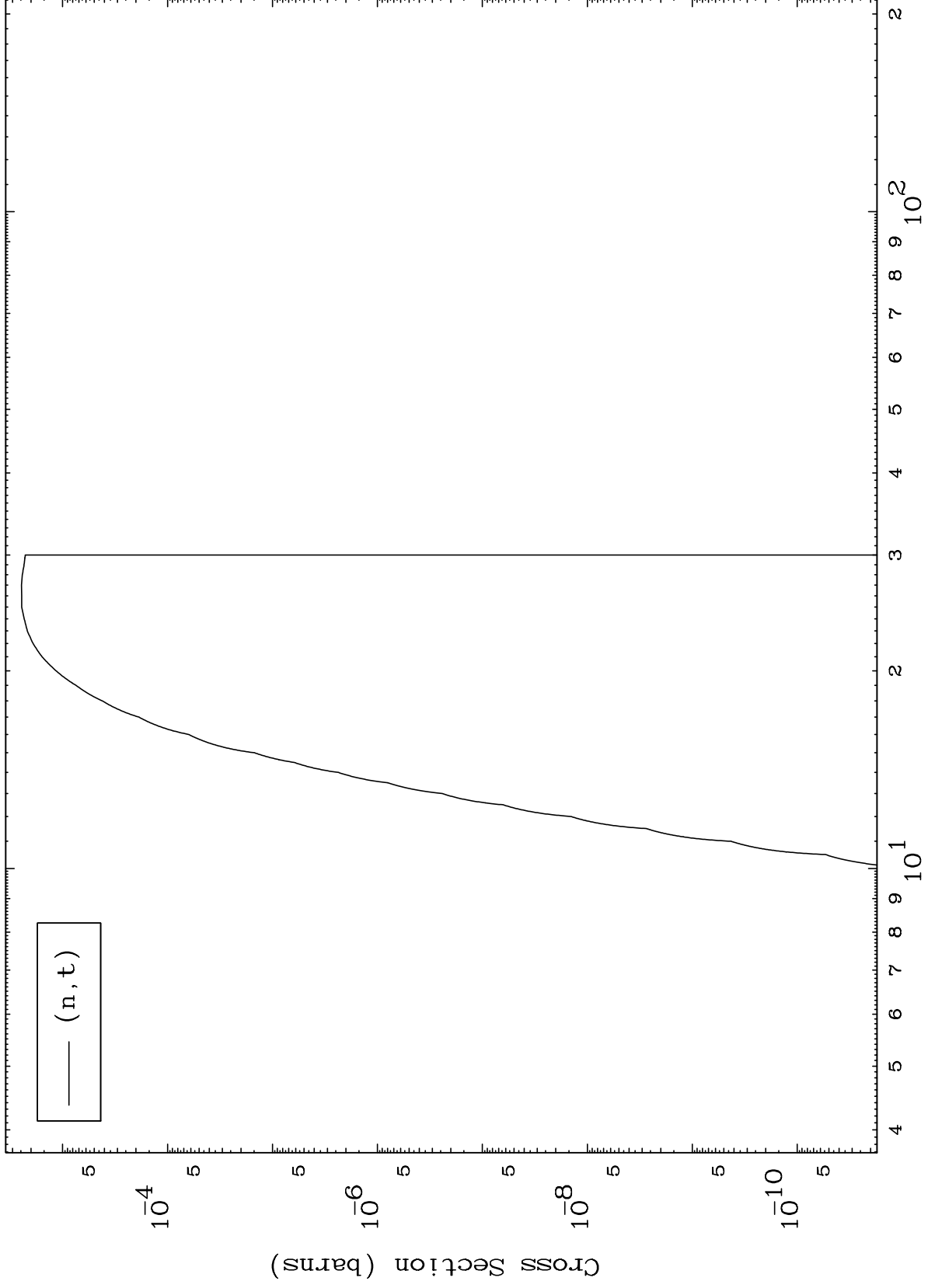
Incident Energy (MeV)

92-U -243

MAT 9252

(n,t) Levels  
293 Kelvin Cross Sections

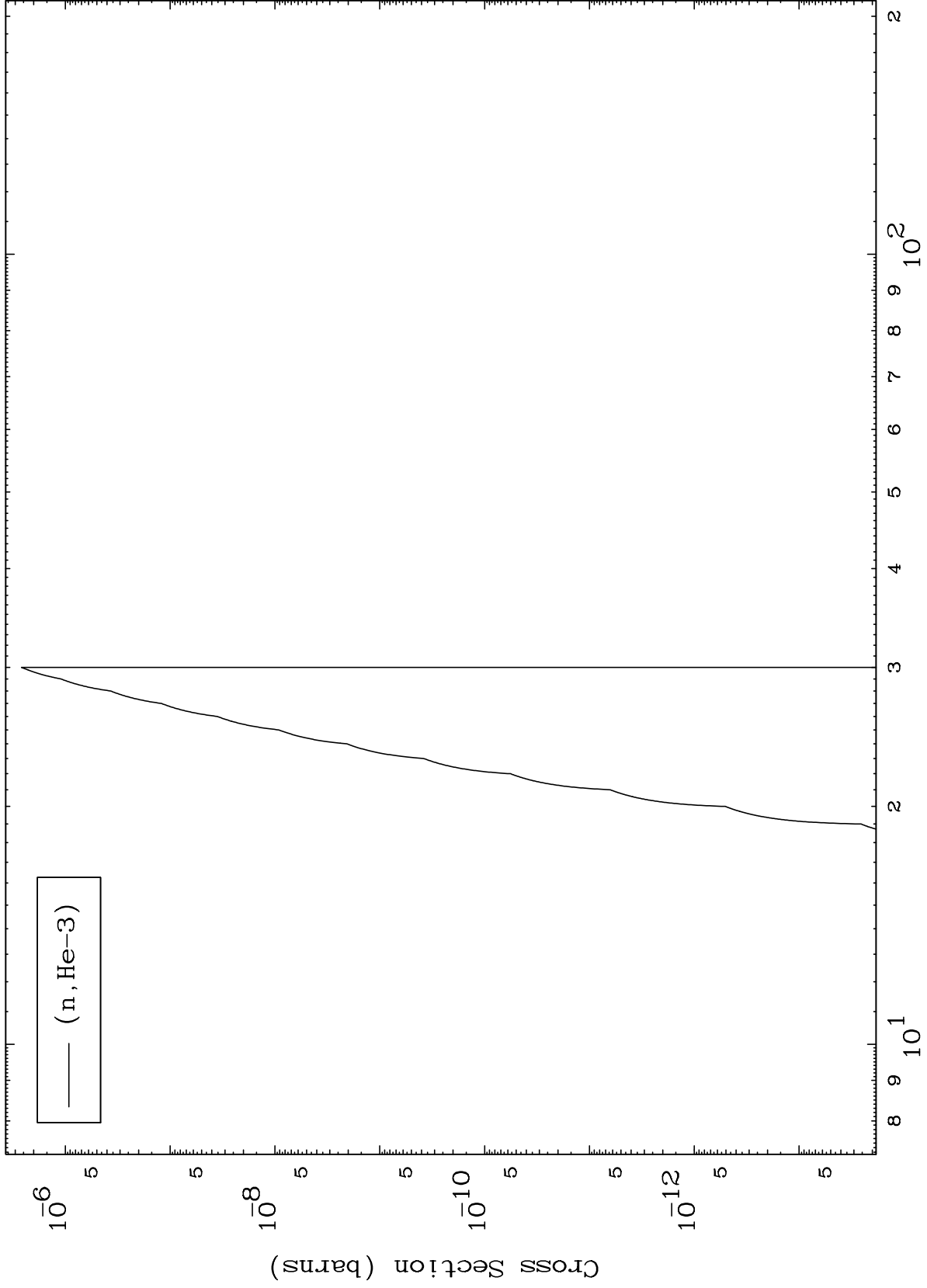
92-U -243



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(n,He3) Levels  
293 Kelvin Cross Sections

92-U -243



12

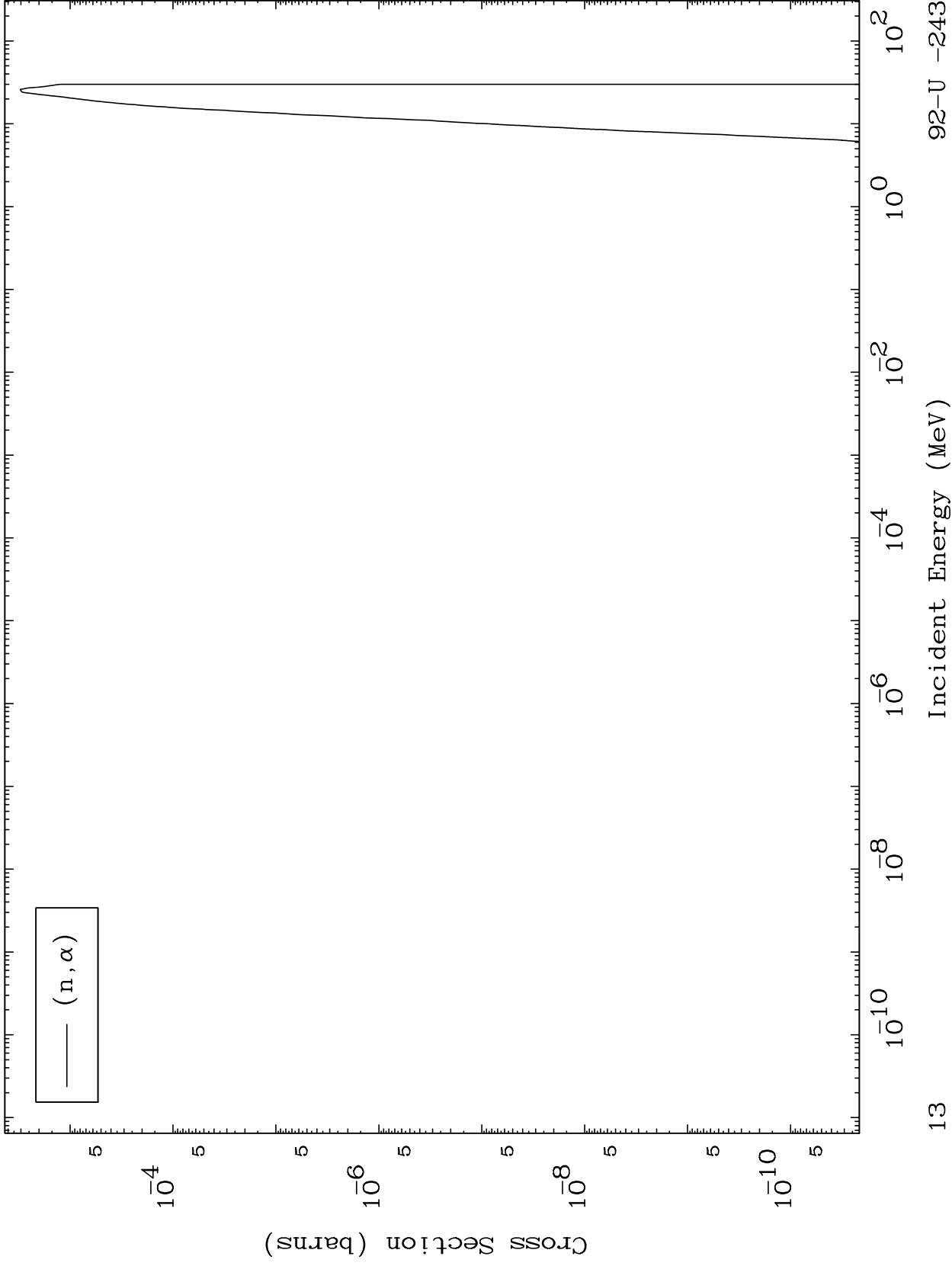
Incident Energy (MeV)

92-U -243

MAT 9252

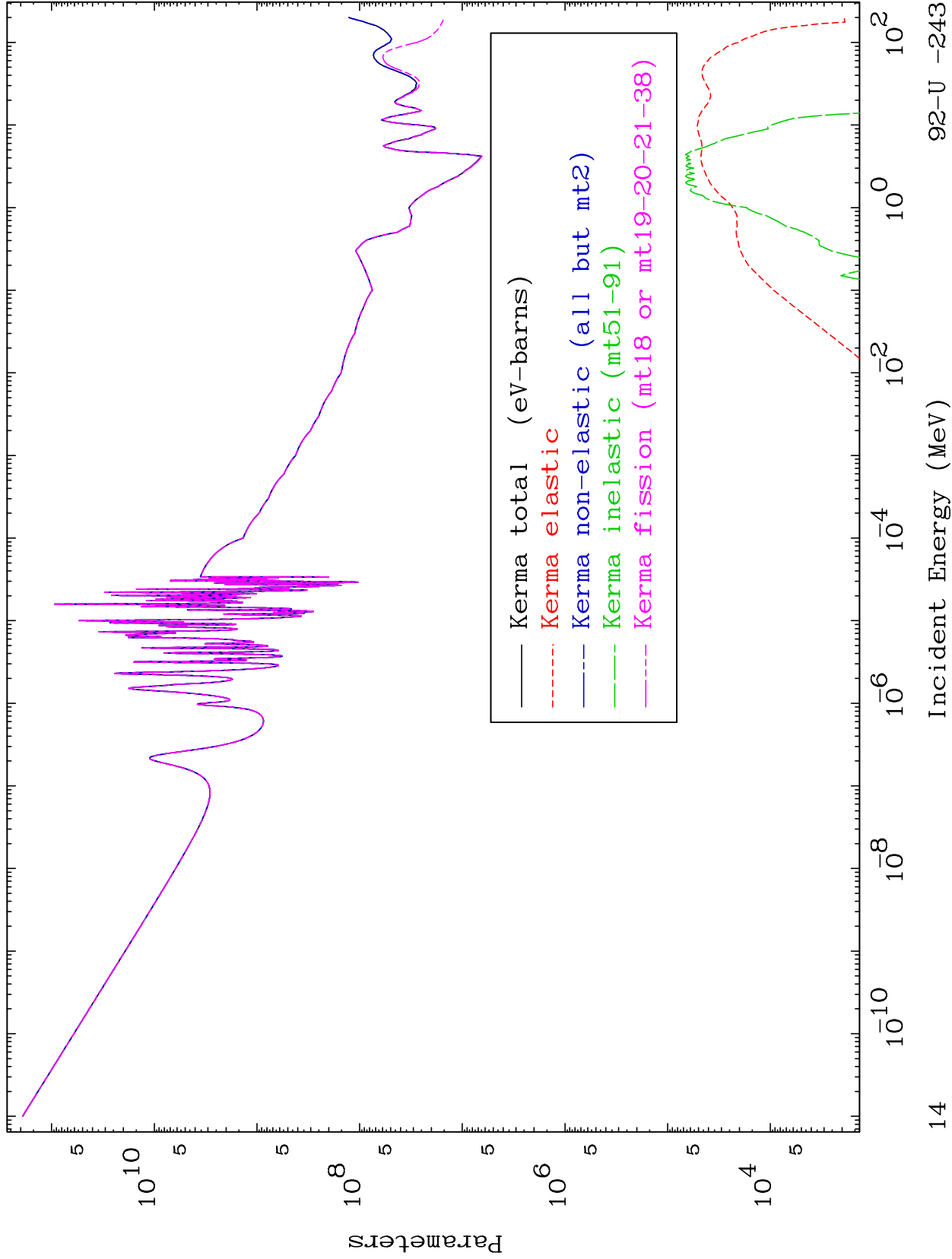
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

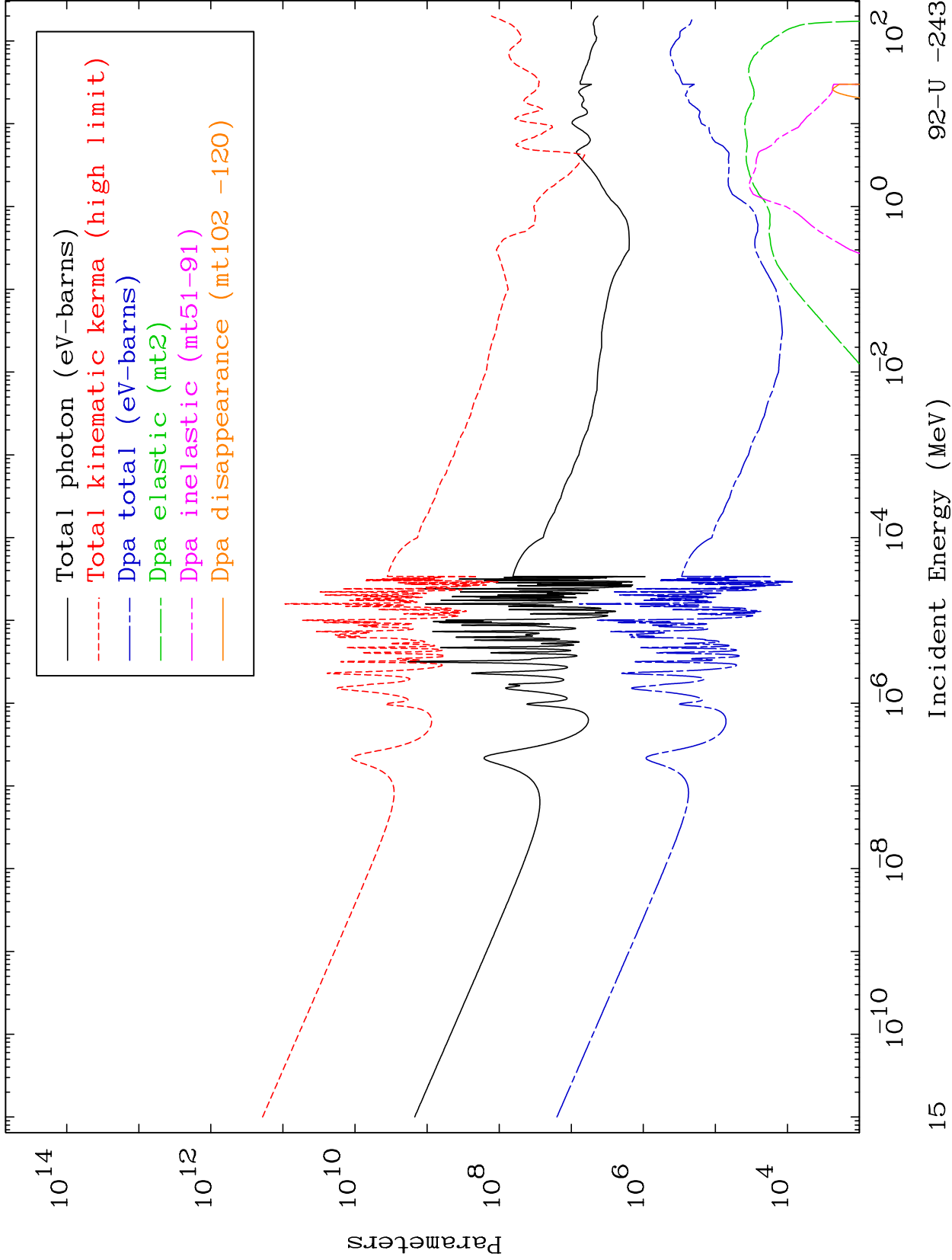
92-U -243



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92-U -243



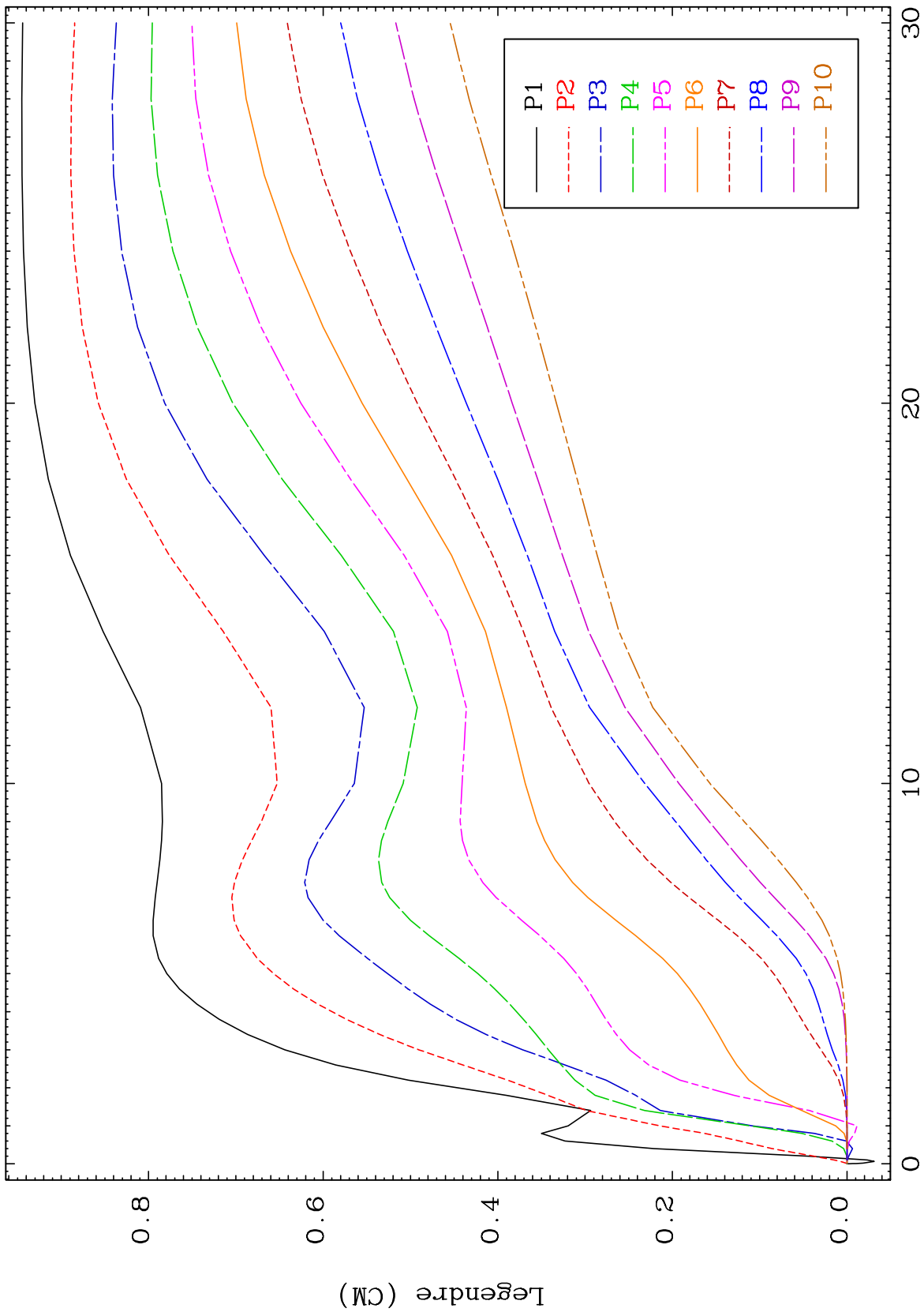




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92-U -243

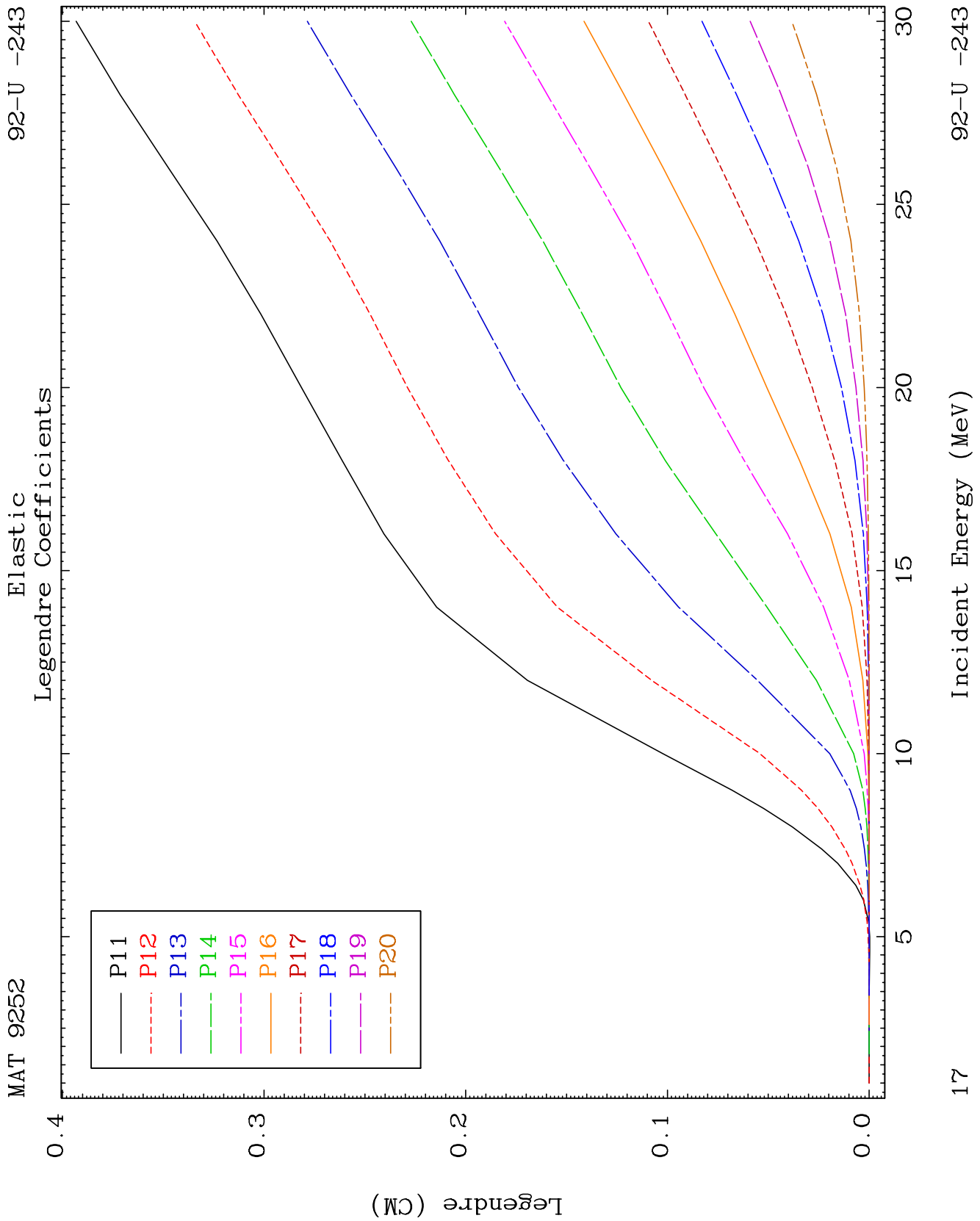
Elastic Legendre Coefficients



92-U -243

Incident Energy (MeV)

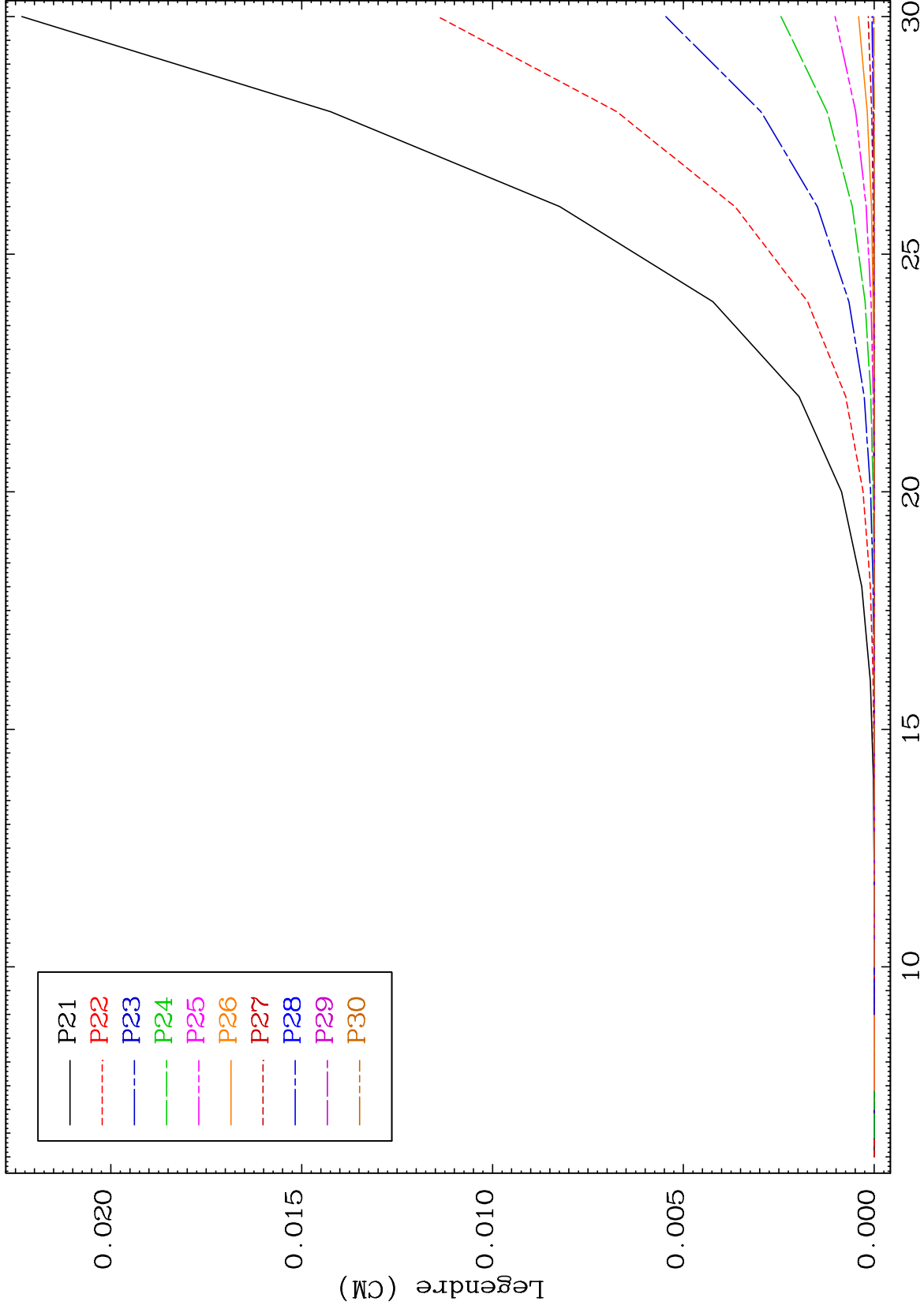
16



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### Elastic Legendre Coefficients

92-U -243



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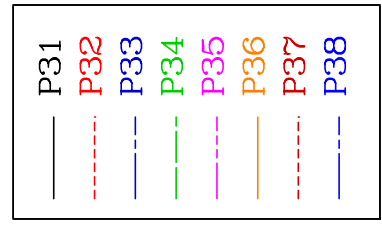
Incident Energy (MeV)

92-U -243

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Elastic Legendre Coefficients

92-U -243



$\times 10^{-6}$

2.0

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

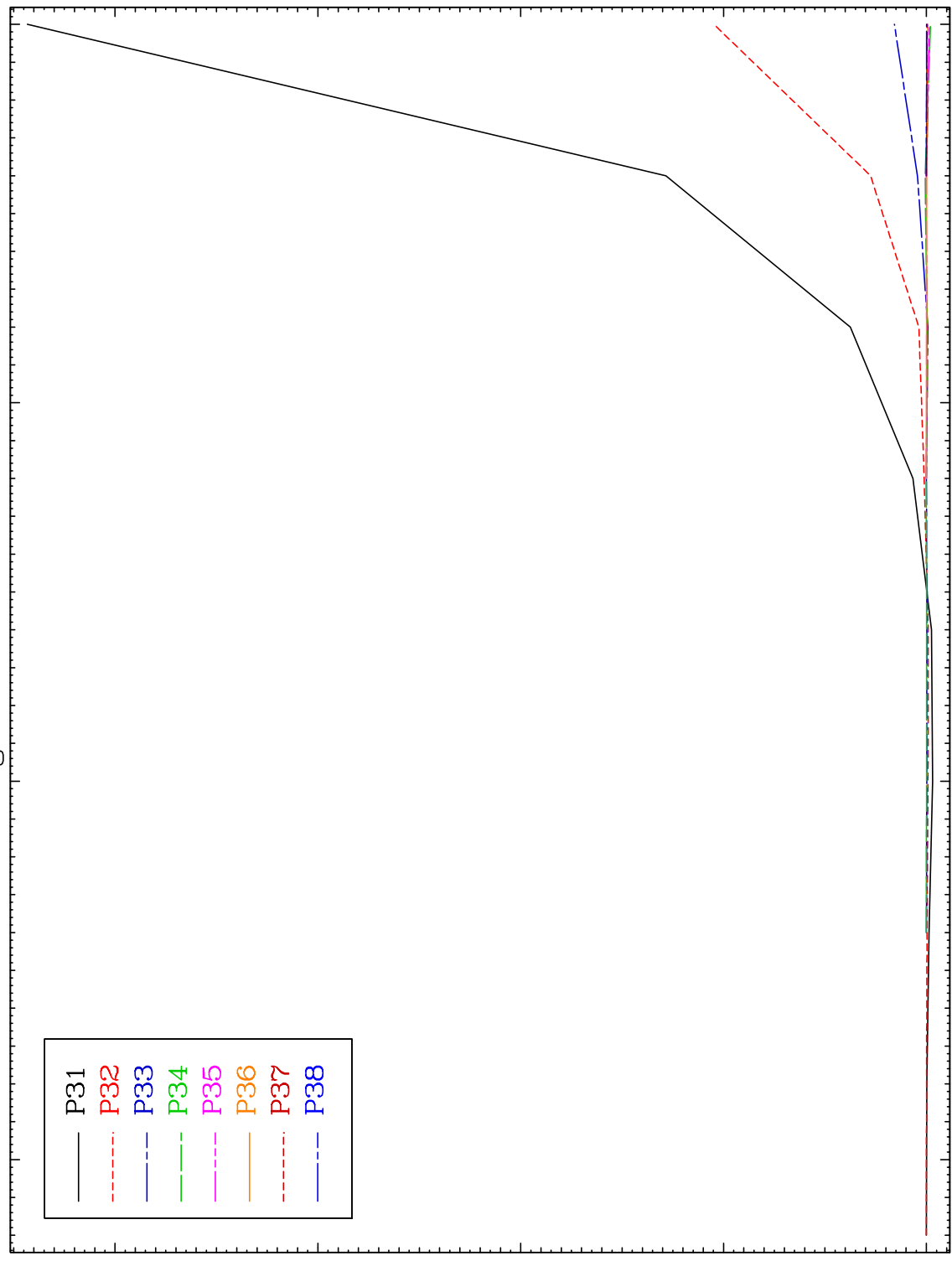
25

30

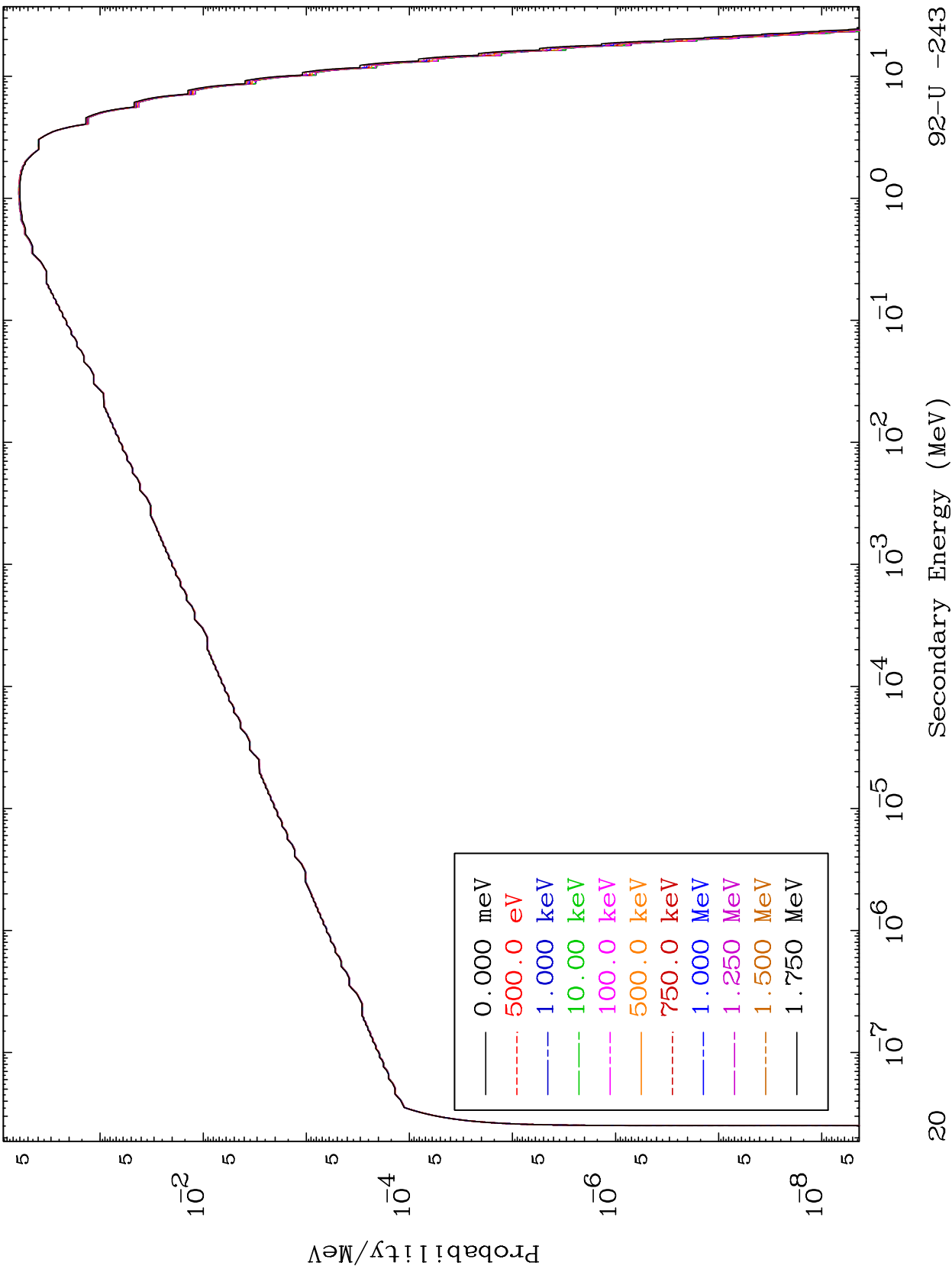
19

Incident Energy (MeV)

92-U -243



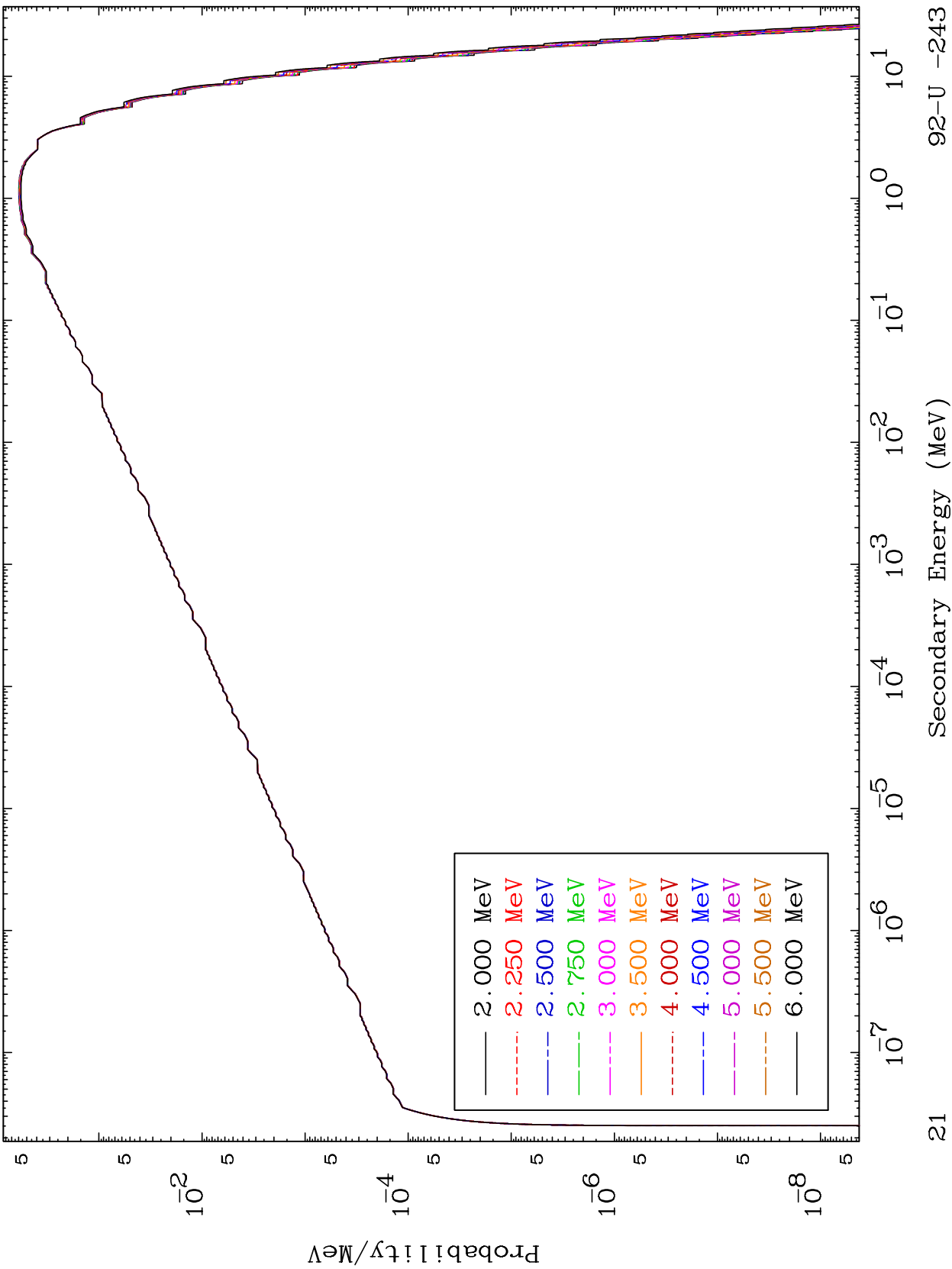
Fission  
Energy Distribution



MAT 9252

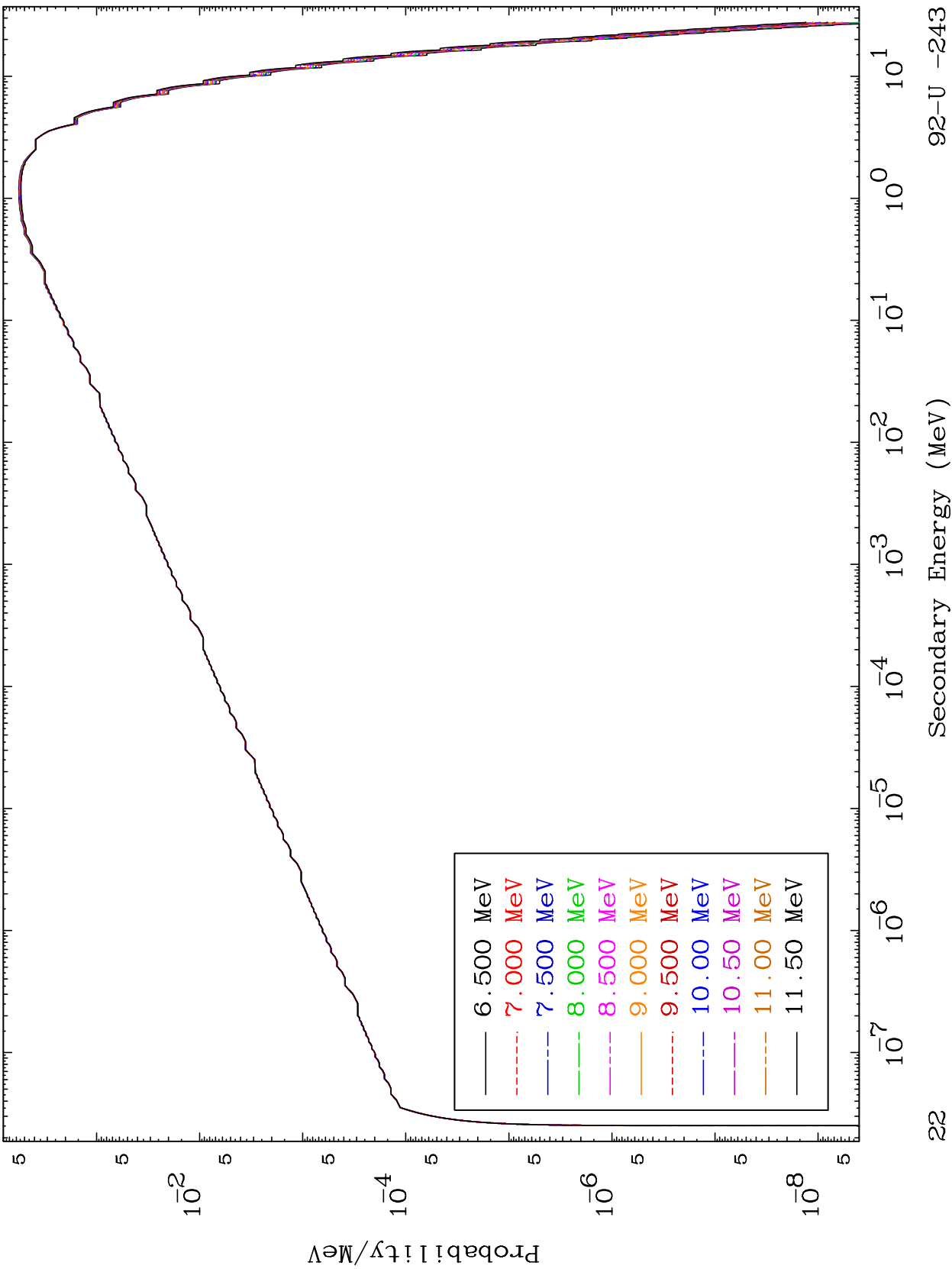
### Fission Energy Distribution

92-U -243

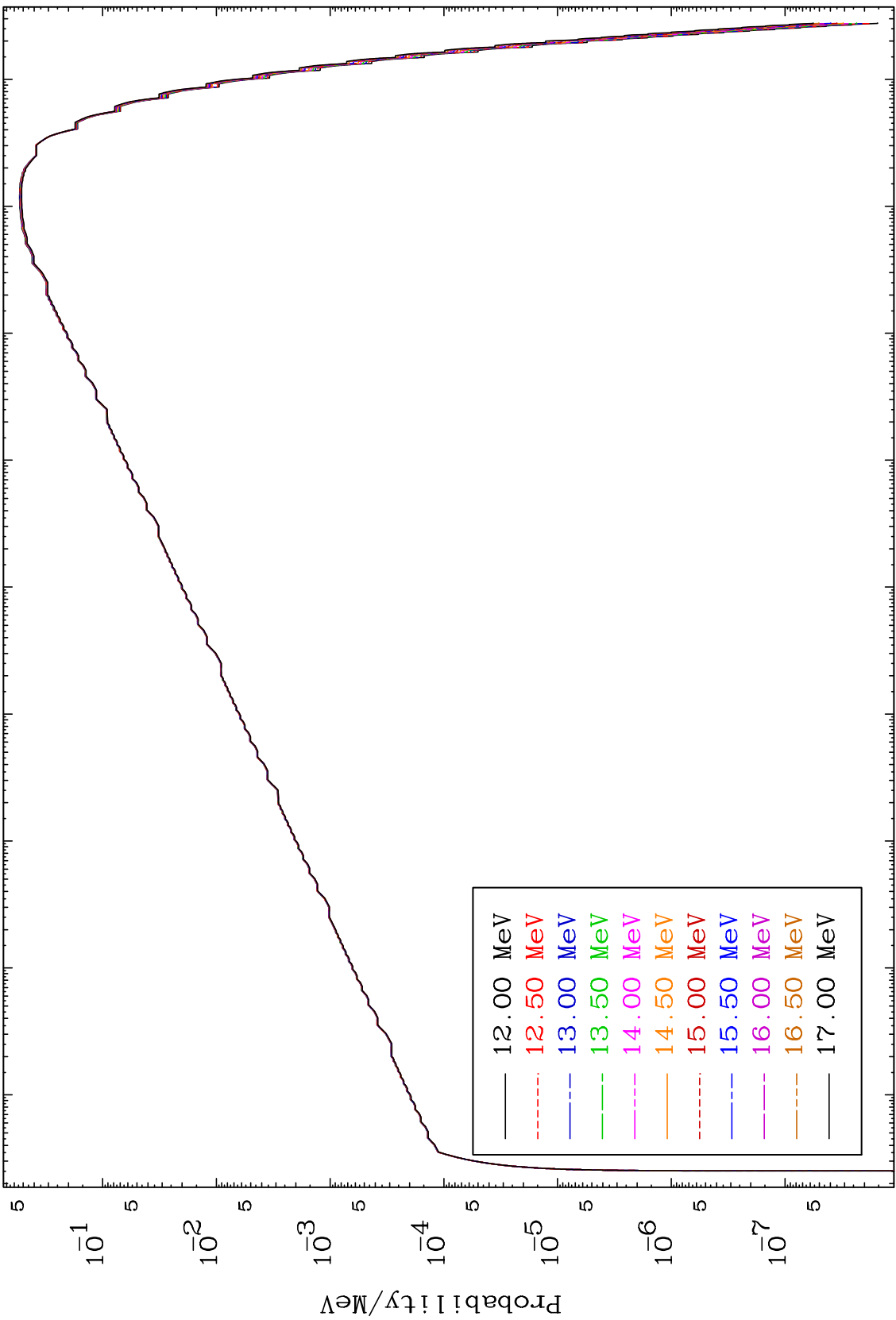


92-U -243

Fission Energy Distribution



Fission  
Energy Distribution





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### Fission Energy Distribution

92-U -243

