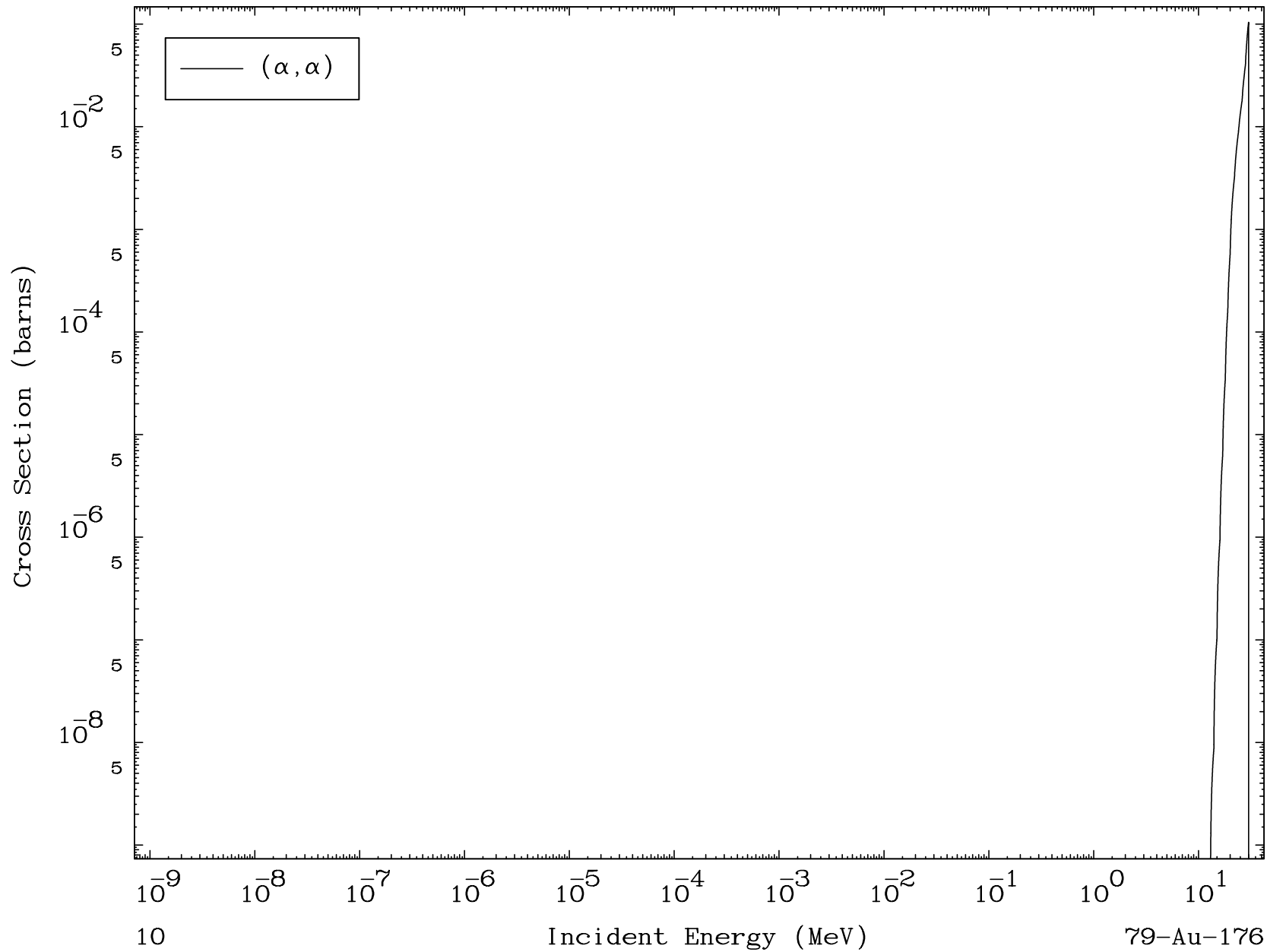


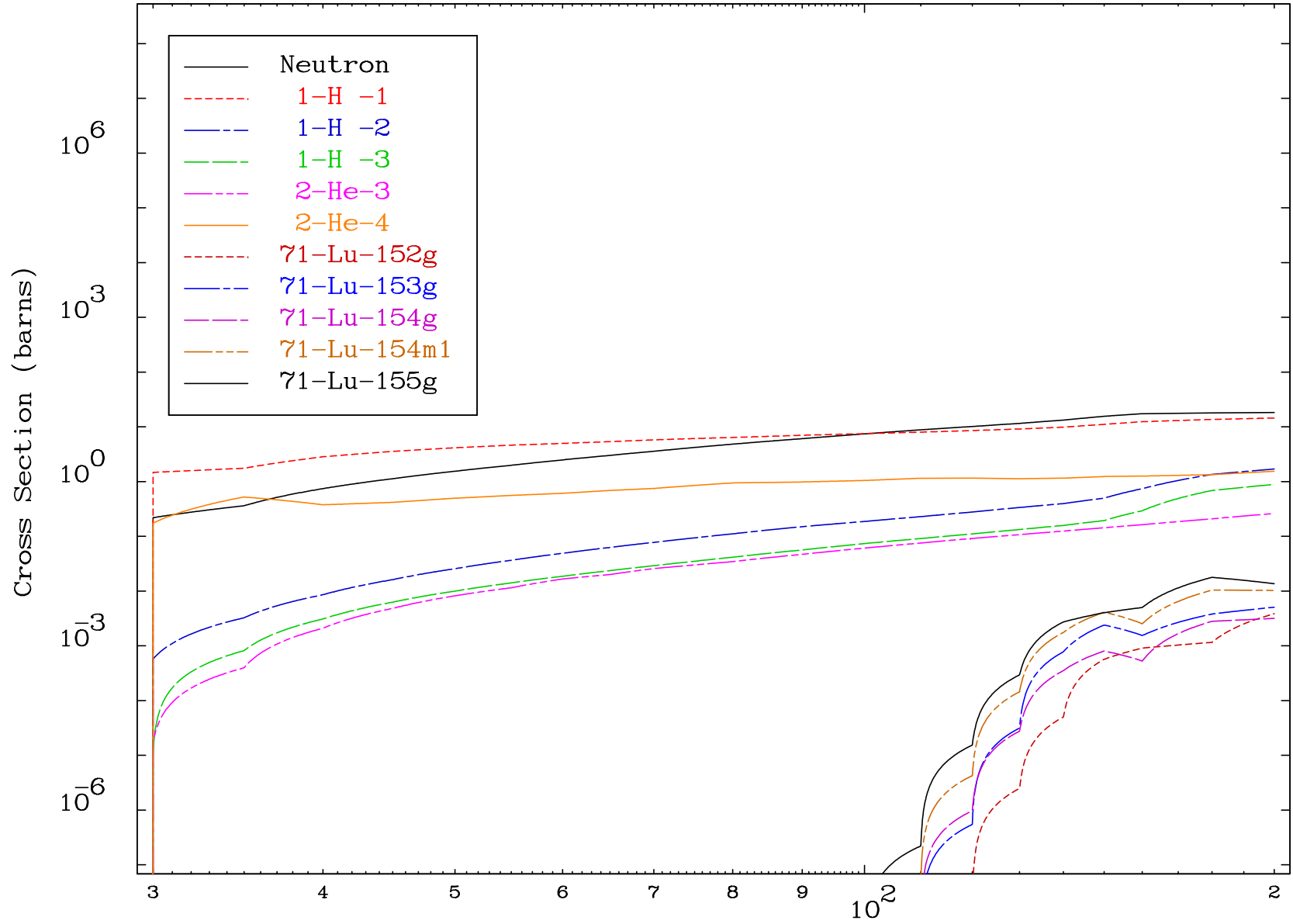
MAT 7863

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

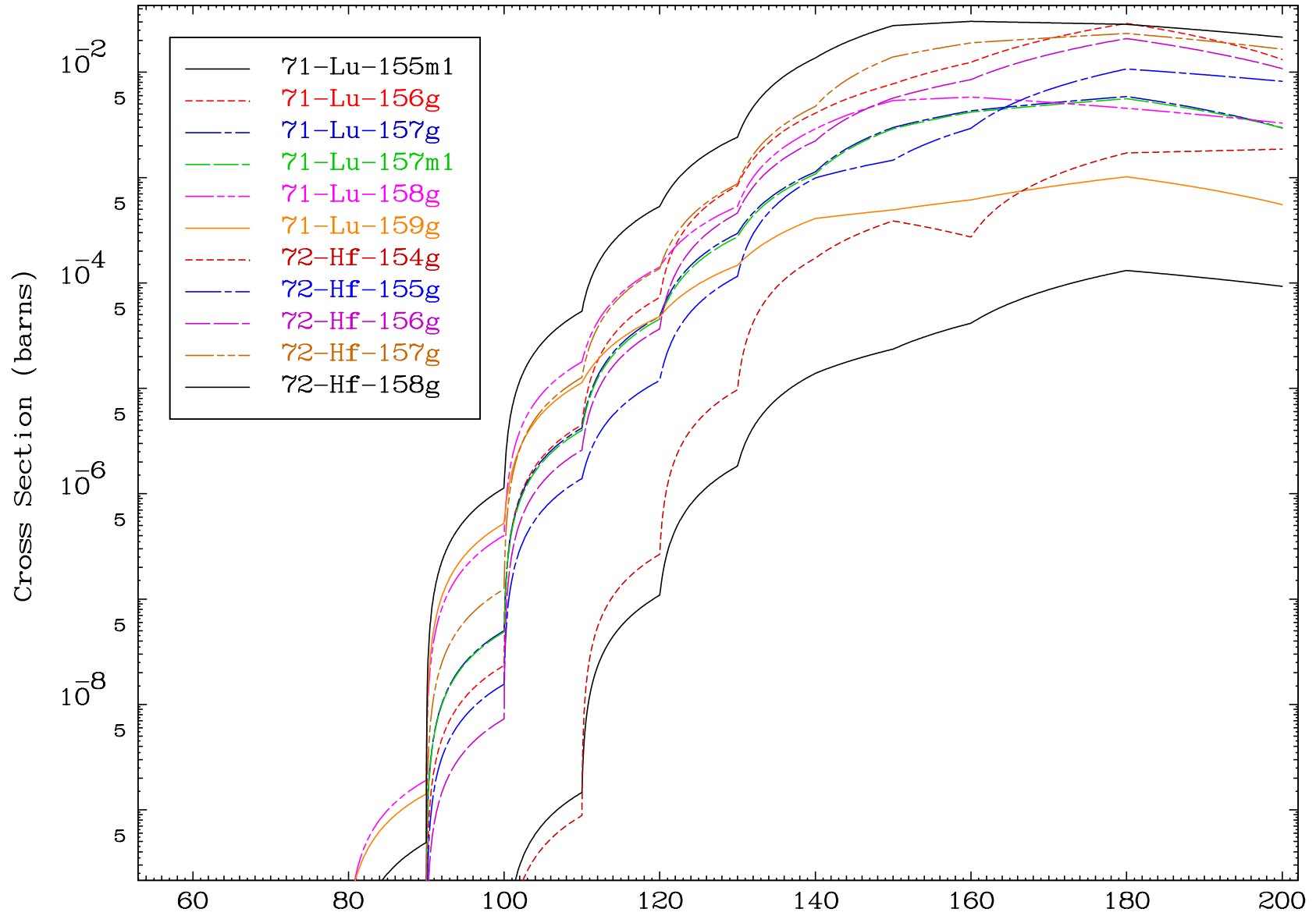
79-Au-176



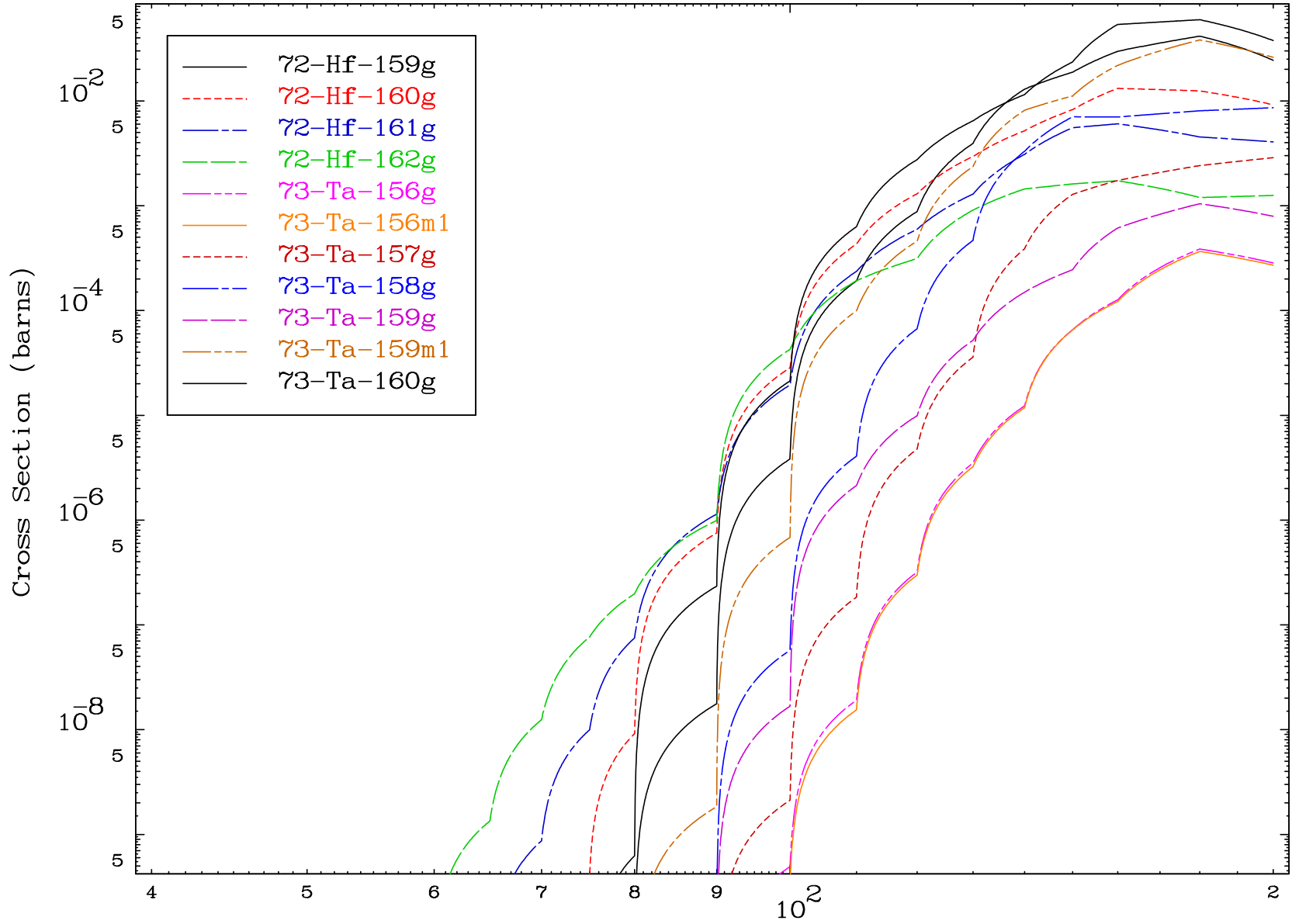
Radionuclide Production Cross Section



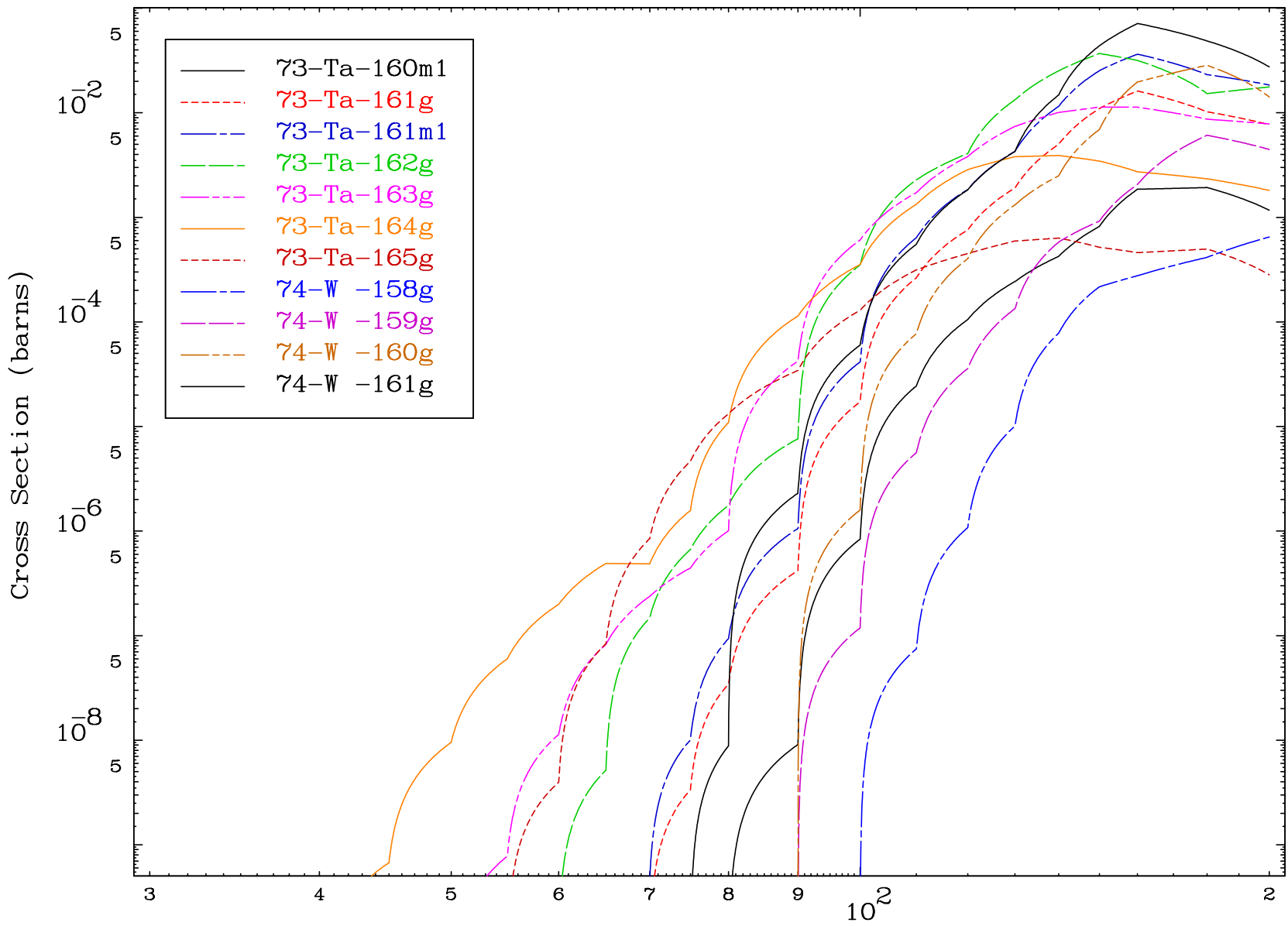
Radionuclide Production Cross Section



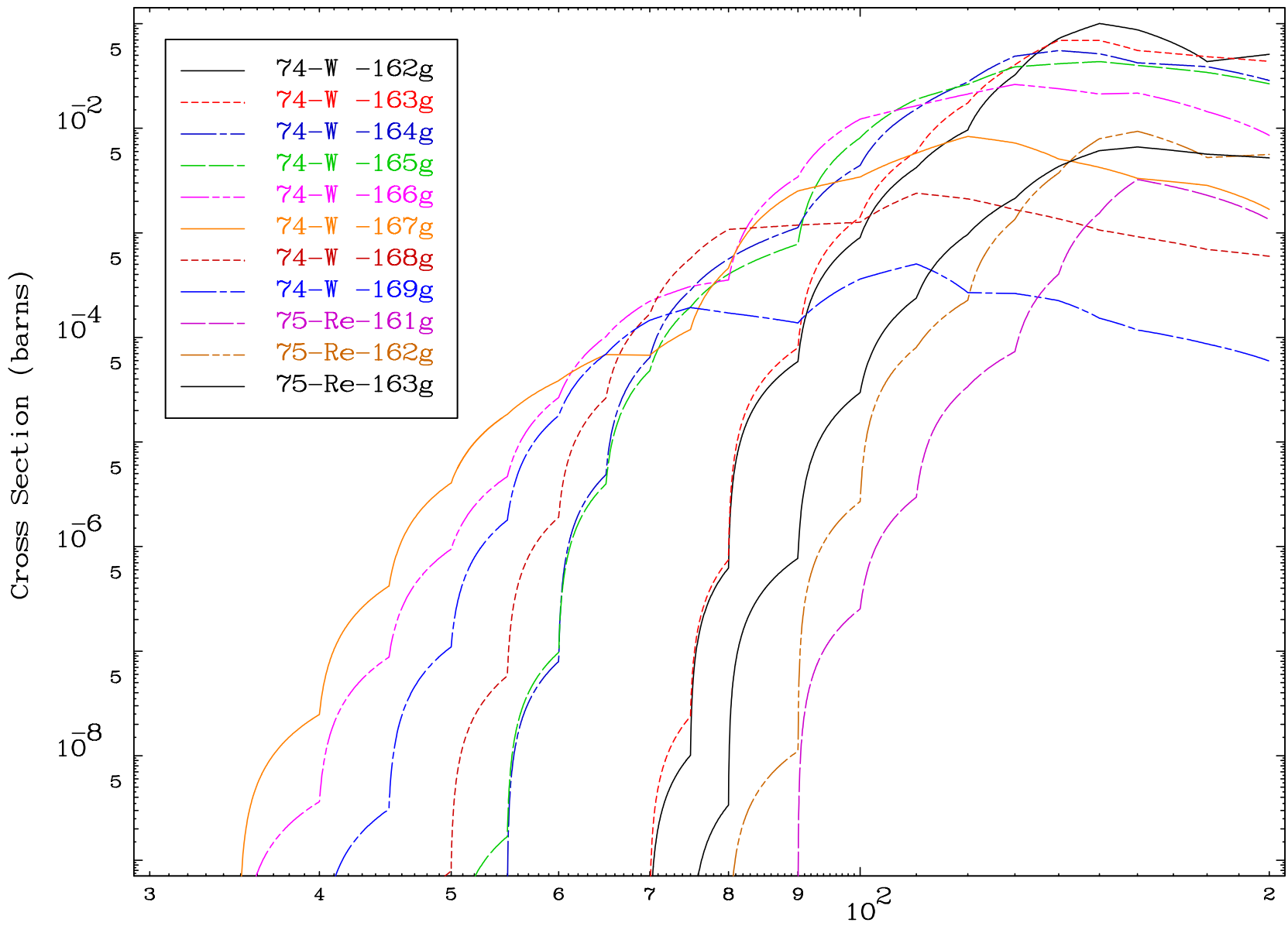
Radionuclide Production Cross Section



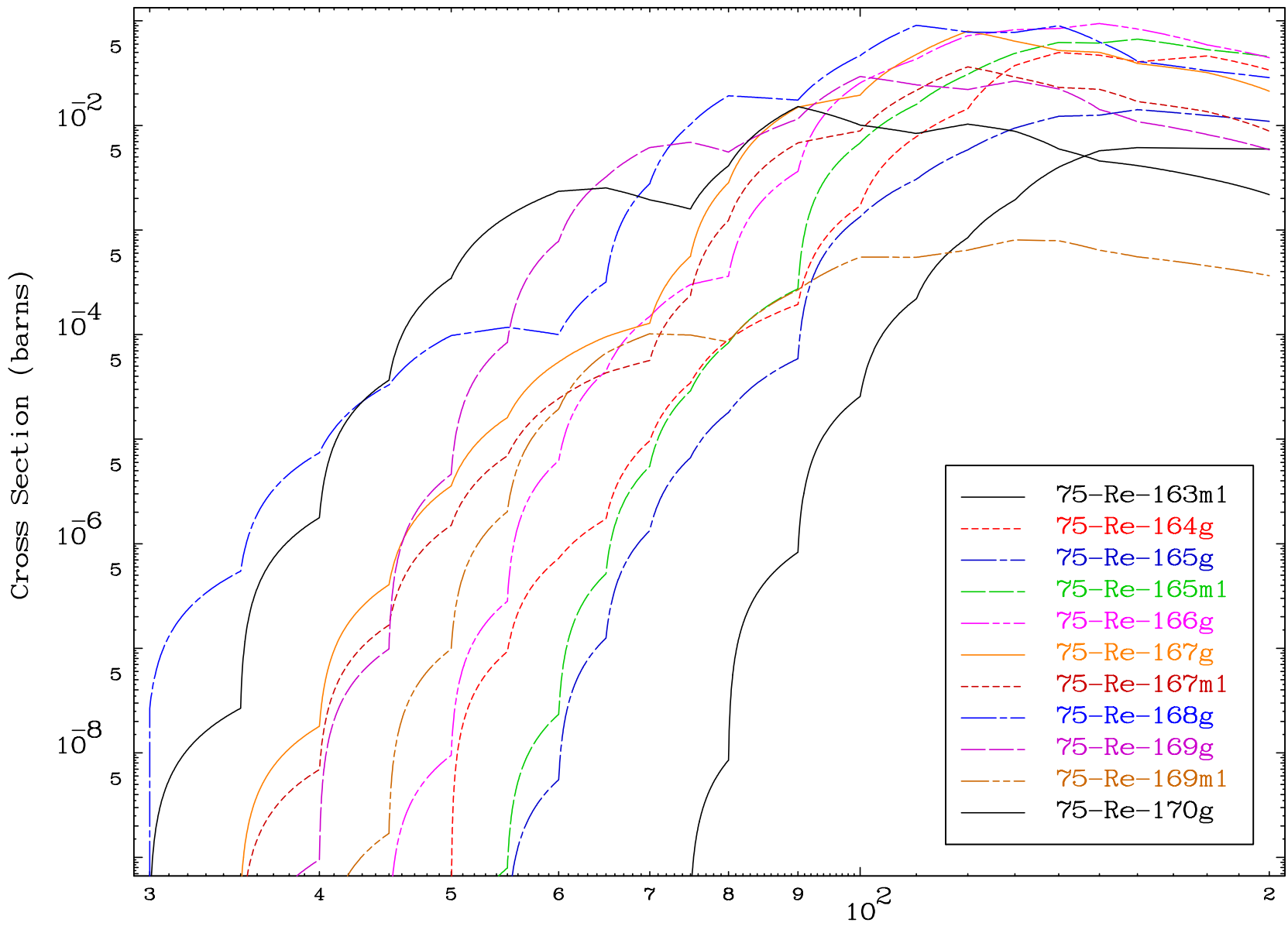
Radionuclide Production Cross Section



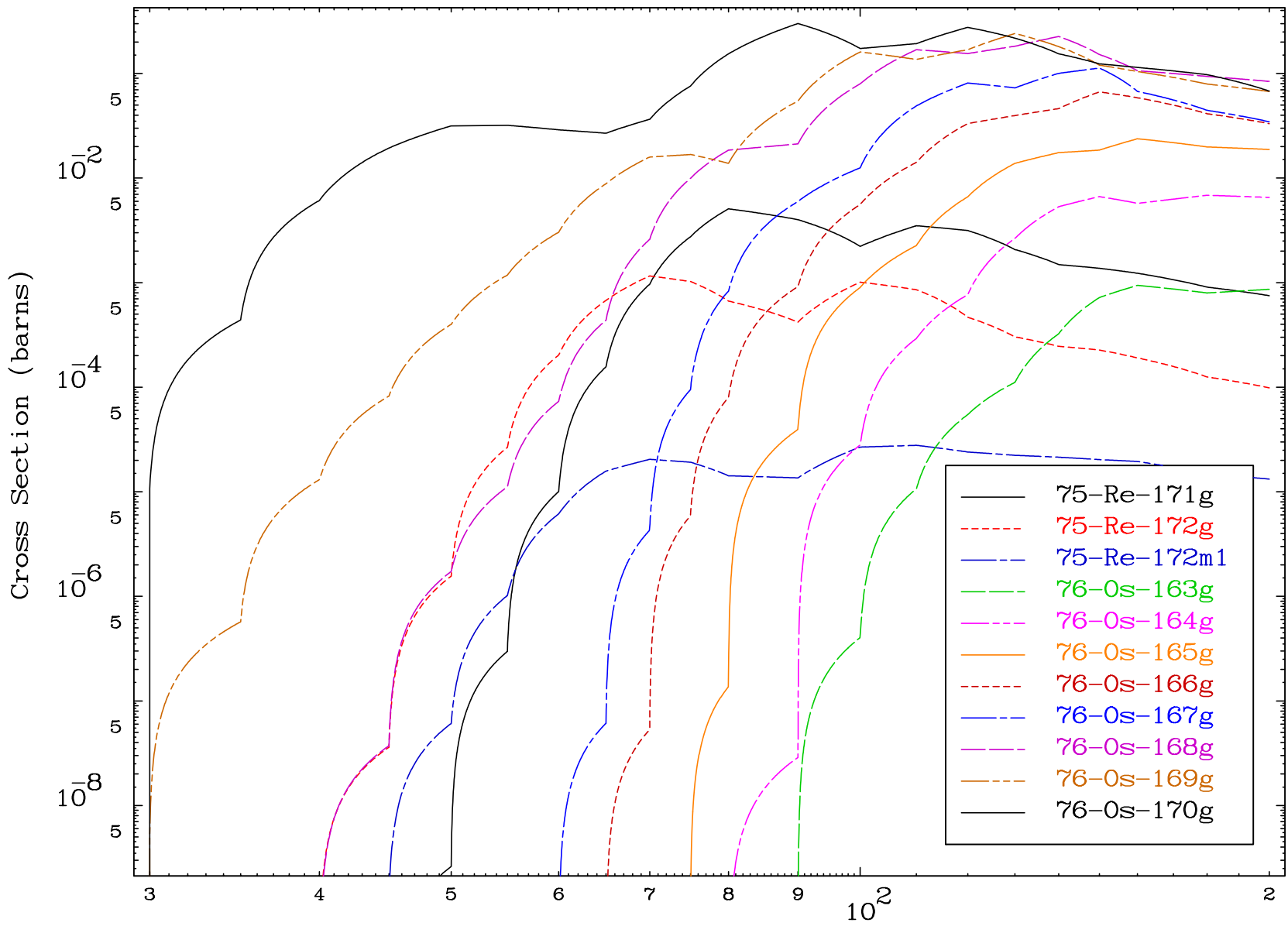
Radionuclide Production Cross Section



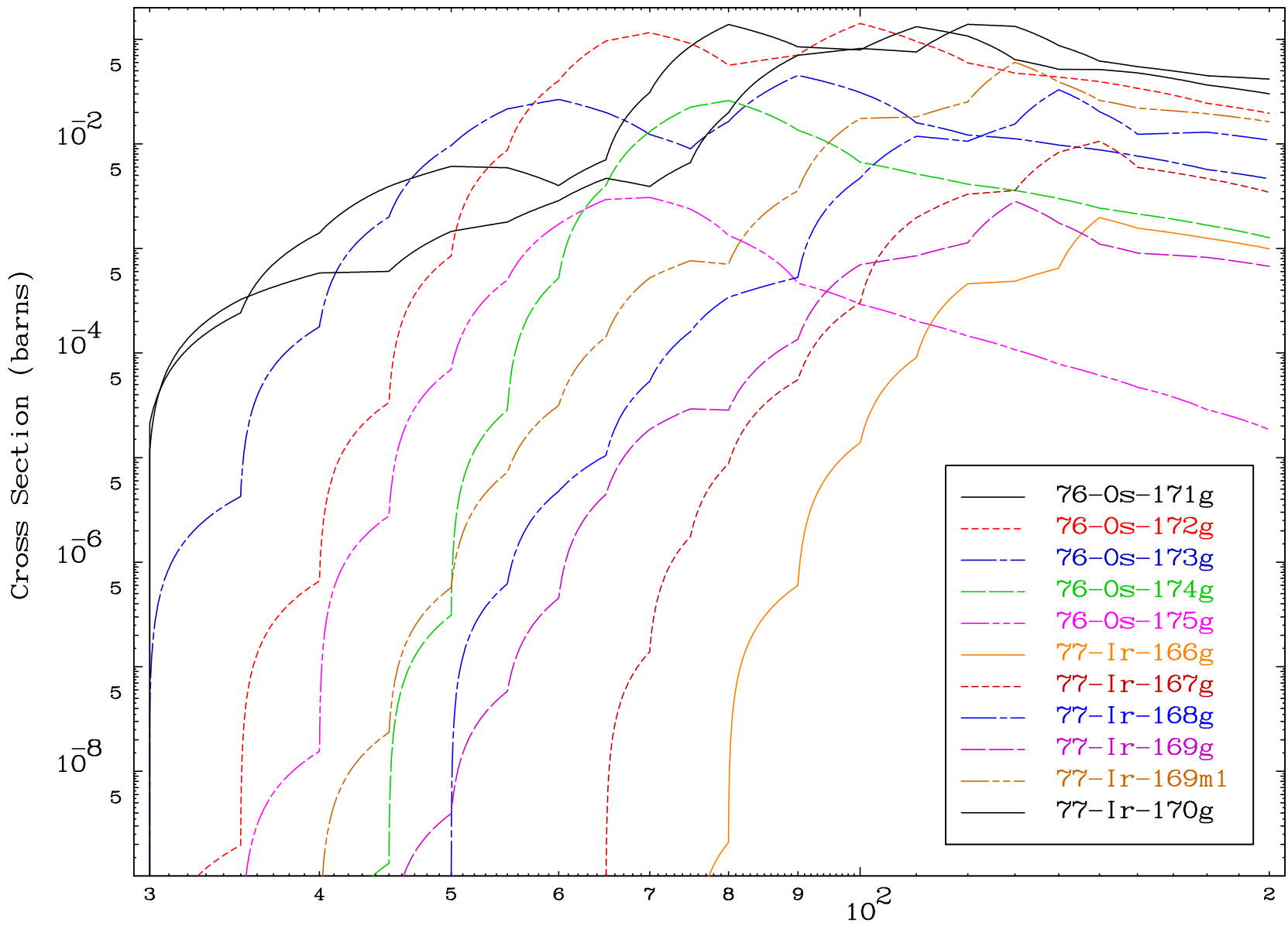
Radionuclide Production Cross Section



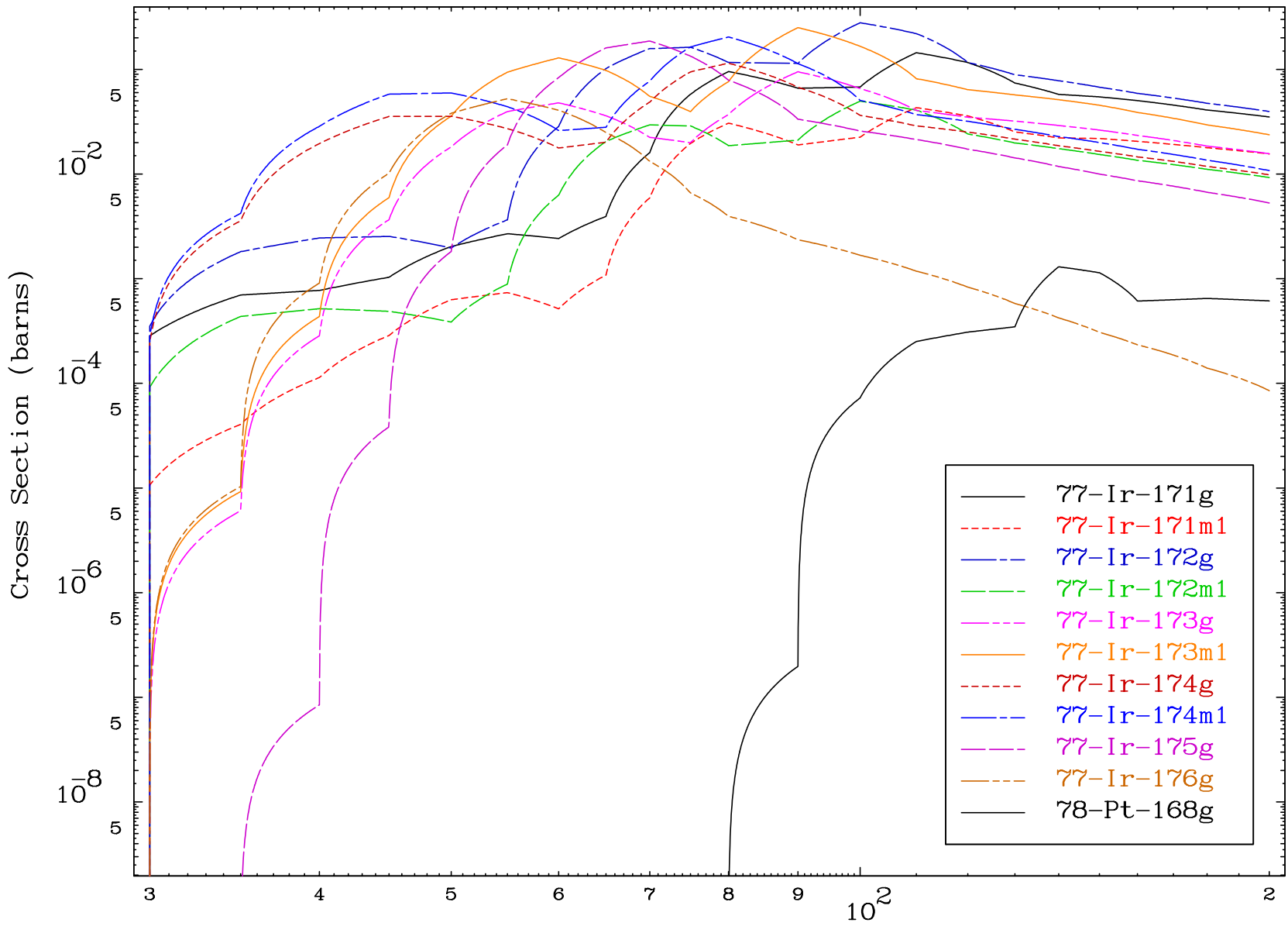




Radionuclide Production Cross Section



Radionuclide Production Cross Section

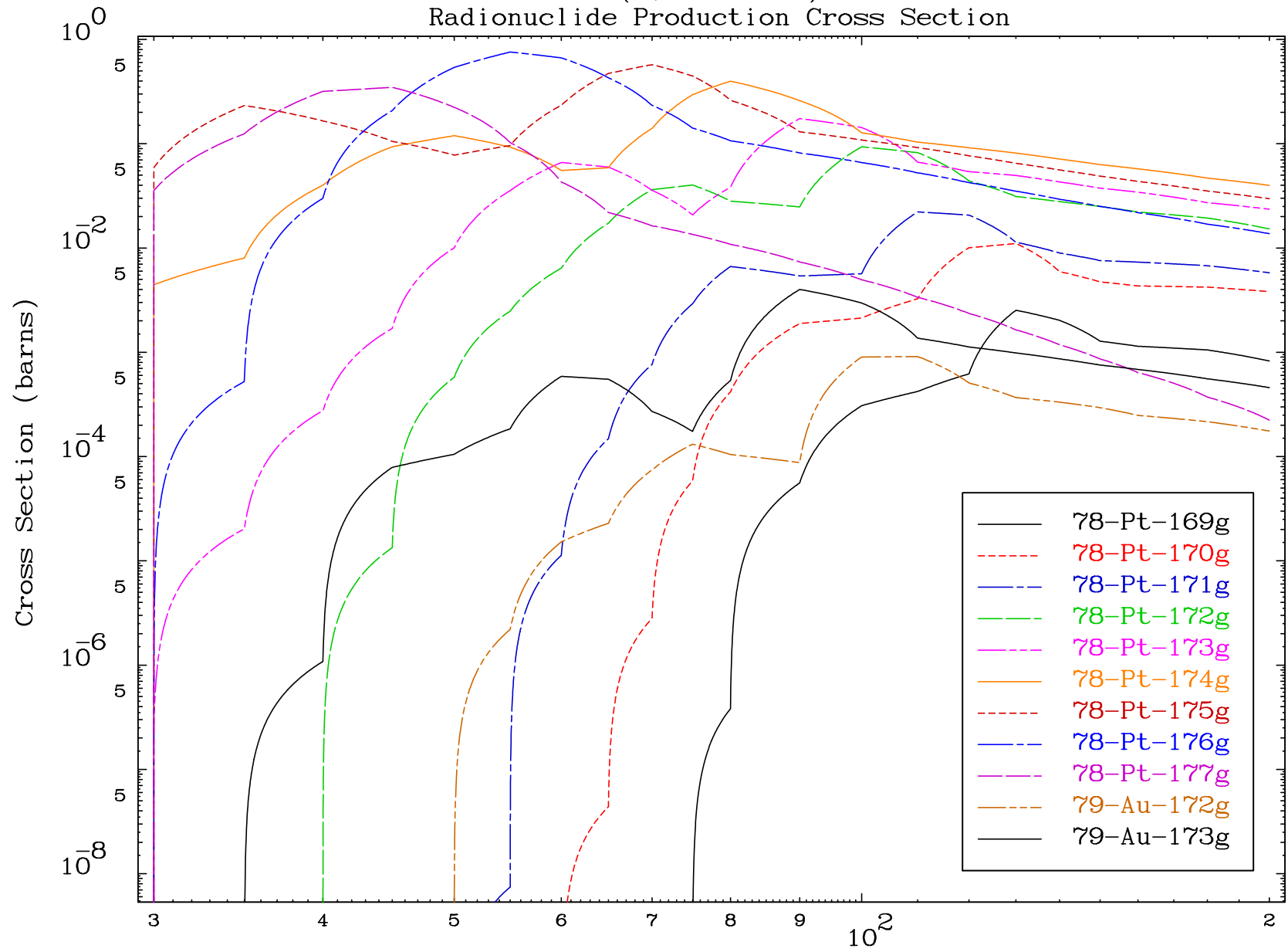


MAT 7863

( $\alpha$ , remainder)

79-Au-176

### Radionuclide Production Cross Section

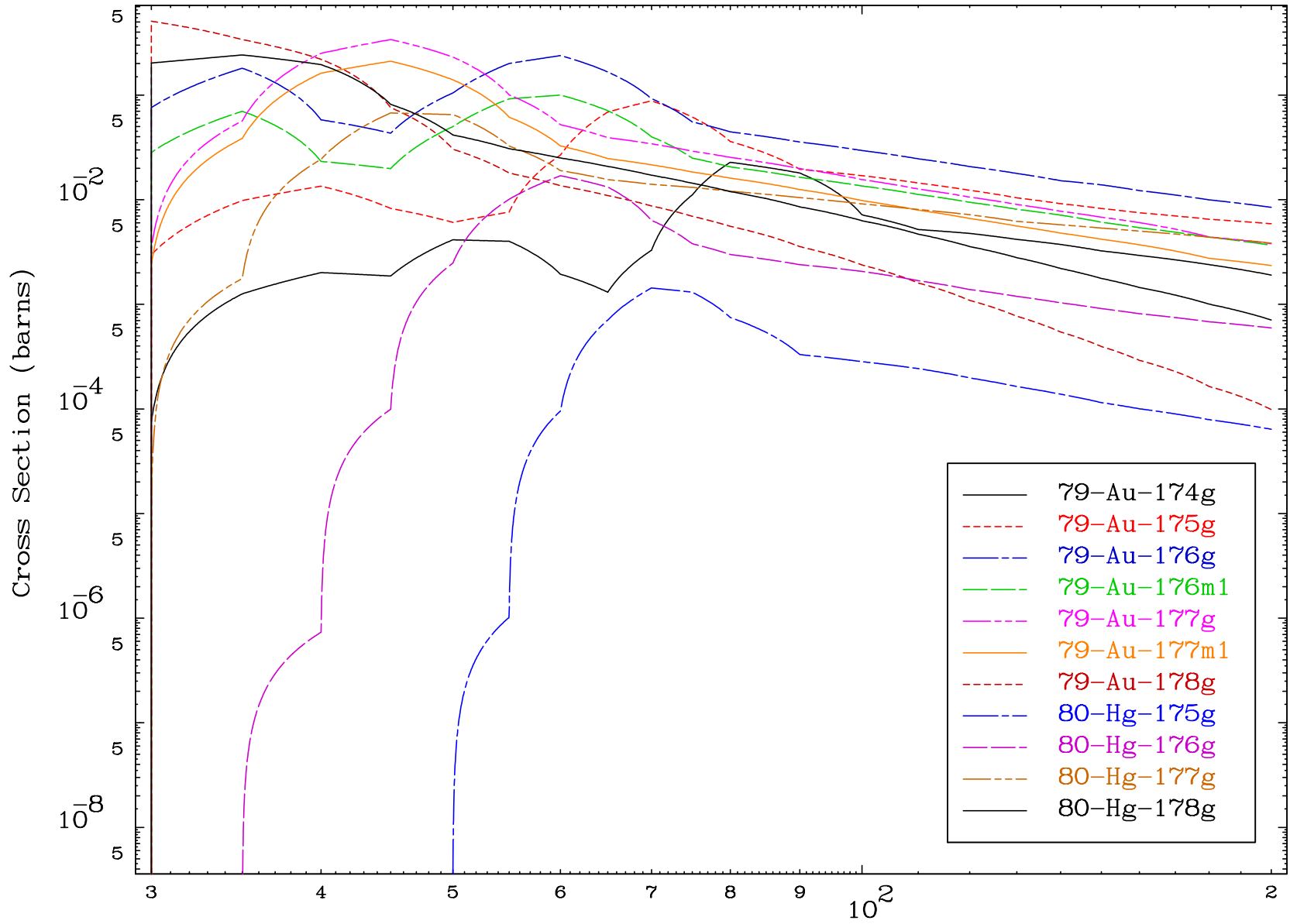


20

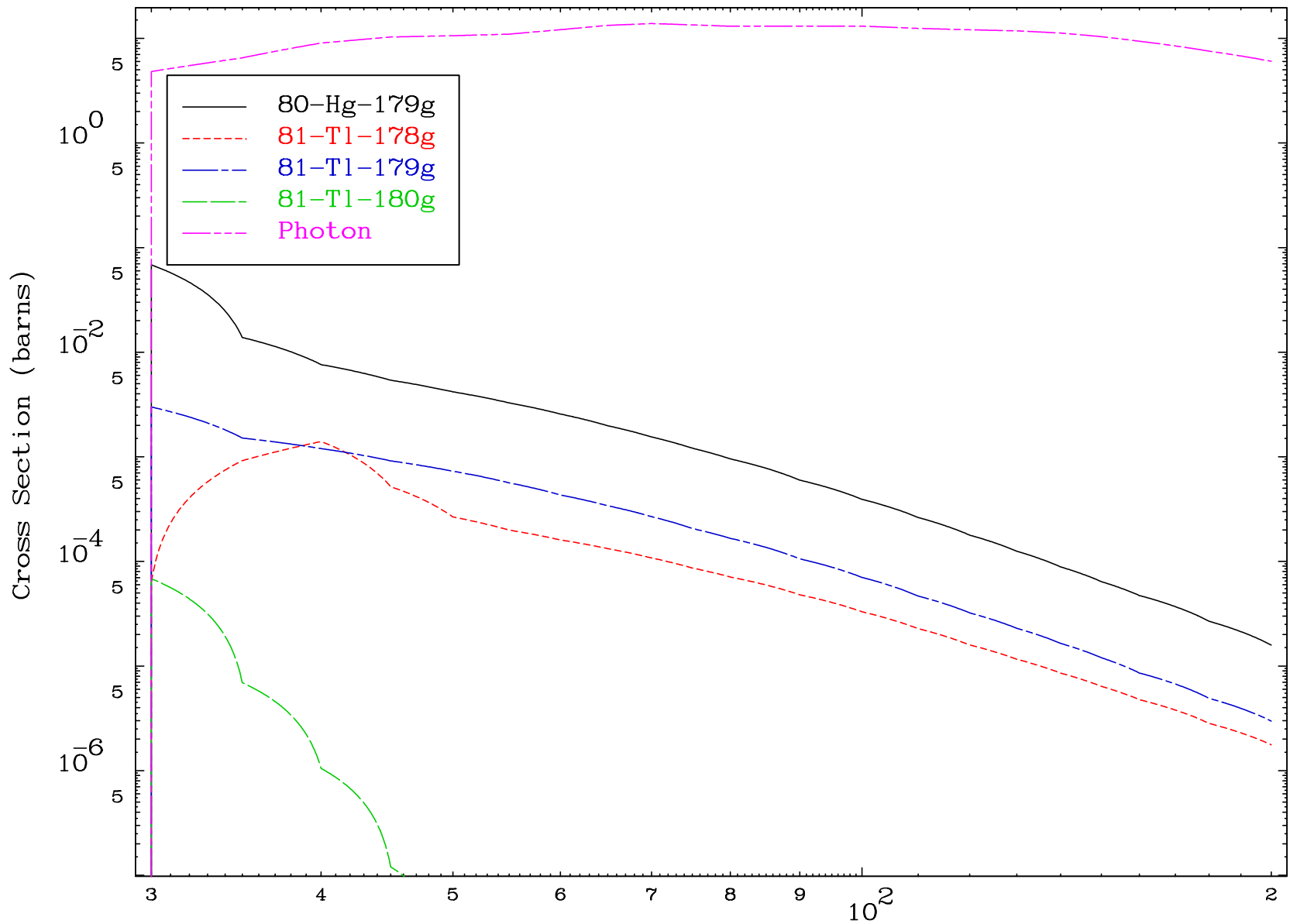
Incident Energy (MeV)

79-Au-176

Radionuclide Production Cross Section



Radionuclide Production Cross Section

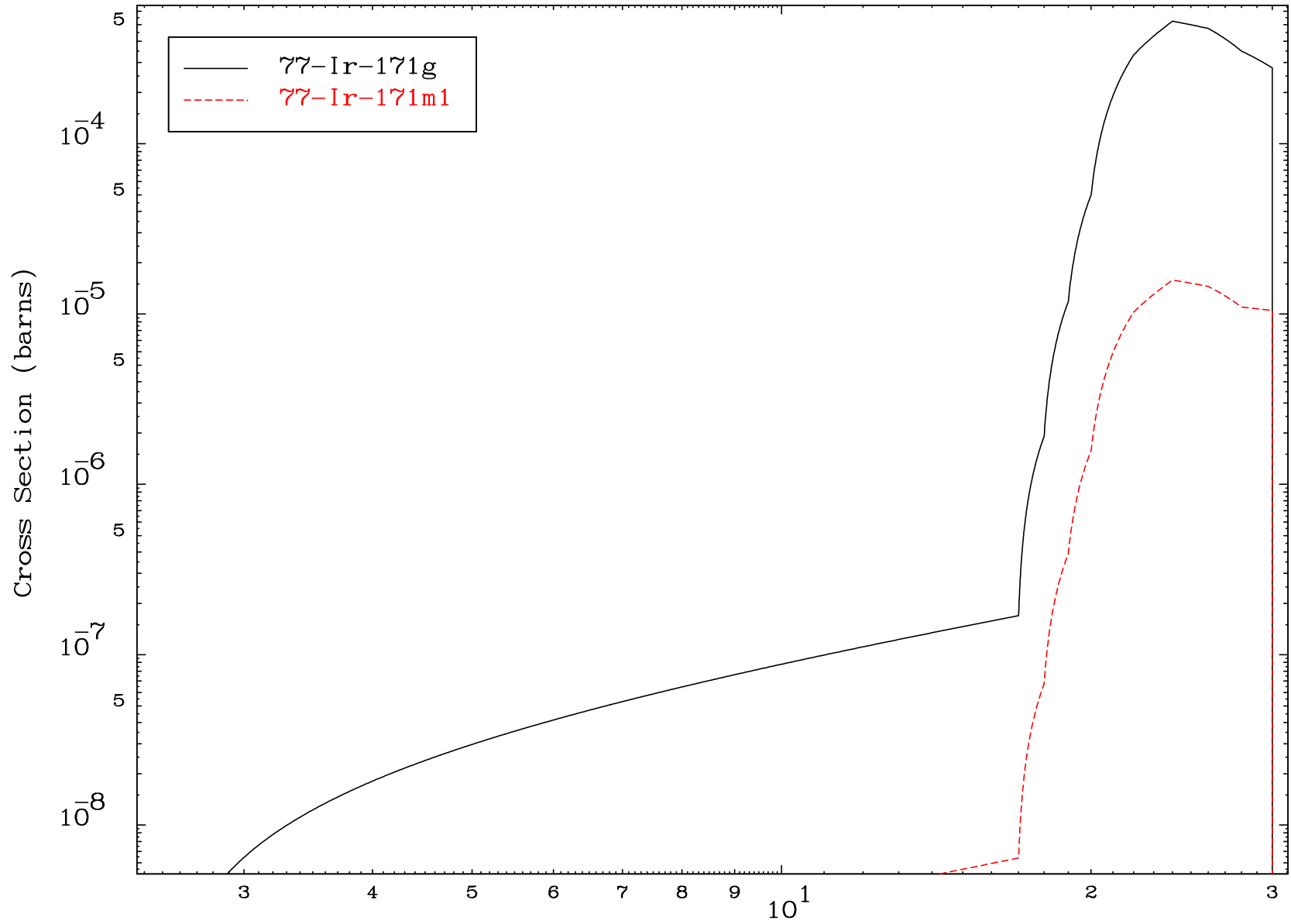


MAT 7863

( $\alpha, n'$ )  $2\alpha$

79-Au-176

Radionuclide Production Cross Section

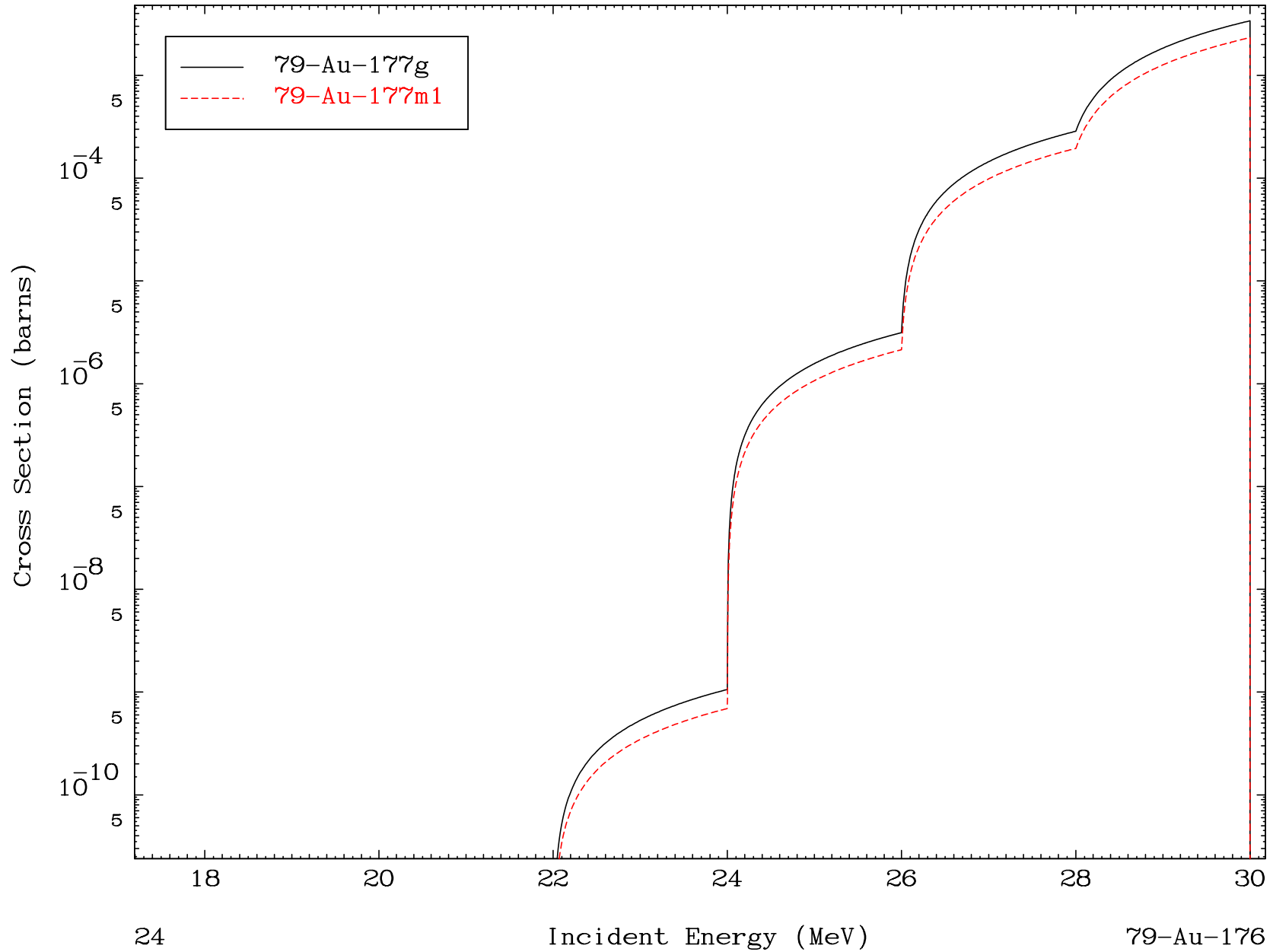


23

Incident Energy (MeV)

79-Au-176

Radionuclide Production Cross Section



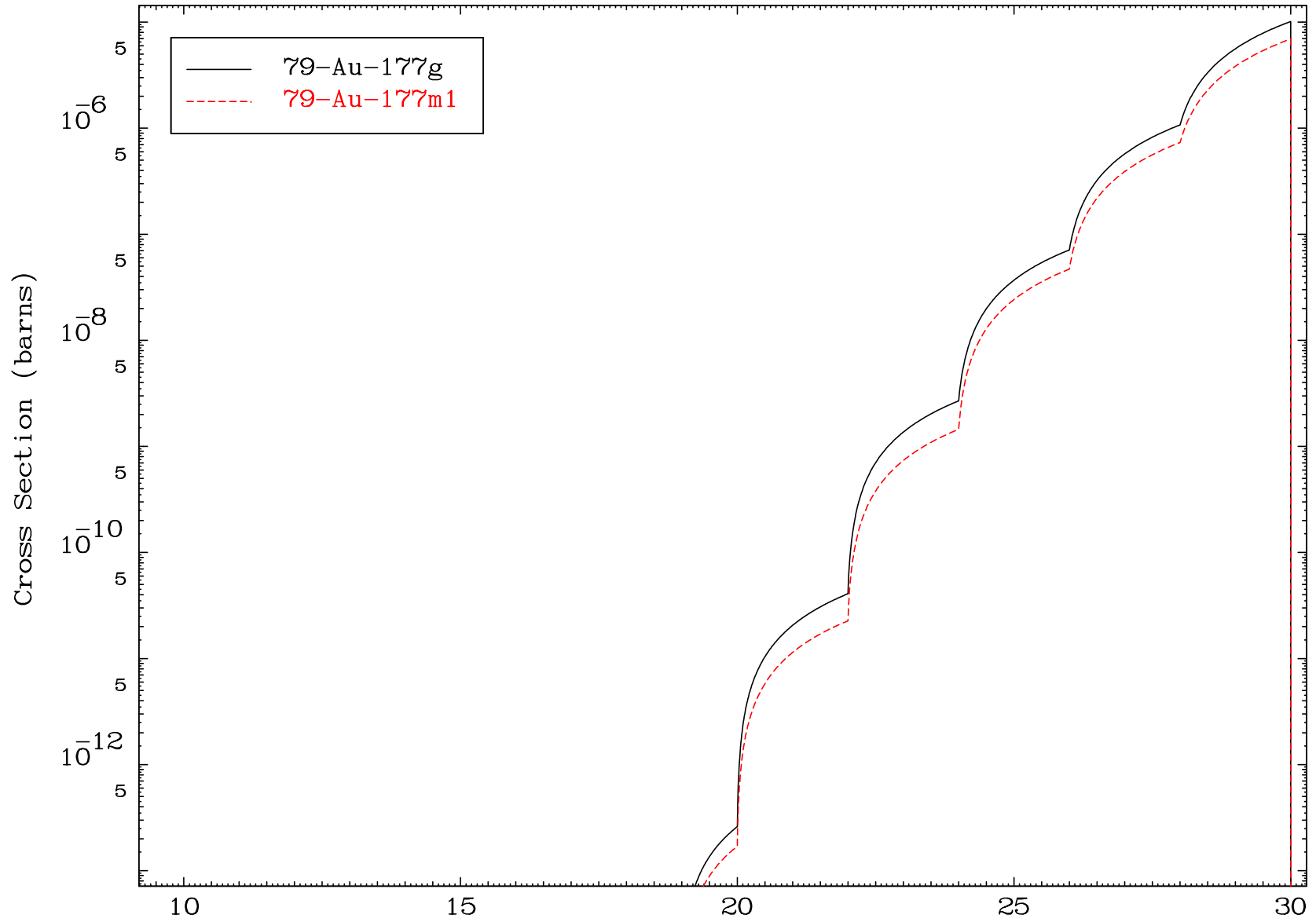


MAT 7863

( $\alpha, \text{He-3}$ )

79-Au-176

Radionuclide Production Cross Section



25

Incident Energy (MeV)

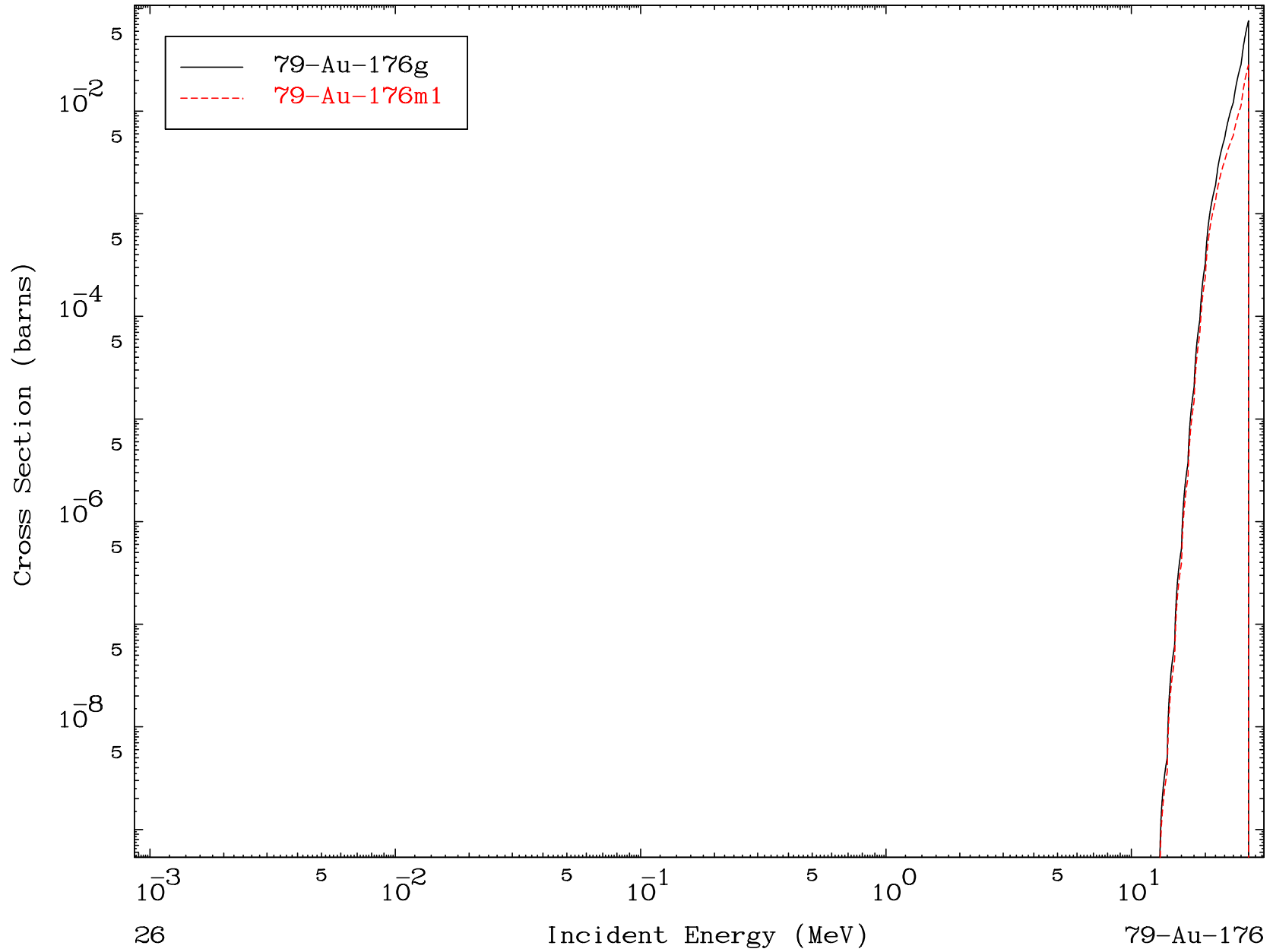
79-Au-176

MAT 7863

( $\alpha, \alpha$ )

79-Au-176

Radionuclide Production Cross Section

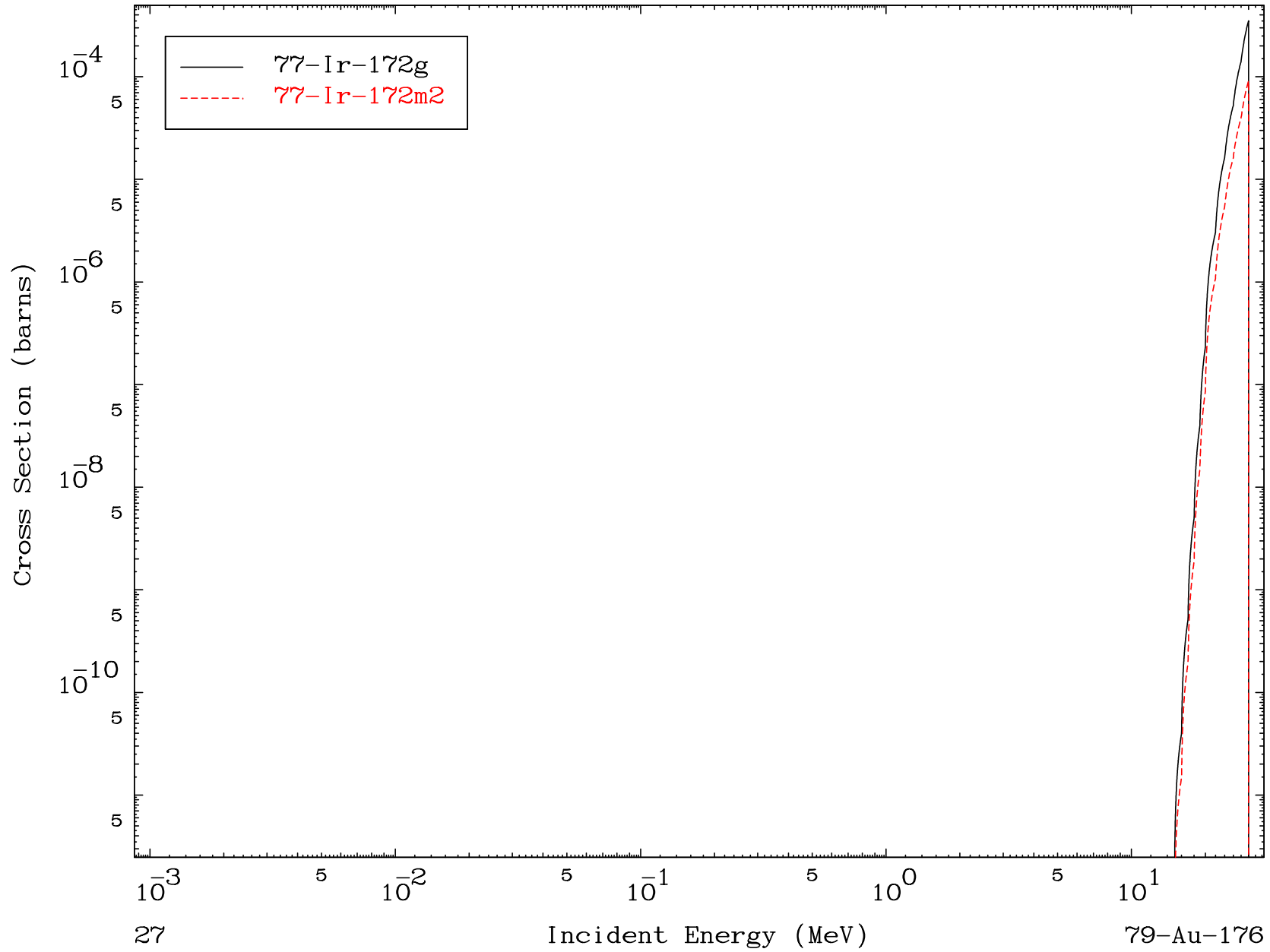


MAT 7863

( $\alpha, 2\alpha$ )

79-Au-176

Radionuclide Production Cross Section



27

Incident Energy (MeV)

79-Au-176

MAT 7863

( $\alpha, p$ ) d

79-Au-176

Radionuclide Production Cross Section

