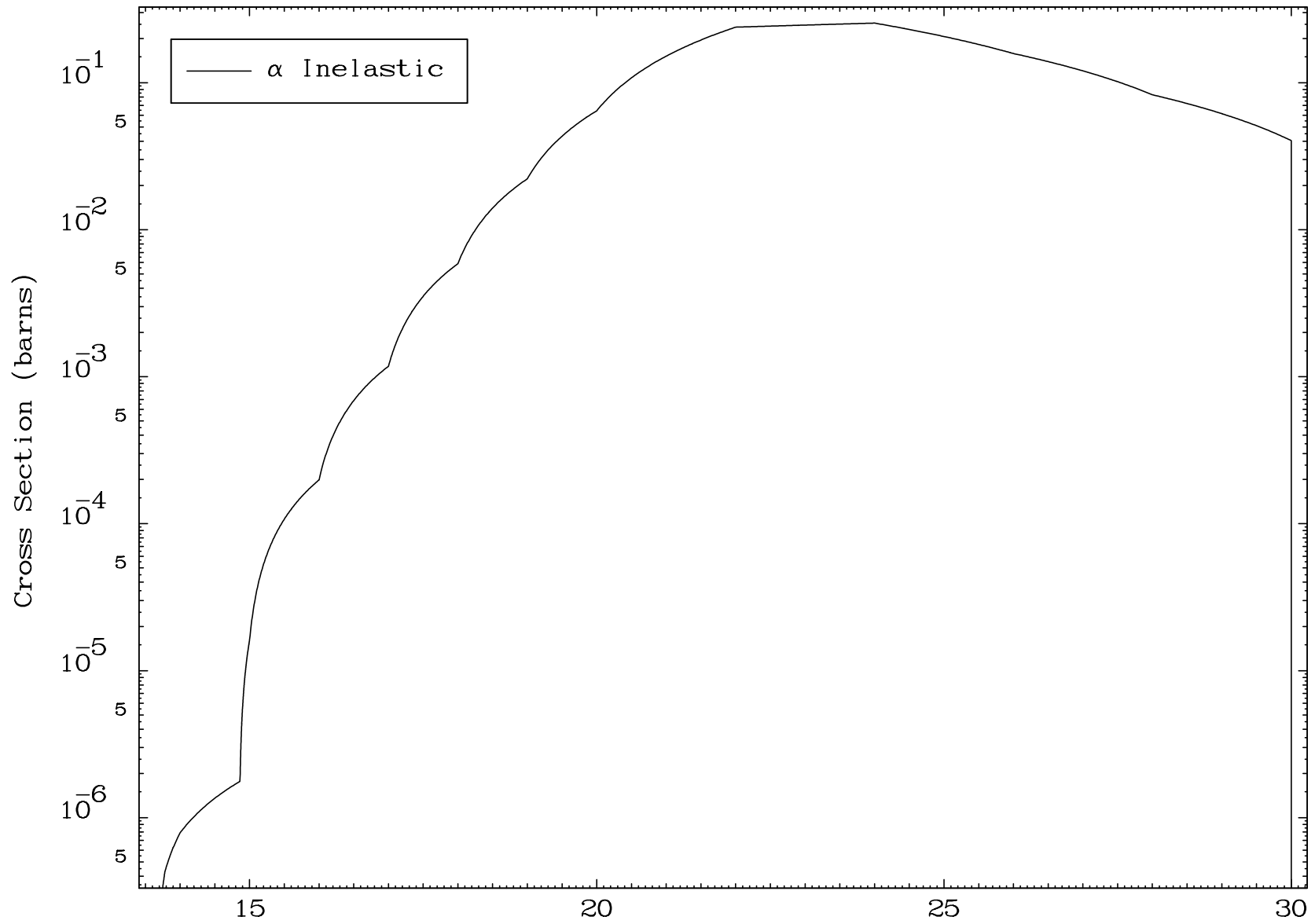


MAT 7901

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

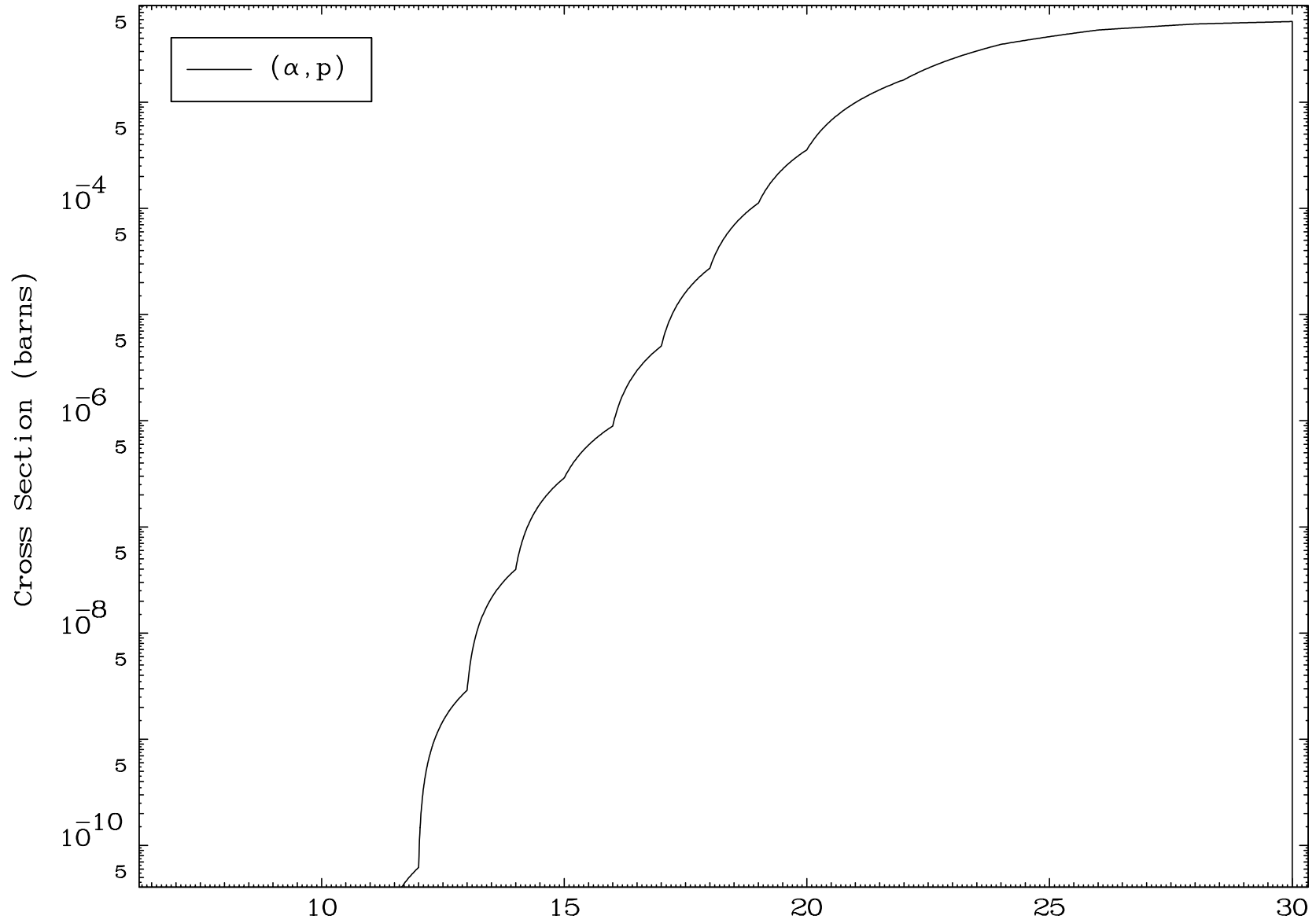
79-Au-189

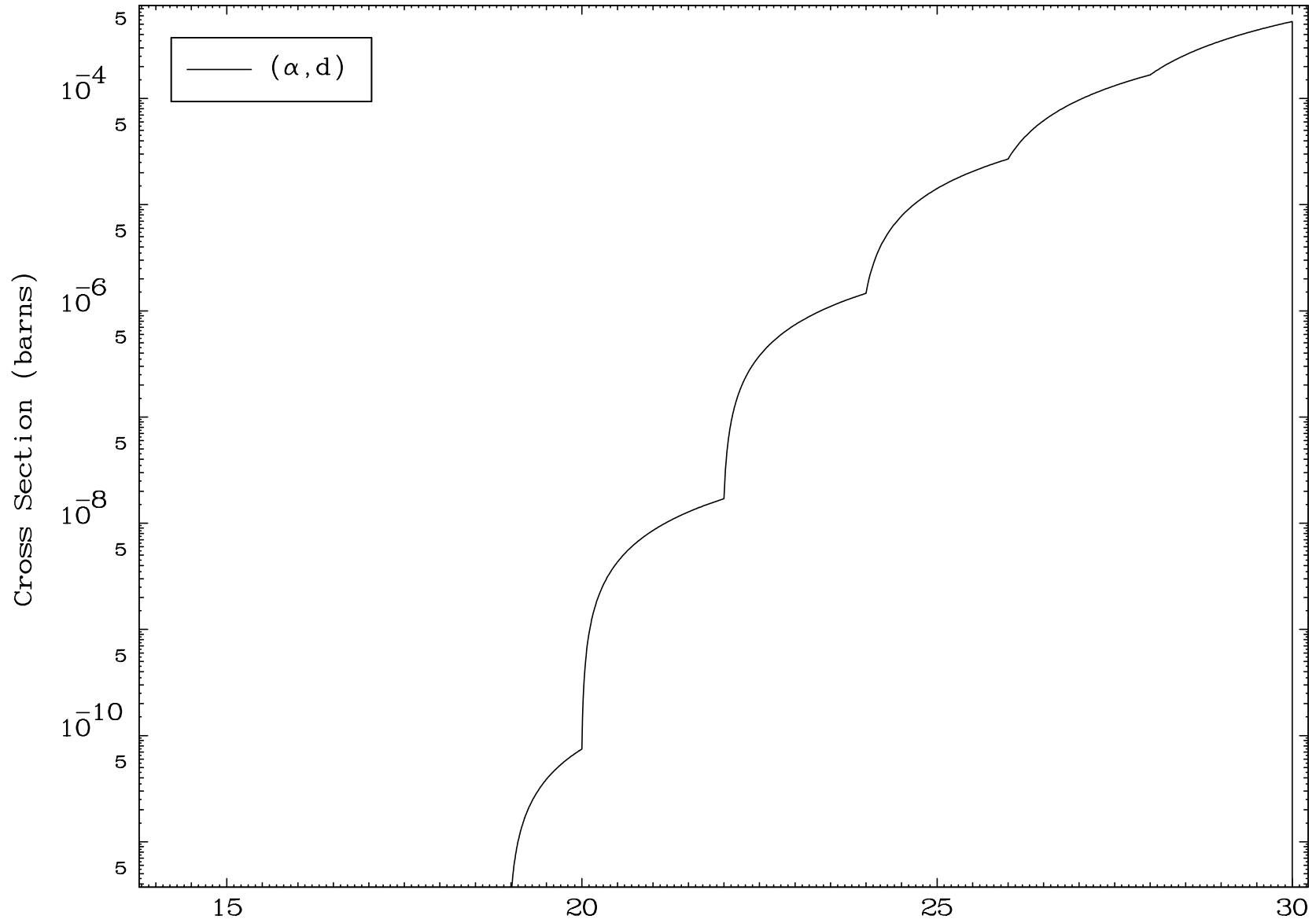


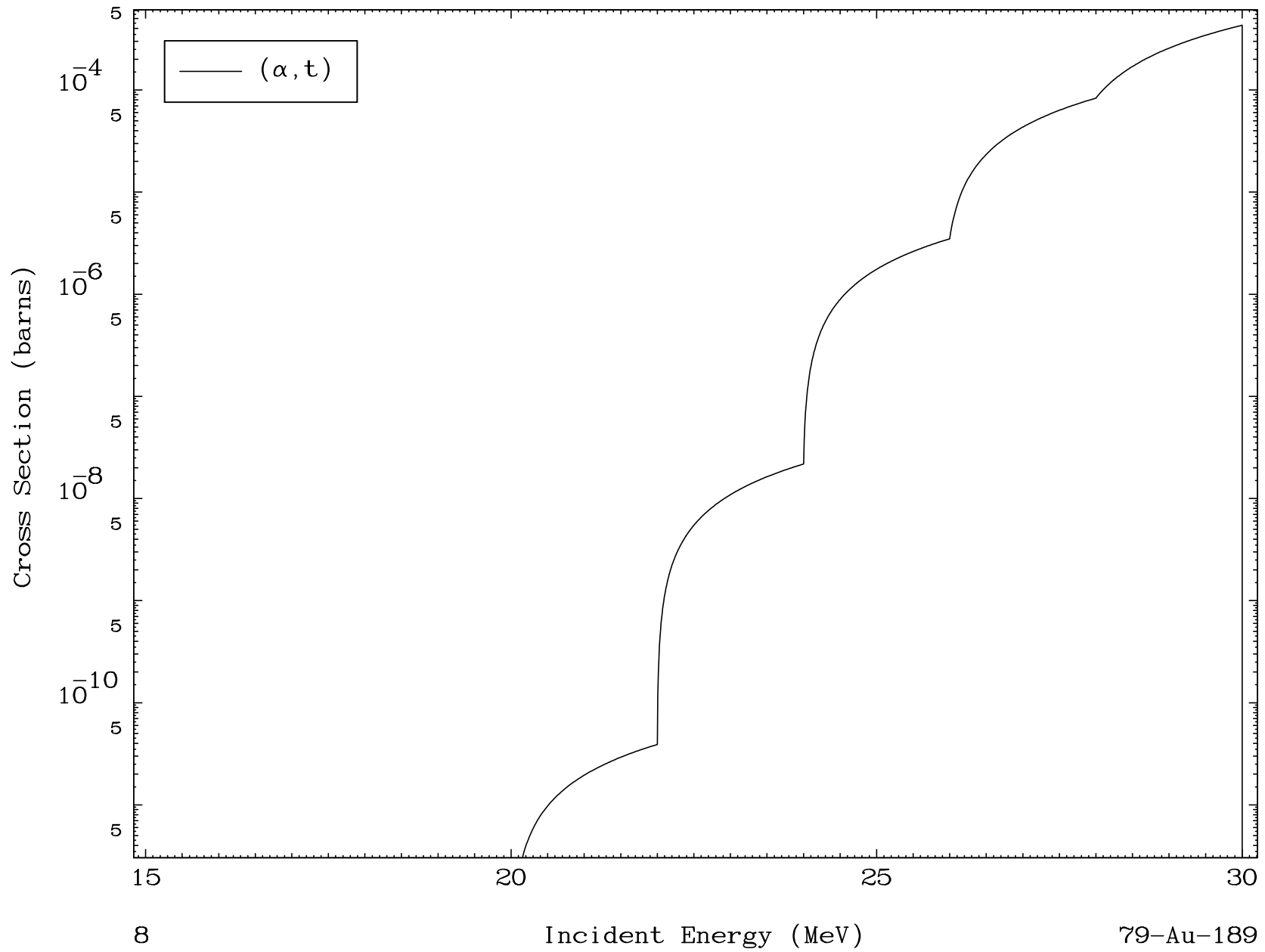
5

Incident Energy (MeV)

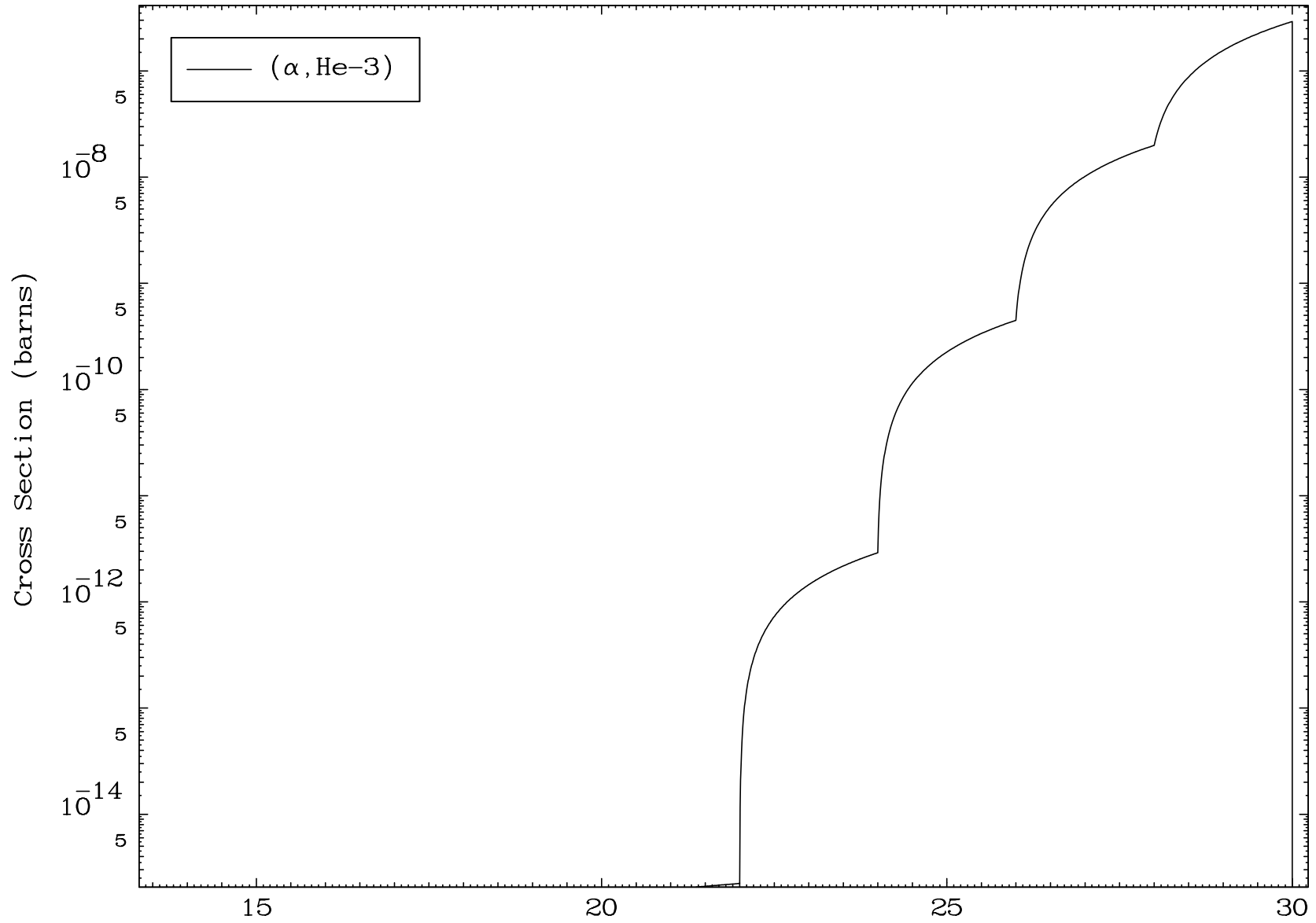
79-Au-189







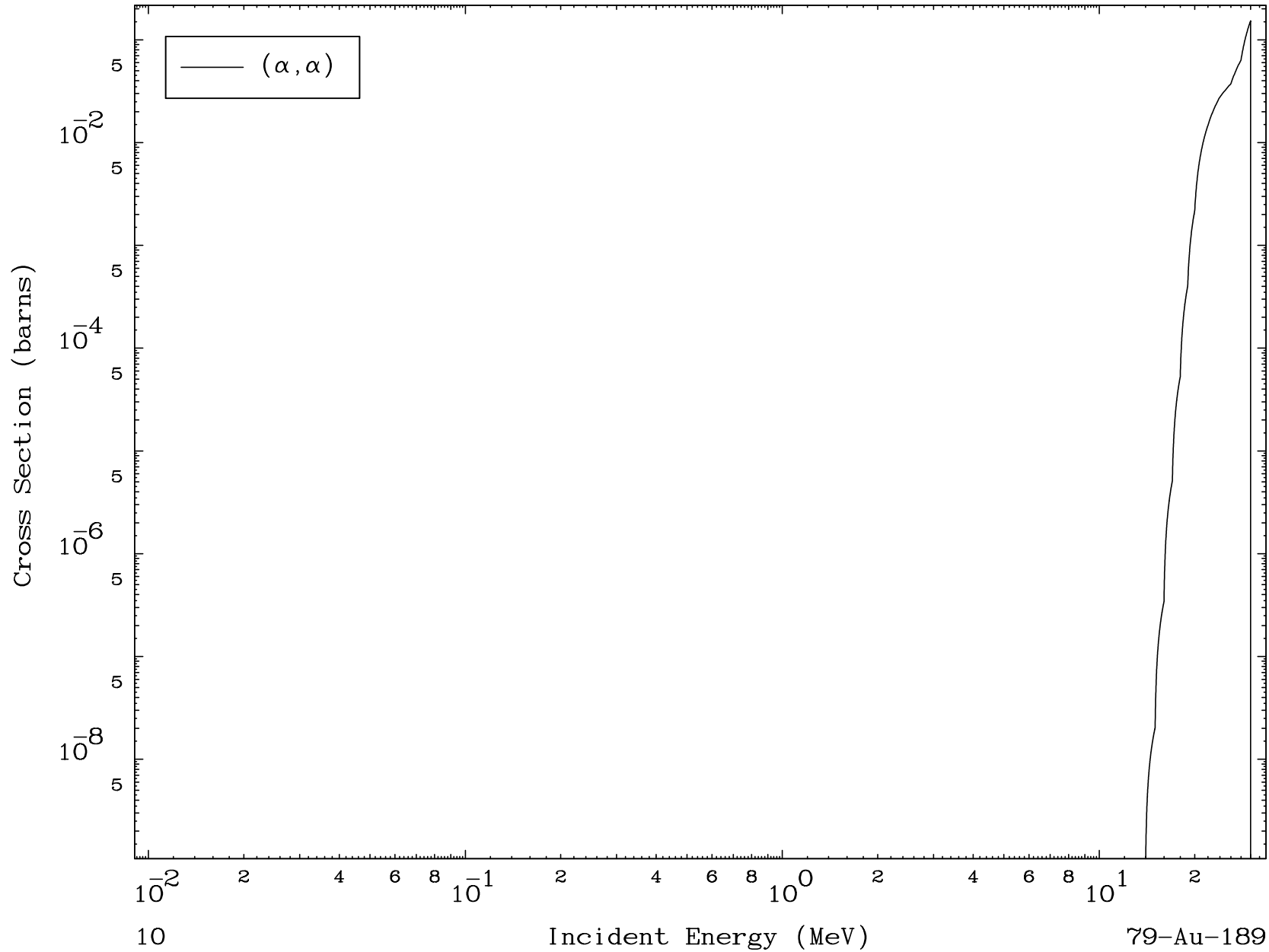




MAT 7901

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

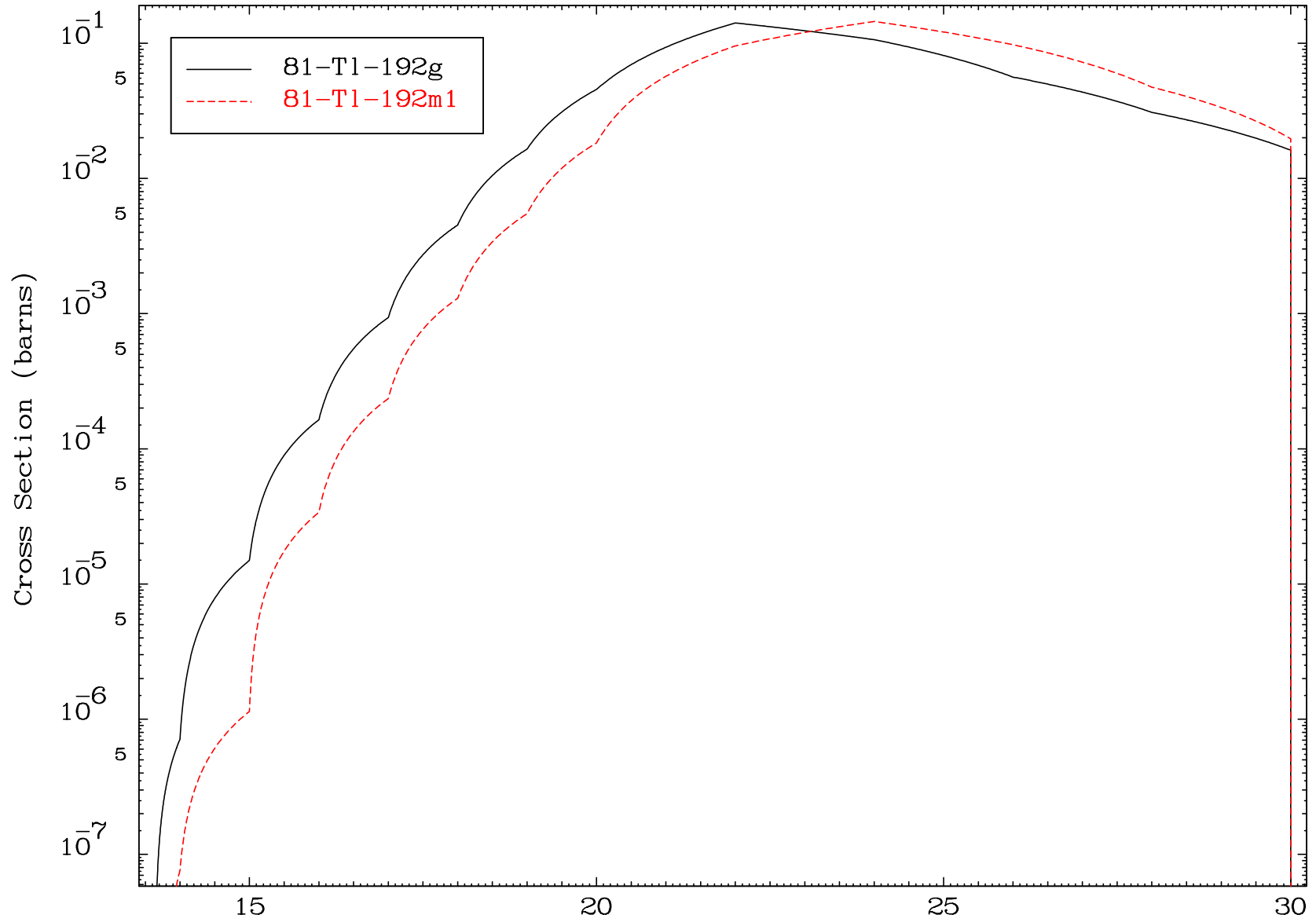
79-Au-189



MAT 7901

$\alpha$  Inelastic  
Radionuclide Production Cross Section

79-Au-189

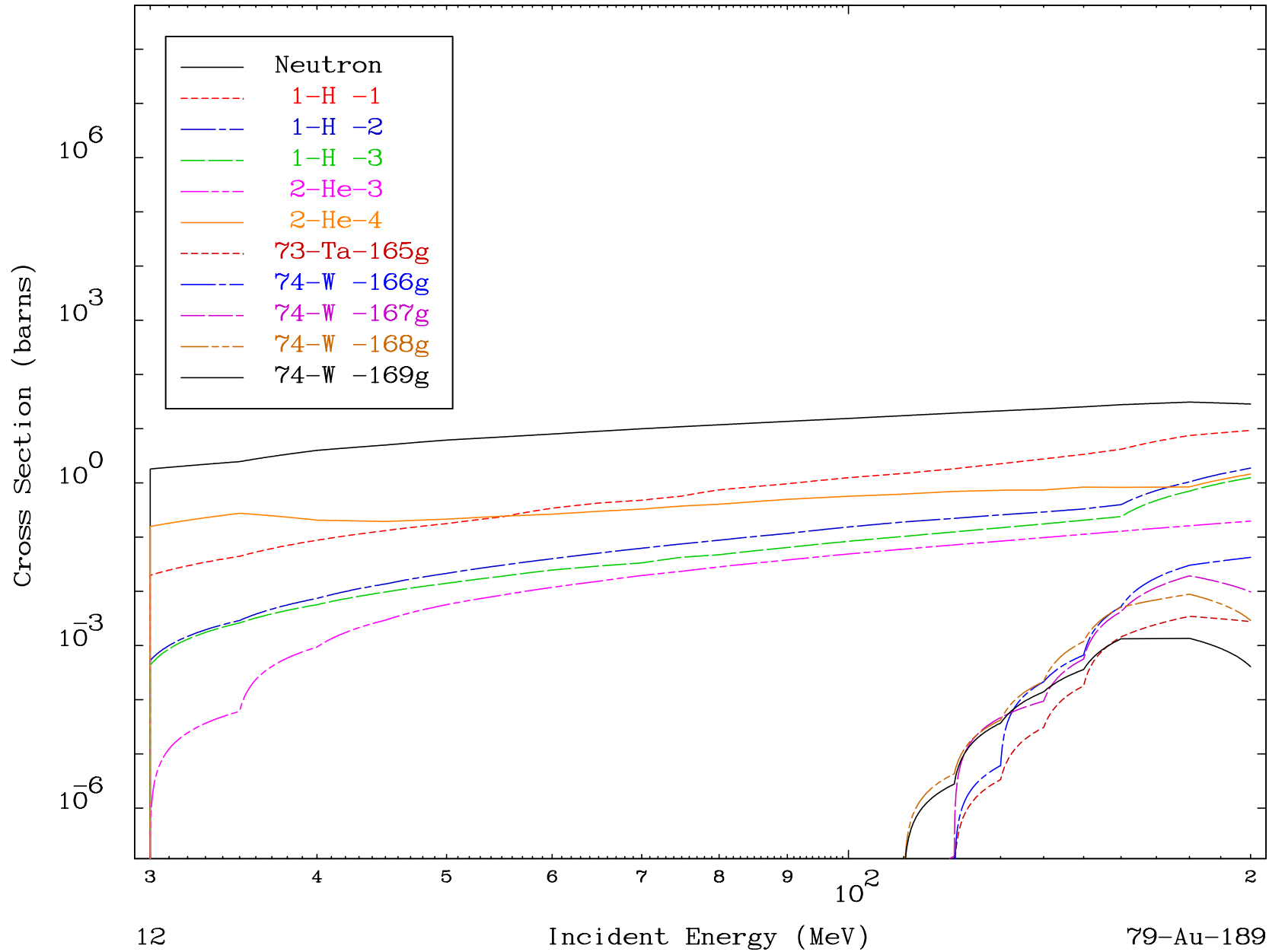


11

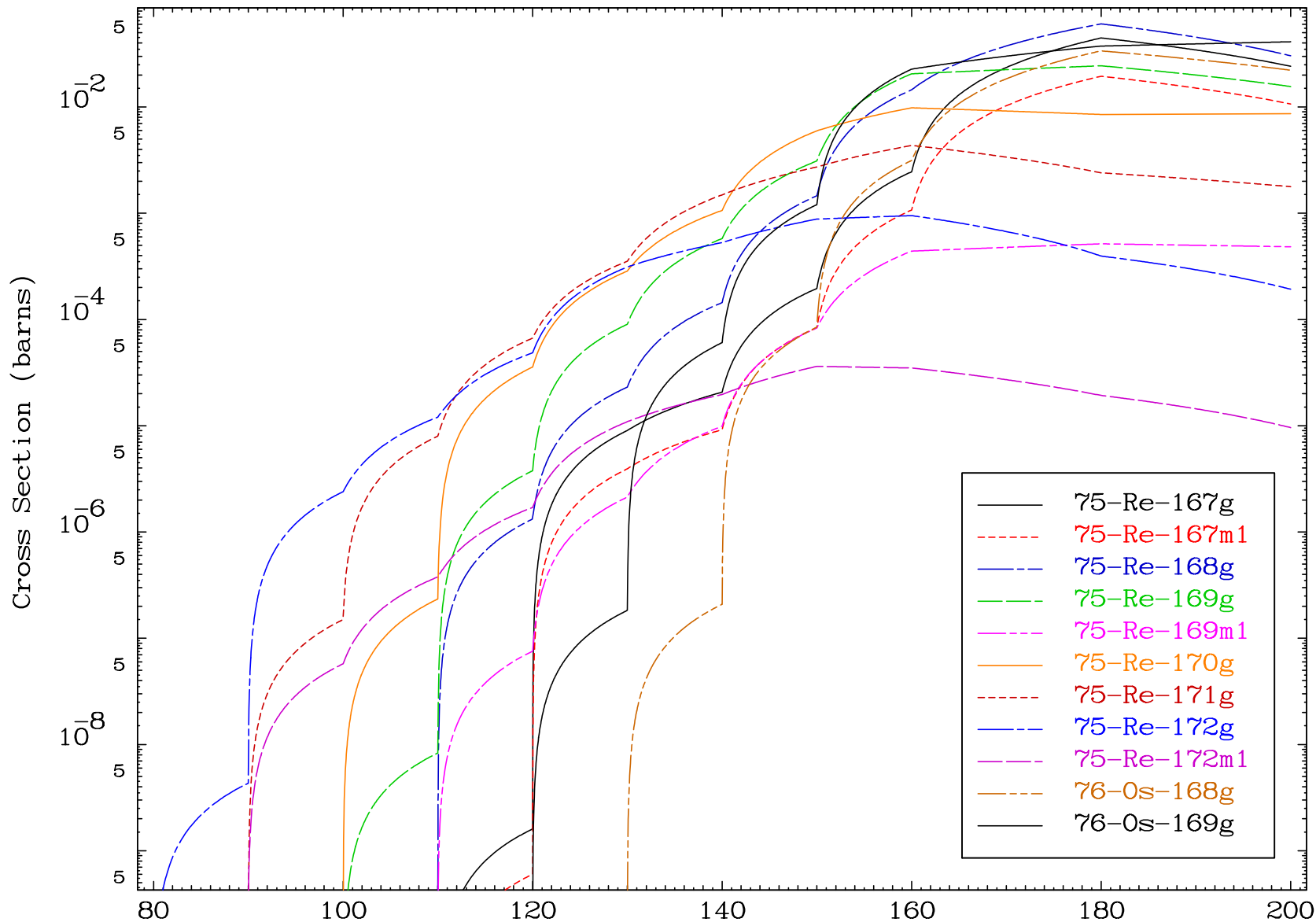
Incident Energy (MeV)

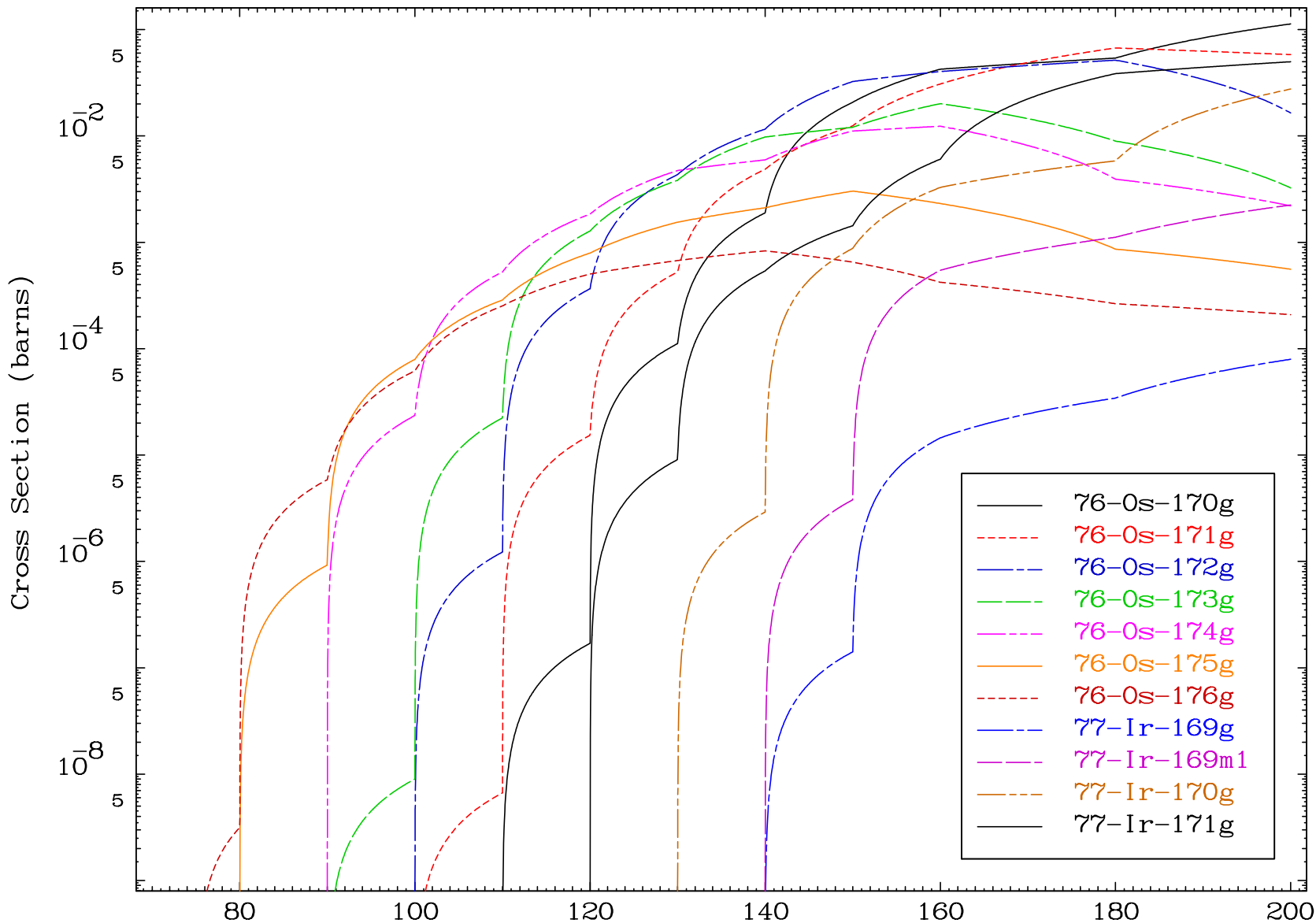
79-Au-189

Radionuclide Production Cross Section

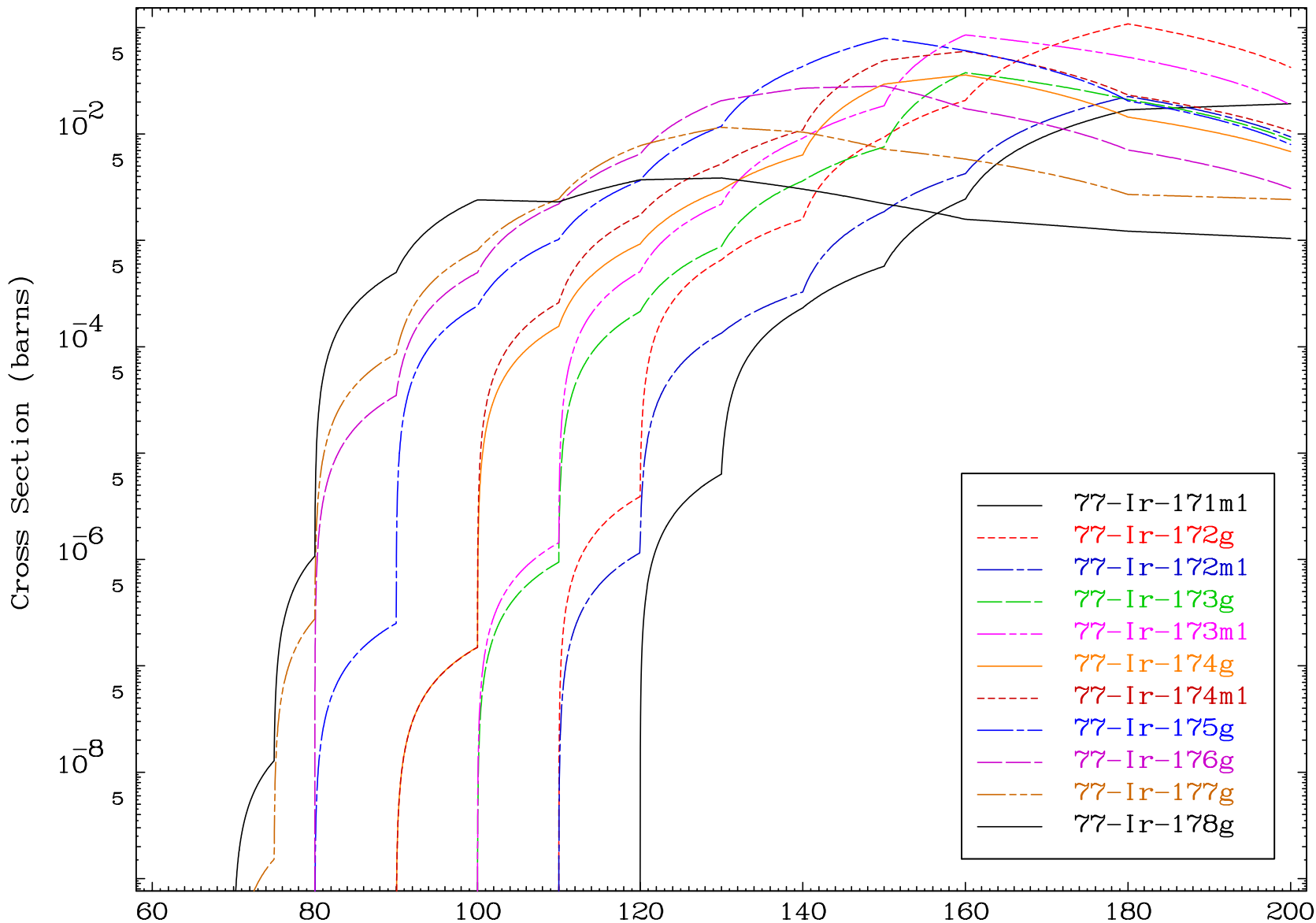


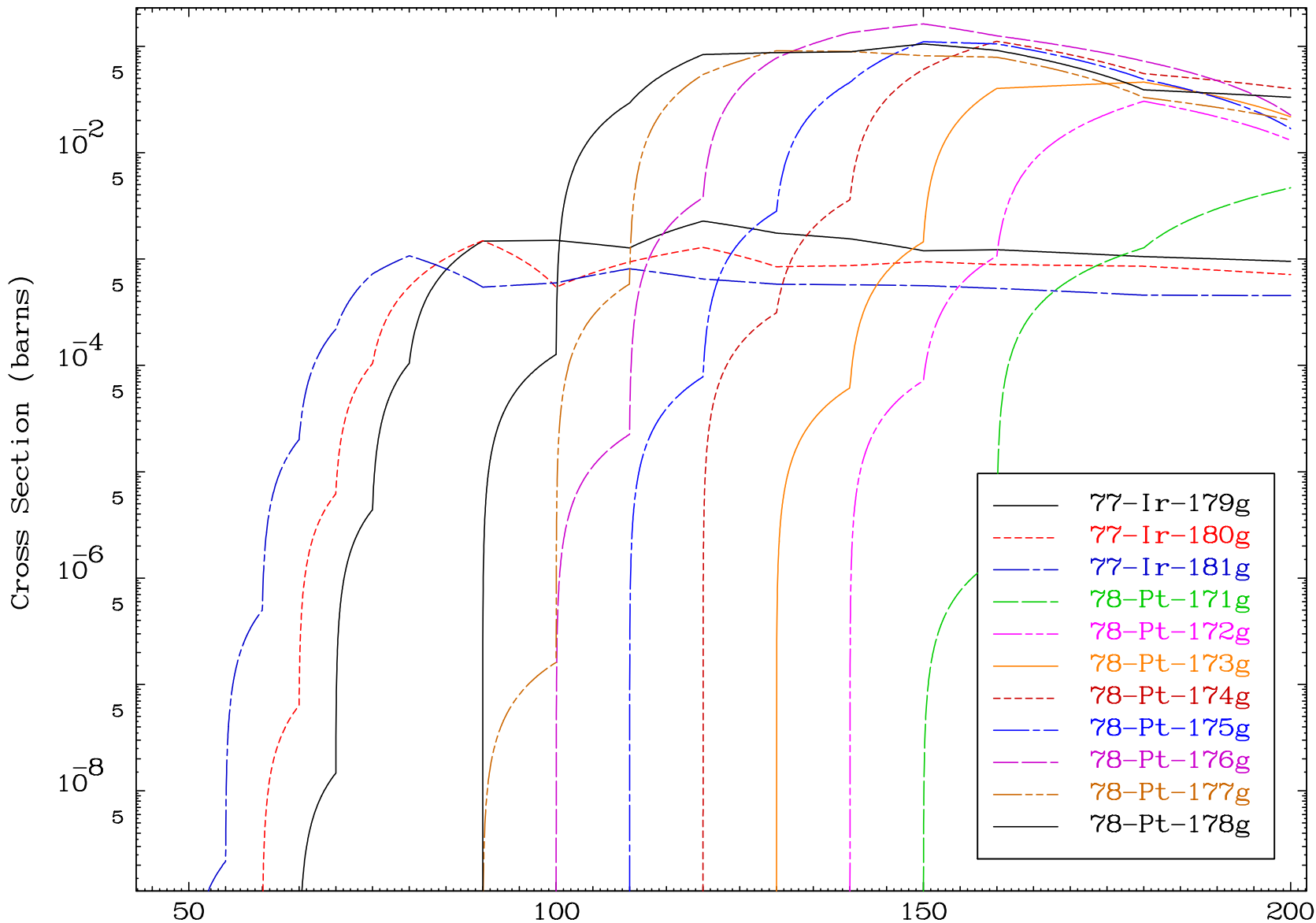
Radionuclide Production Cross Section





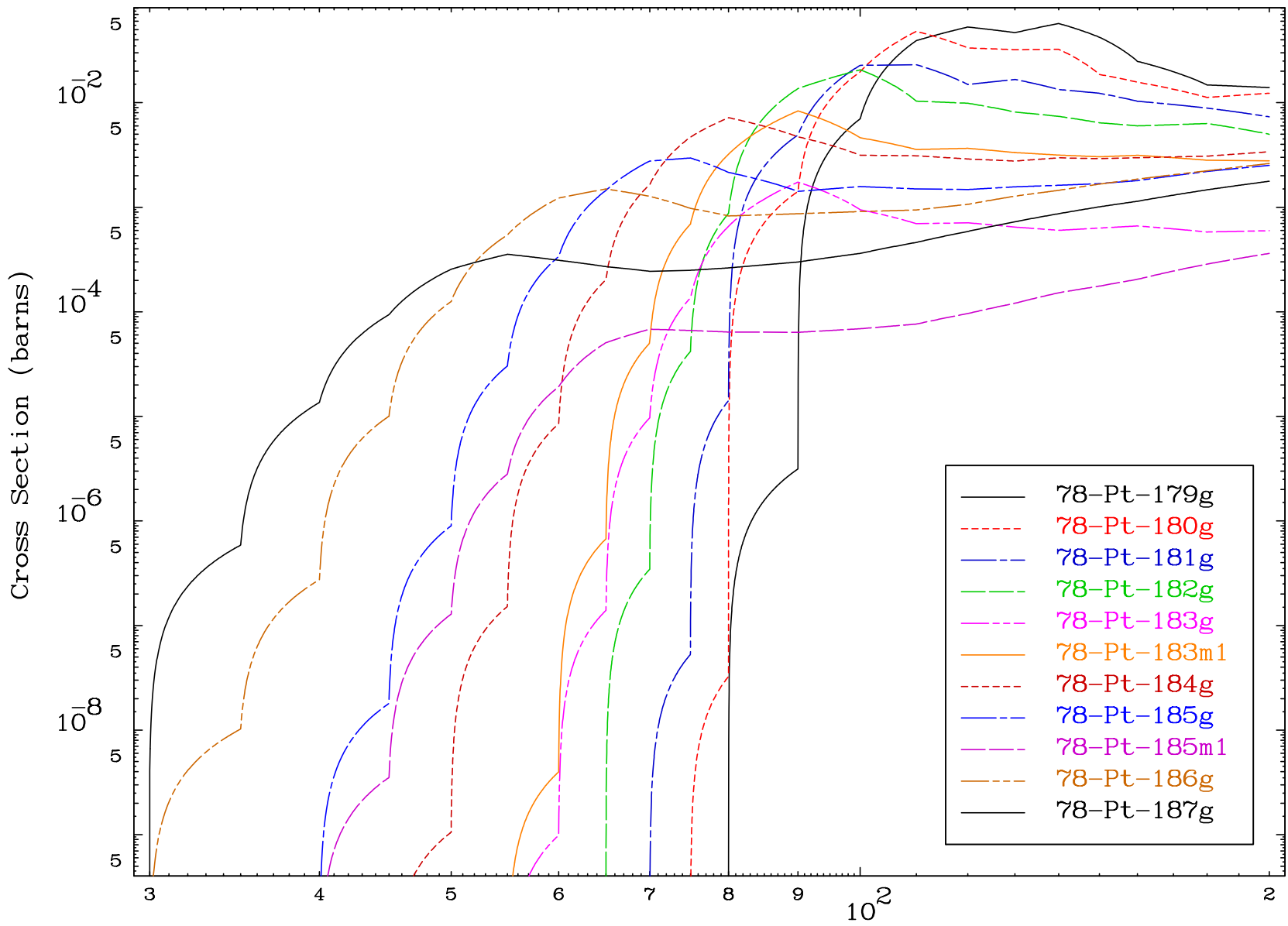
Radionuclide Production Cross Section

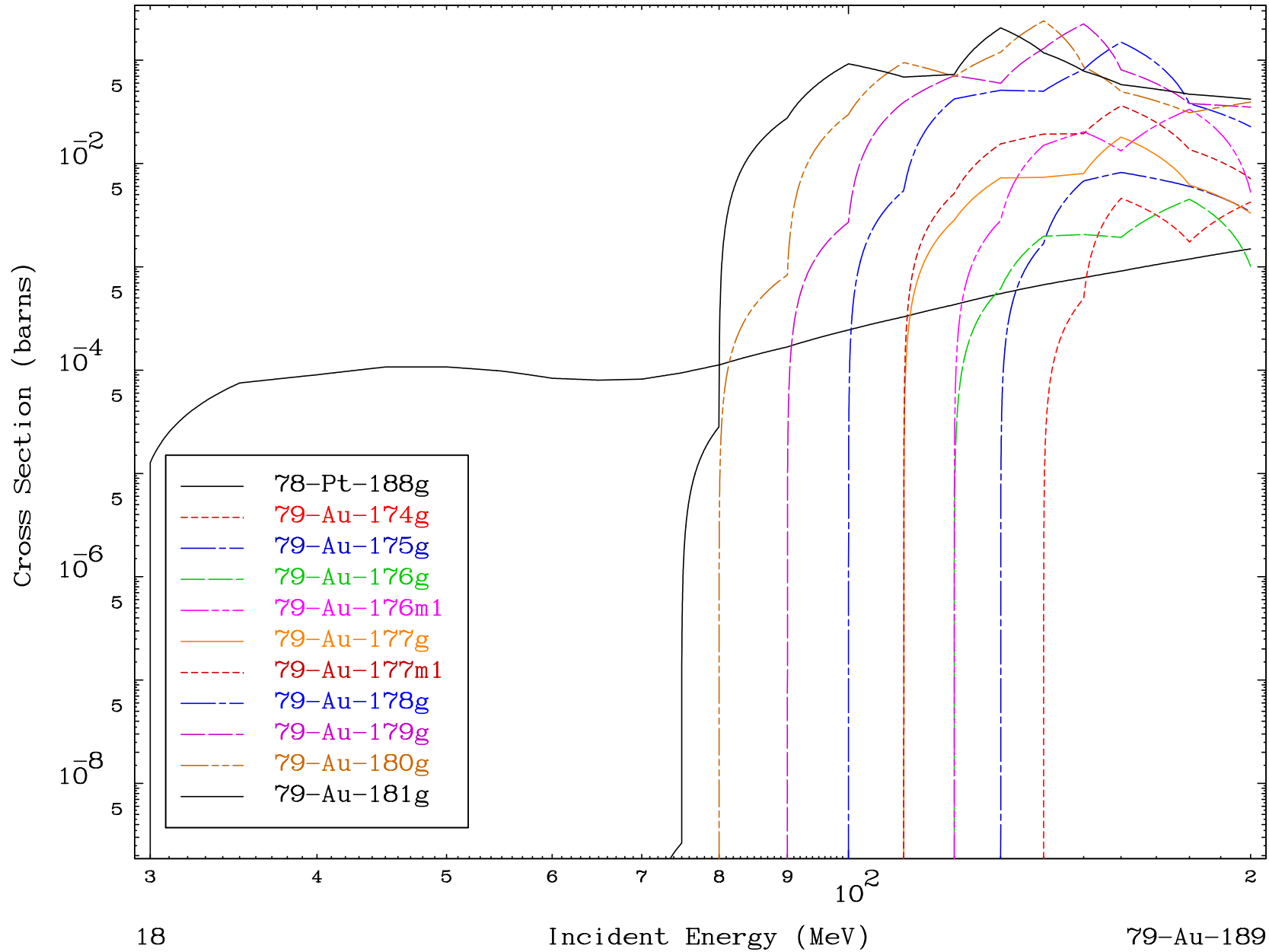




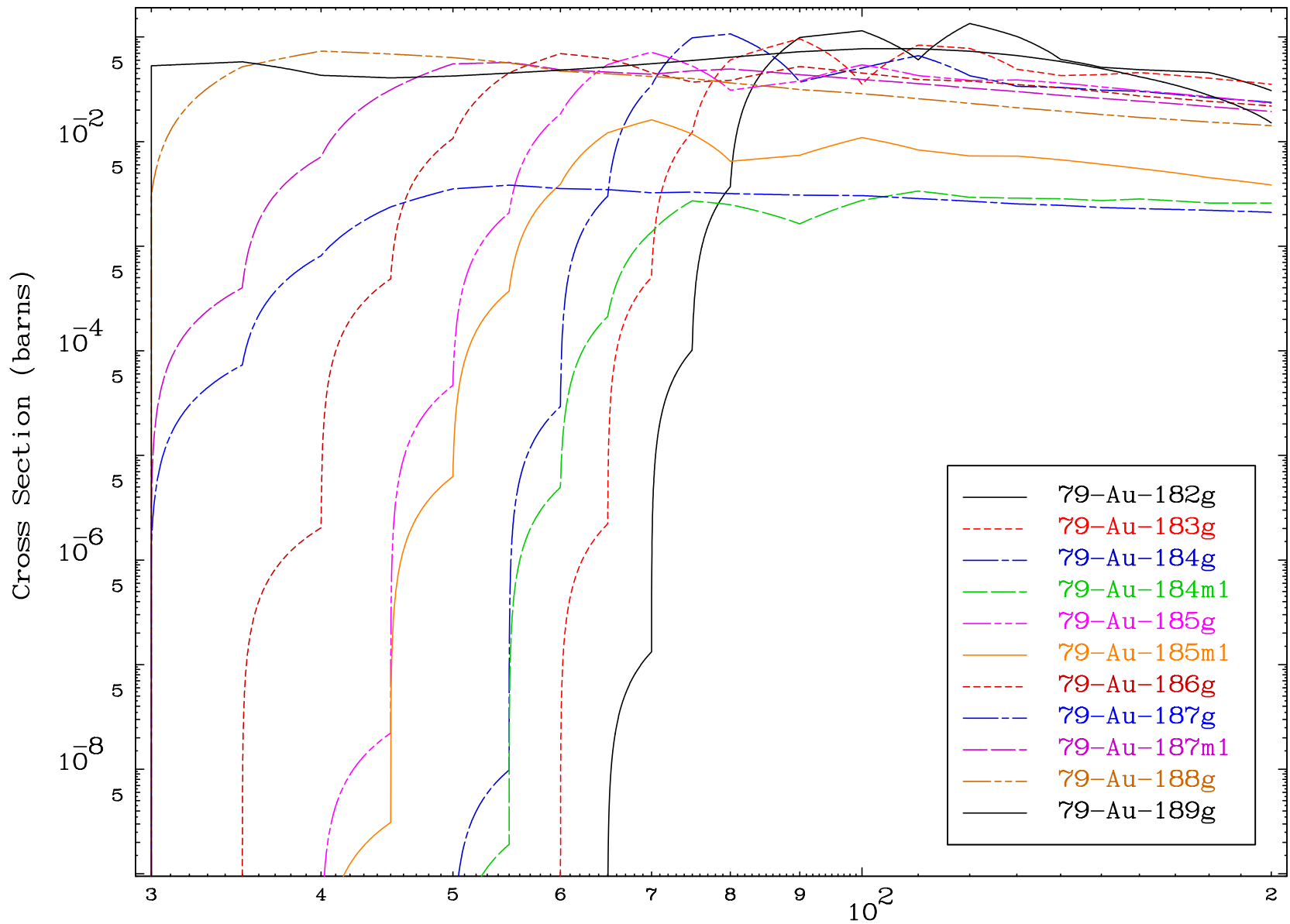


Radionuclide Production Cross Section

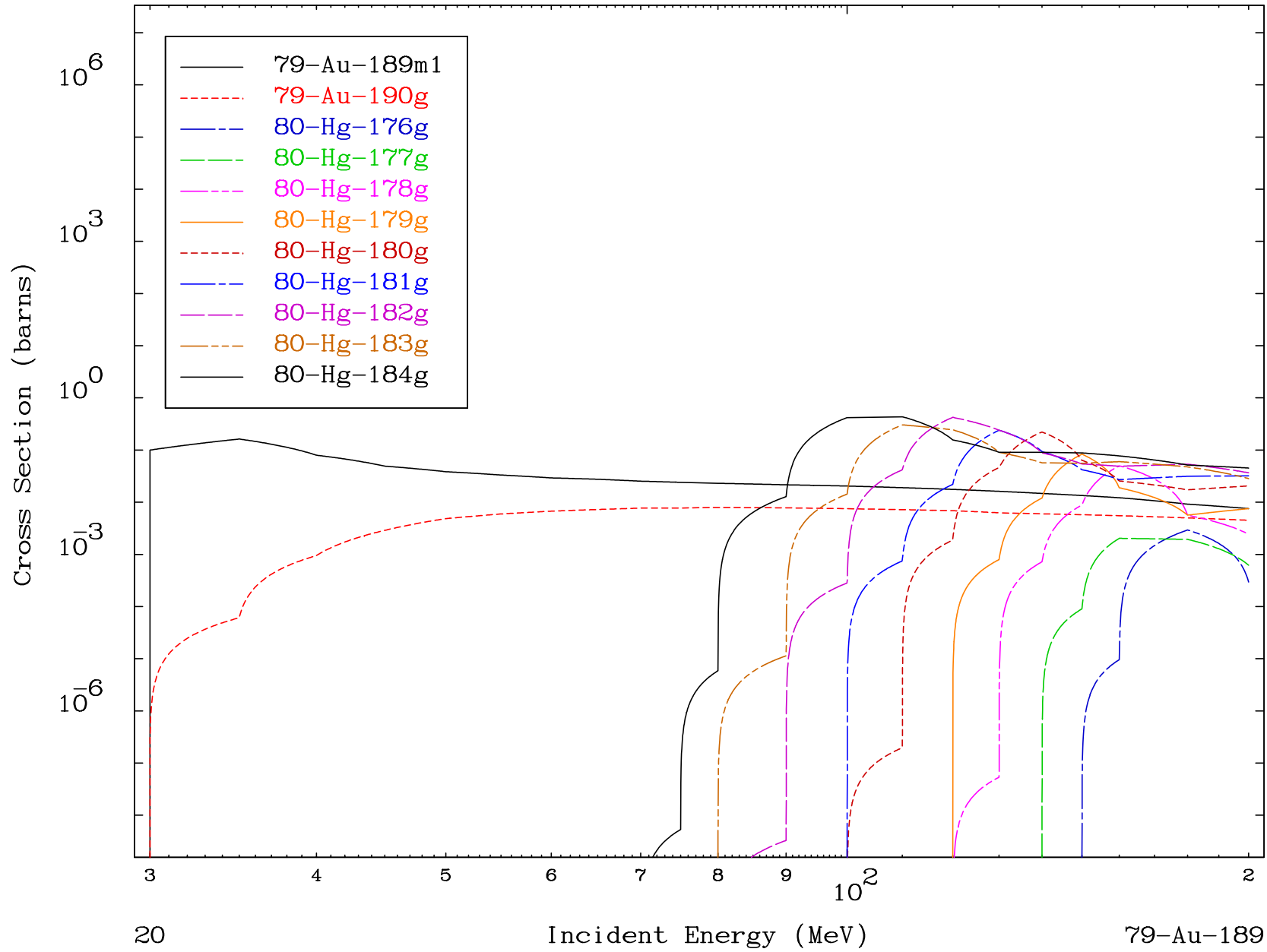




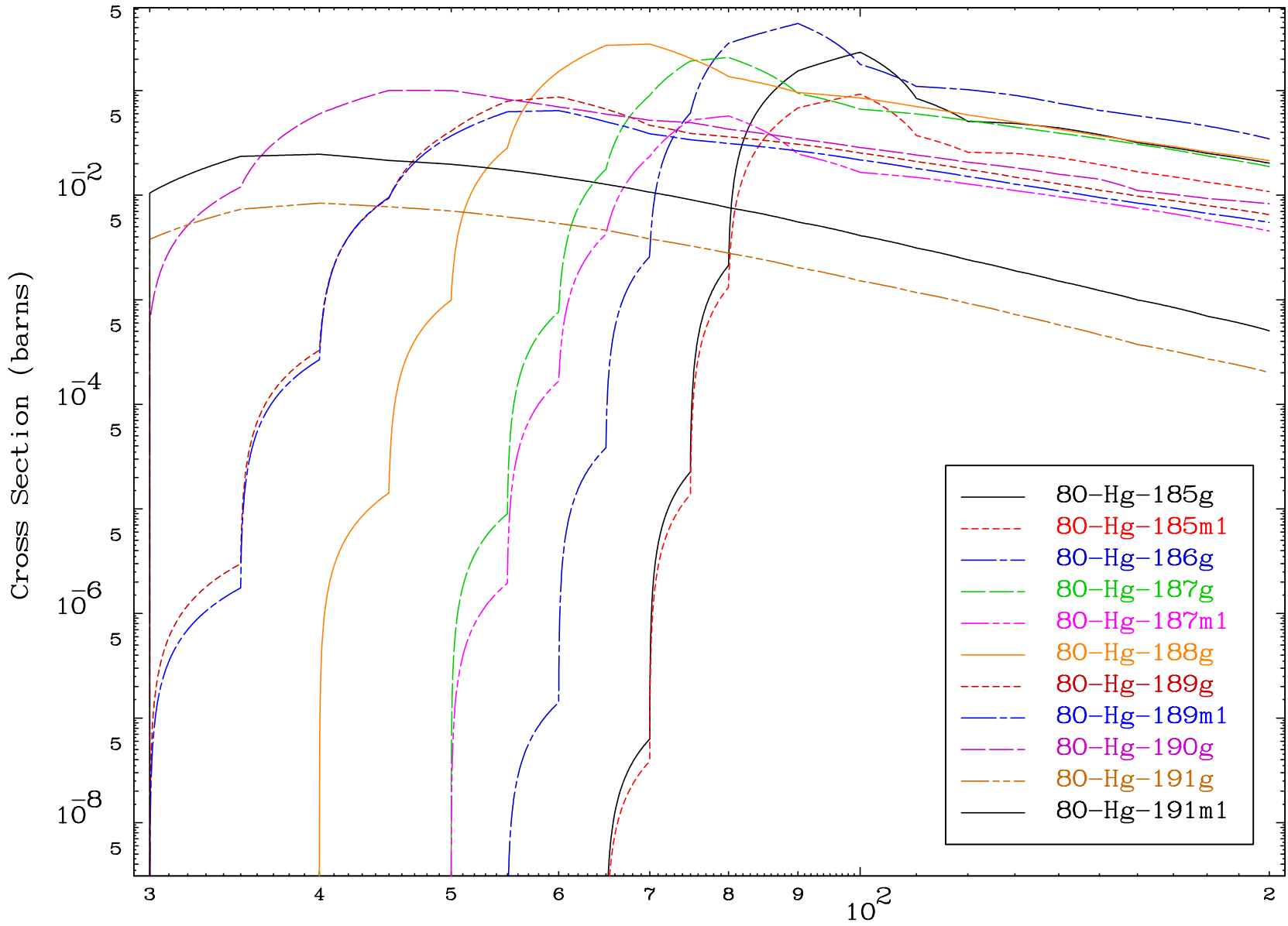
Radionuclide Production Cross Section

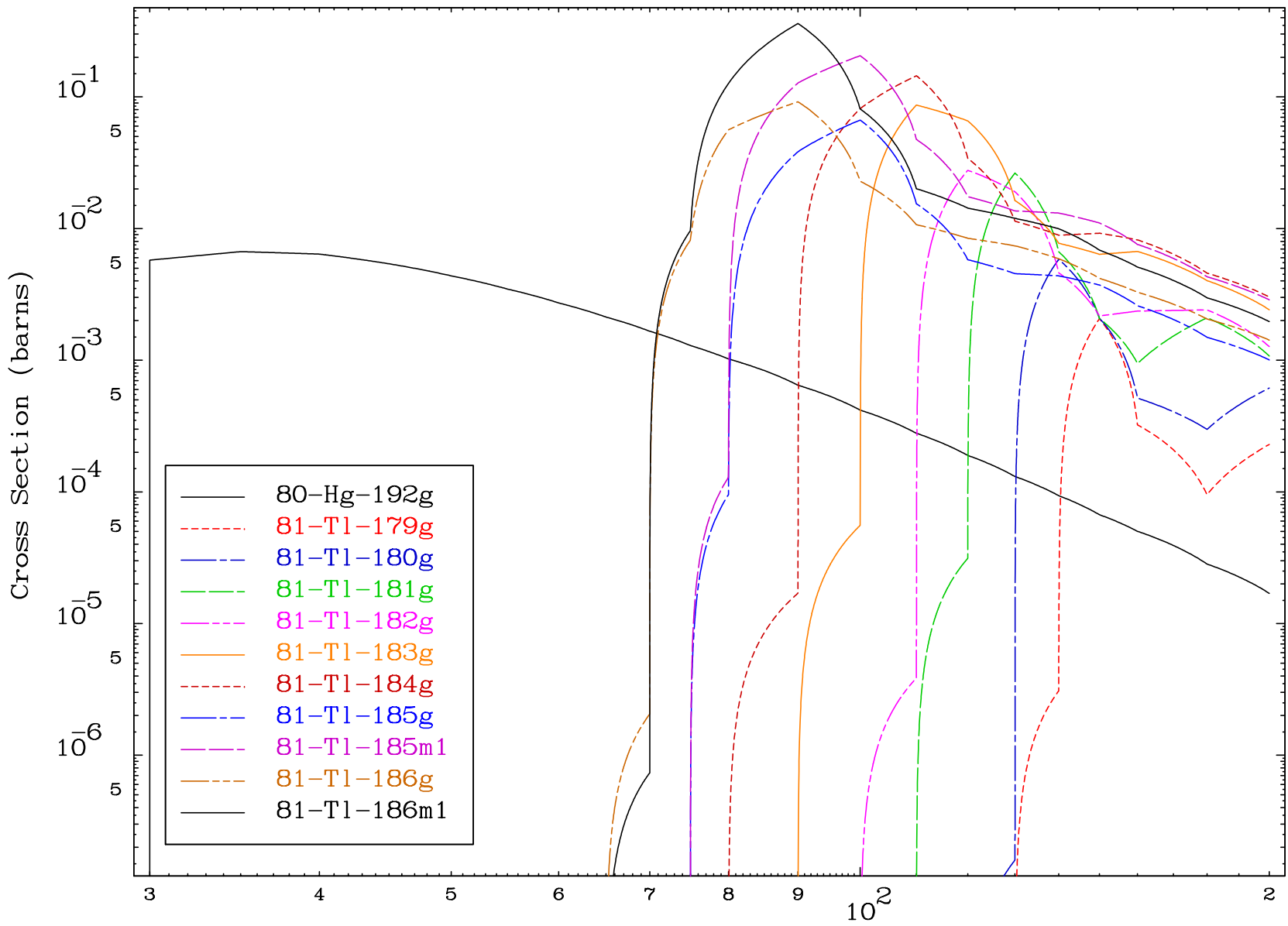


Radionuclide Production Cross Section

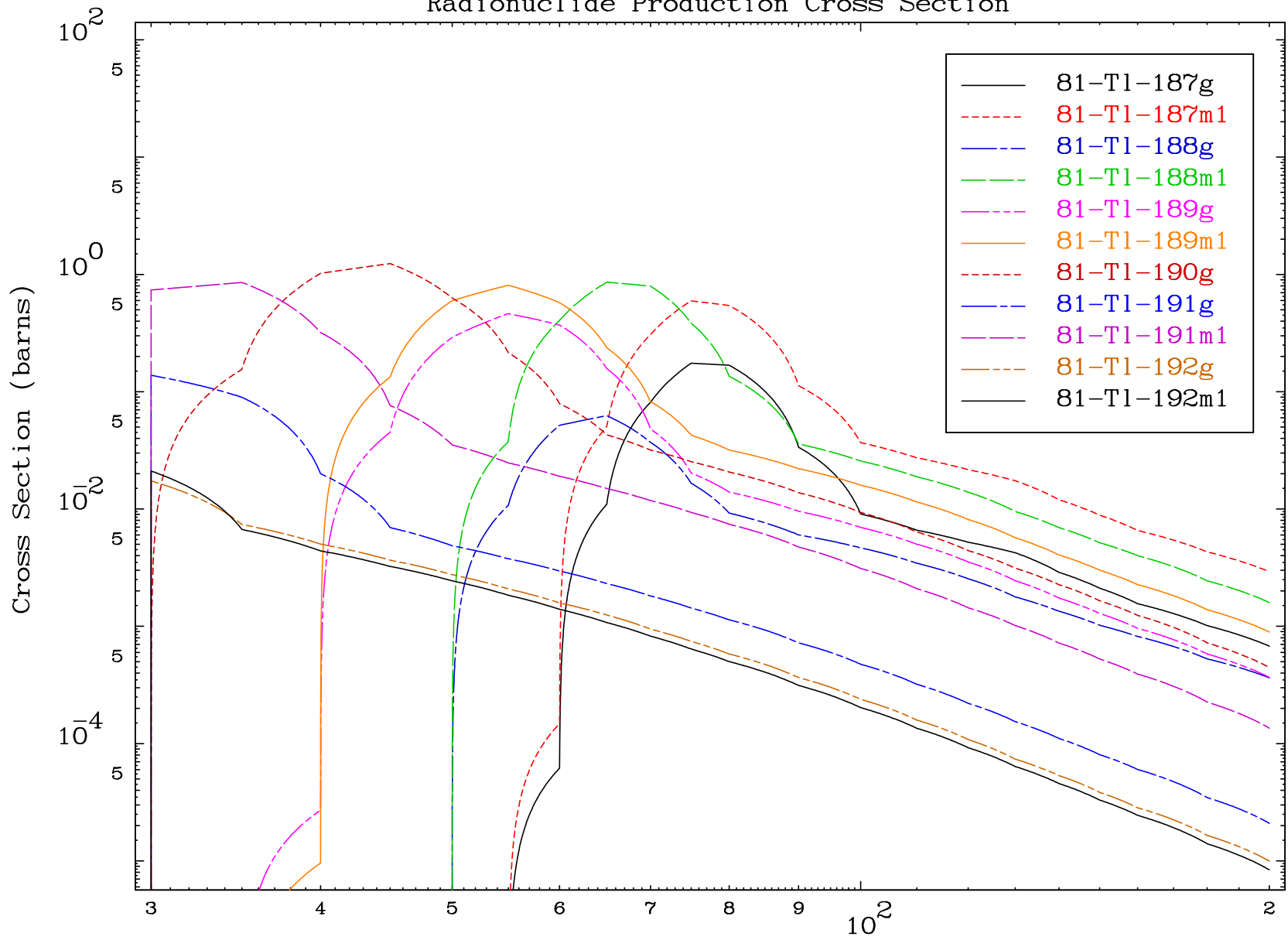


Radionuclide Production Cross Section

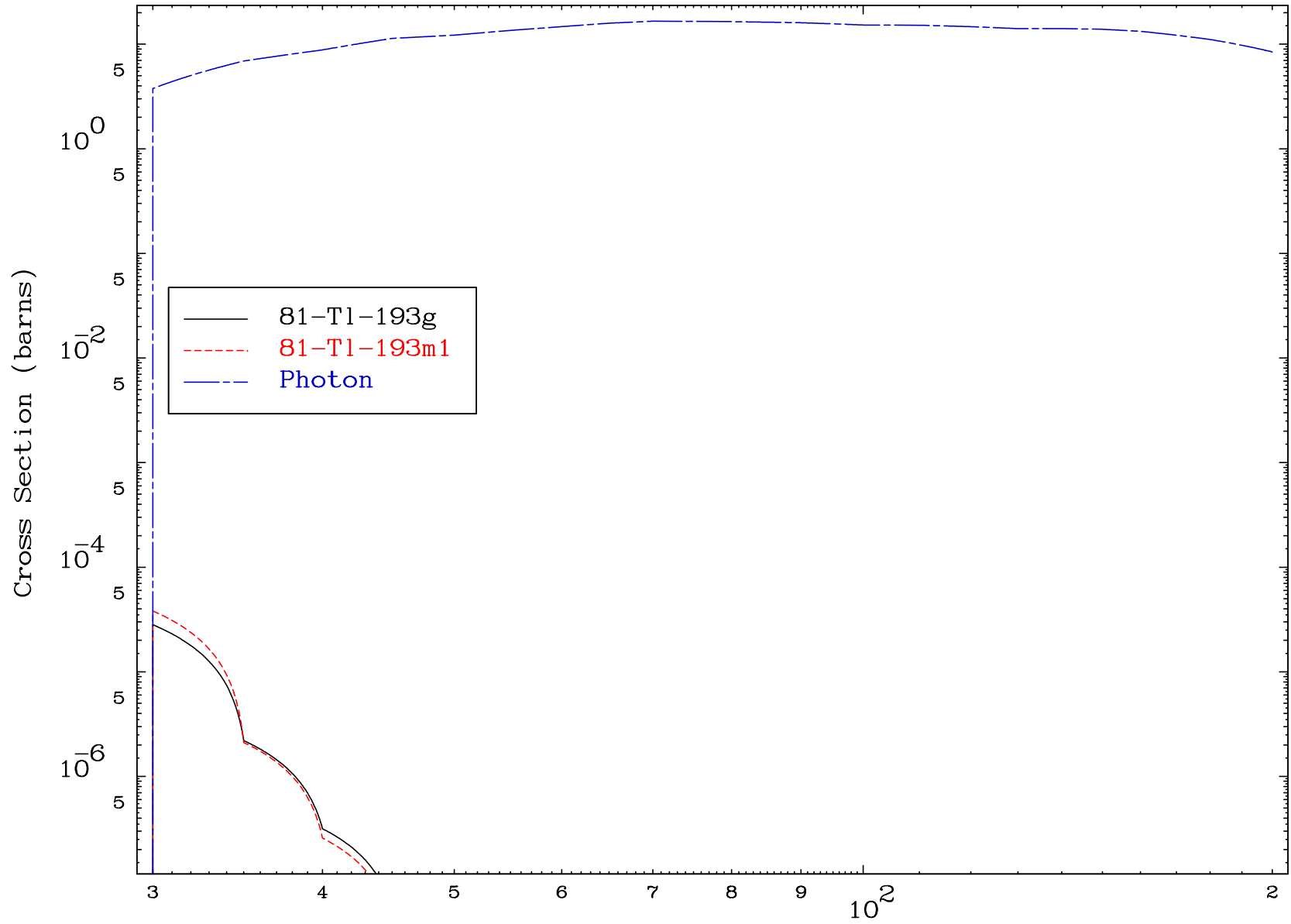




Radionuclide Production Cross Section



Radionuclide Production Cross Section



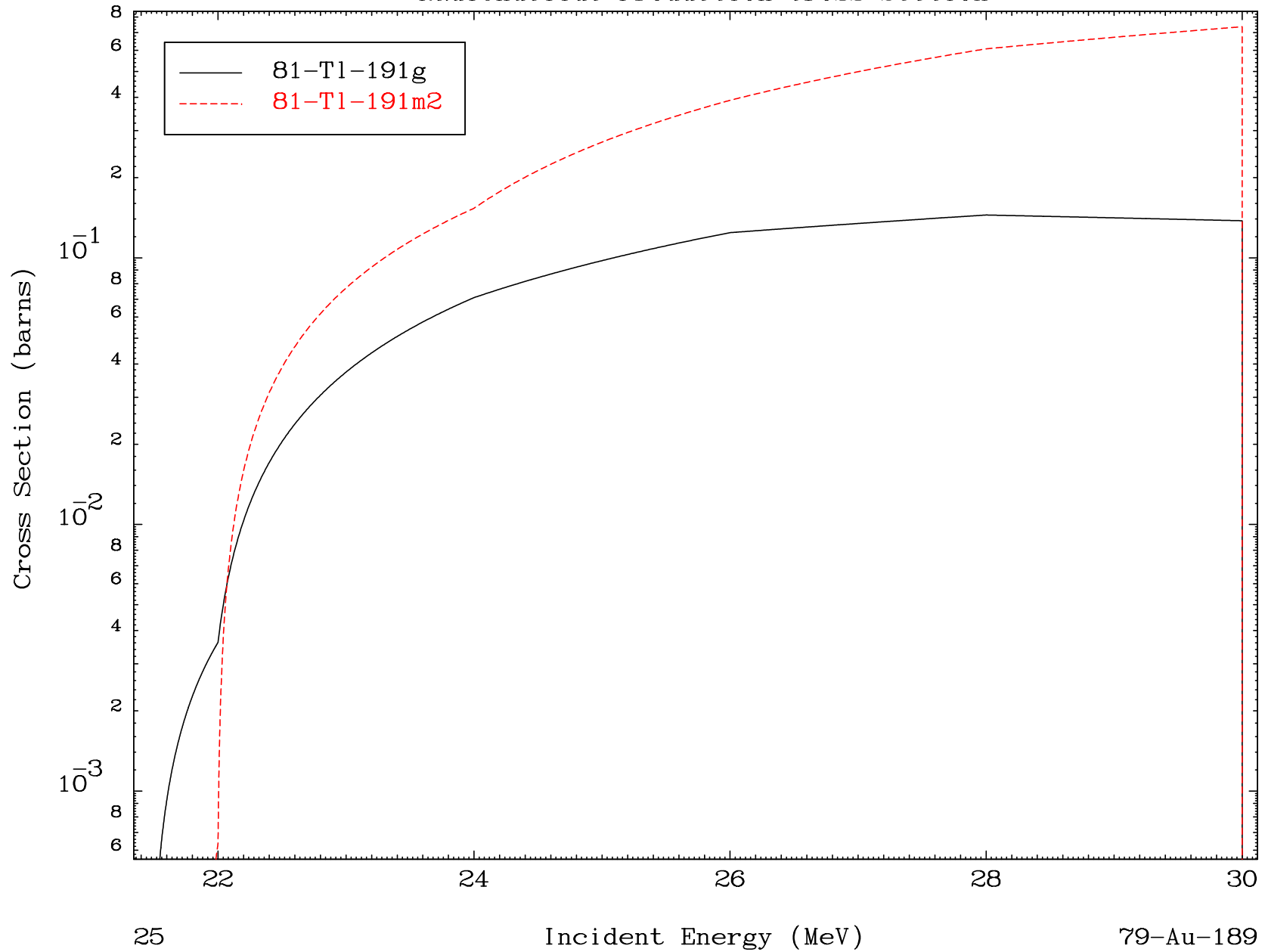


MAT 7901

( $\alpha, 2n$ )

79-Au-189

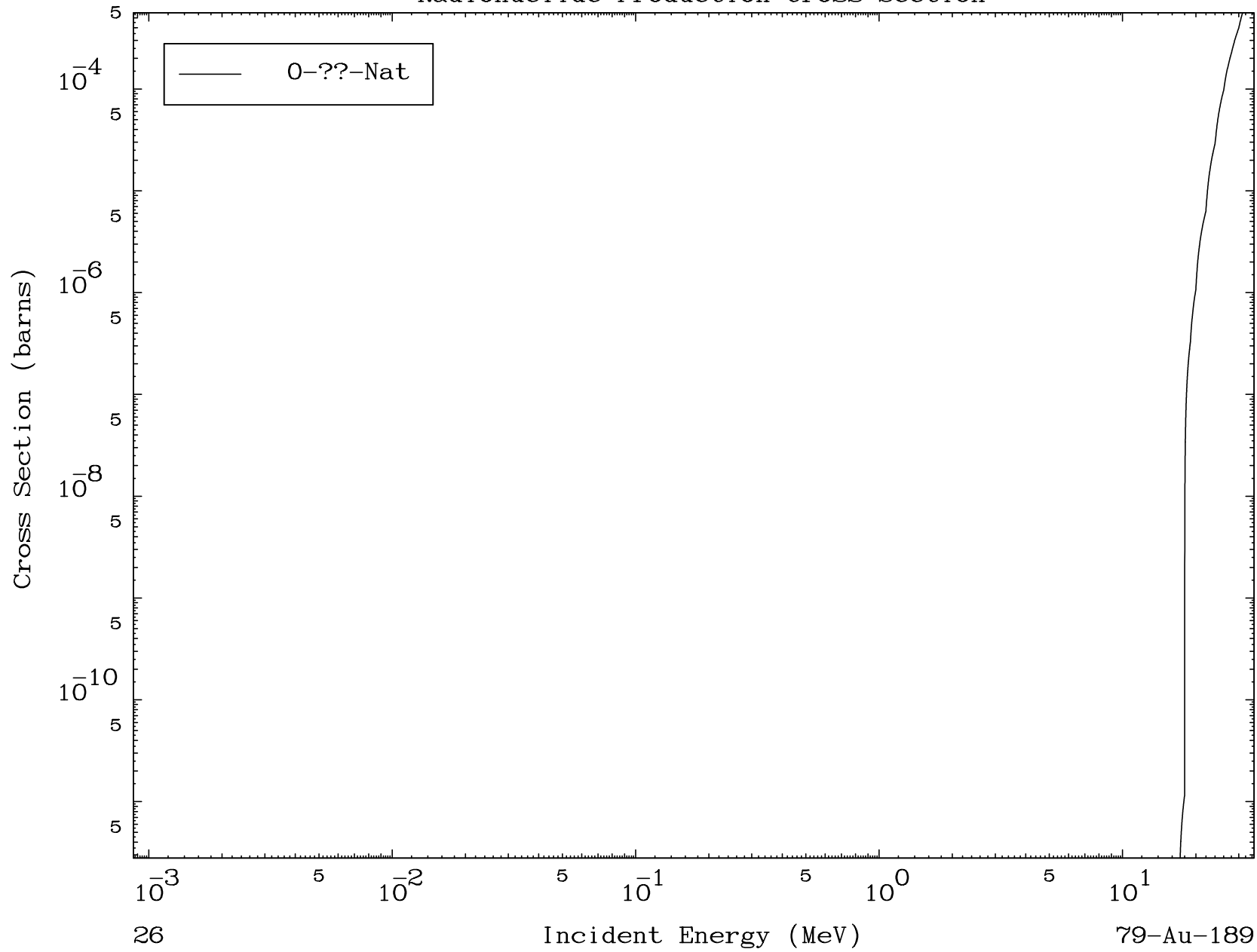
Radionuclide Production Cross Section



MAT 7901

$\alpha$  Fission  
Radionuclide Production Cross Section

79-Au-189

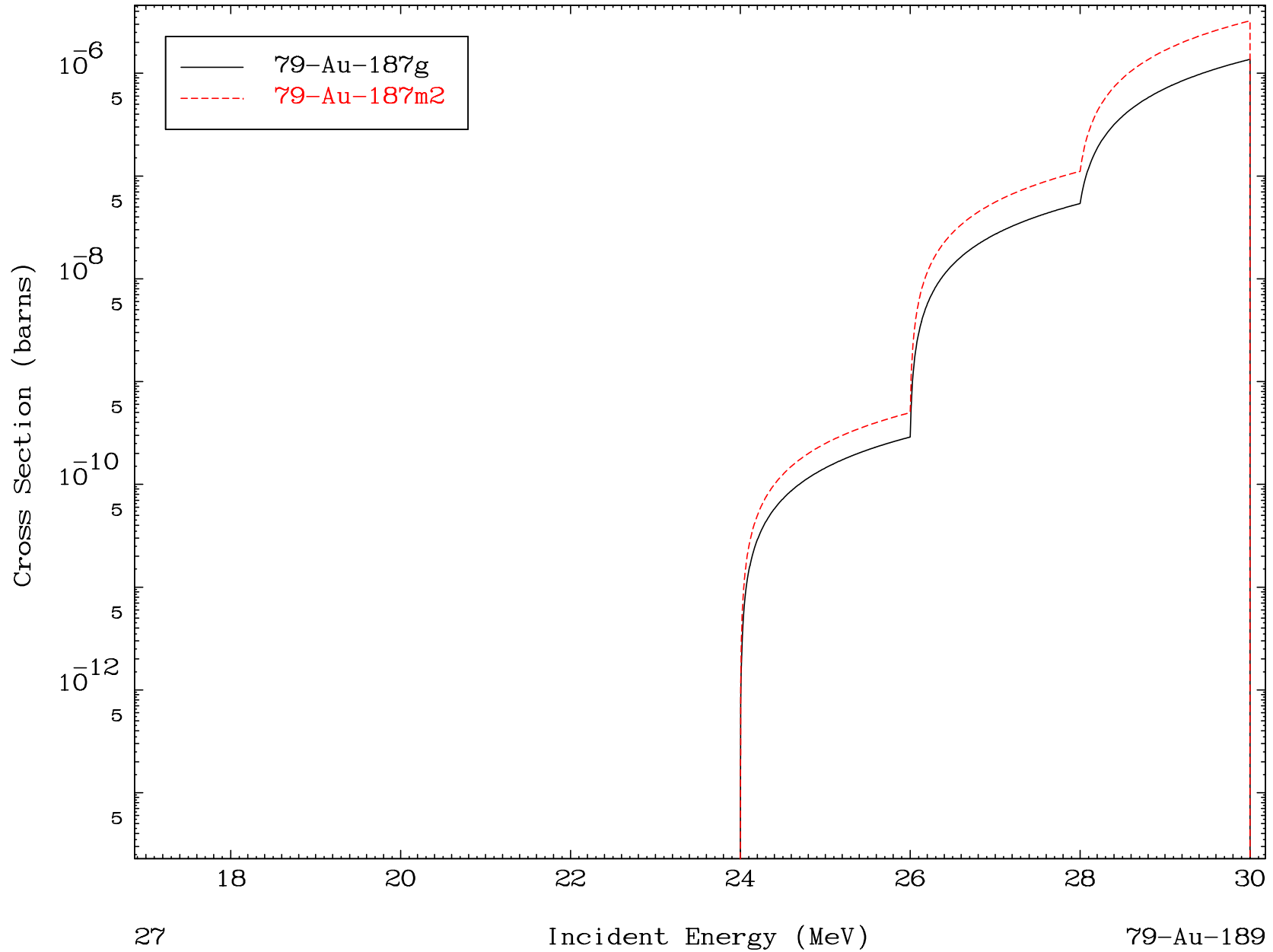


26

Incident Energy (MeV)

79-Au-189

Radionuclide Production Cross Section

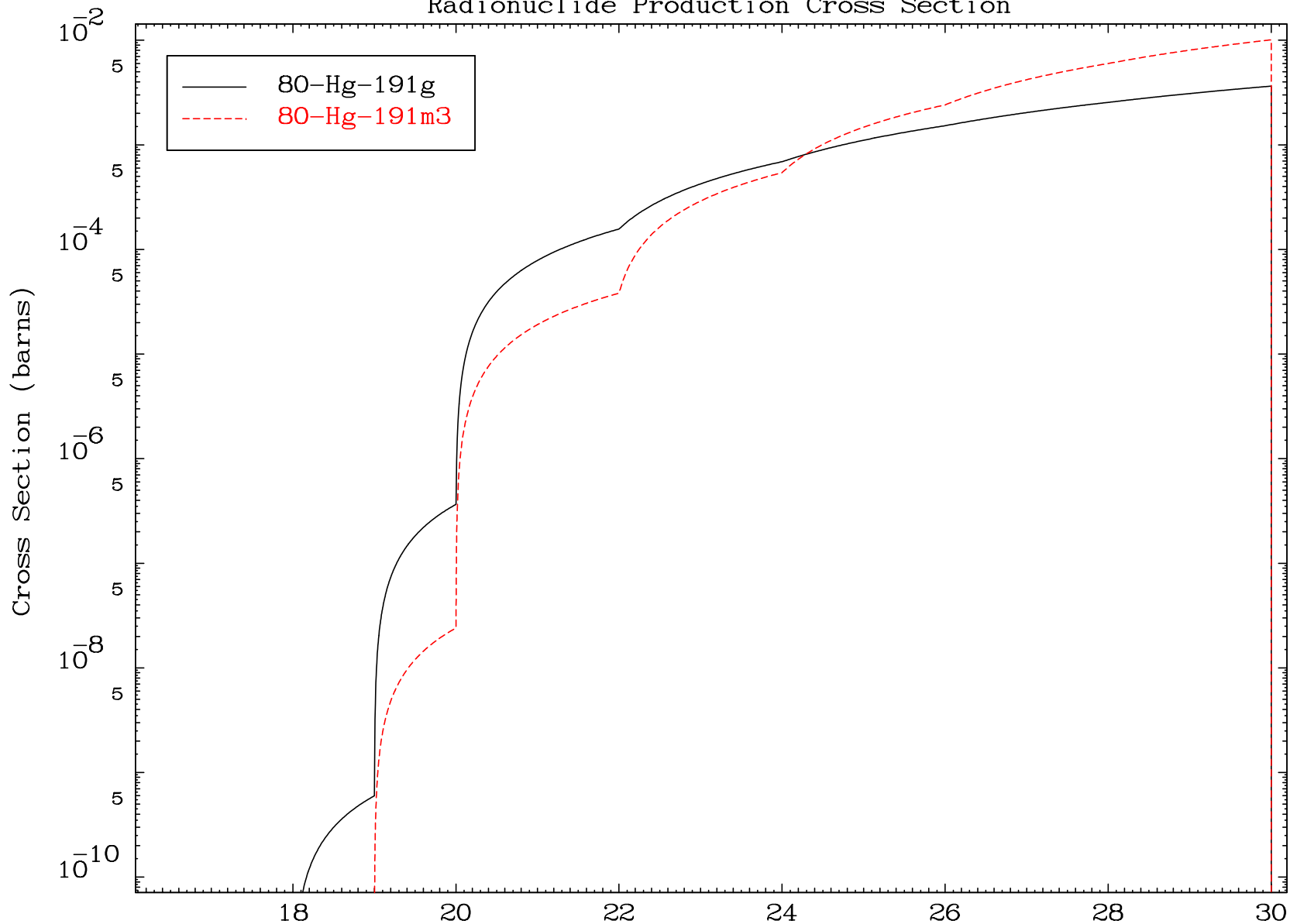


MAT 7901

( $\alpha, n'$ ) p

79-Au-189

Radionuclide Production Cross Section



28

Incident Energy (MeV)

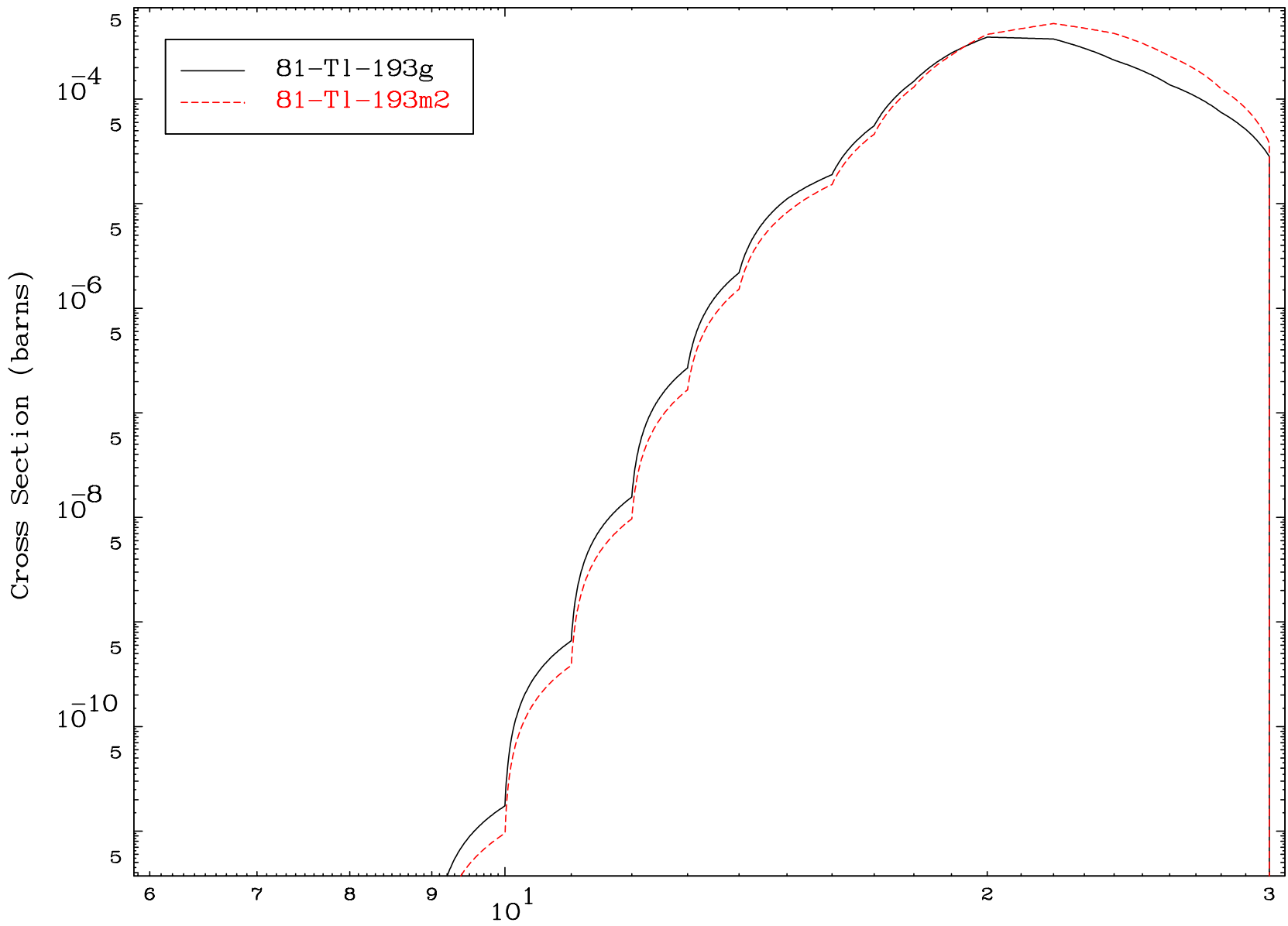
79-Au-189

MAT 7901

( $\alpha, \gamma$ )

79-Au-189

Radionuclide Production Cross Section



29

Incident Energy (MeV)

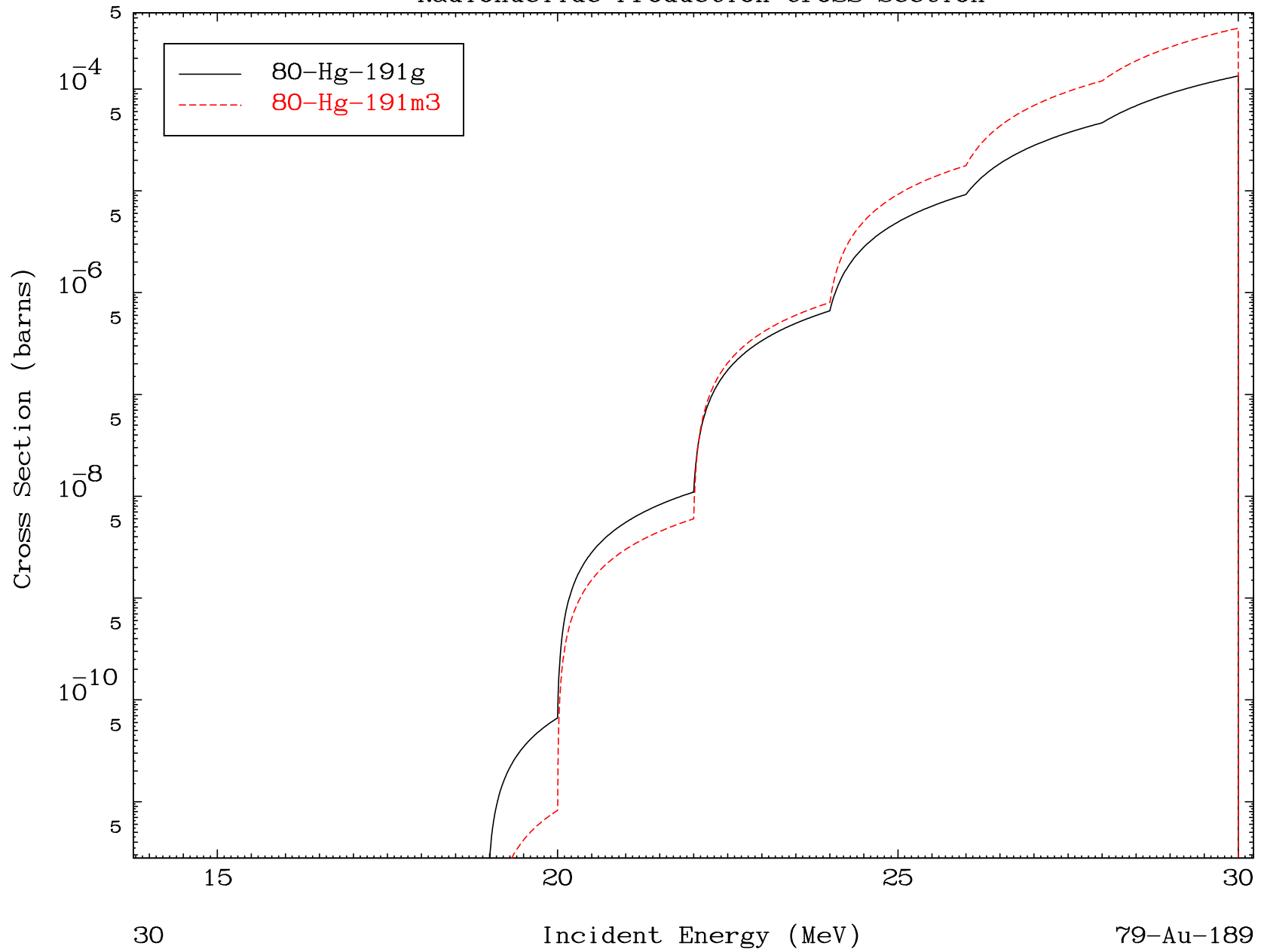
79-Au-189

MAT 7901

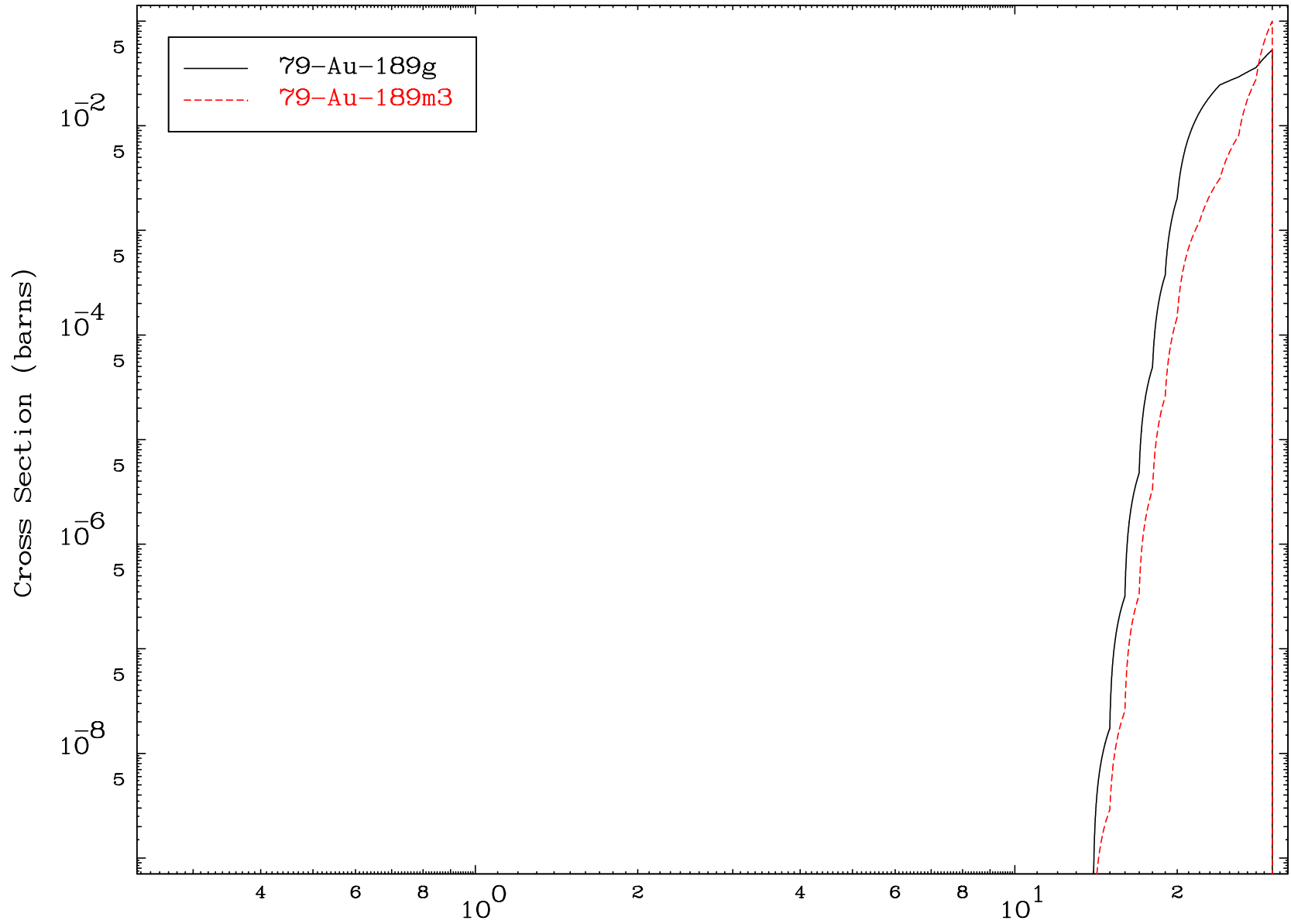
( $\alpha, d$ )

79-Au-189

Radionuclide Production Cross Section



Radionuclide Production Cross Section



MAT 7901

( $\alpha, 2p$ )

79-Au-189

Radionuclide Production Cross Section

