

Program EVALPLOT  
(Version 2017-1)

by

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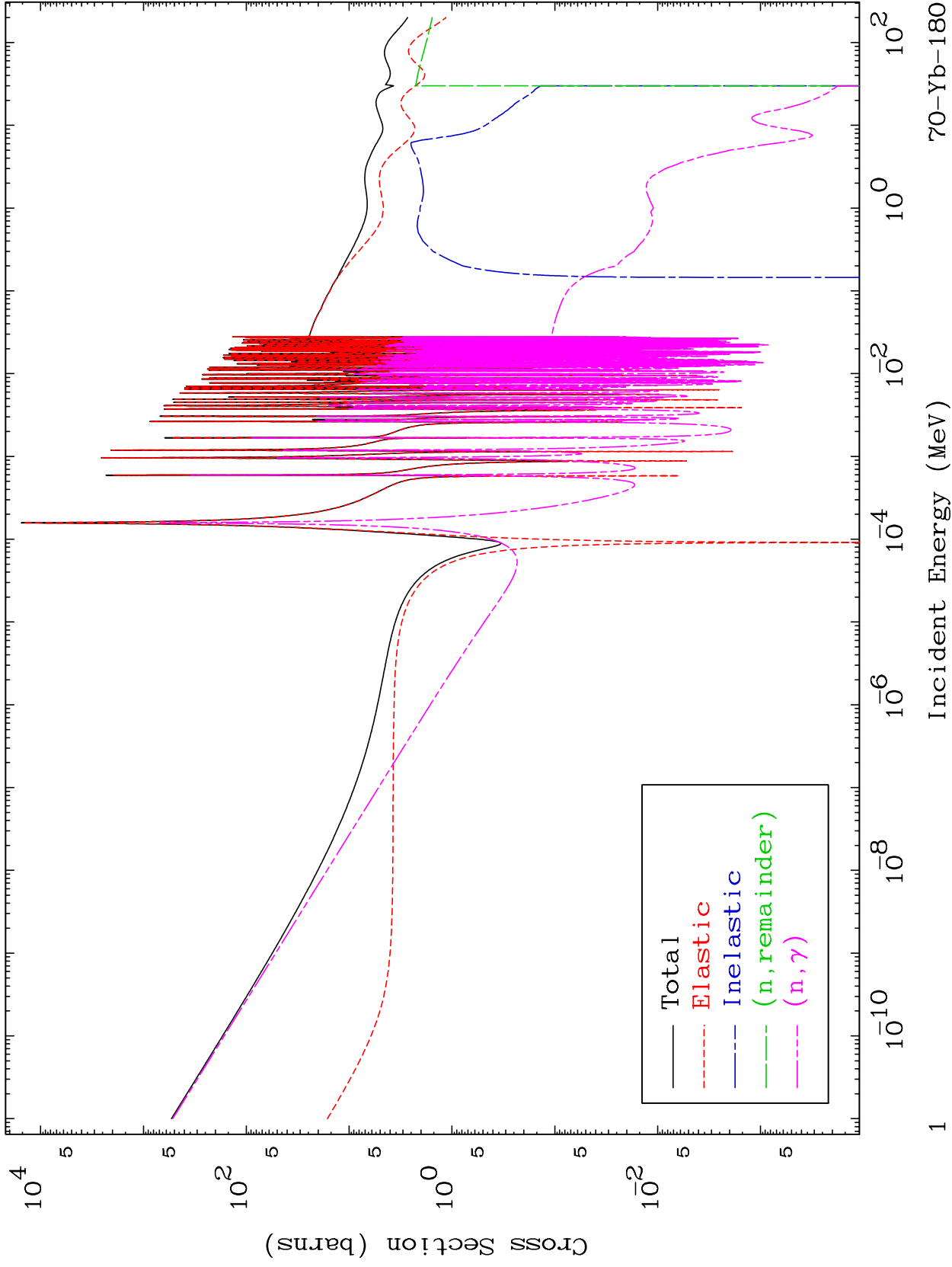
Press Mouse Button to Start

MAT 7061

Major

293 Kelvin Cross Sections

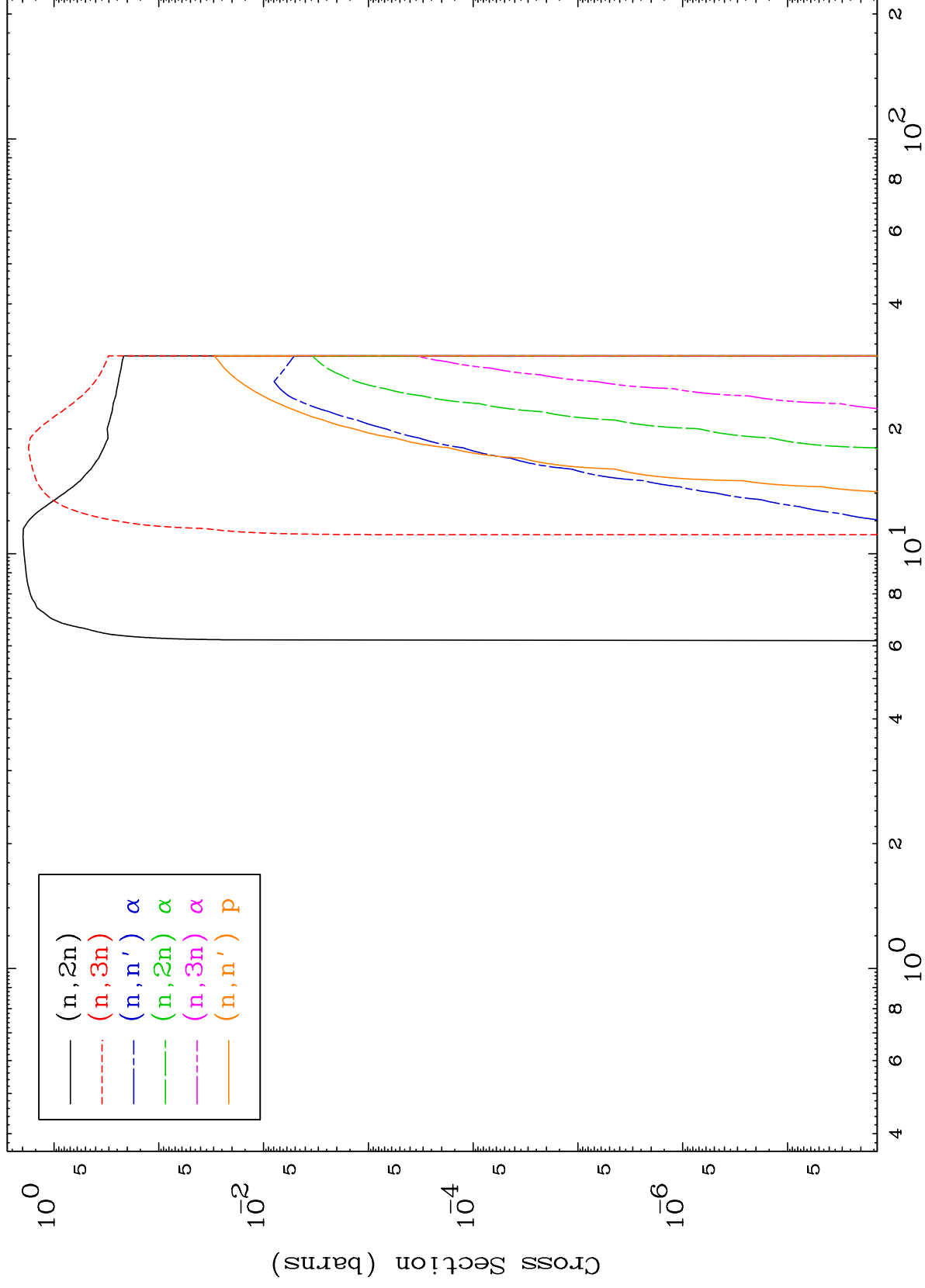
70-Yb-180



MAT 7061

Neutron Production  
293 Kelvin Cross Sections

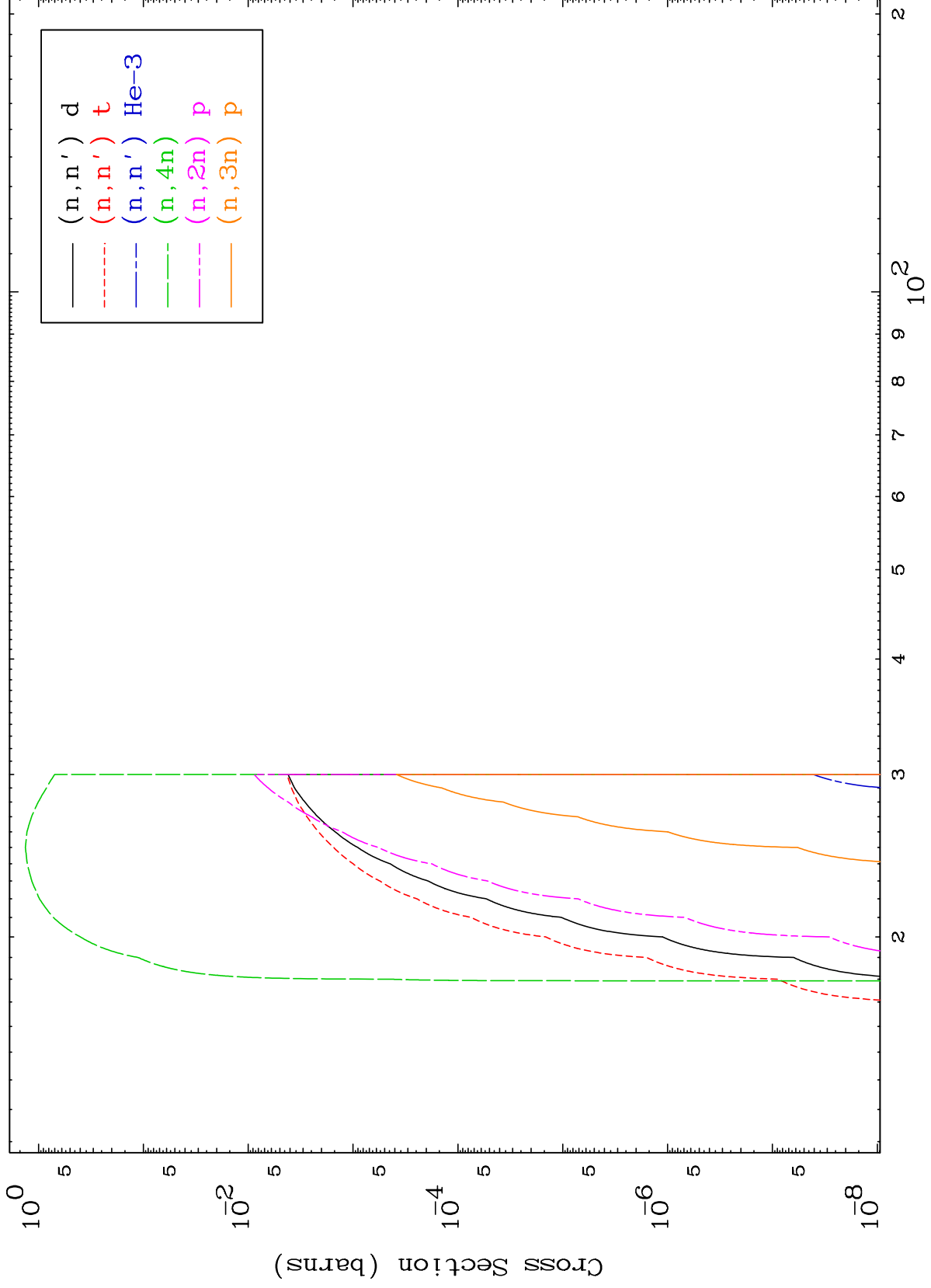
70-Yb-180



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Incident Energy (MeV)

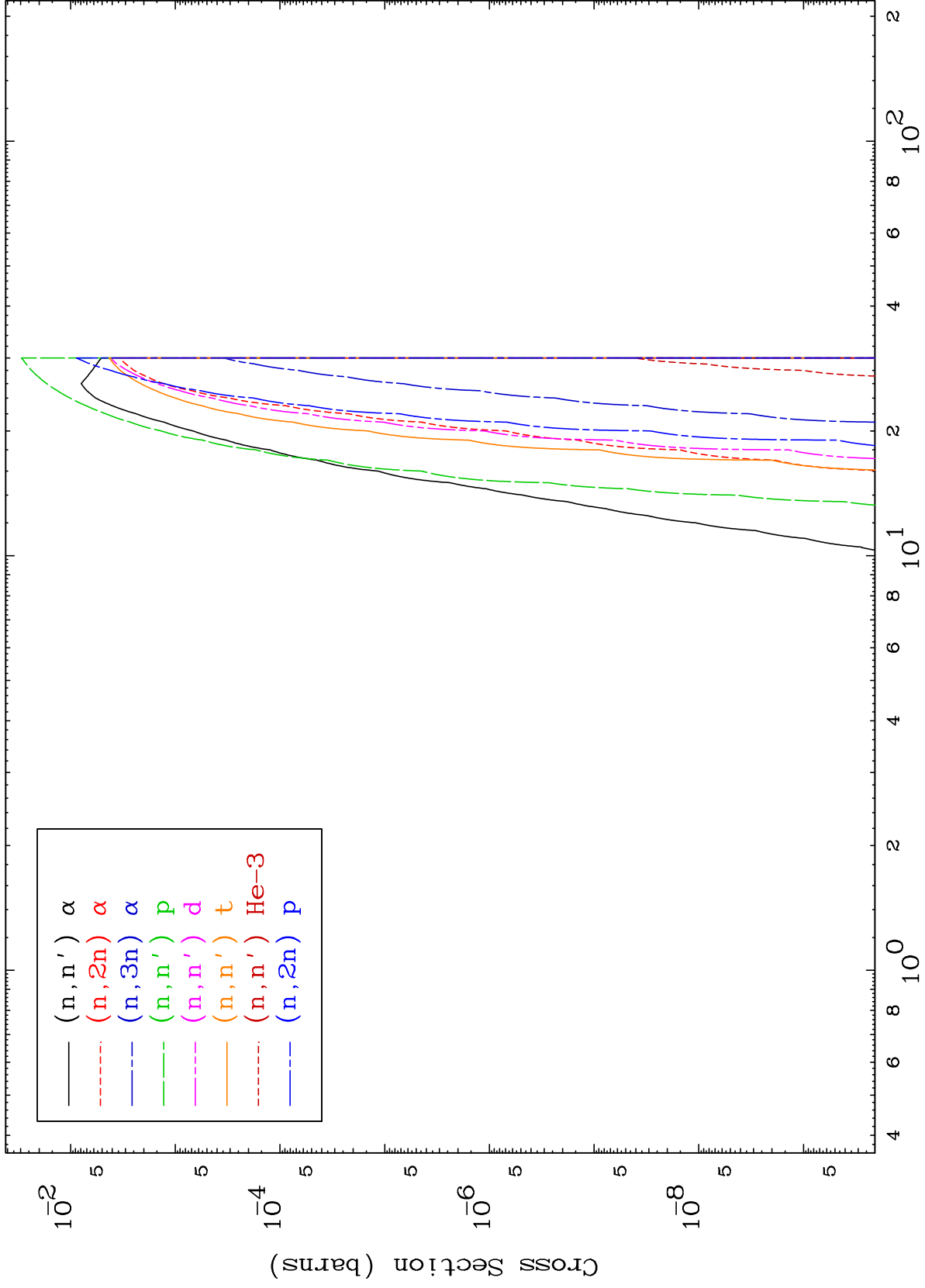
70-Yb-180



MAT 7061

Charged Particle  
293 Kelvin Cross Sections

70-Yb-180



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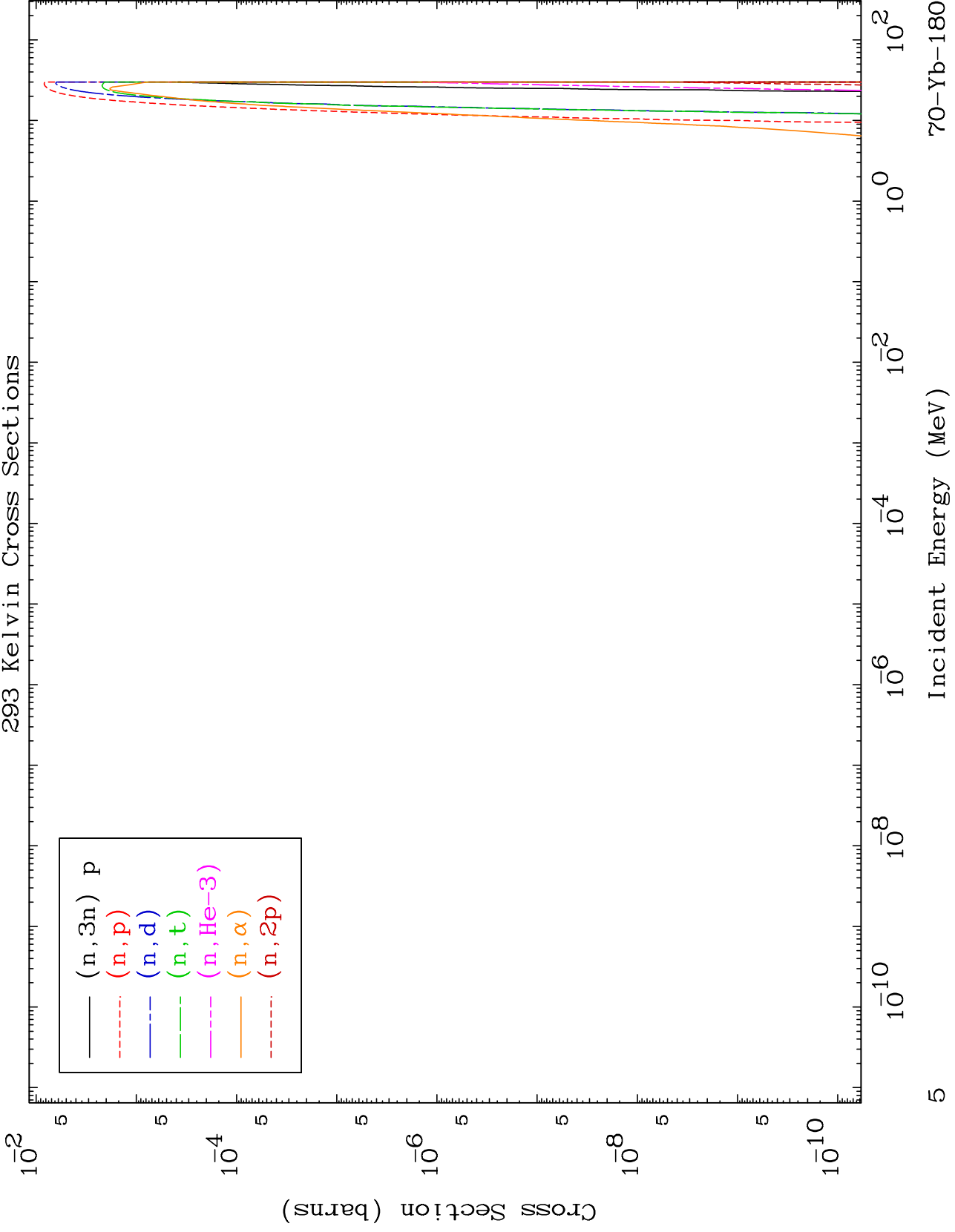
Incident Energy (MeV)

70-Yb-180

MAT 7061

Charged Particle  
293 Kelvin Cross Sections

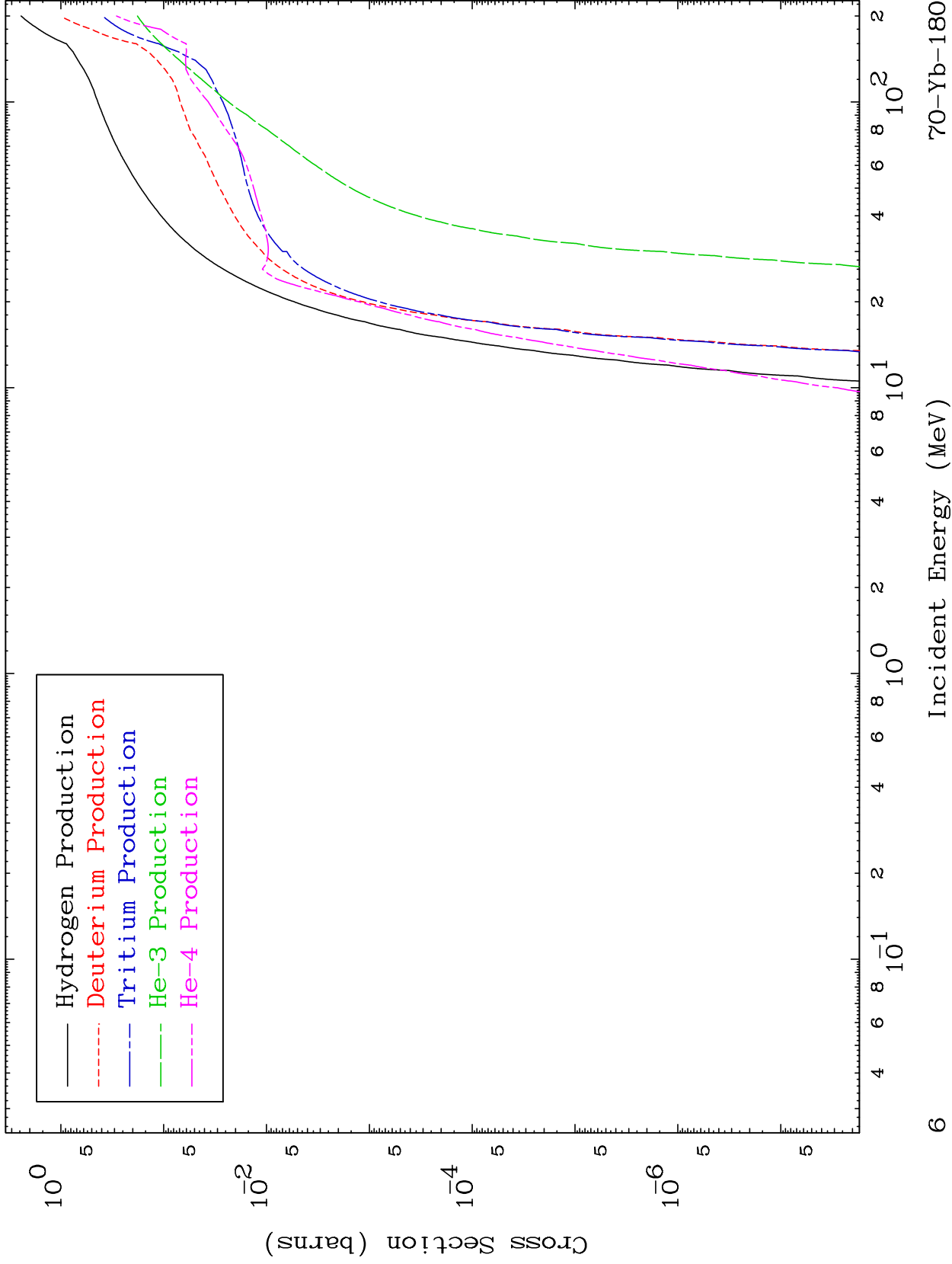
70-Yb-180



MAT 7061

Particle Production  
293 Kelvin Cross Sections

70-Yb-180

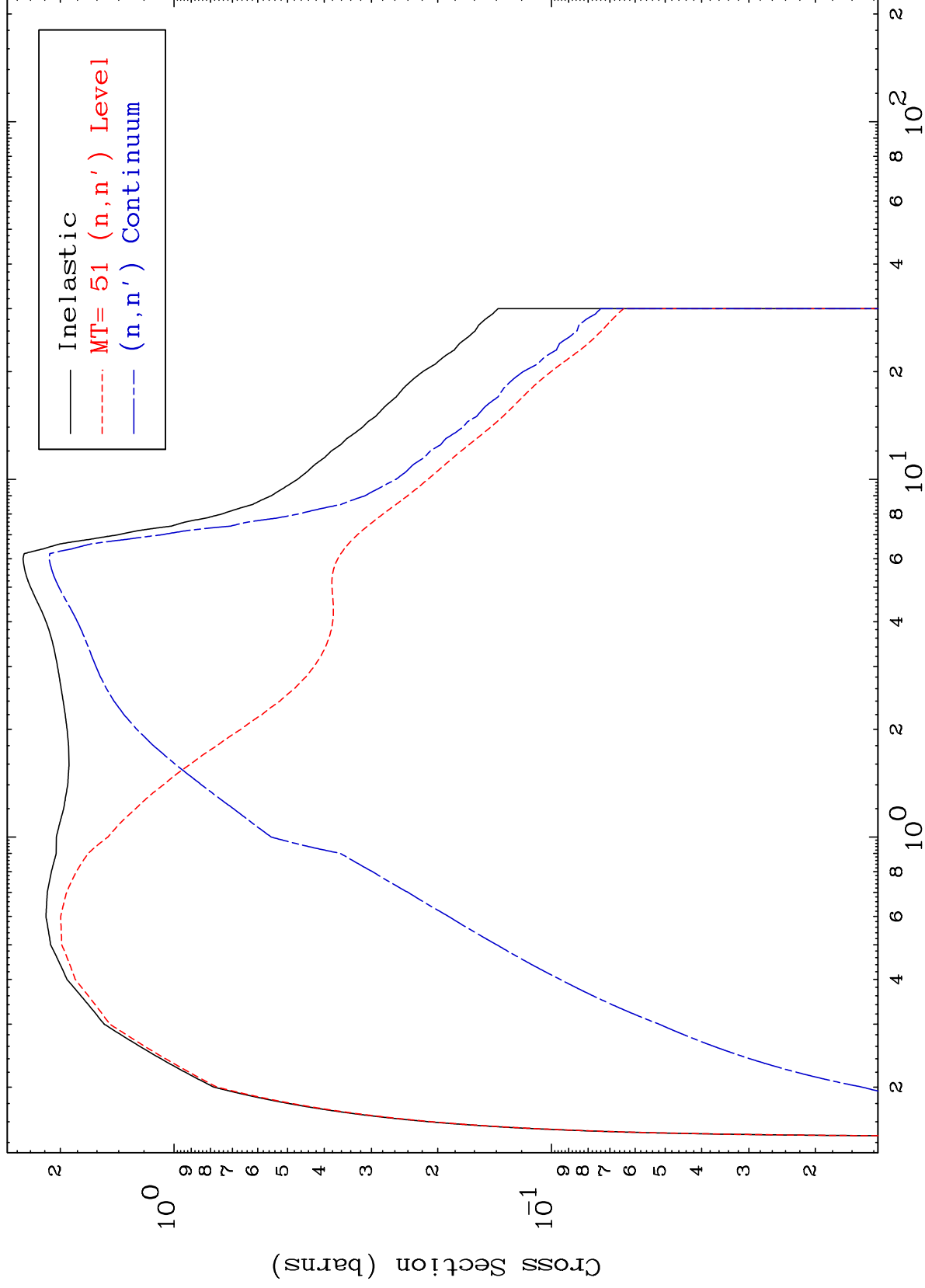


MAT 7061

(n,n') Level

70-Yb-180

293 Kelvin Cross Sections

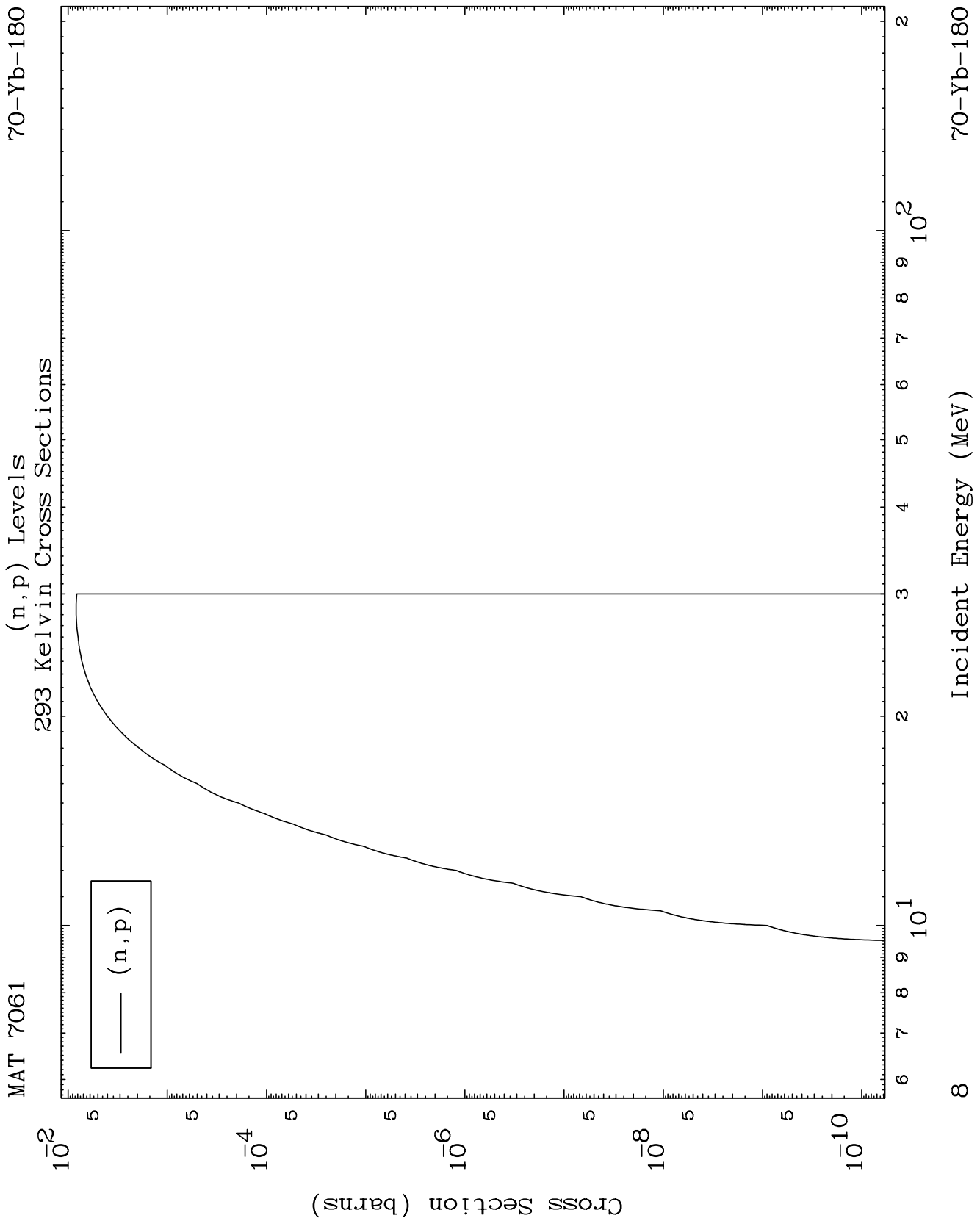


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Incident Energy (MeV)

70-Yb-180

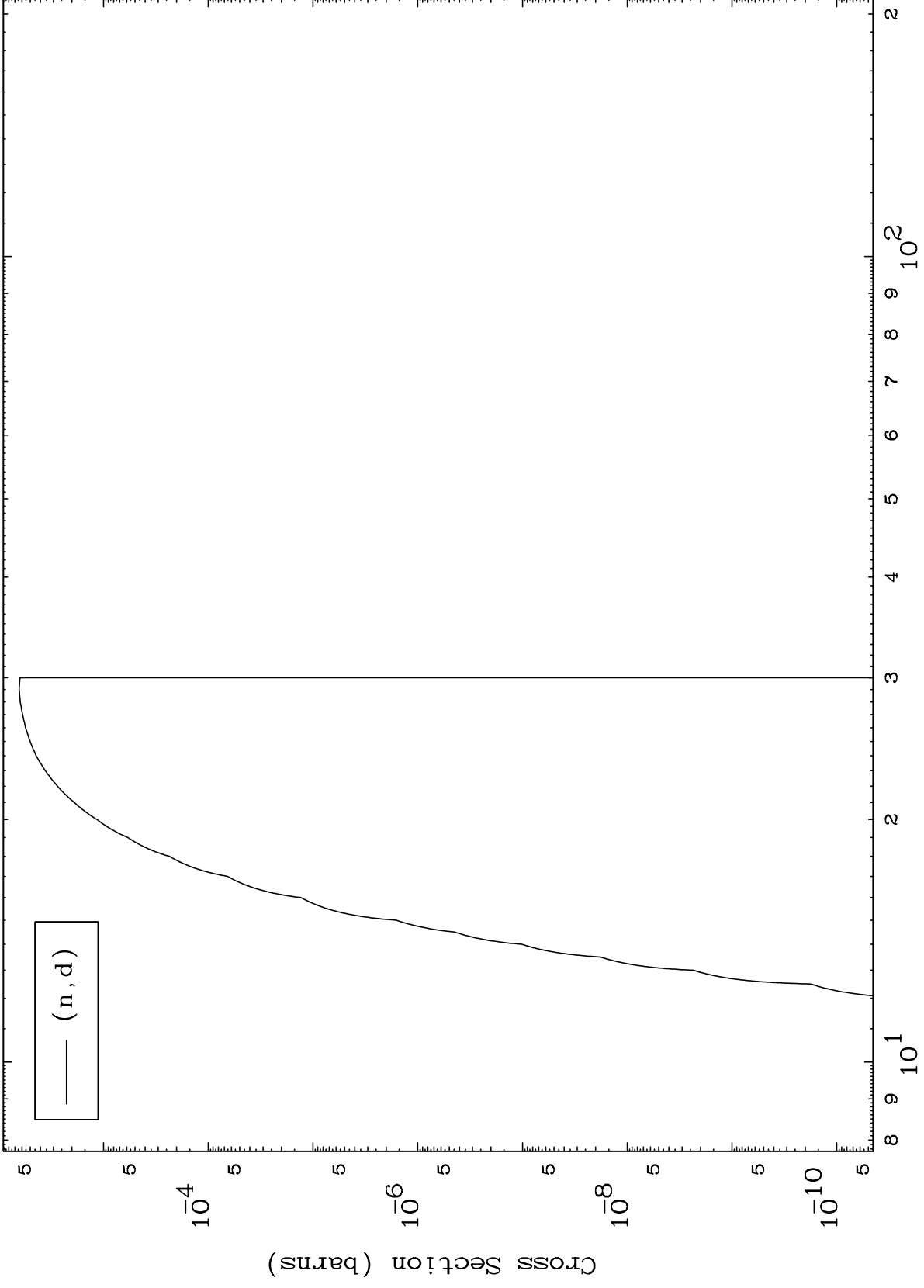




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(n,d) Levels  
293 Kelvin Cross Sections

70-Yb-180



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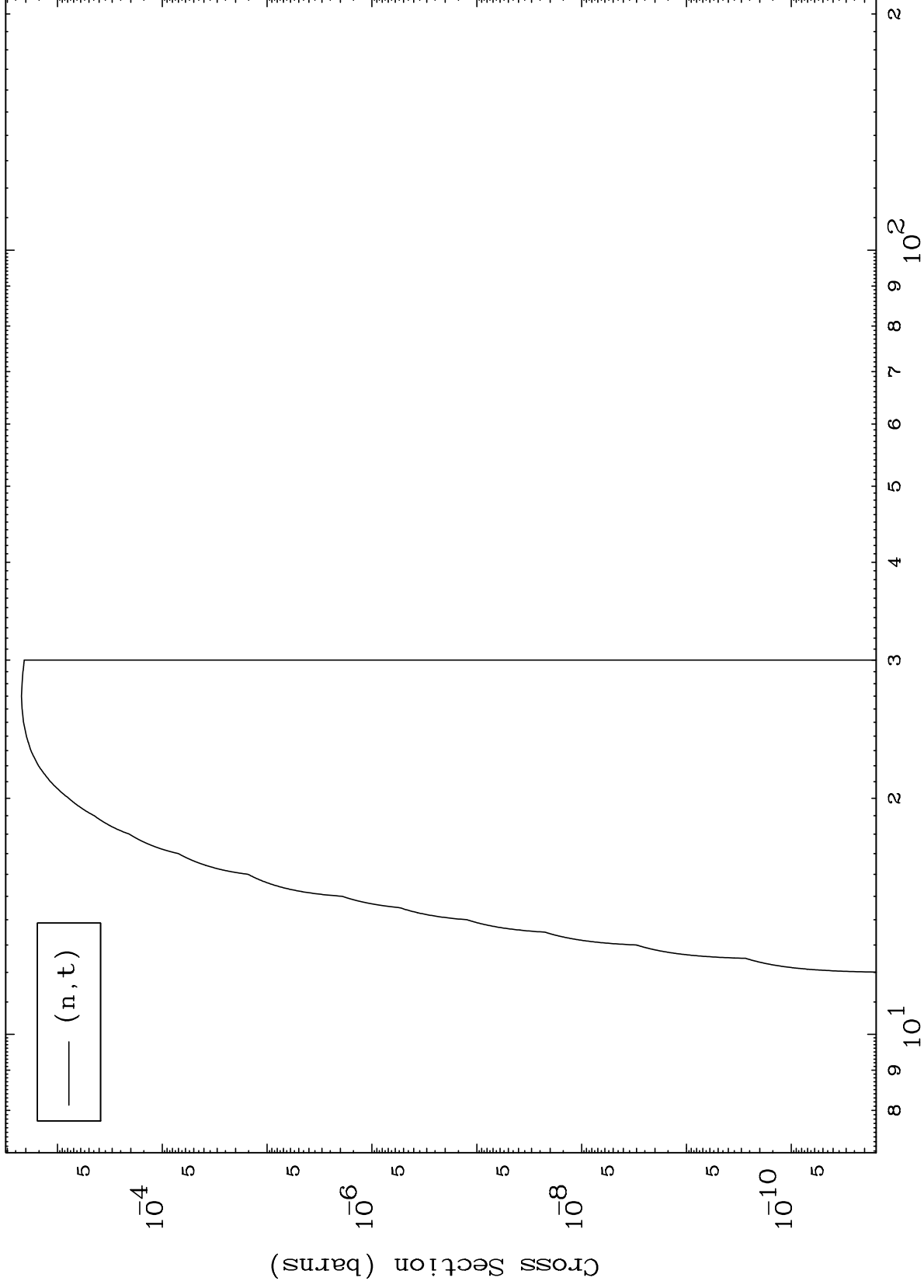
Incident Energy (MeV)

70-Yb-180

MAT 7061

(n,t) Levels  
293 Kelvin Cross Sections

70-Yb-180



10

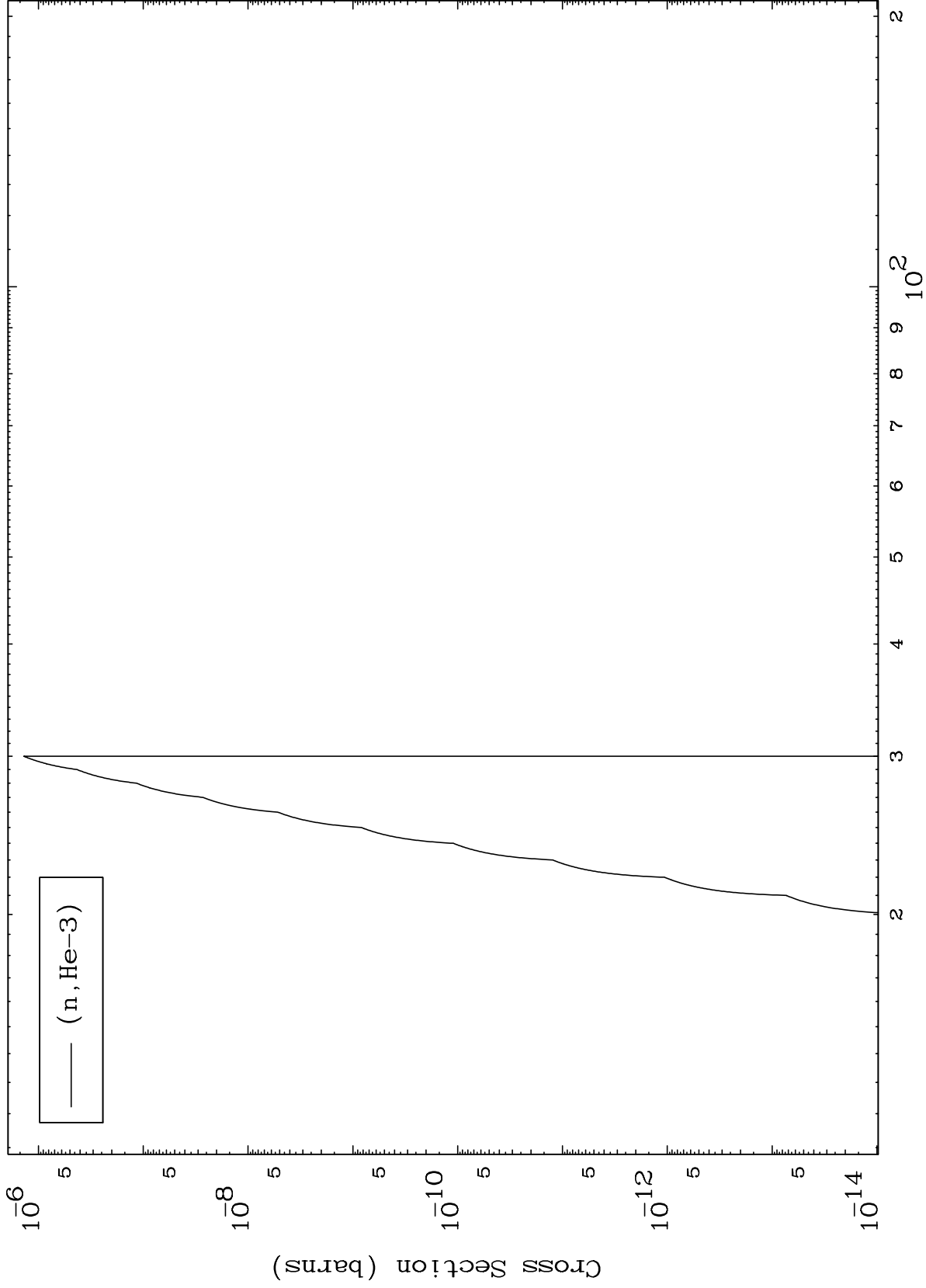
Incident Energy (MeV)

70-Yb-180

MAT 7061

(n,He3) Levels  
293 Kelvin Cross Sections

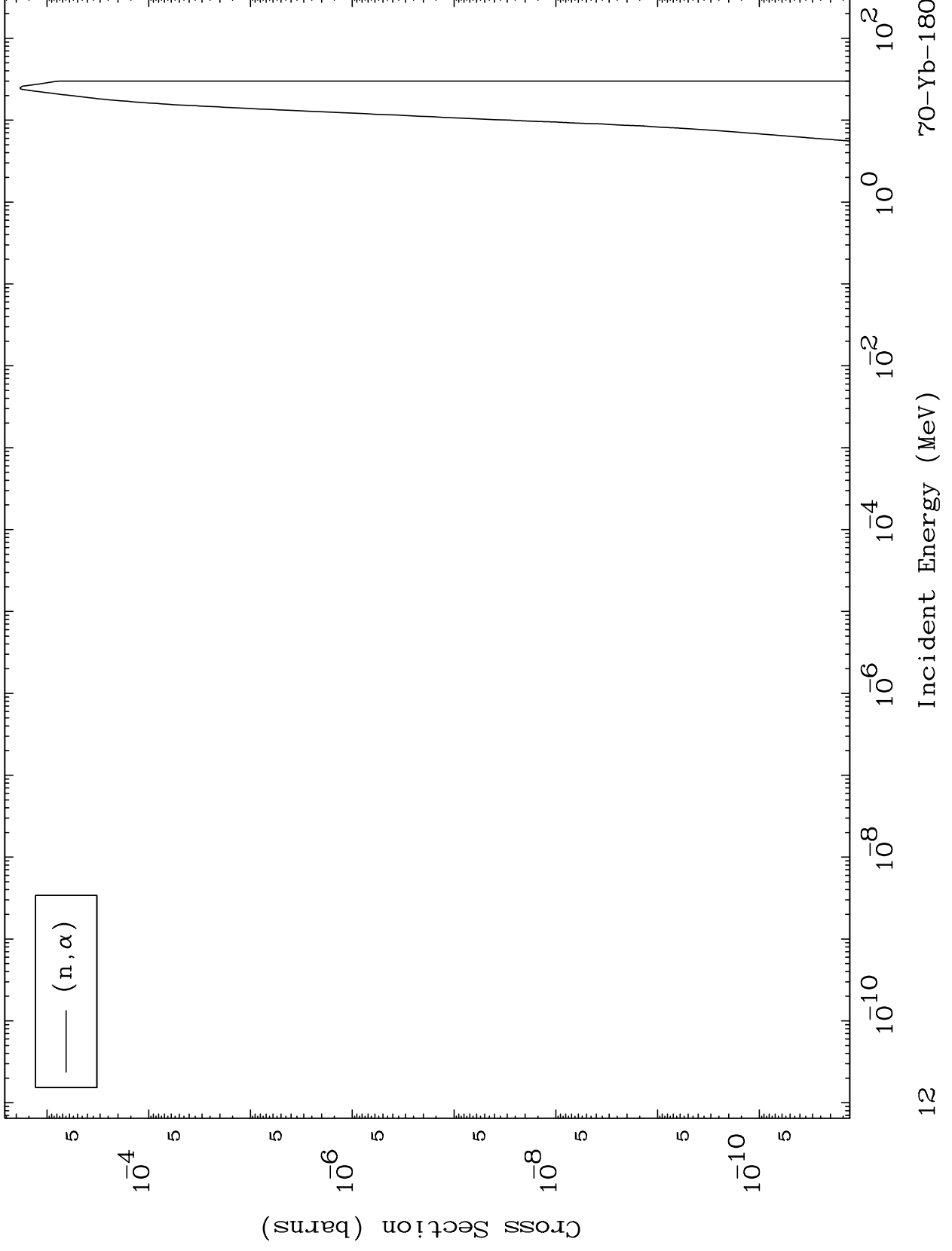
70-Yb-180

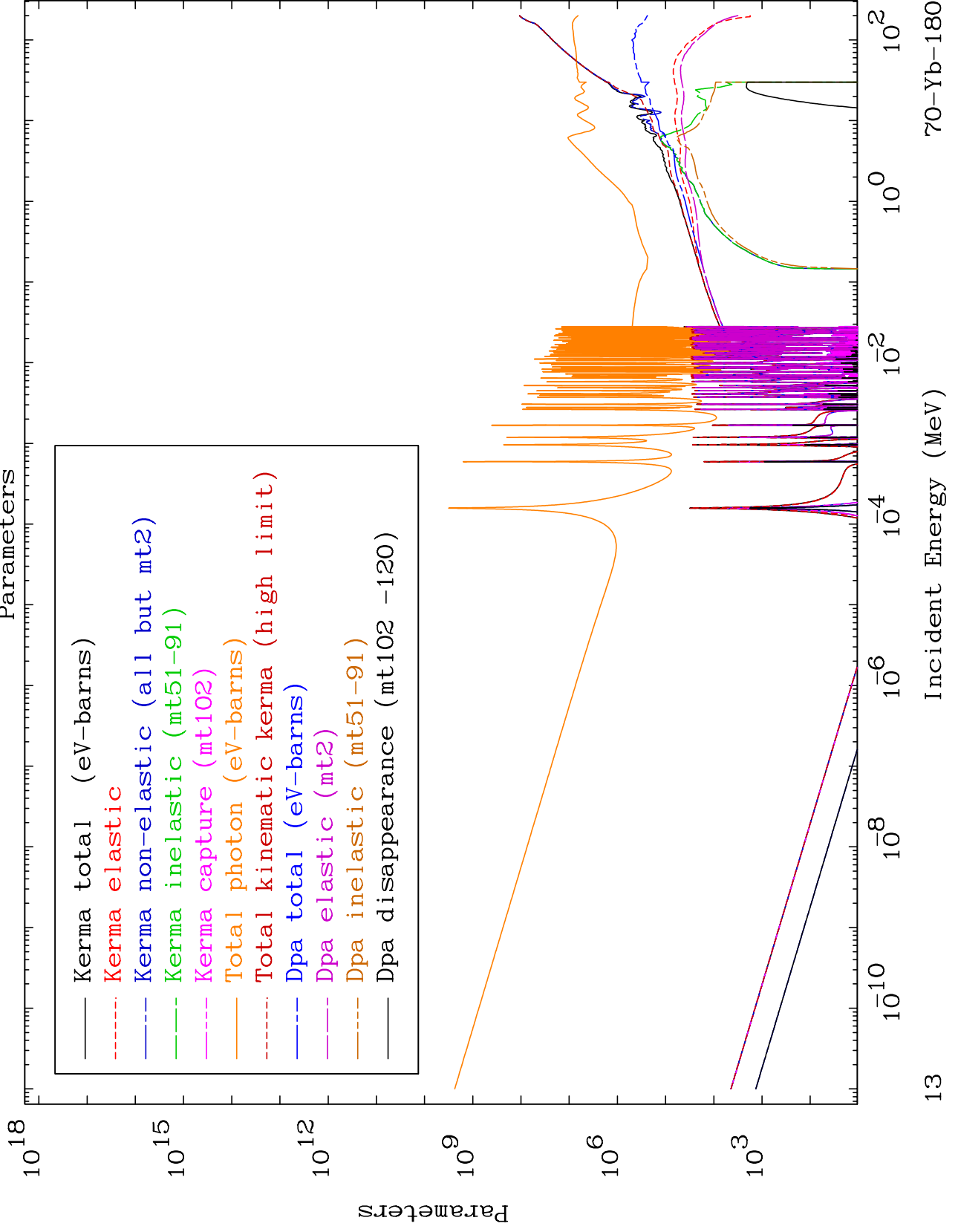


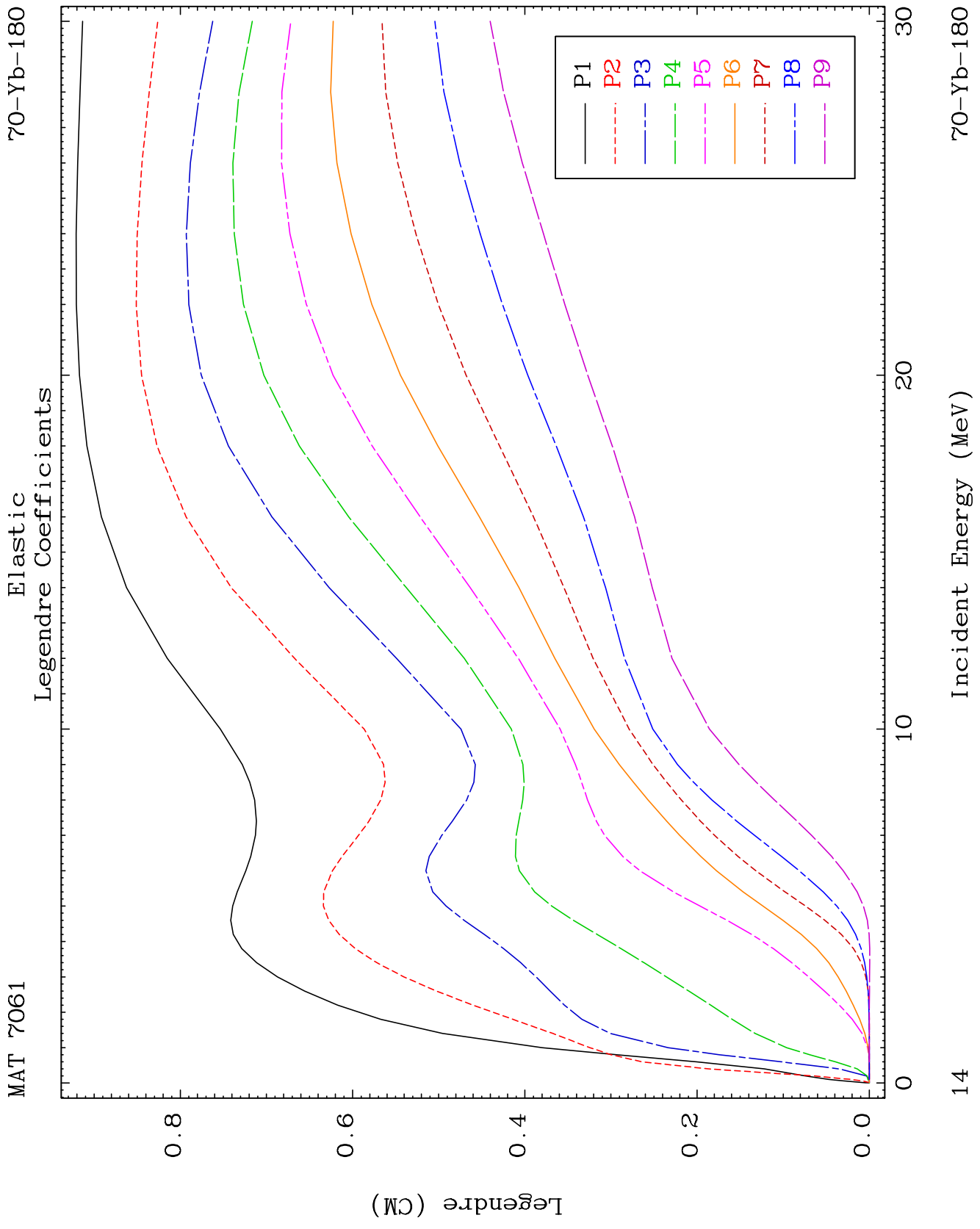
MAT 7061

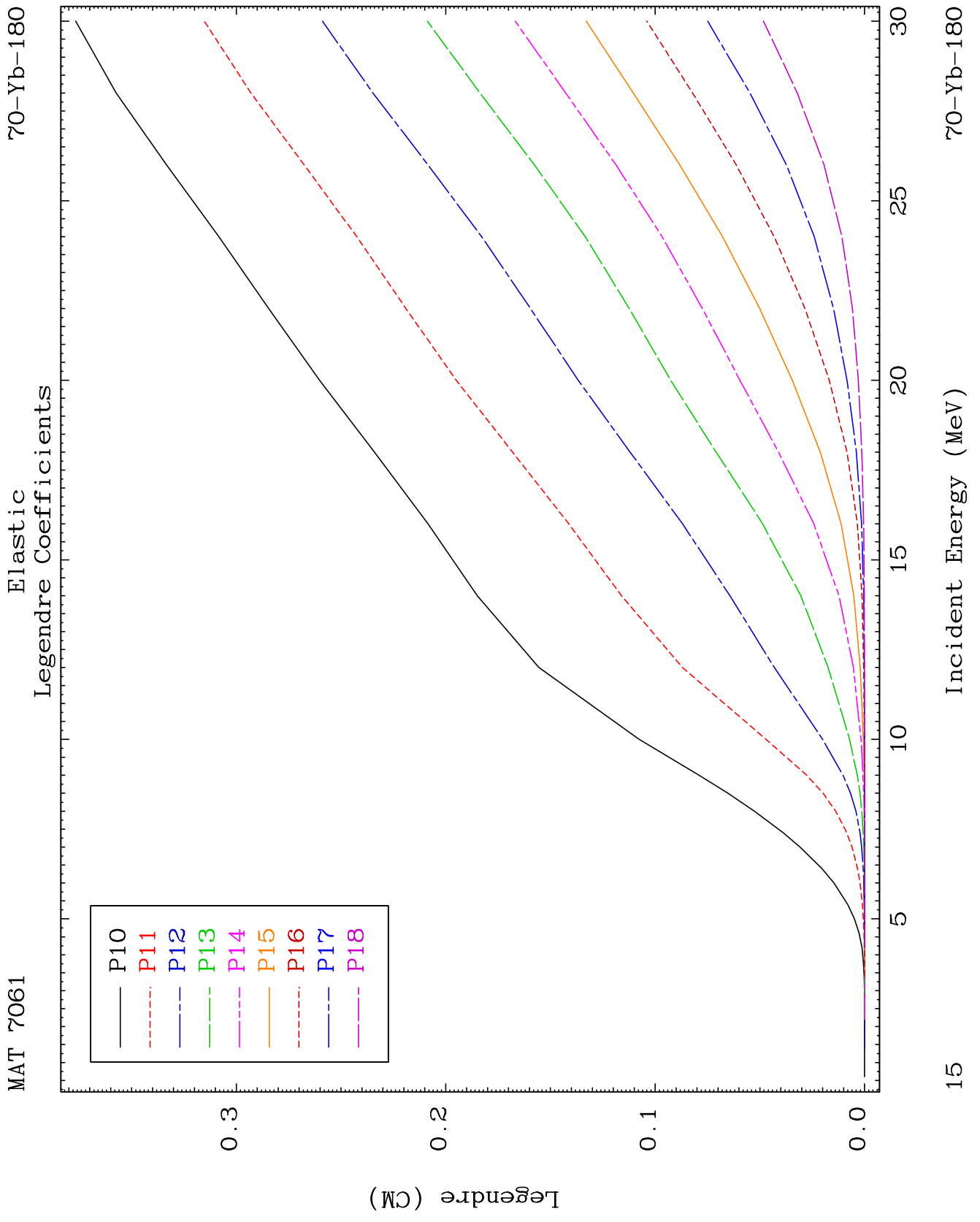
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

70-Yb-180







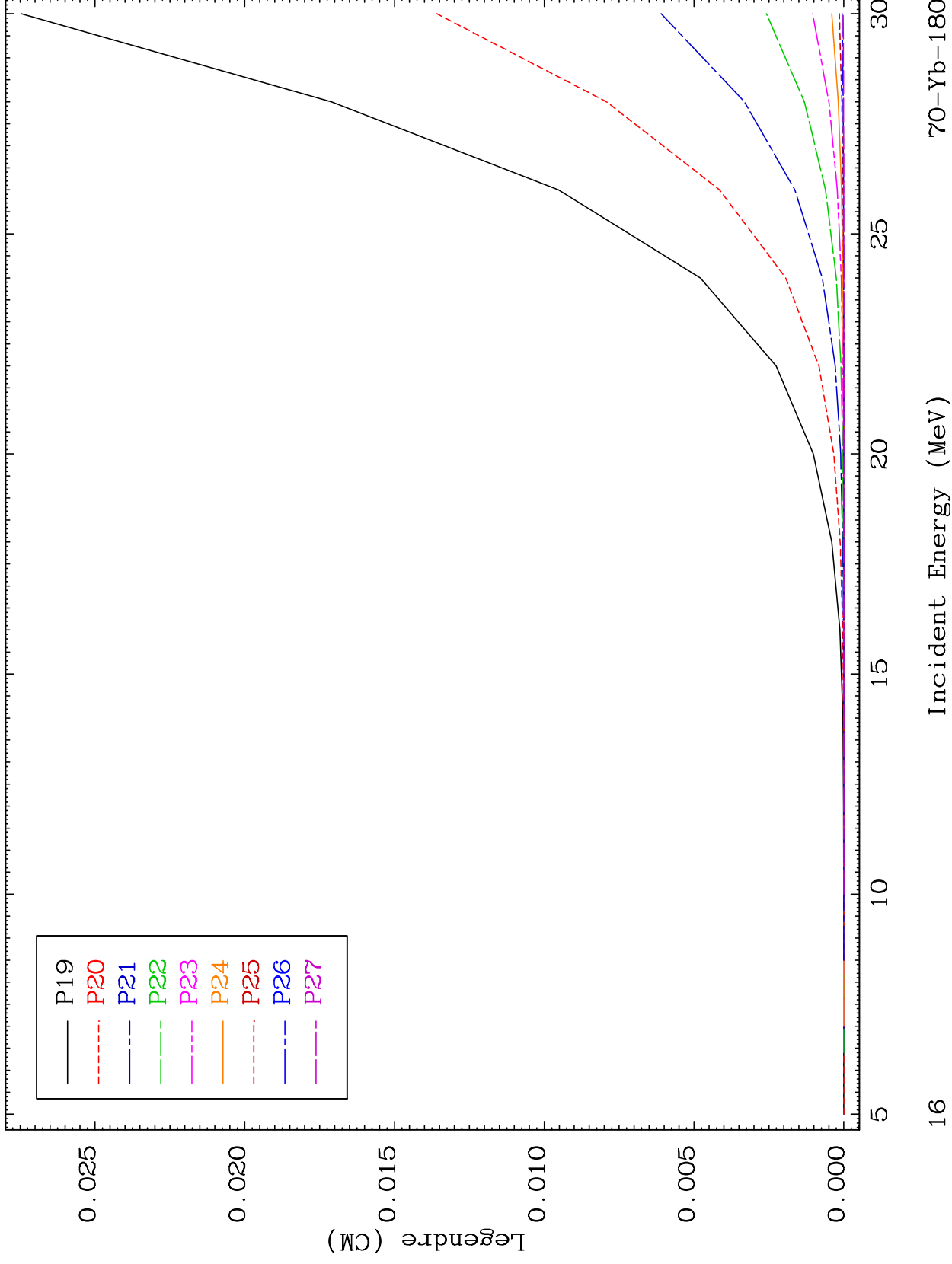




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Elastic  
Legendre Coefficients

70-Yb-180

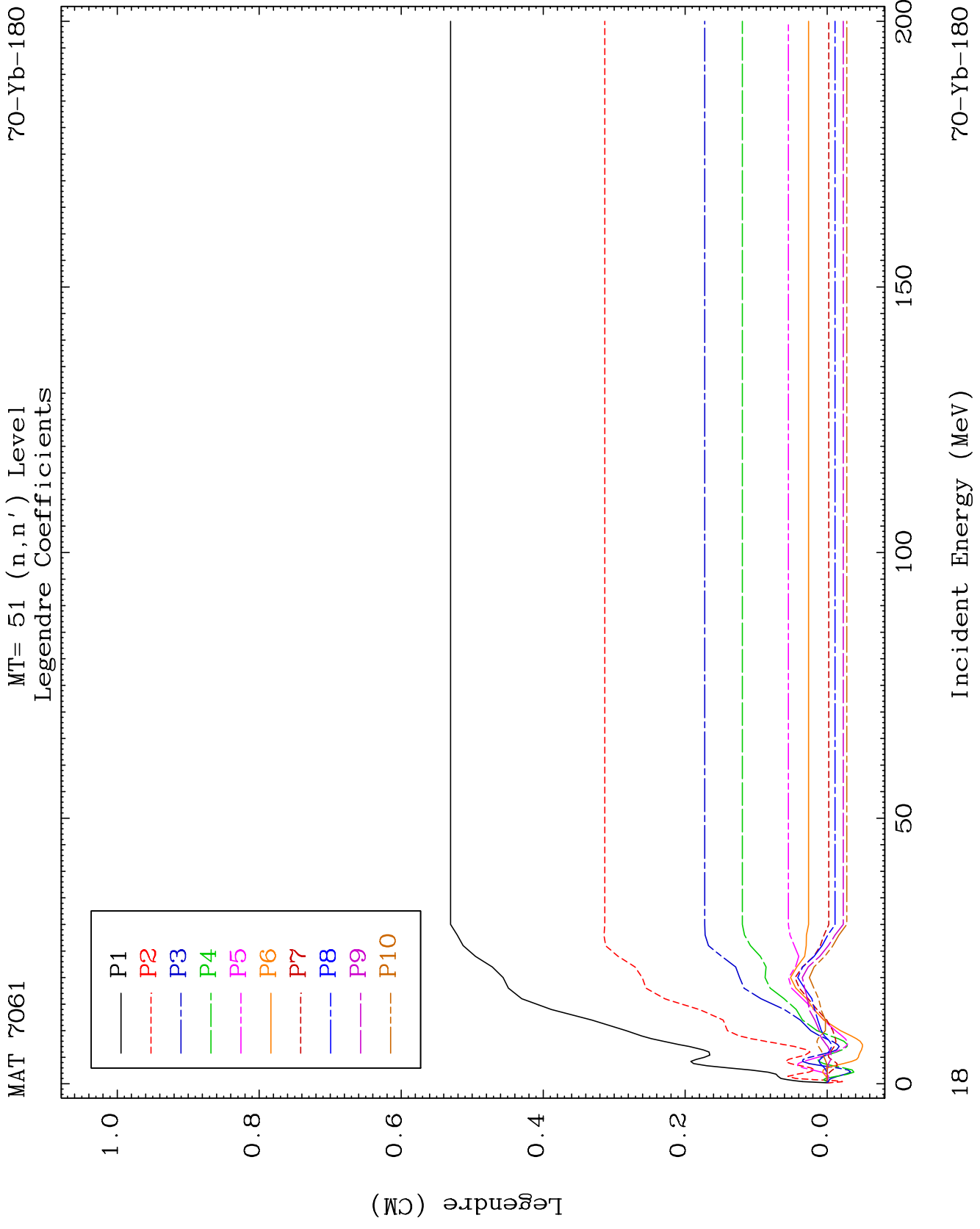


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Incident Energy (MeV)

70-Yb-180

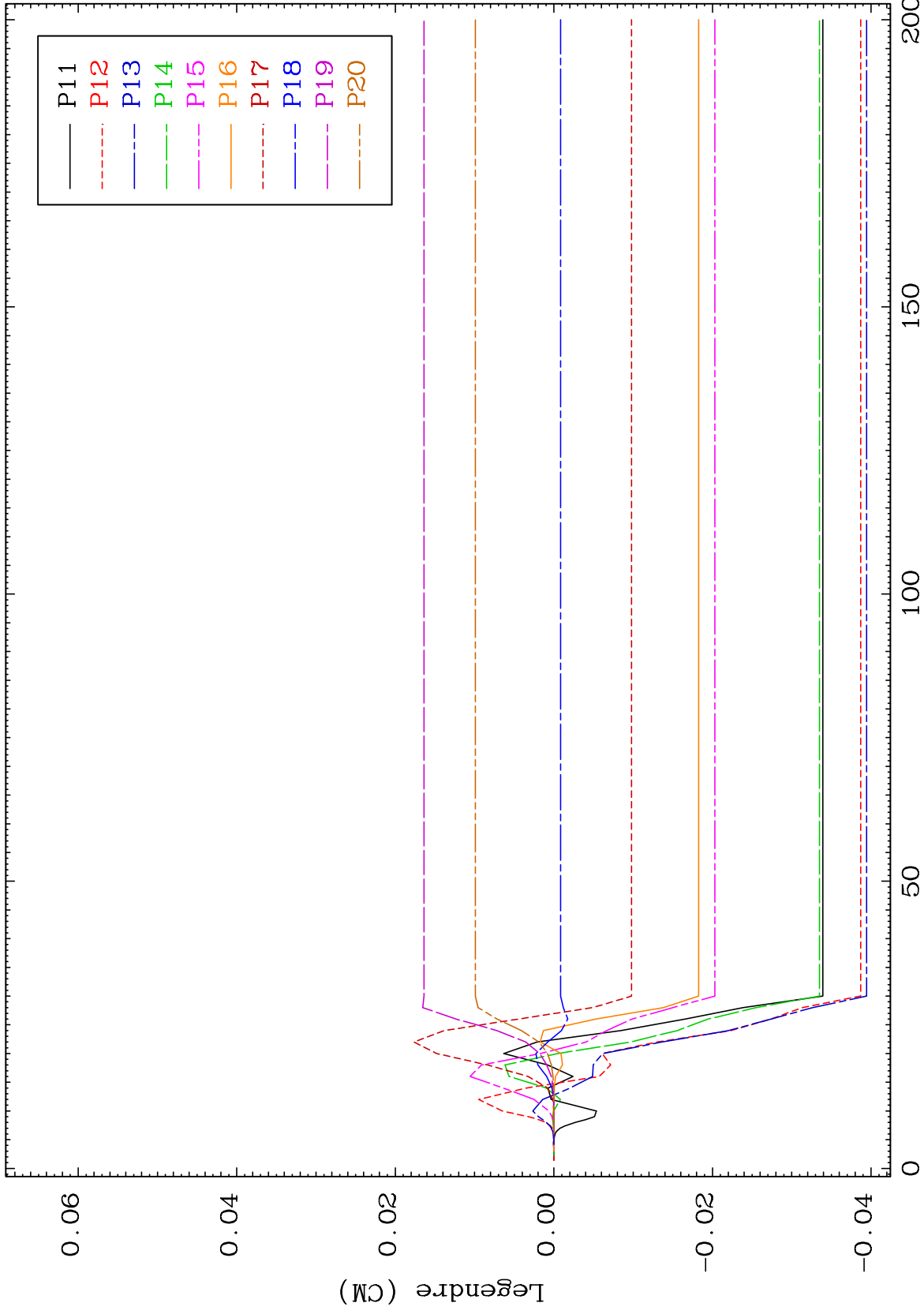




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MT= 51 (n,n') Level  
Legendre Coefficients

70-Yb-180



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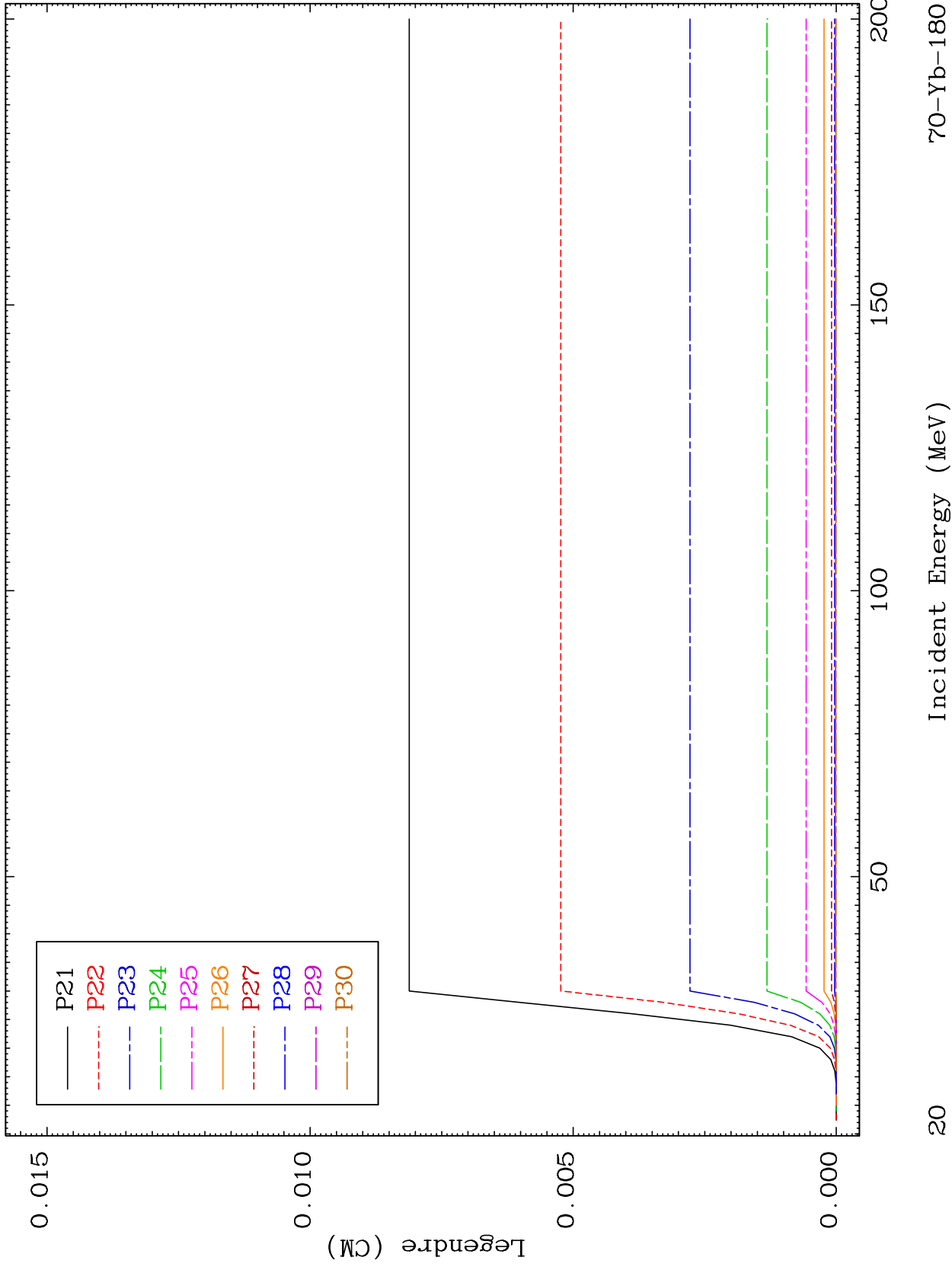
Incident Energy (MeV)

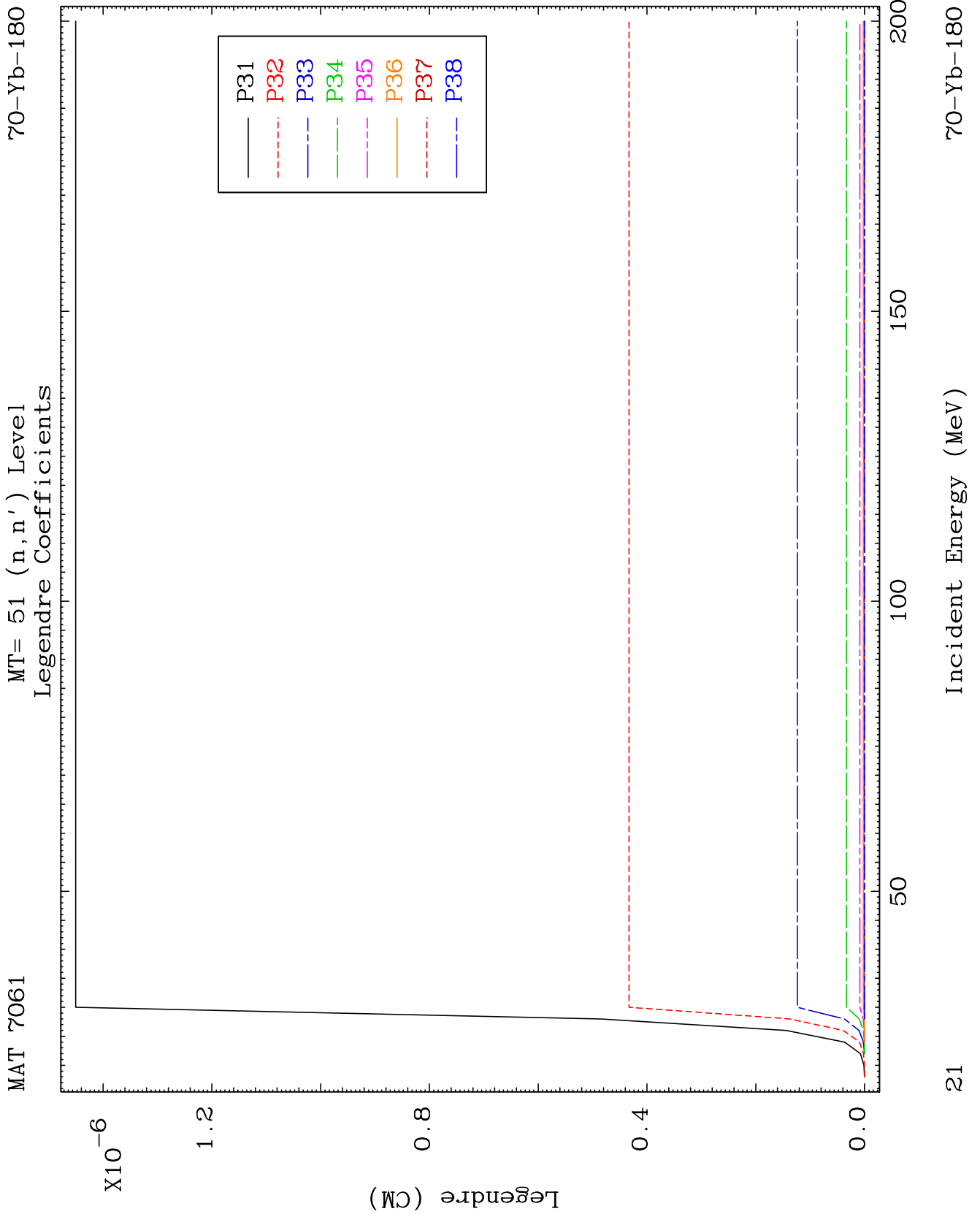
70-Yb-180

MAT 7061

MT= 51 (n,n') Level  
Legendre Coefficients

70-Yb-180



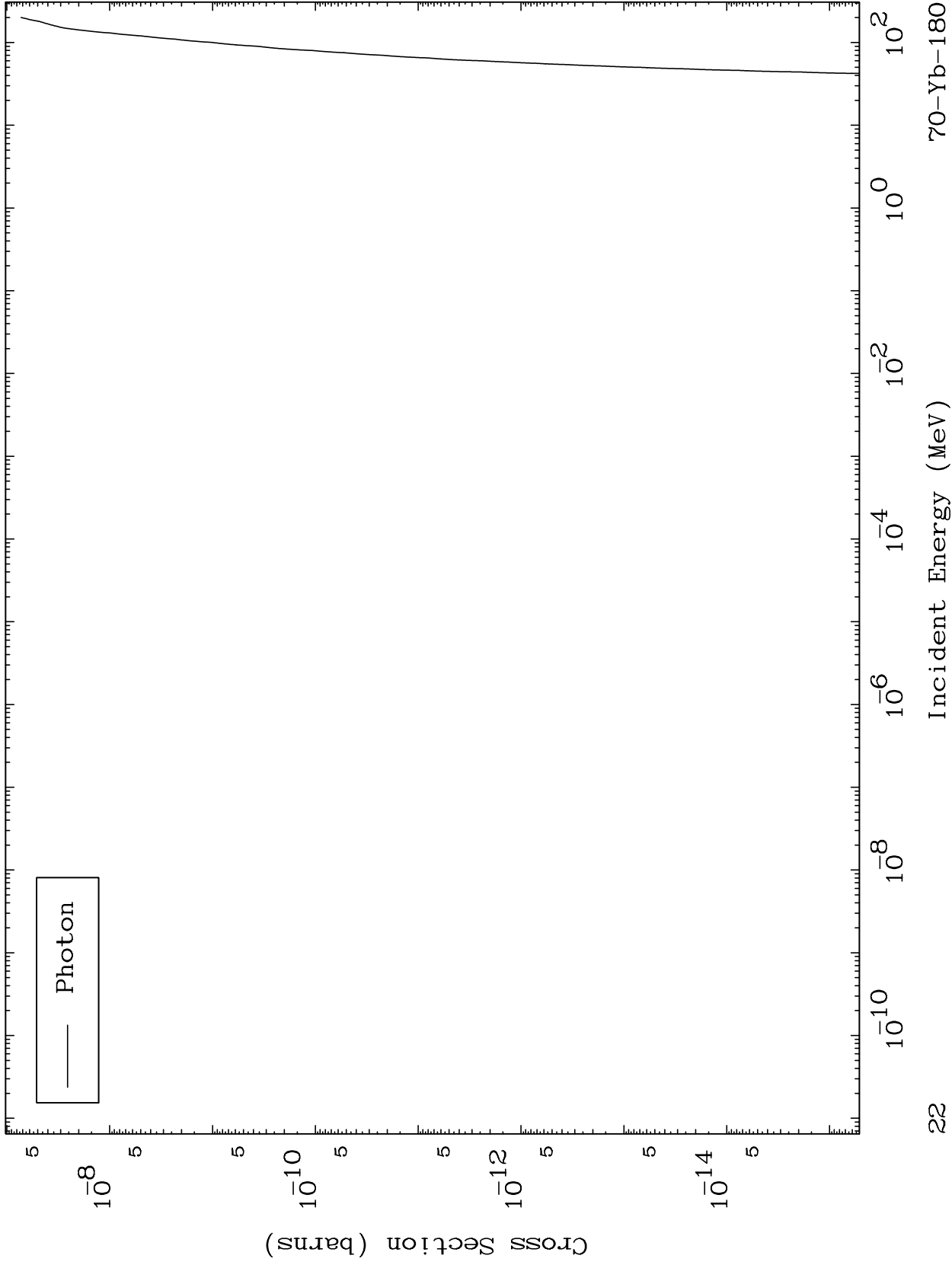


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Fission

70-Yb-180

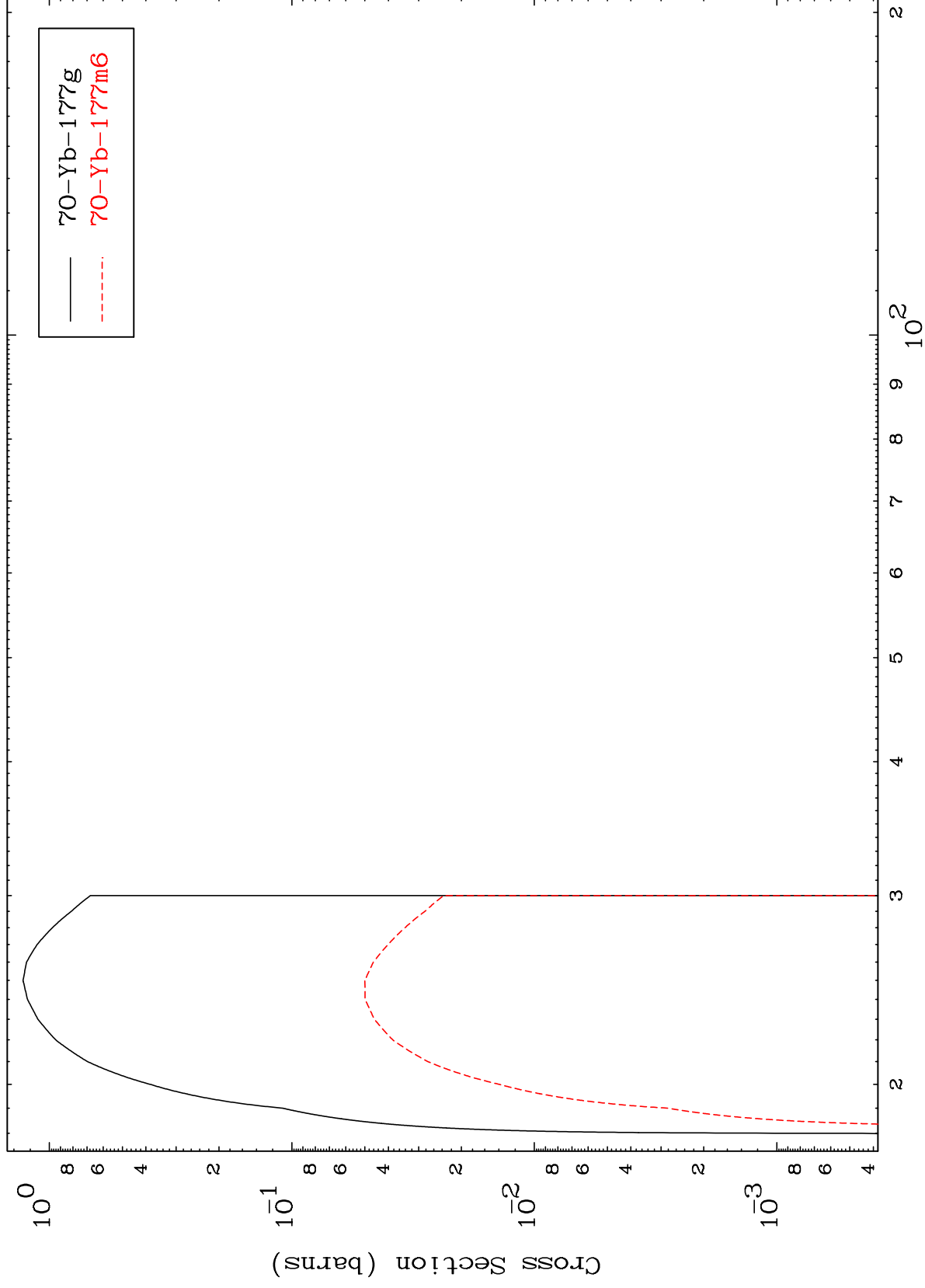
Radionuclide Production Cross Section



MAT 7061

70-Yb-180

(n,4n)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

70-Yb-180