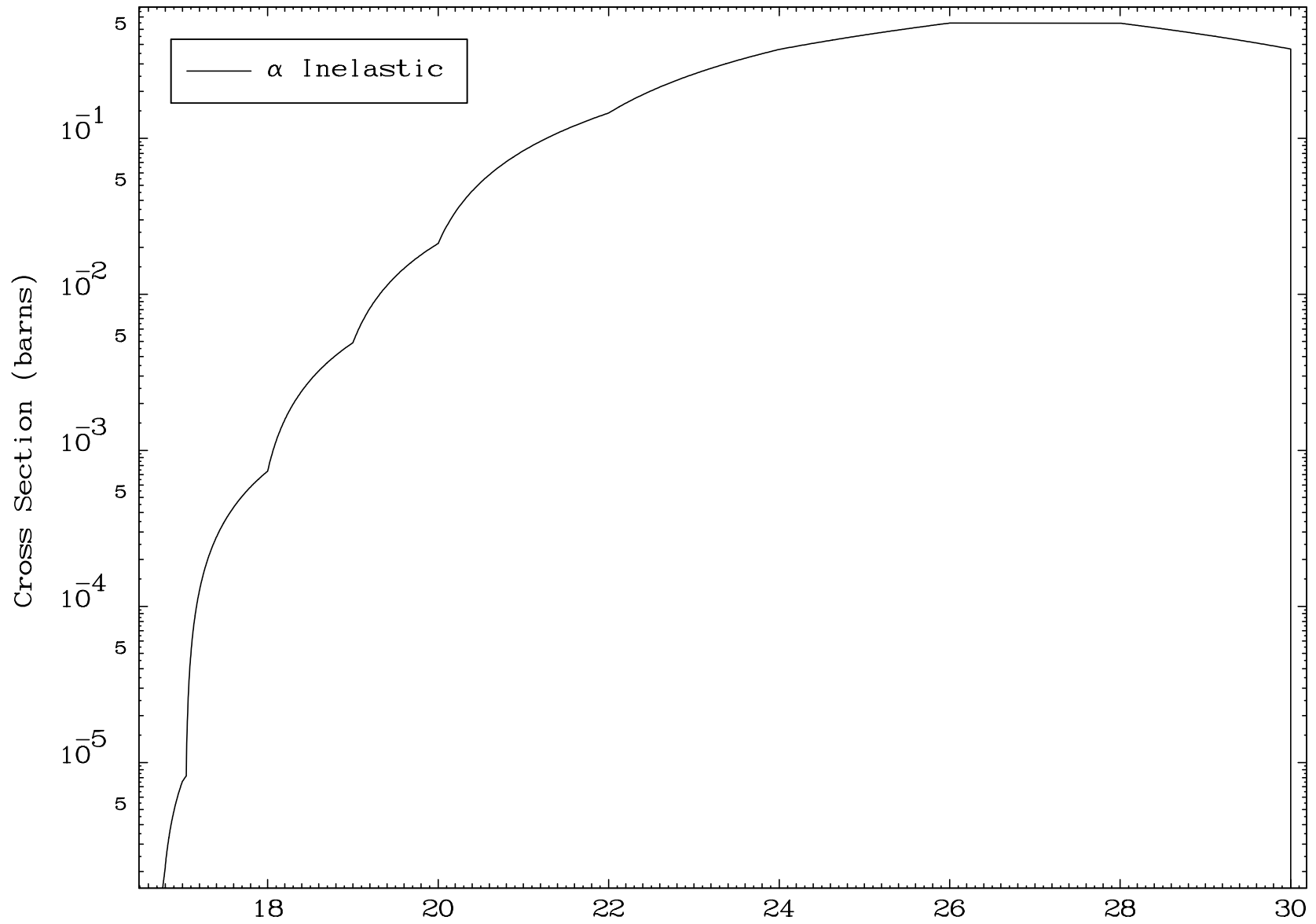


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( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

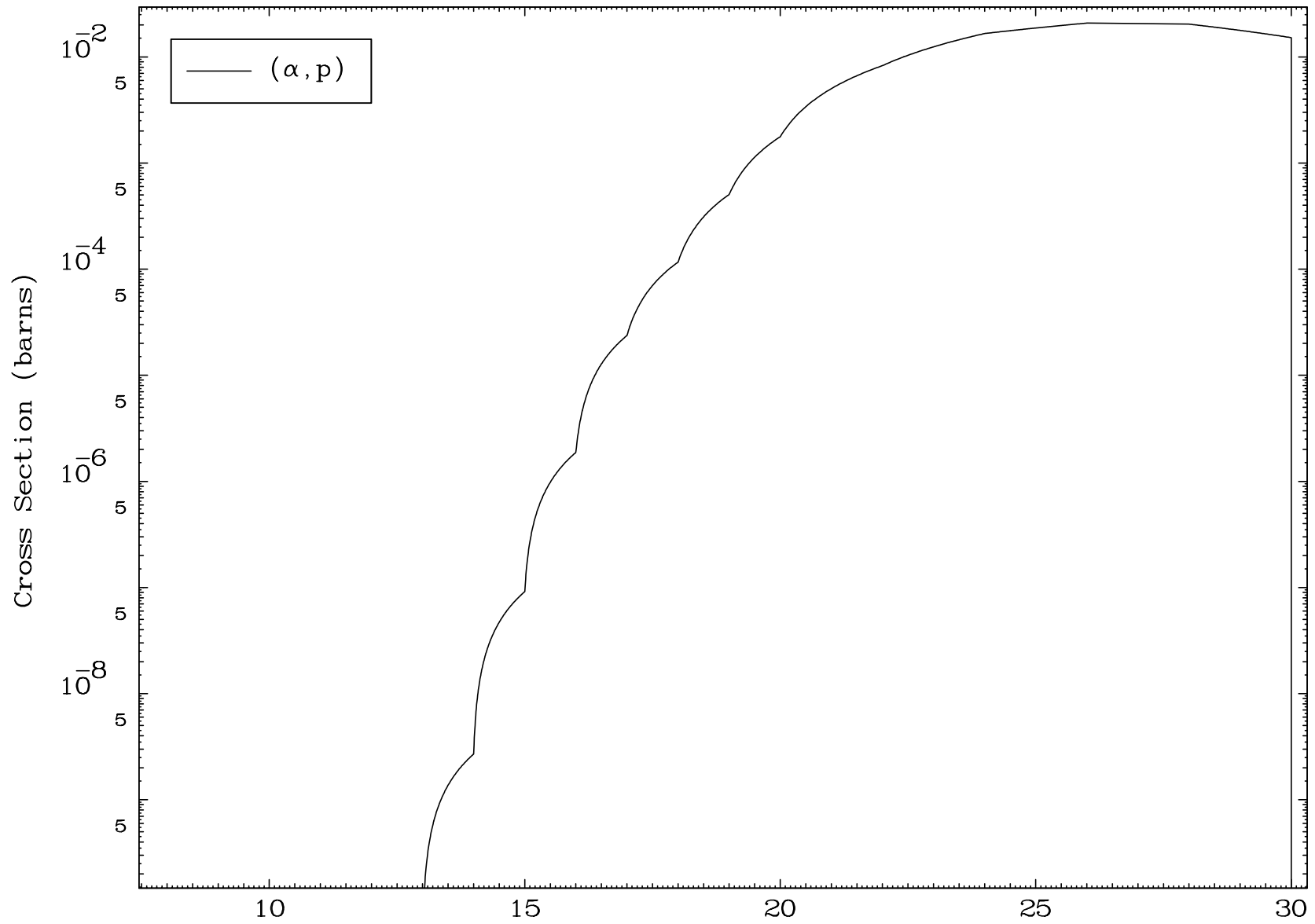
83-Bi-197

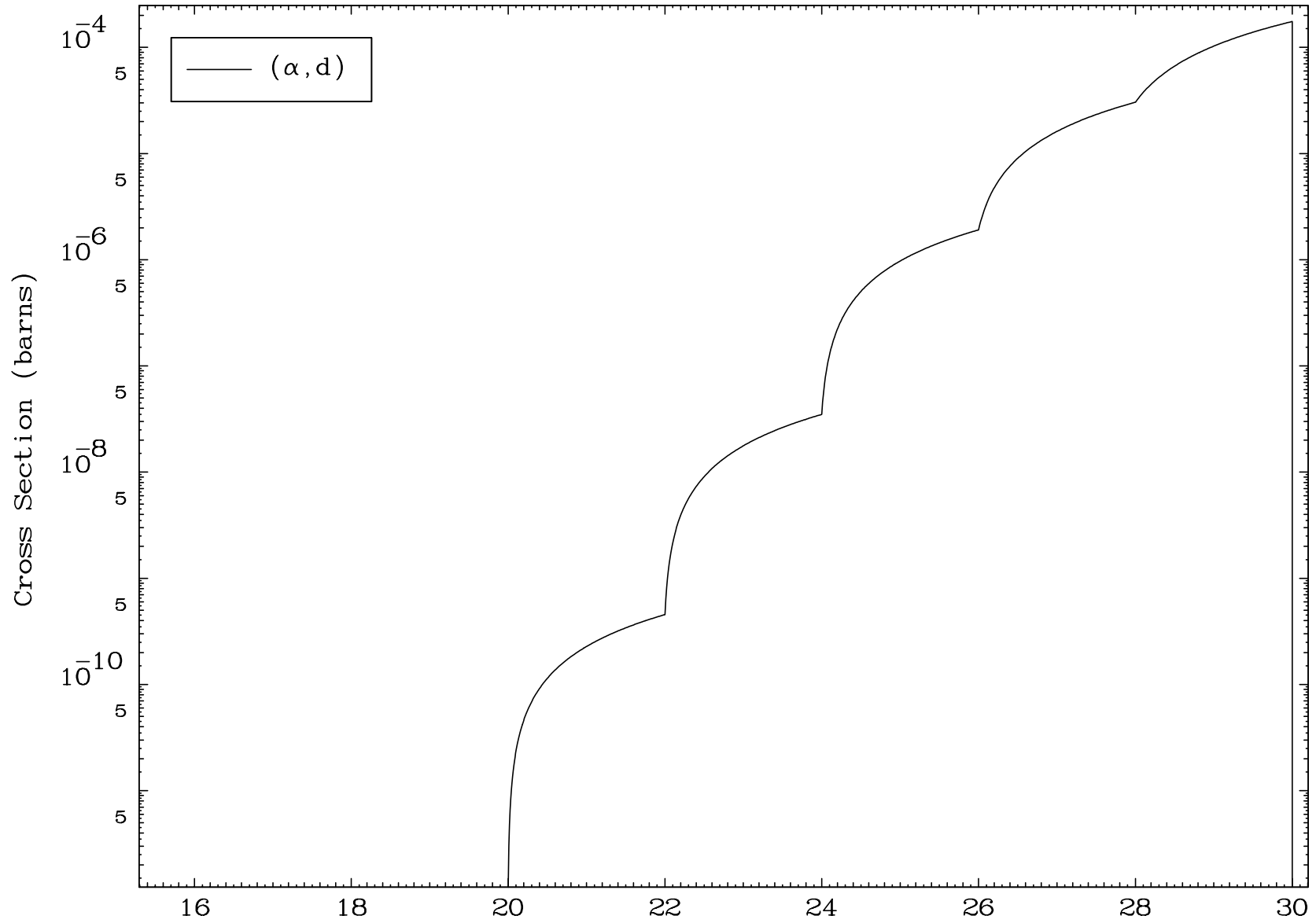


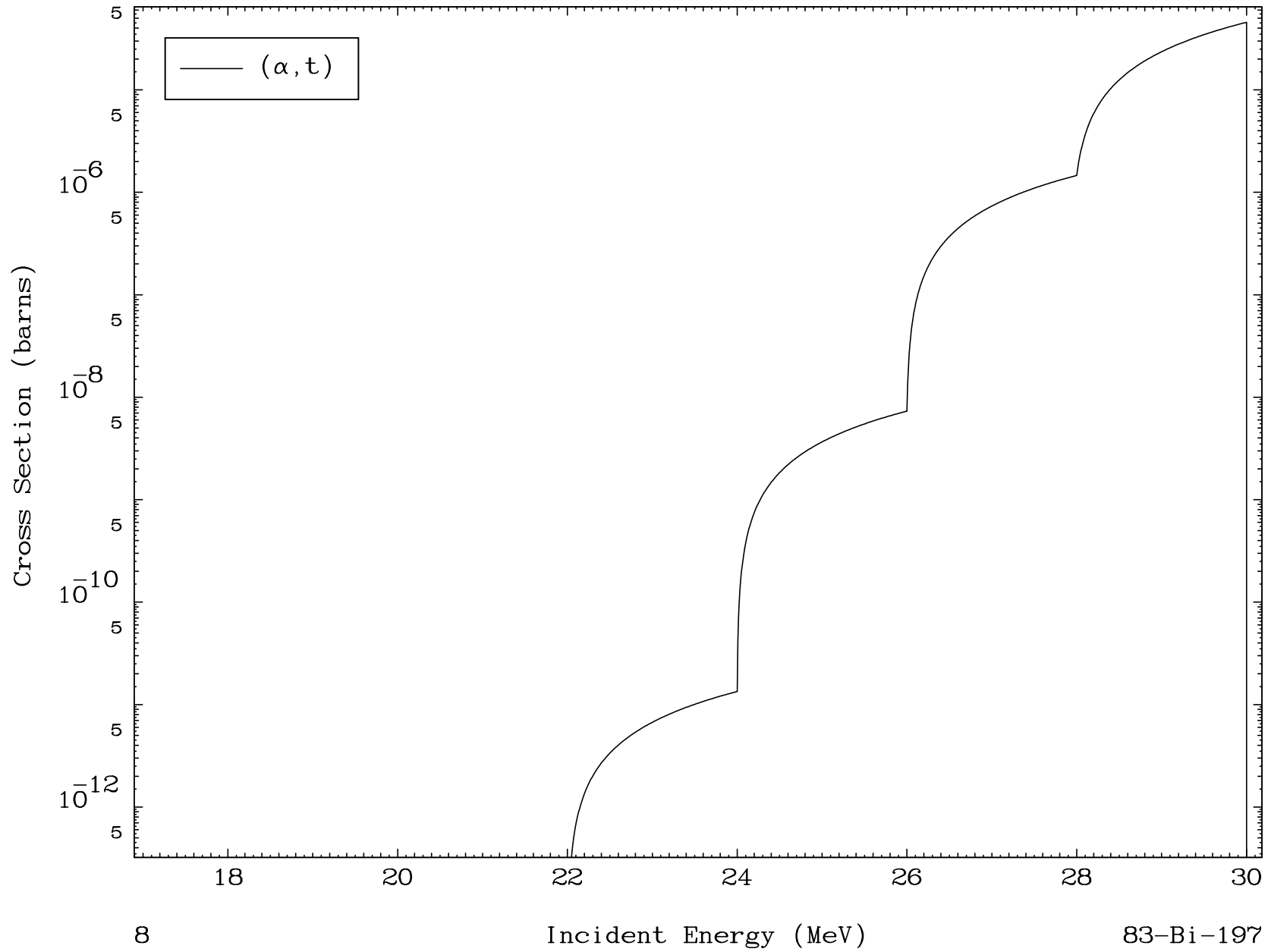
5

Incident Energy (MeV)

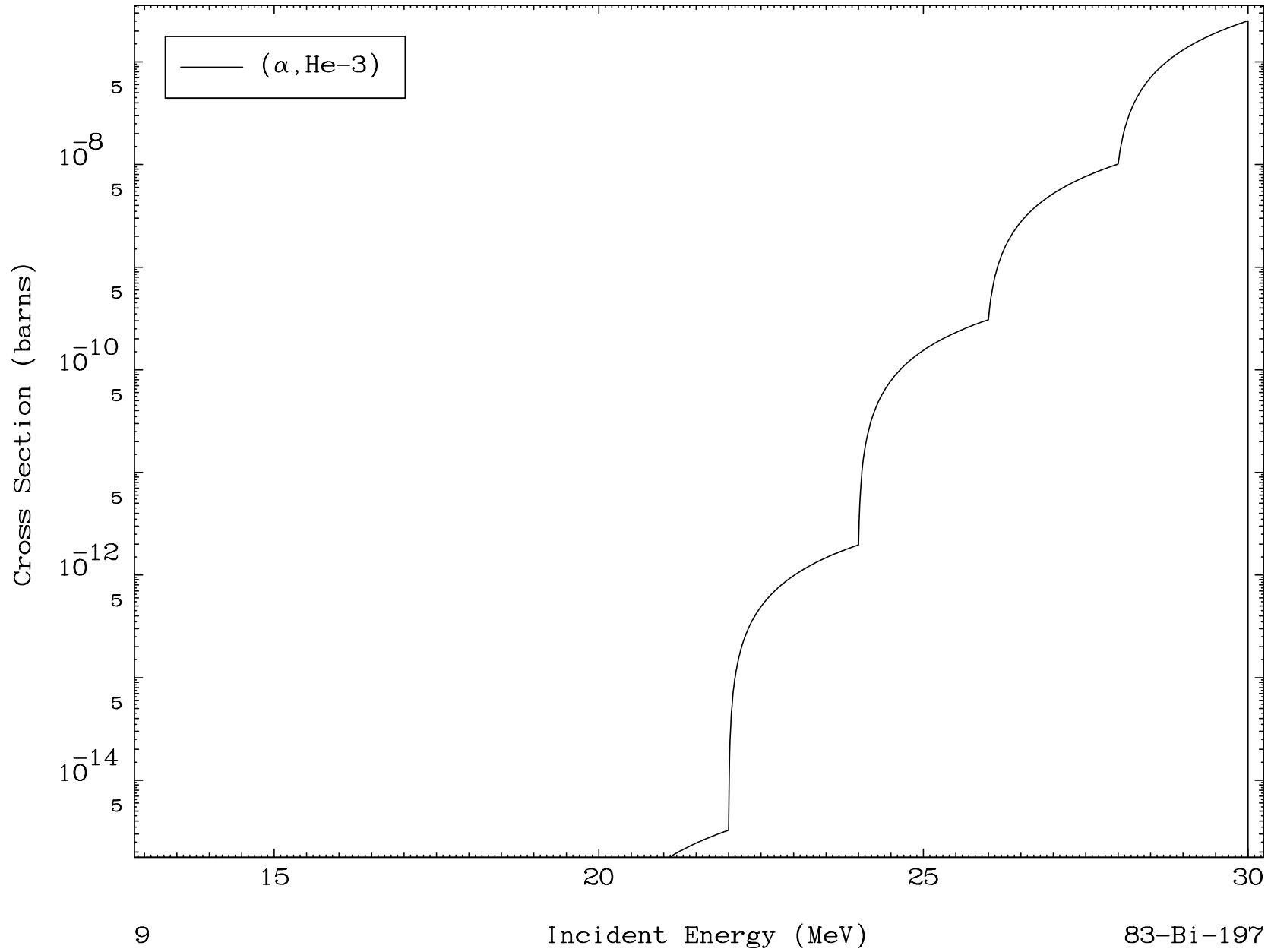
83-Bi-197







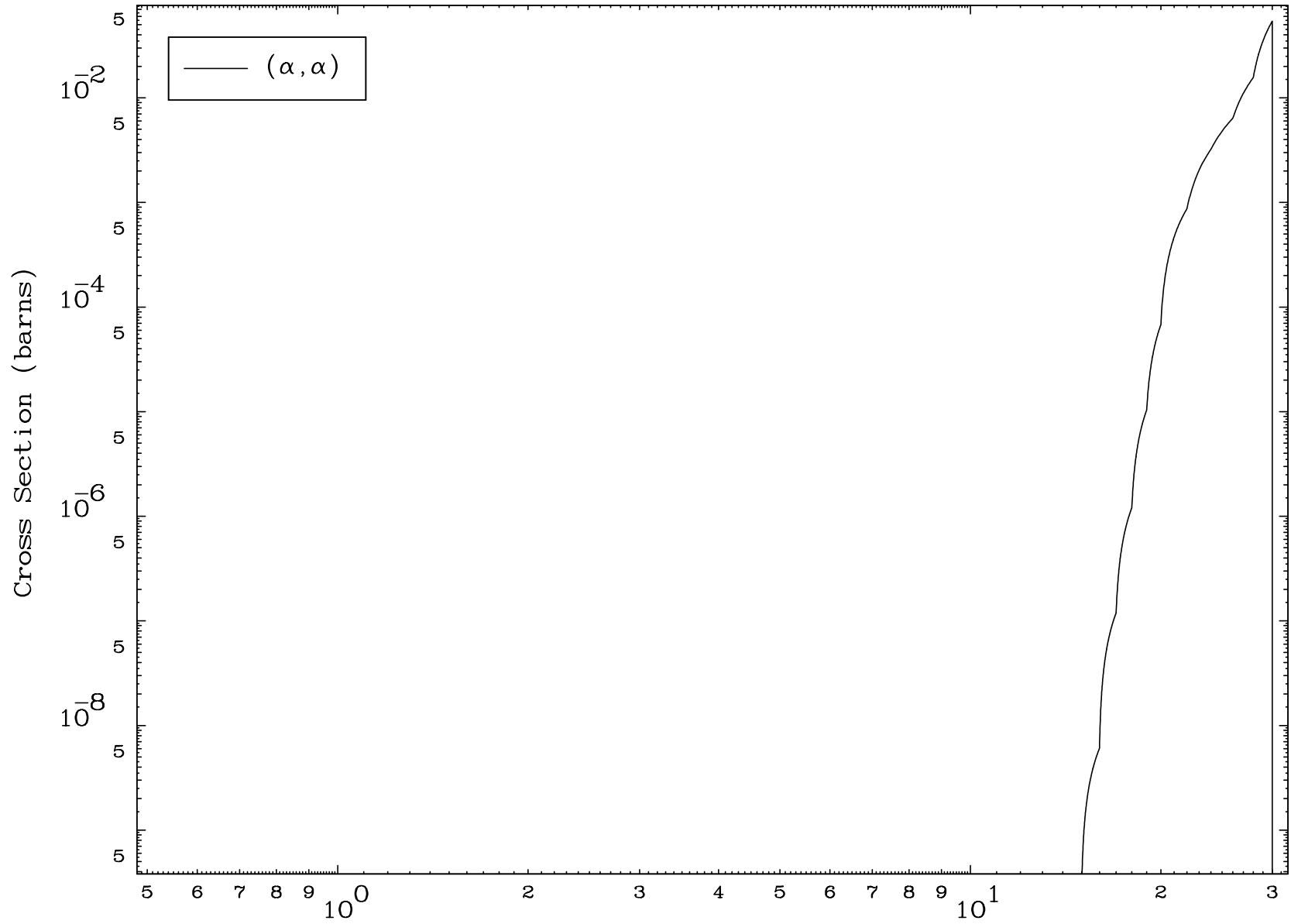




MAT 8289

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

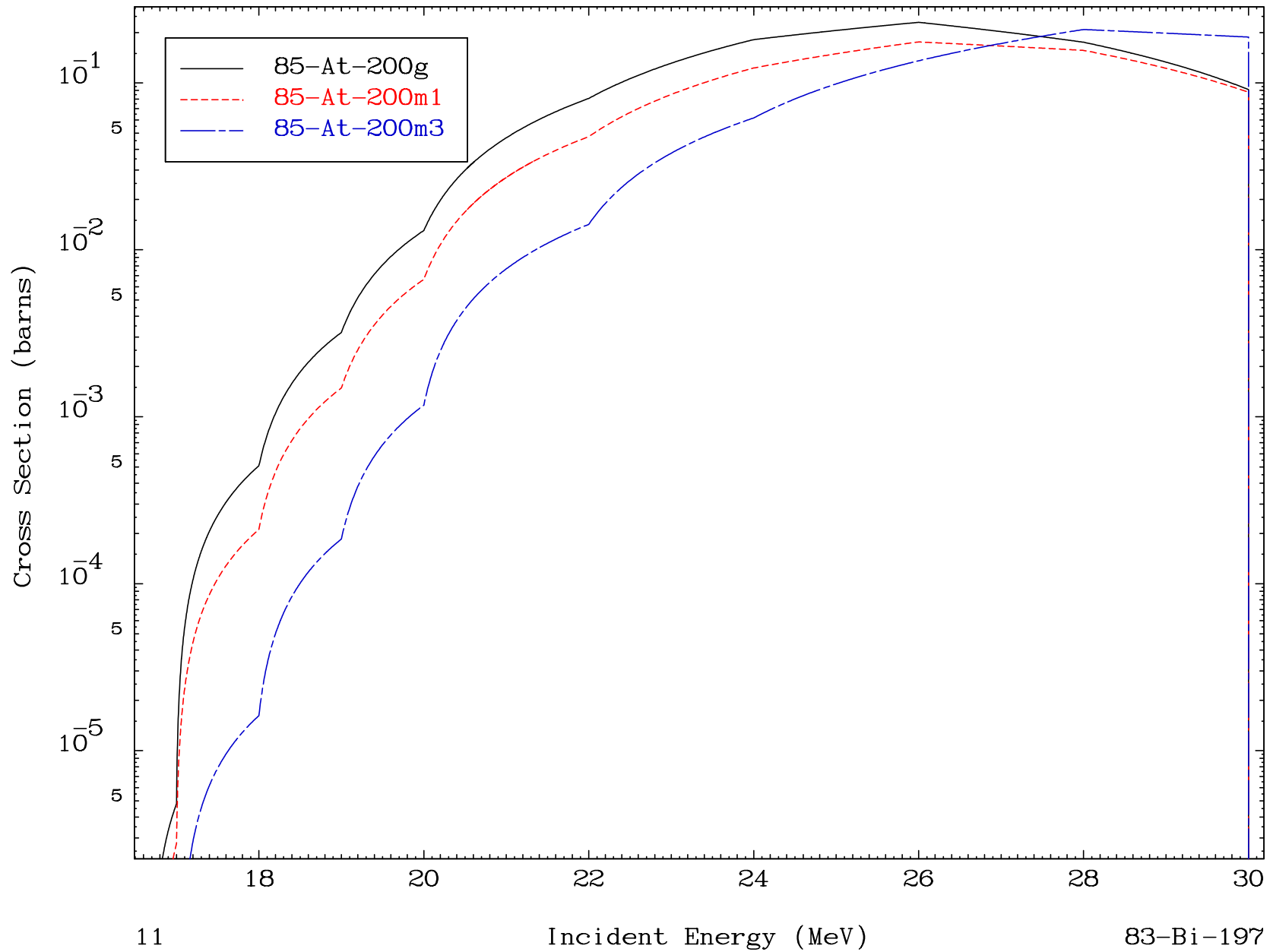
83-Bi-197



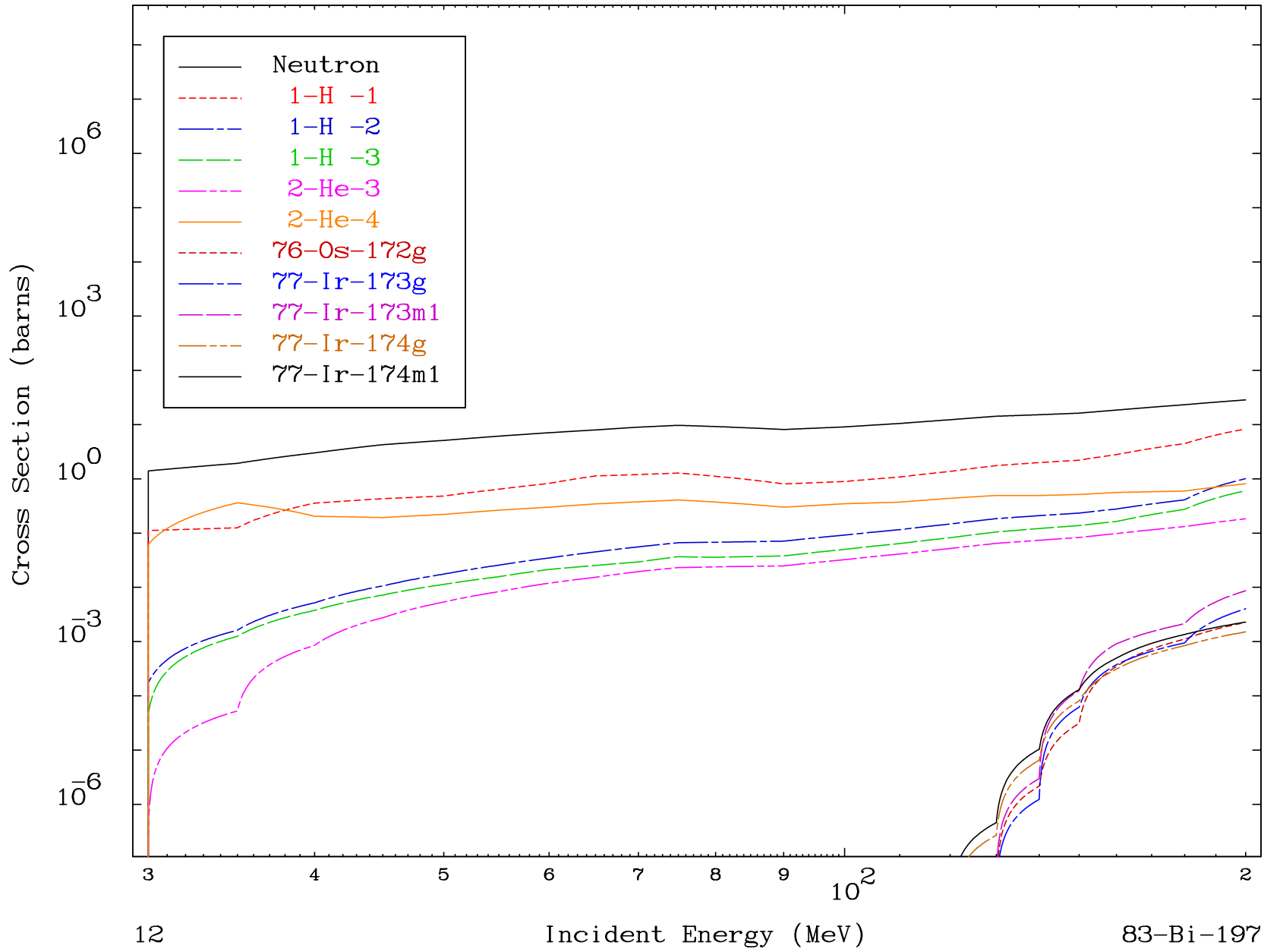
10

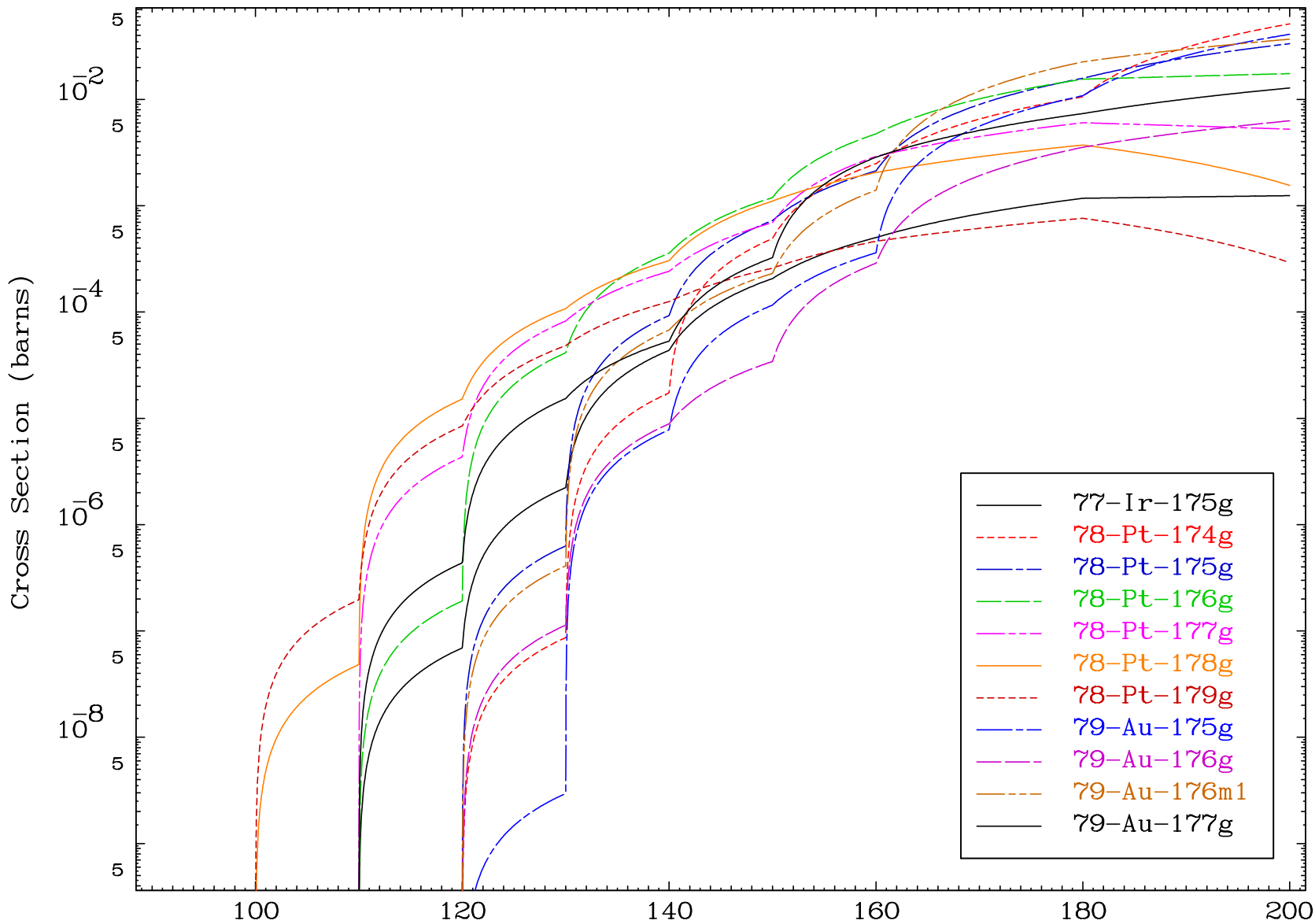
Incident Energy (MeV)

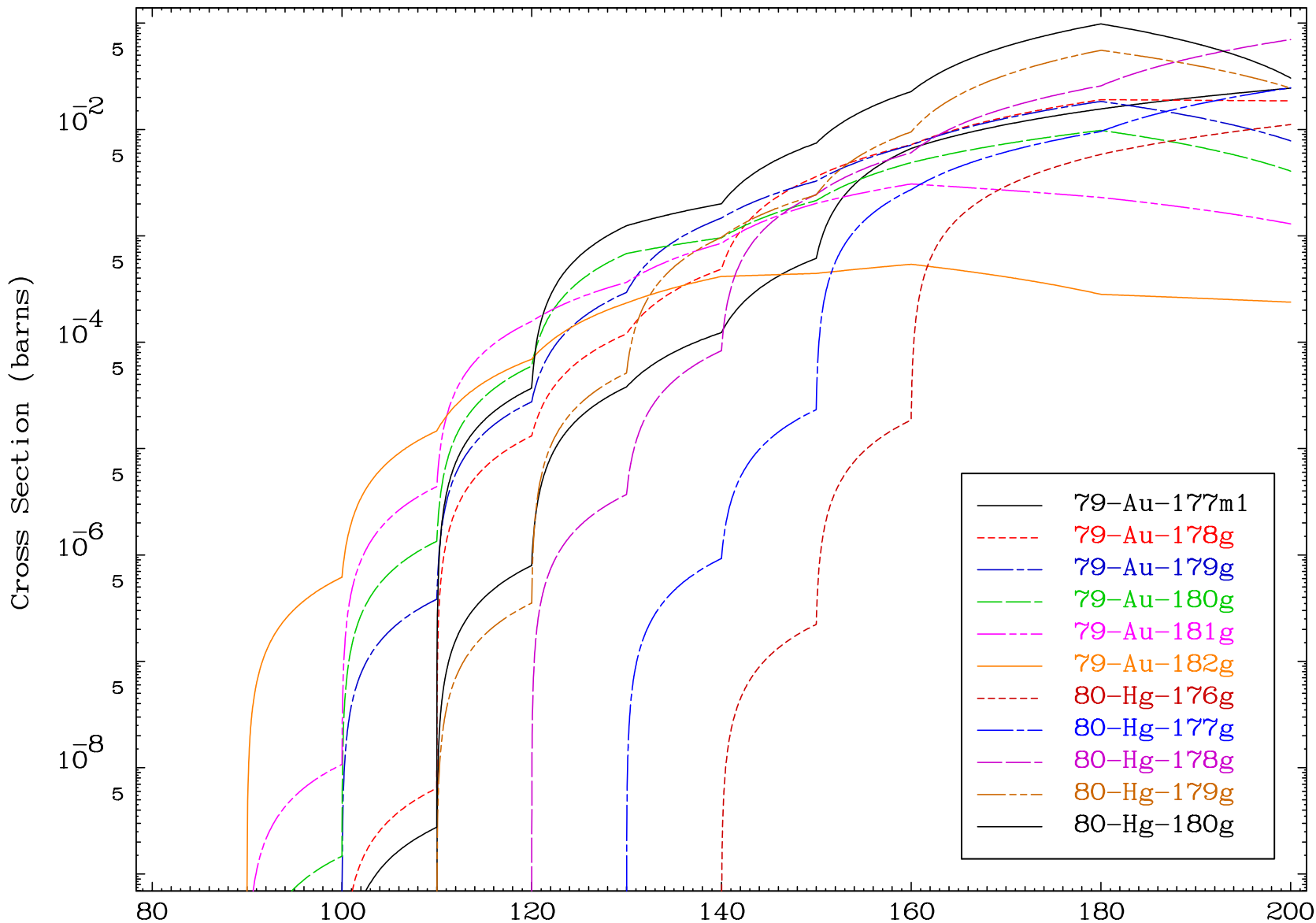
83-Bi-197

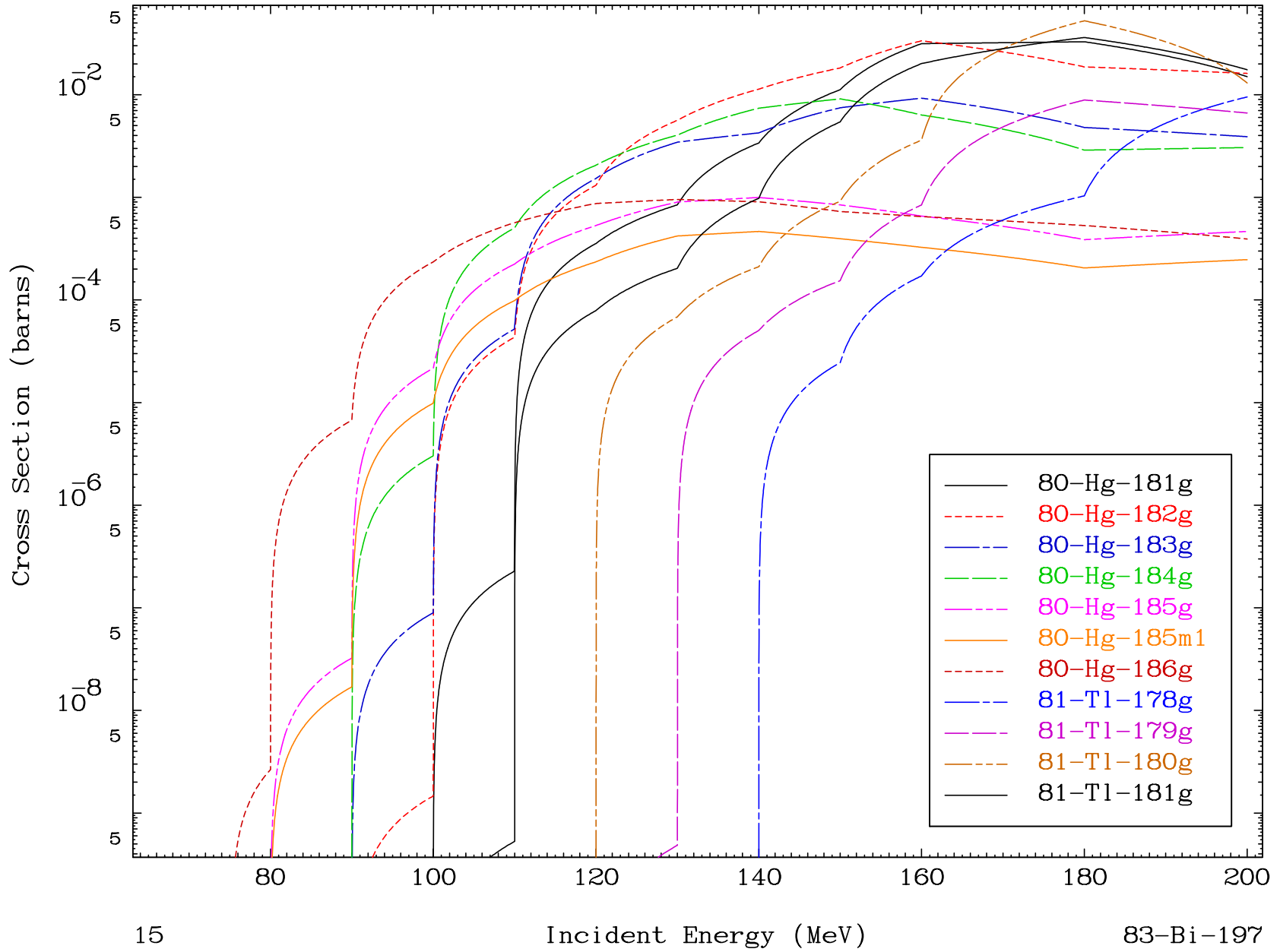


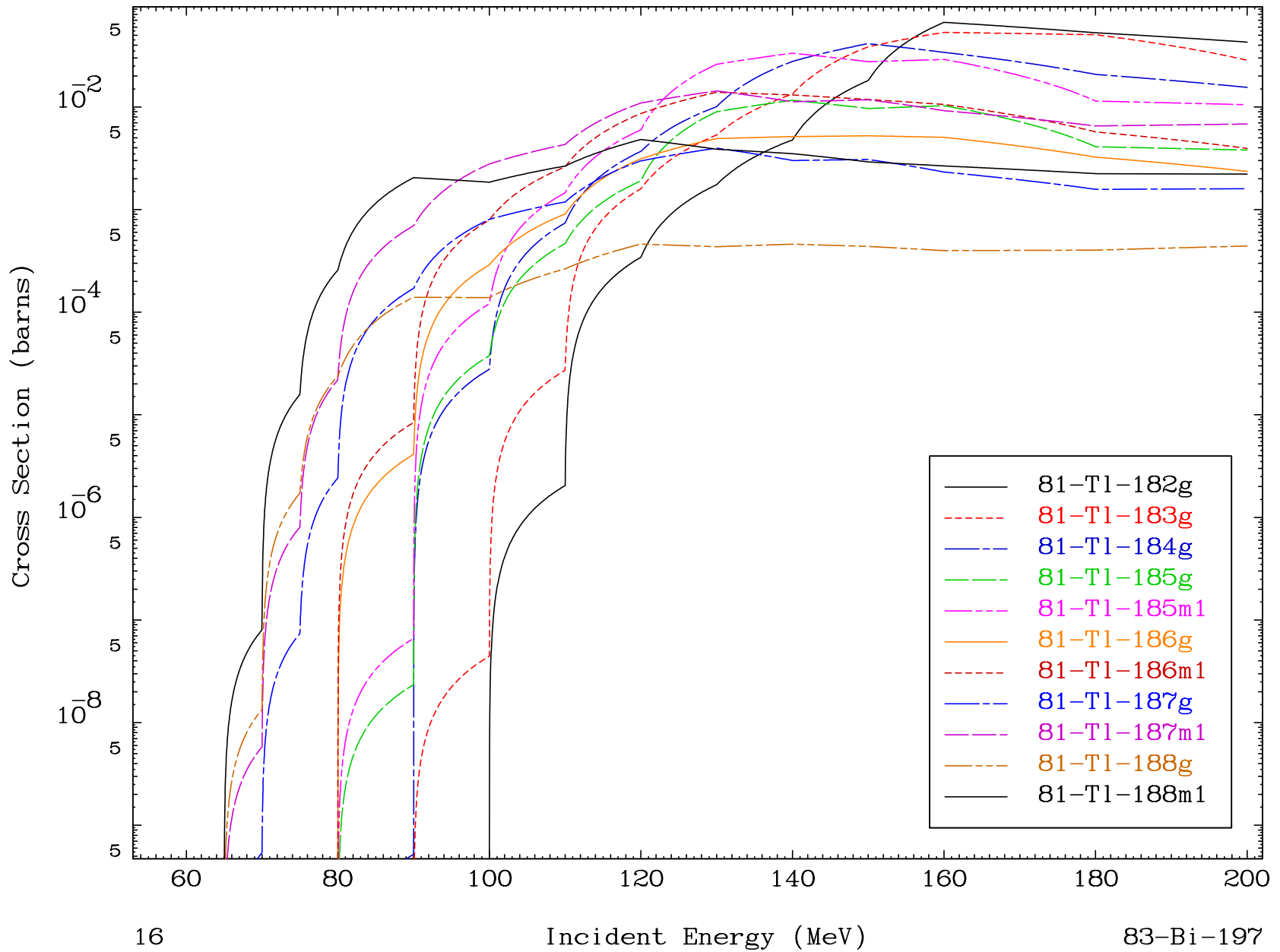
Radionuclide Production Cross Section



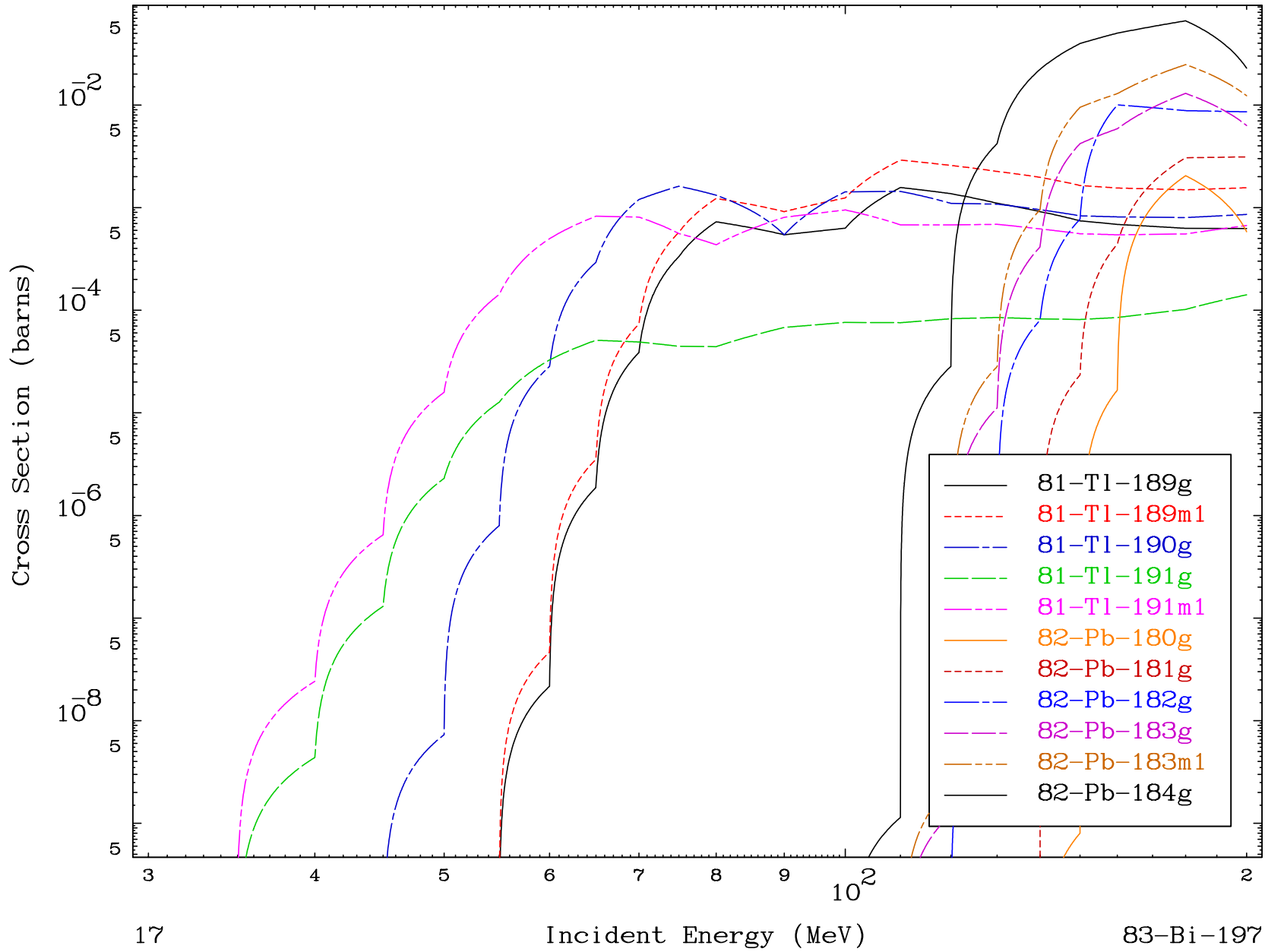


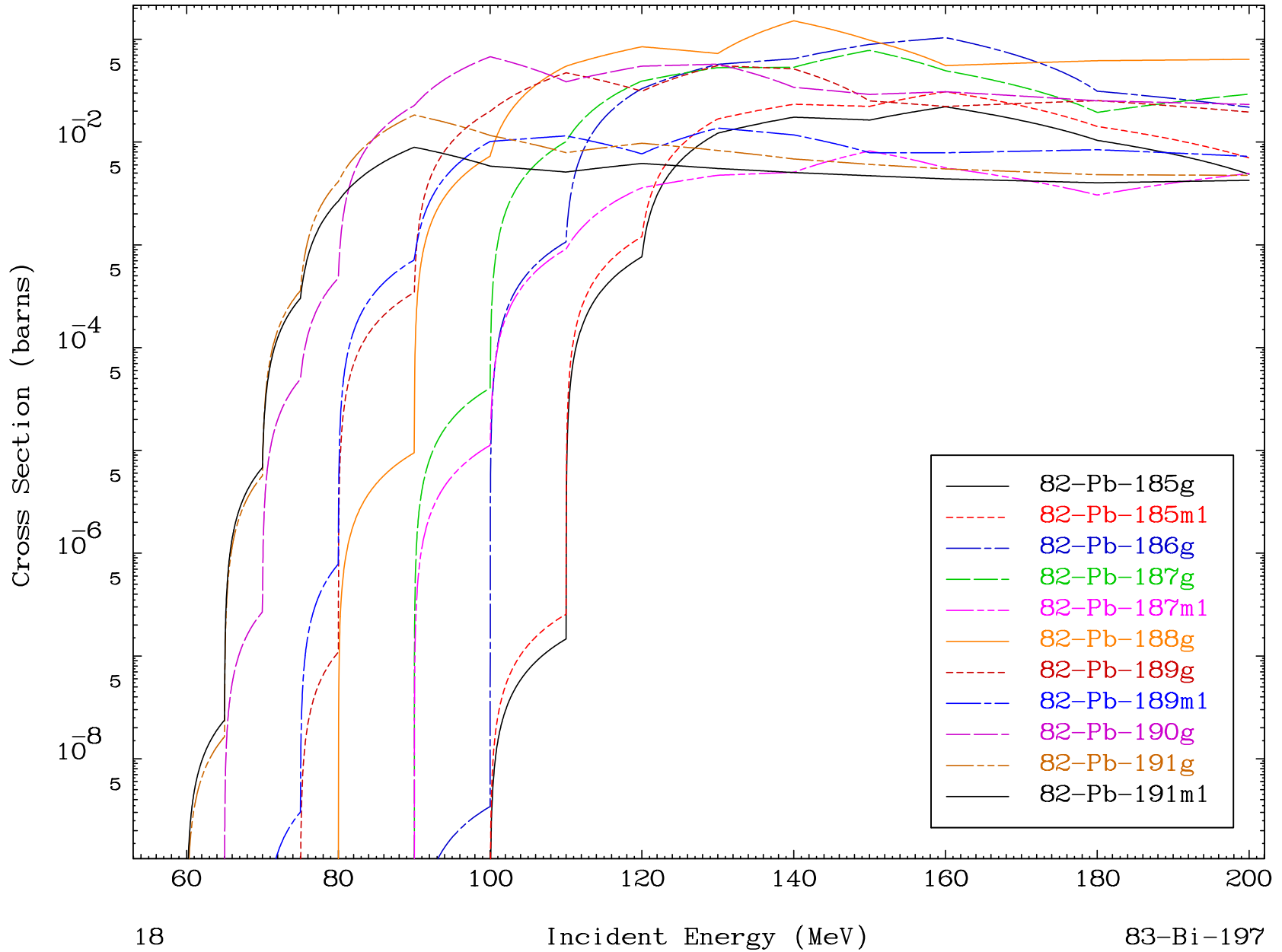




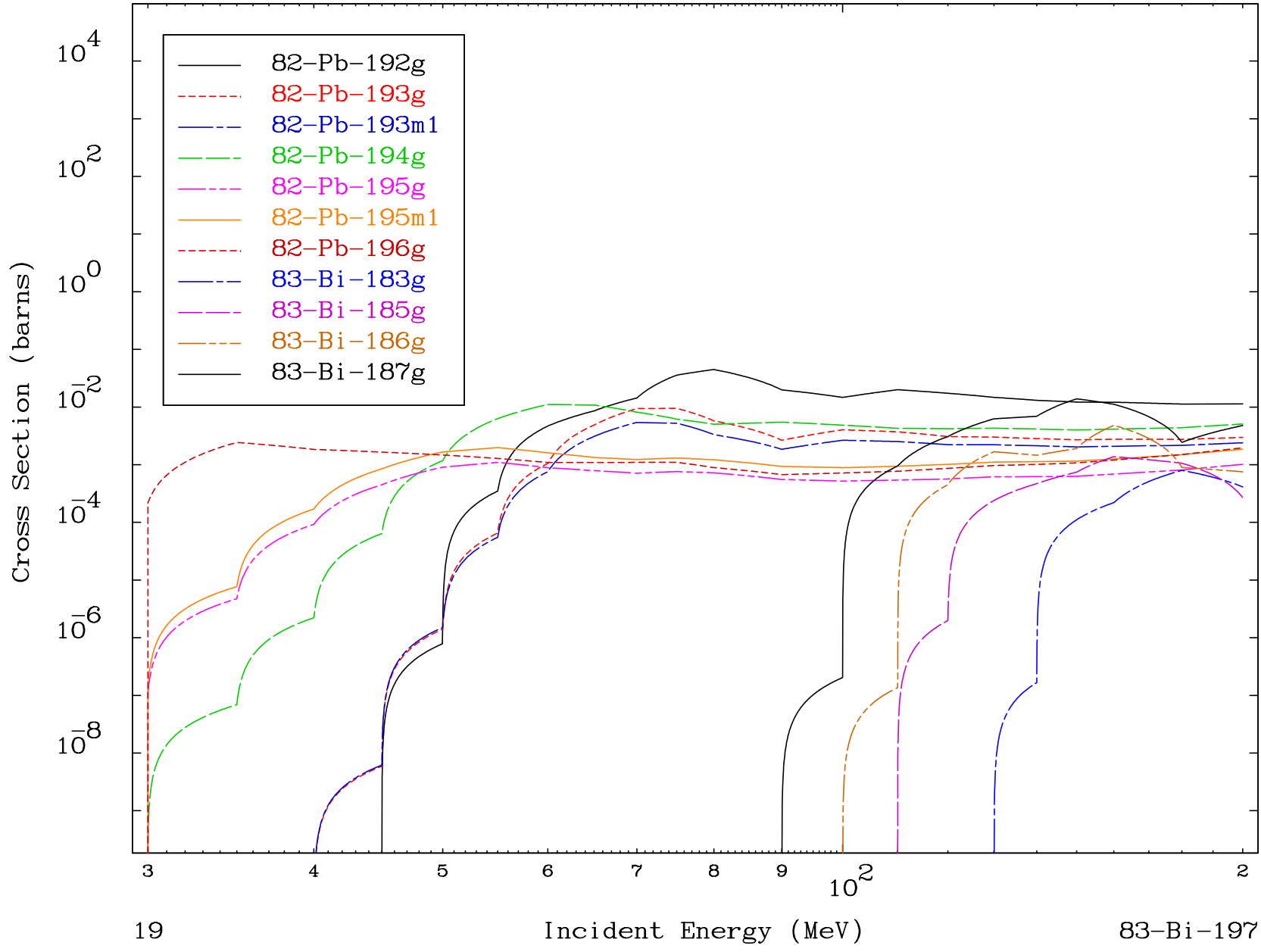


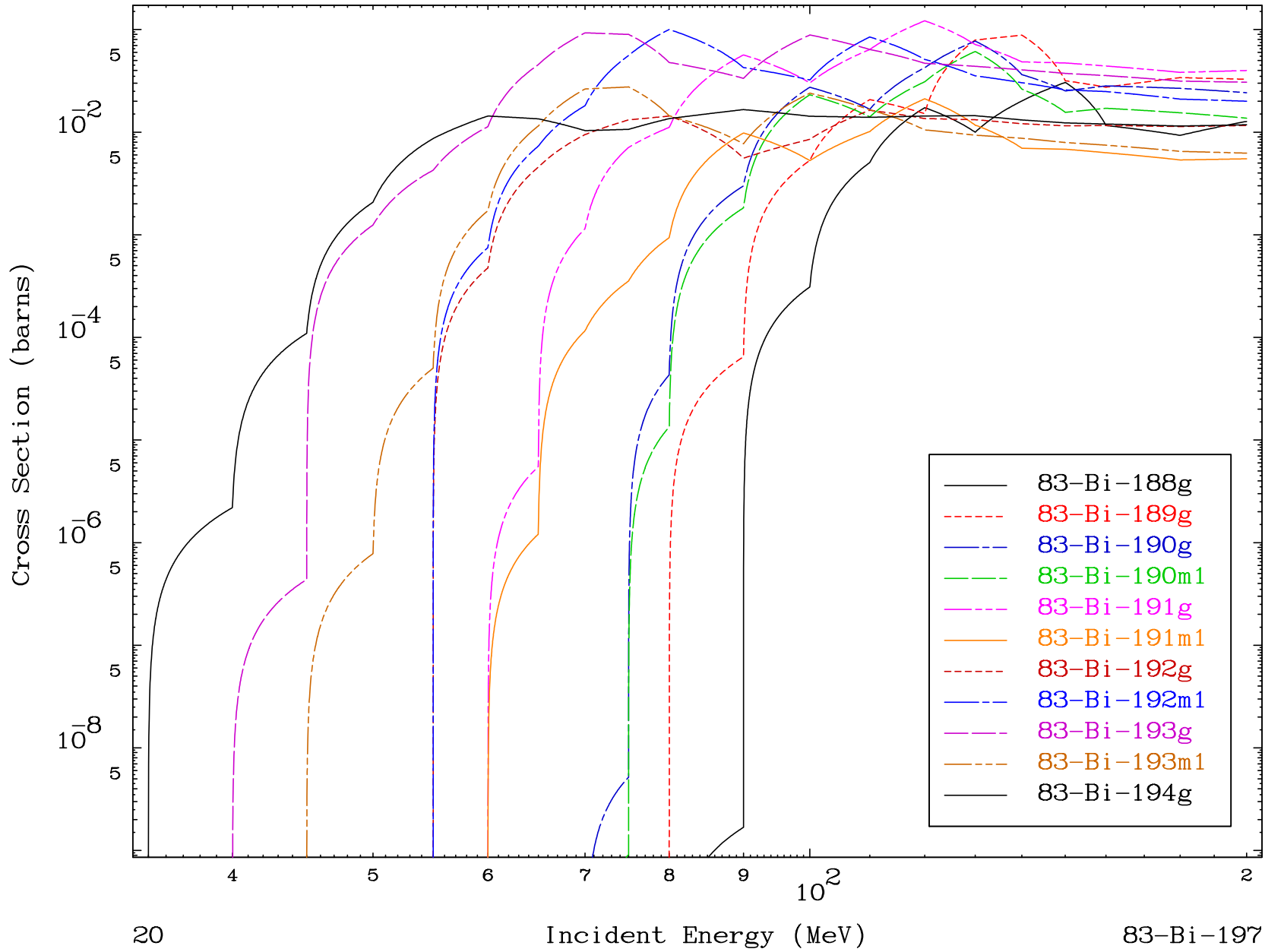


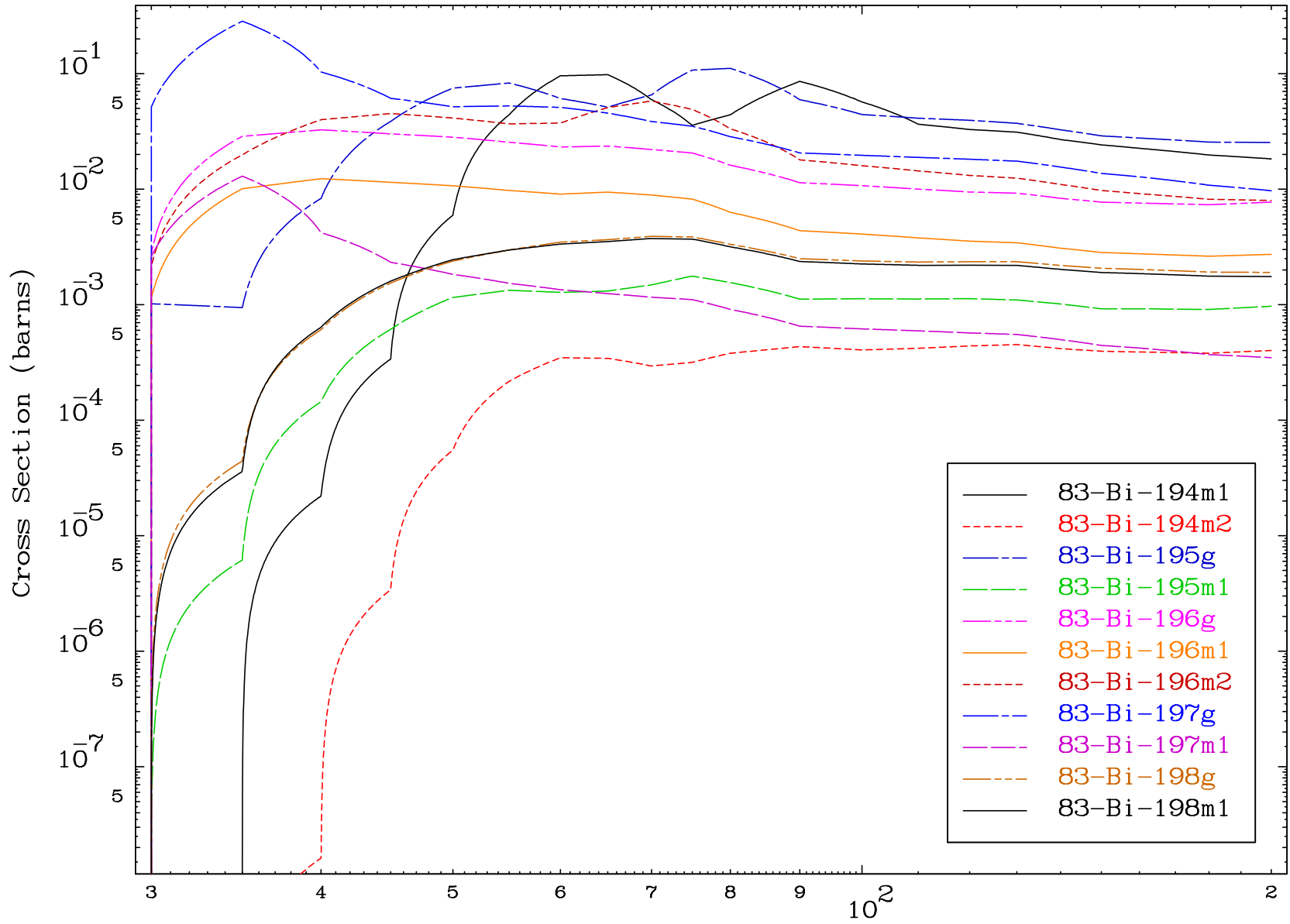


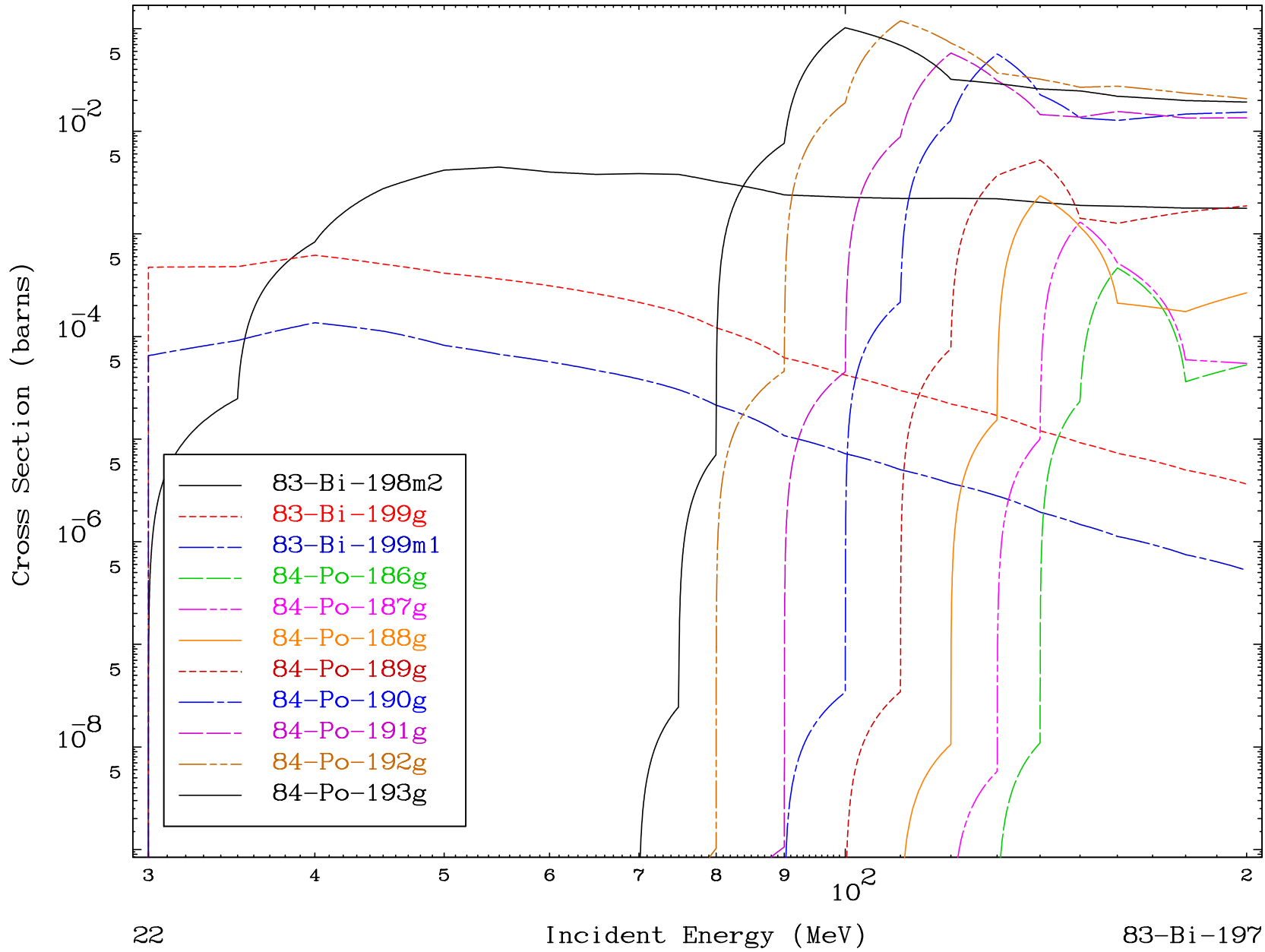


Radionuclide Production Cross Section







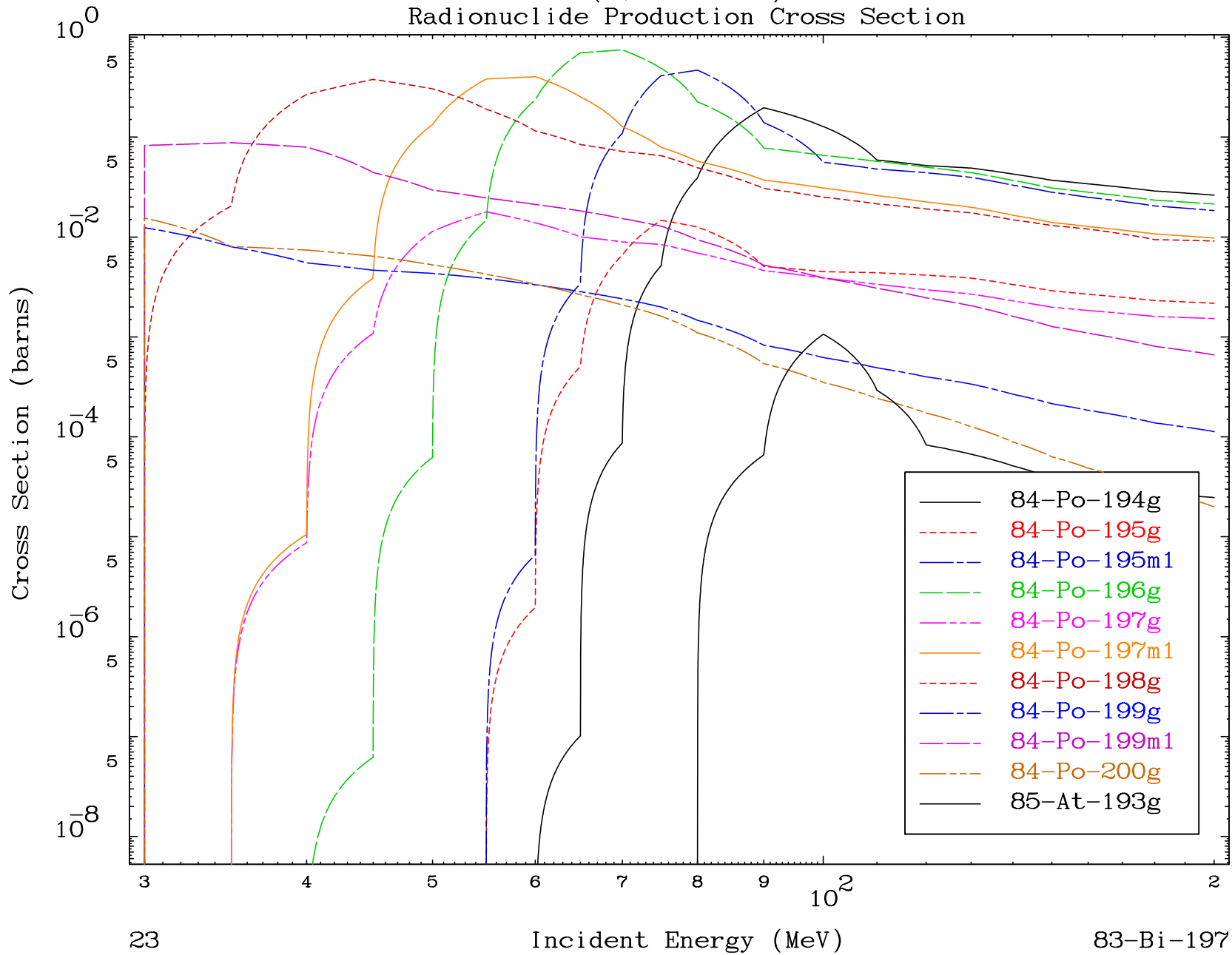


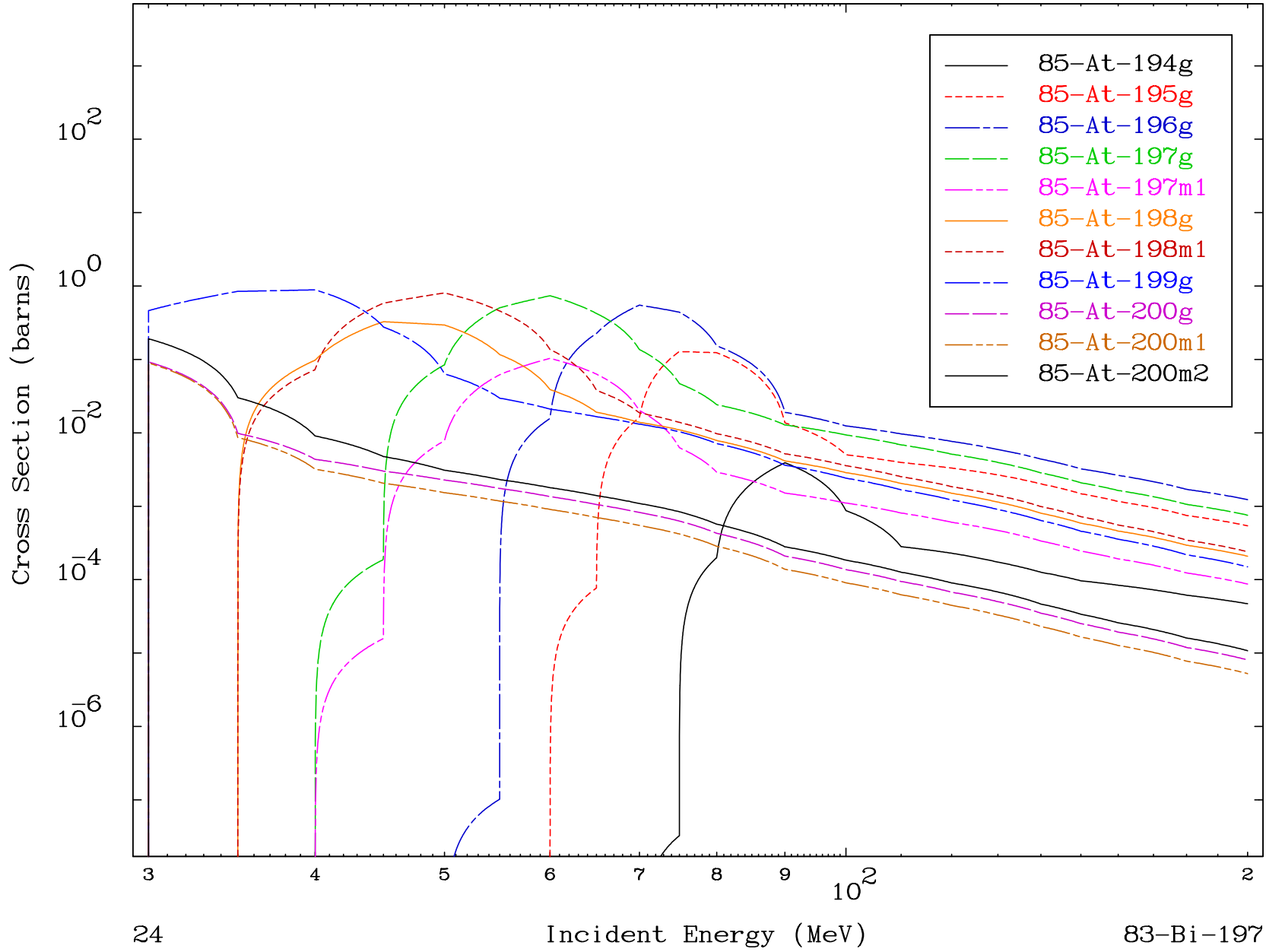
MAT 8289

( $\alpha$ , remainder)

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### Radionuclide Production Cross Section





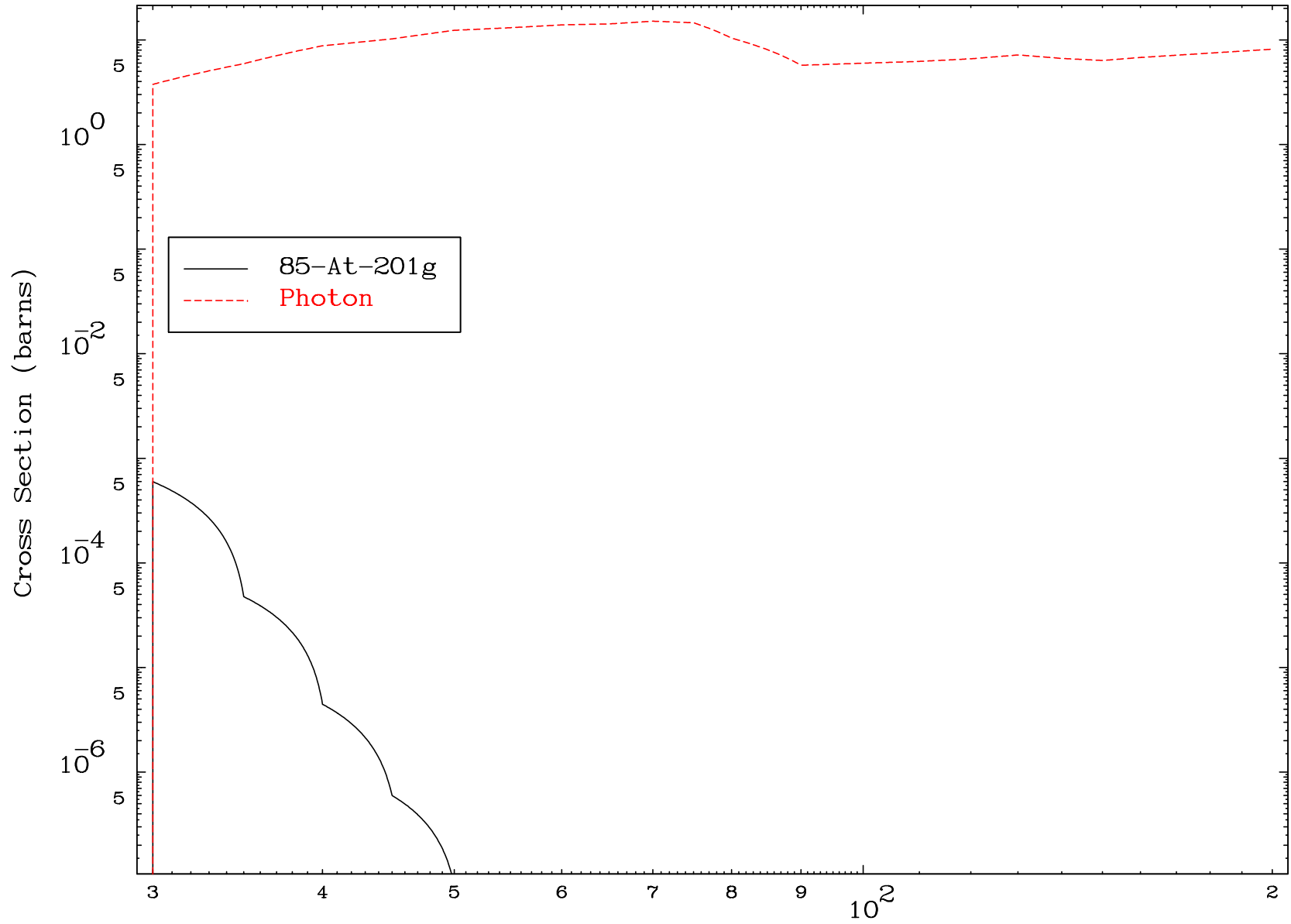


MAT 8289

( $\alpha$ , remainder)

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Radionuclide Production Cross Section



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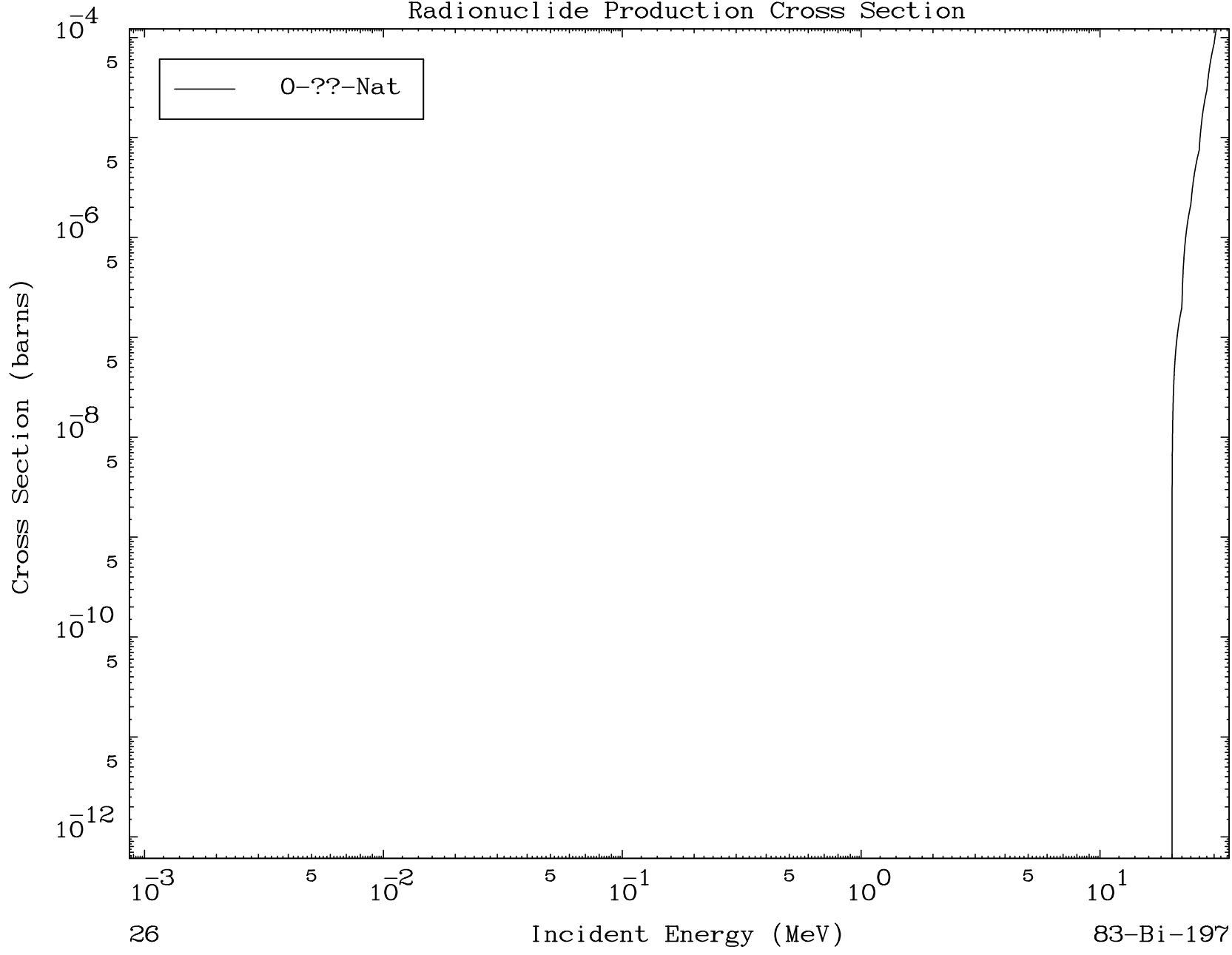
Incident Energy (MeV)

83-Bi-197

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$\alpha$  Fission  
Radionuclide Production Cross Section

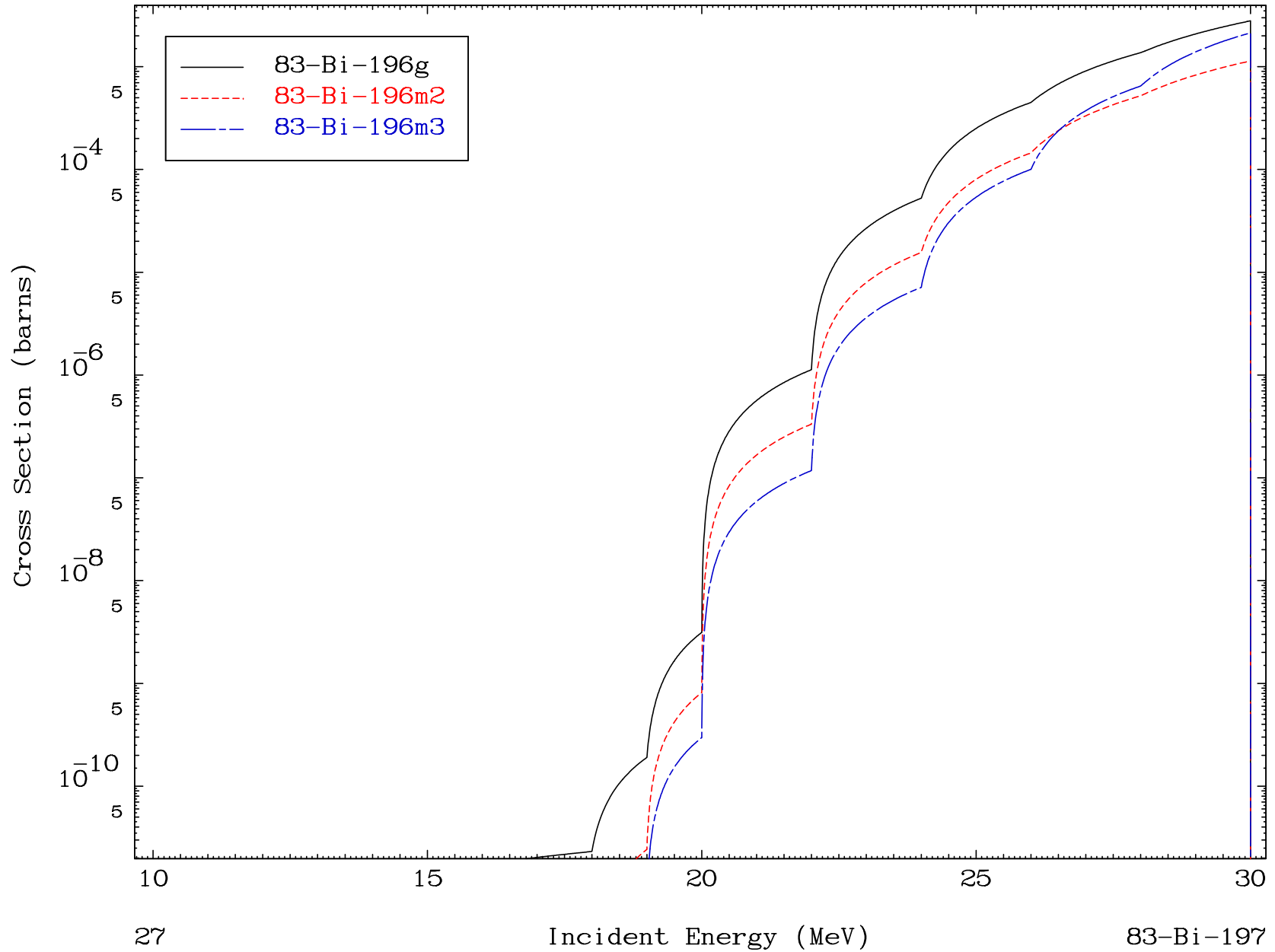
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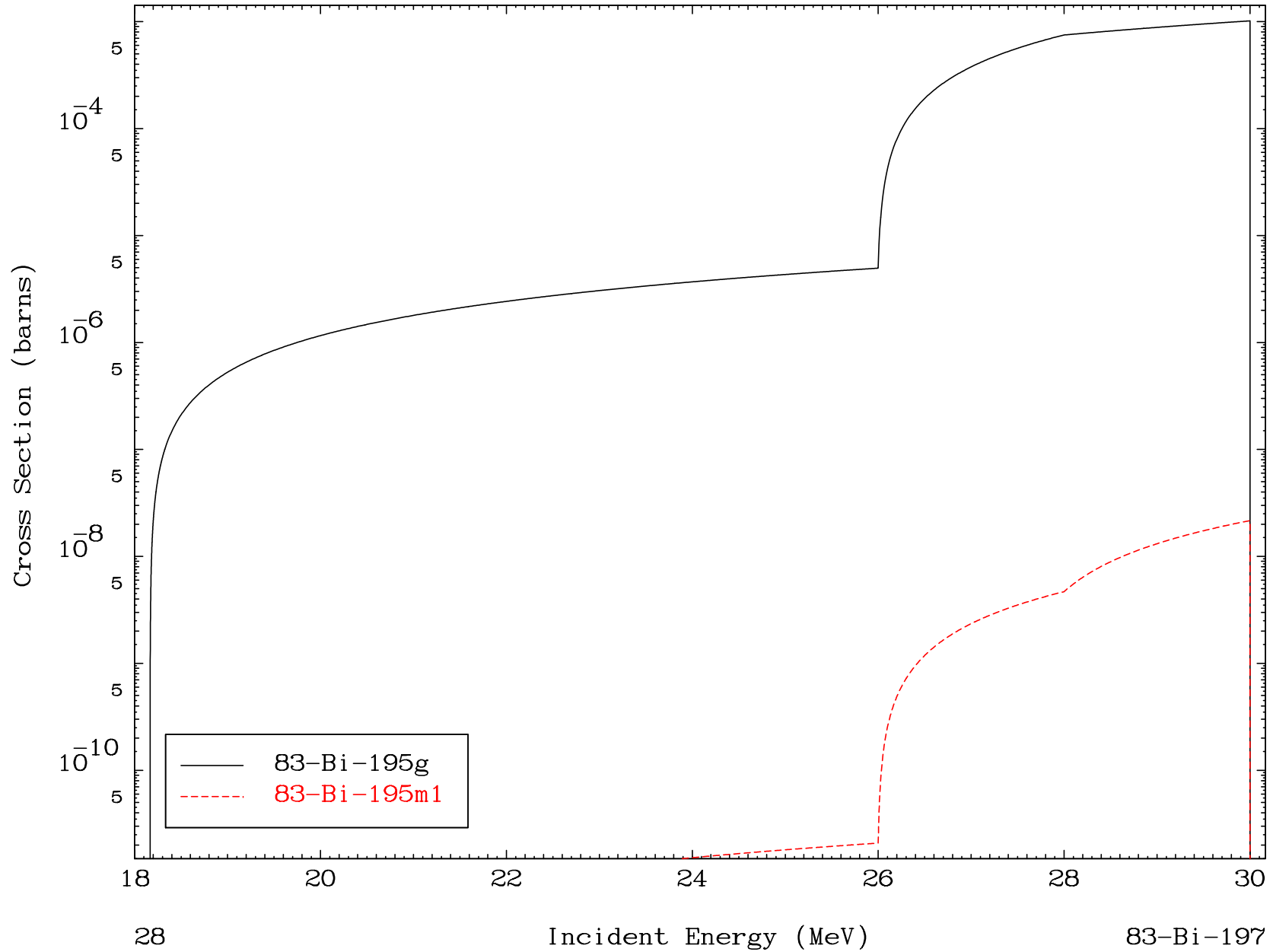
26

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

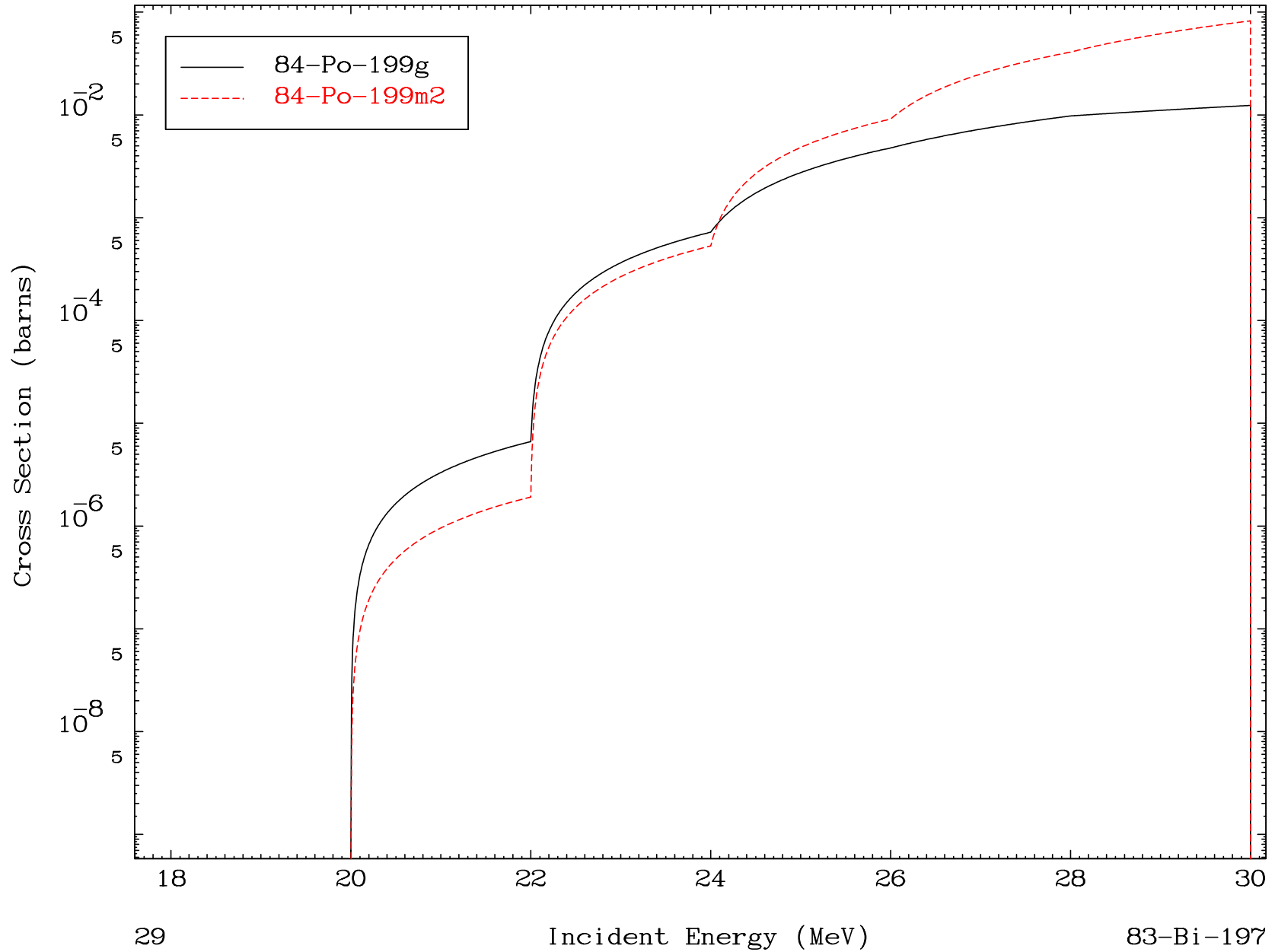


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( $\alpha, n'$ ) p

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Radionuclide Production Cross Section

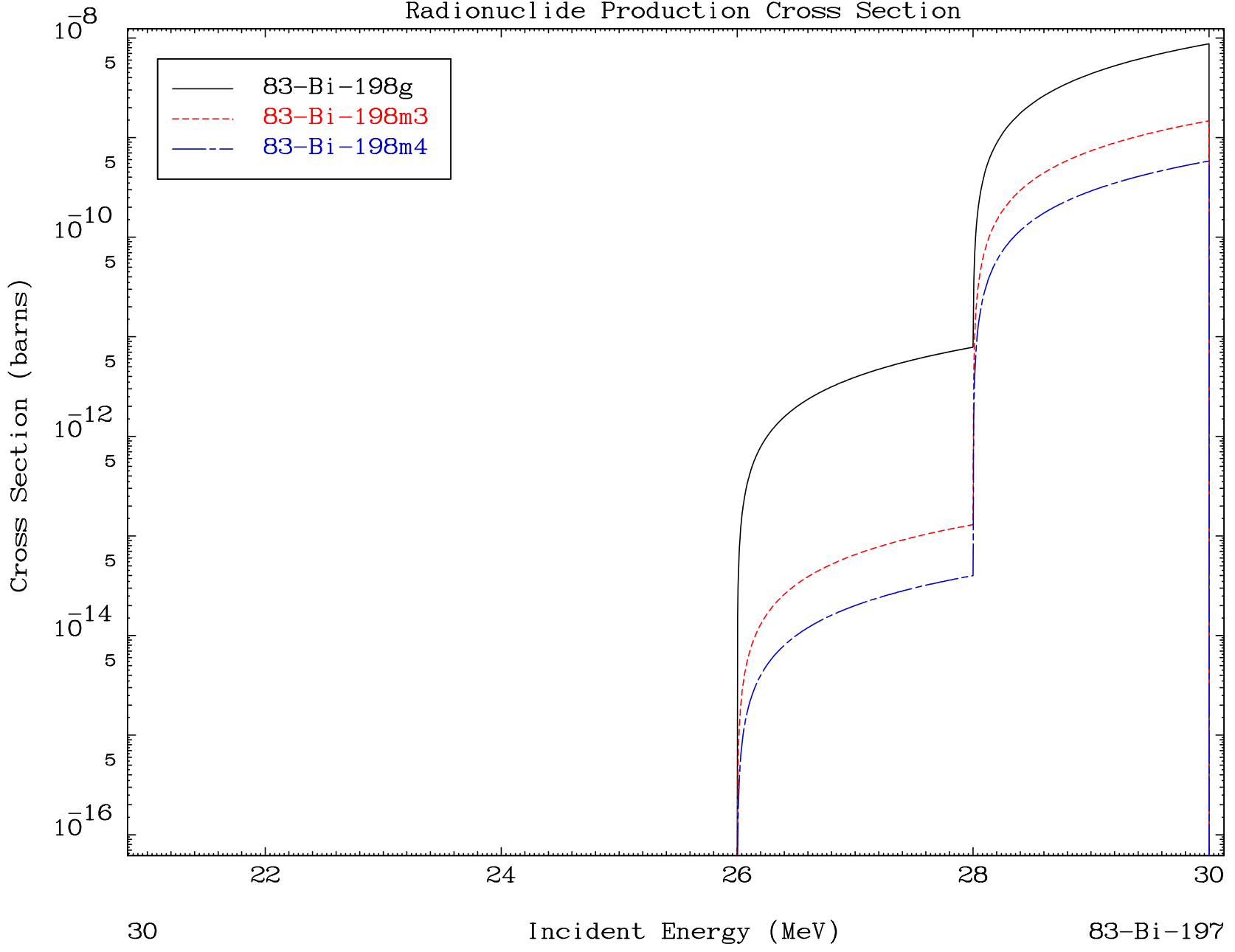


MAT 8289

( $\alpha, 2n$ ) p

83-Bi-197

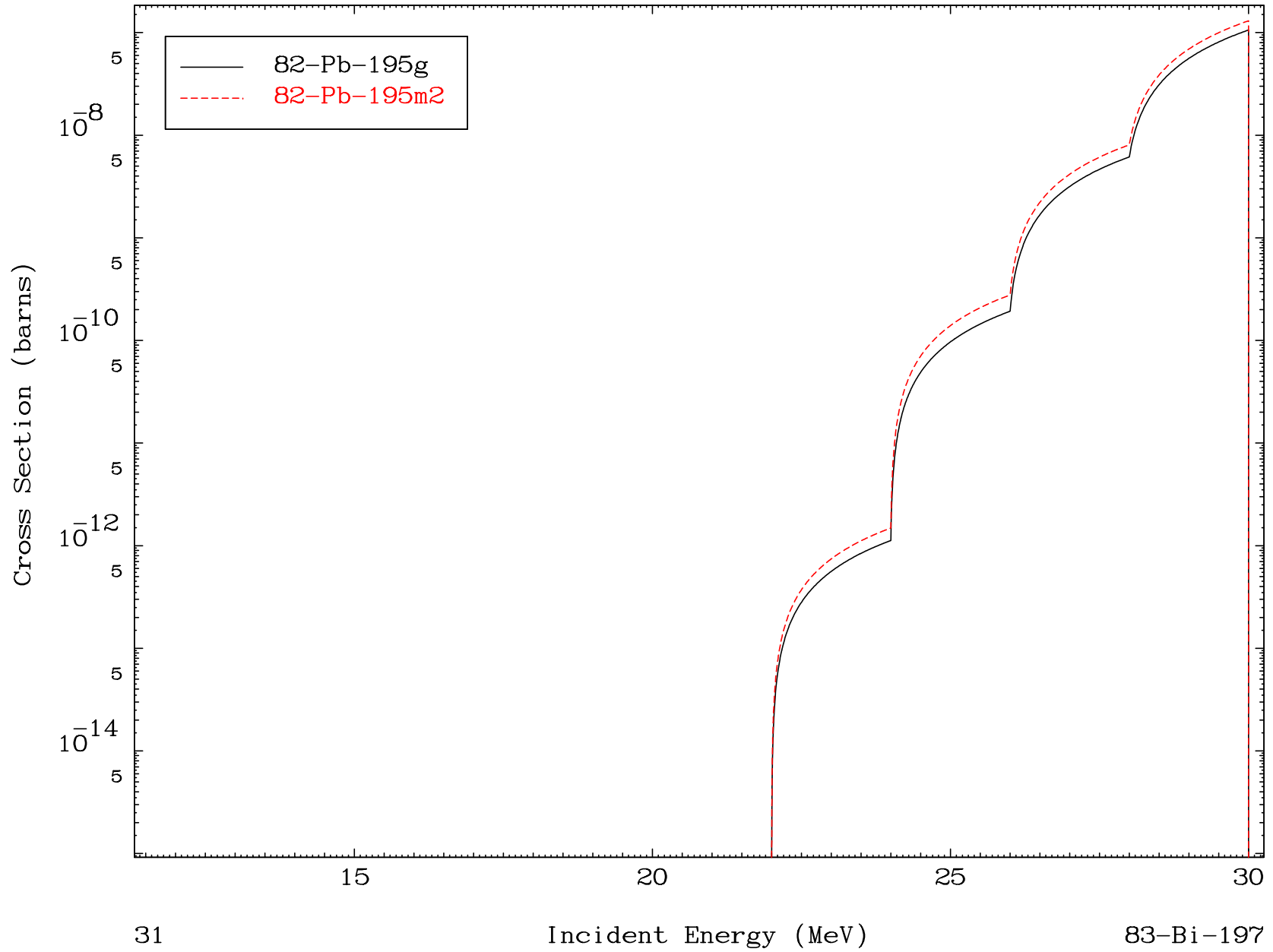
Radionuclide Production Cross Section



MAT 8289

( $\alpha, n'$ ) p  $\alpha$   
Radionuclide Production Cross Section

83-Bi-197



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Incident Energy (MeV)

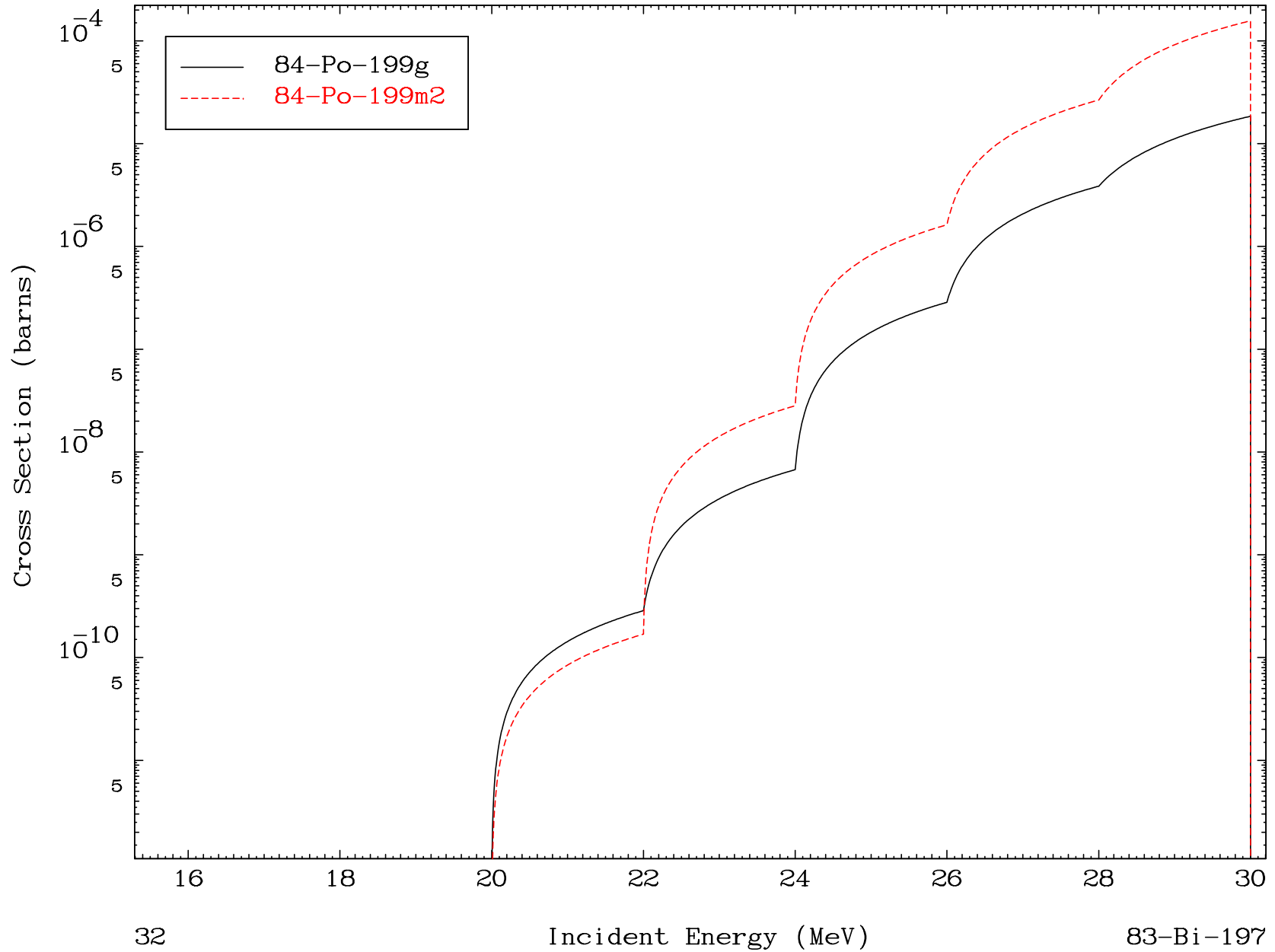
83-Bi-197

MAT 8289

( $\alpha, d$ )

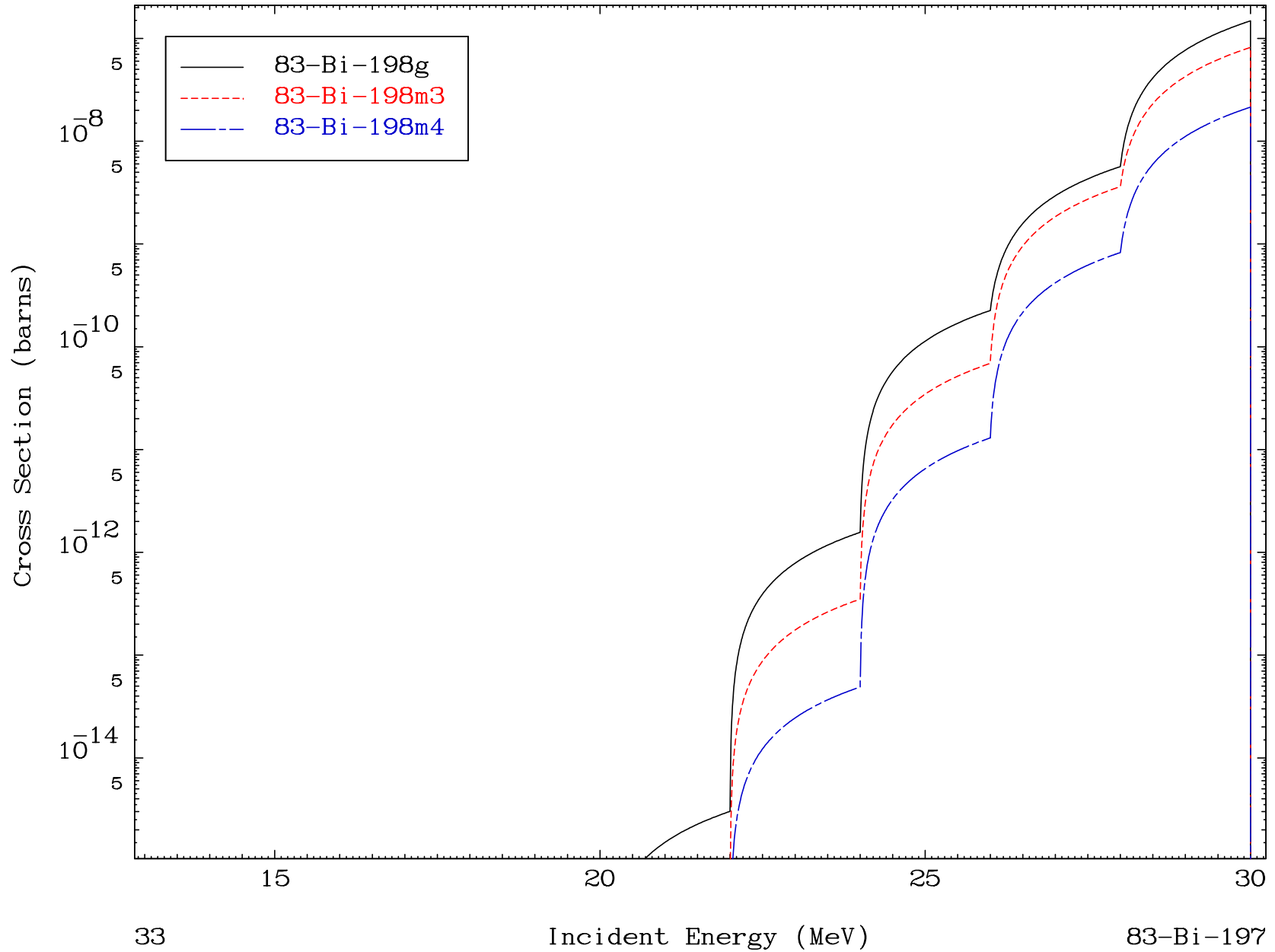
83-Bi-197

### Radionuclide Production Cross Section

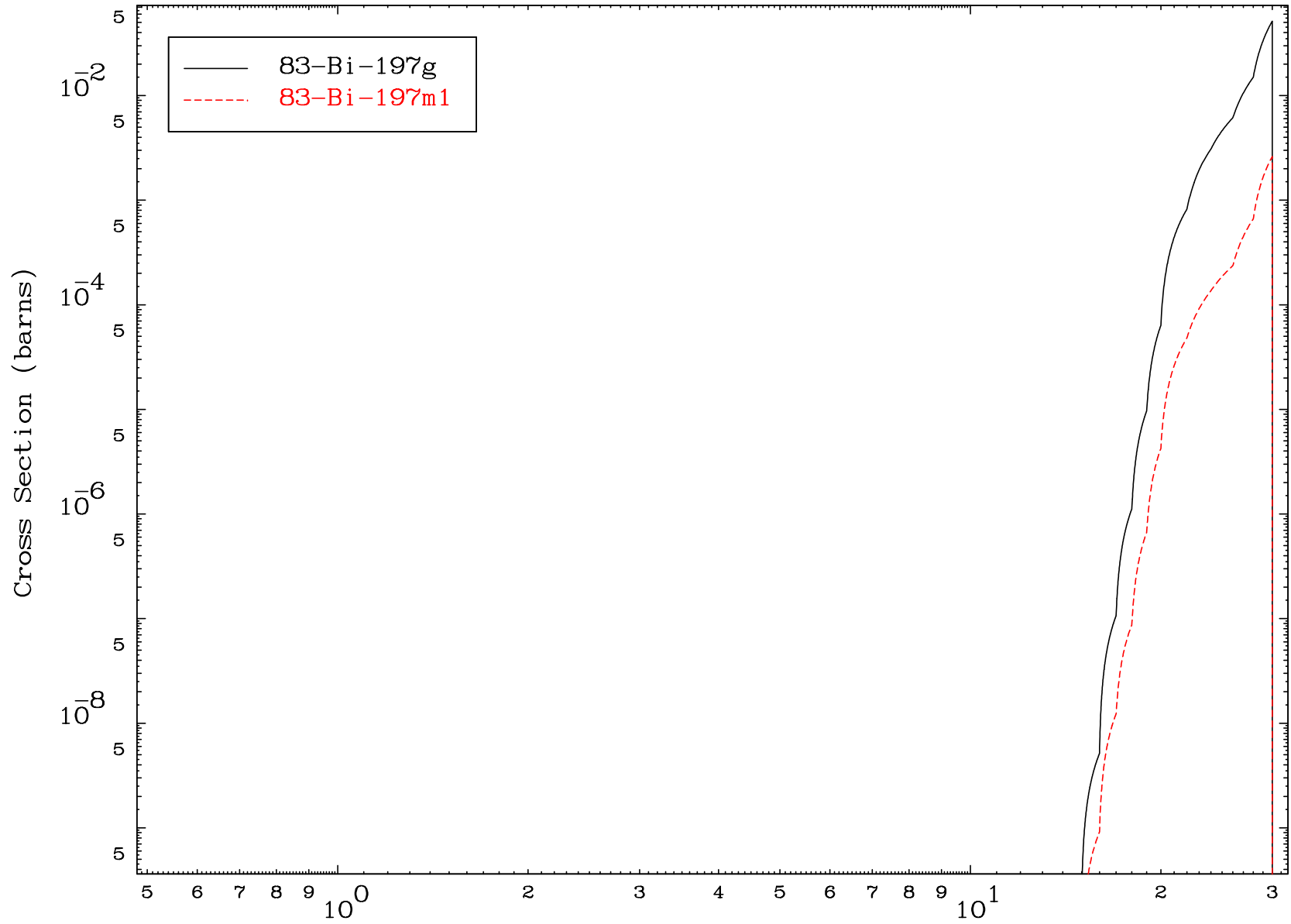




Radionuclide Production Cross Section



Radionuclide Production Cross Section

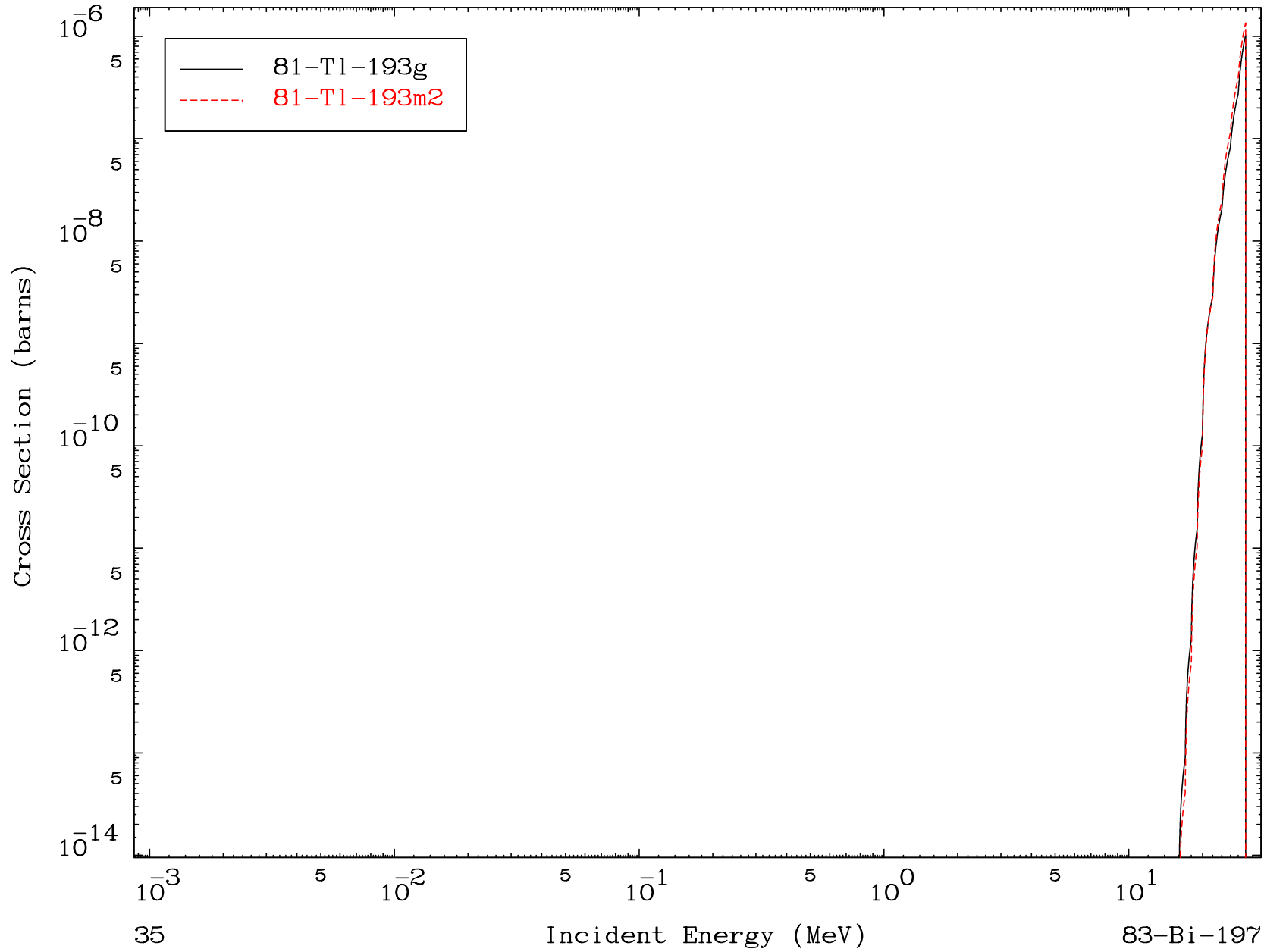


MAT 8289

( $\alpha, 2\alpha$ )

83-Bi-197

Radionuclide Production Cross Section



Radionuclide Production Cross Section

