

Program EVALPLOT
(Version 2017-1)

by

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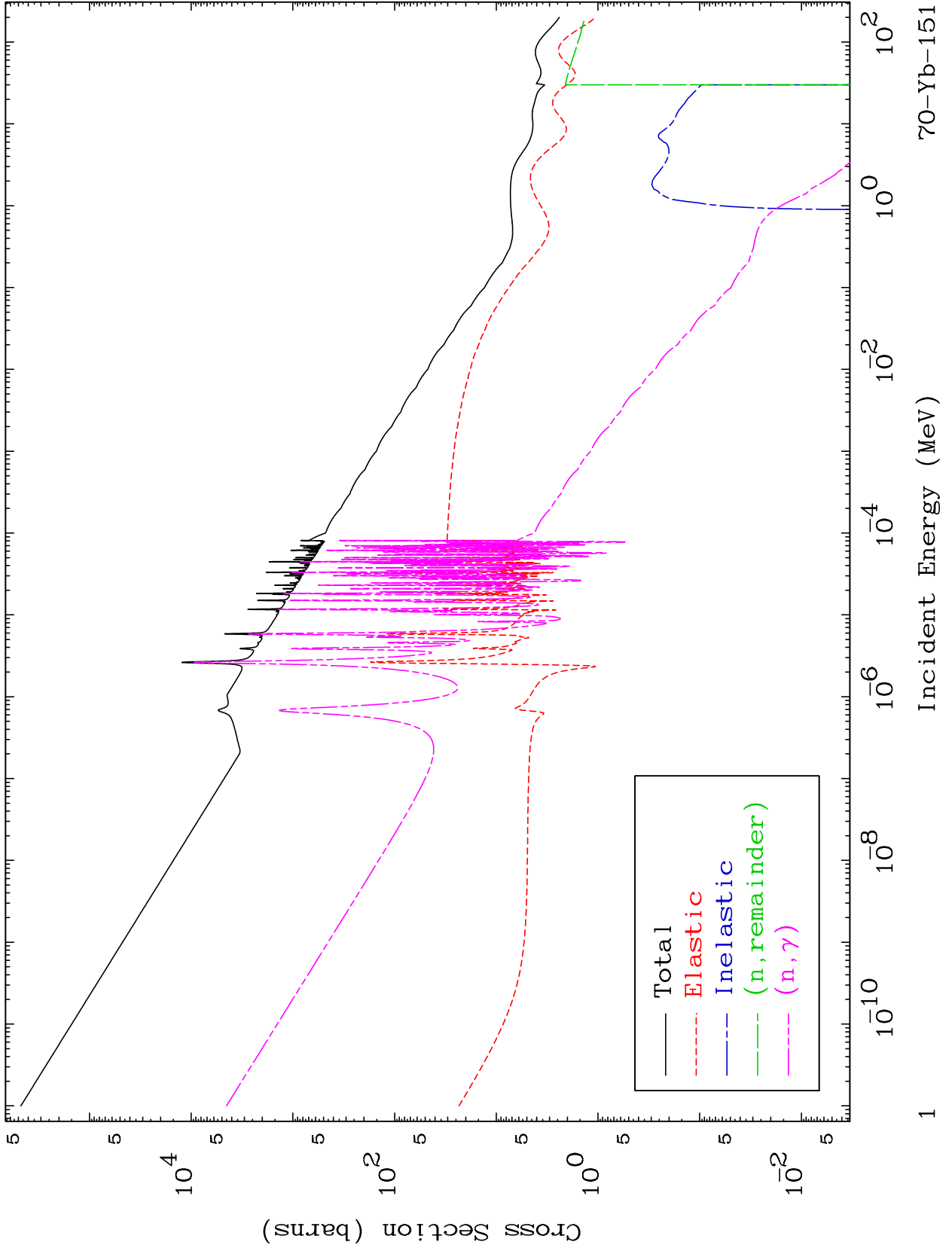
Press Mouse Button to Start

MAT 6974

Major

293 Kelvin Cross Sections

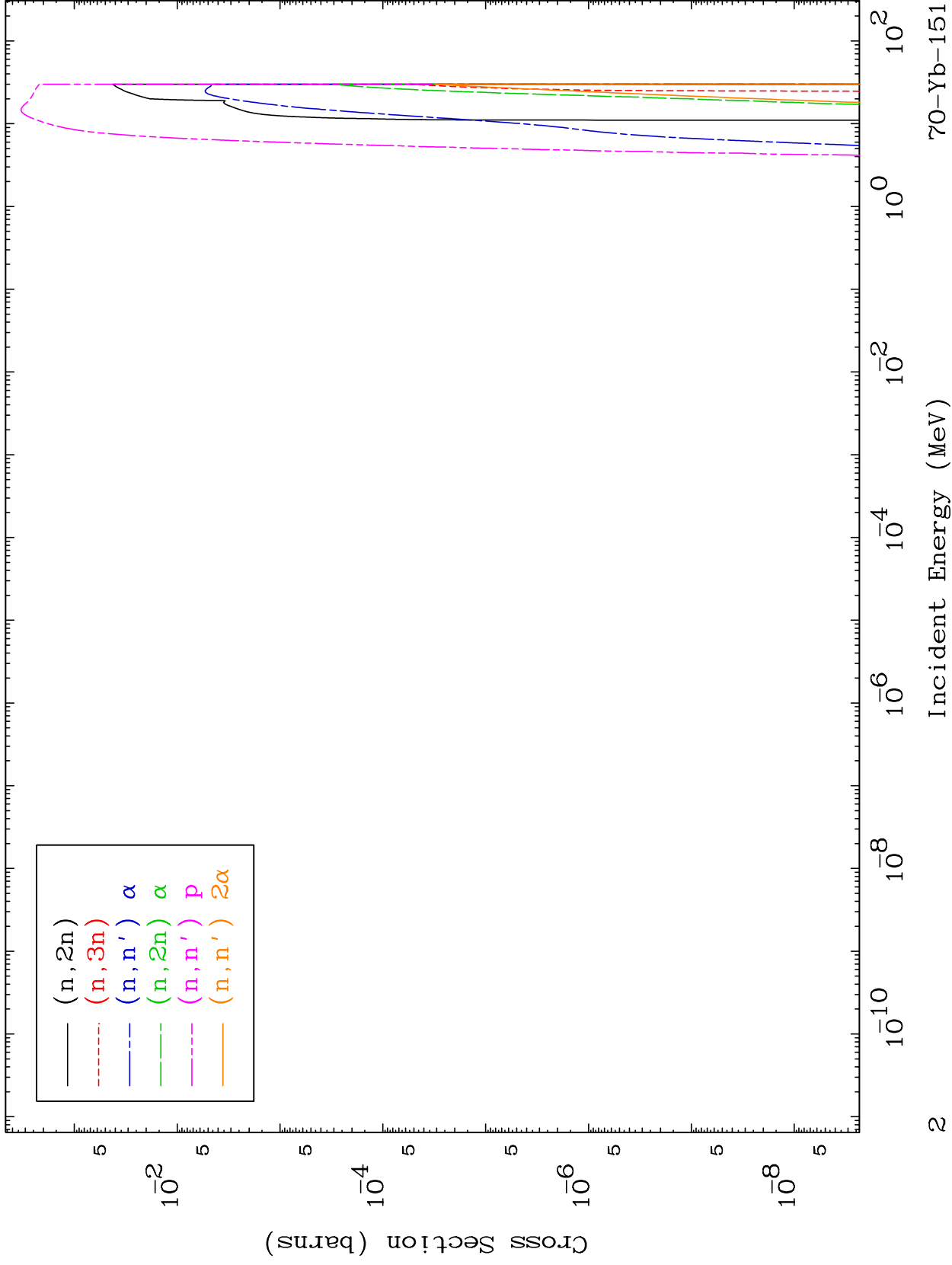
70-Yb-151

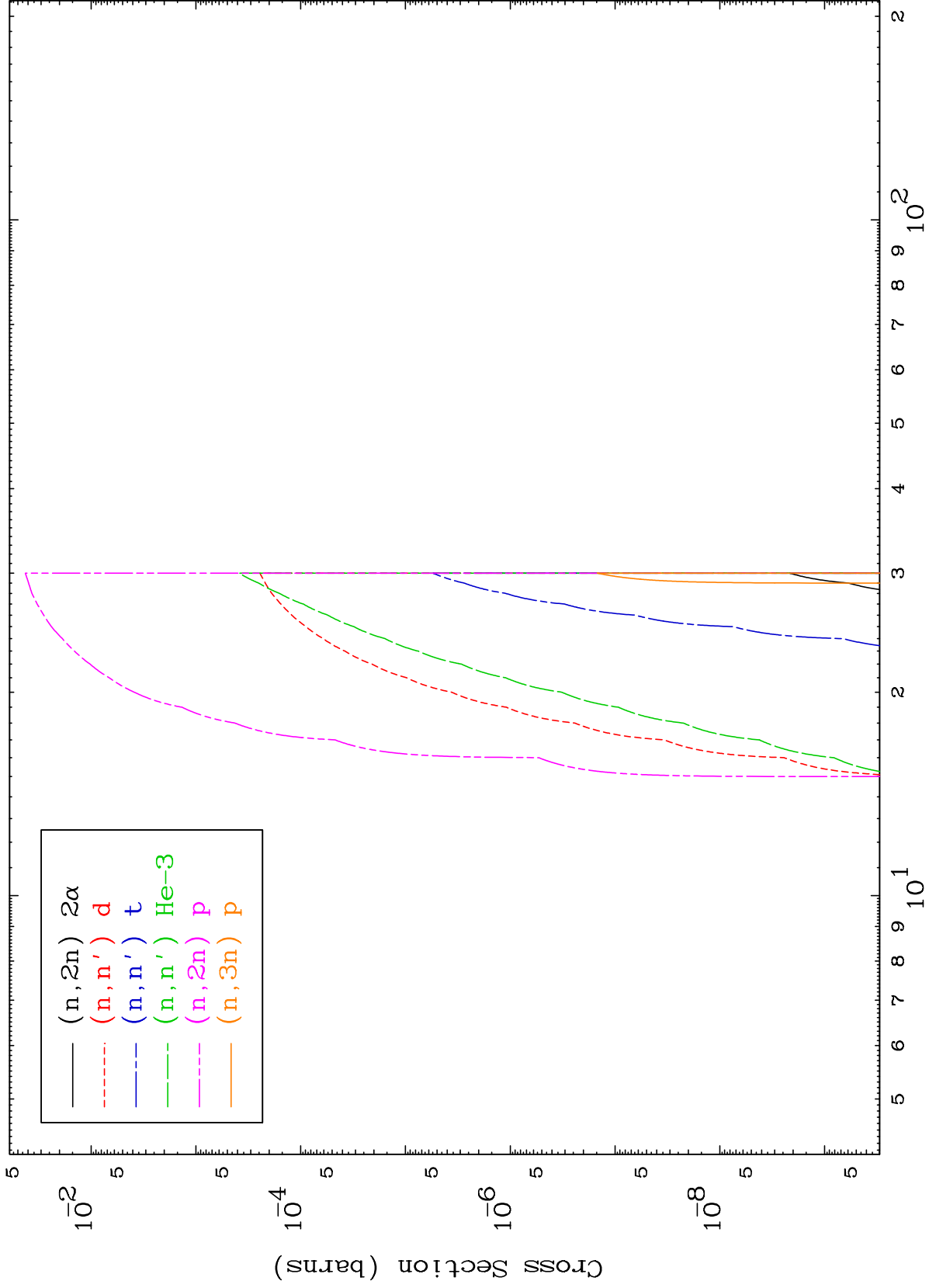


MAT 6974

Neutron Production
293 Kelvin Cross Sections

70-Yb-151

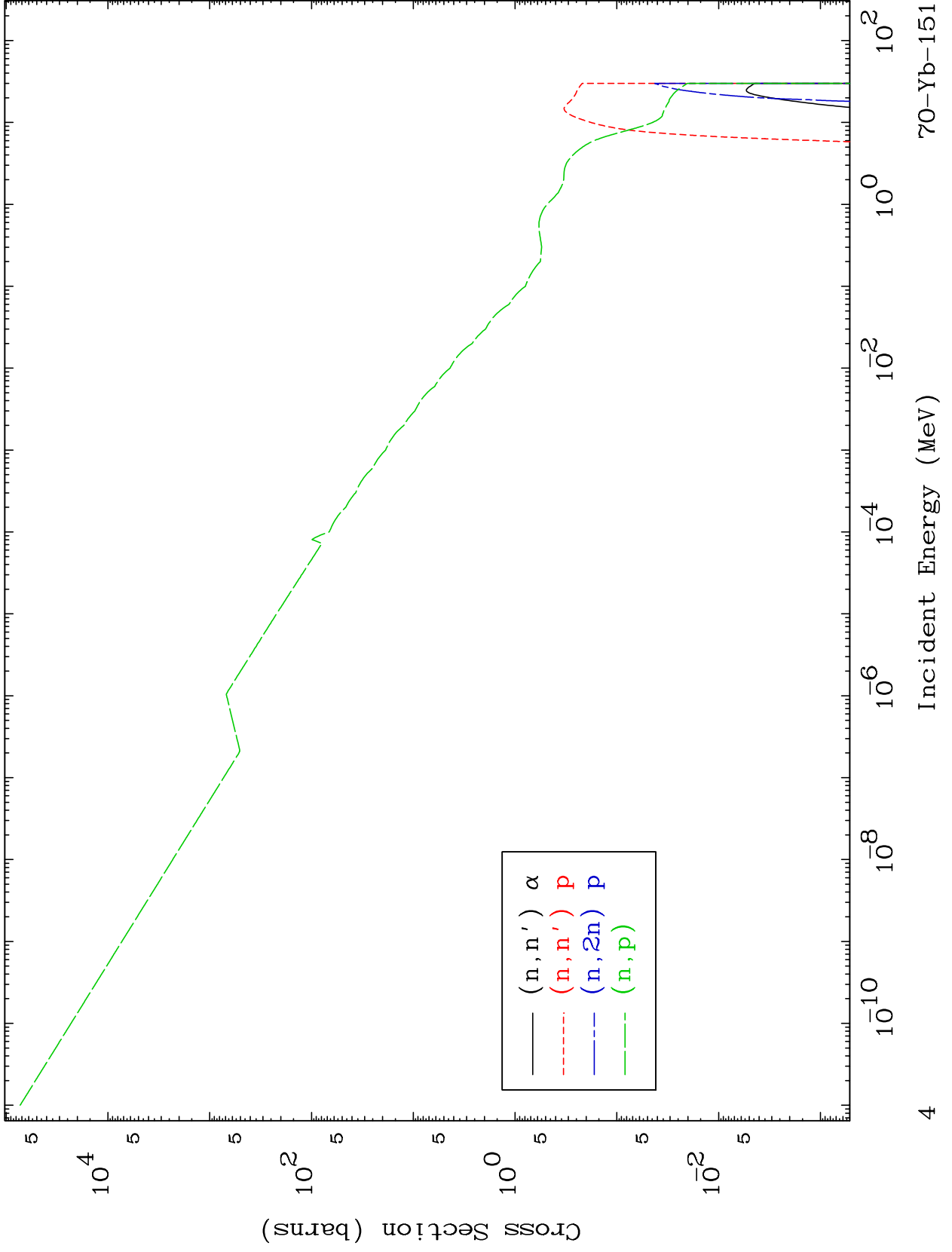




MAT 6974

Charged Particle
293 Kelvin Cross Sections

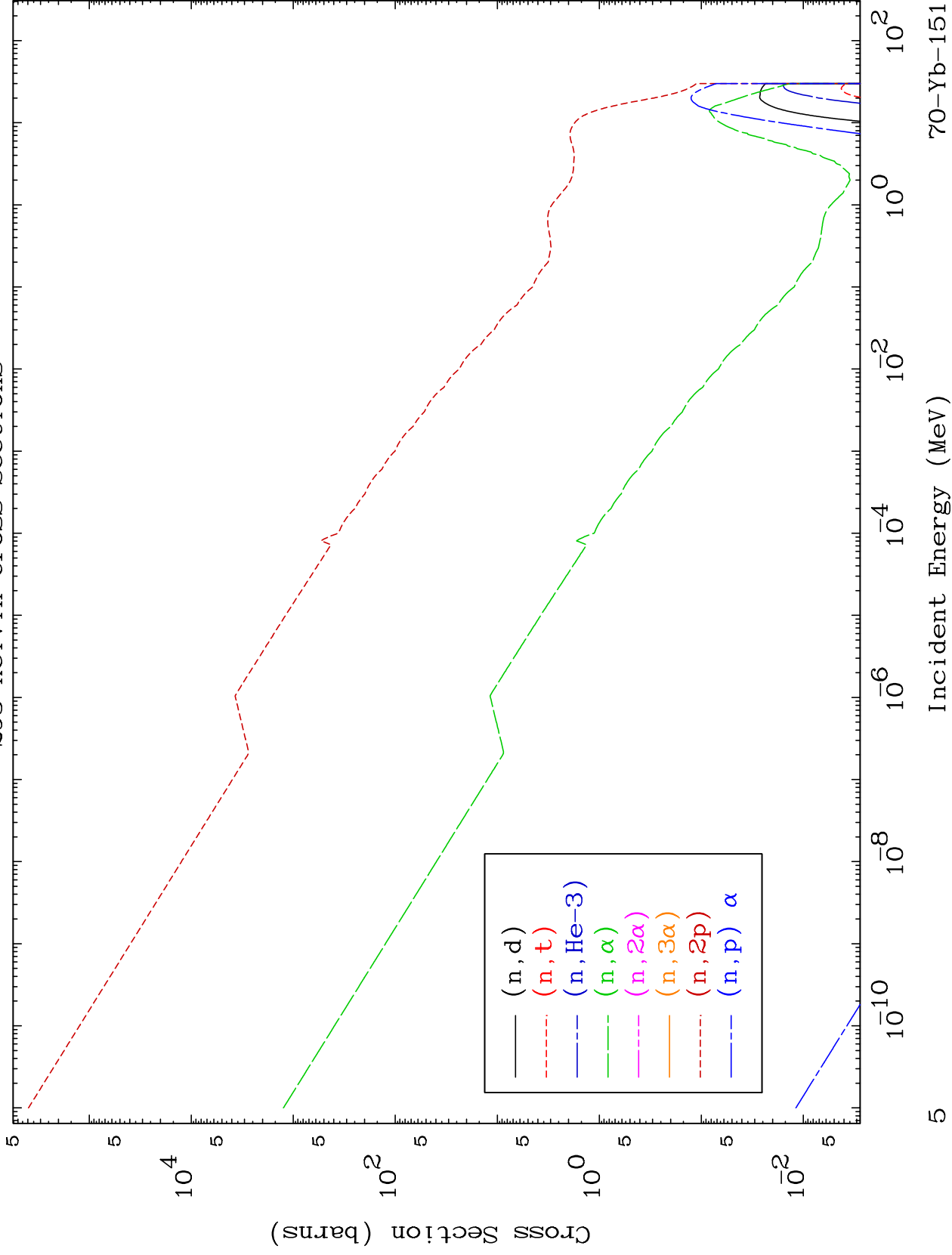
70-Yb-151



MAT 6974

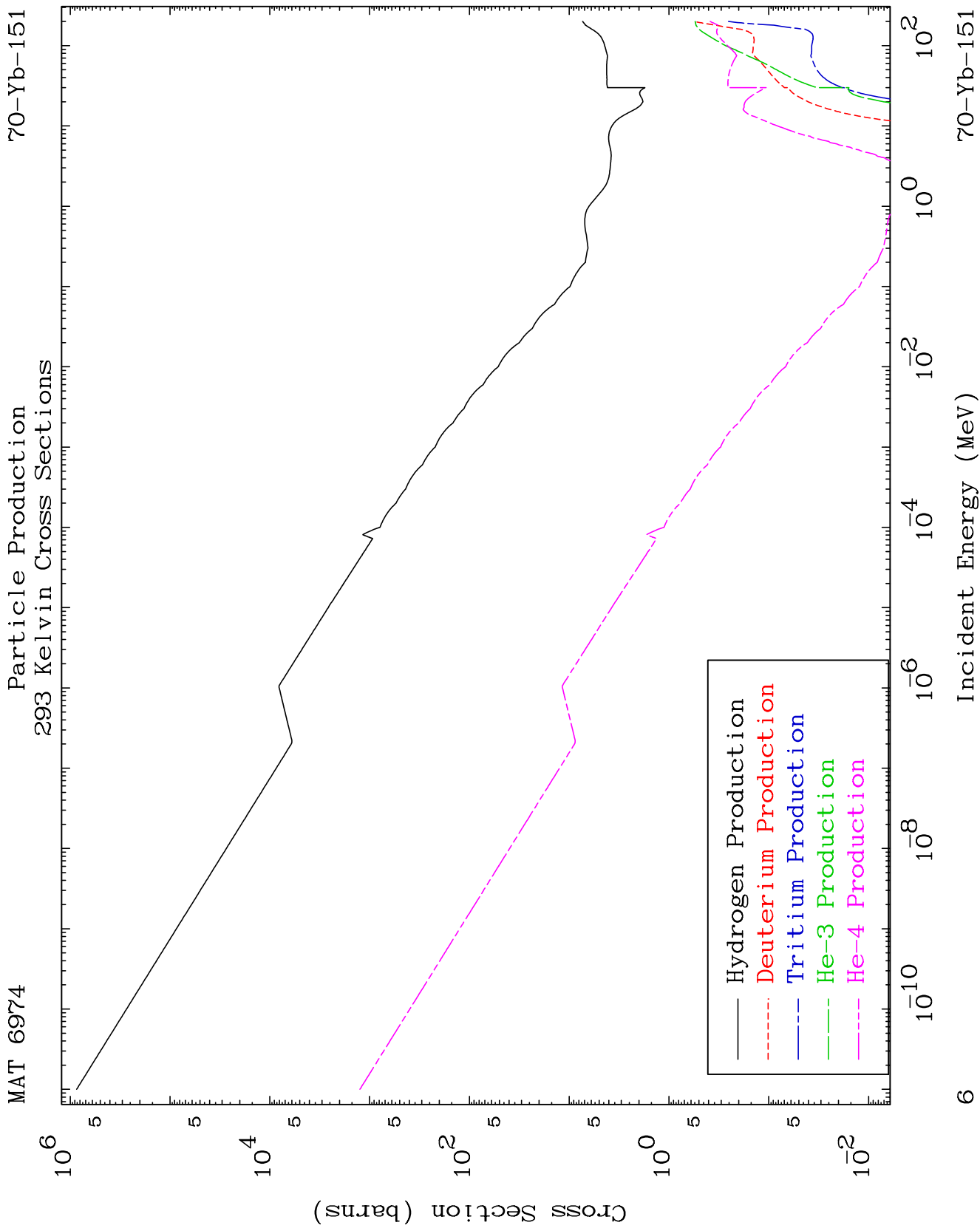
Charged Particle
293 Kelvin Cross Sections

70-Yb-151



5

70-Yb-151

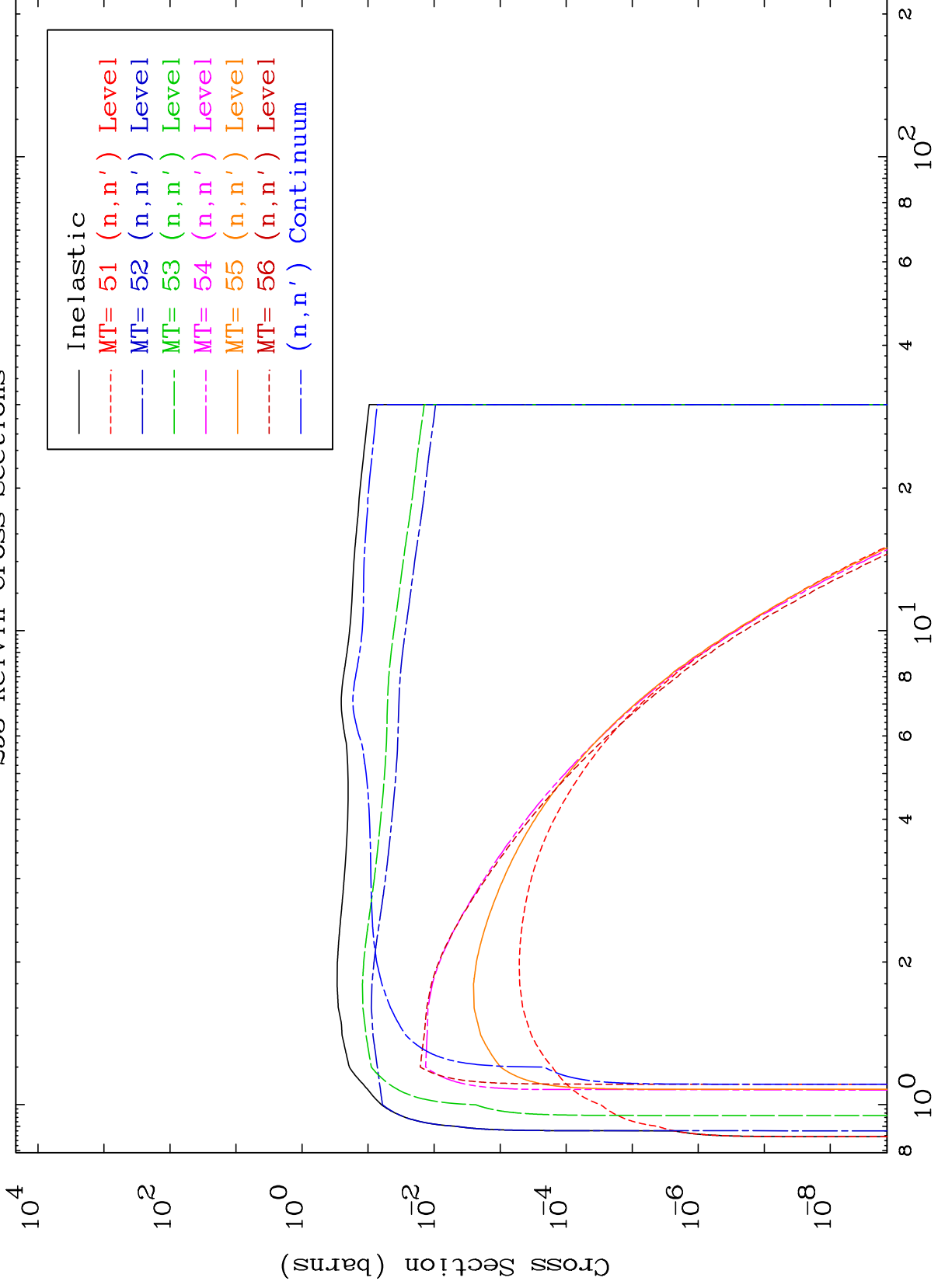


MAT 6974

(n,n') Level

70-Yb-151

293 Kelvin Cross Sections



Incident Energy (MeV)

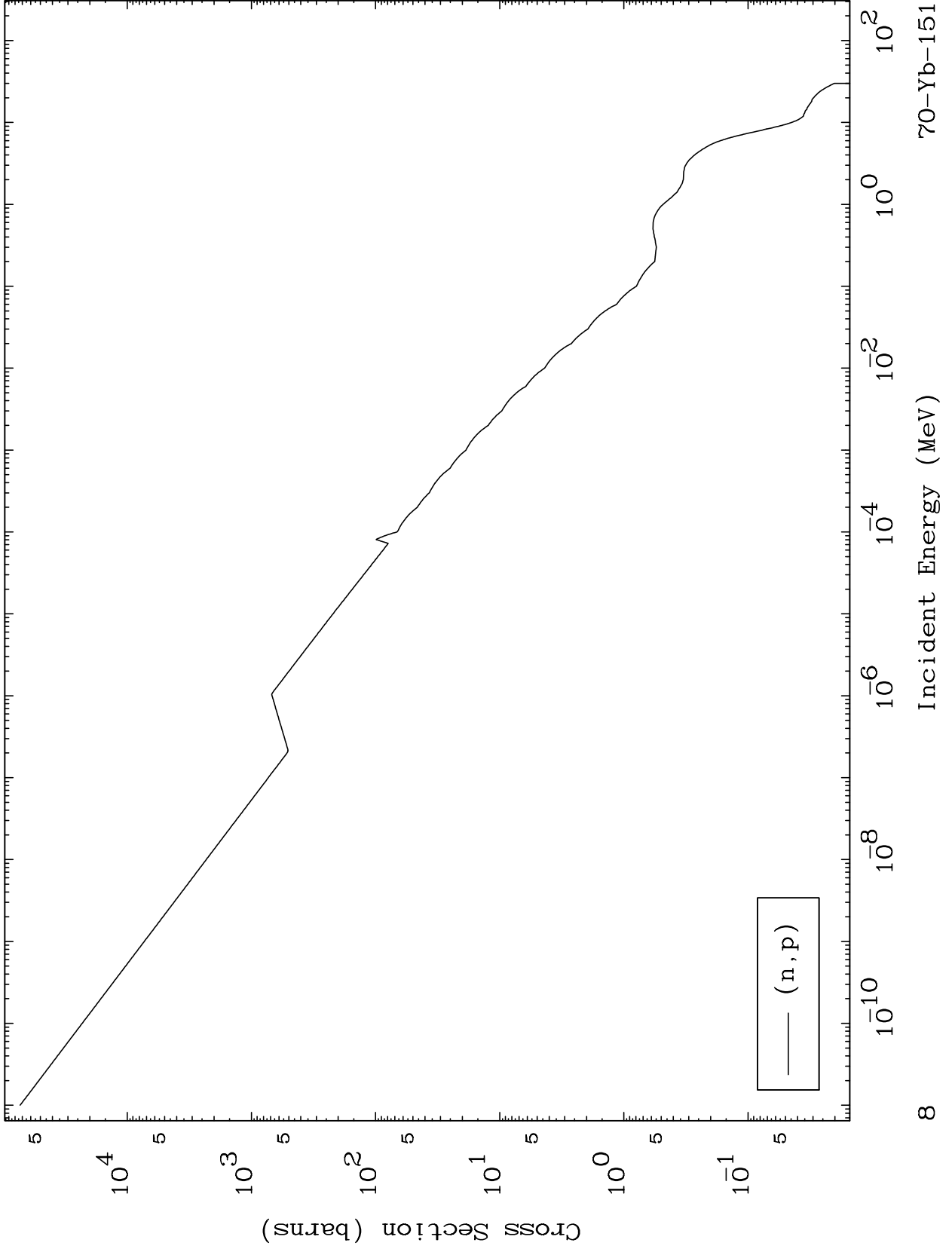
70-Yb-151

7

MAT 6974

(n,p) Levels
293 Kelvin Cross Sections

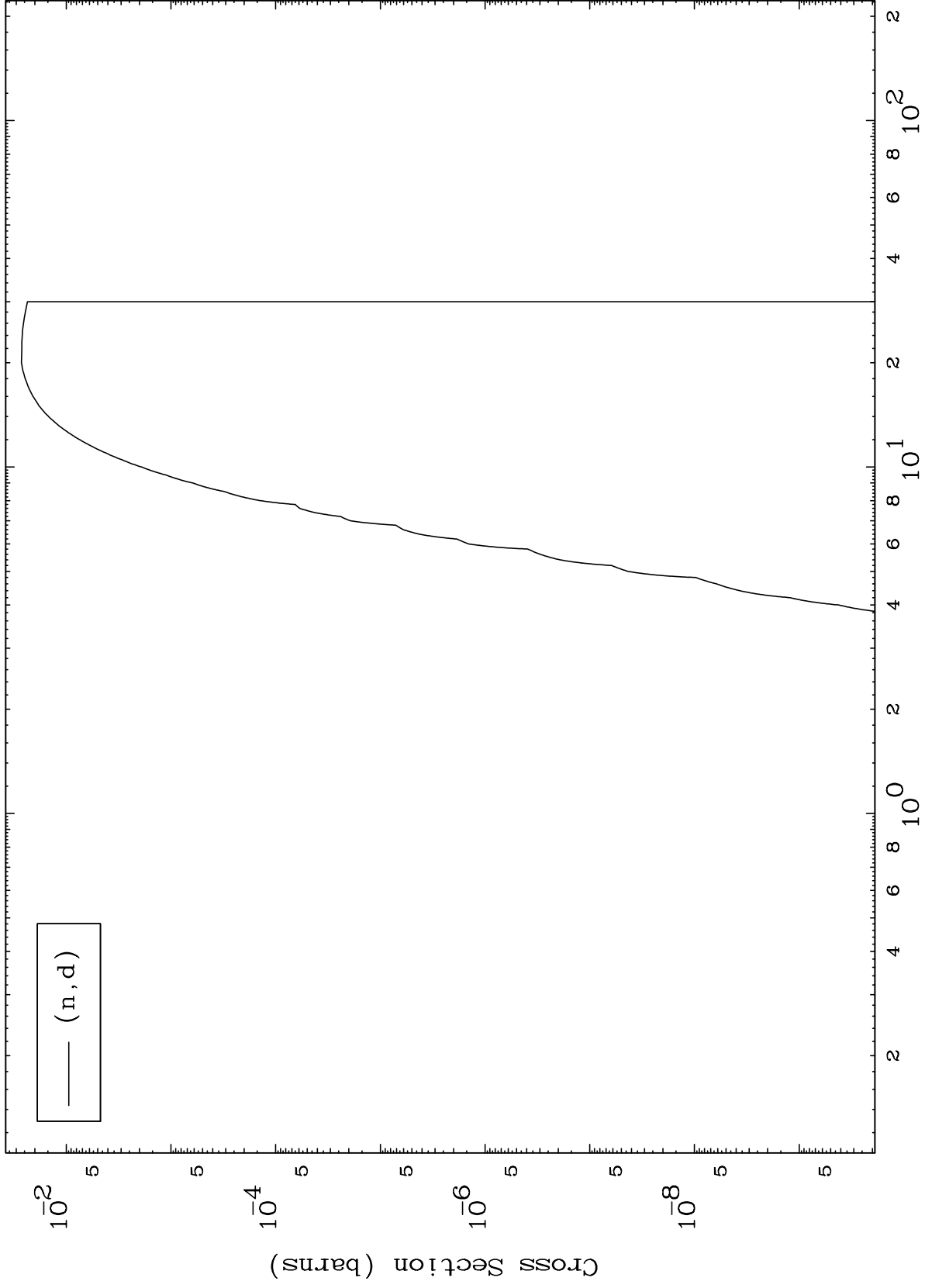
70-Yb-151



MAT 6974

(n,d) Levels
293 Kelvin Cross Sections

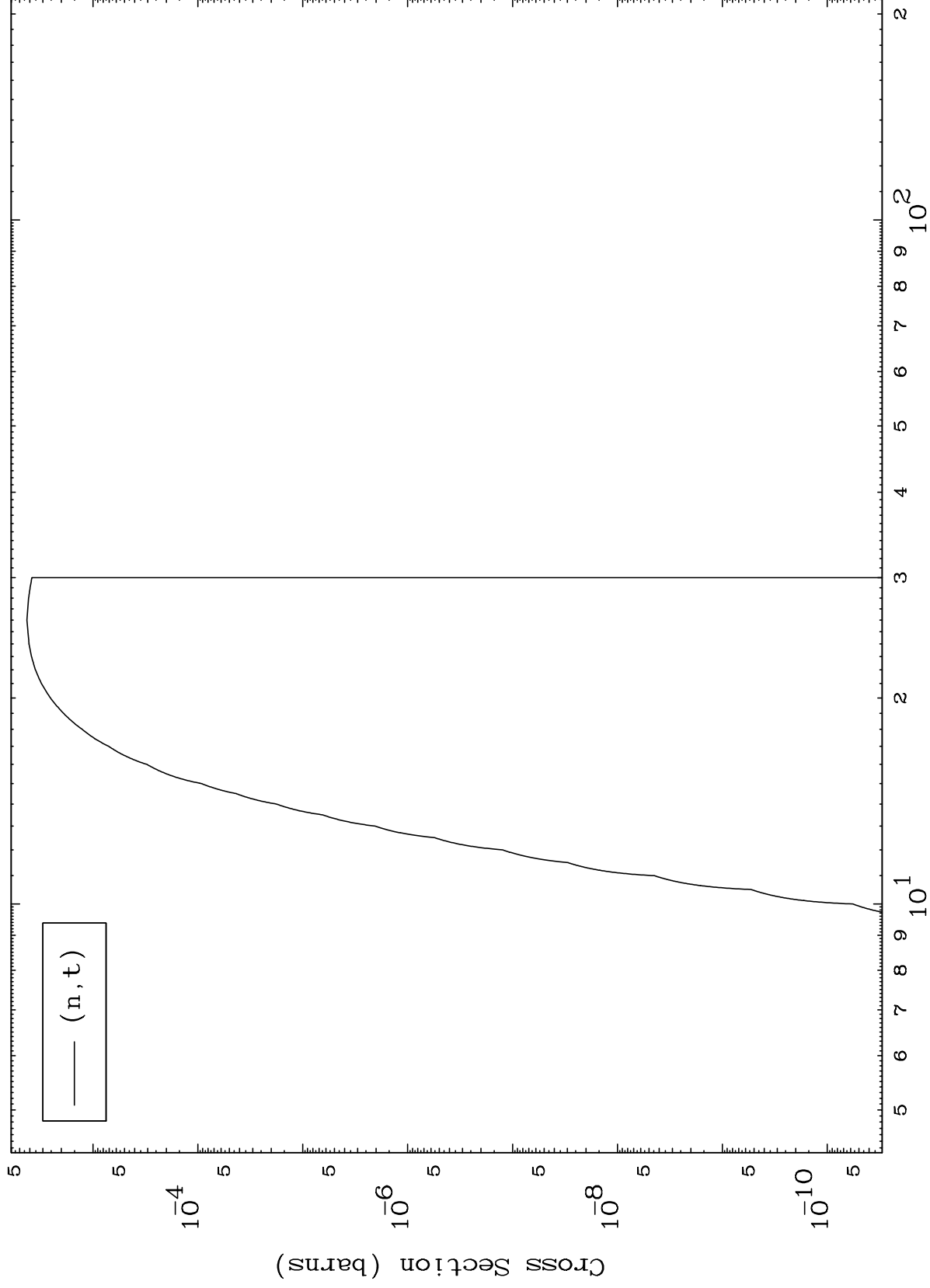
70-Yb-151



MAT 6974

(n,t) Levels
293 Kelvin Cross Sections

70-Yb-151



10

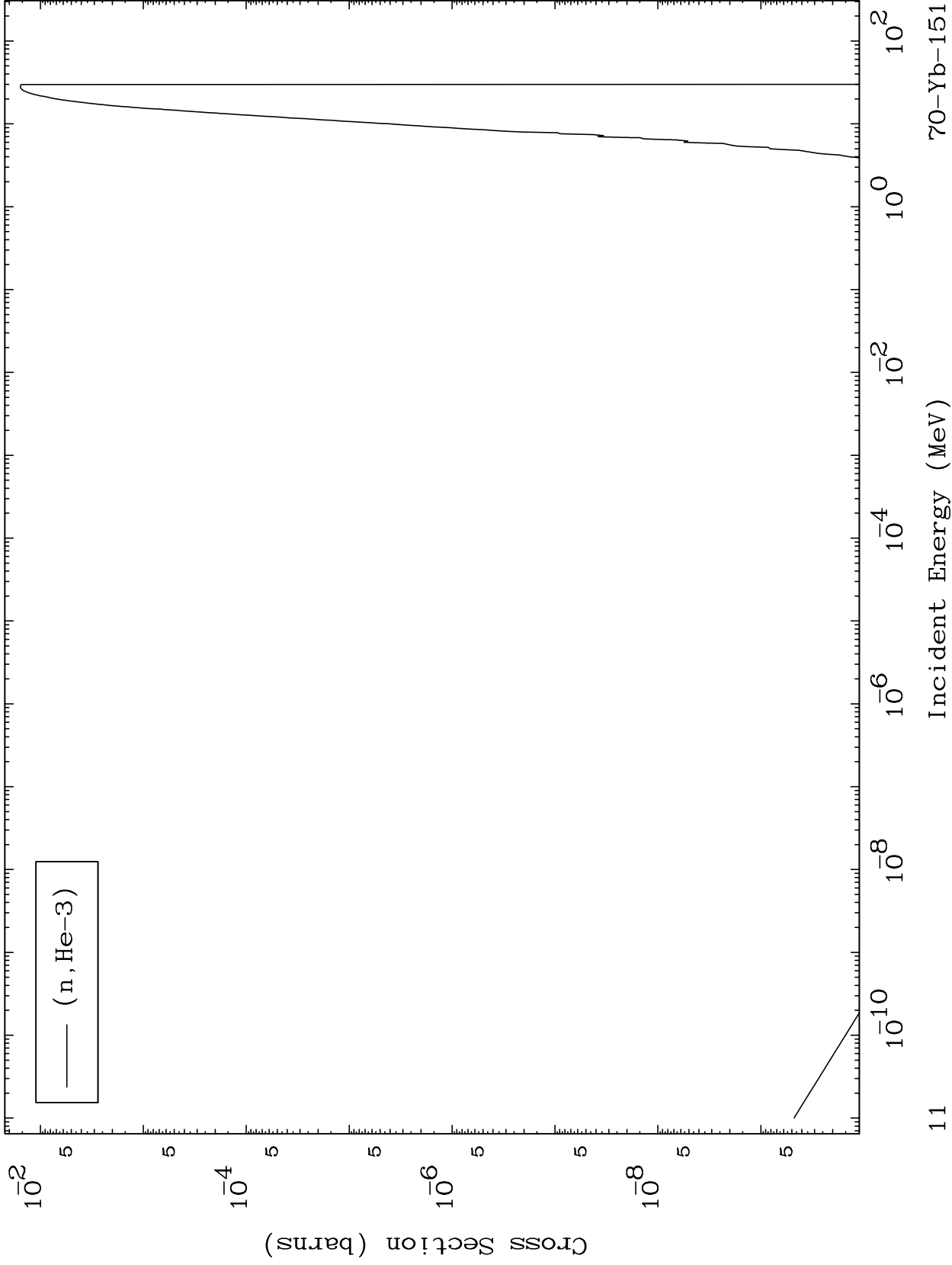
Incident Energy (MeV)

70-Yb-151

MAT 6974

(n,He3) Levels
293 Kelvin Cross Sections

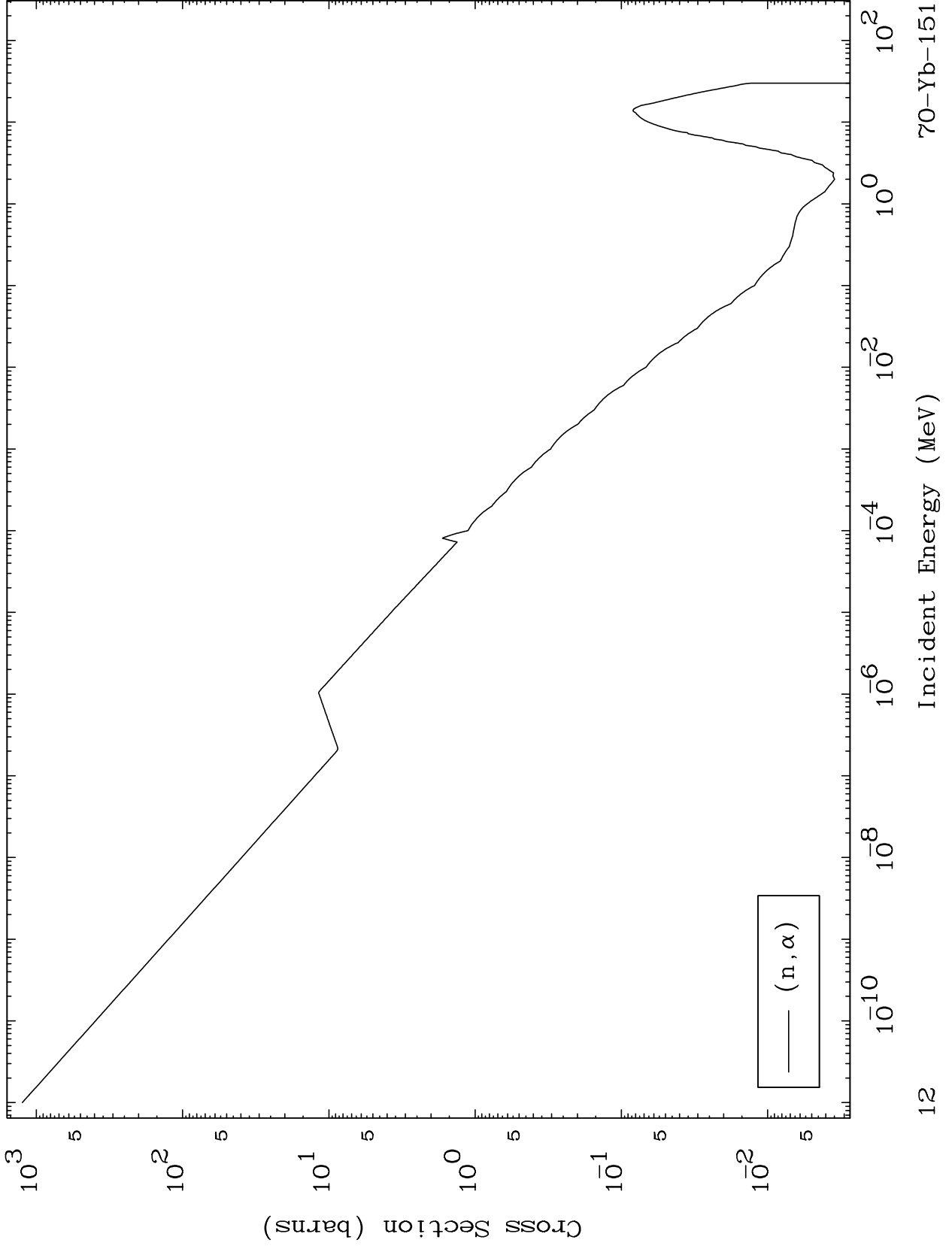
70-Yb-151

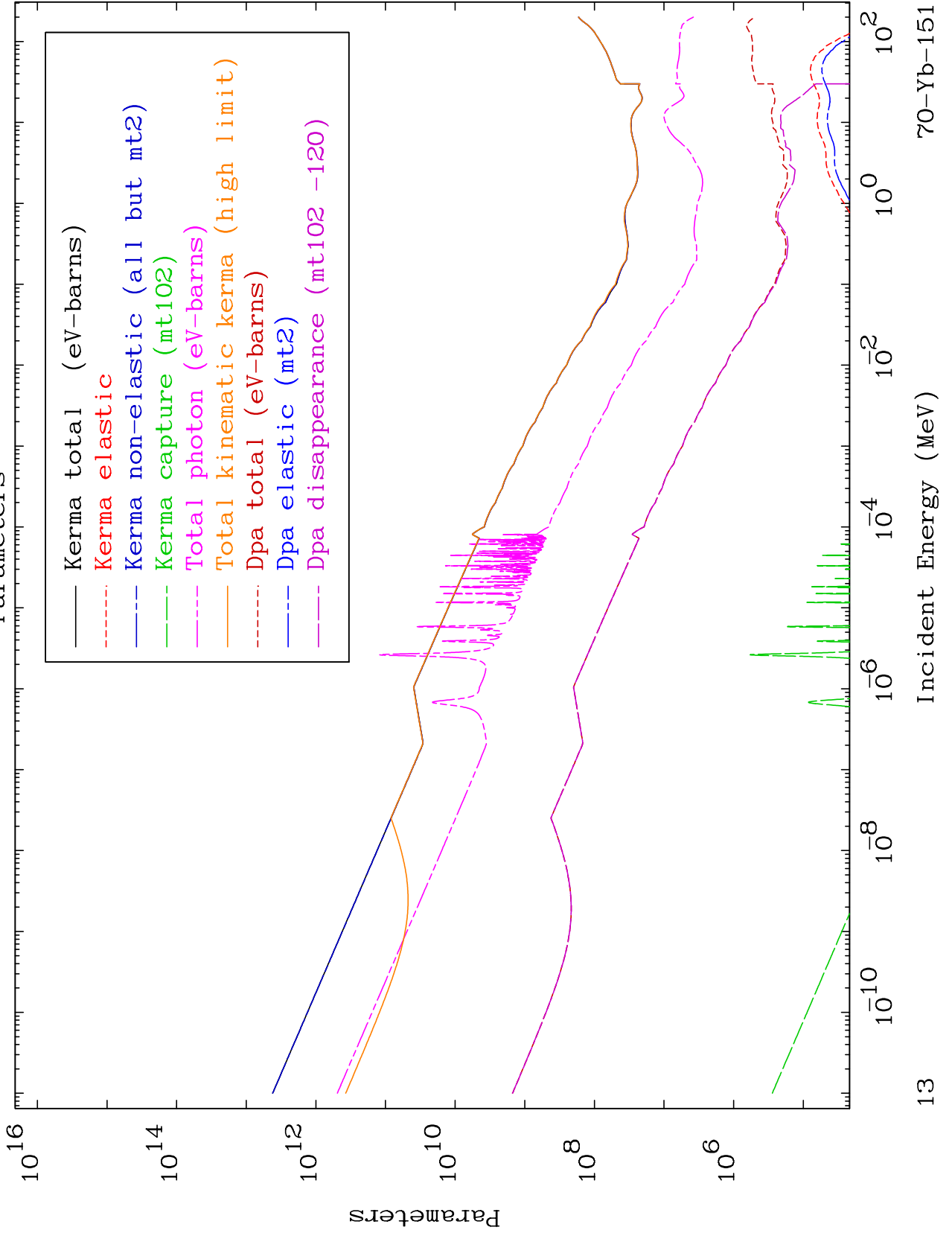


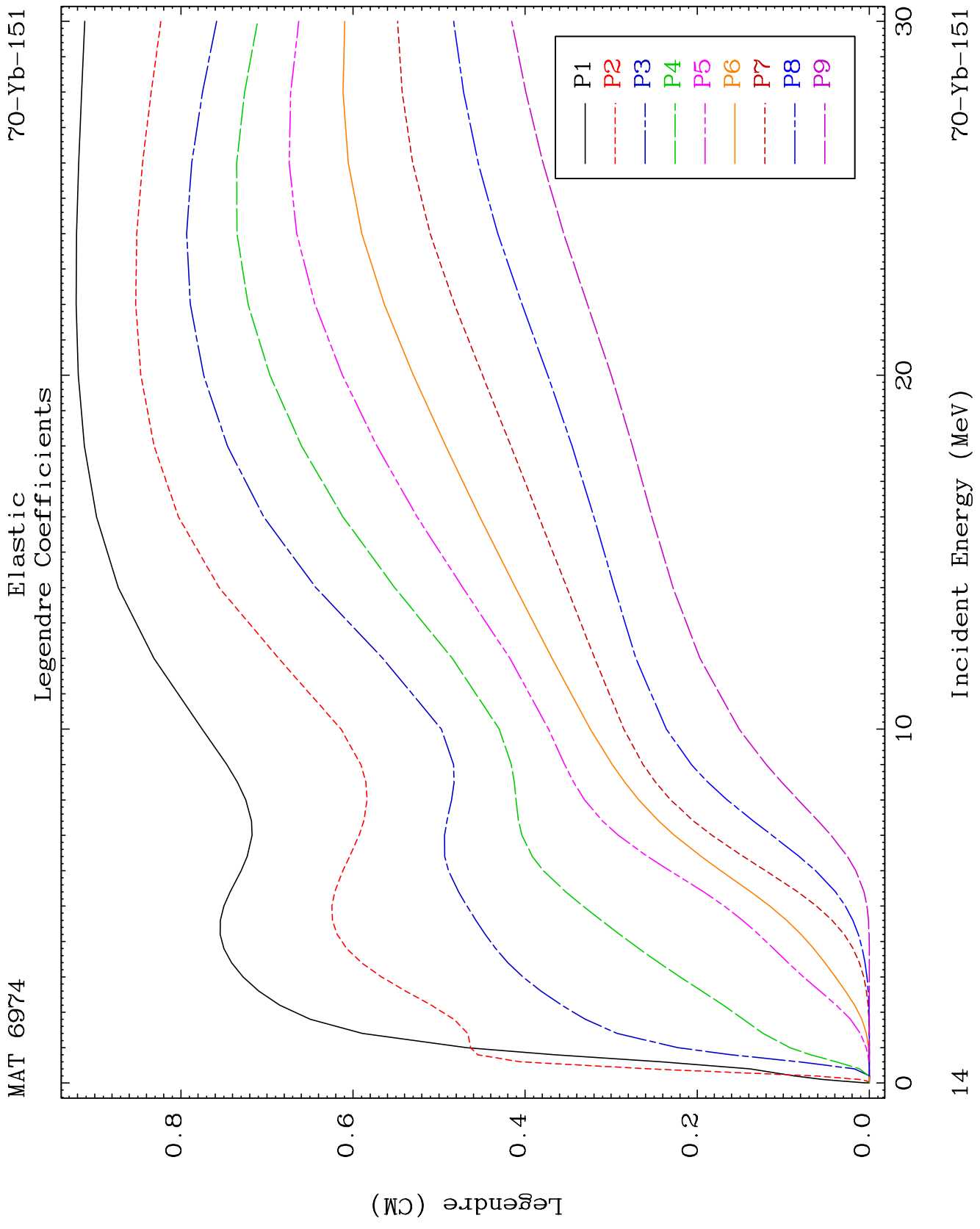
MAT 6974

(n, α) Levels
293 Kelvin Cross Sections

70-Yb-151



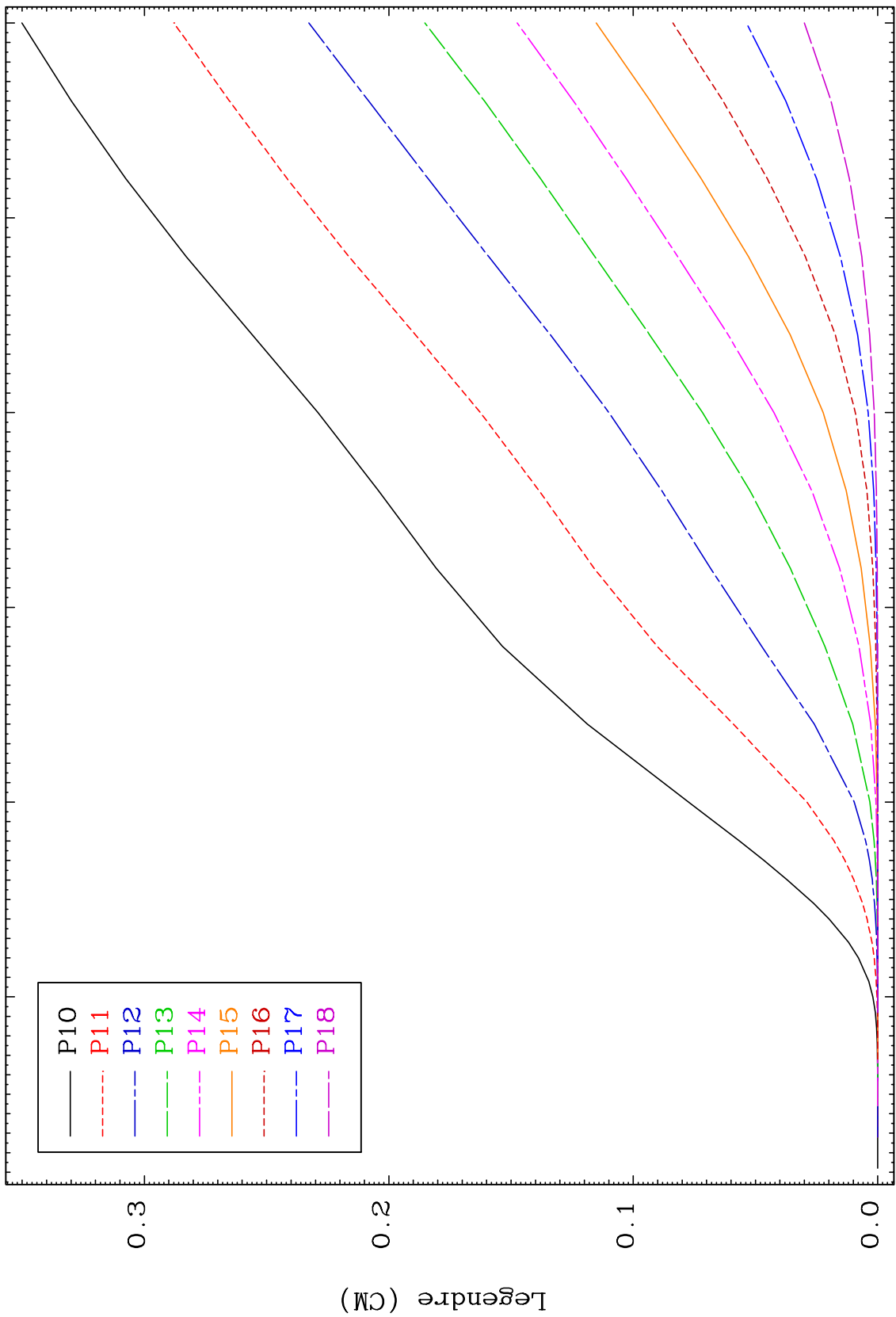




MAT 6974

Elastic Legendre Coefficients

70-Yb-151



15

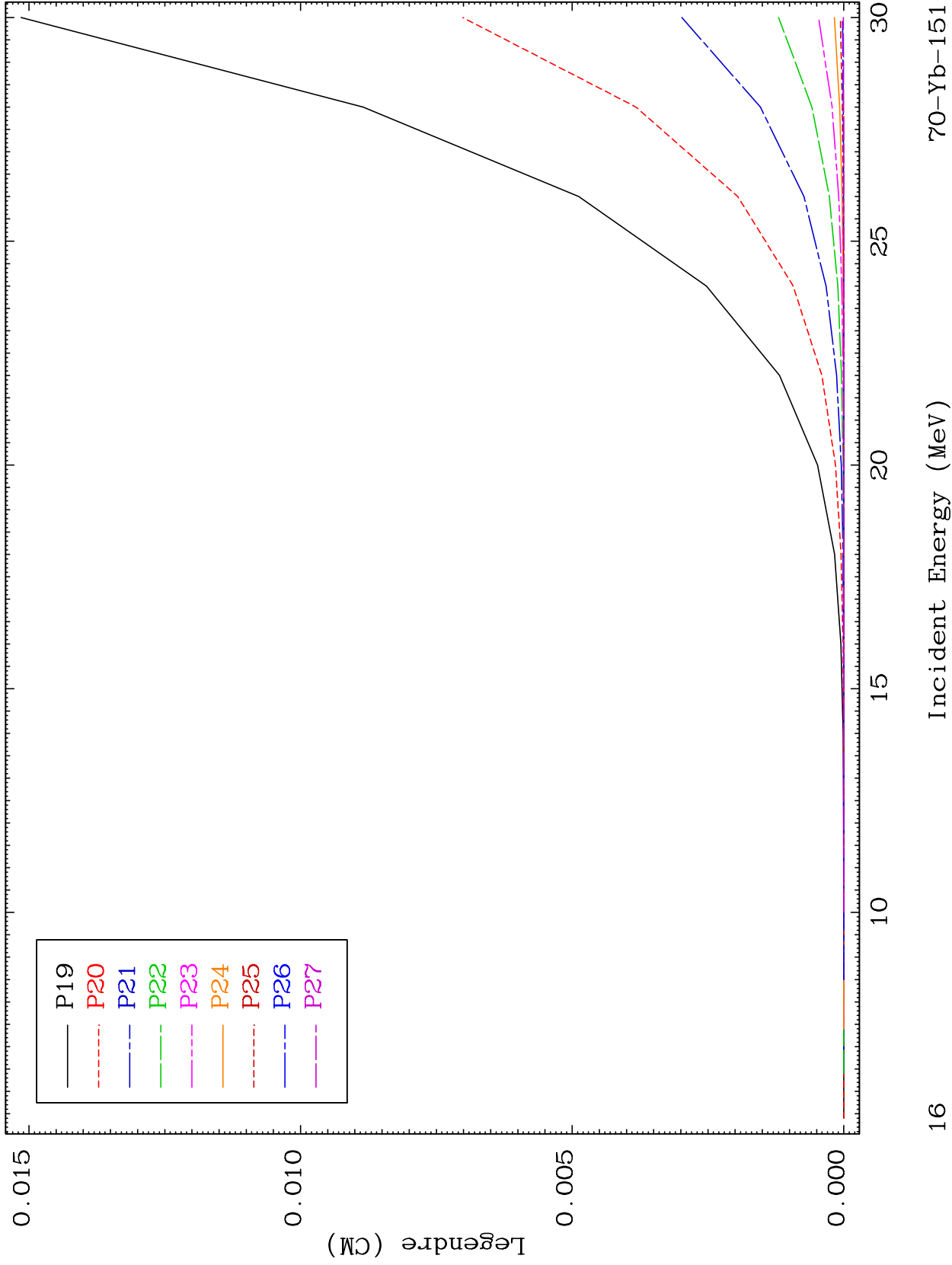
Incident Energy (MeV)

70-Yb-151

MAT 6974

Elastic
Legendre Coefficients

70-Yb-151



16

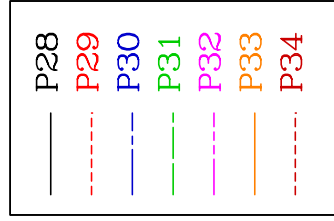
Incident Energy (MeV)

70-Yb-151

MAT 6974

Elastic
Legendre Coefficients

70-Yb-151



$\times 10^{-6}$

2.0

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

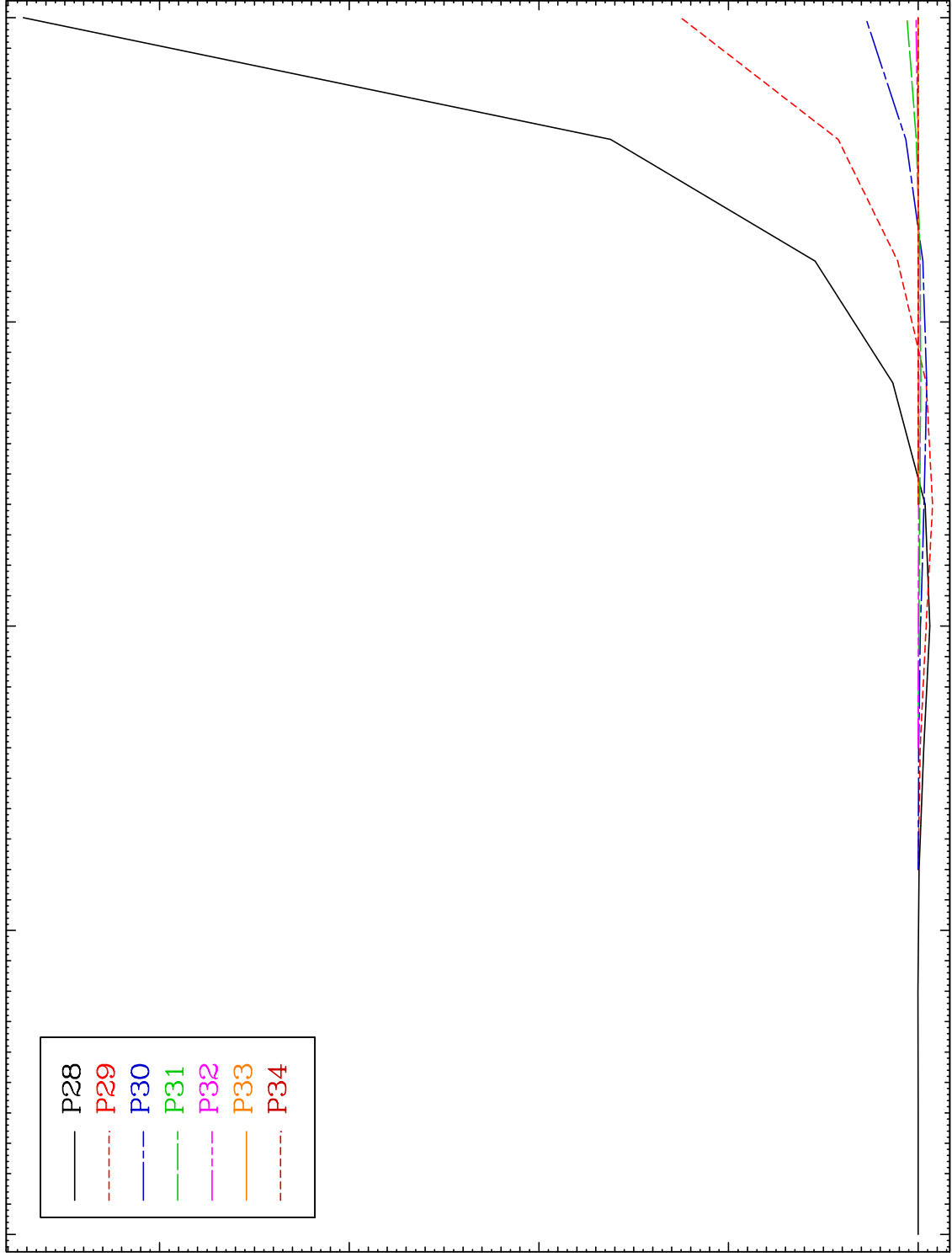
25

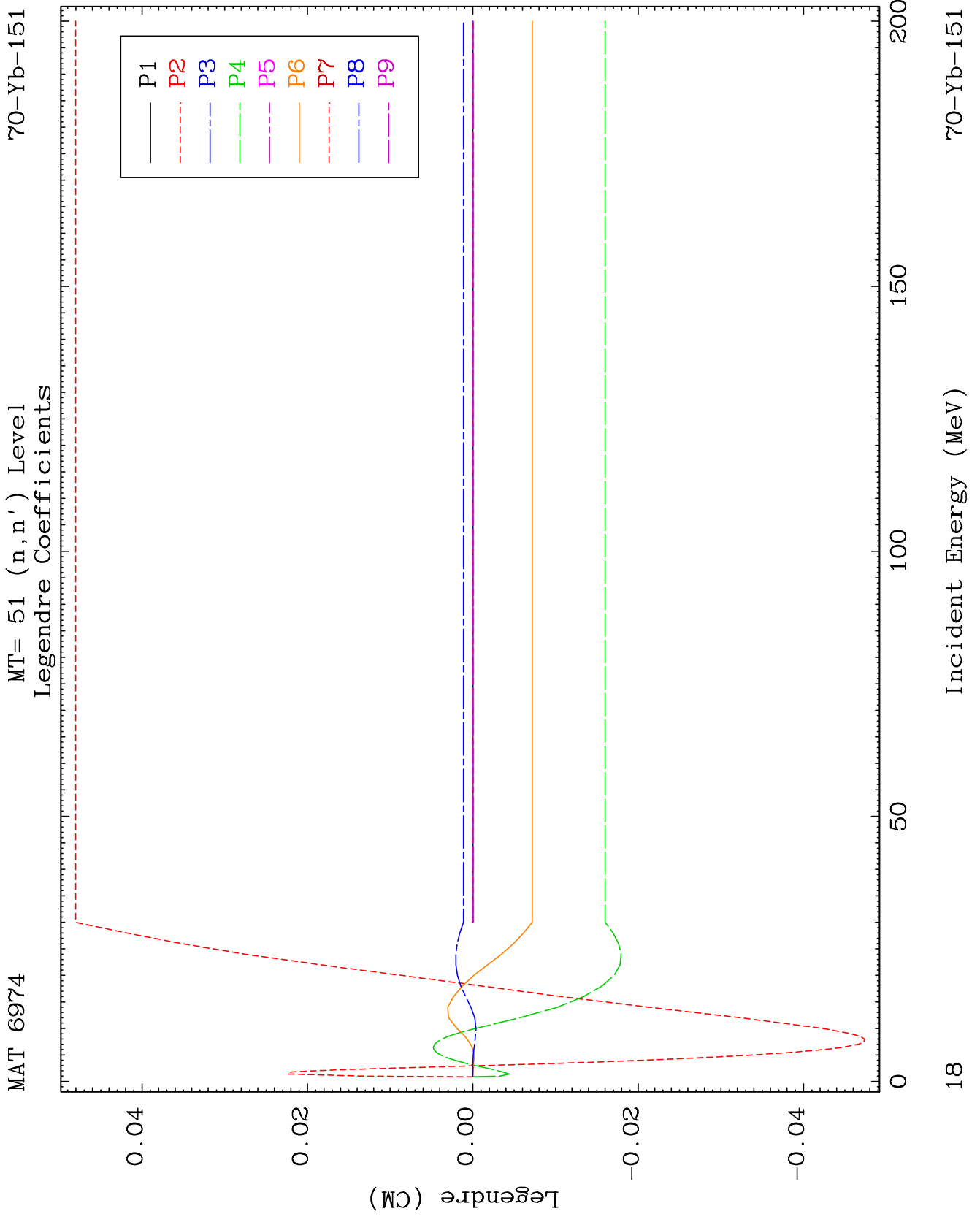
30

17

Incident Energy (MeV)

70-Yb-151

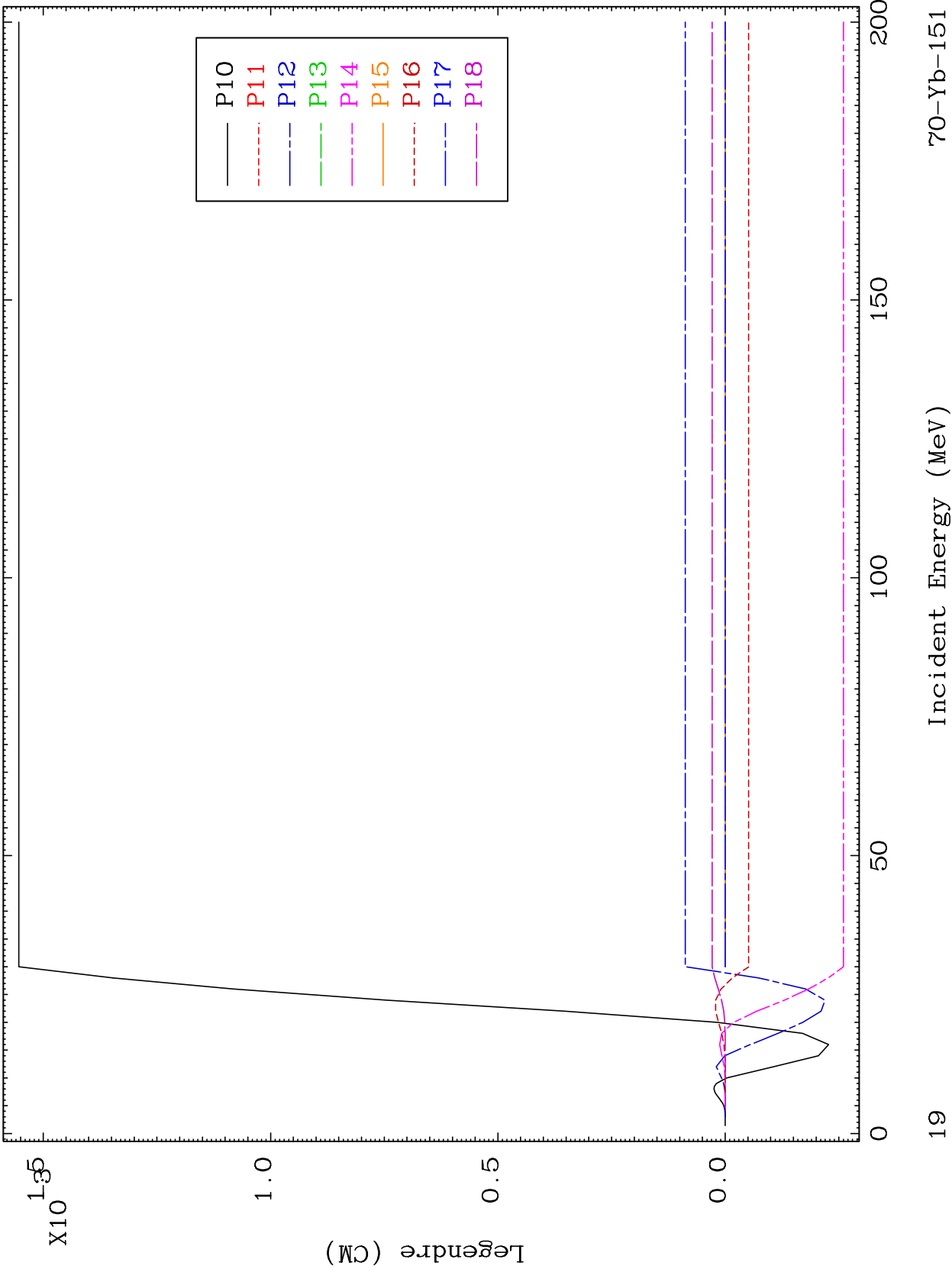




MAT 6974

MT= 51 (n,n') Level
Legendre Coefficients

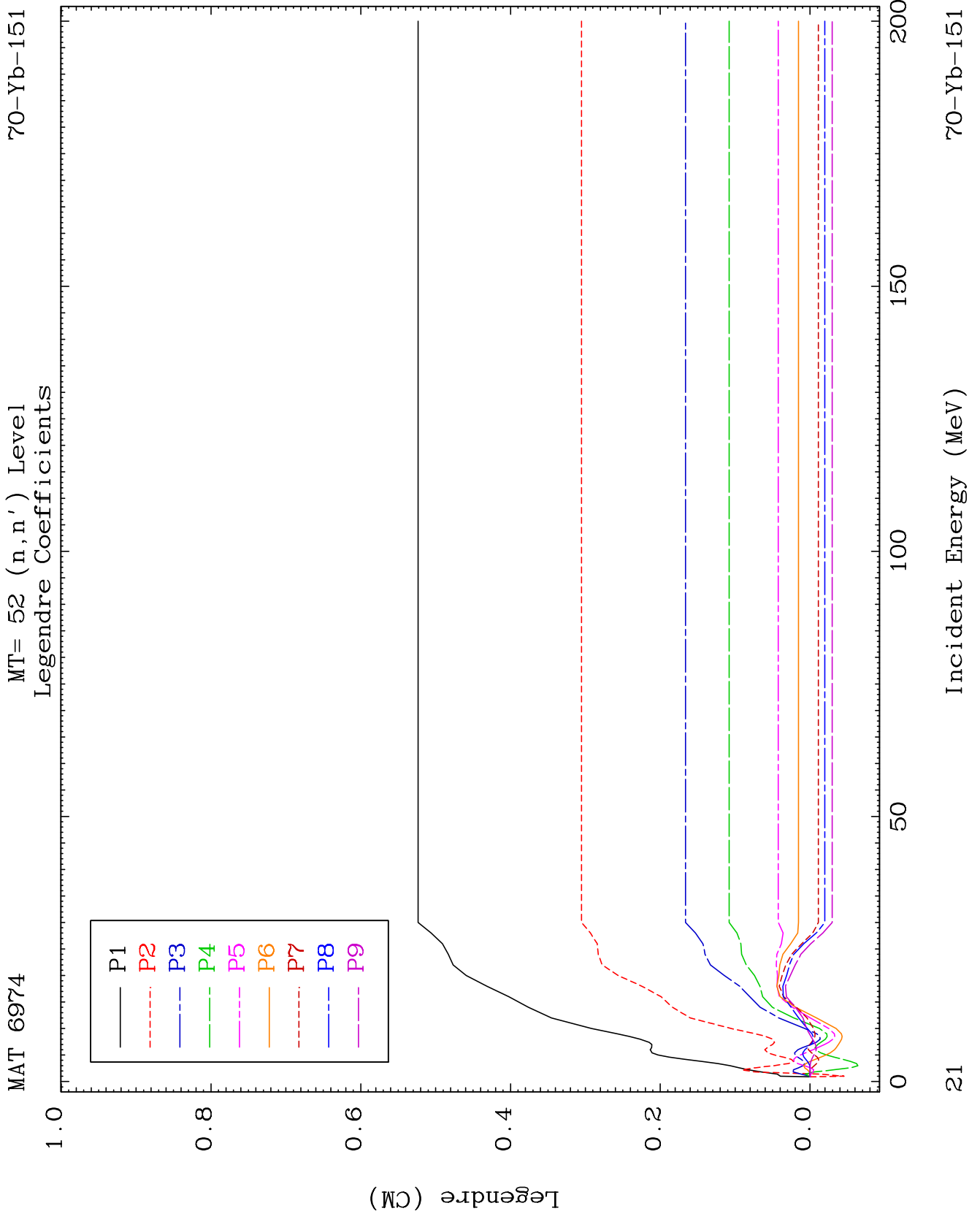
70-Yb-151



19

Incident Energy (MeV)

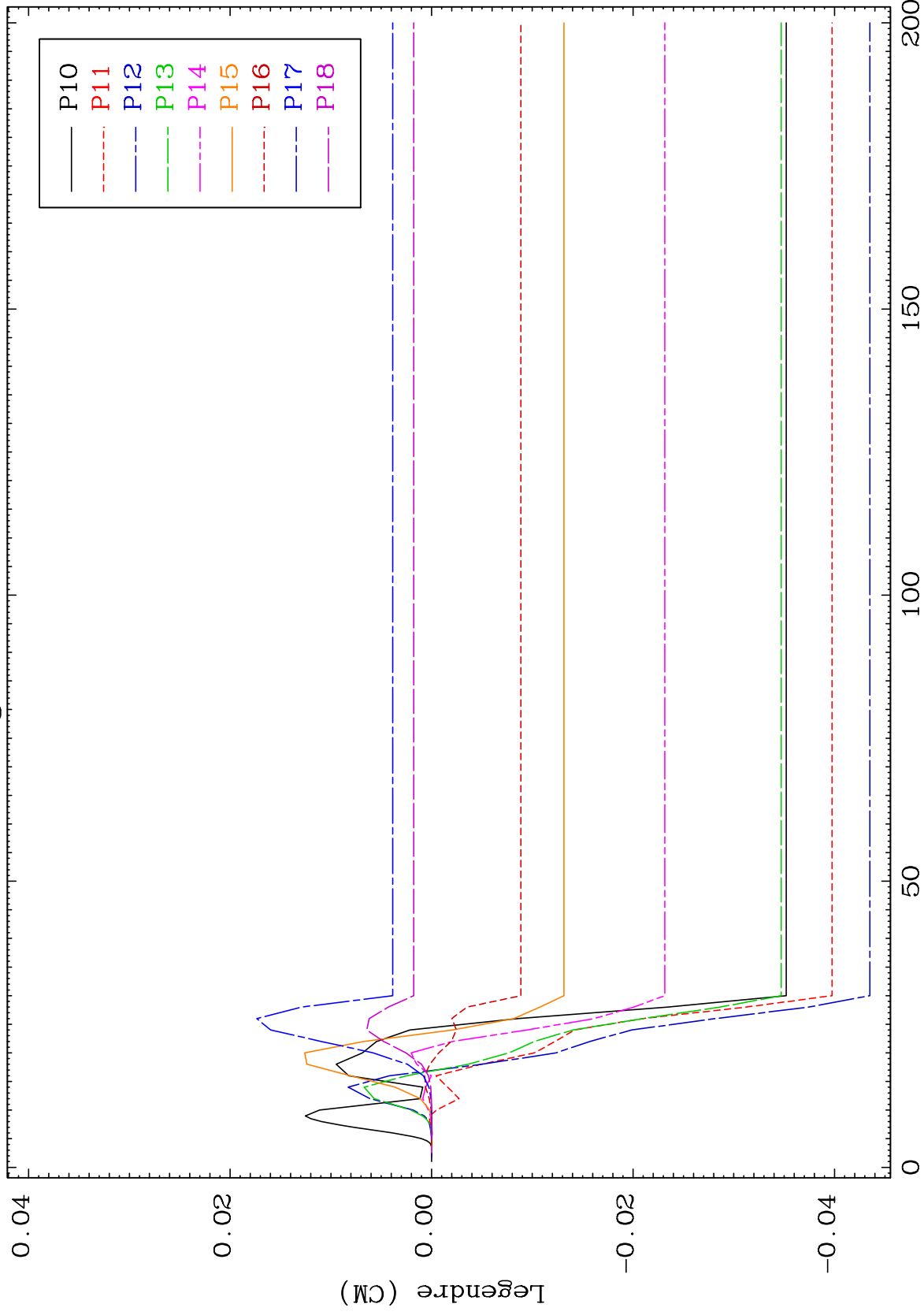
70-Yb-151



MAT 6974

MT= 52 (n,n') Level
Legendre Coefficients

70-Yb-151



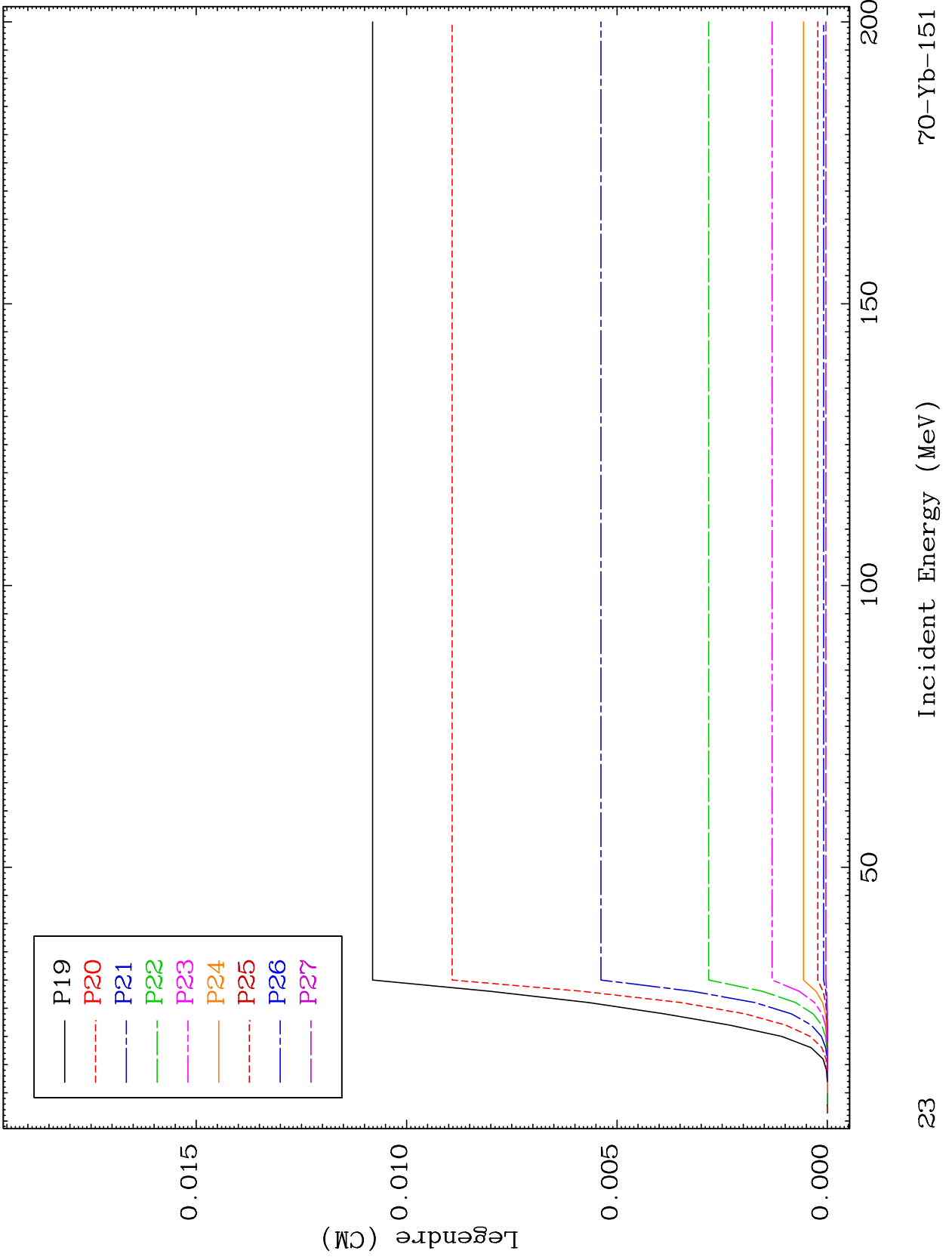
22

70-Yb-151

MAT 6974

MT= 52 (n,n') Level
Legendre Coefficients

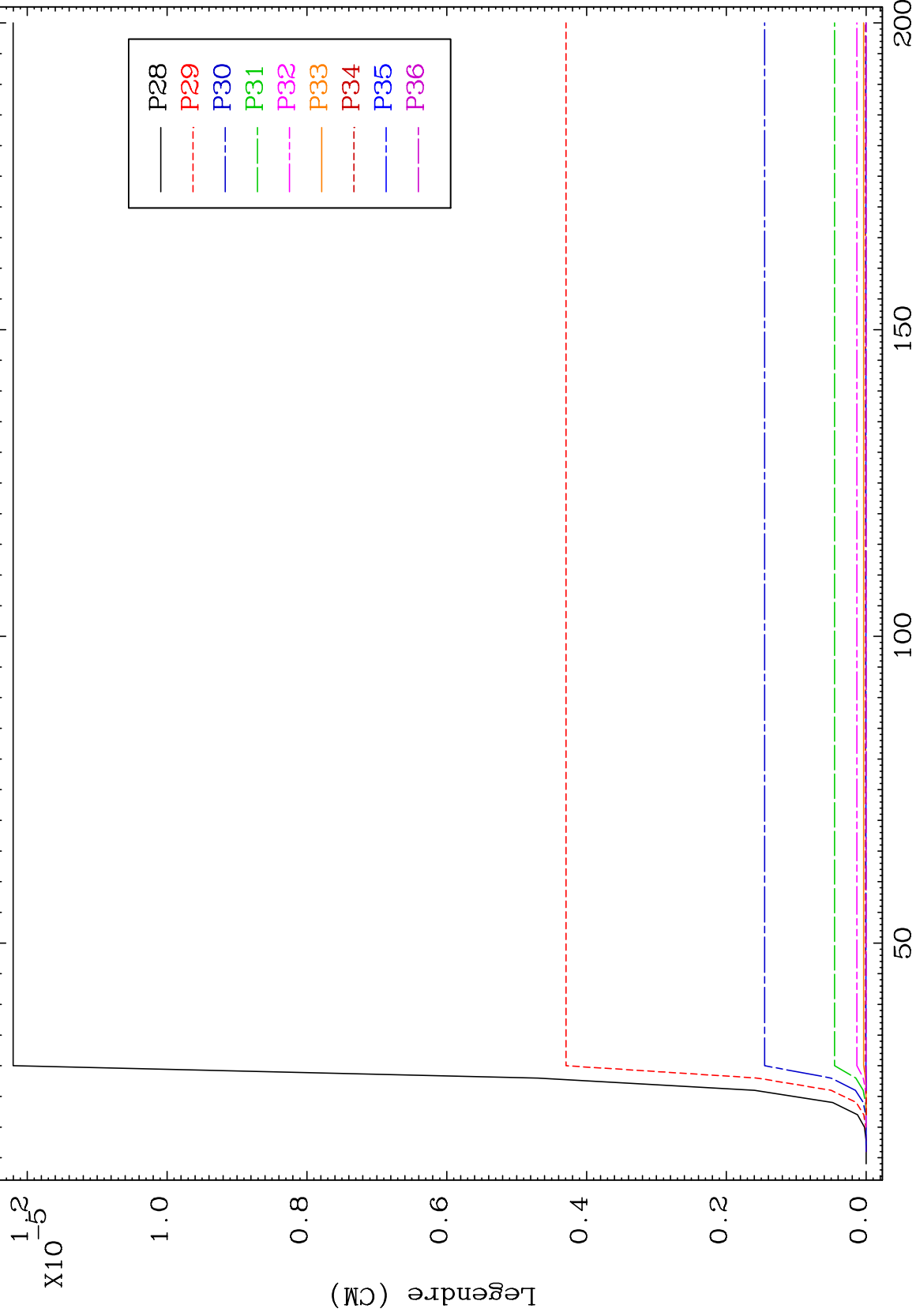
70-Yb-151



MAT 6974

MT= 52 (n,n') Level
Legendre Coefficients

70-Yb-151



24

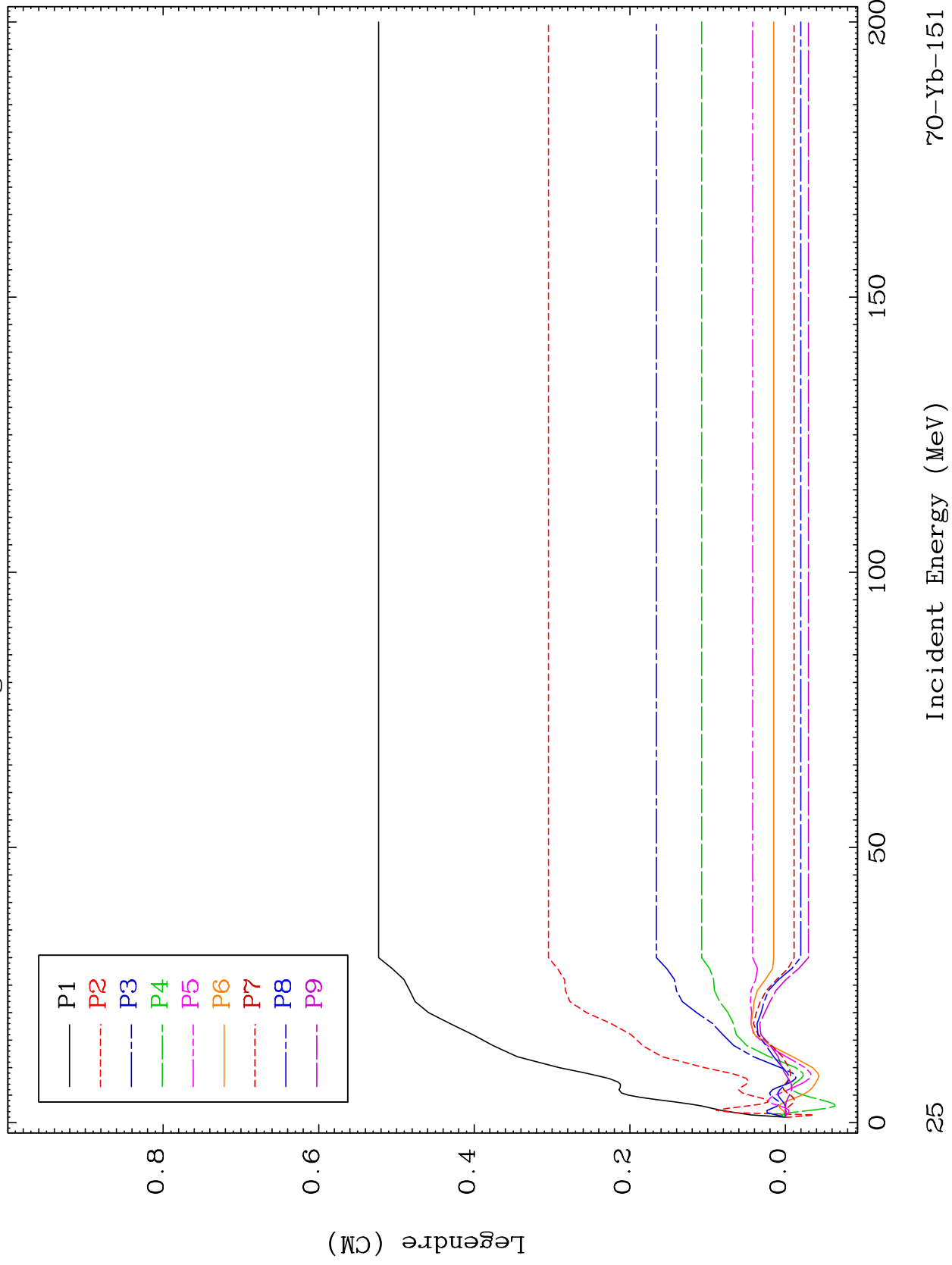
Incident Energy (MeV)

70-Yb-151

MAT 6974

MT= 53 (n,n') Level
Legendre Coefficients

70-Yb-151



70-Yb-151

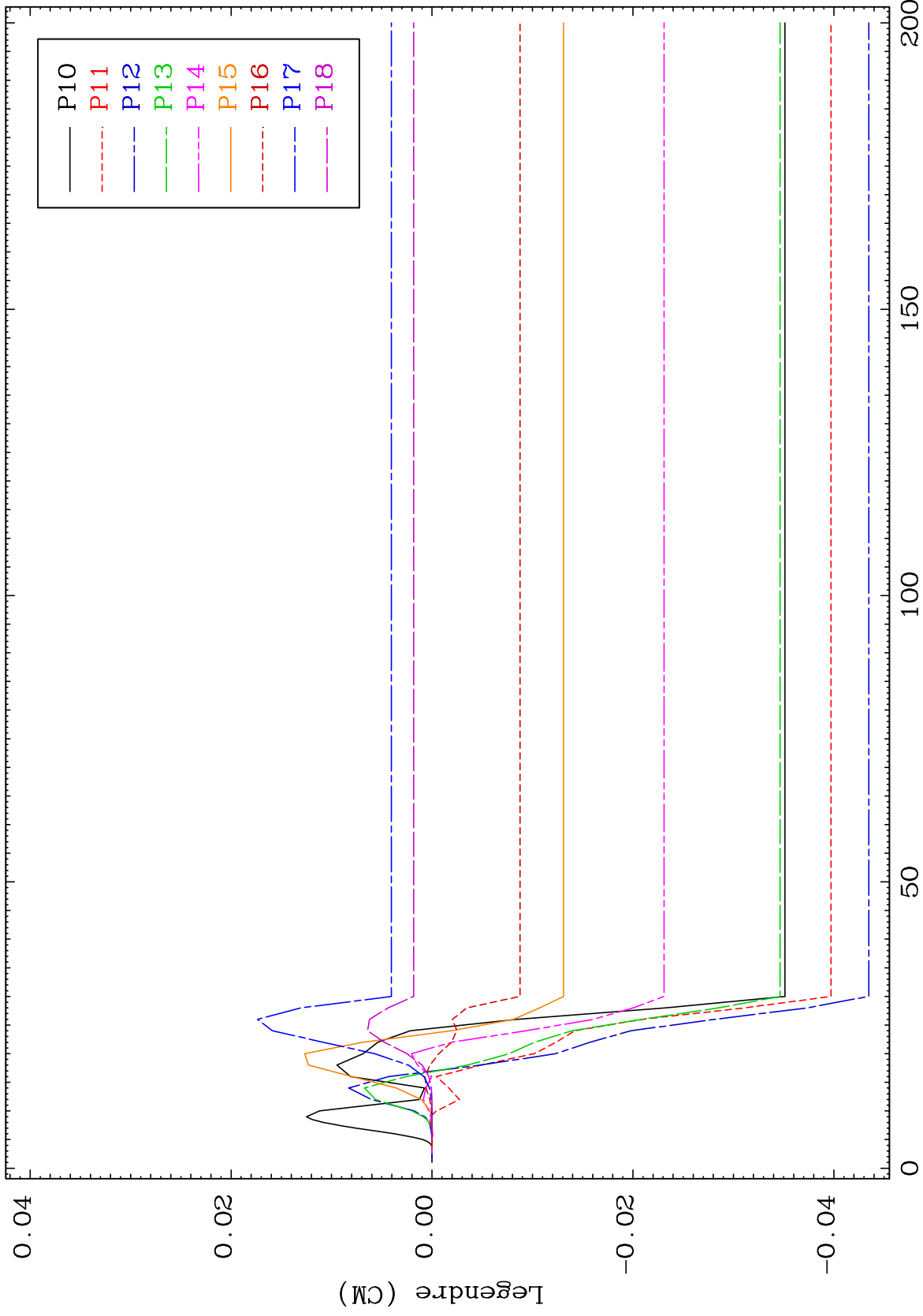
Incident Energy (MeV)

25

MAT 6974

MT= 53 (n,n') Level
Legendre Coefficients

70-Yb-151



26

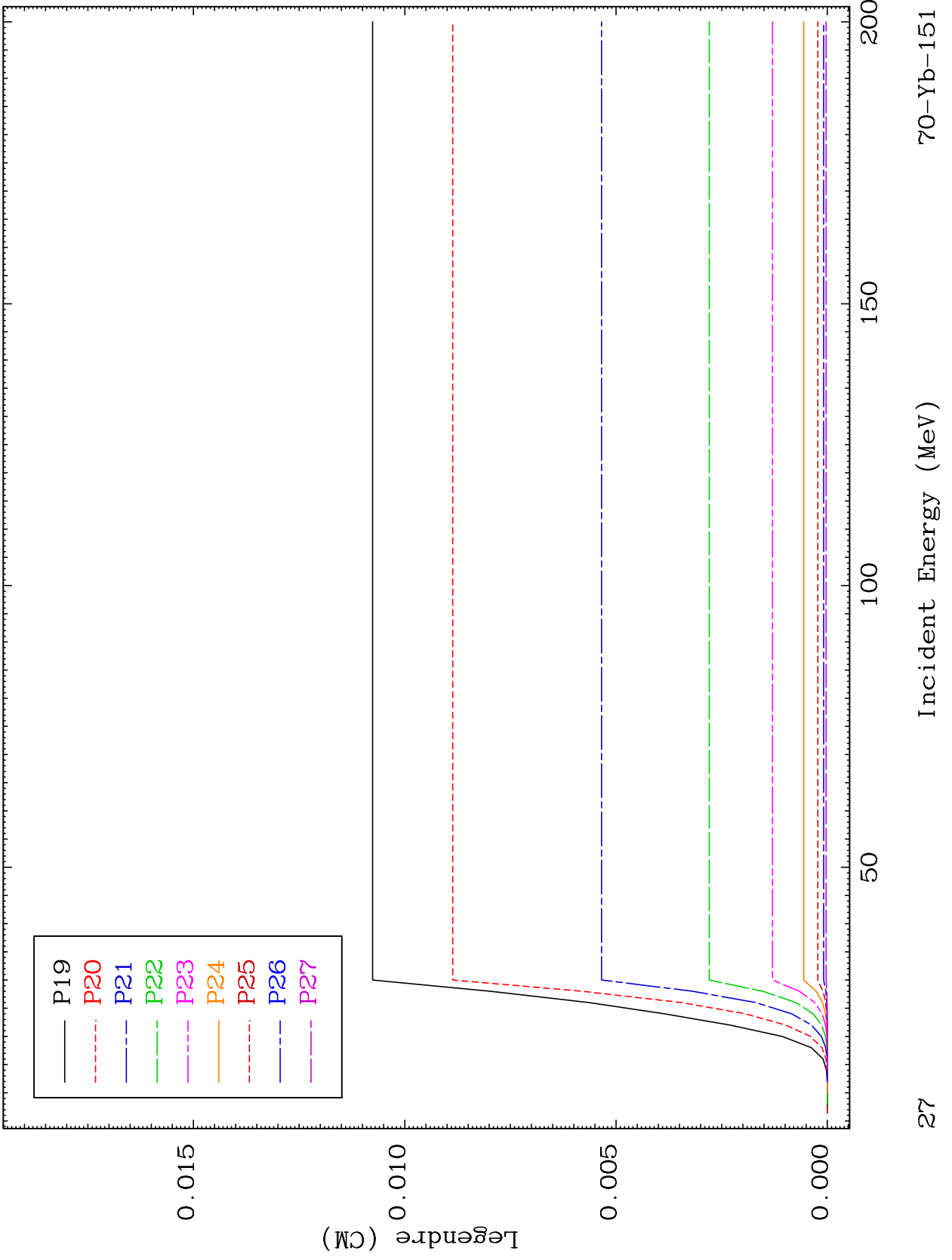
Incident Energy (MeV)

70-Yb-151

MAT 6974

MT= 53 (n,n') Level
Legendre Coefficients

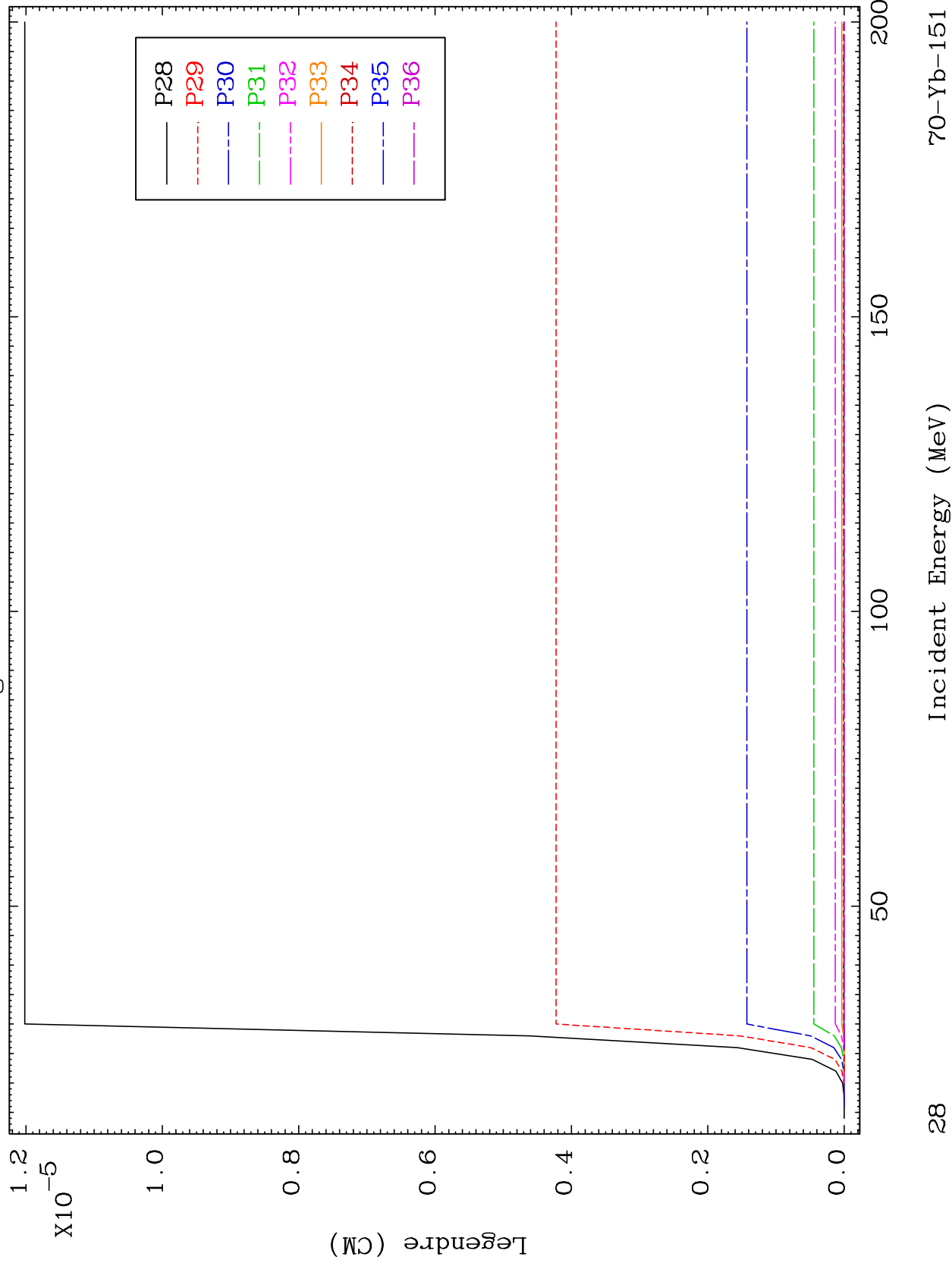
70-Yb-151



MAT 6974

MT= 53 (n,n') Level
Legendre Coefficients

70-Yb-151



28

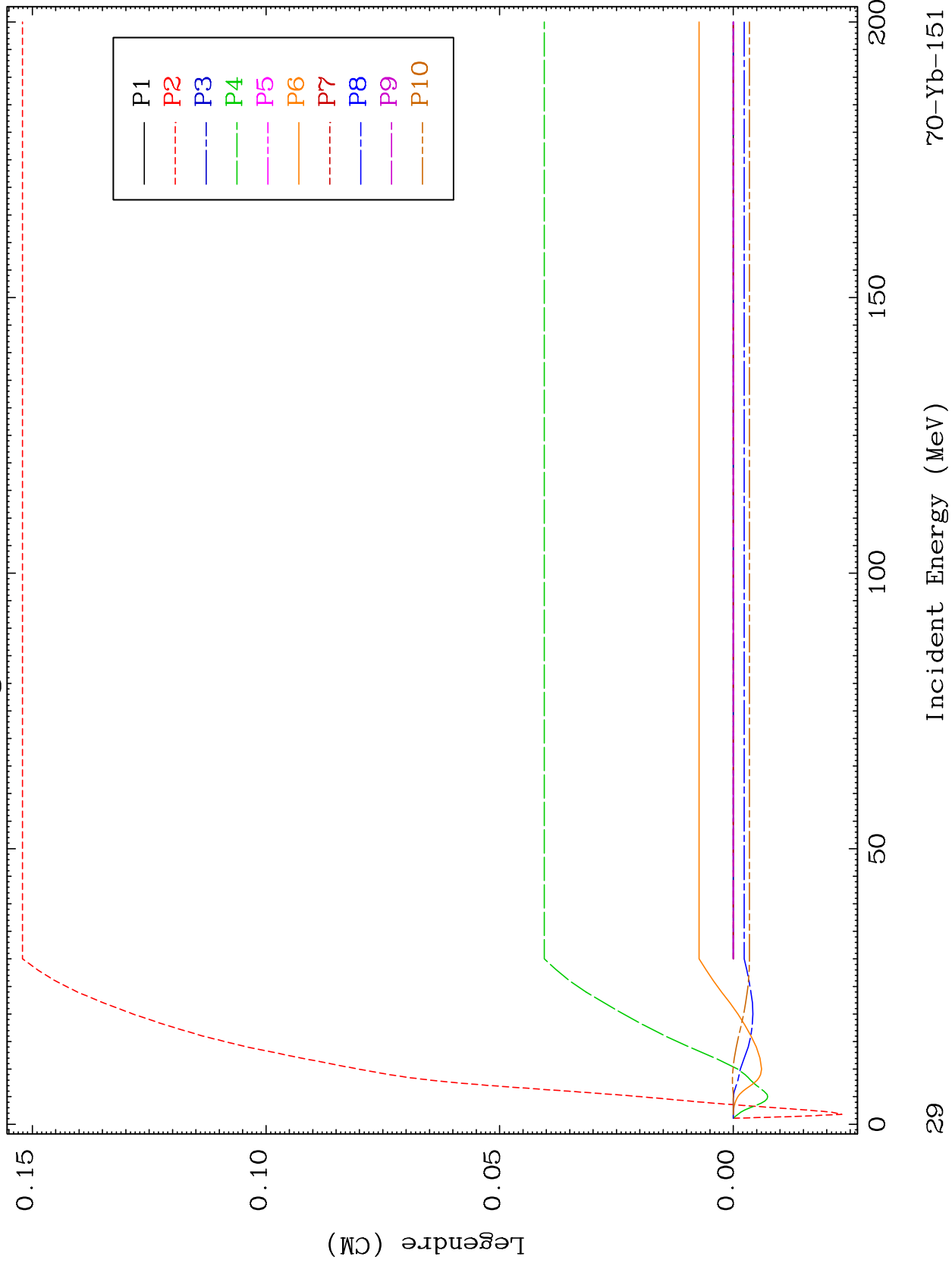
Incident Energy (MeV)

70-Yb-151

MAT 6974

MT= 54 (n,n') Level
Legendre Coefficients

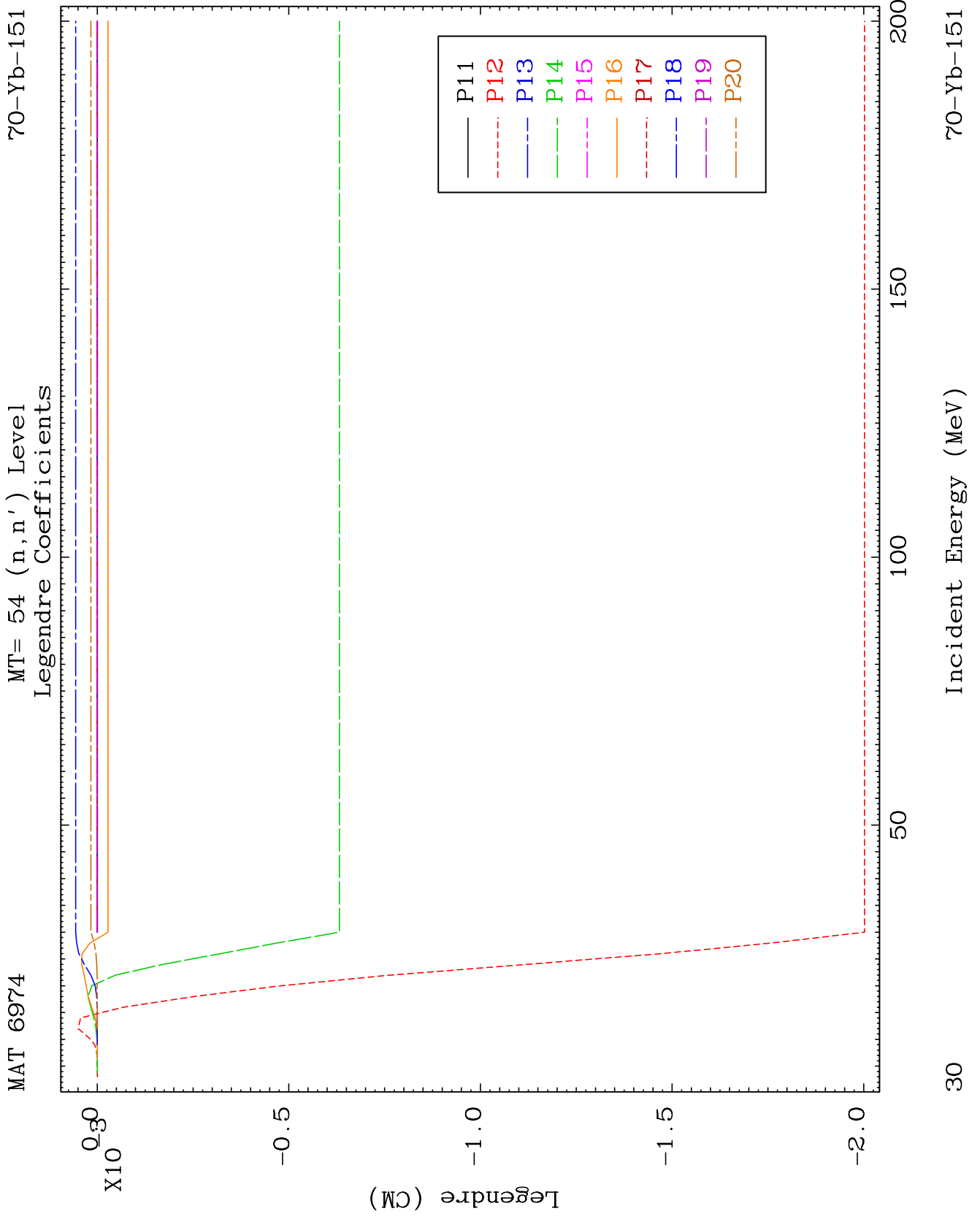
70-Yb-151

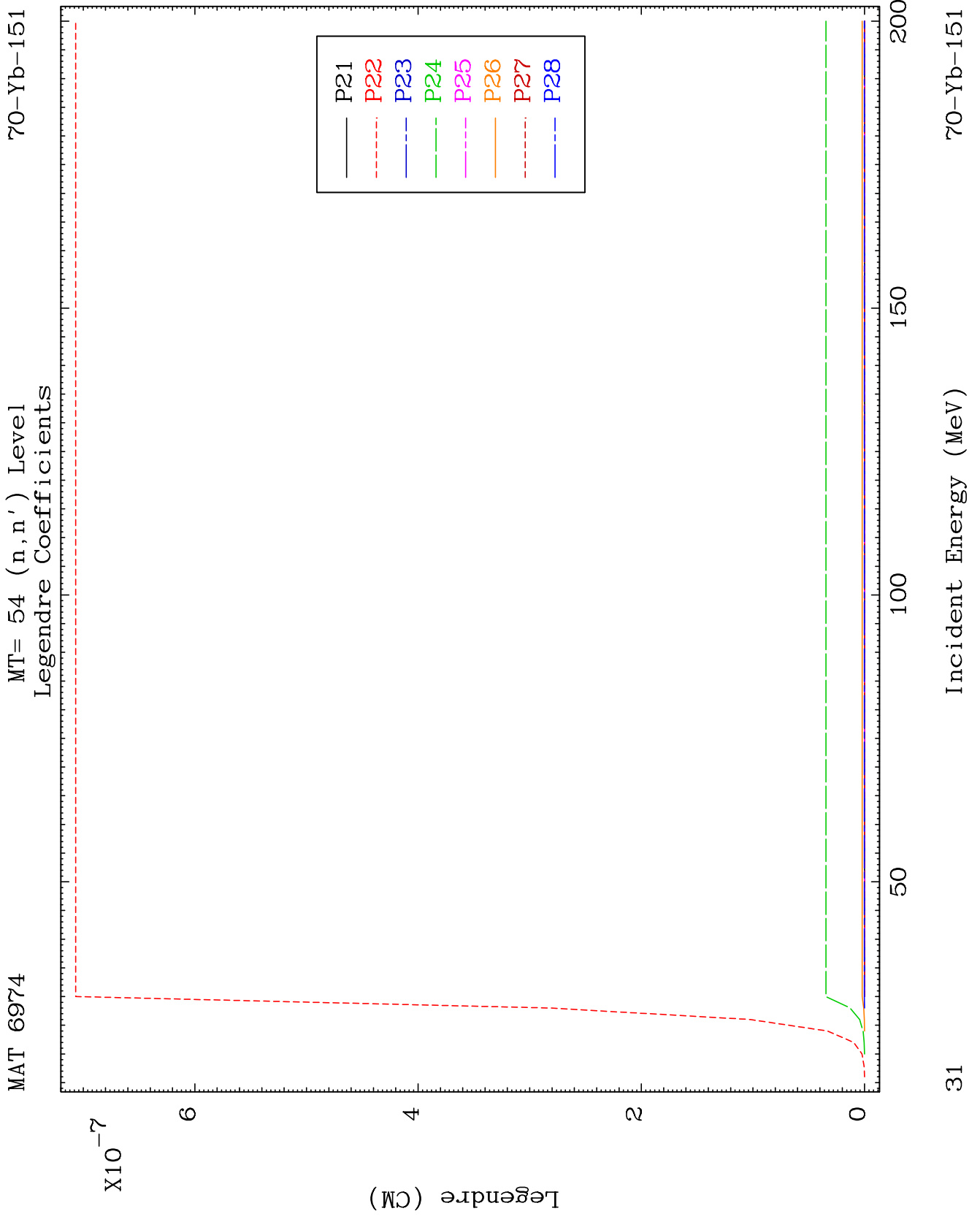


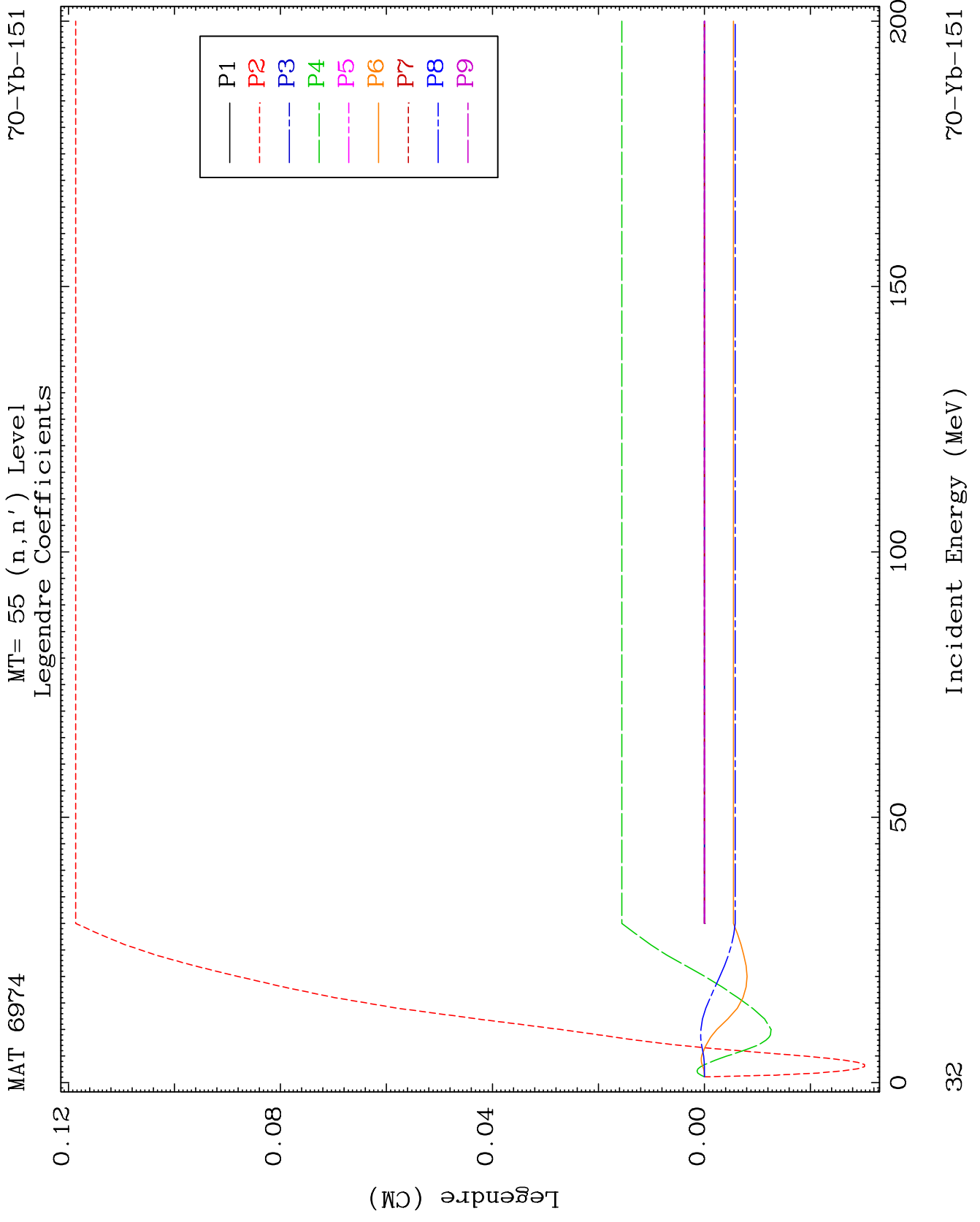
29

Incident Energy (MeV)

70-Yb-151



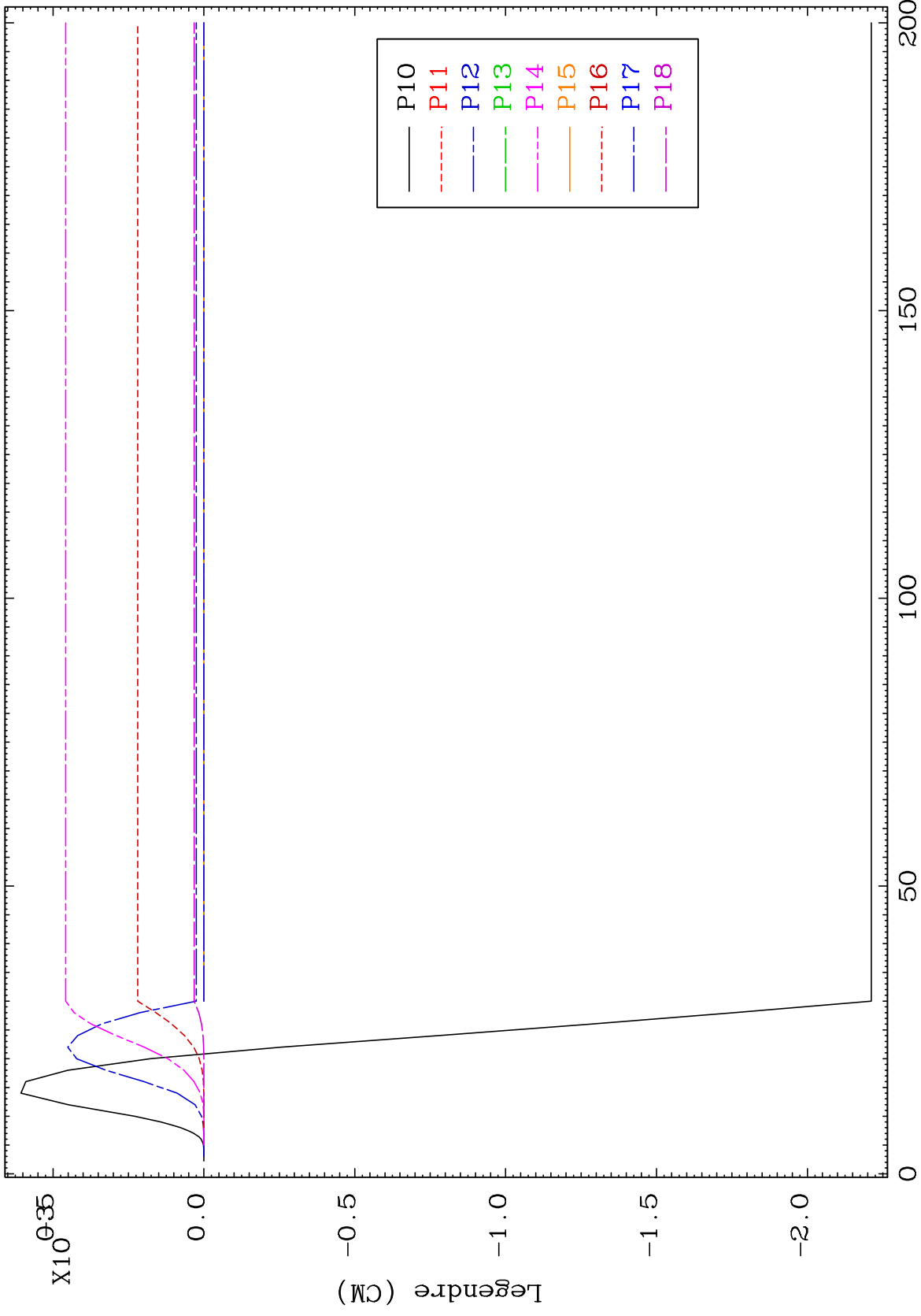




MAT 6974

MT= 55 (n,n') Level
Legendre Coefficients

70-Yb-151



33

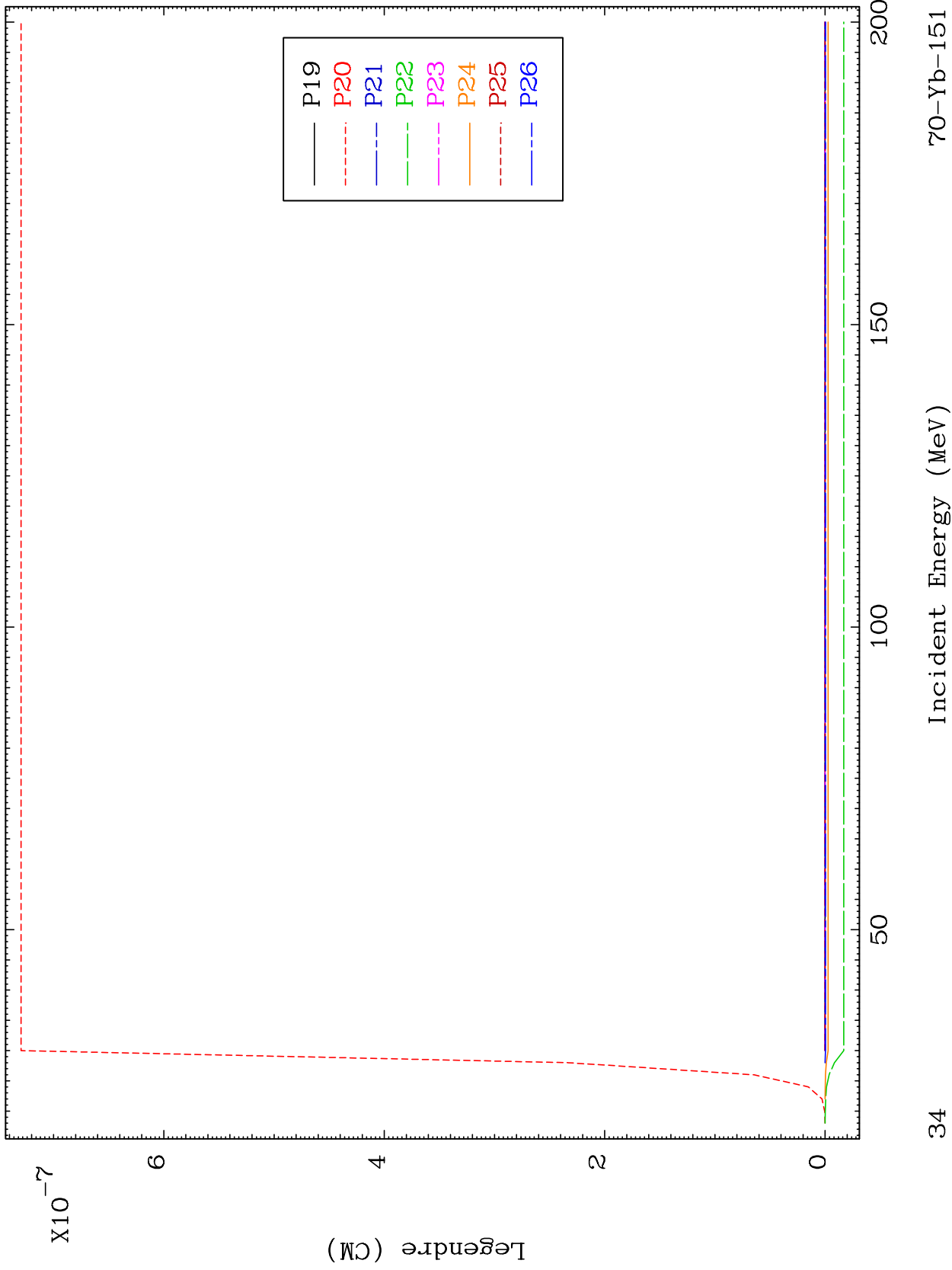
Incident Energy (MeV)

70-Yb-151

MAT 6974

MT= 55 (n,n') Level
Legendre Coefficients

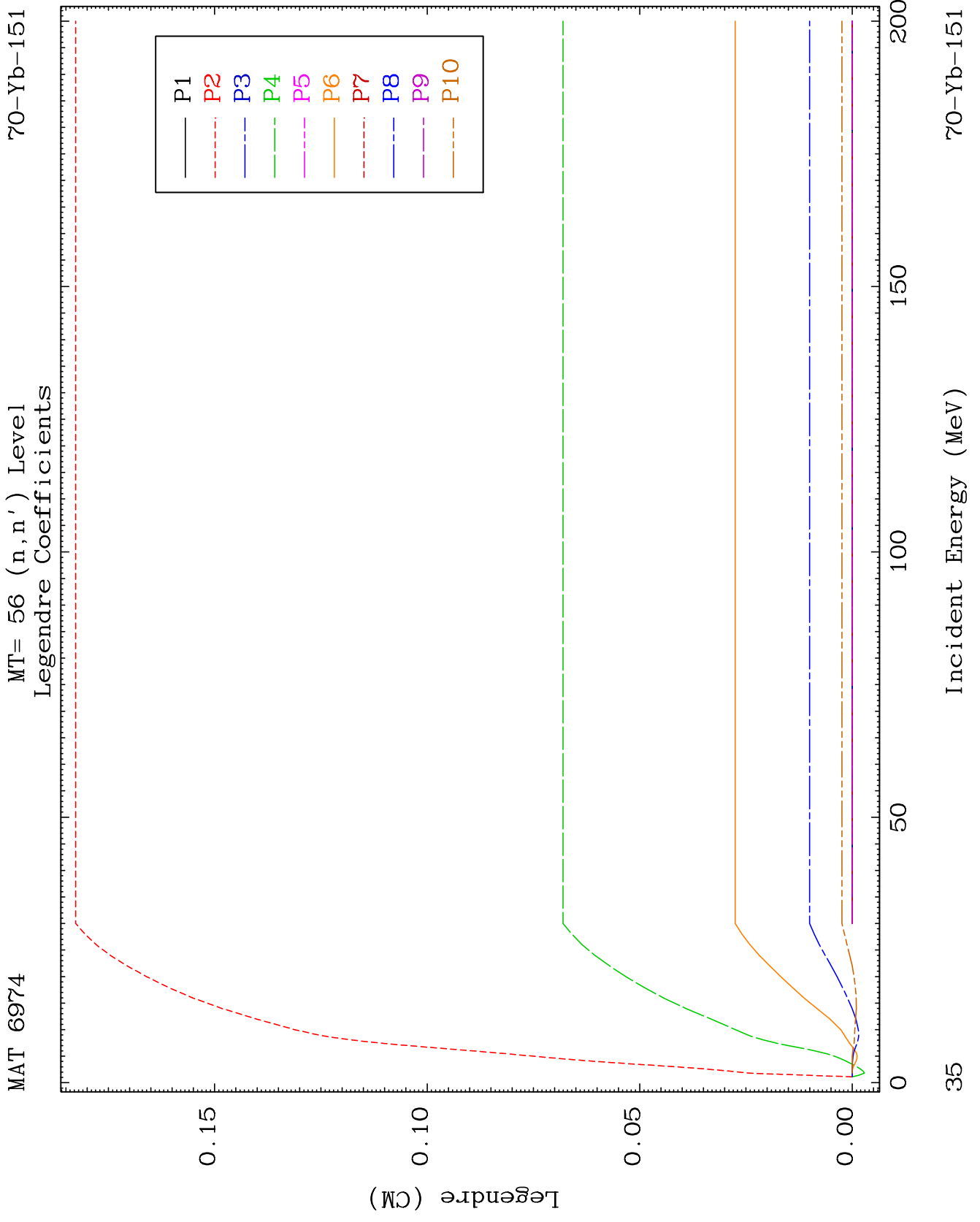
70-Yb-151



34

Incident Energy (MeV)

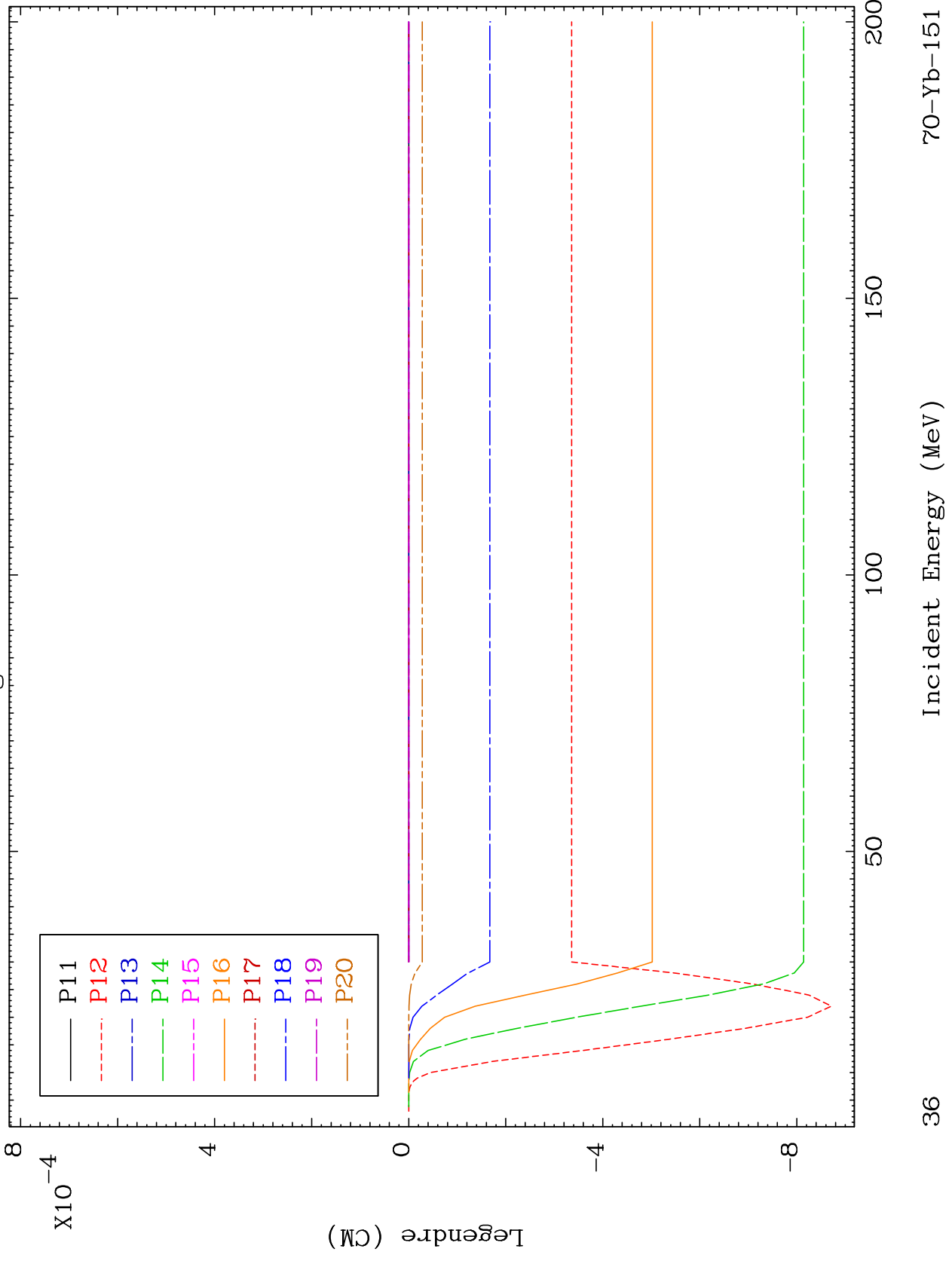
70-Yb-151

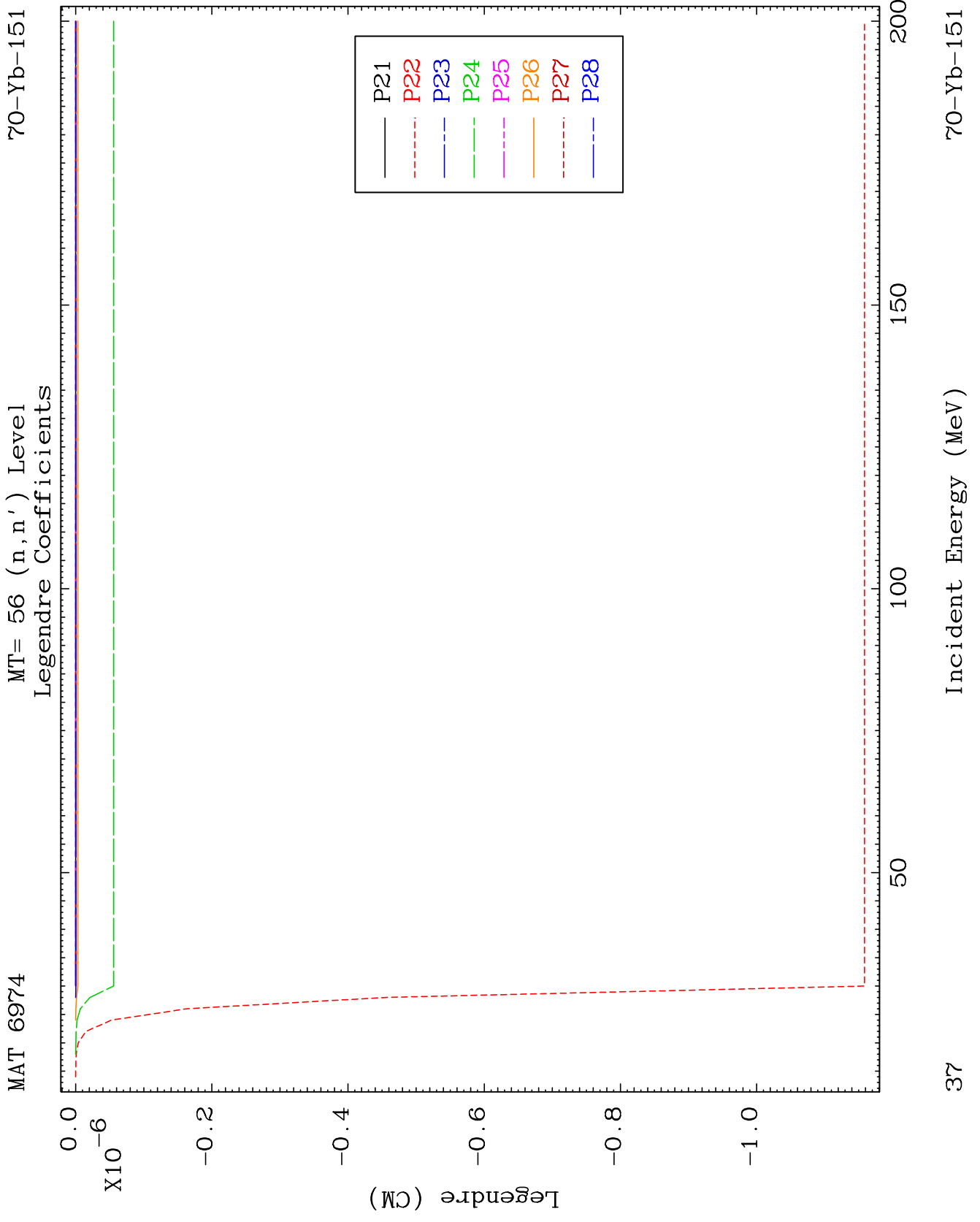


MAT 6974

MT= 56 (n,n') Level
Legendre Coefficients

70-Yb-151

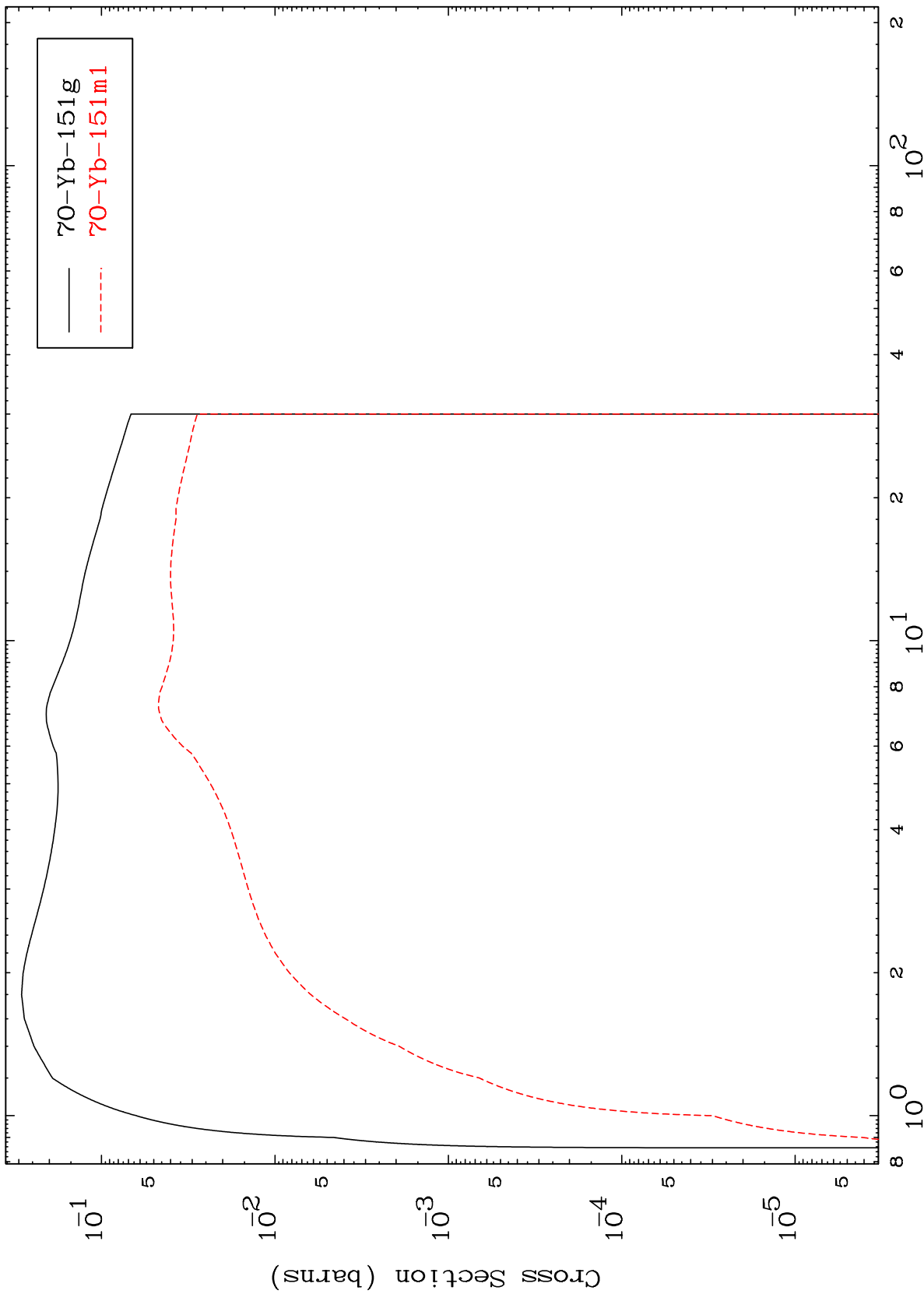




MAT 6974

70-Yb-151

Inelastic
Radionuclide Production Cross Section



Incident Energy (MeV)

70-Yb-151

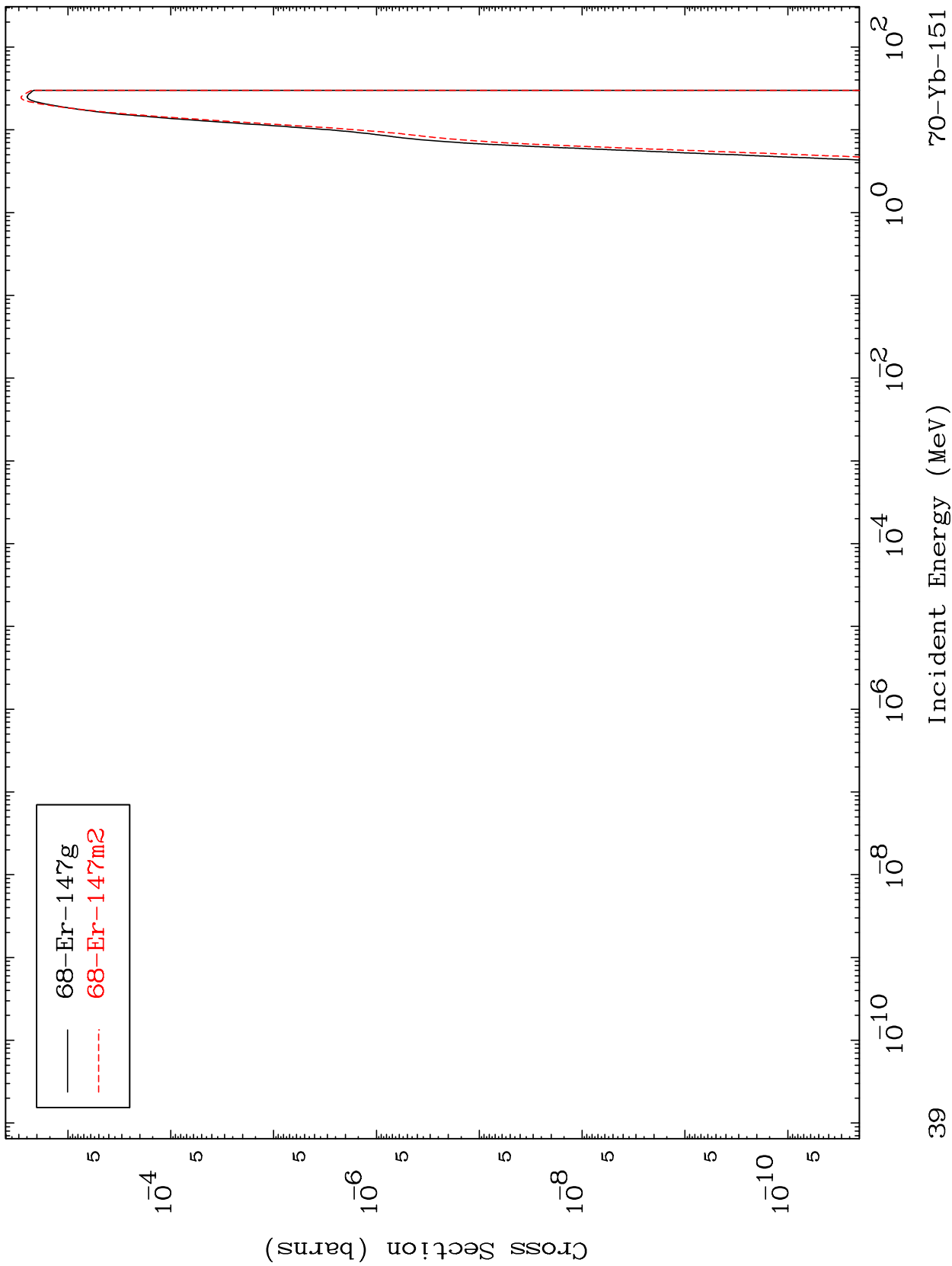
38

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$(n, n') \alpha$

70-Yb-151

Radionuclide Production Cross Section

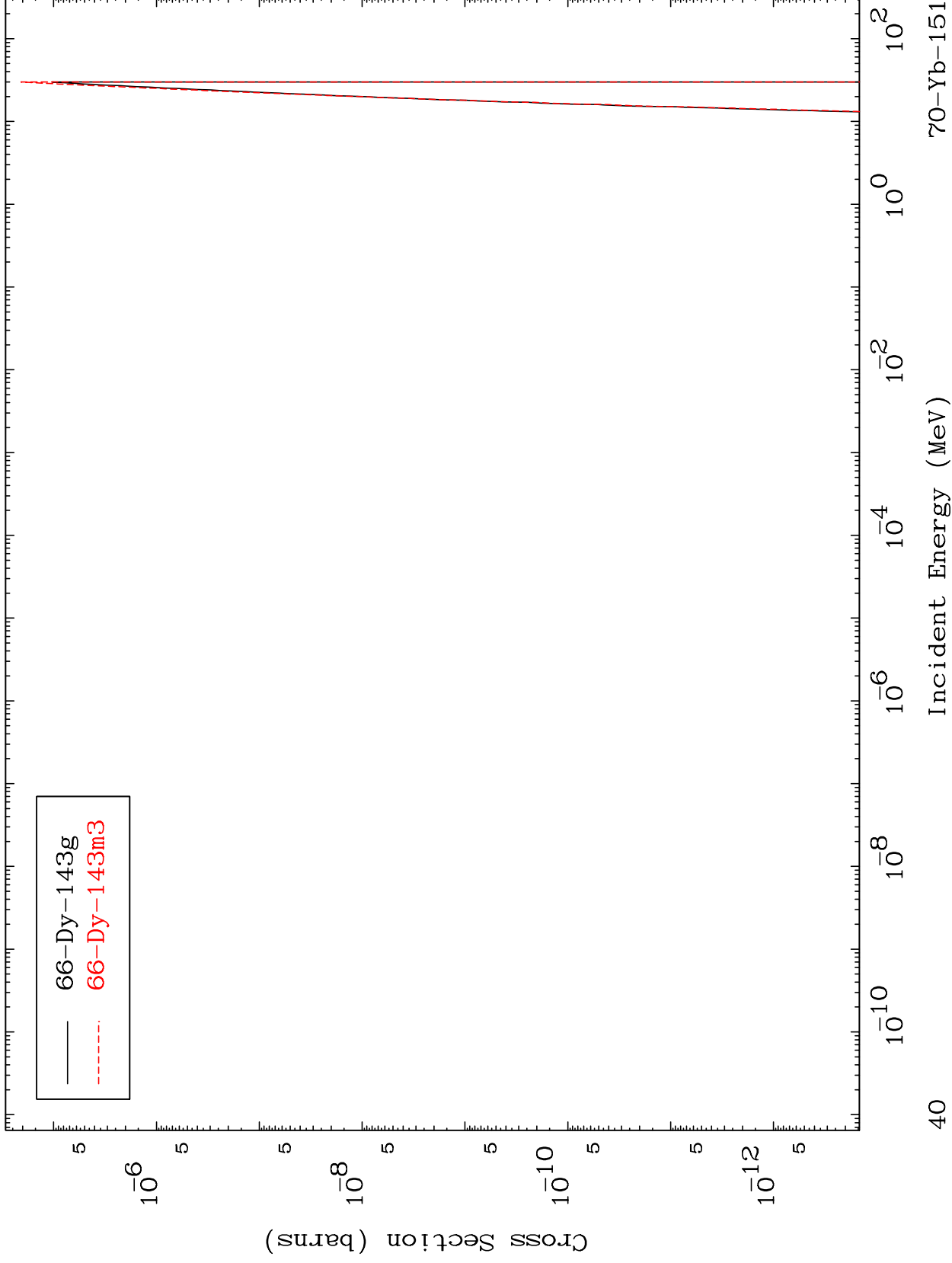


MAT 6974

(n,n') 2α

70-Yb-151

Radionuclide Production Cross Section



40

Incident Energy (MeV)

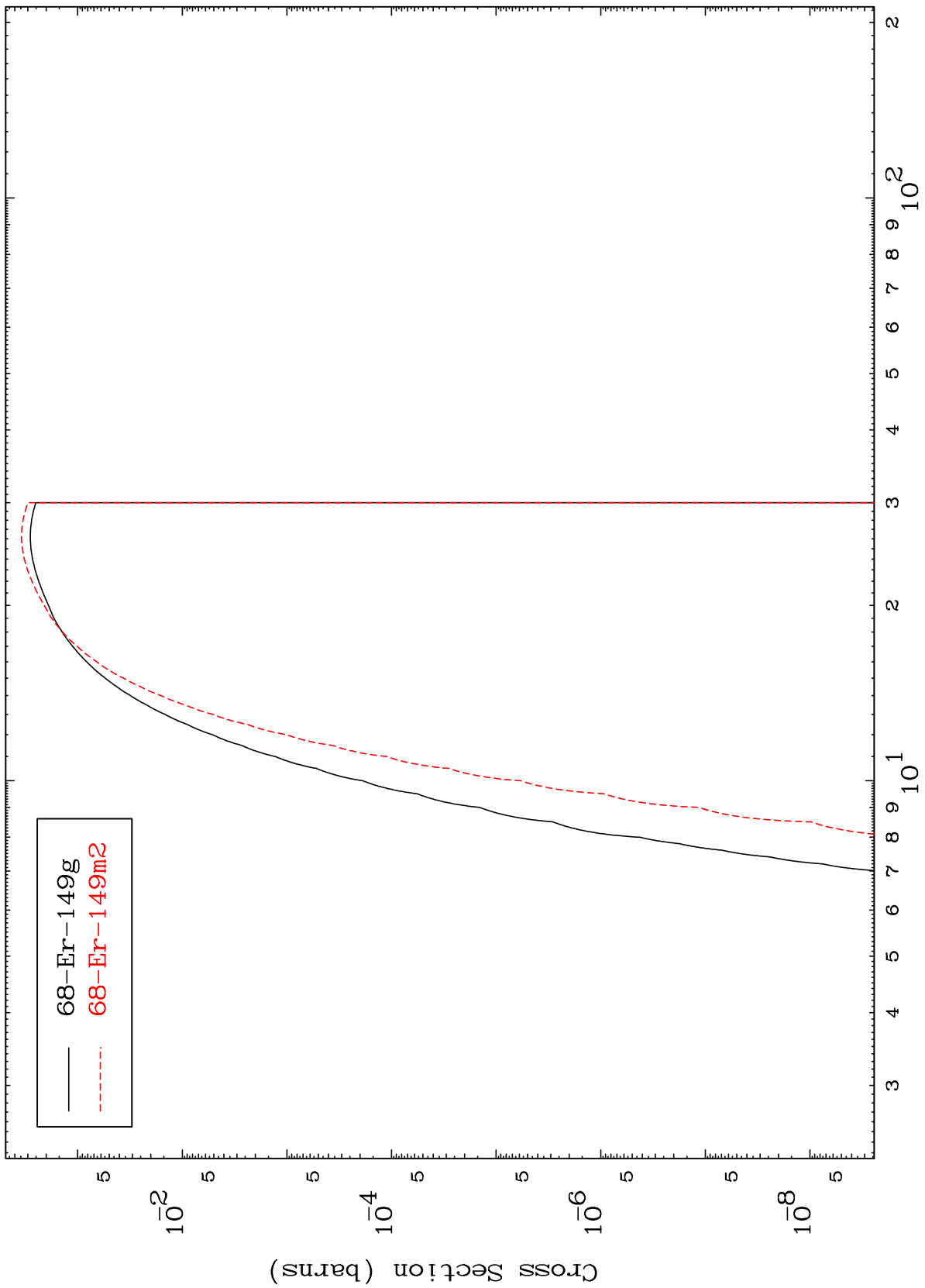
70-Yb-151

MAT 6974

(n,2n) p

70-Yb-151

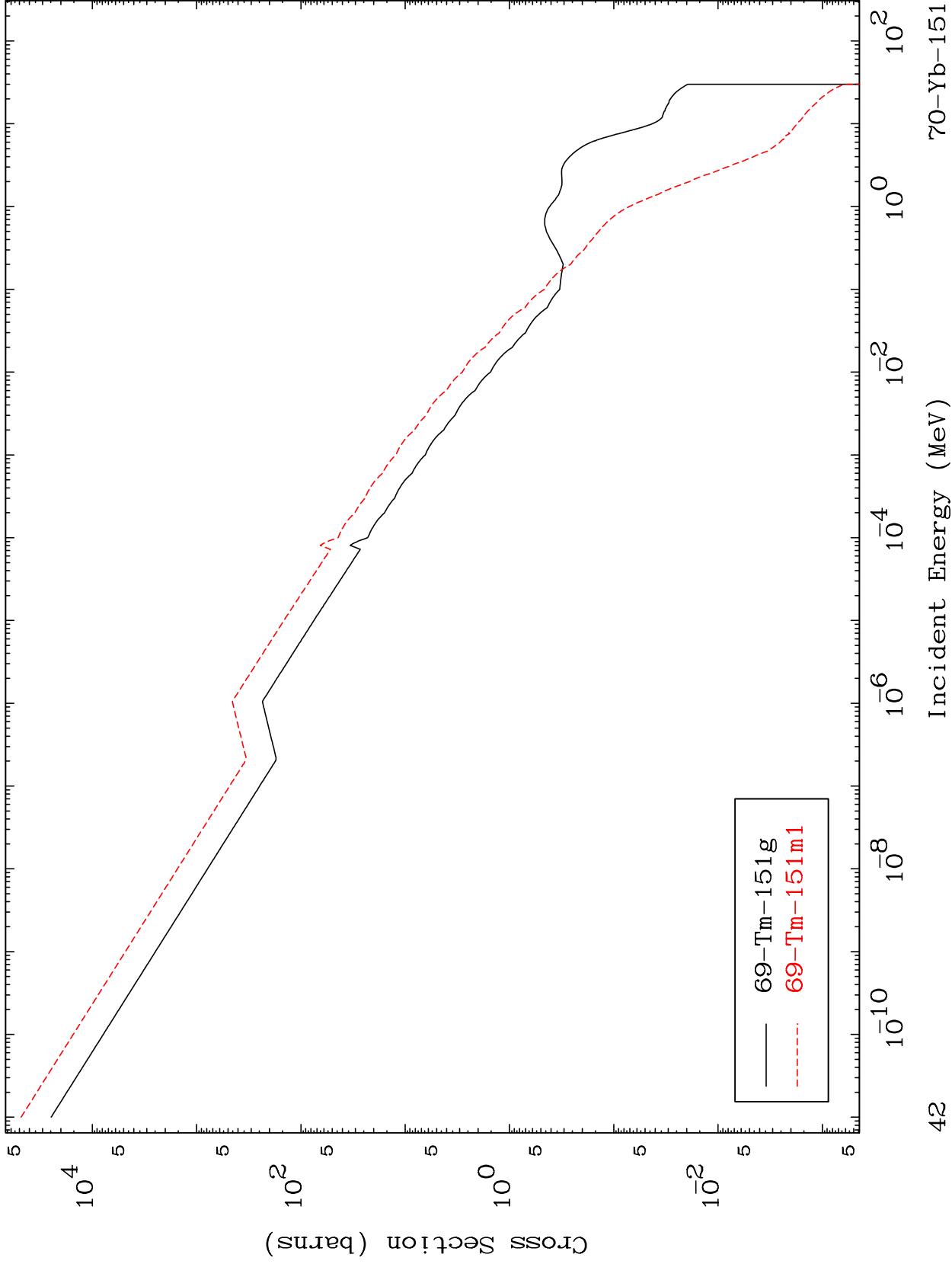
Radionuclide Production Cross Section



MAT 6974

70-Yb-151

(n,p)
Radionuclide Production Cross Section

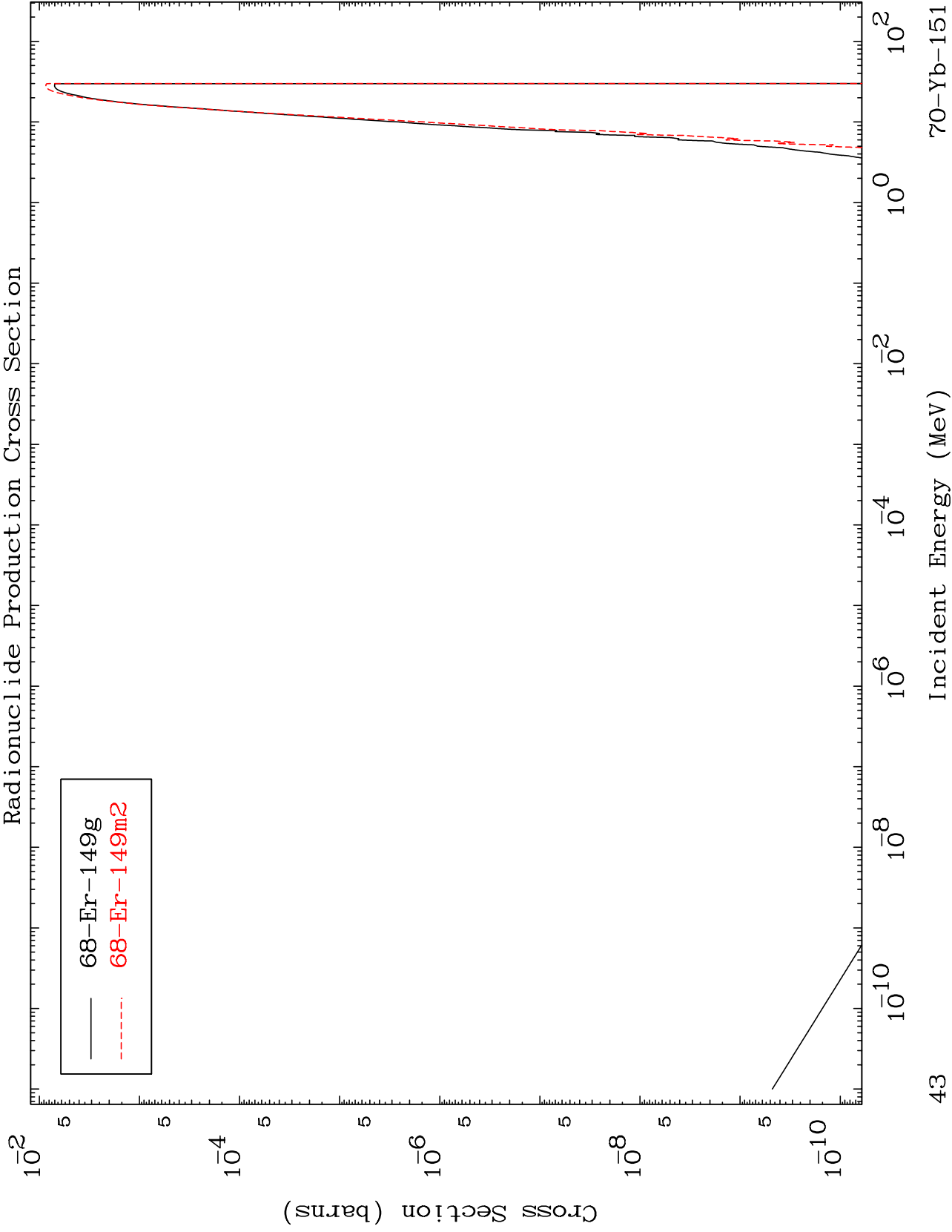


MAT 6974

(n,He-3)

70-Yb-151

Radionuclide Production Cross Section

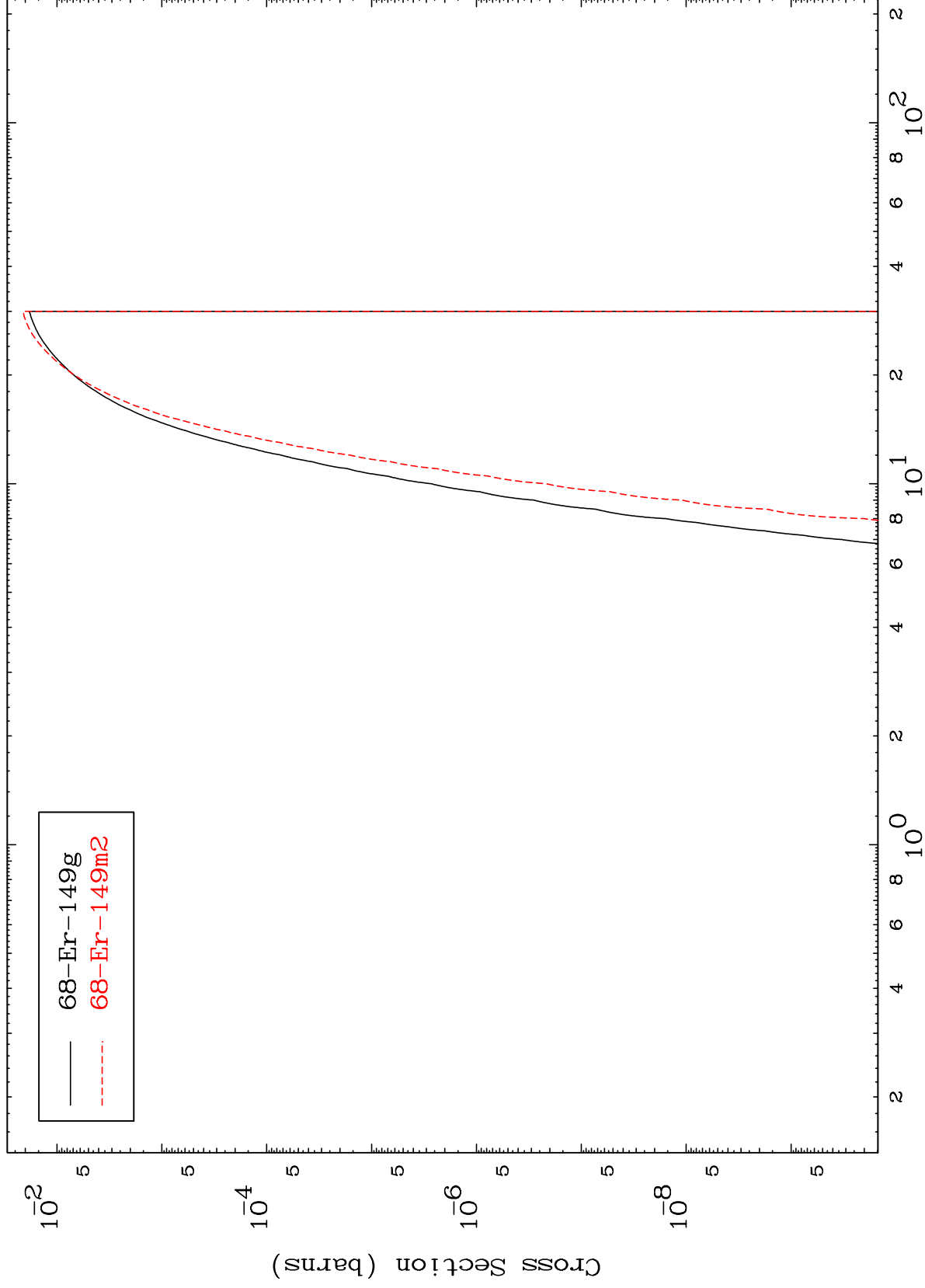


MAT 6974

(n,p) d

70-Yb-151

Radionuclide Production Cross Section



44

Incident Energy (MeV)

70-Yb-151