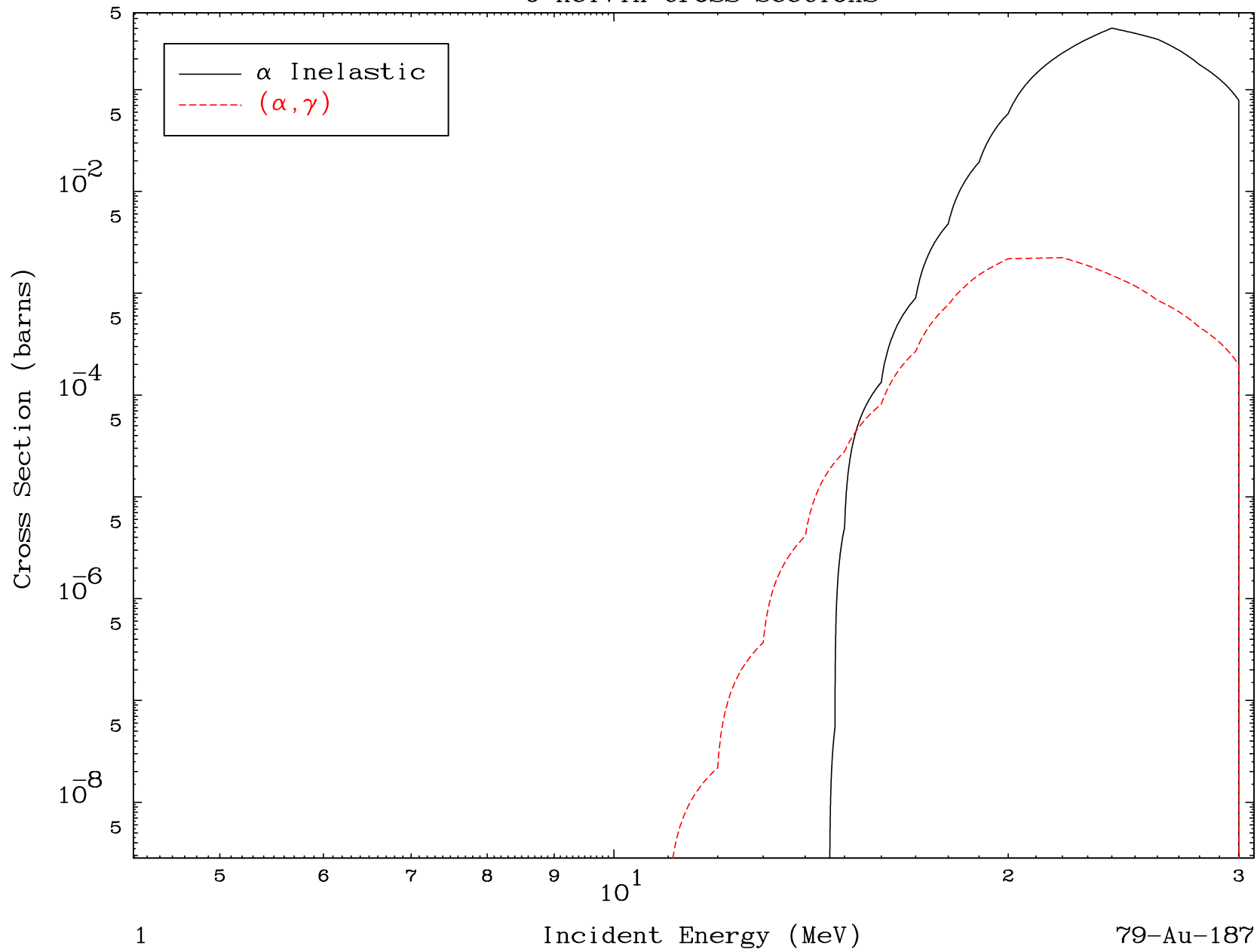


MAT 7895

$\alpha$  Major  
0 Kelvin Cross Sections

79-Au-187



1

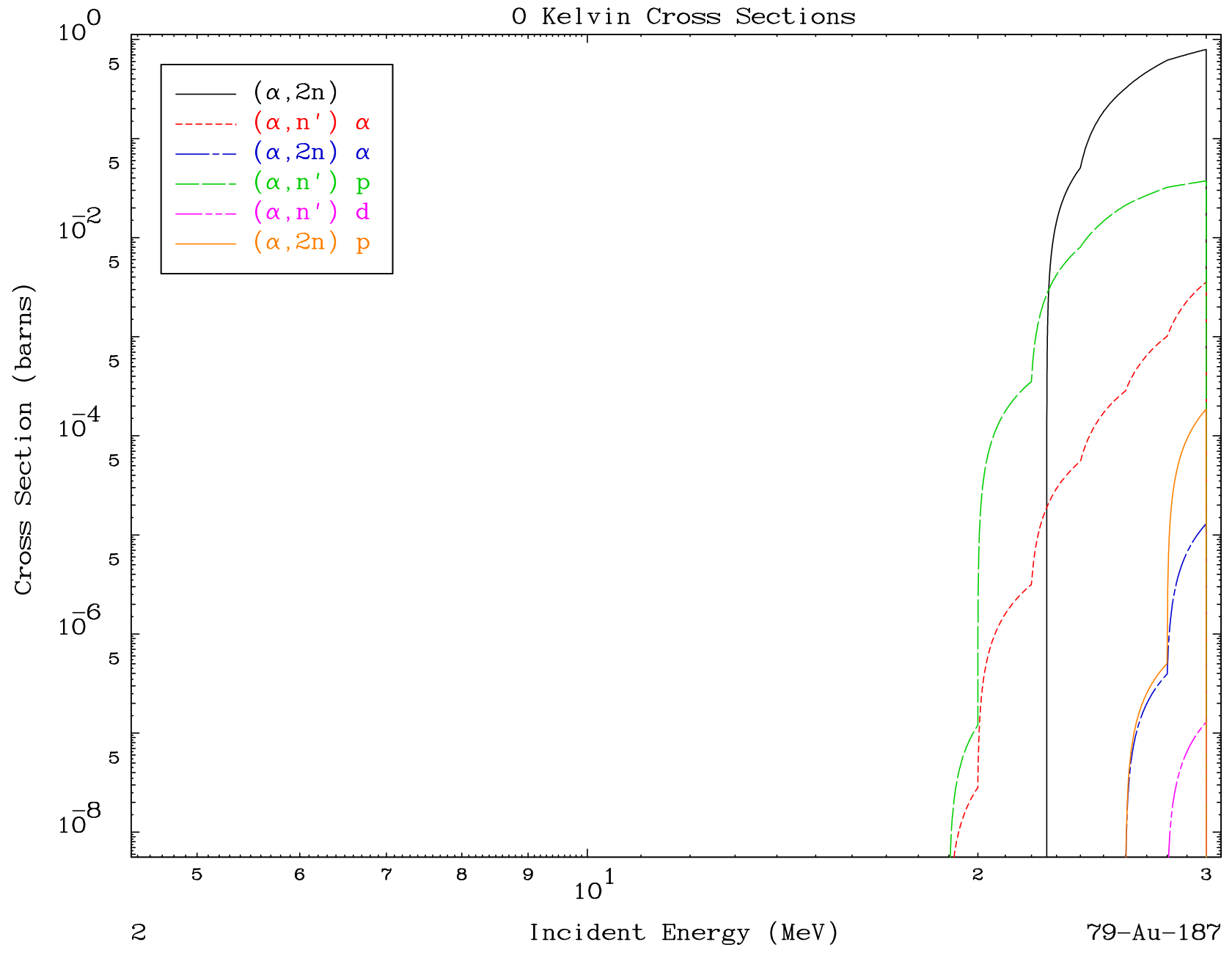
Incident Energy (MeV)

79-Au-187

MAT 7895

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

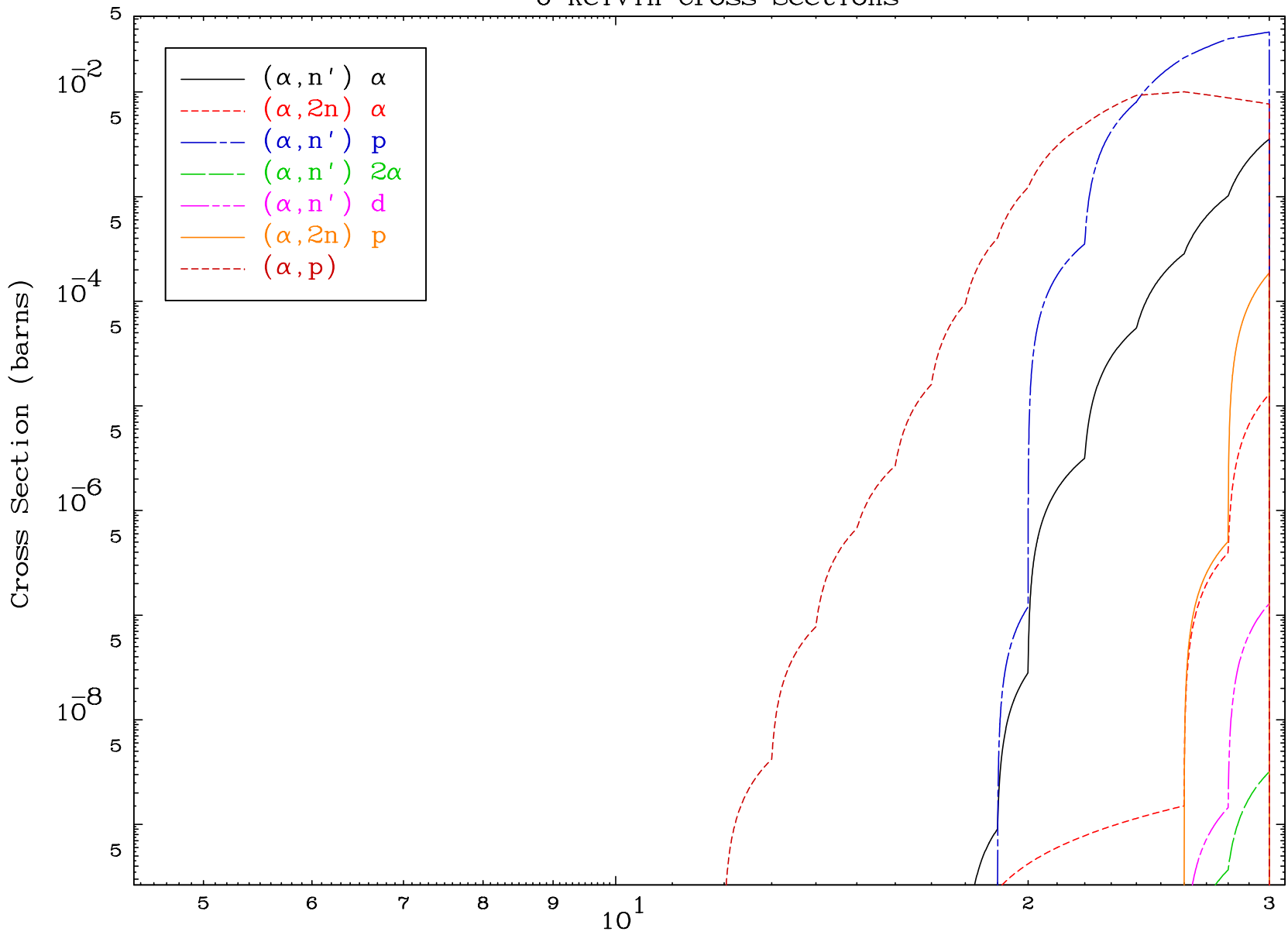
79-Au-187

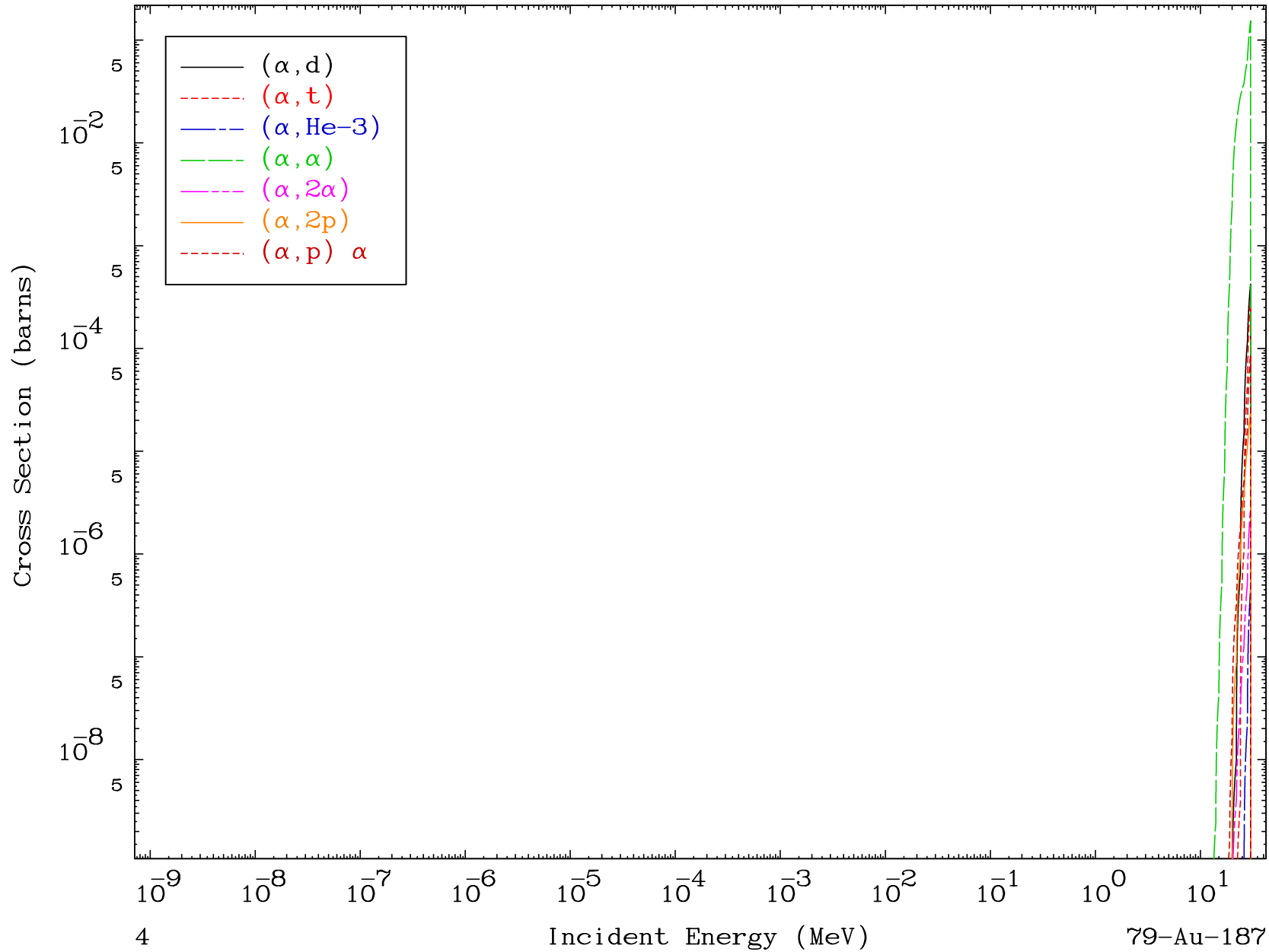


2

Incident Energy (MeV)

79-Au-187

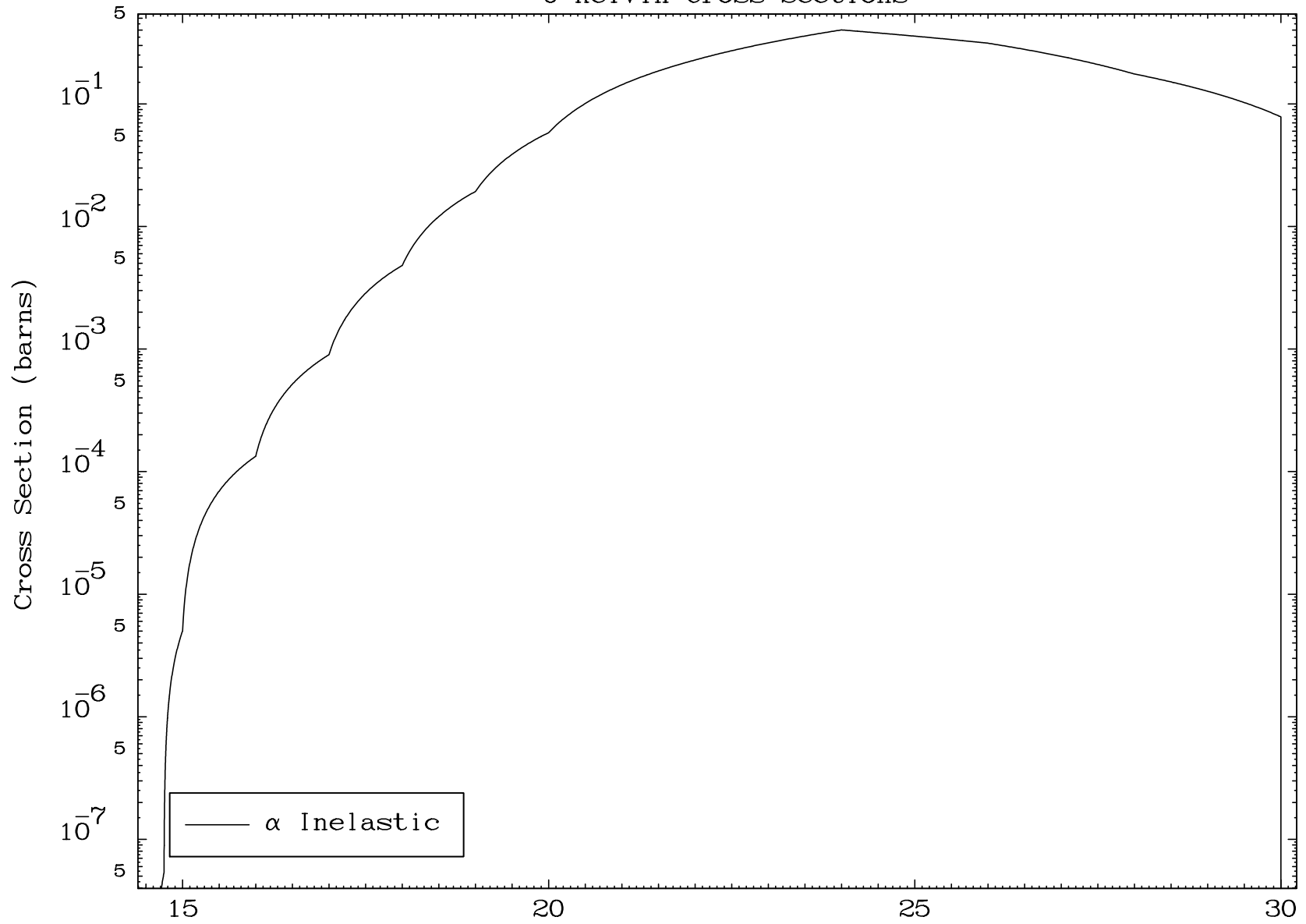




MAT 7895

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

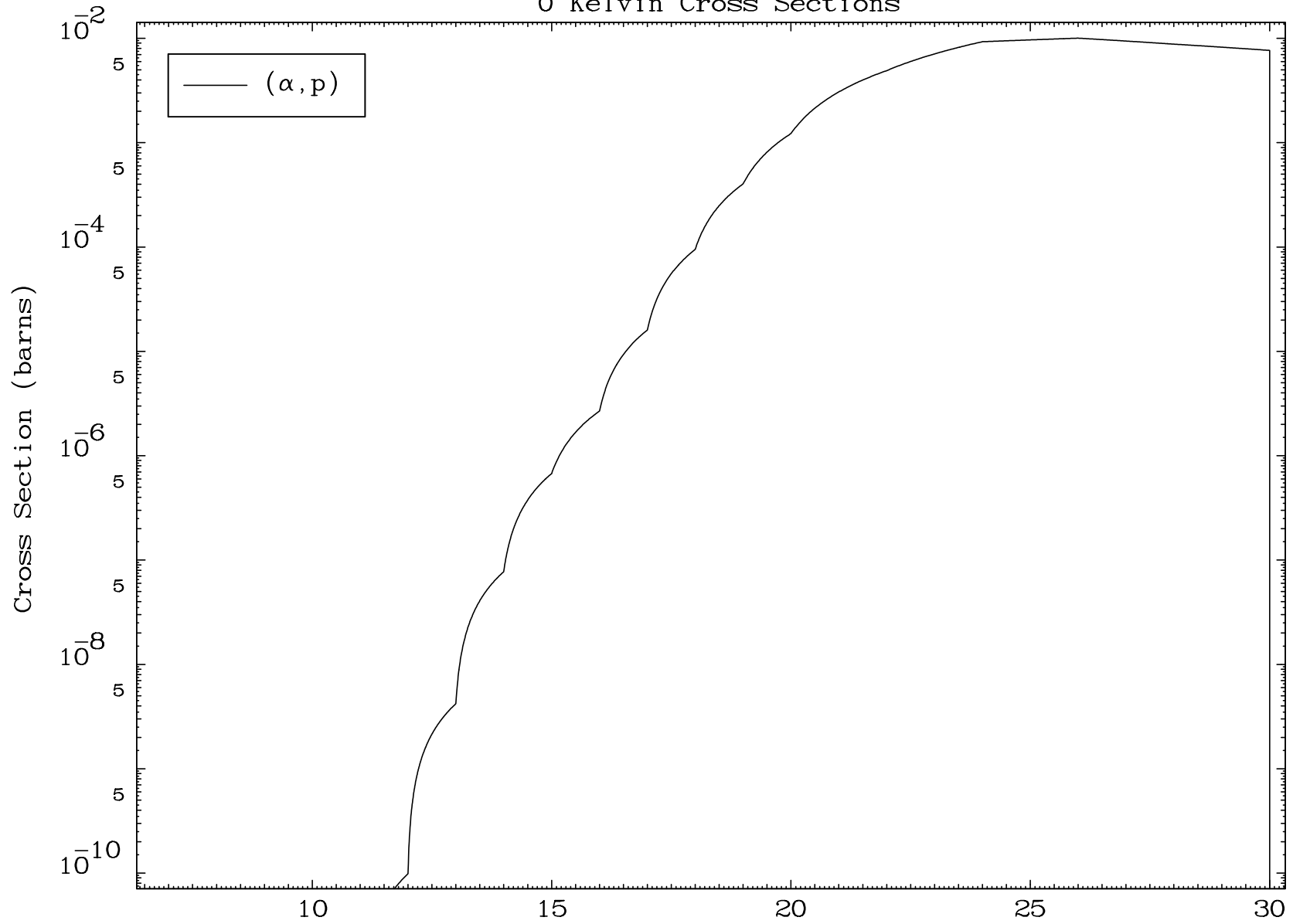
79-Au-187



MAT 7895

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

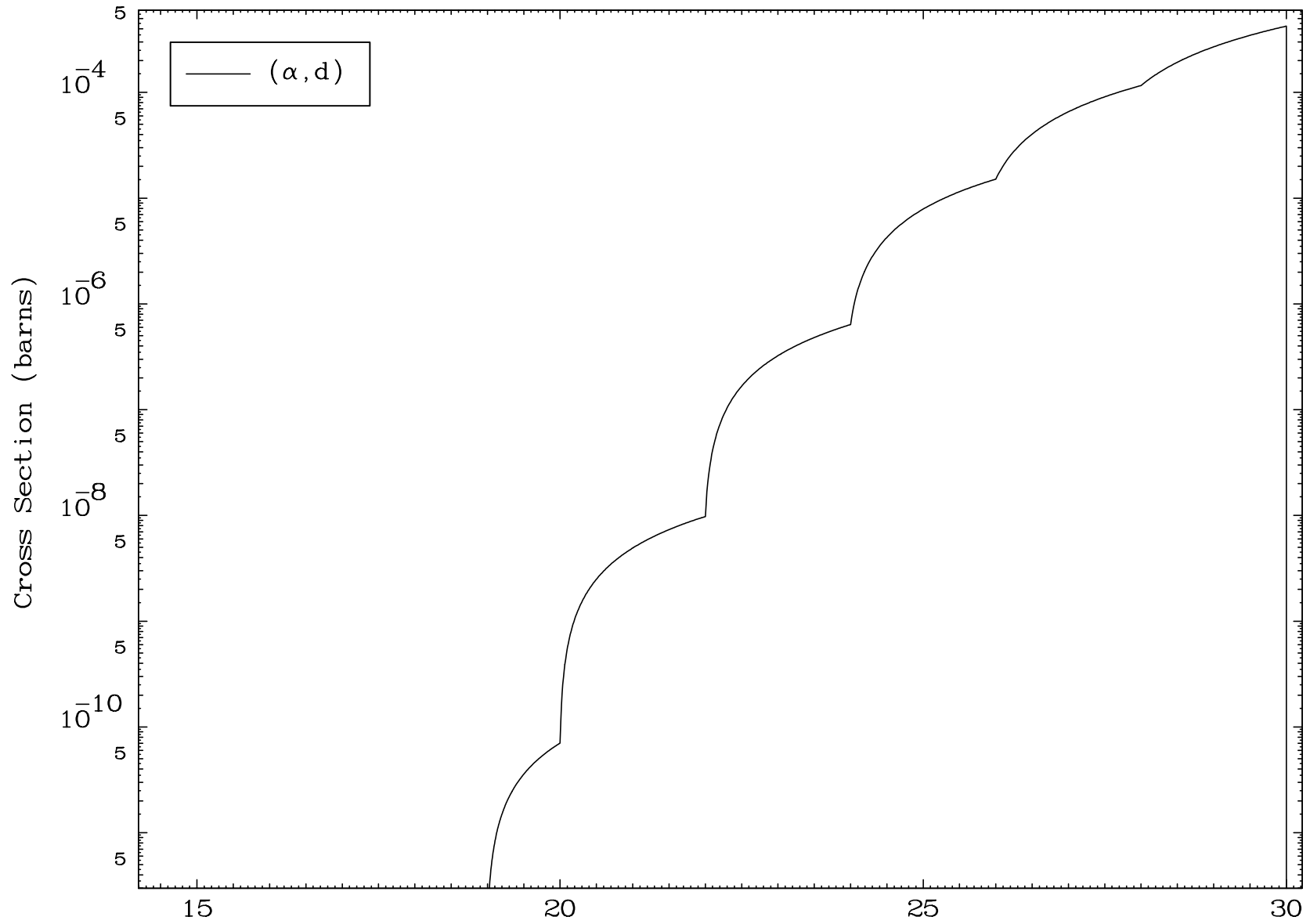
79-Au-187

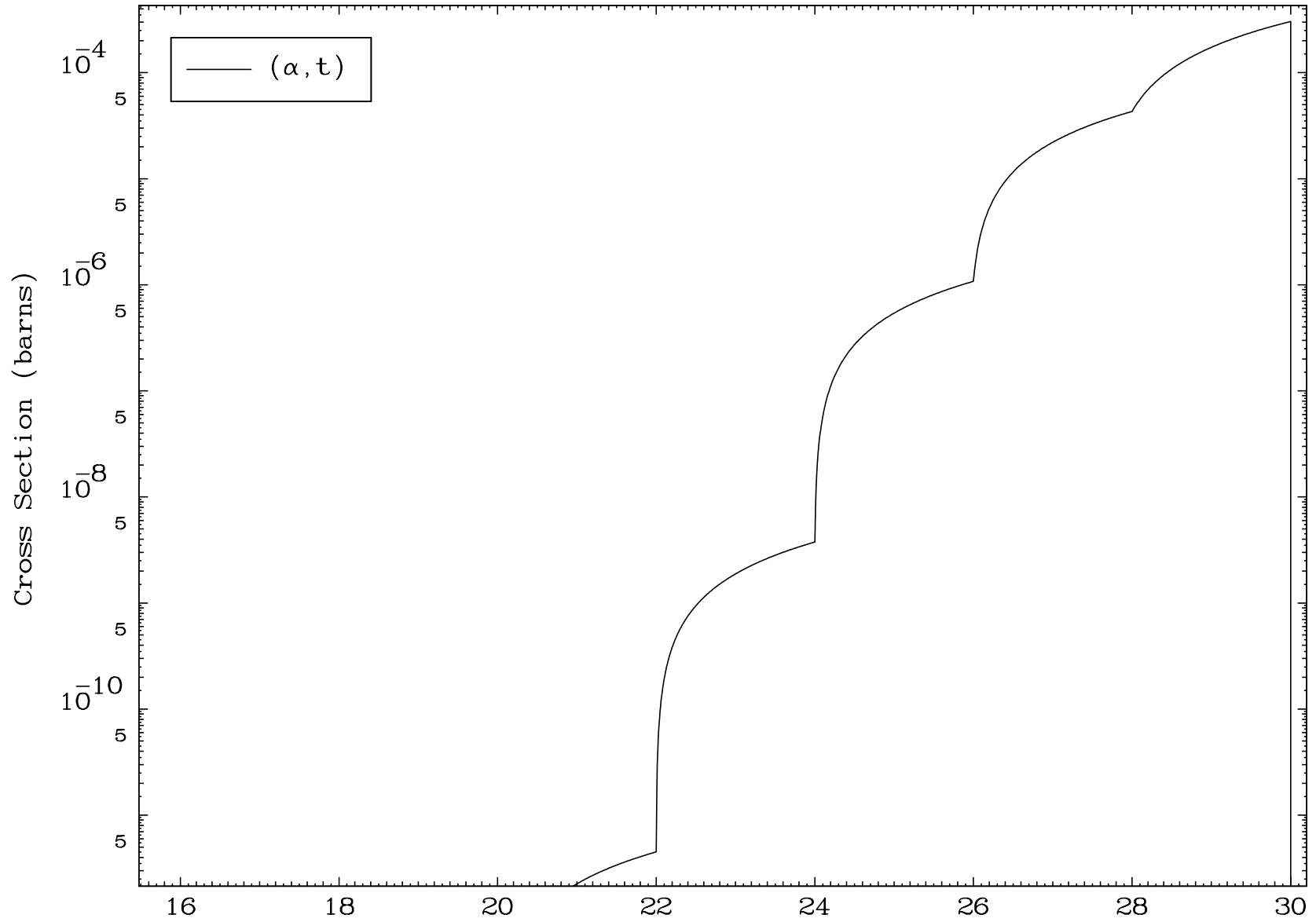


6

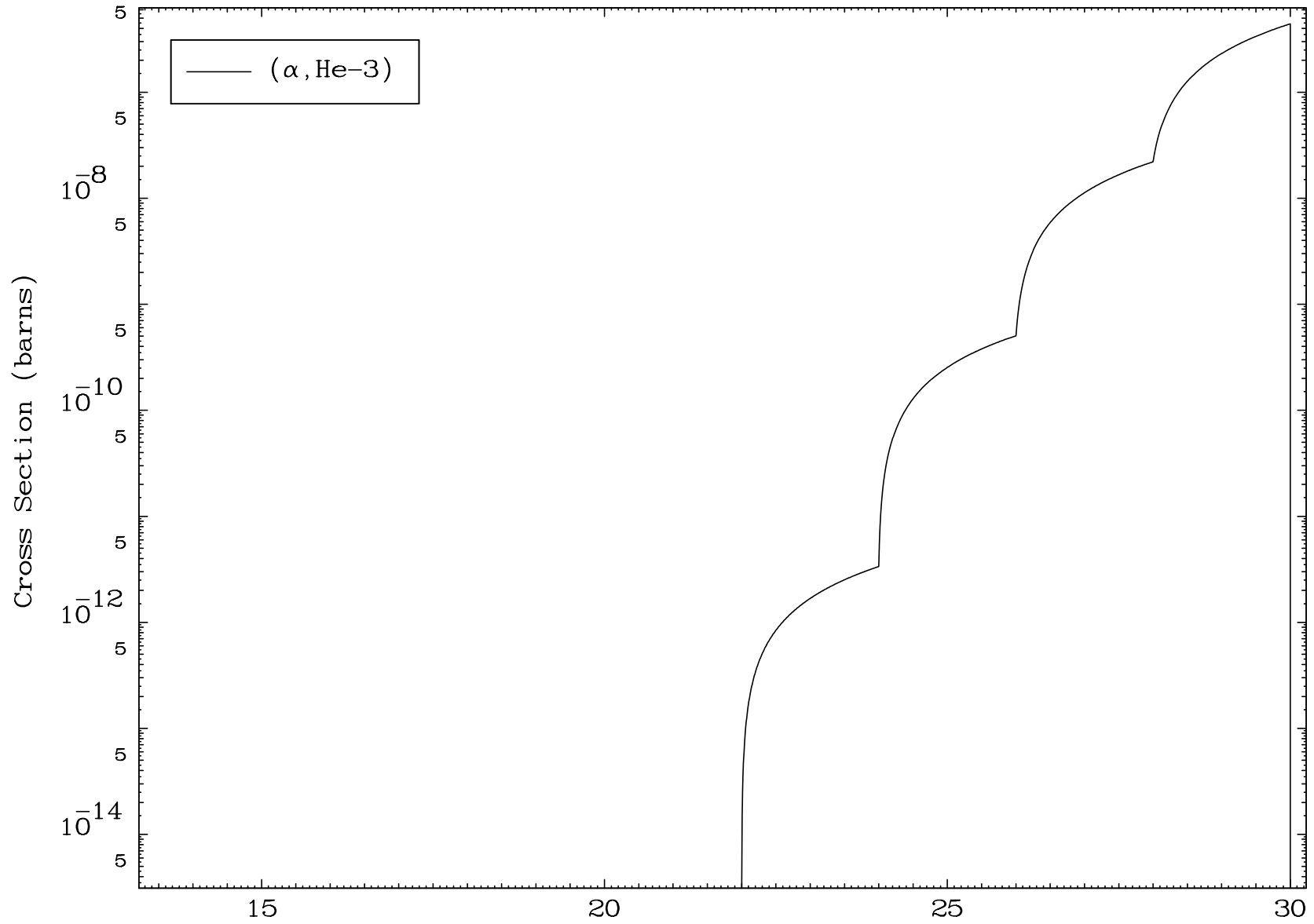
Incident Energy (MeV)

79-Au-187





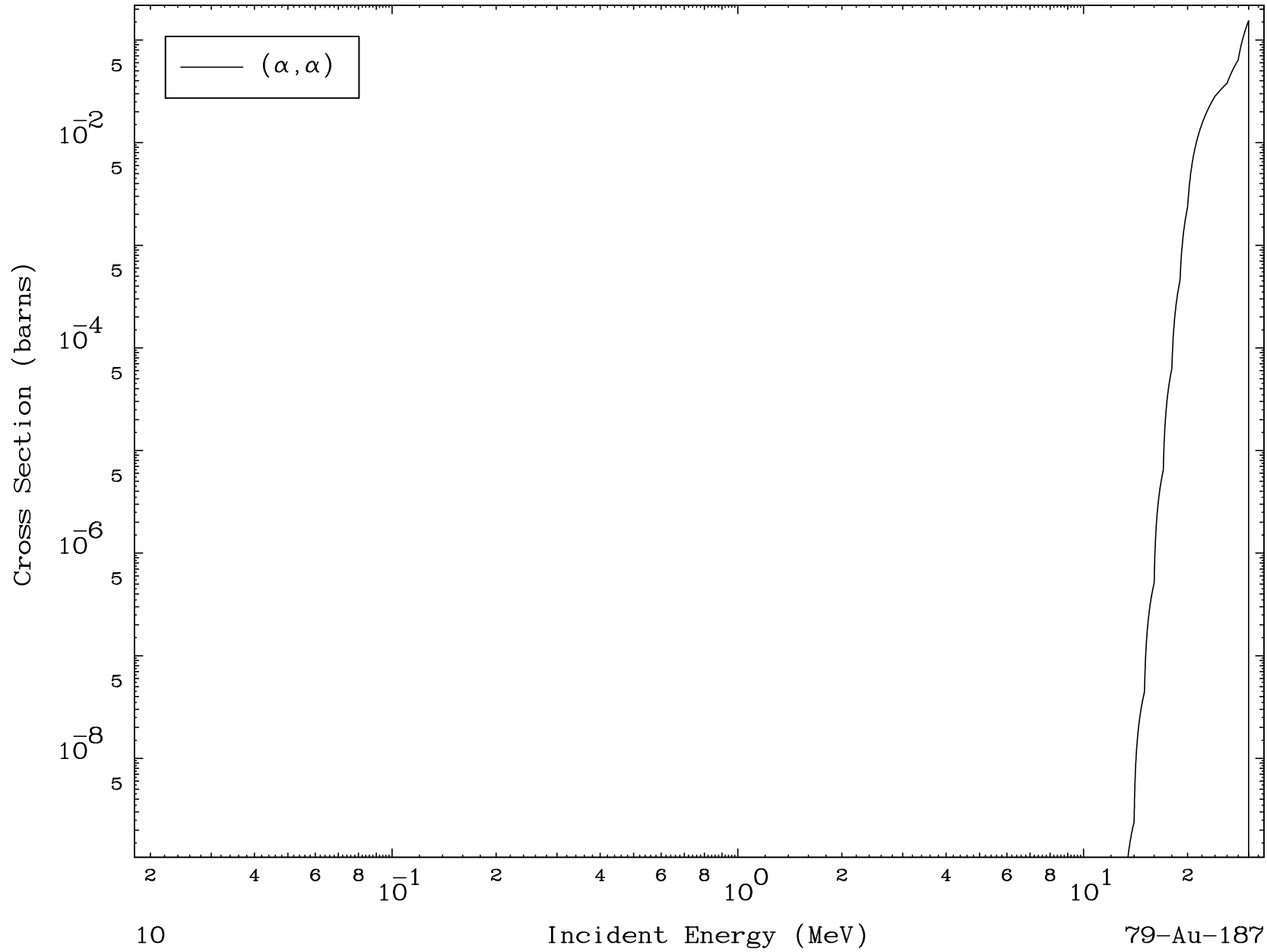




MAT 7895

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-187

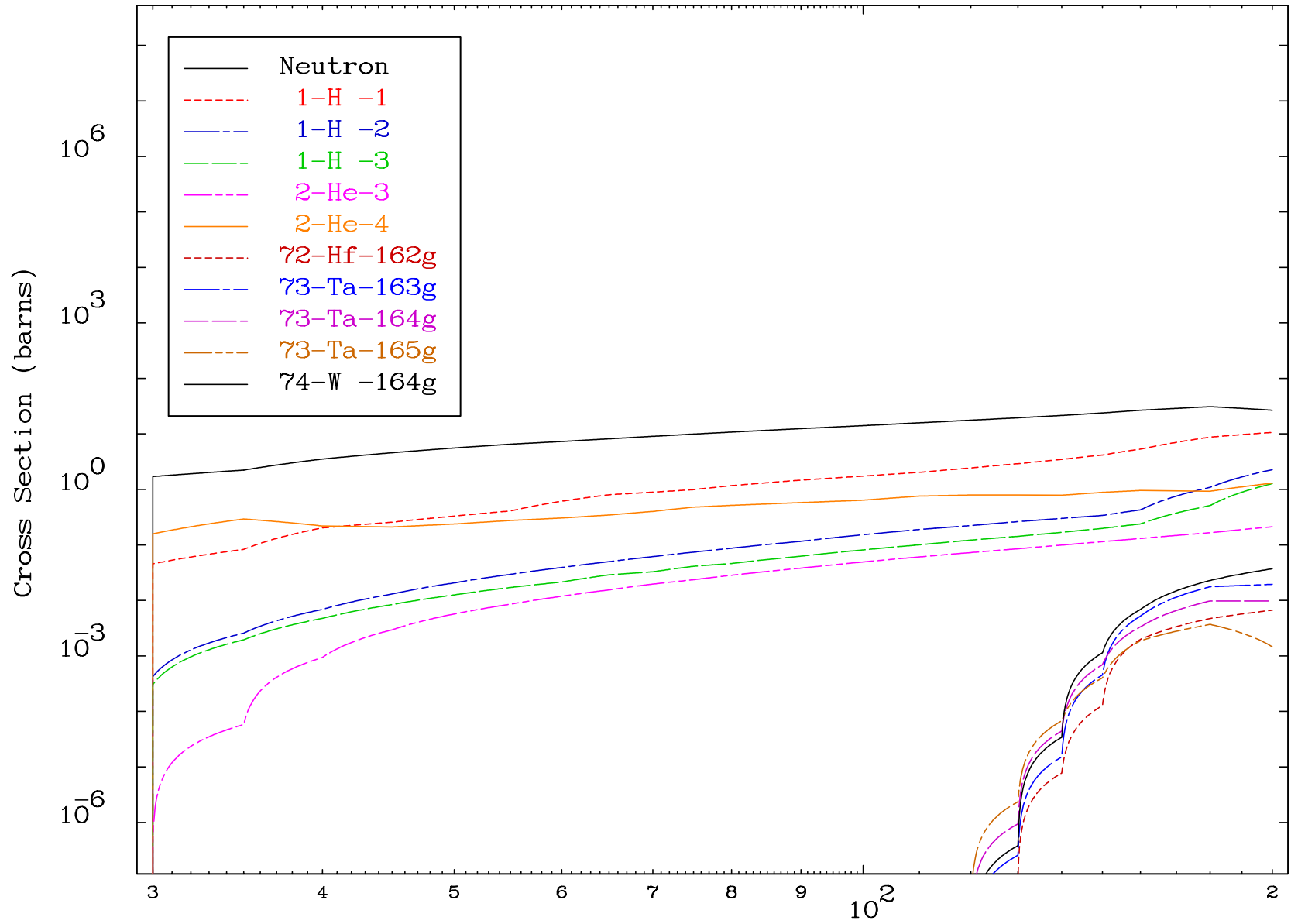


10

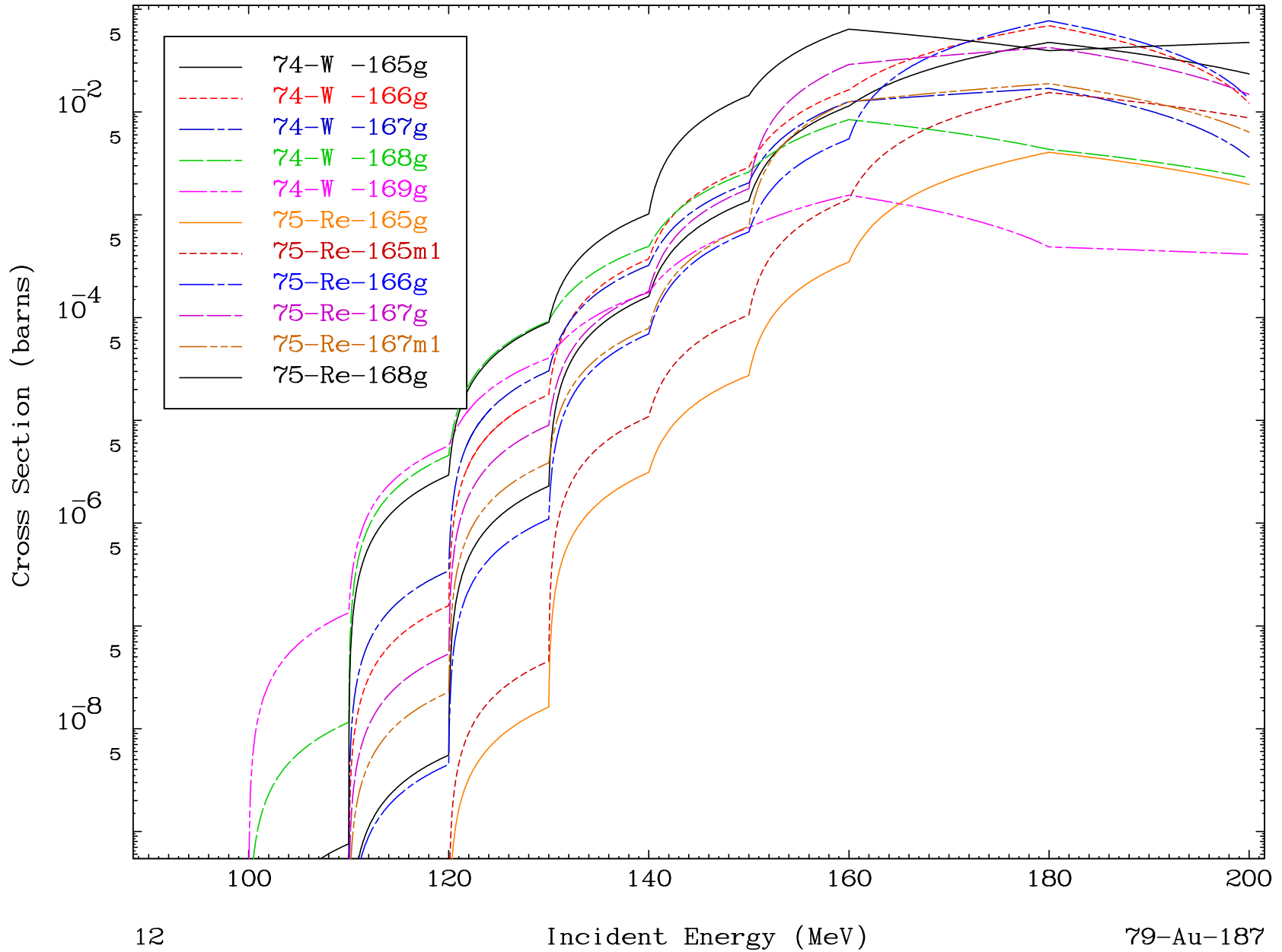
Incident Energy (MeV)

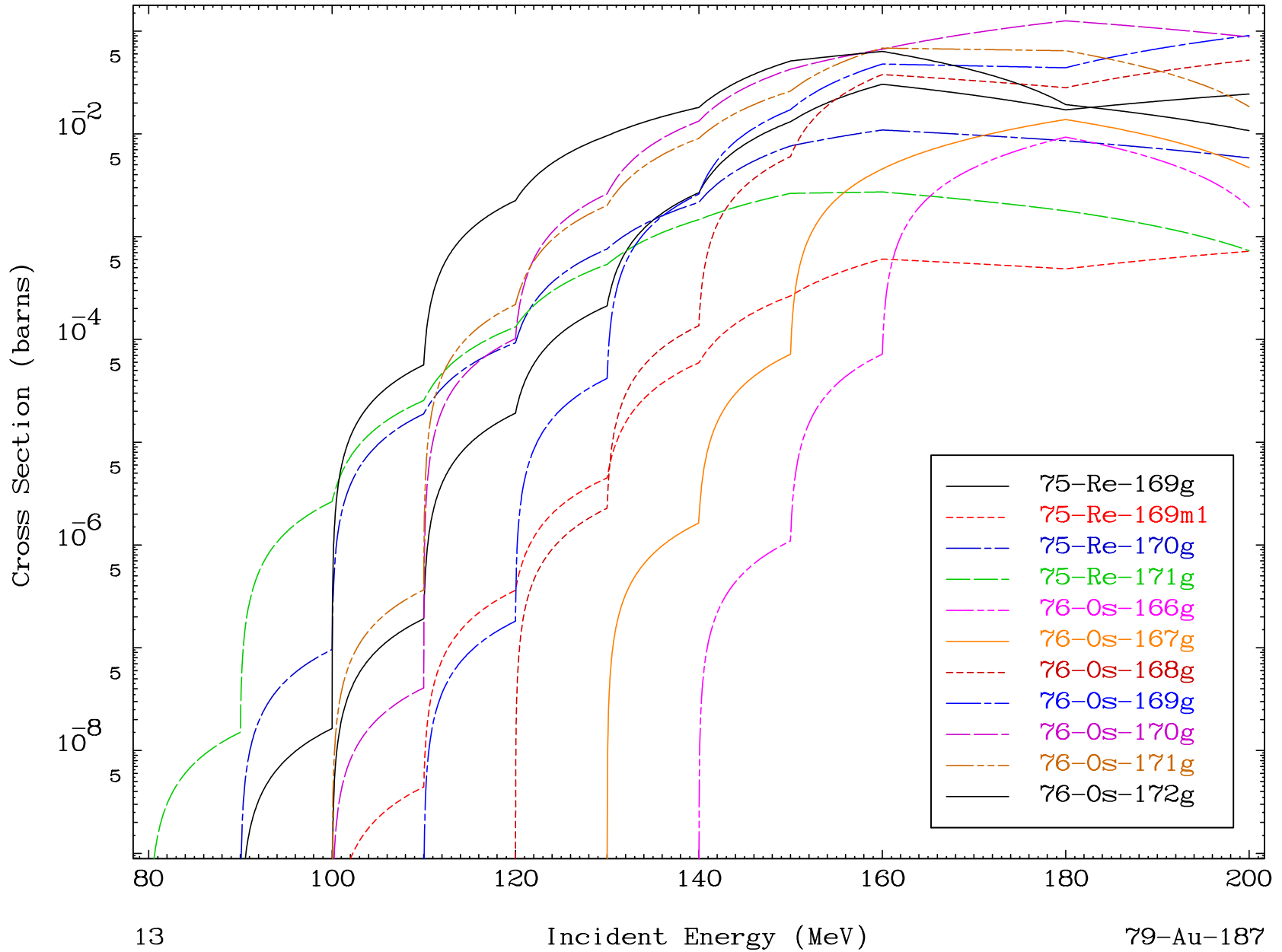
79-Au-187

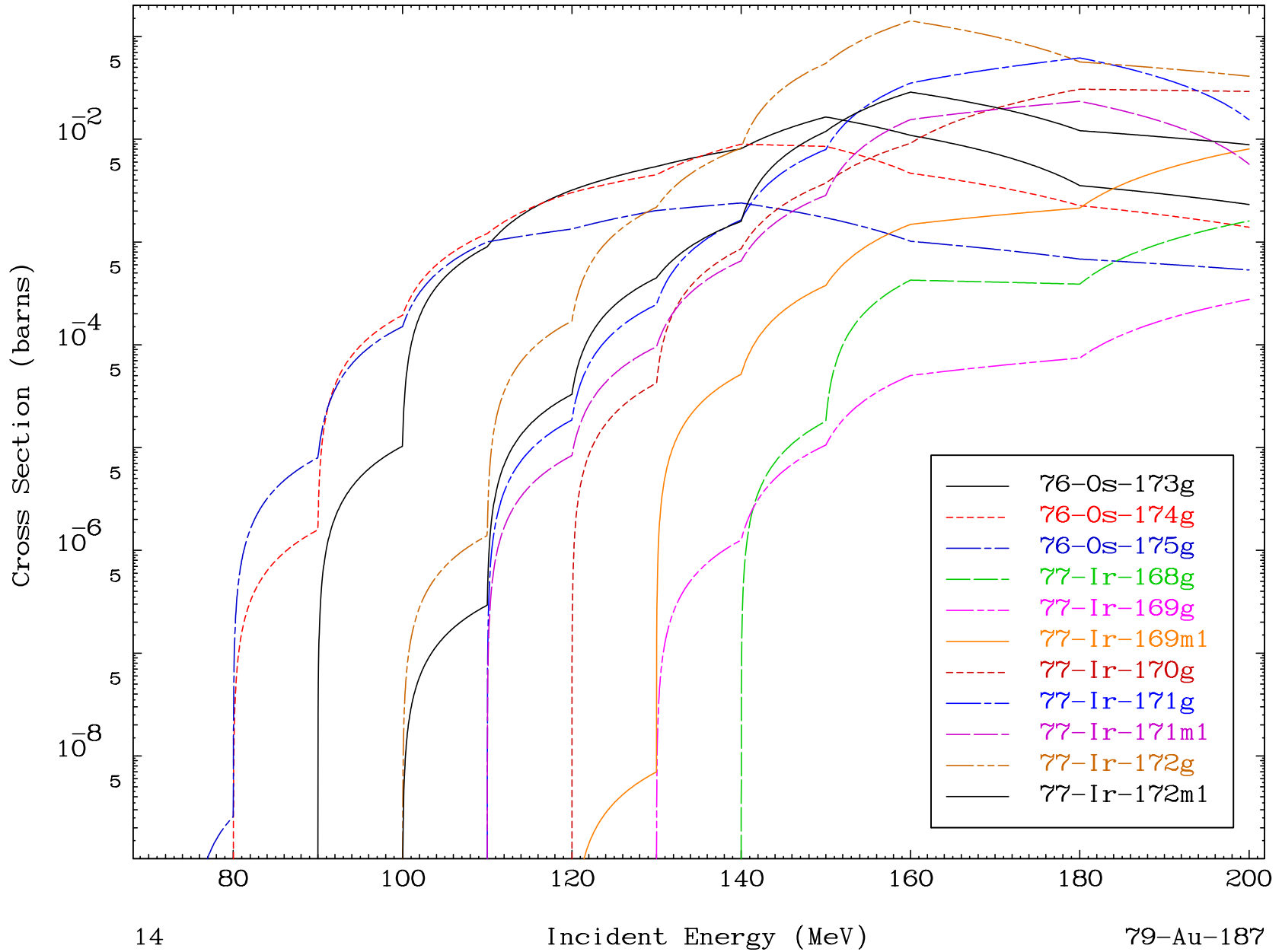
Radionuclide Production Cross Section



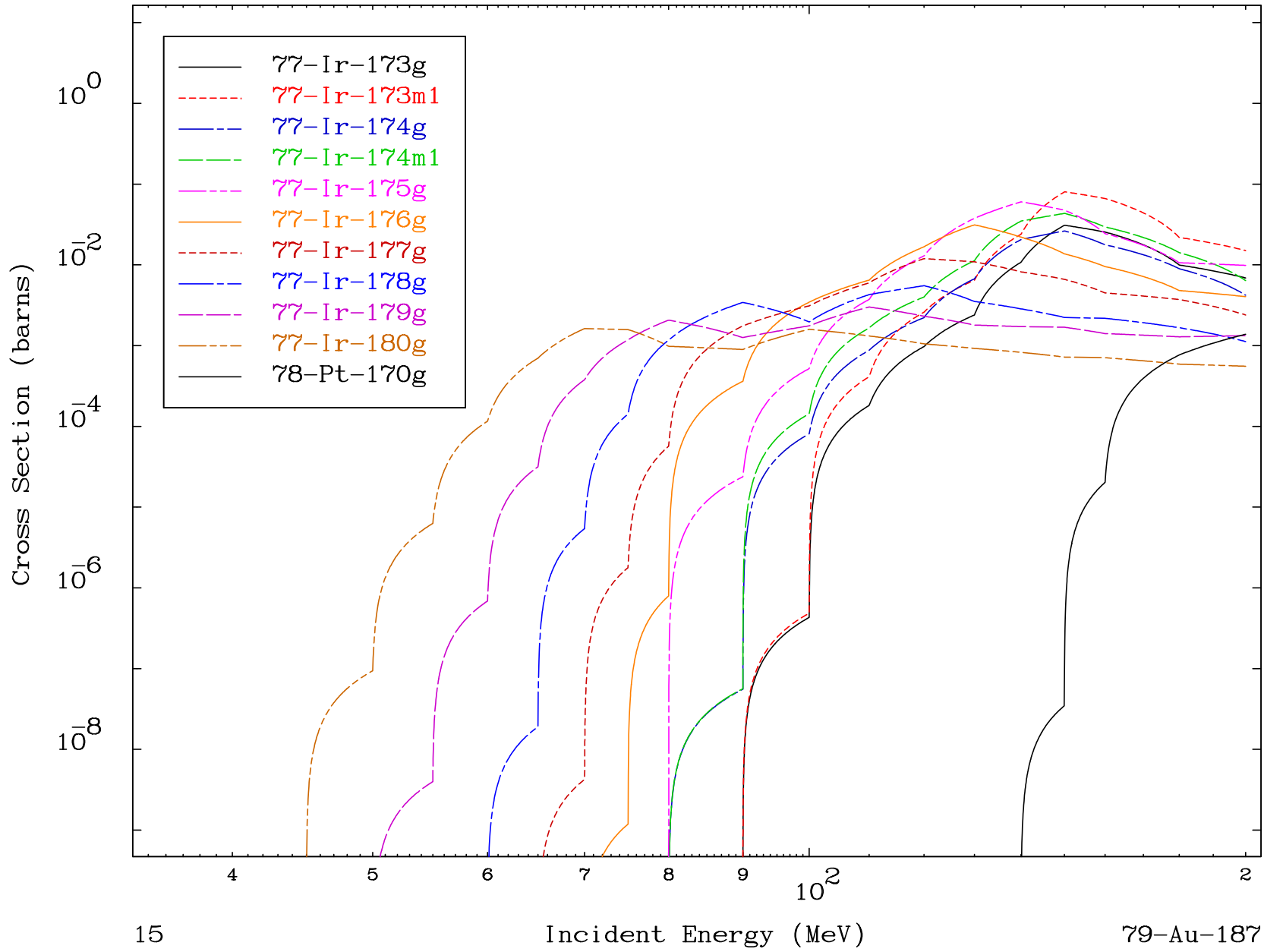
Radionuclide Production Cross Section



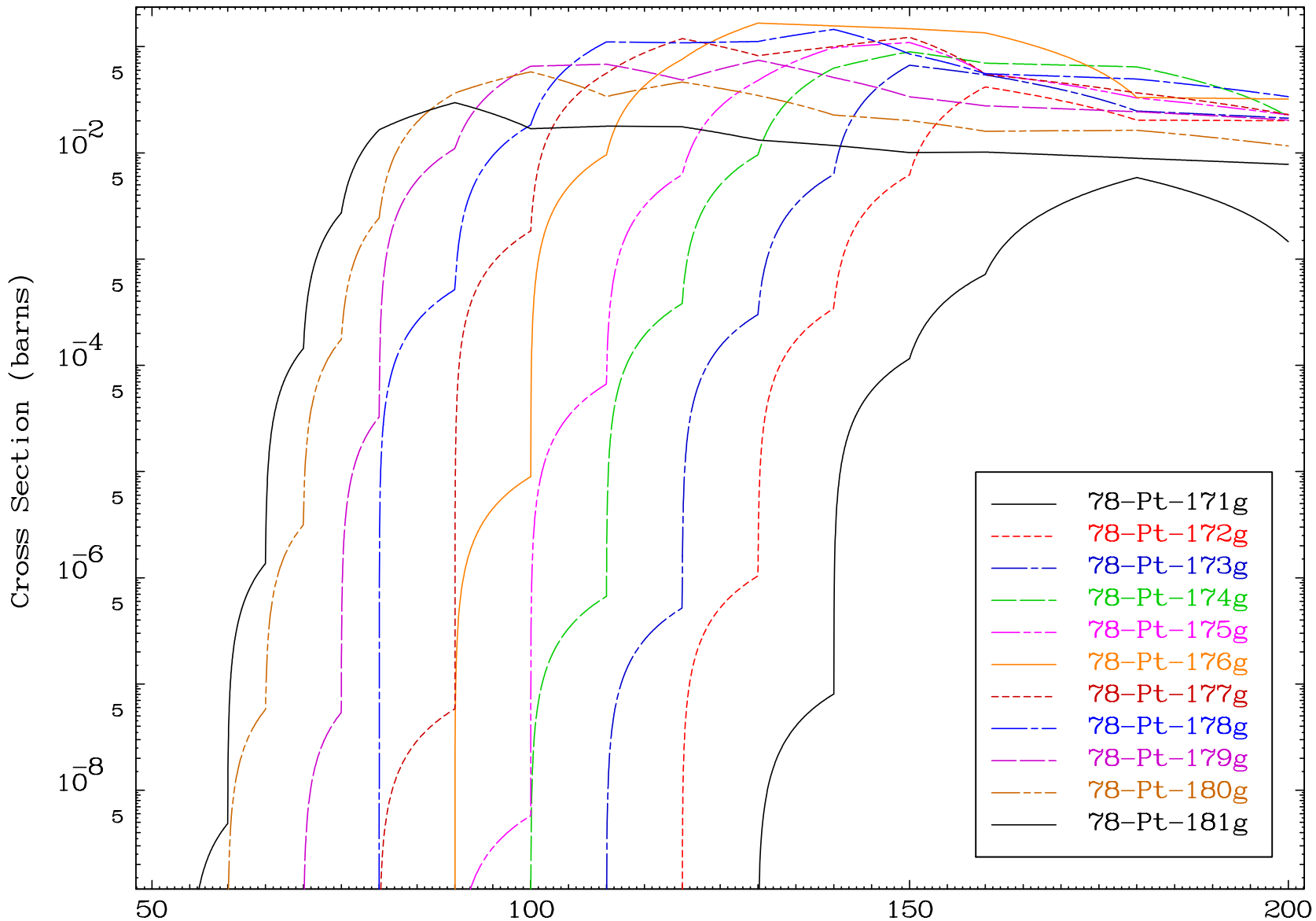




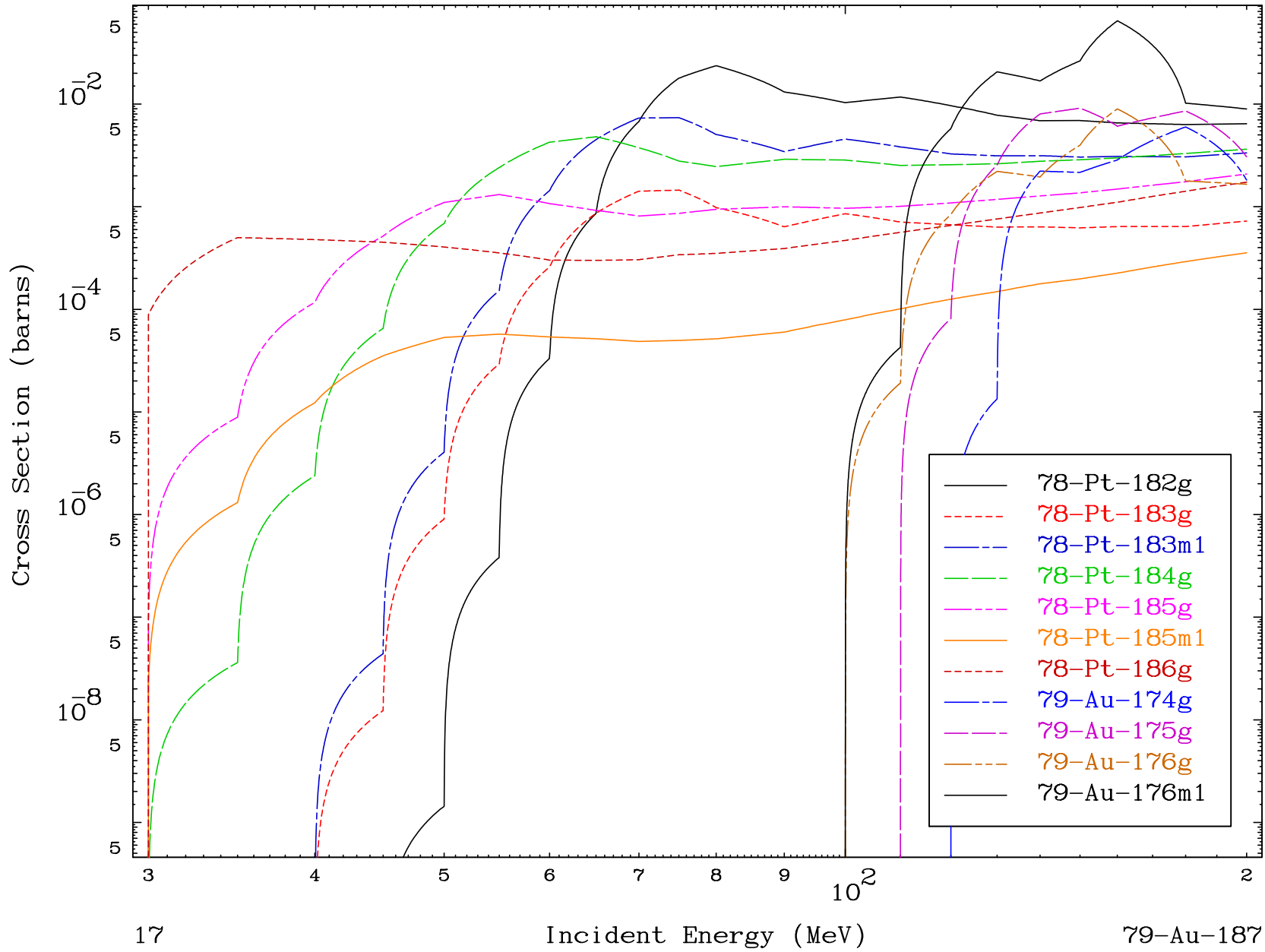
Radionuclide Production Cross Section

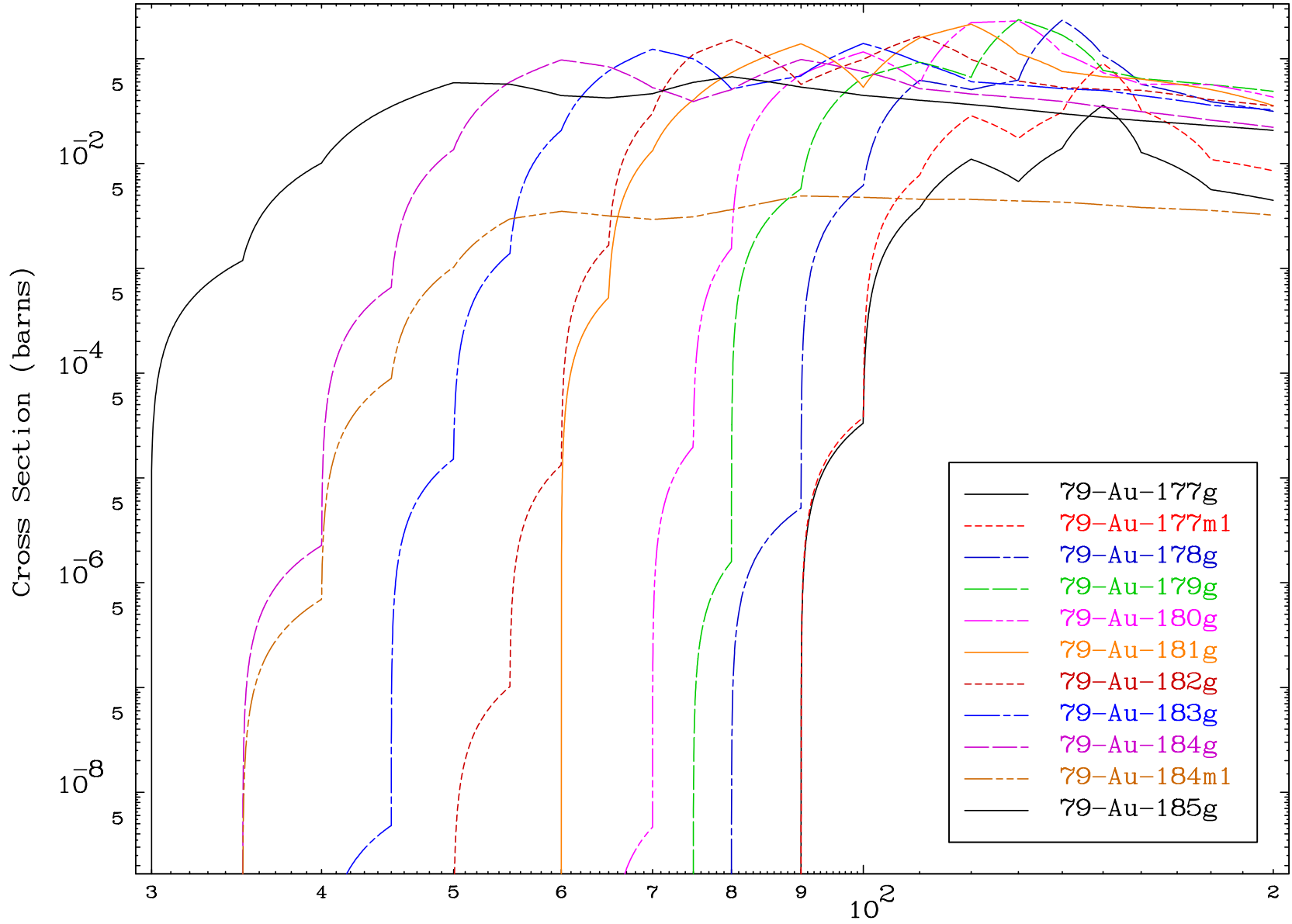


Radionuclide Production Cross Section







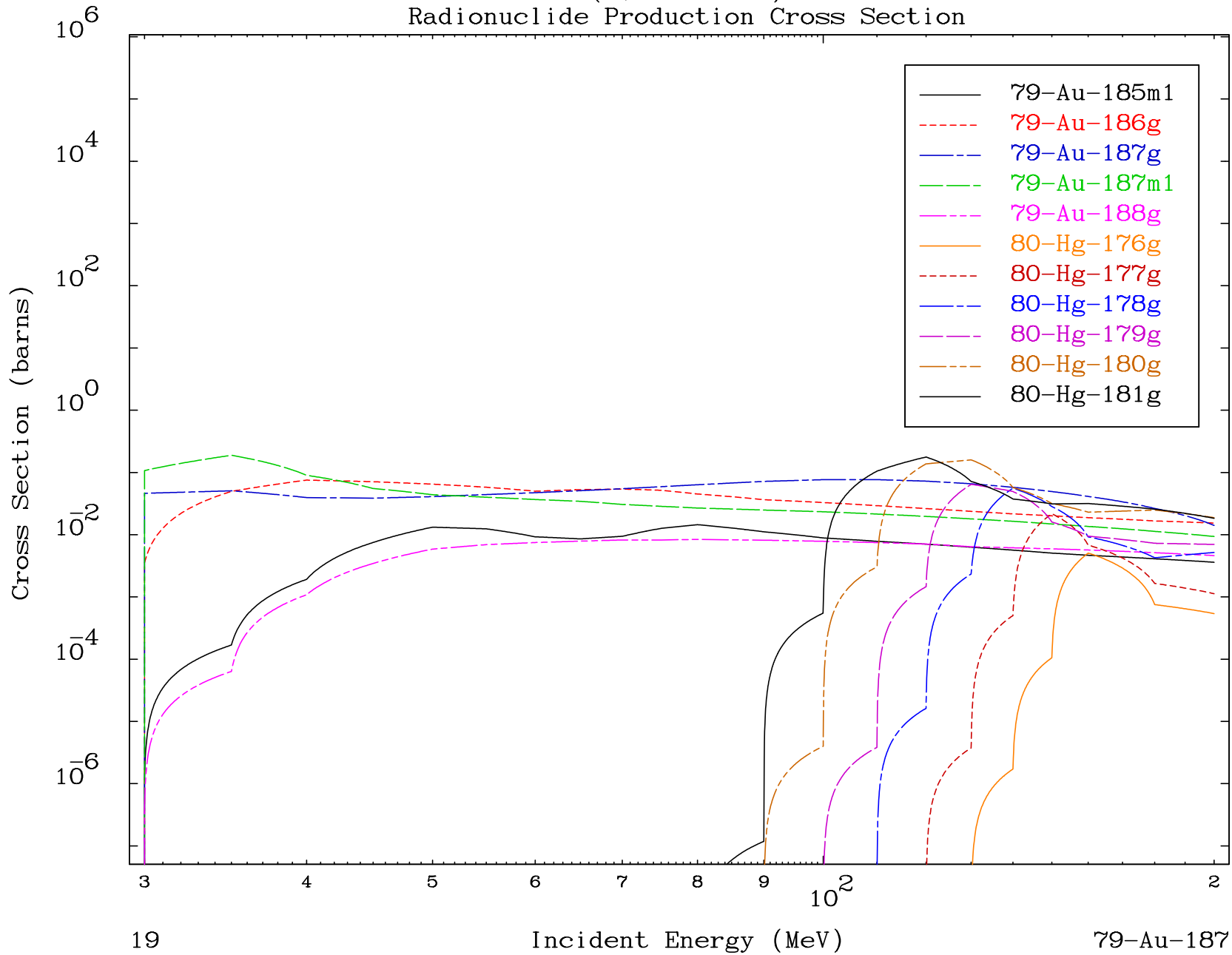


MAT 7895

( $\alpha$ , remainder)

79-Au-187

### Radionuclide Production Cross Section

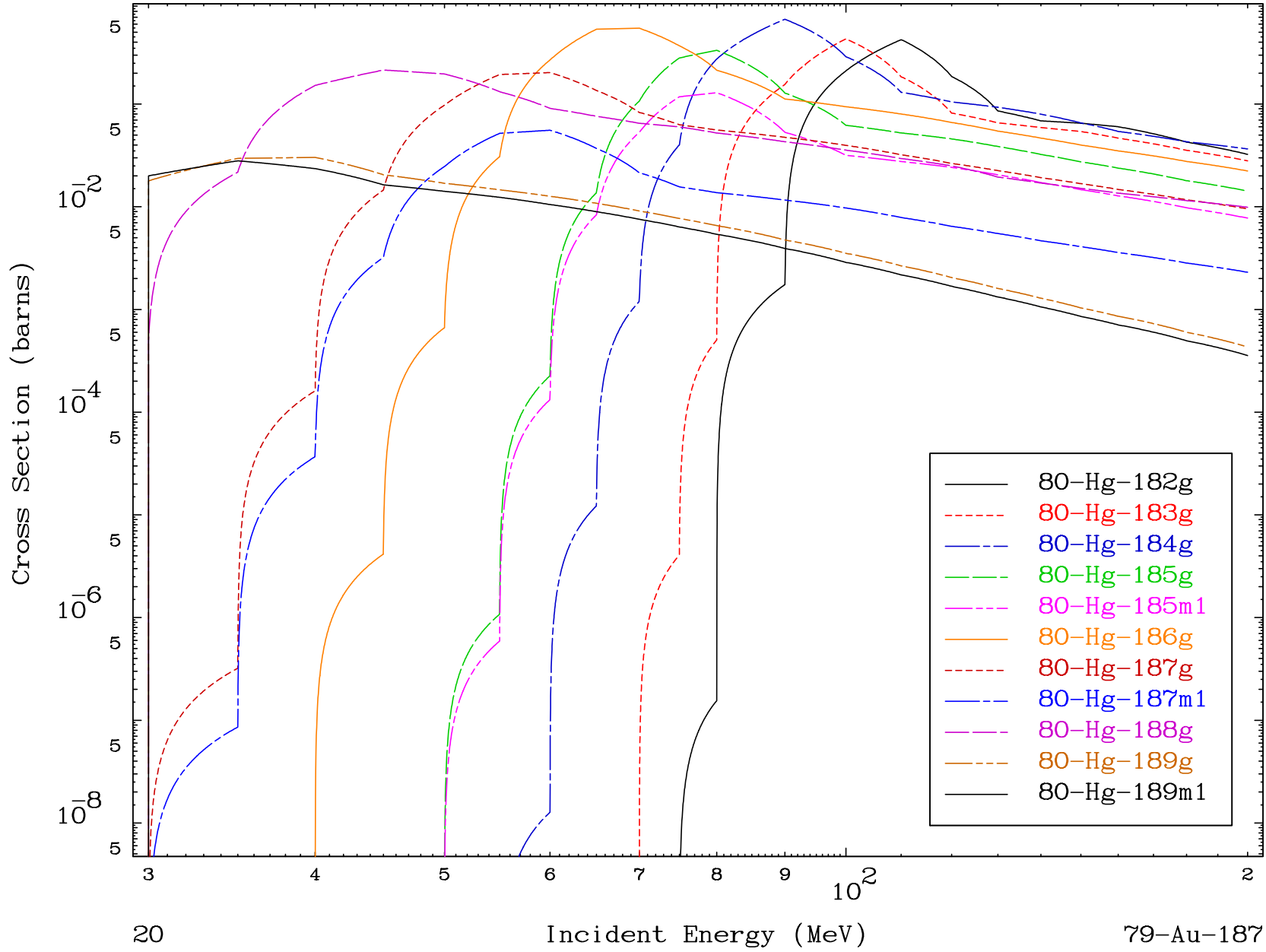


19

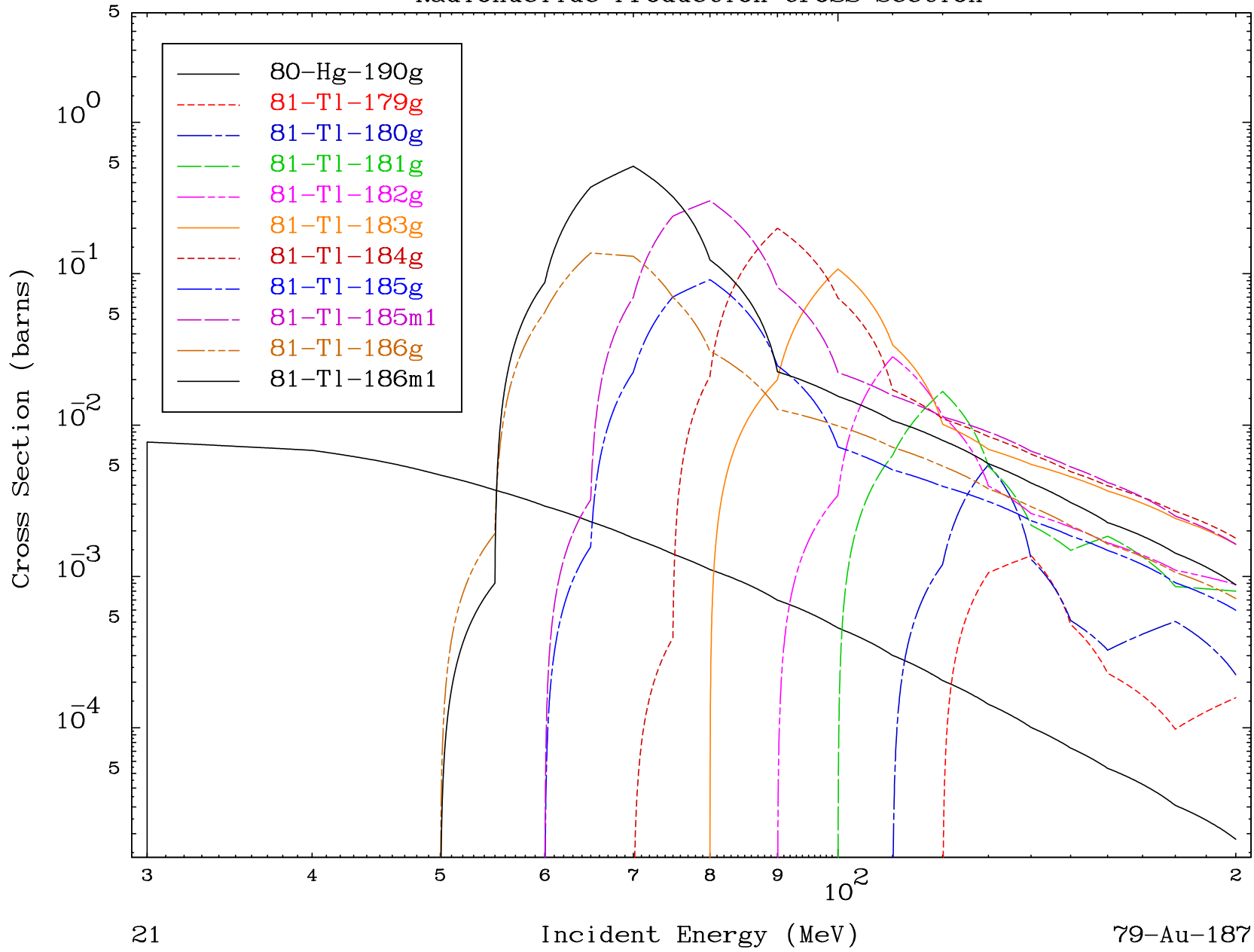
Incident Energy (MeV)

79-Au-187

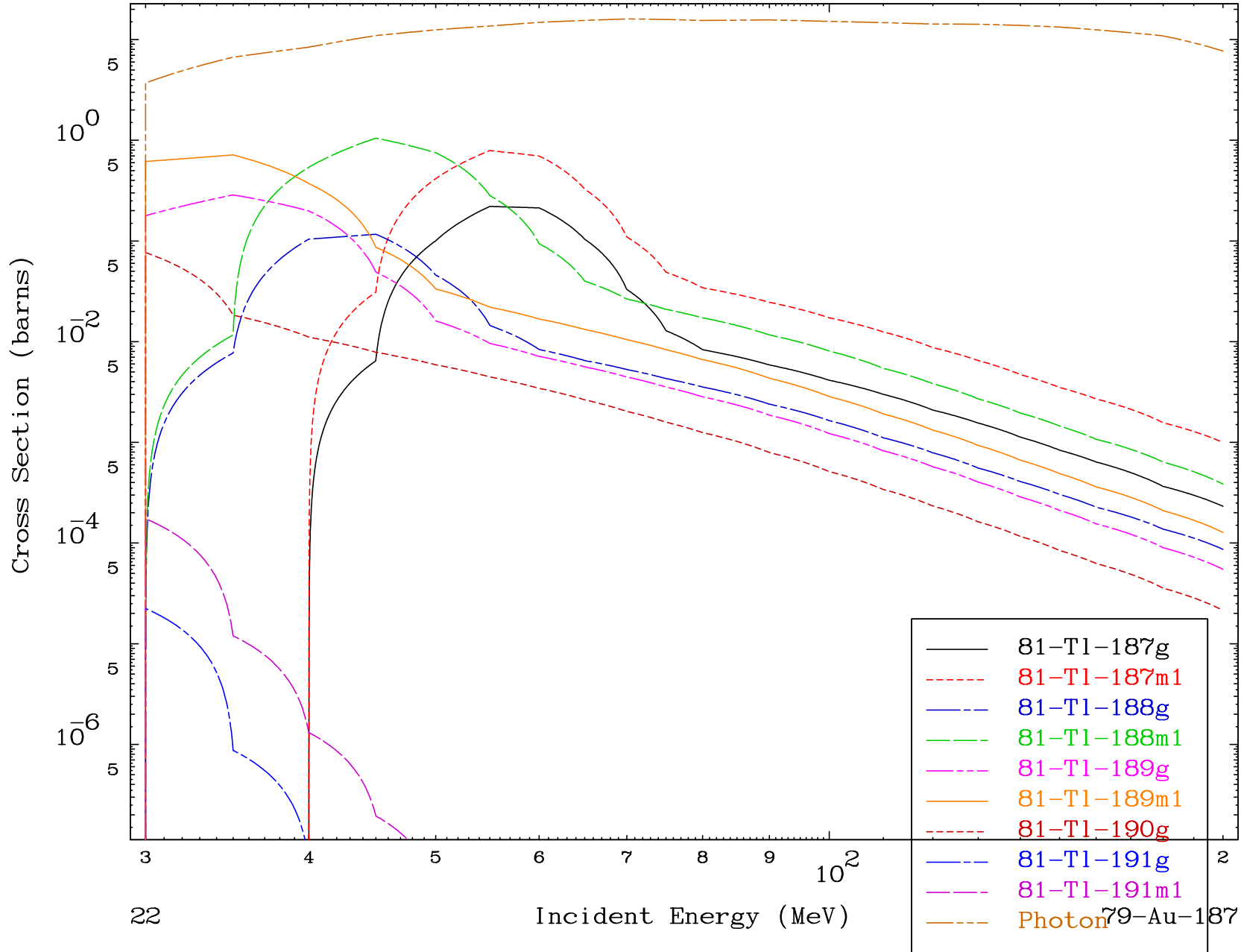
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

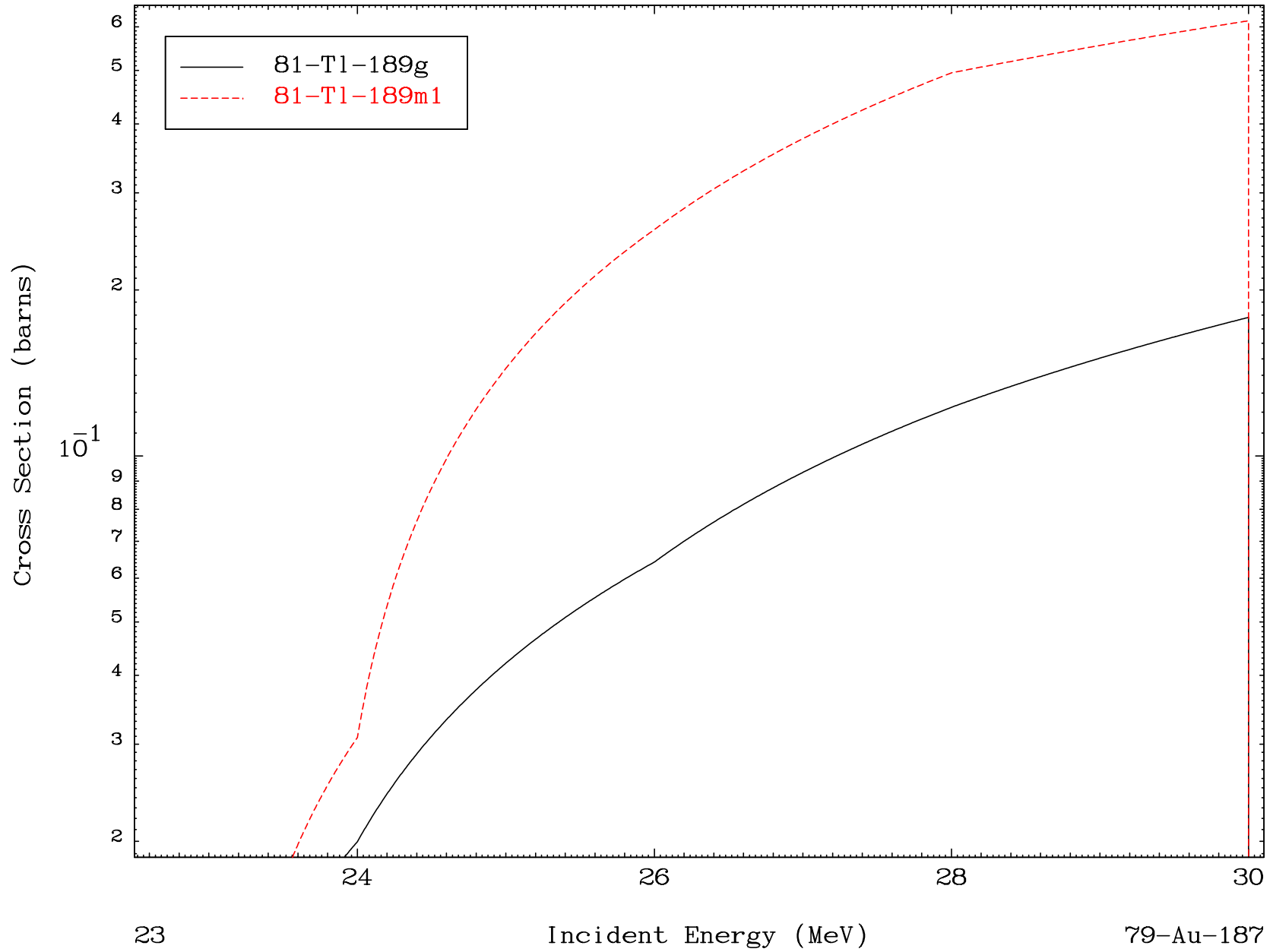


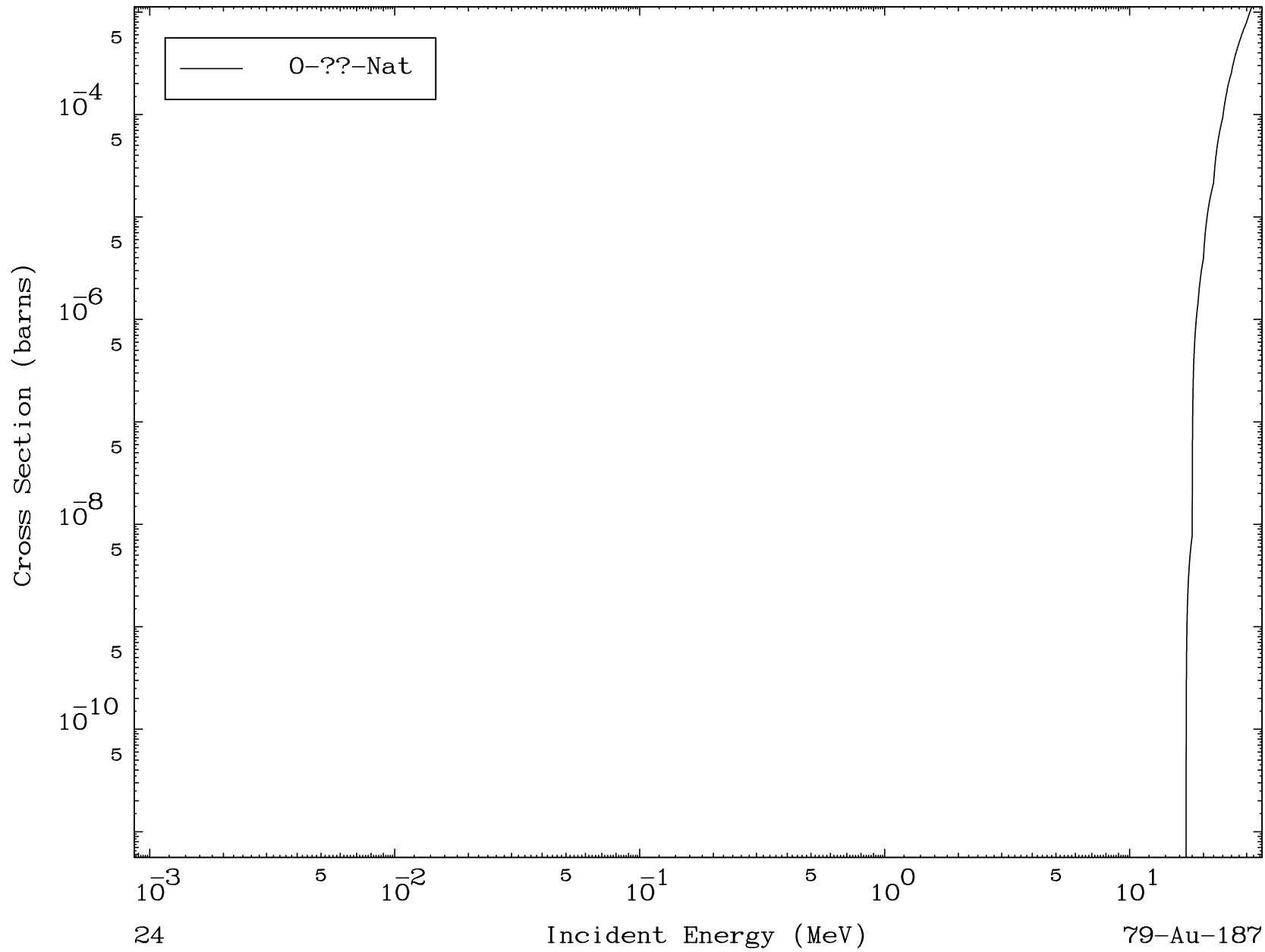
MAT 7895

( $\alpha, 2n$ )

<sup>79</sup>Au-187

### Radionuclide Production Cross Section





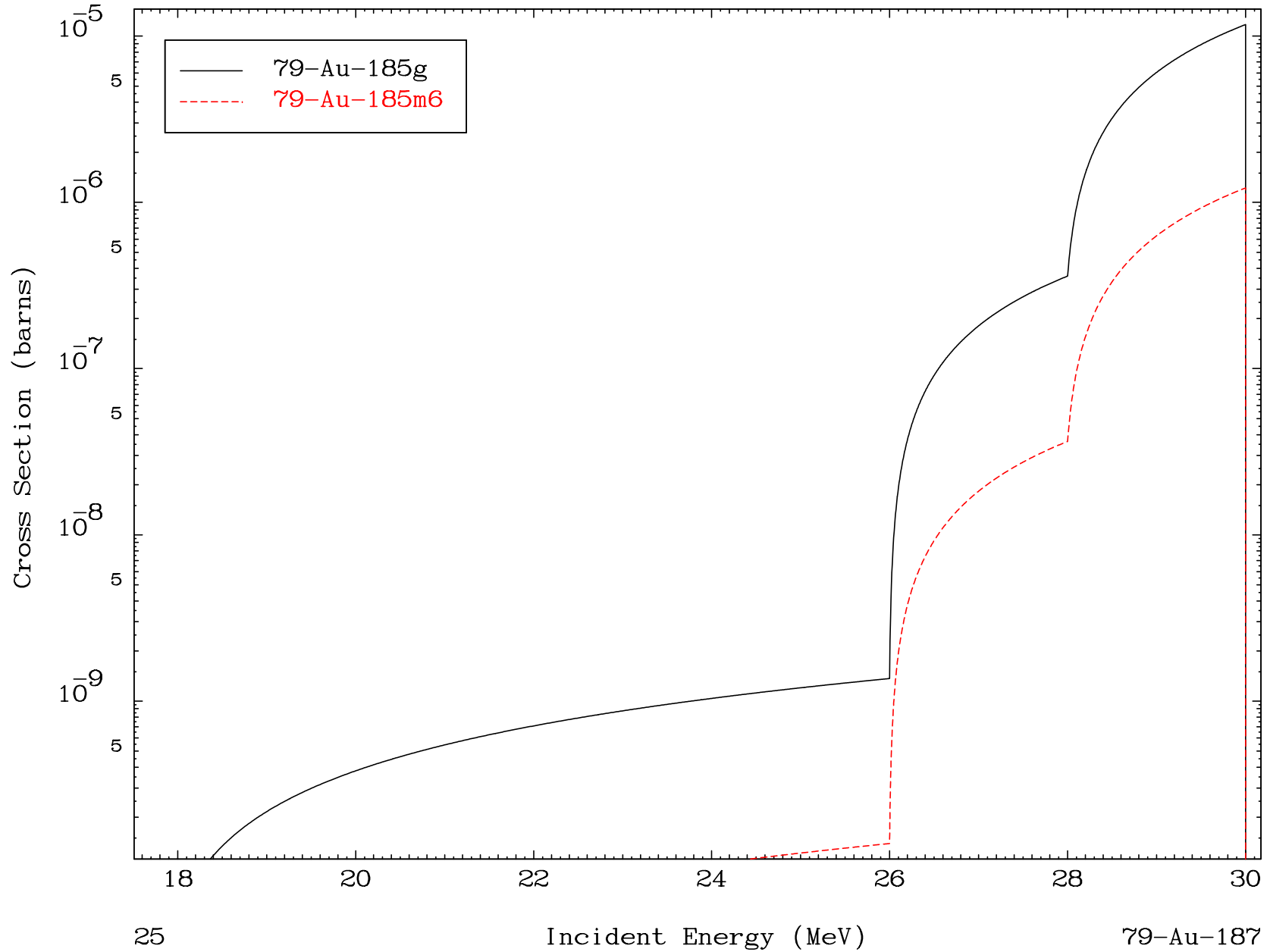


MAT 7895

( $\alpha, 2n$ )  $\alpha$

79-Au-187

Radionuclide Production Cross Section

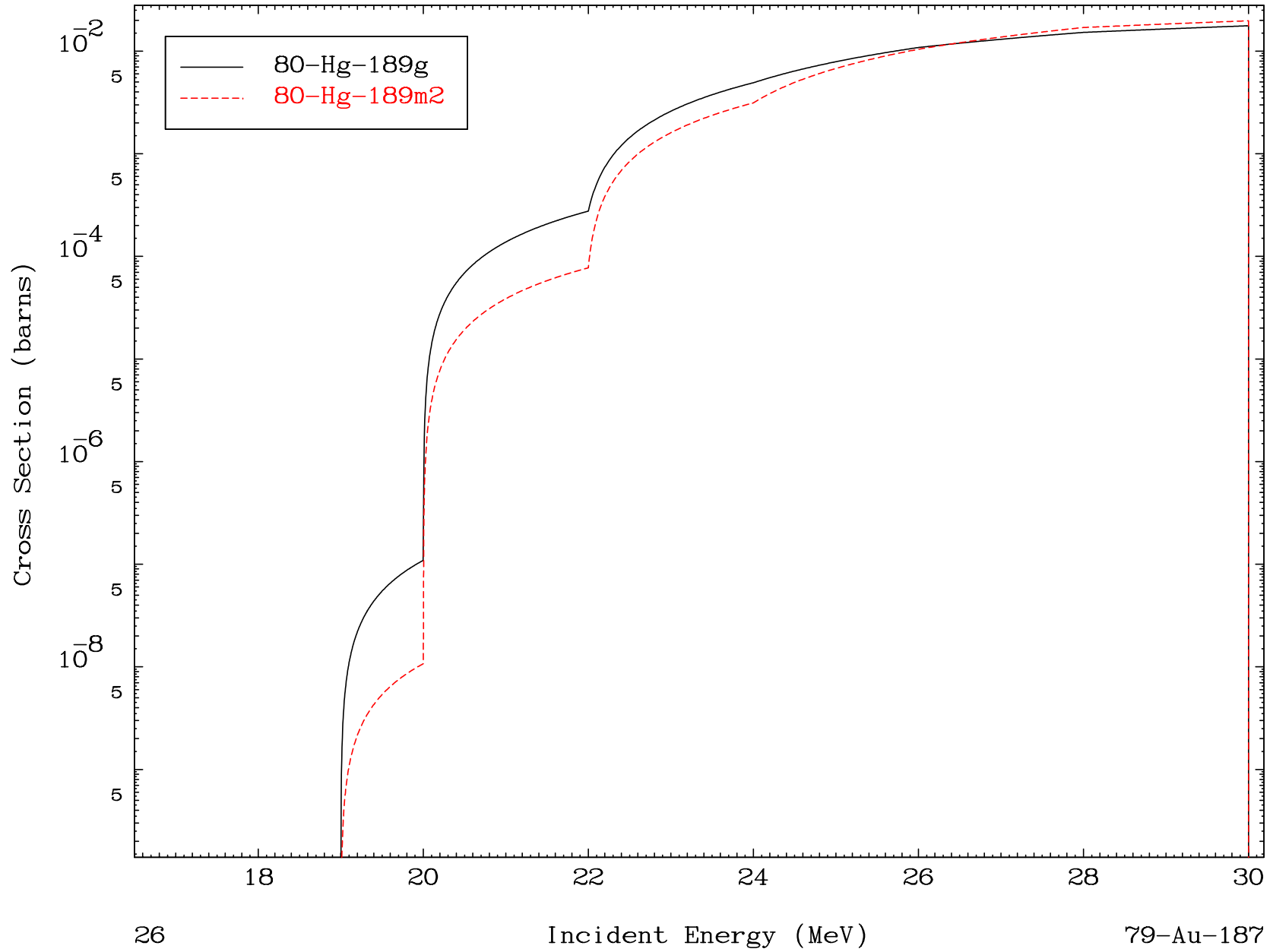


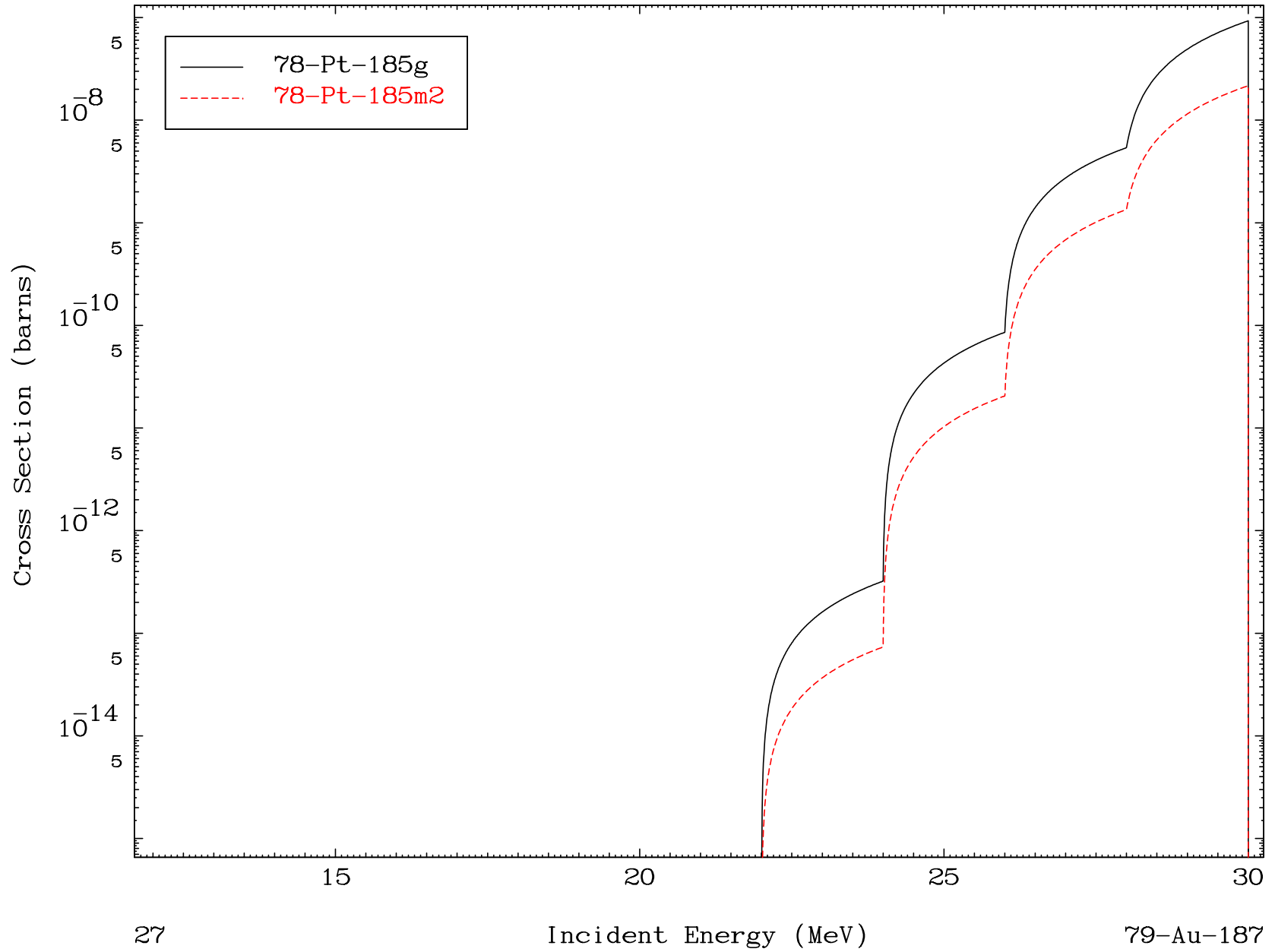
MAT 7895

( $\alpha, n'$ ) p

79-Au-187

Radionuclide Production Cross Section



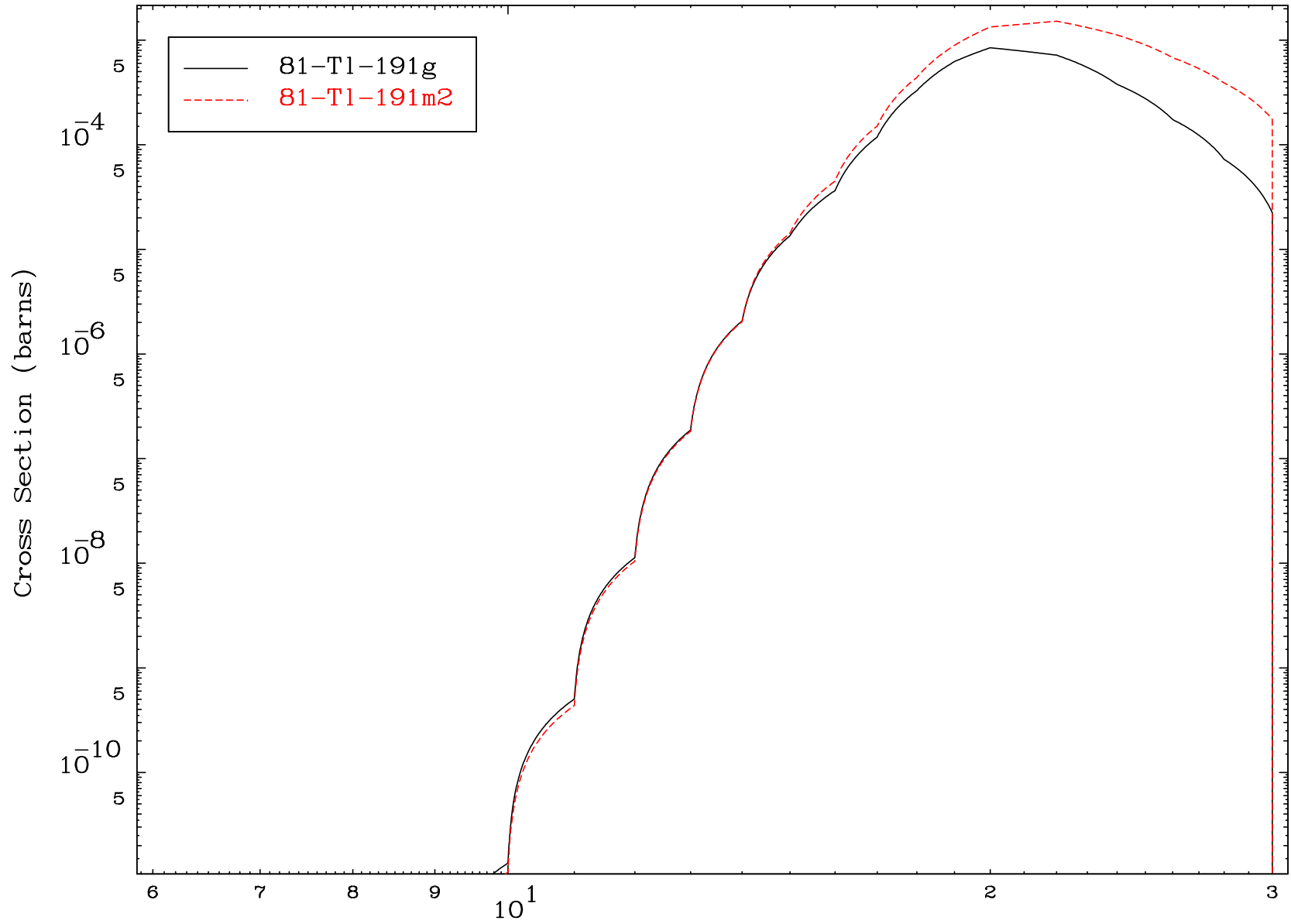


MAT 7895

( $\alpha, \gamma$ )

79-Au-187

### Radionuclide Production Cross Section



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Incident Energy (MeV)

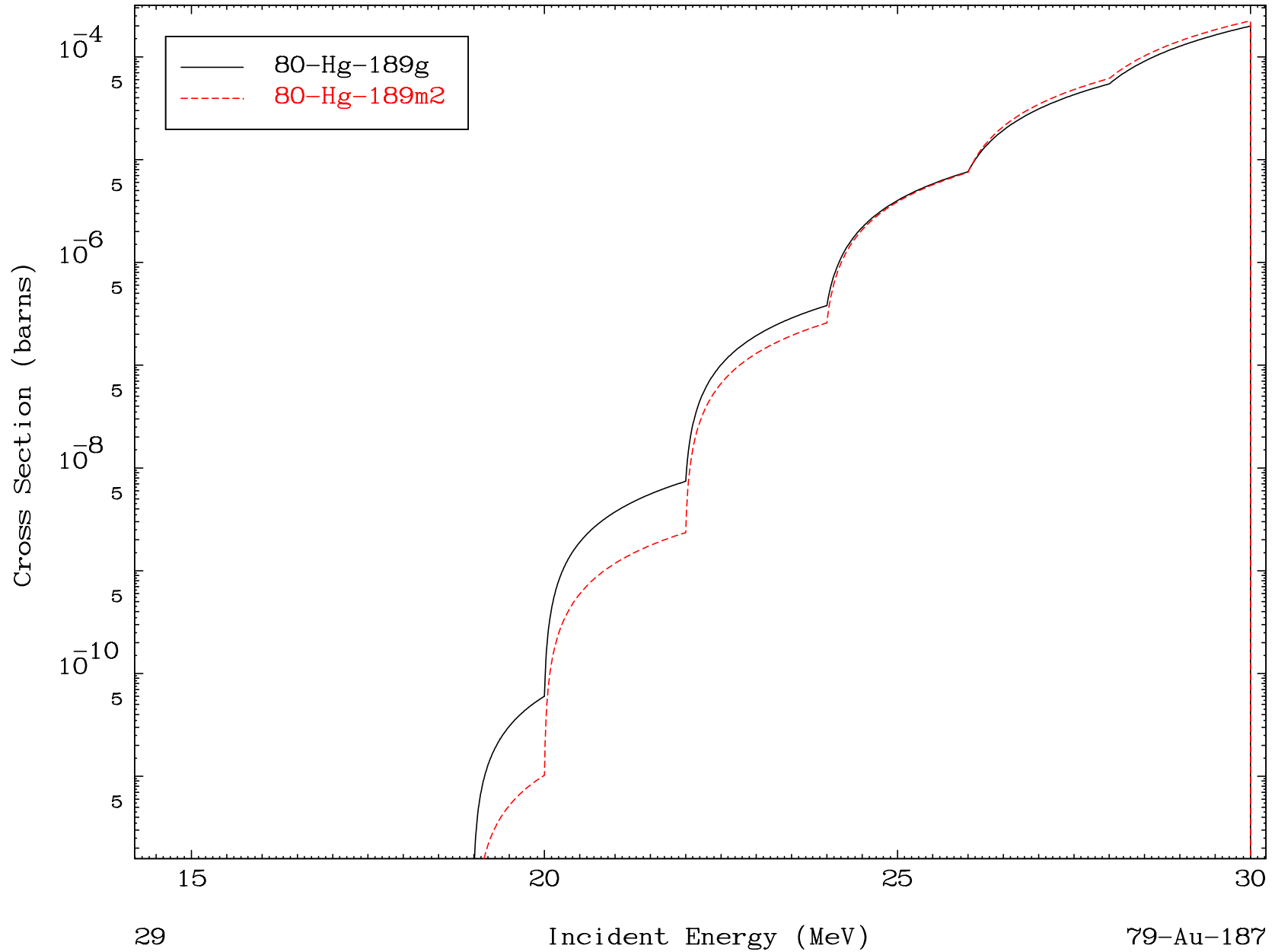
79-Au-187

MAT 7895

( $\alpha, d$ )

79-Au-187

### Radionuclide Production Cross Section

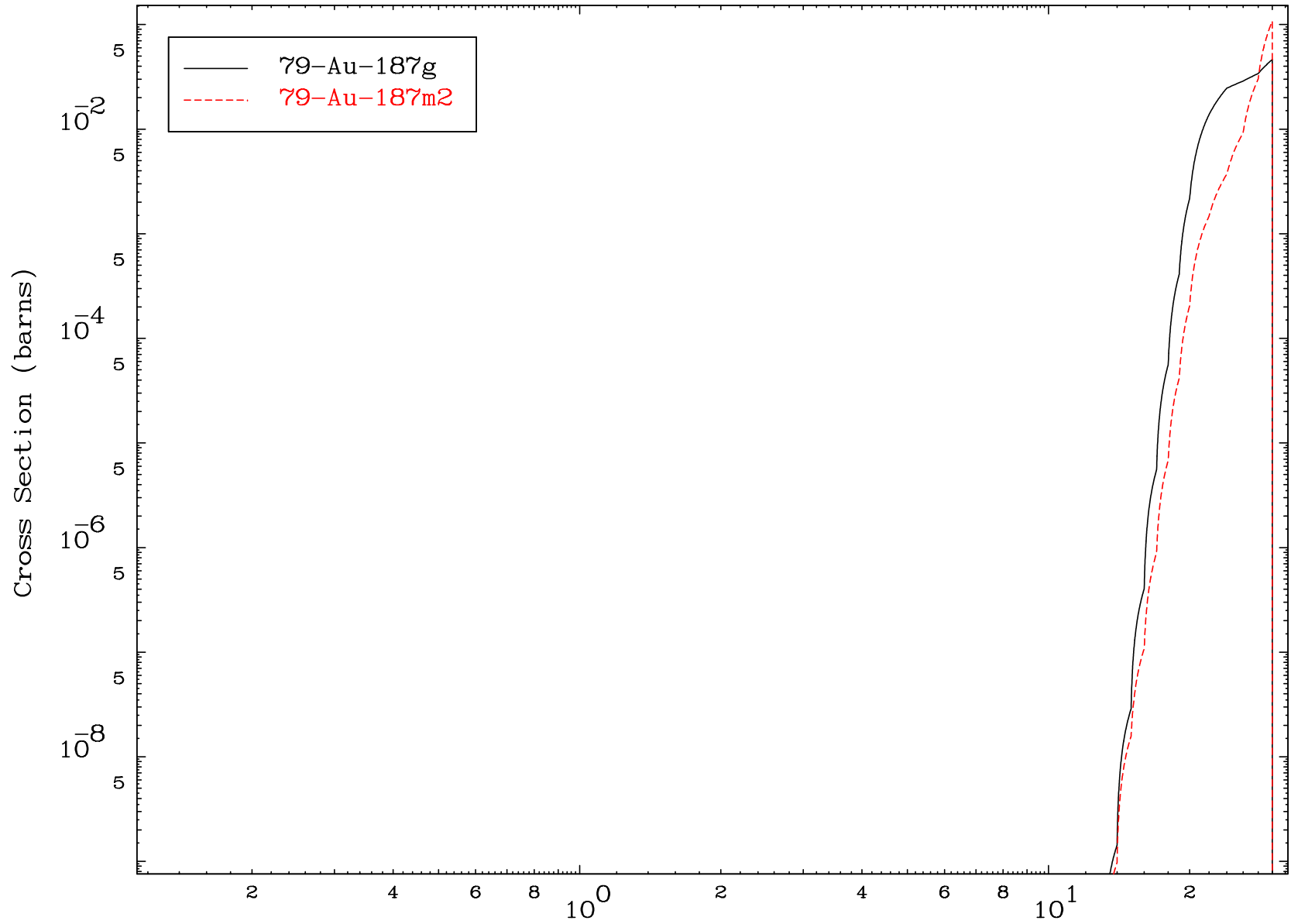


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Incident Energy (MeV)

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

