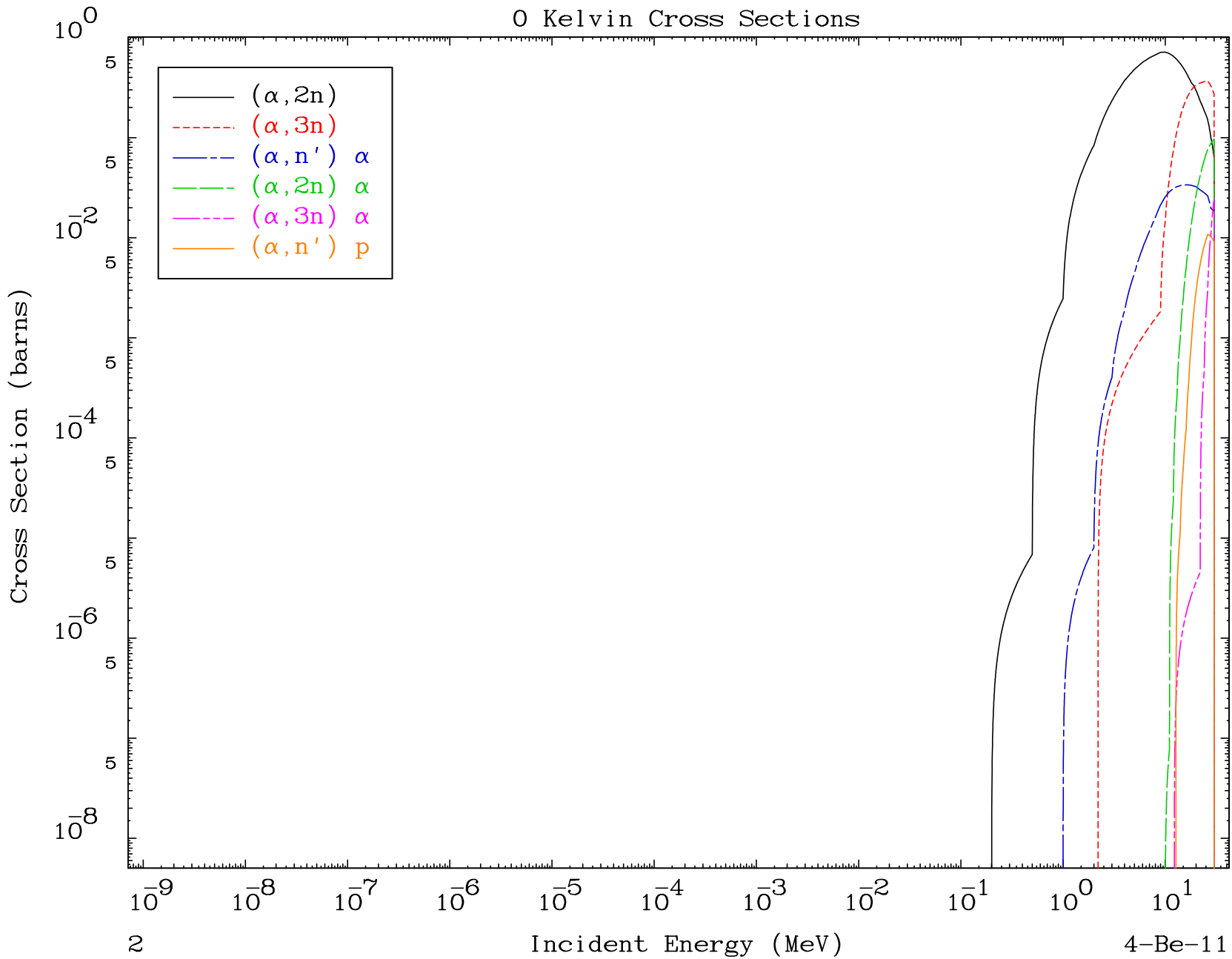


MAT 431

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

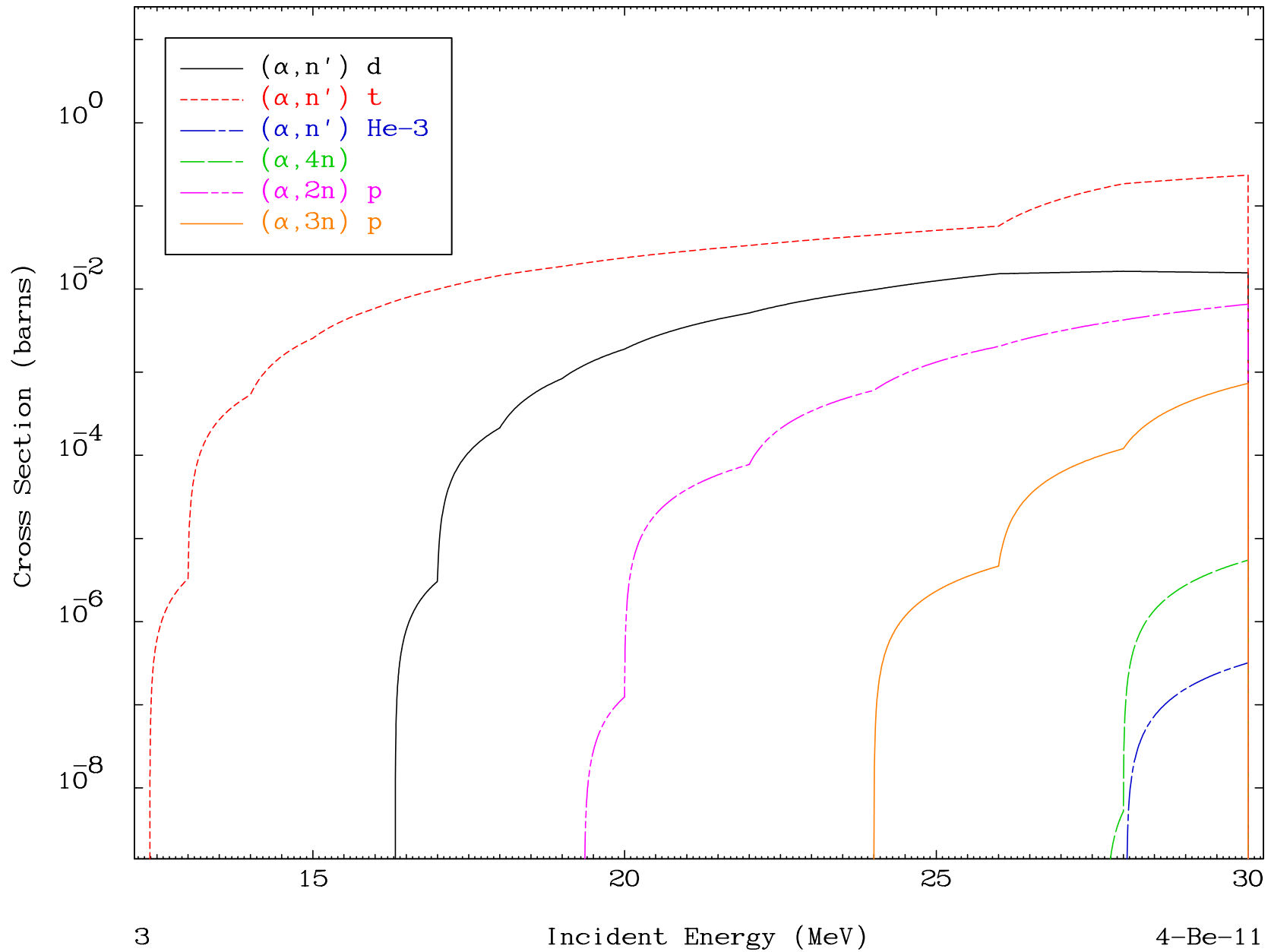
4-Be-11

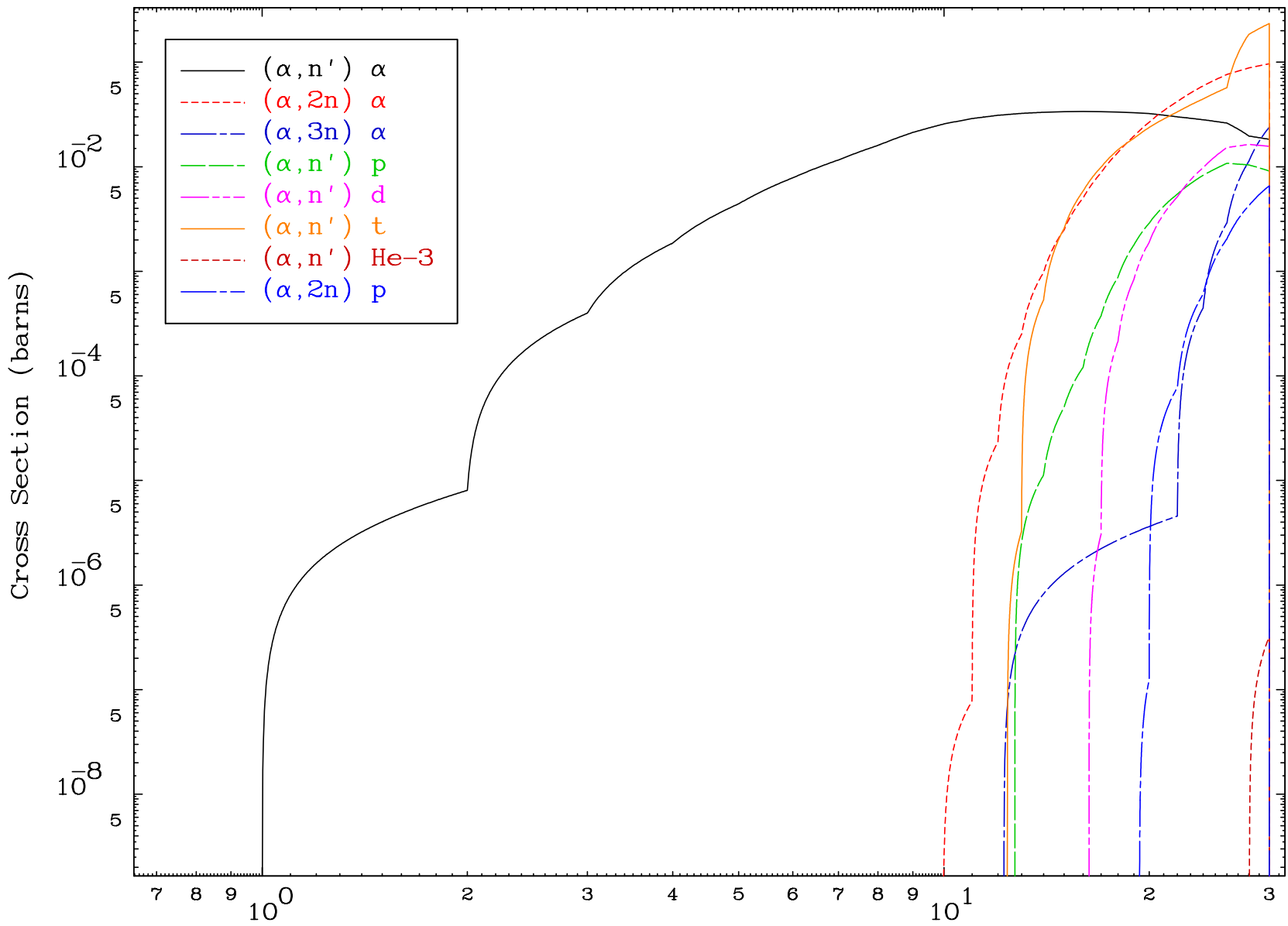


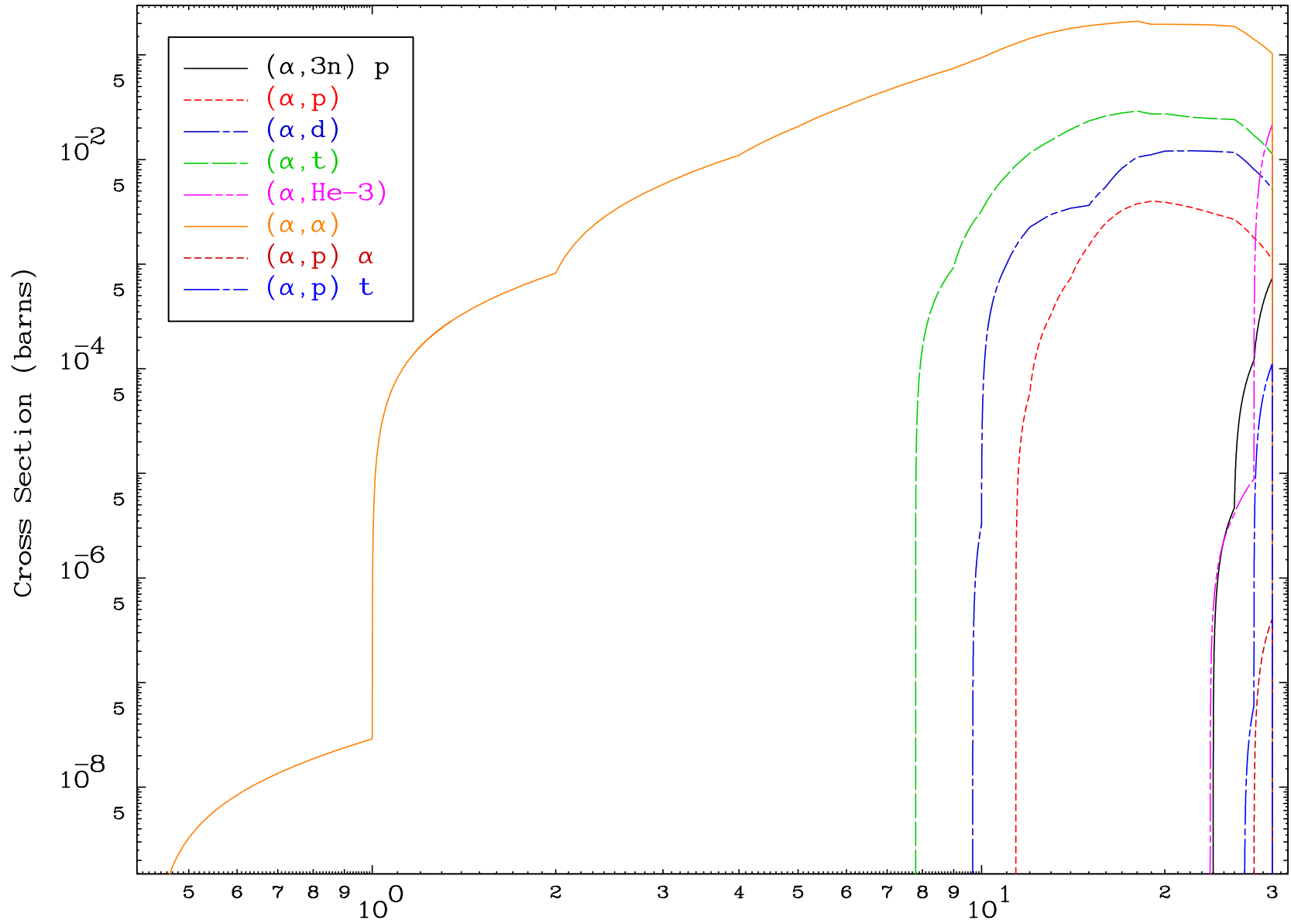
2

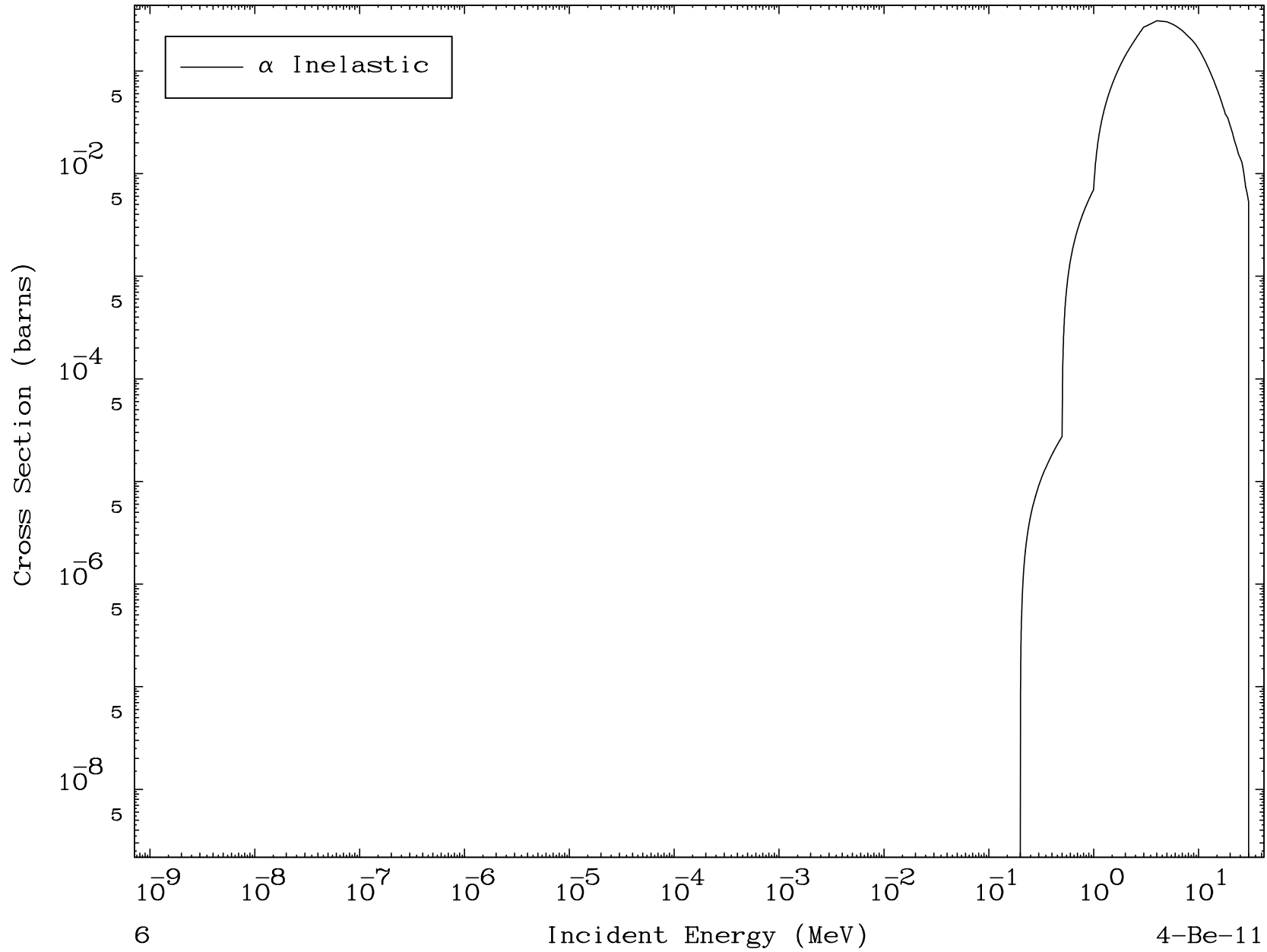
Incident Energy (MeV)

4-Be-11





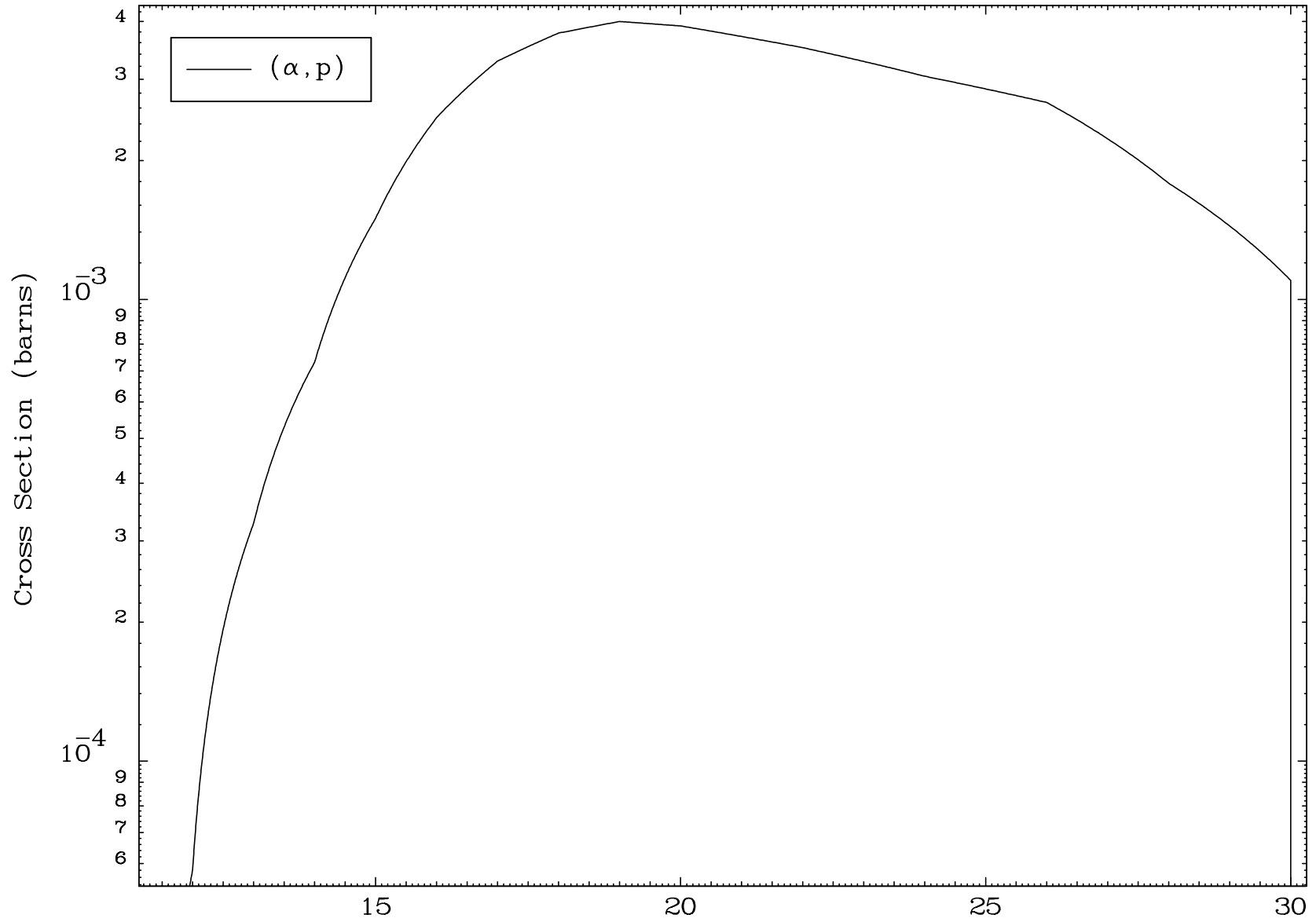




MAT 431

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

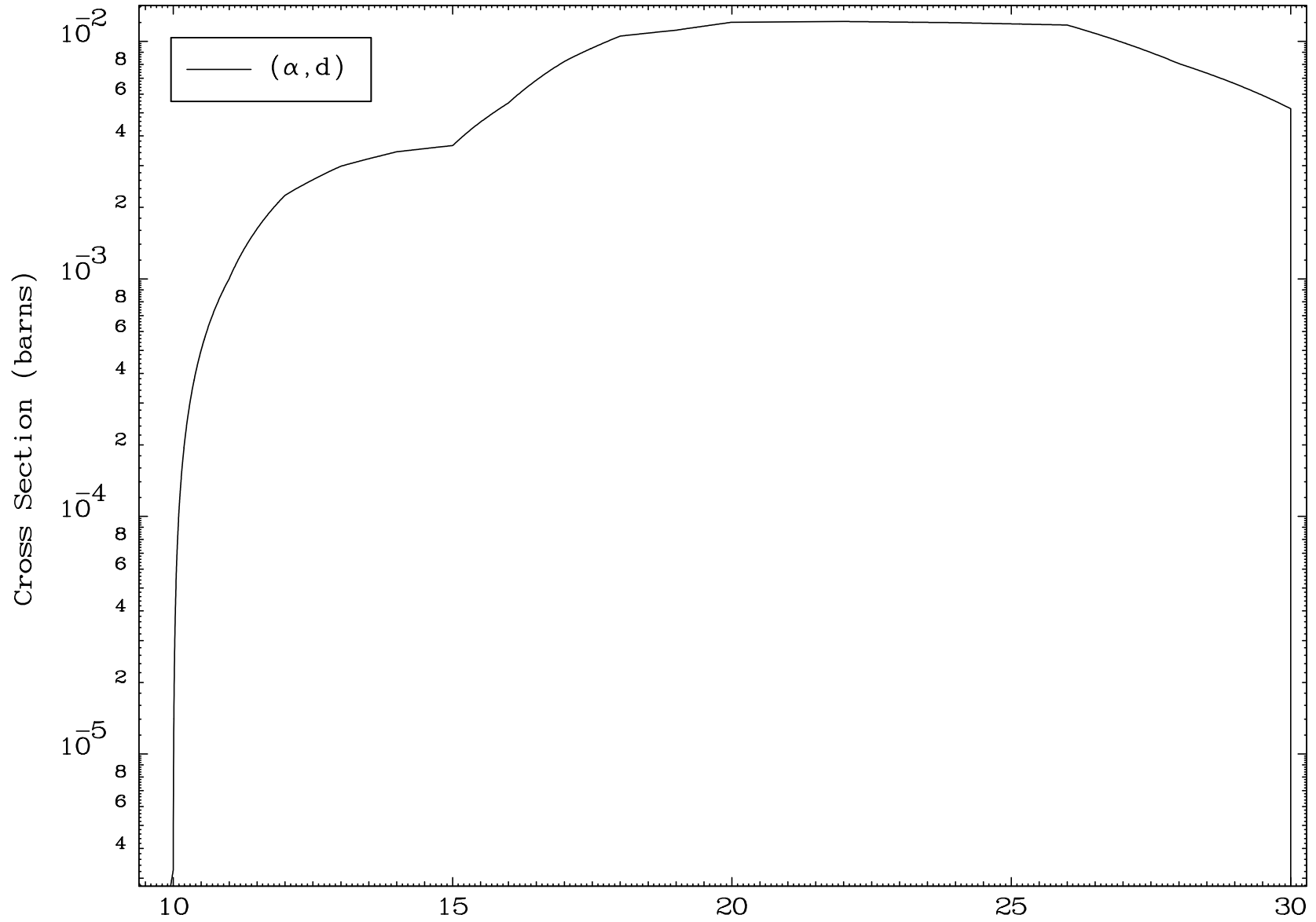
4-Be-11



7

Incident Energy (MeV)

4-Be-11

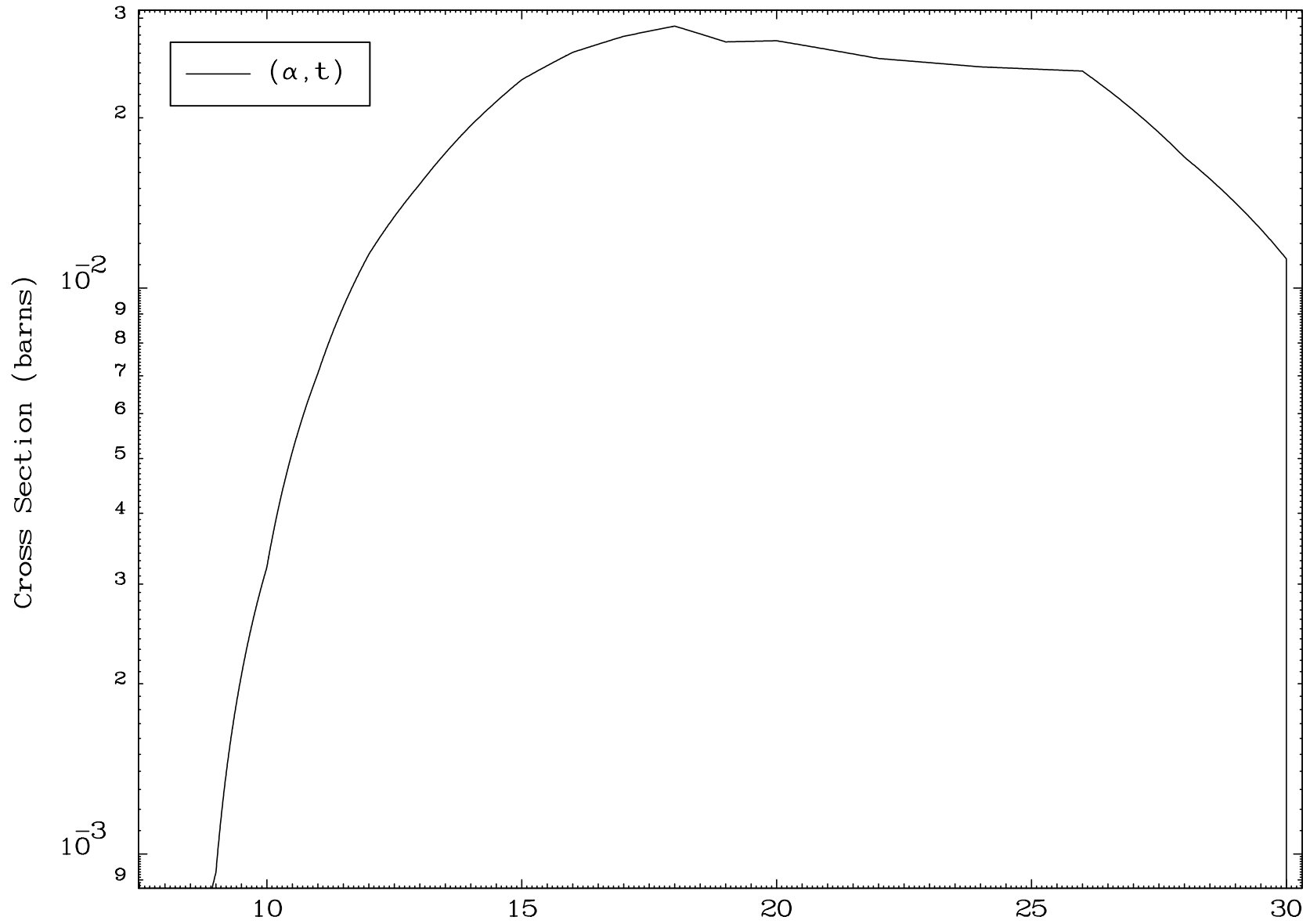




MAT 431

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

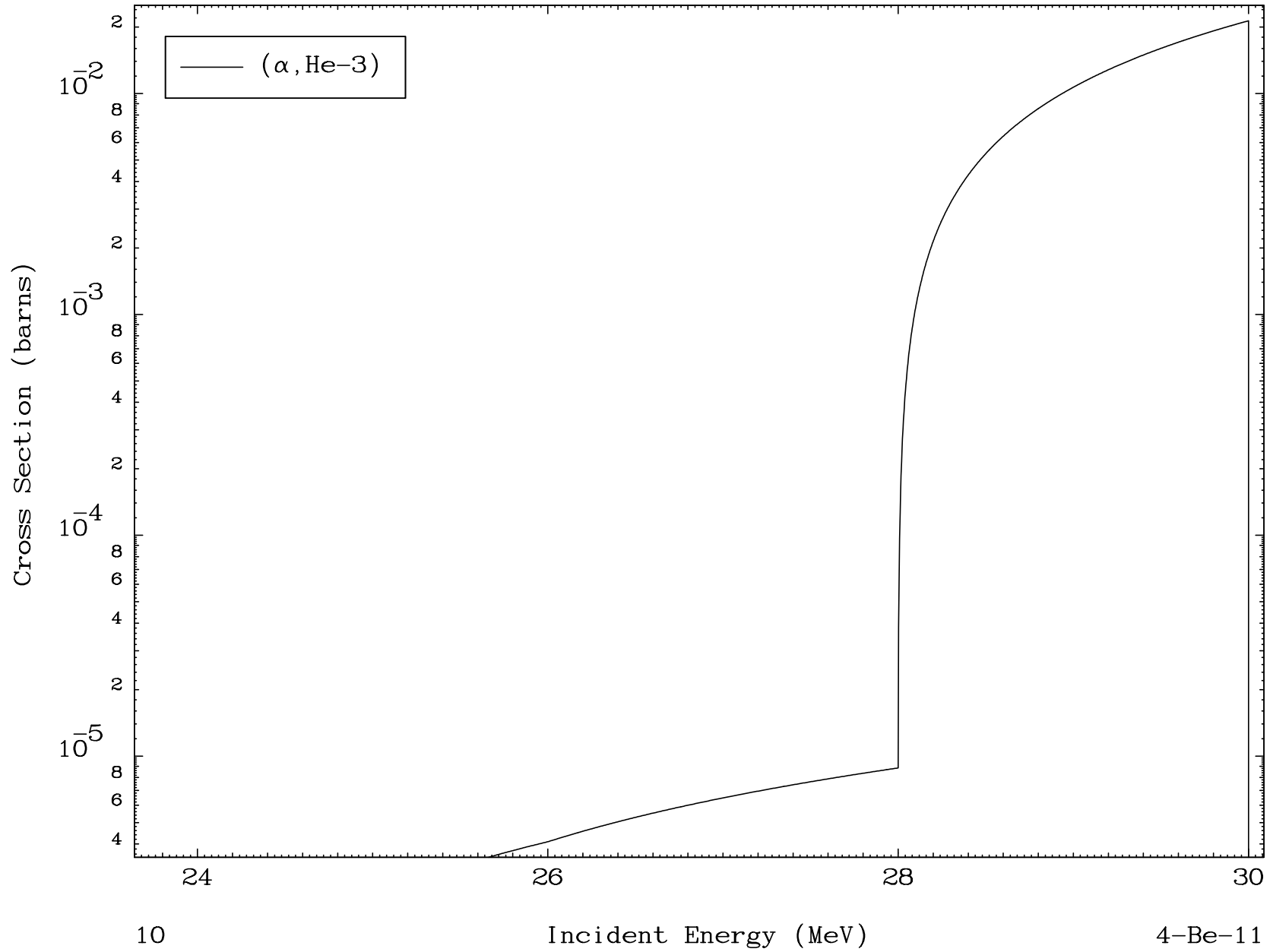
4-Be-11

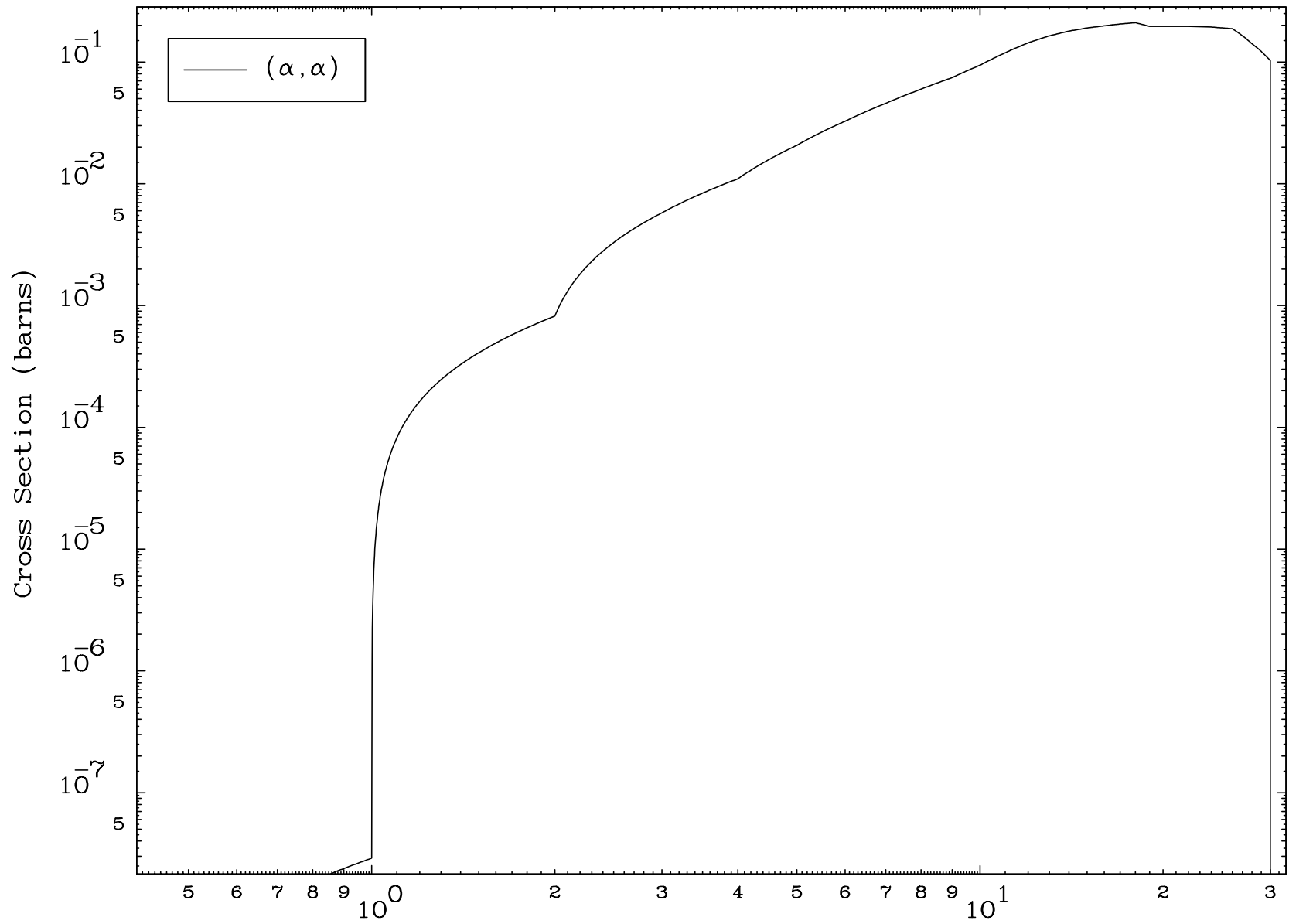


9

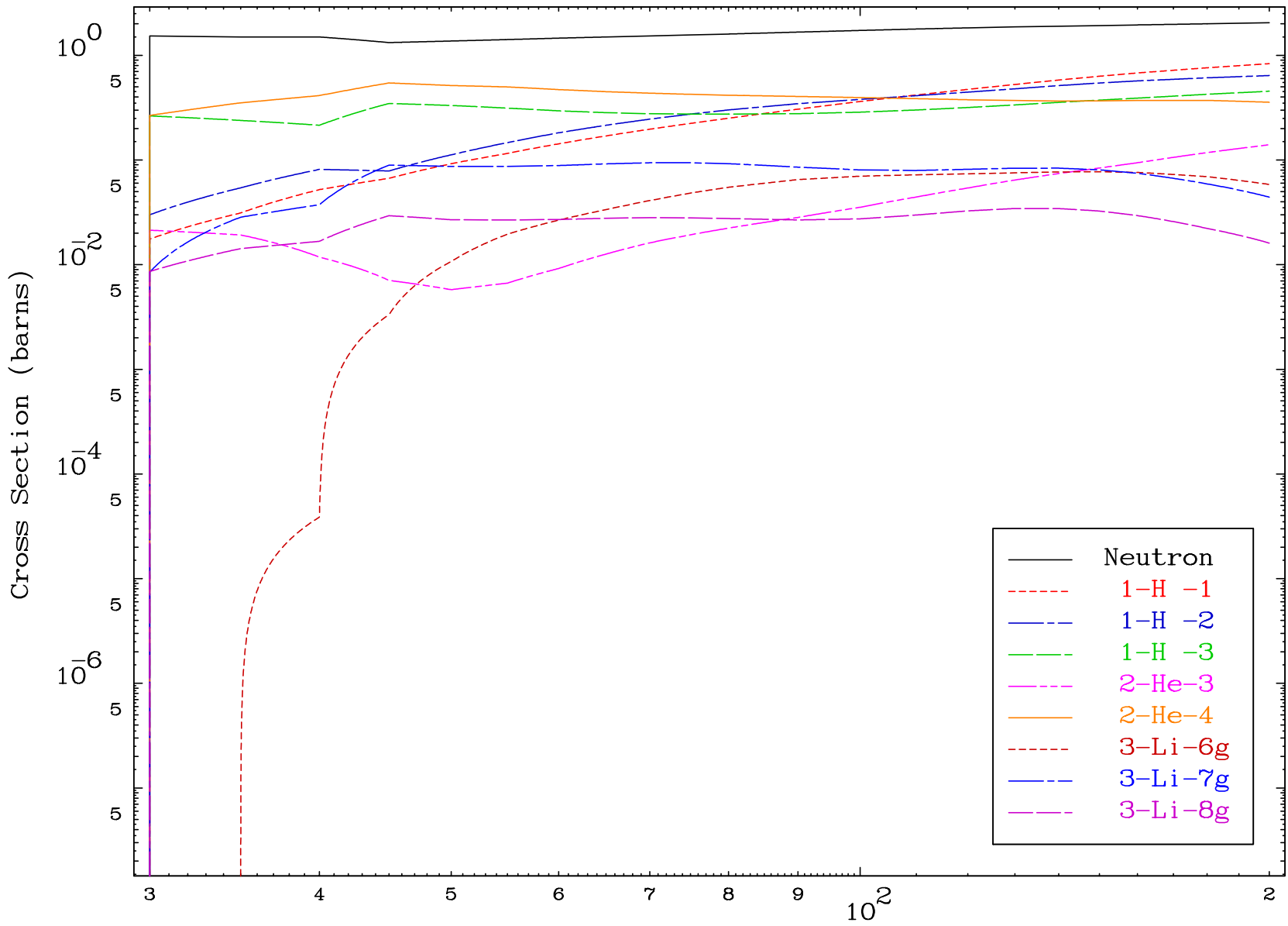
Incident Energy (MeV)

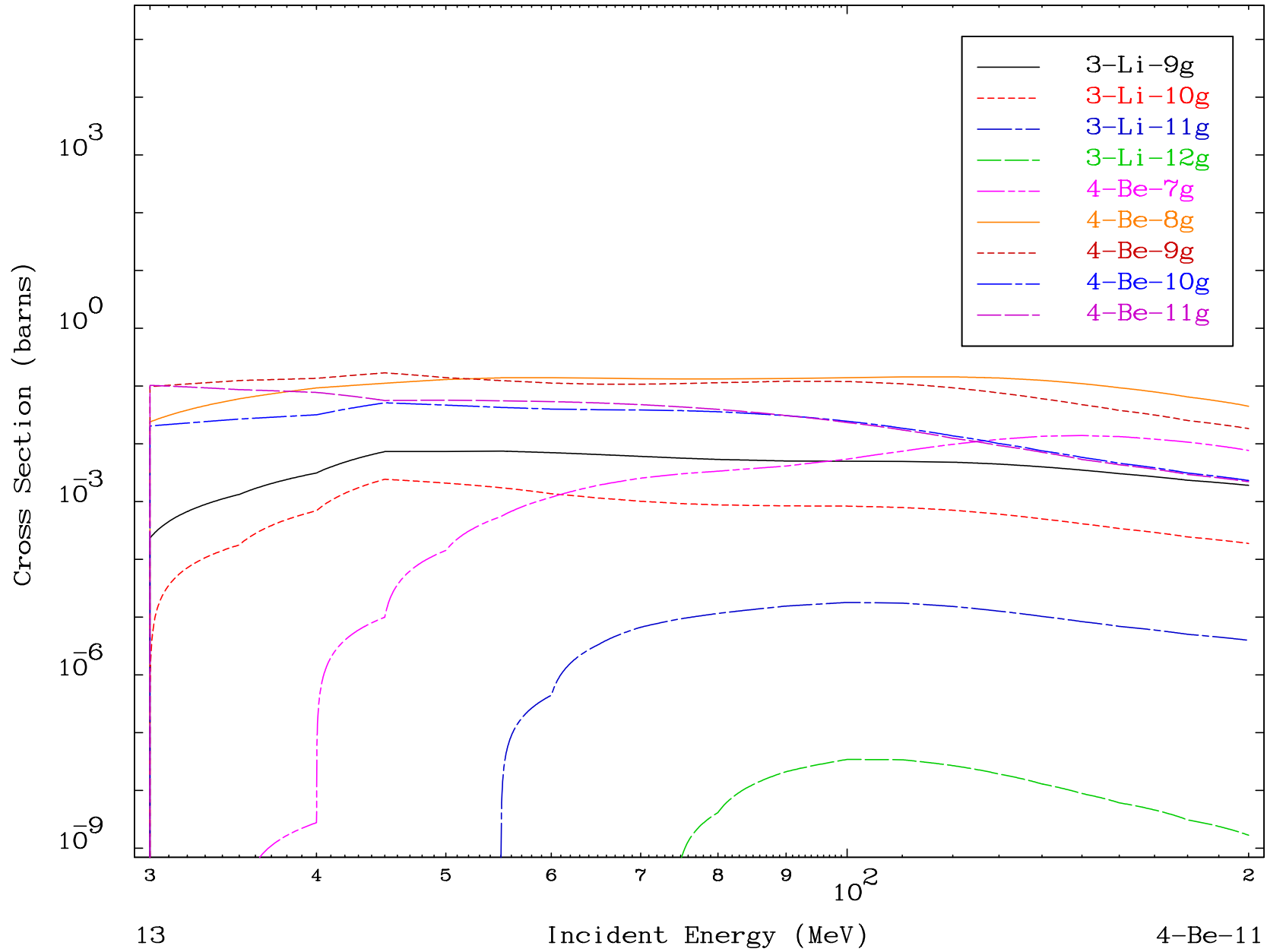
4-Be-11





Radionuclide Production Cross Section





Radionuclide Production Cross Section

