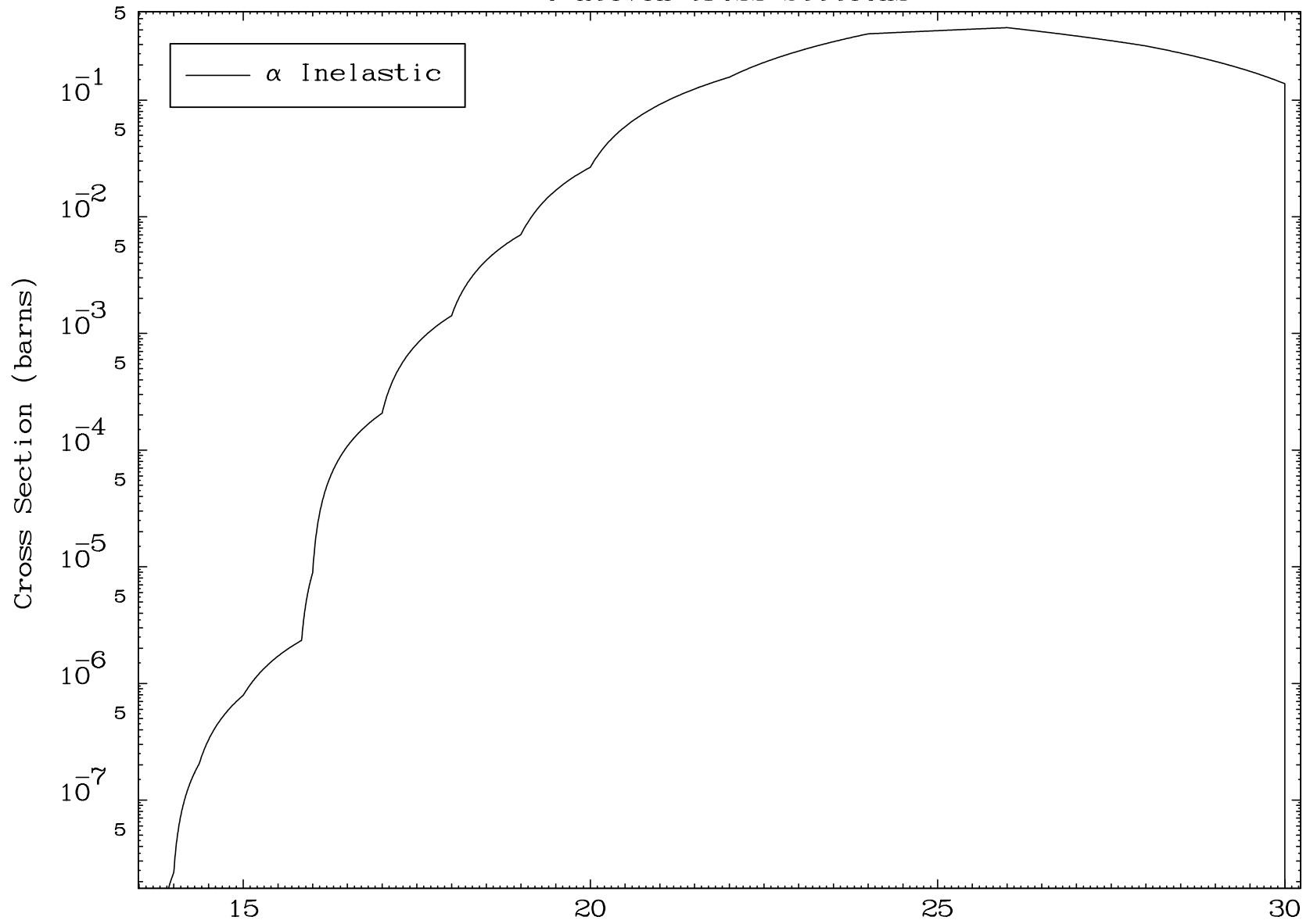


MAT 8293

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

83-Bi-198



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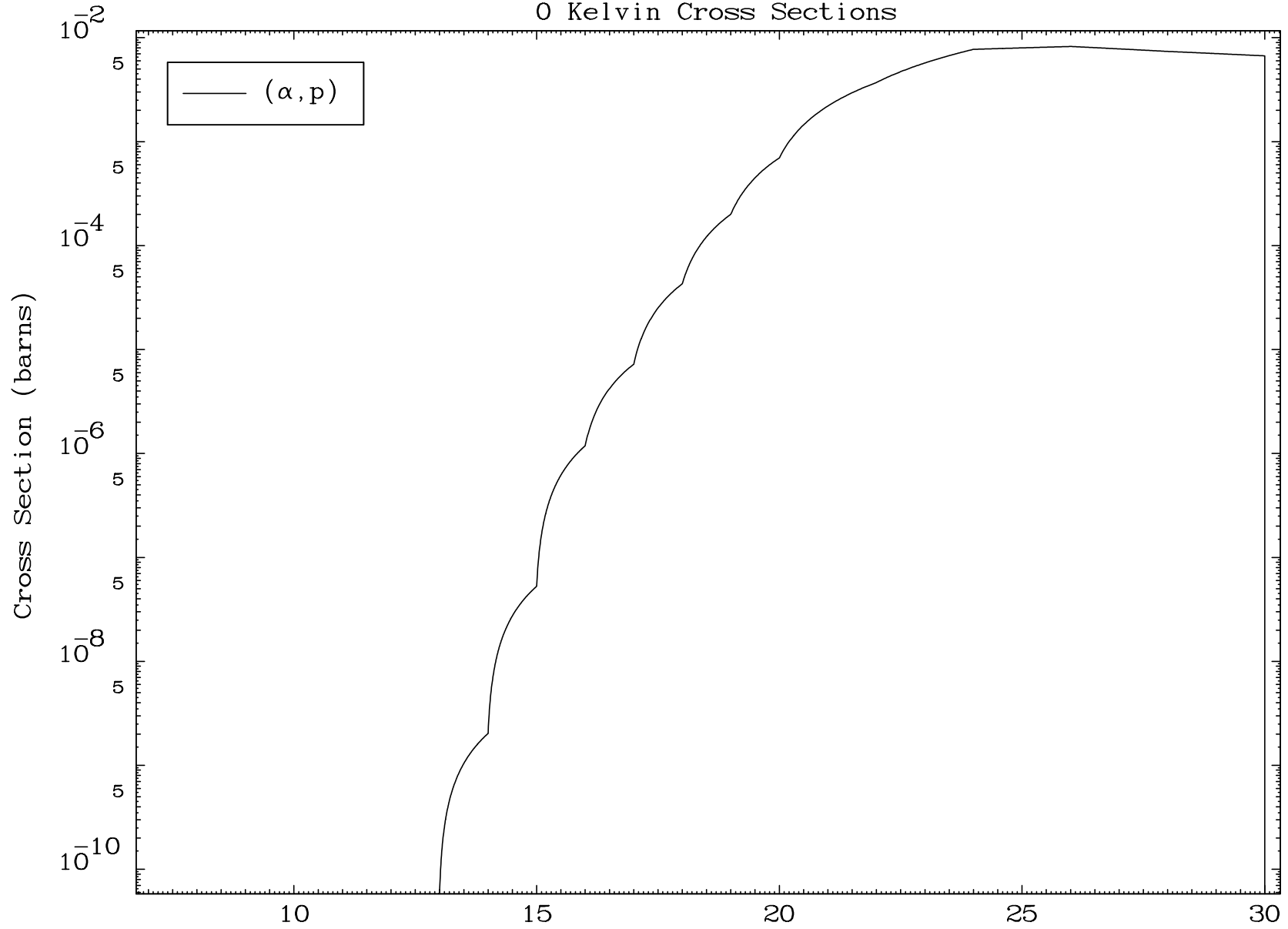
Incident Energy (MeV)

83-Bi-198

MAT 8293

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

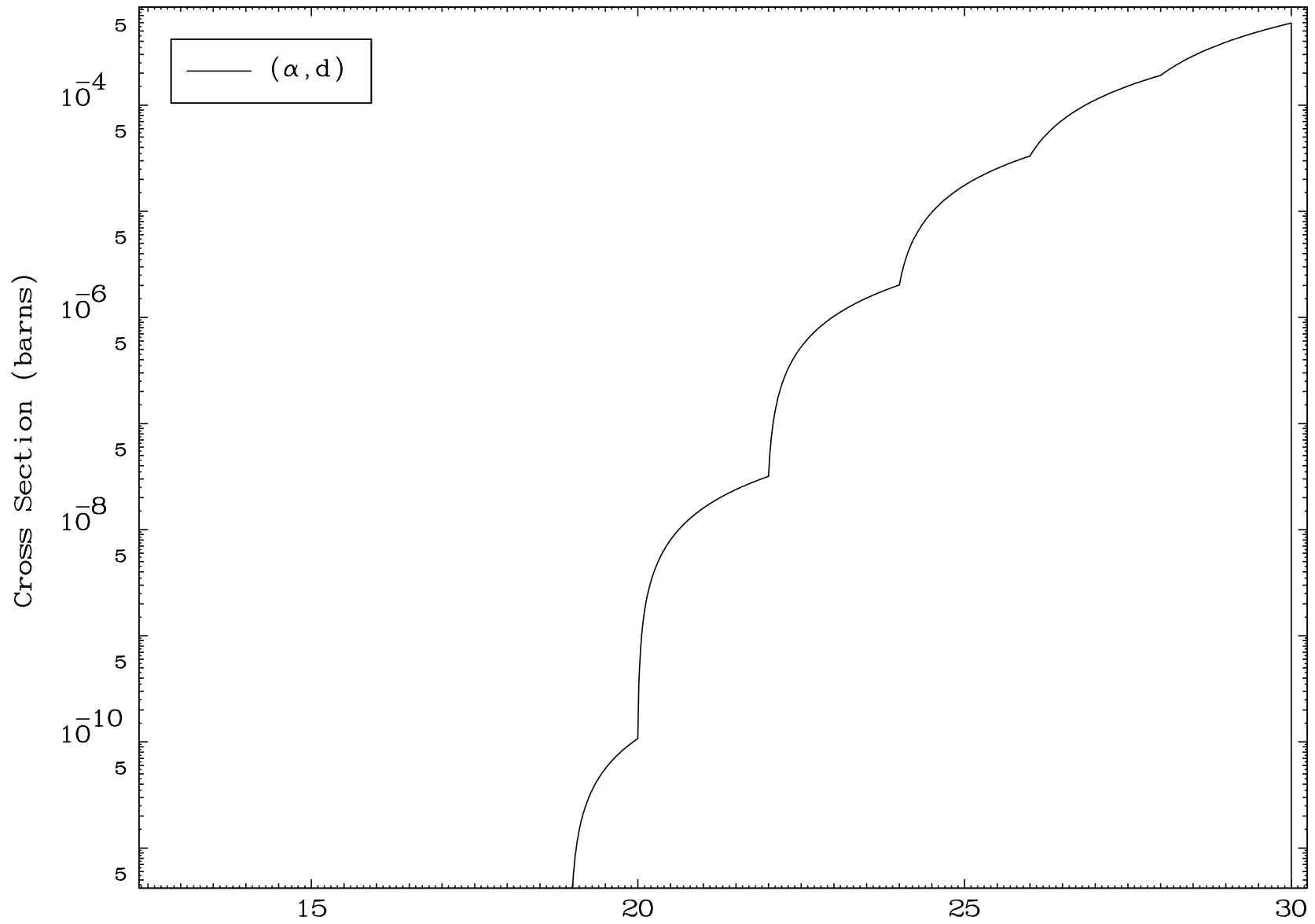
83-Bi-198

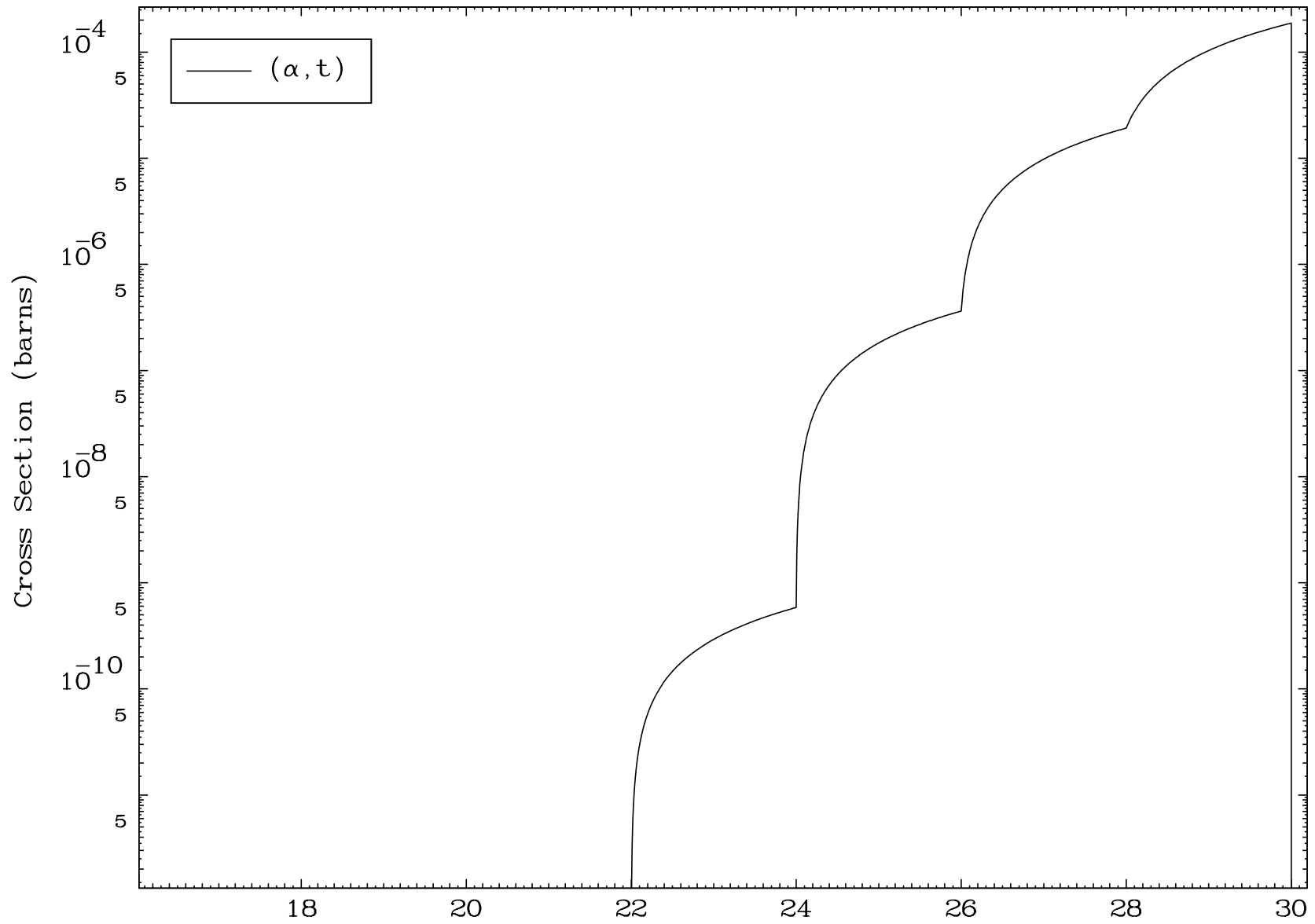


6

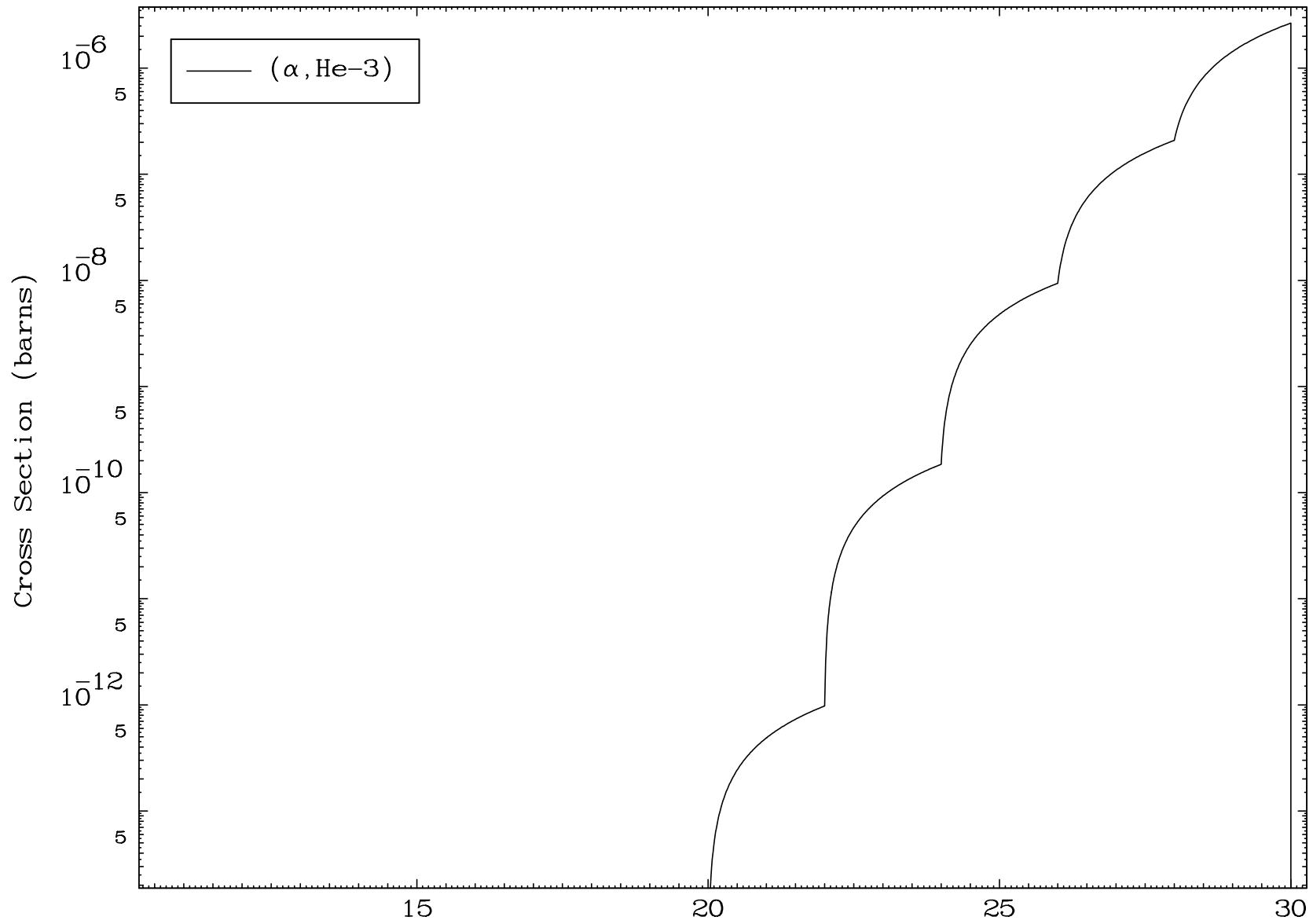
Incident Energy (MeV)

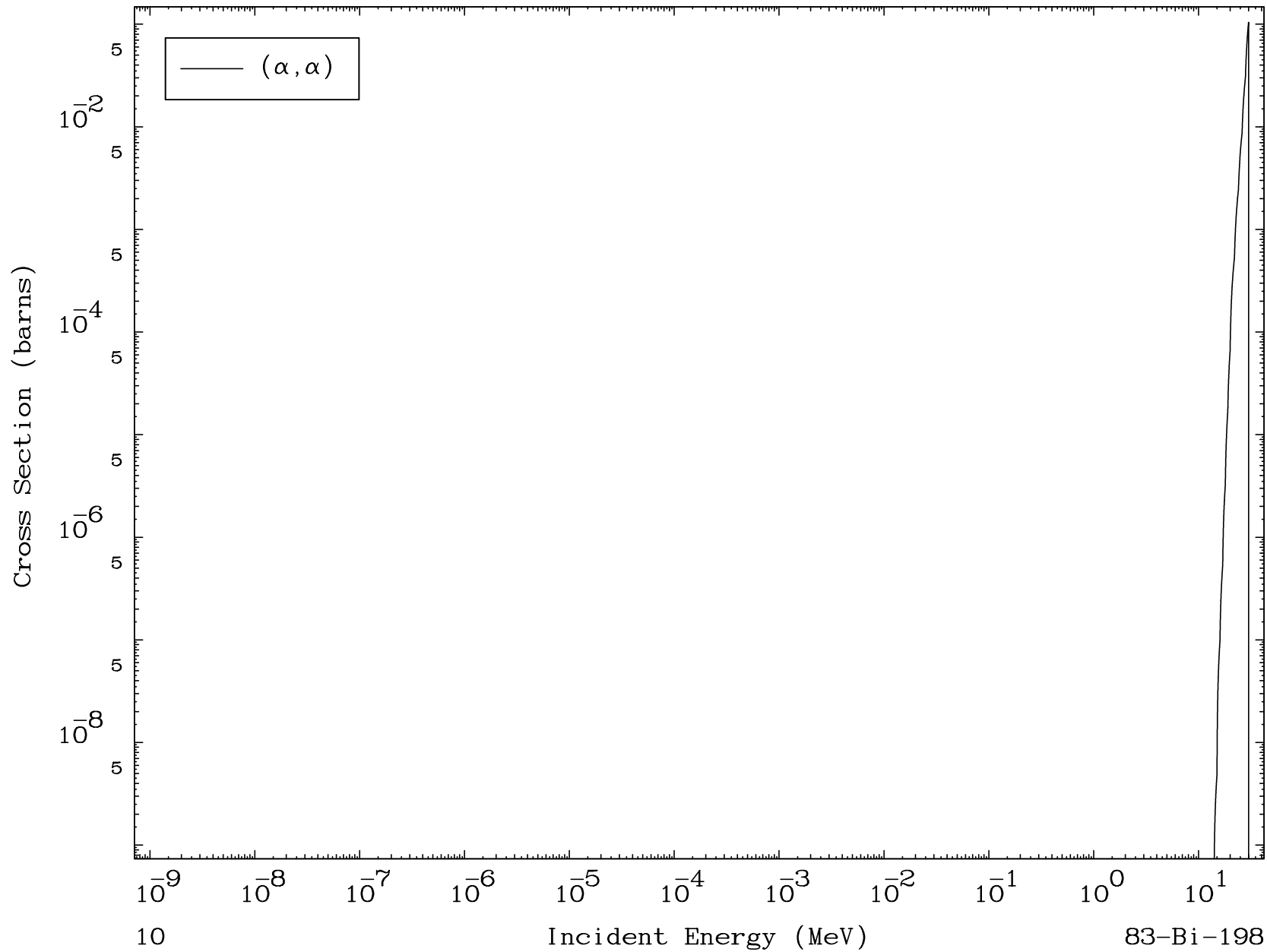
83-Bi-198



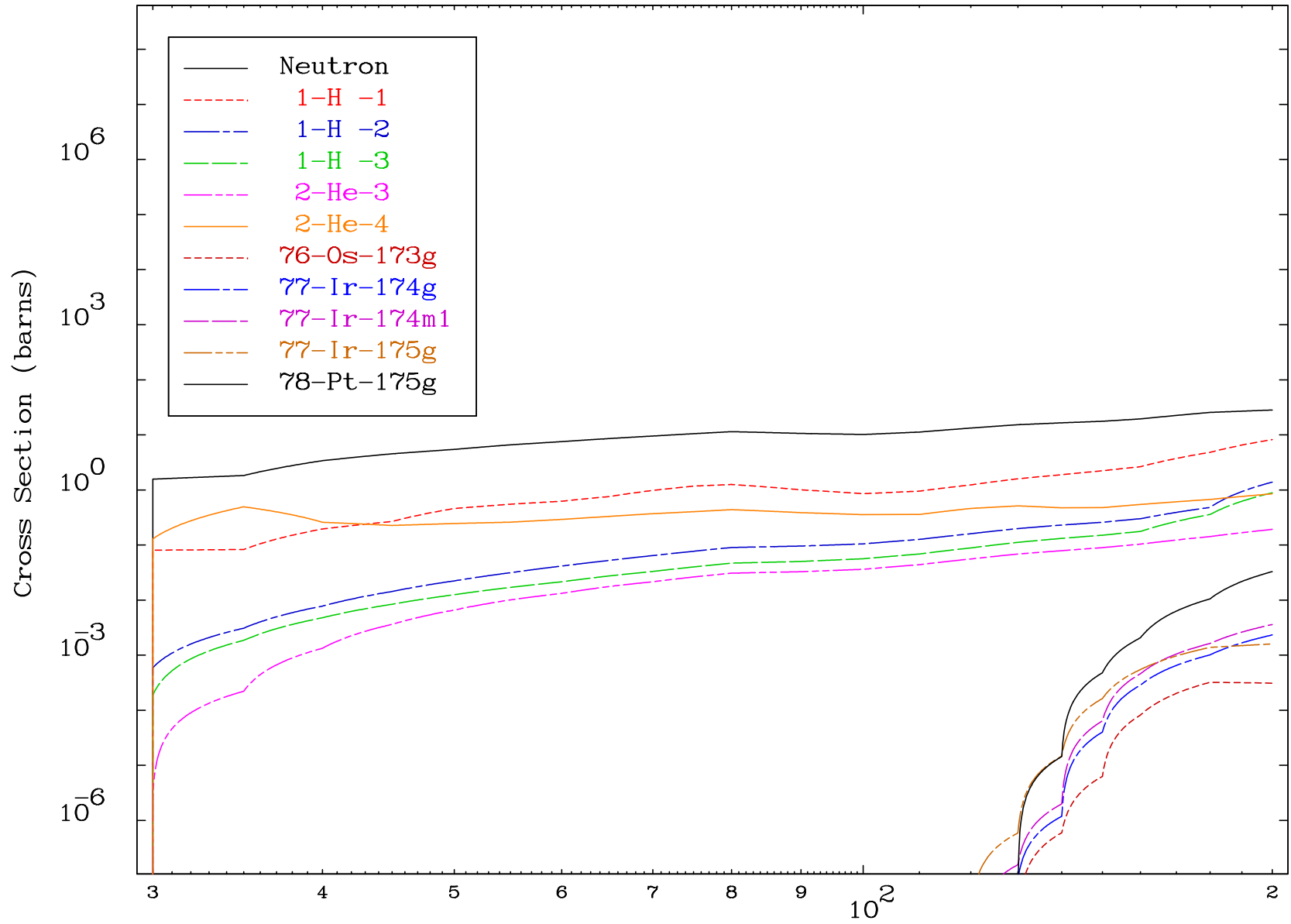




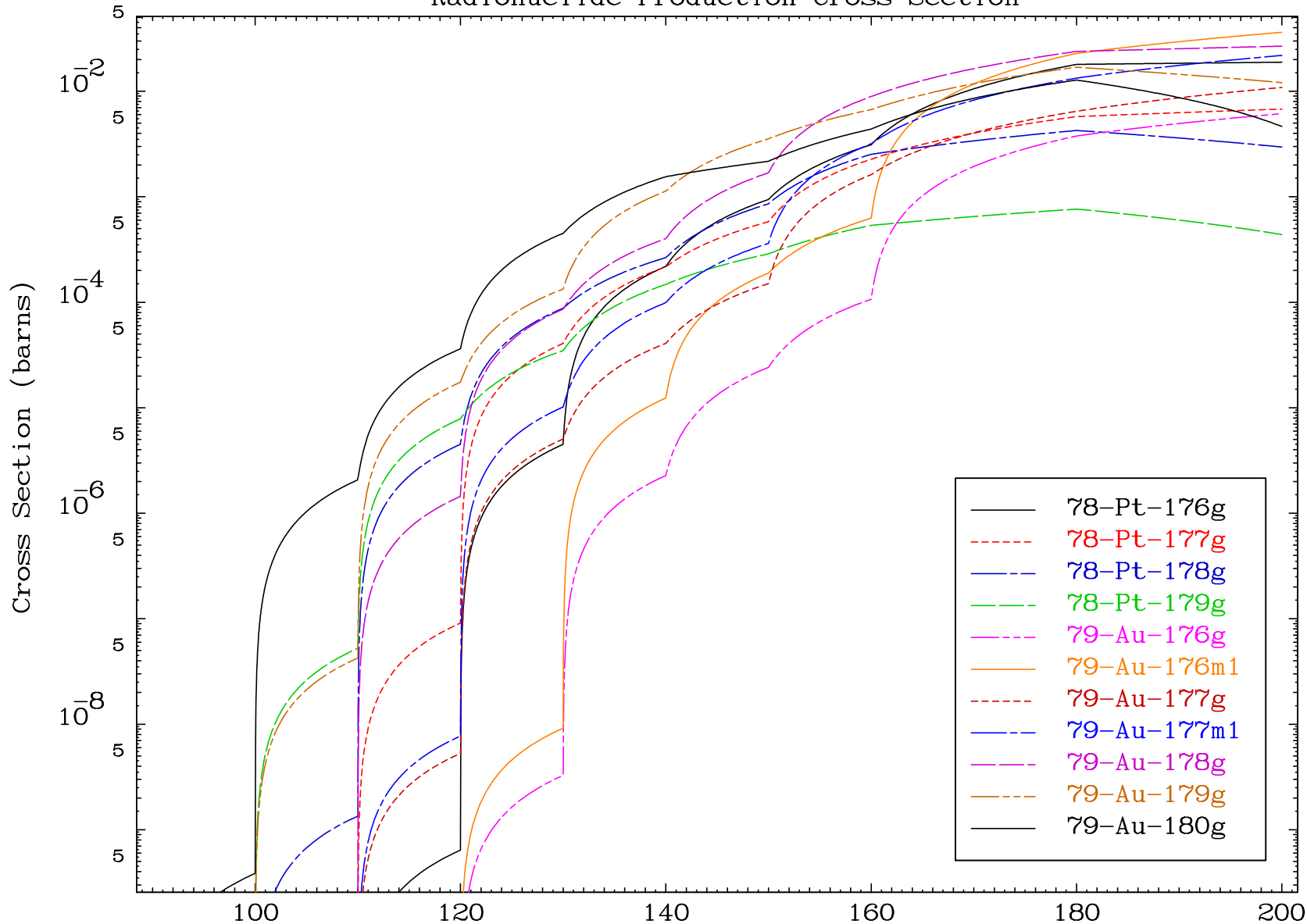




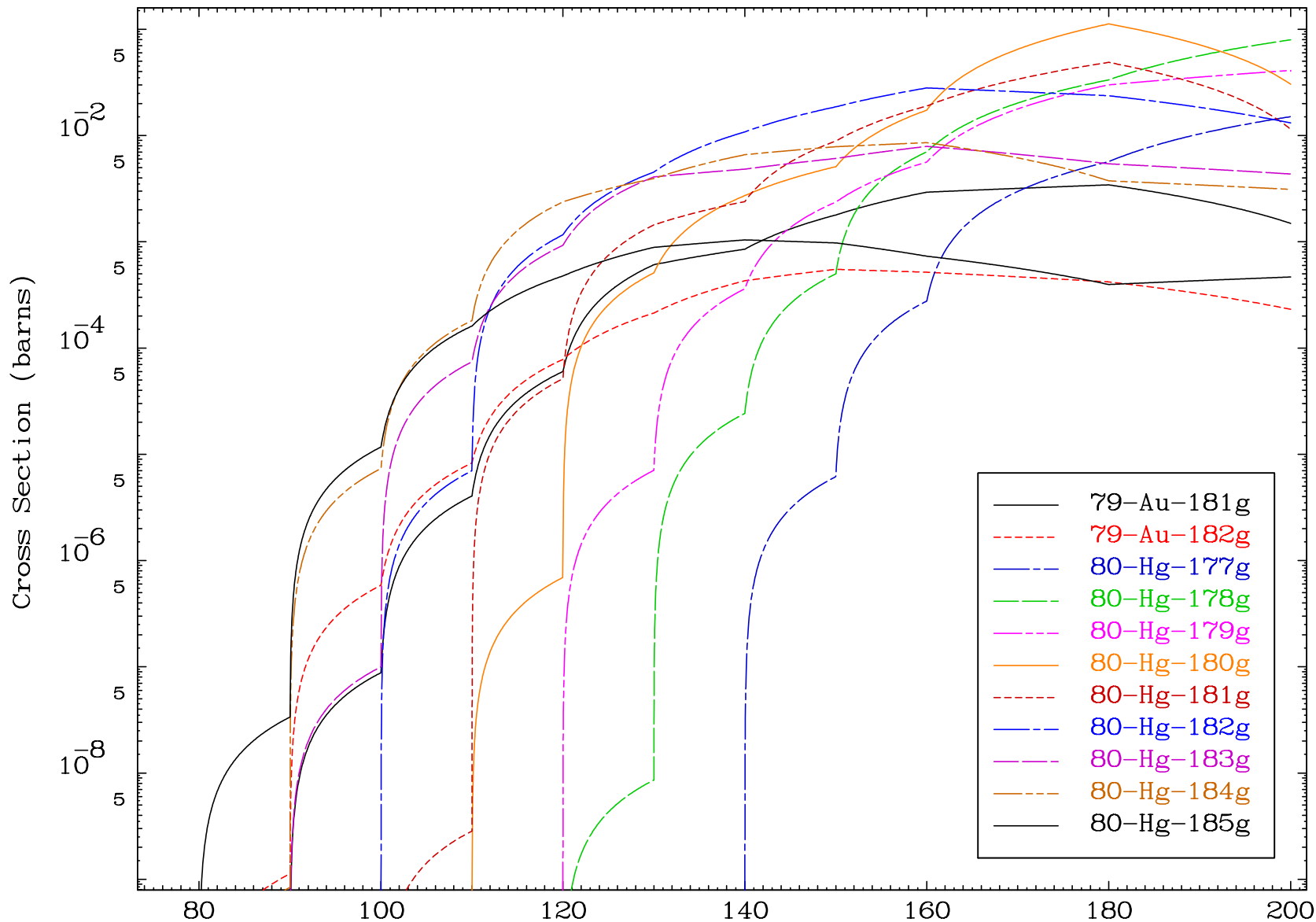
Radionuclide Production Cross Section



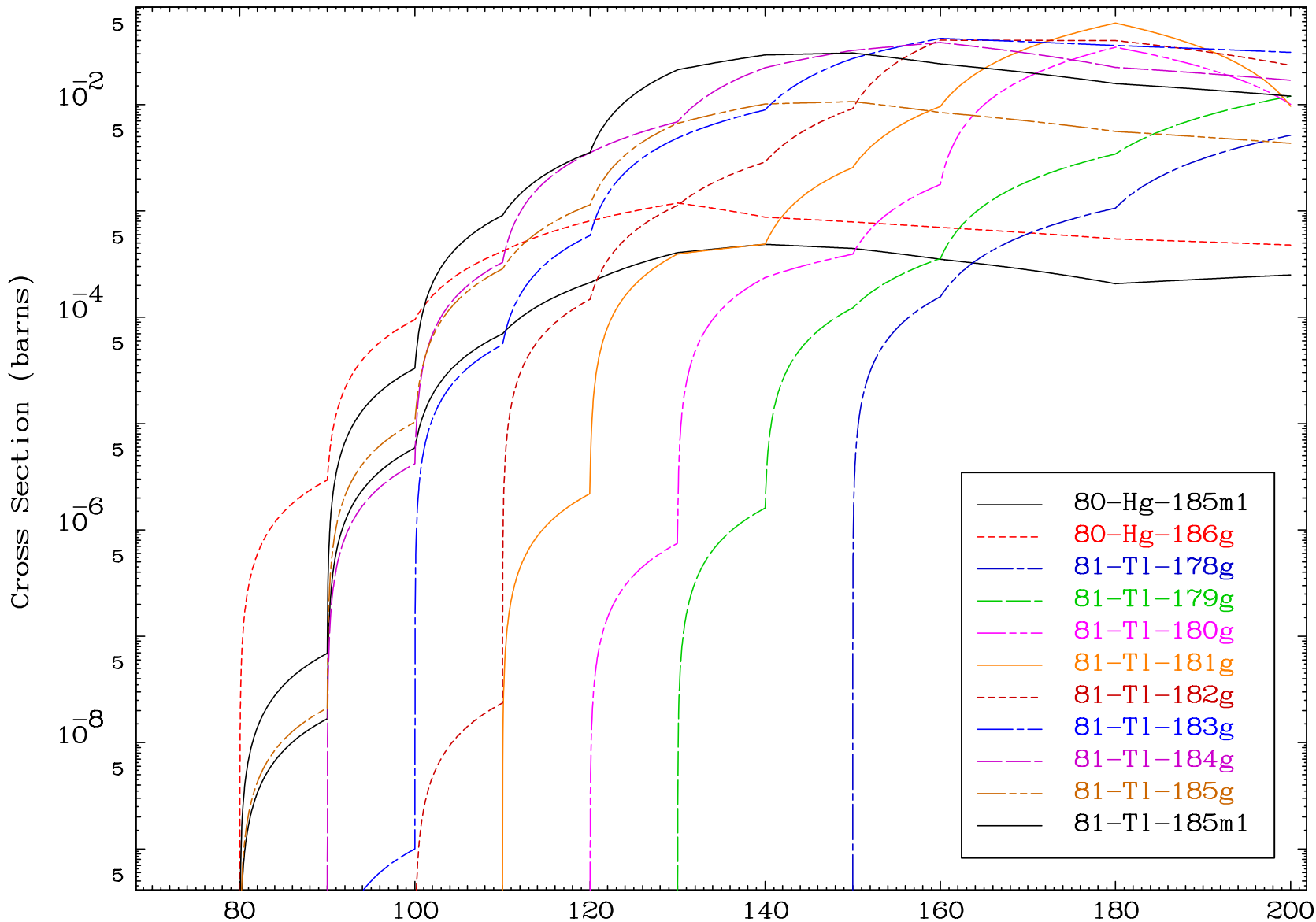
Radionuclide Production Cross Section



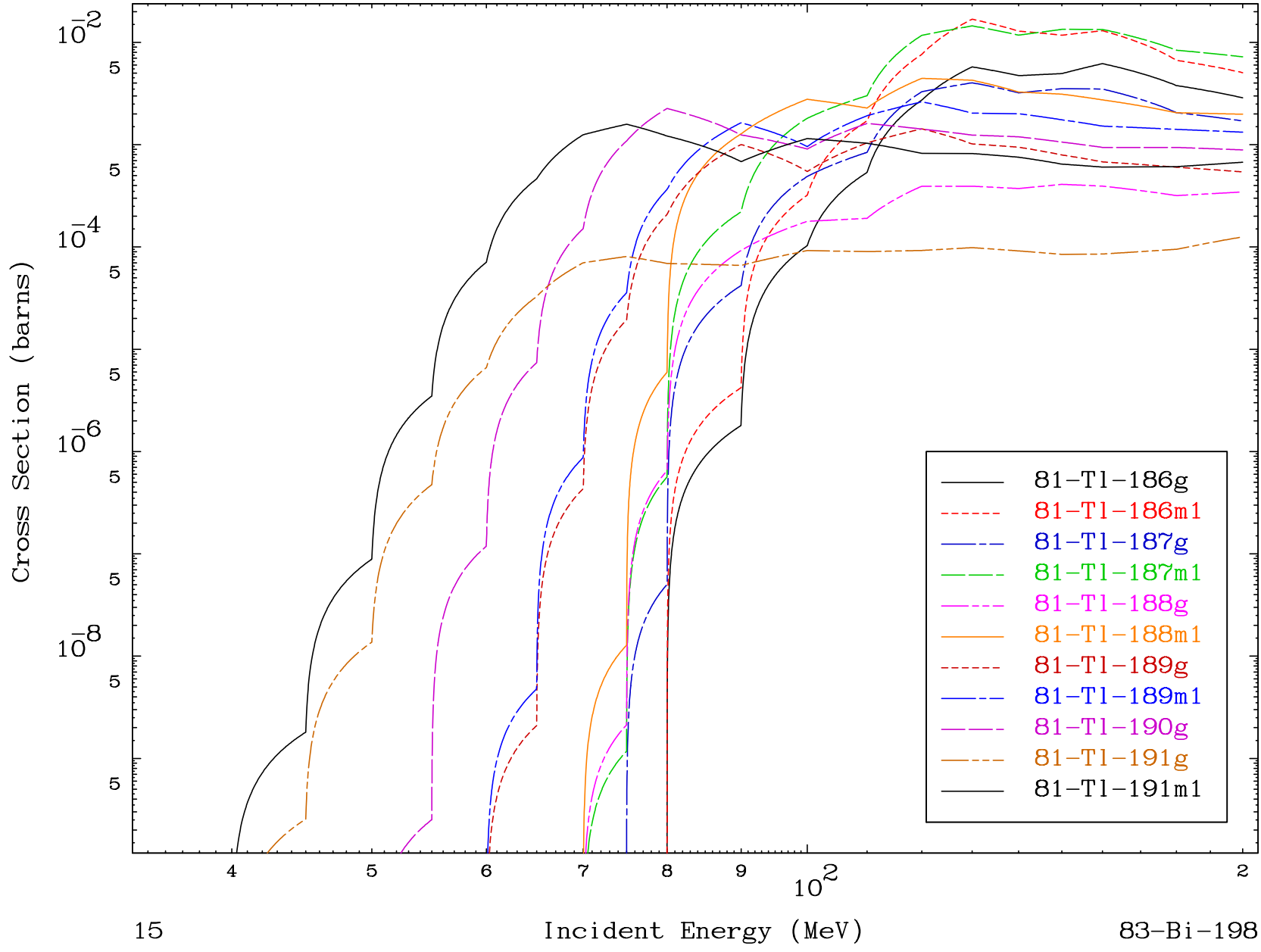
Radionuclide Production Cross Section



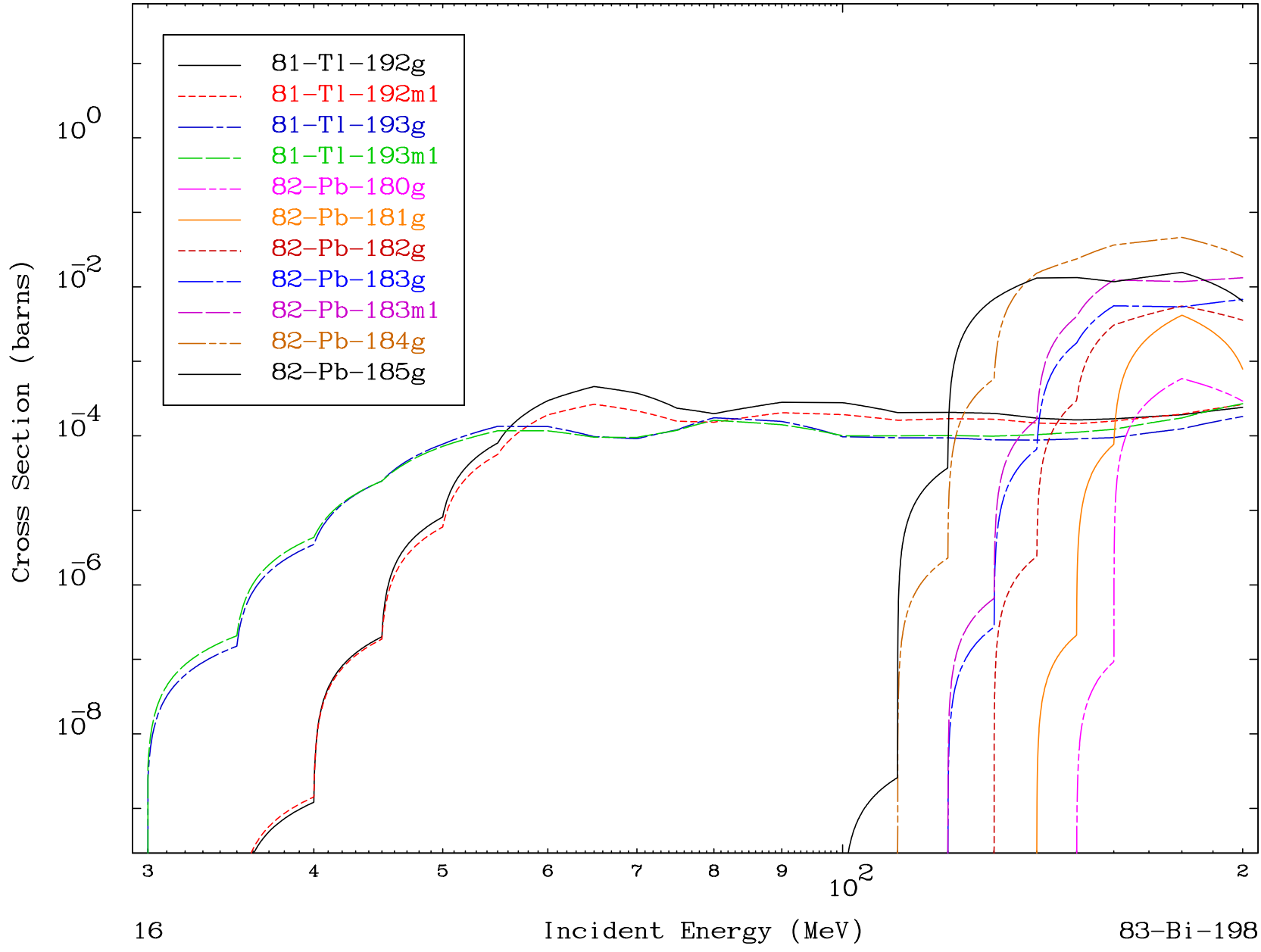
Radionuclide Production Cross Section



Radionuclide Production Cross Section

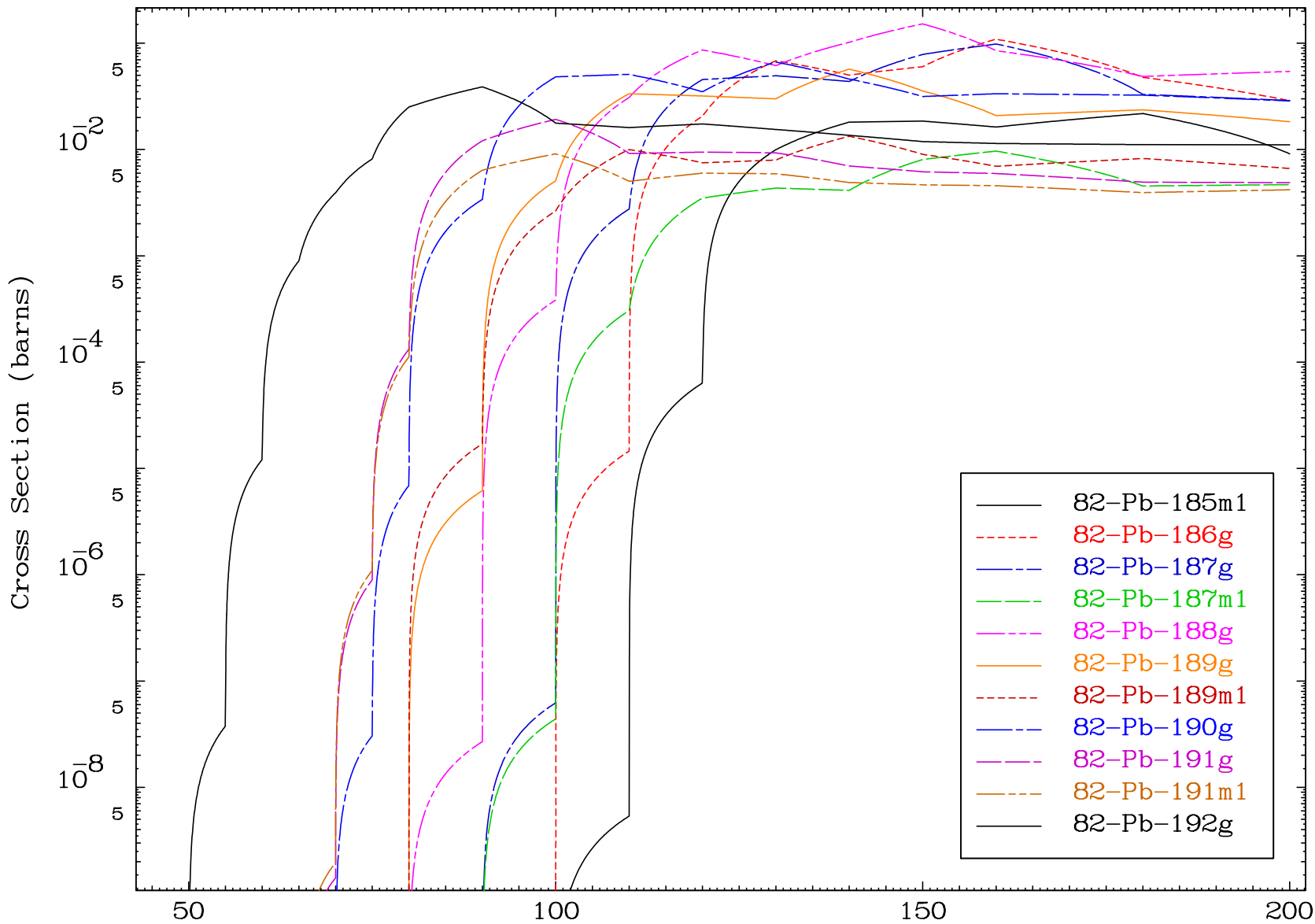


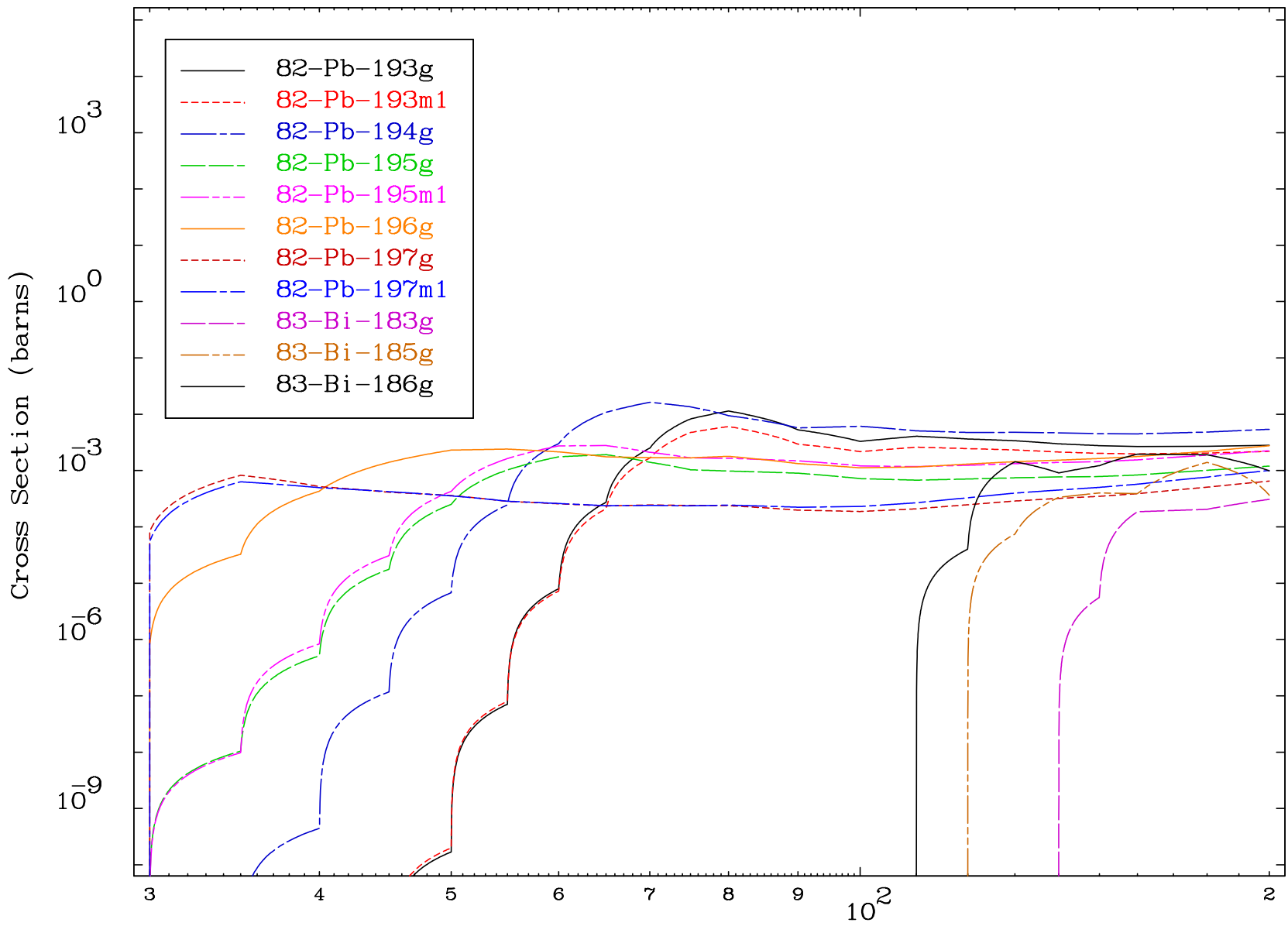
Radionuclide Production Cross Section



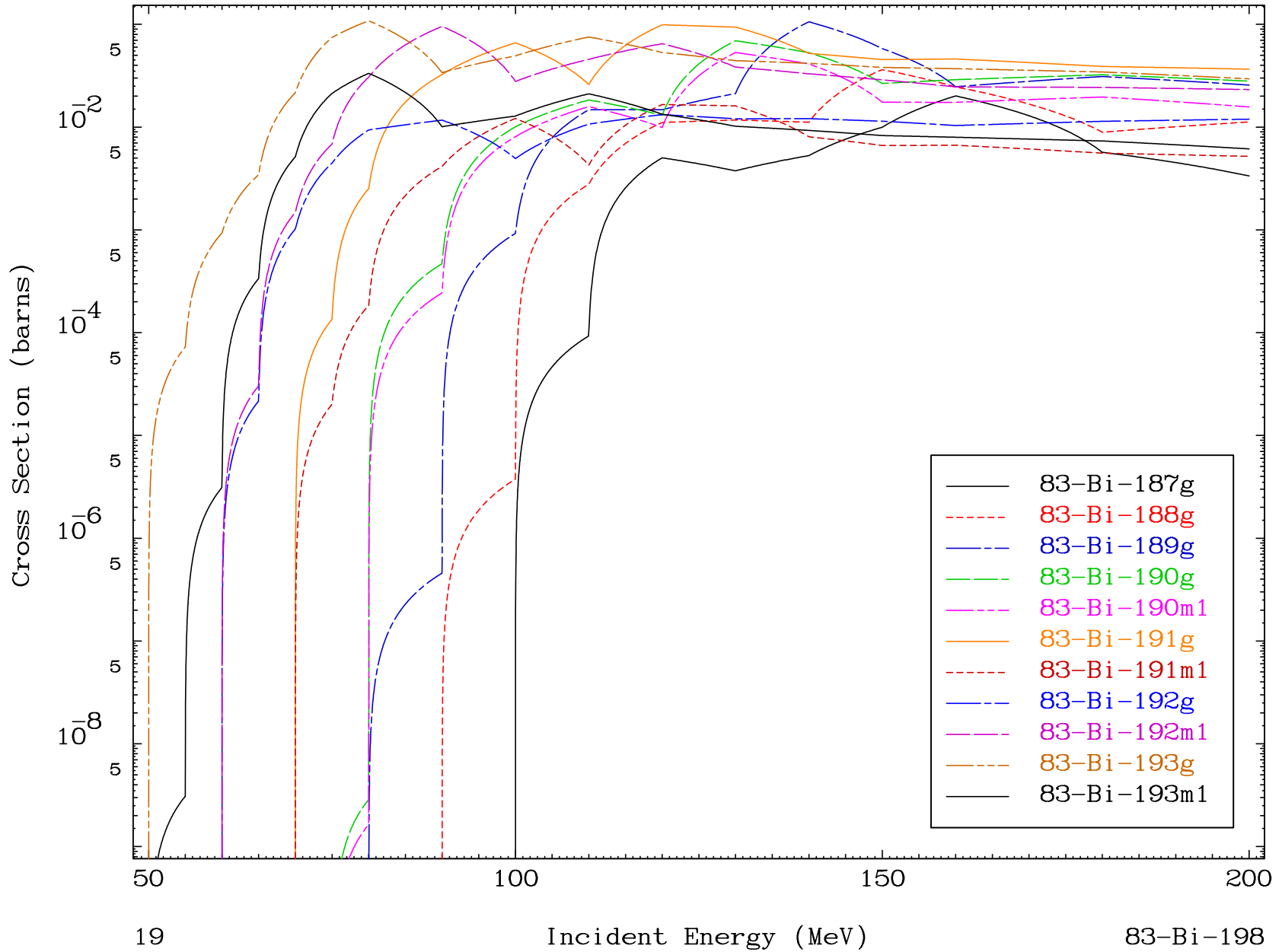


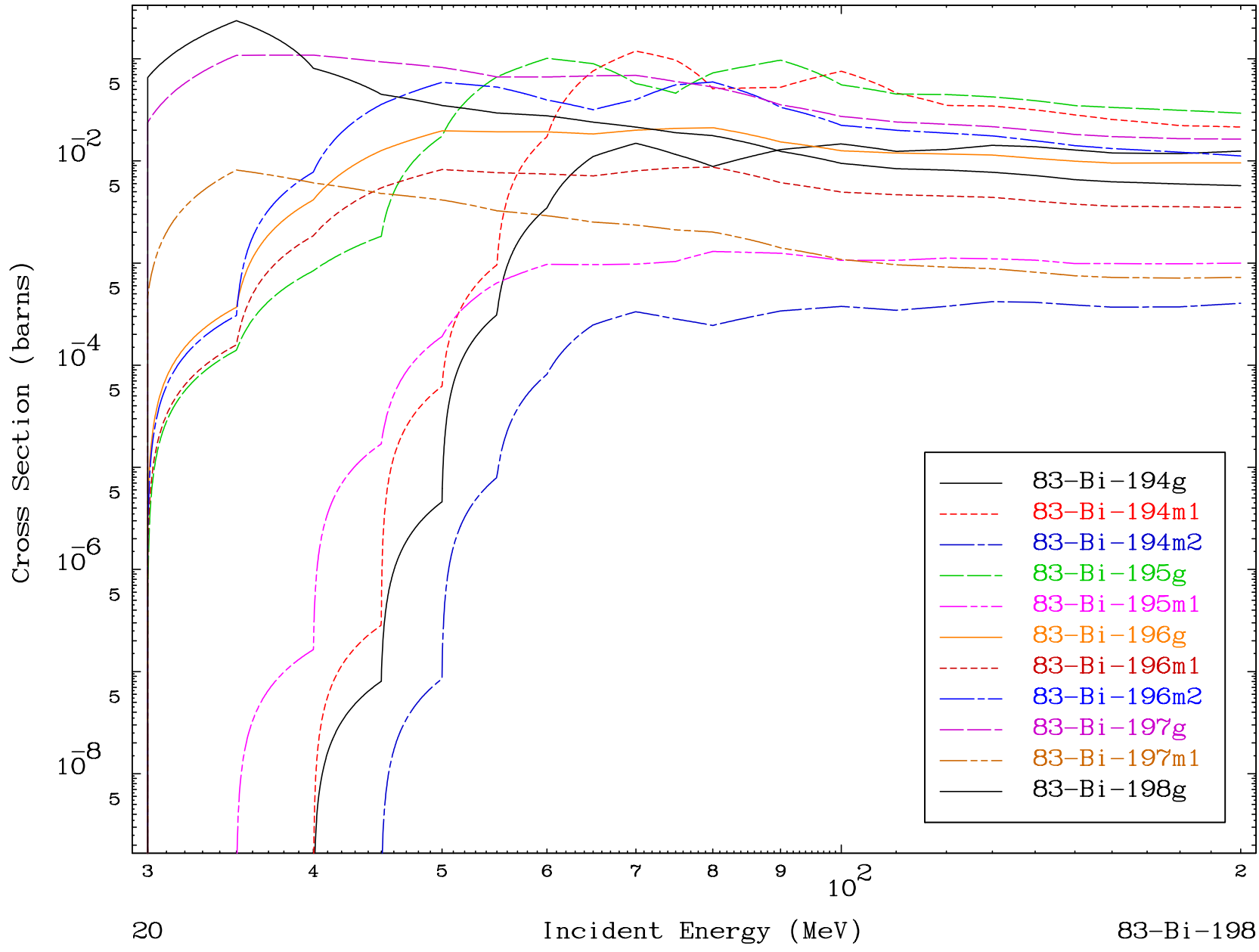
Radionuclide Production Cross Section



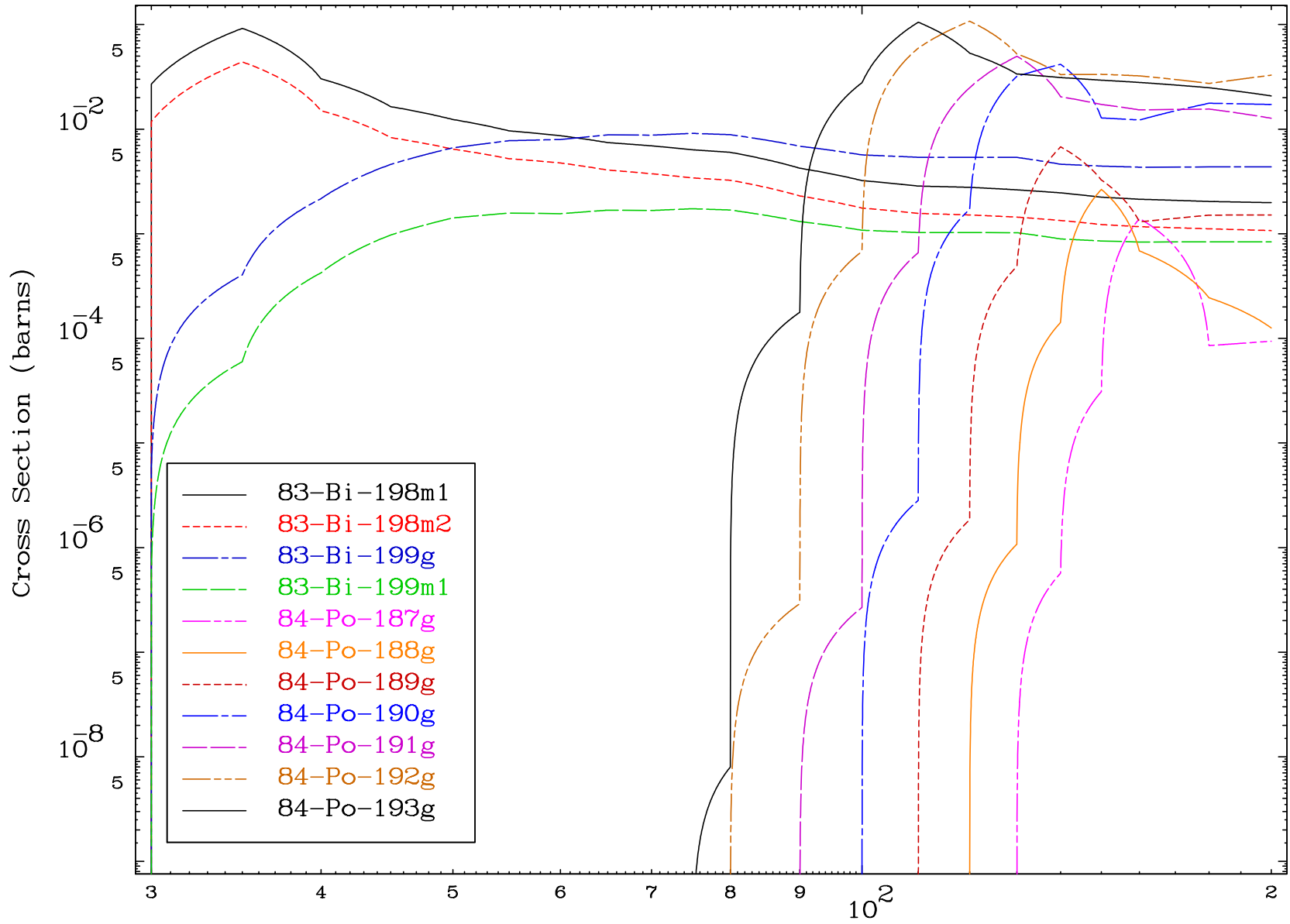


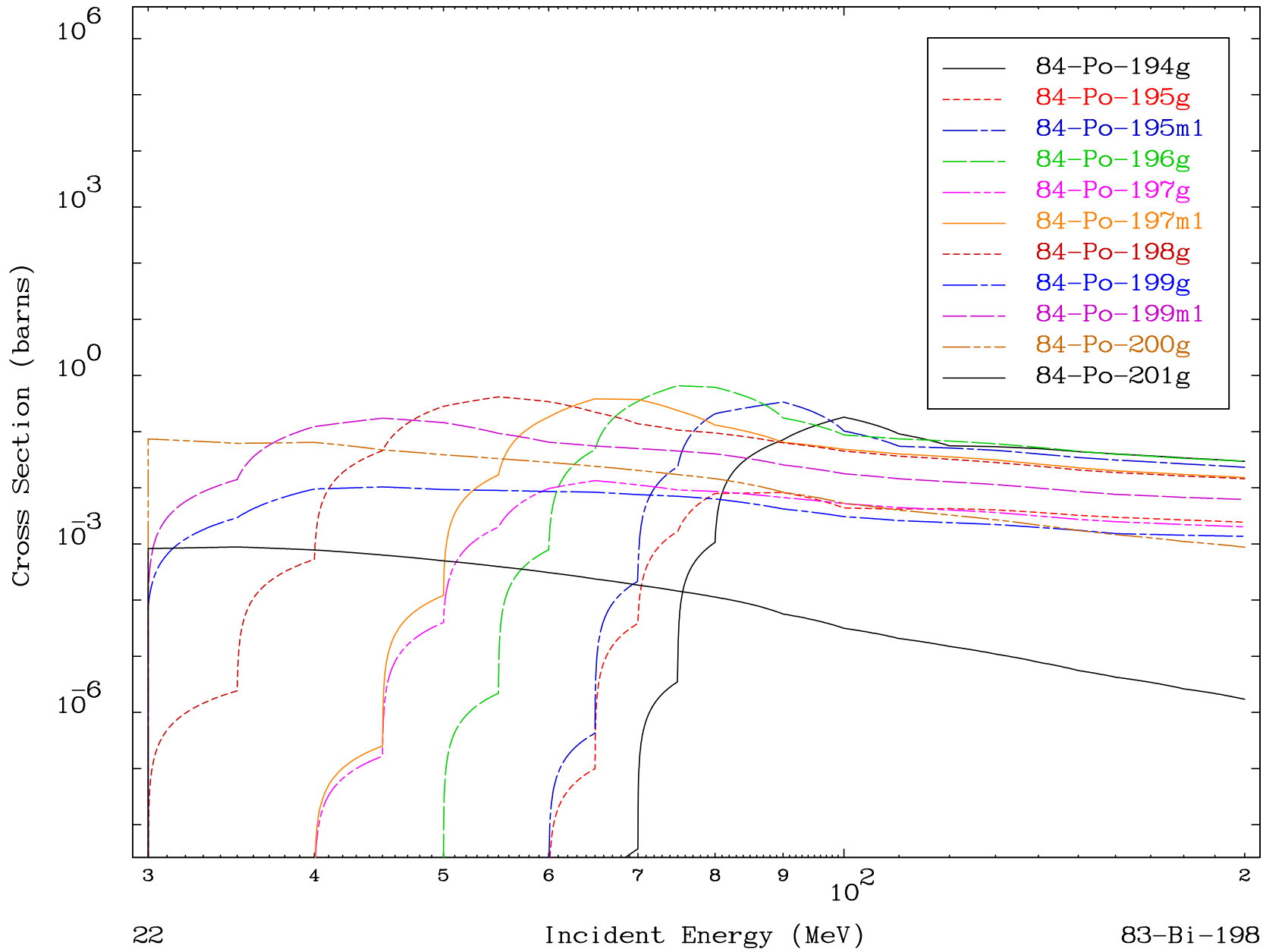
Radionuclide Production Cross Section

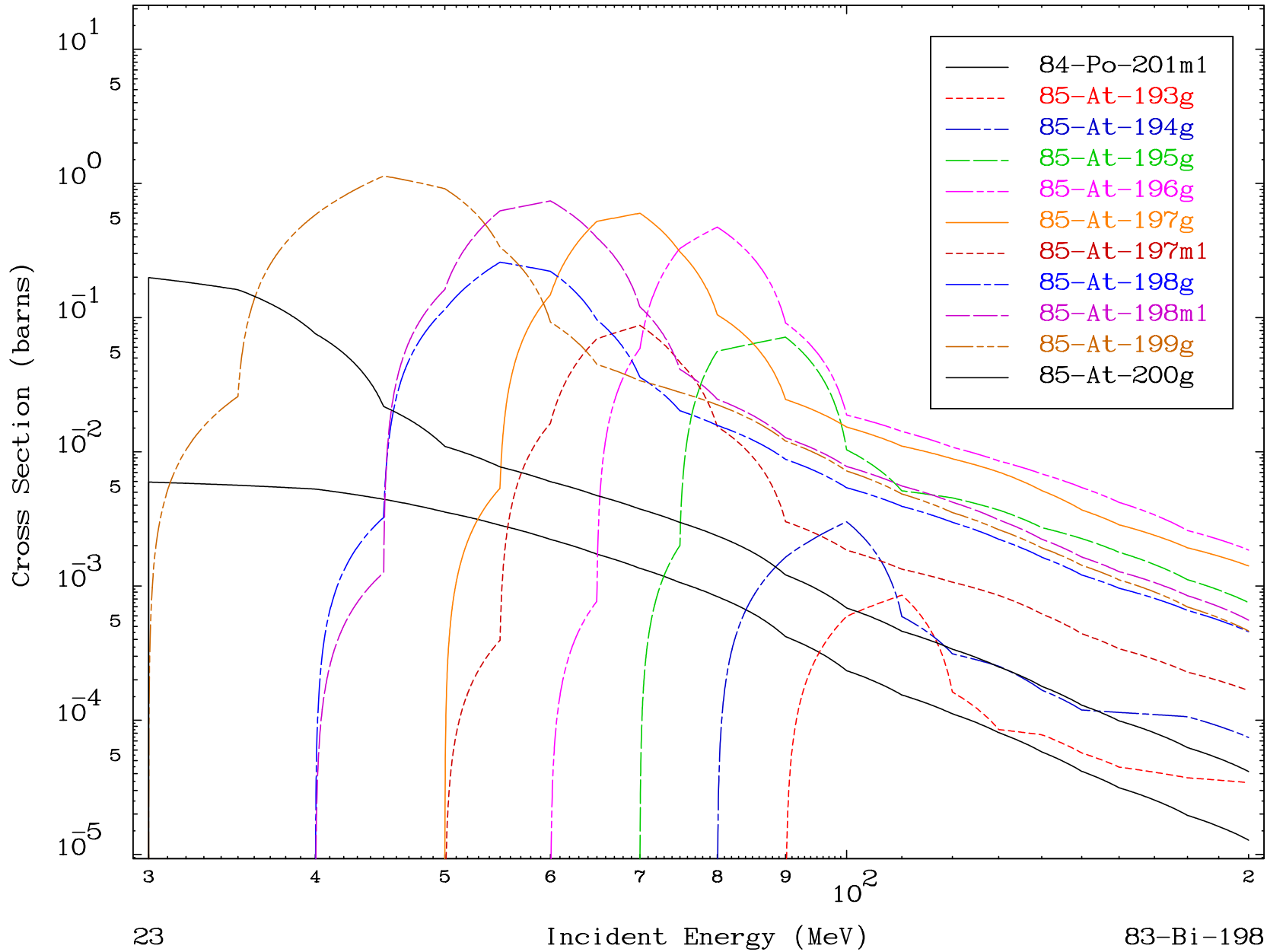




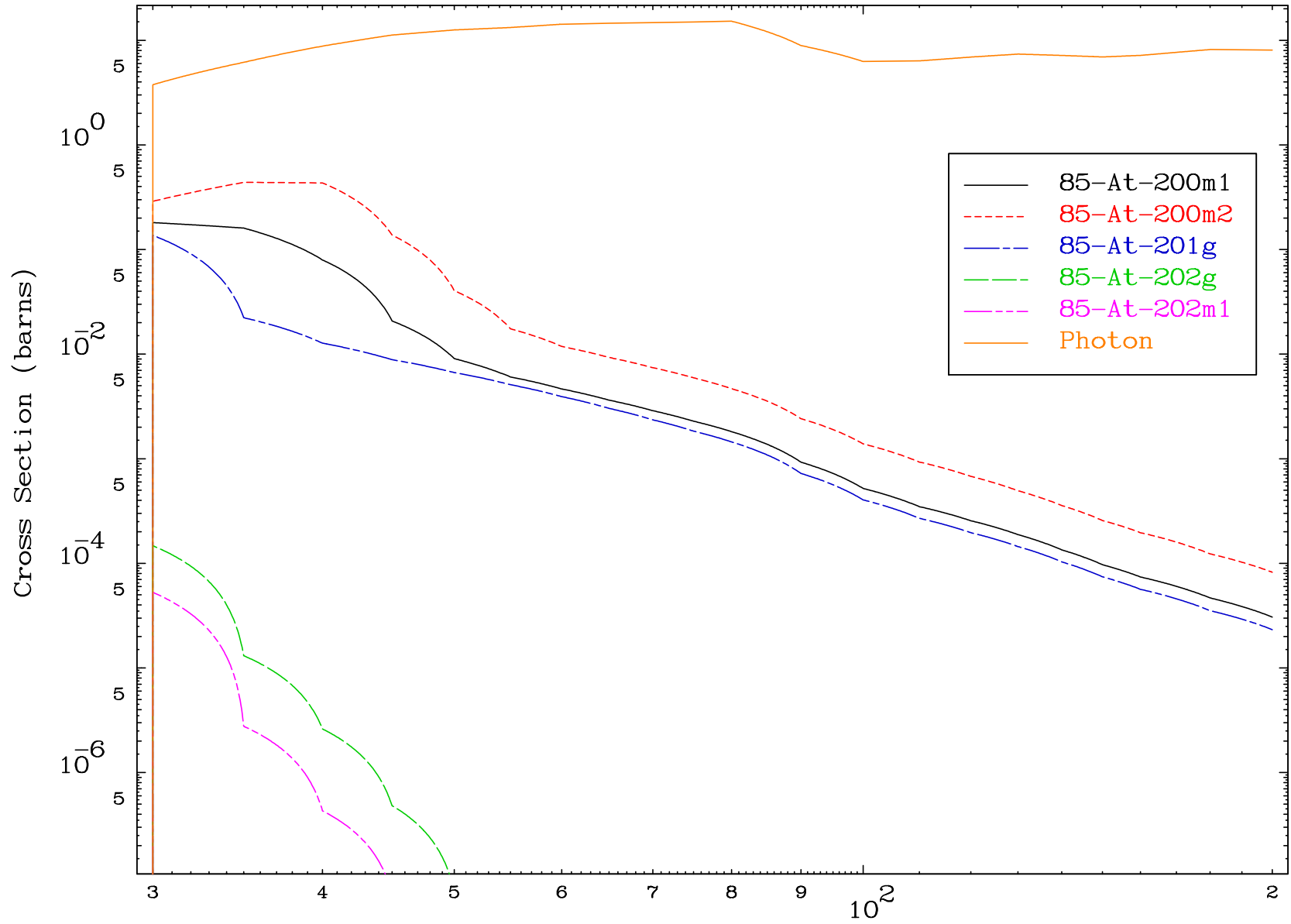
Radionuclide Production Cross Section







Radionuclide Production Cross Section



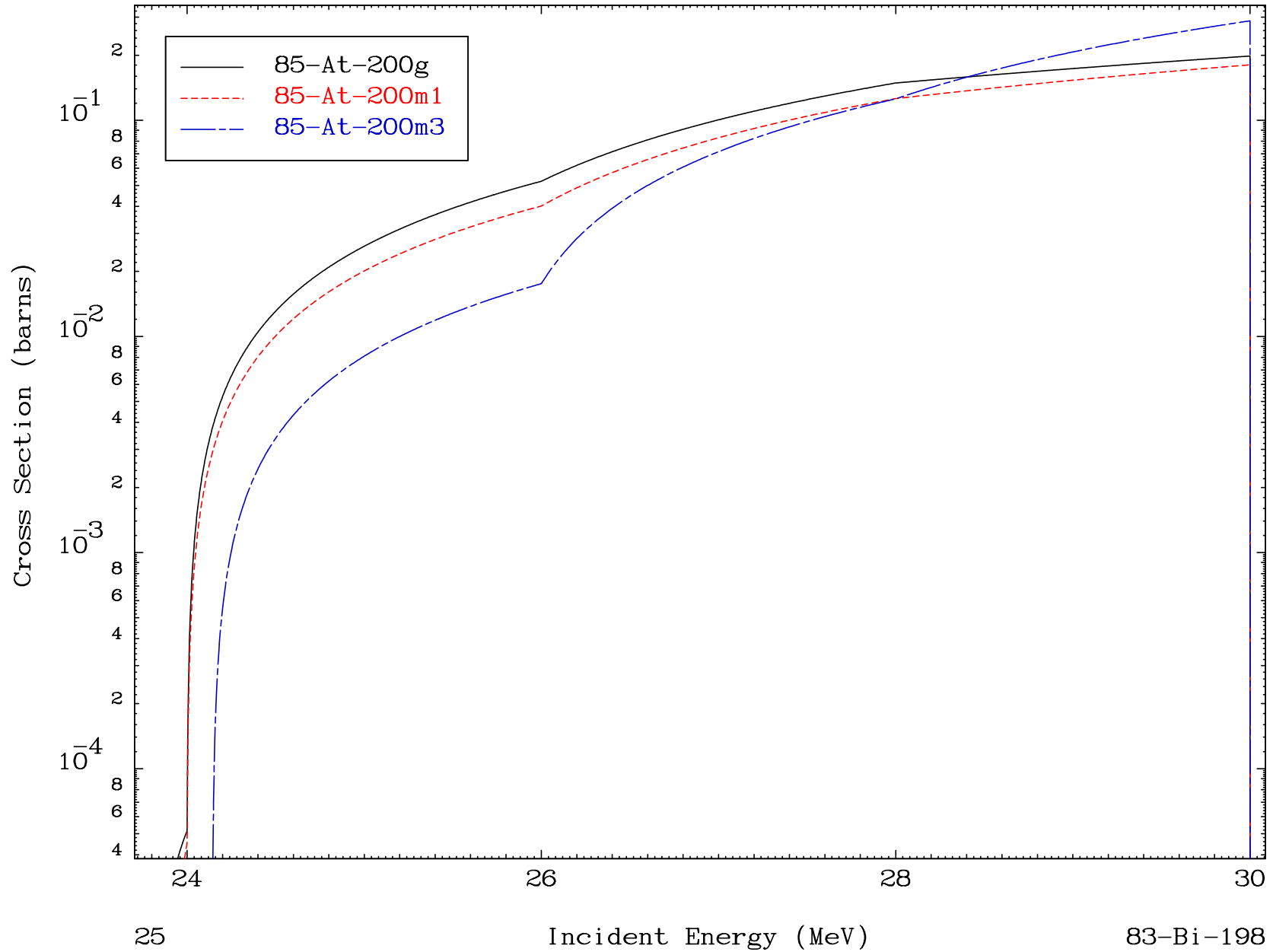


MAT 8293

( $\alpha, 2n$ )

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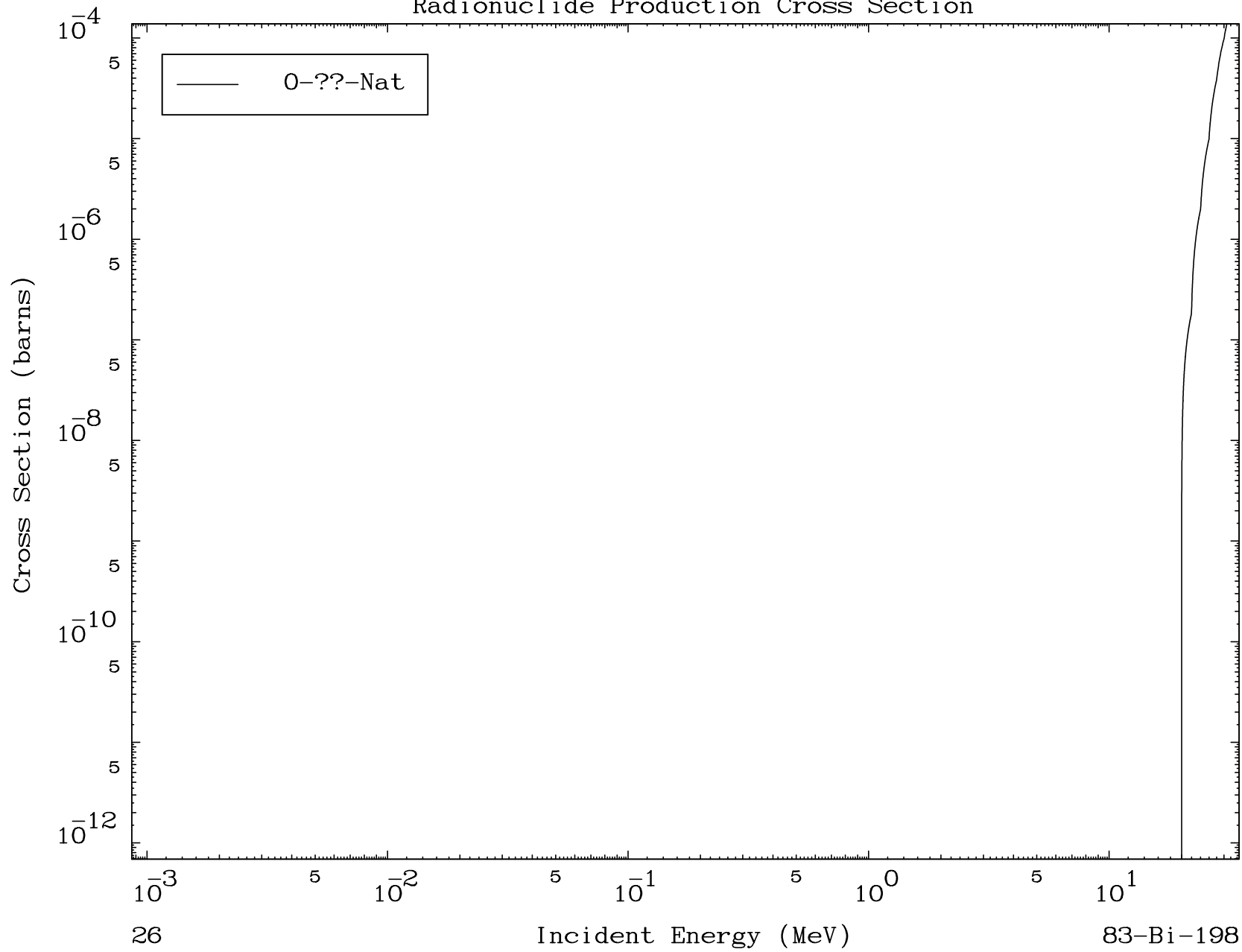
Radionuclide Production Cross Section



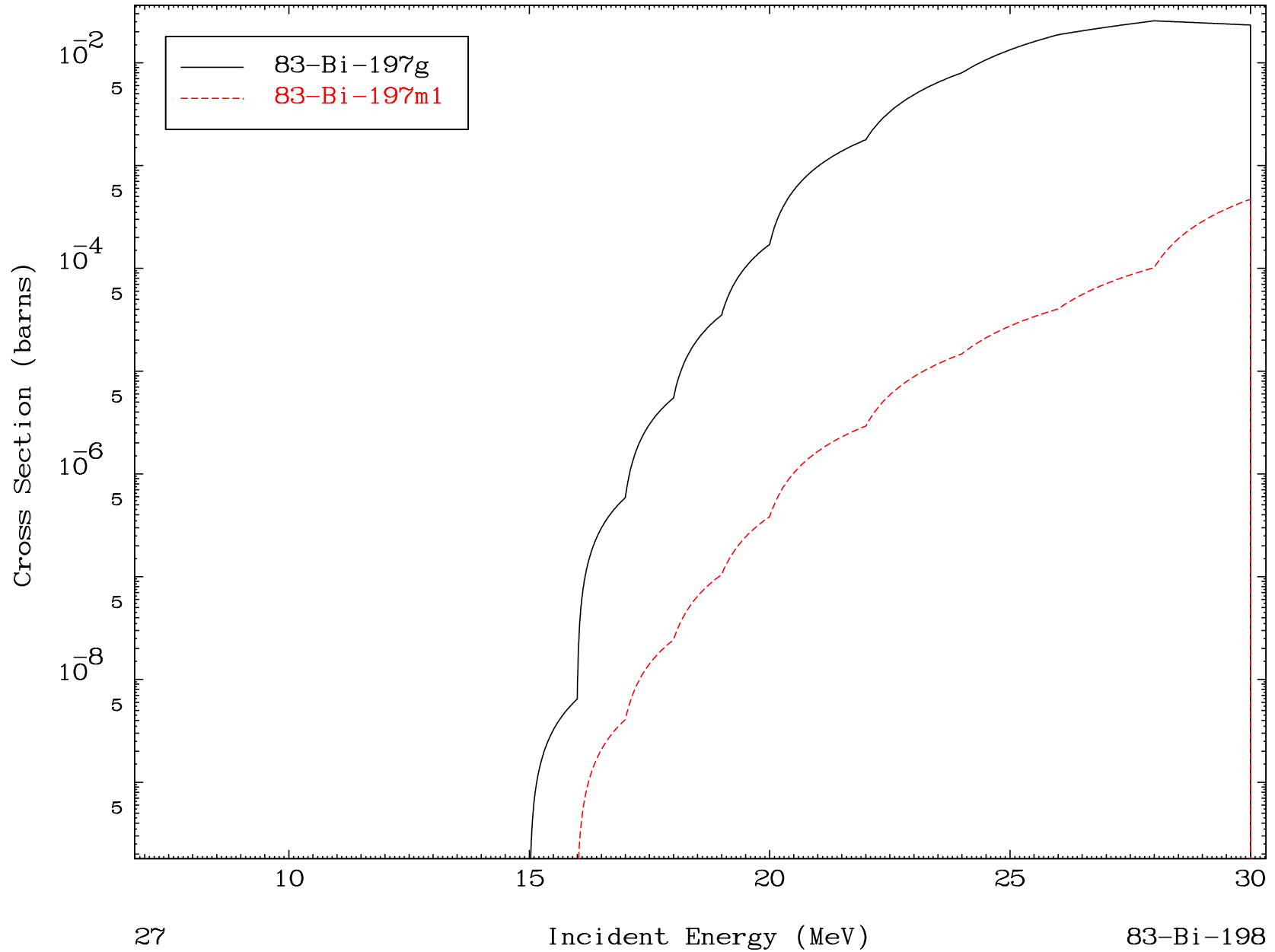
MAT 8293

$\alpha$  Fission  
Radionuclide Production Cross Section

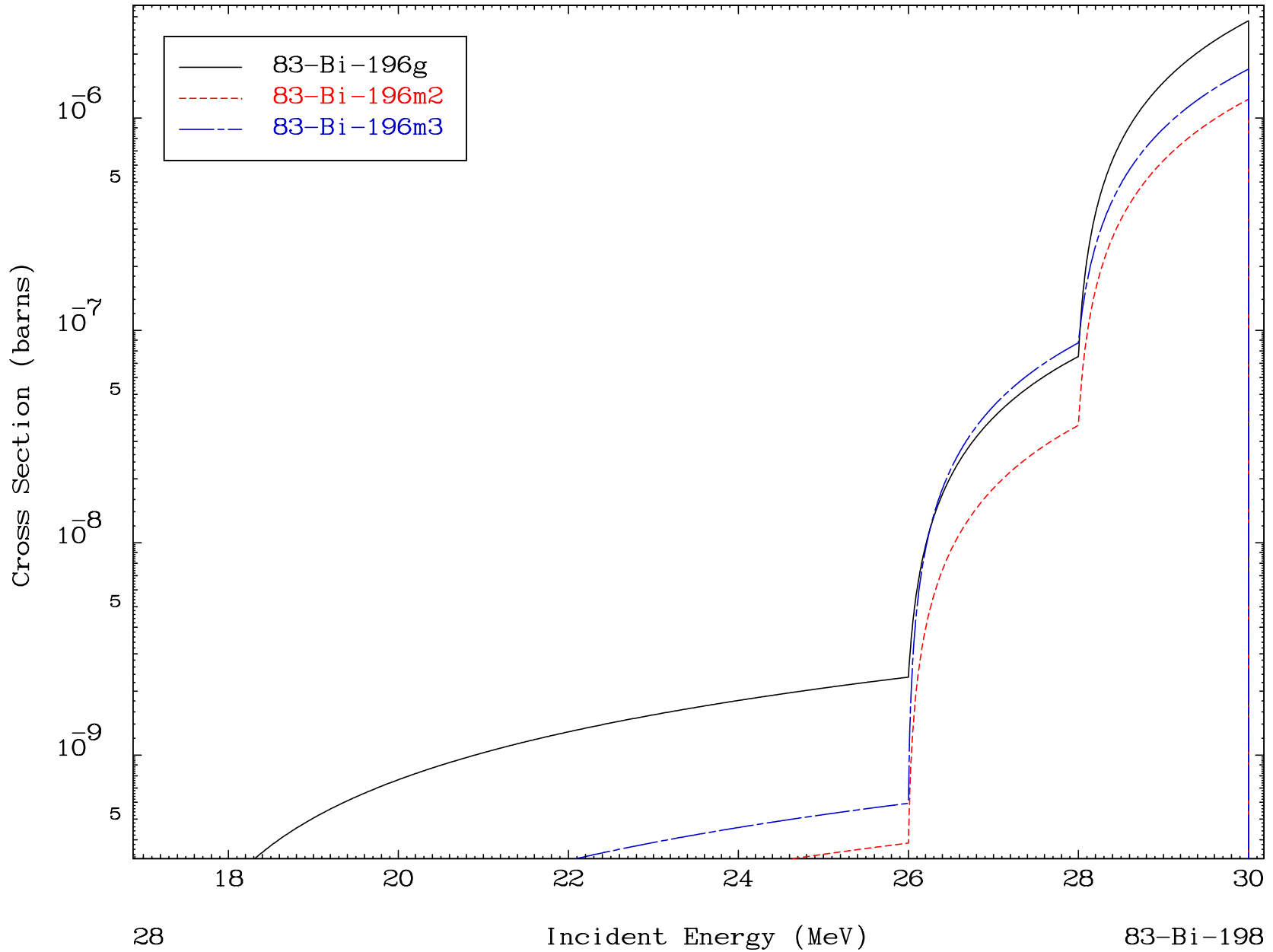
83-Bi-198



Radionuclide Production Cross Section



Radionuclide Production Cross Section

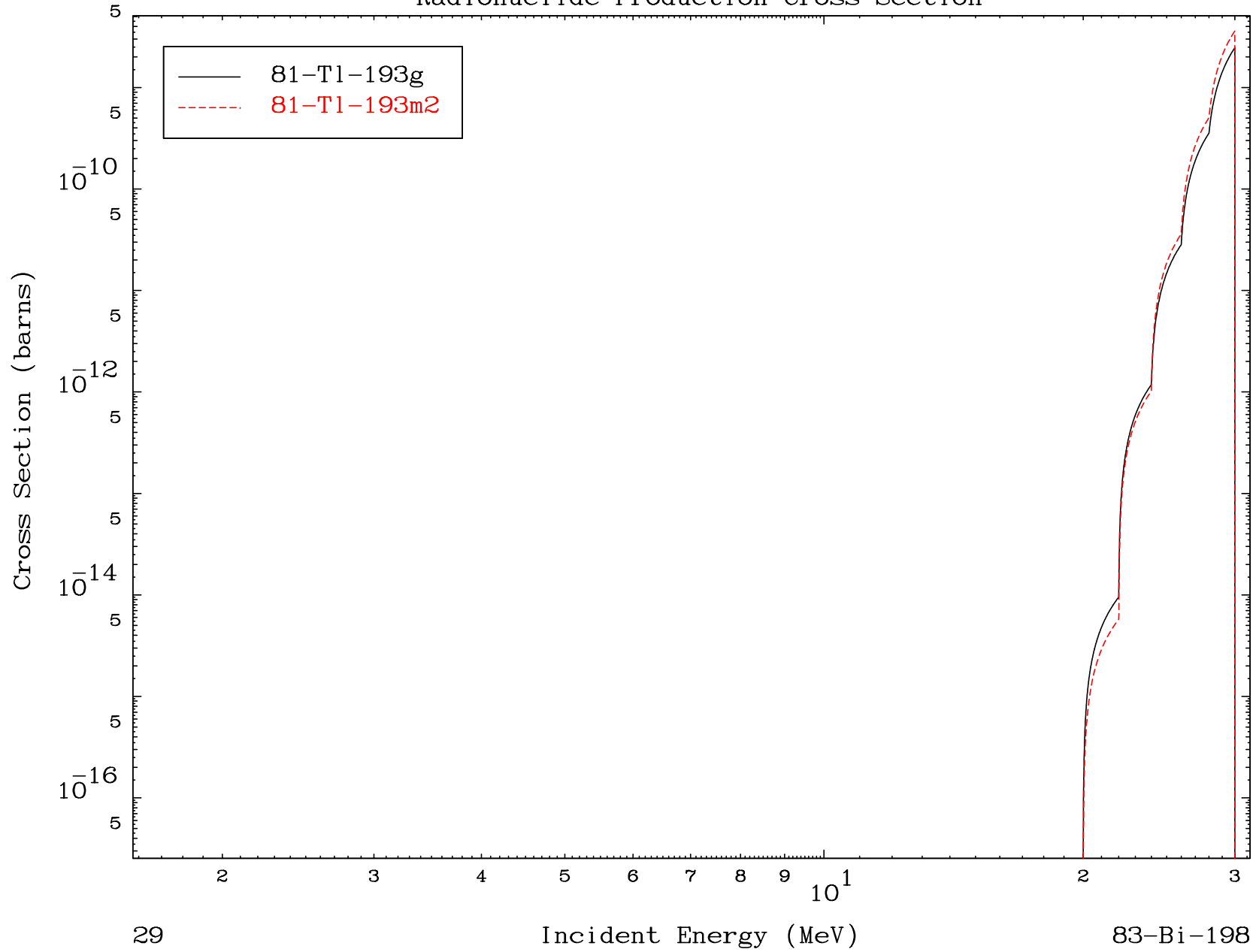


MAT 8293

$(\alpha, n')$   $2\alpha$

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Radionuclide Production Cross Section

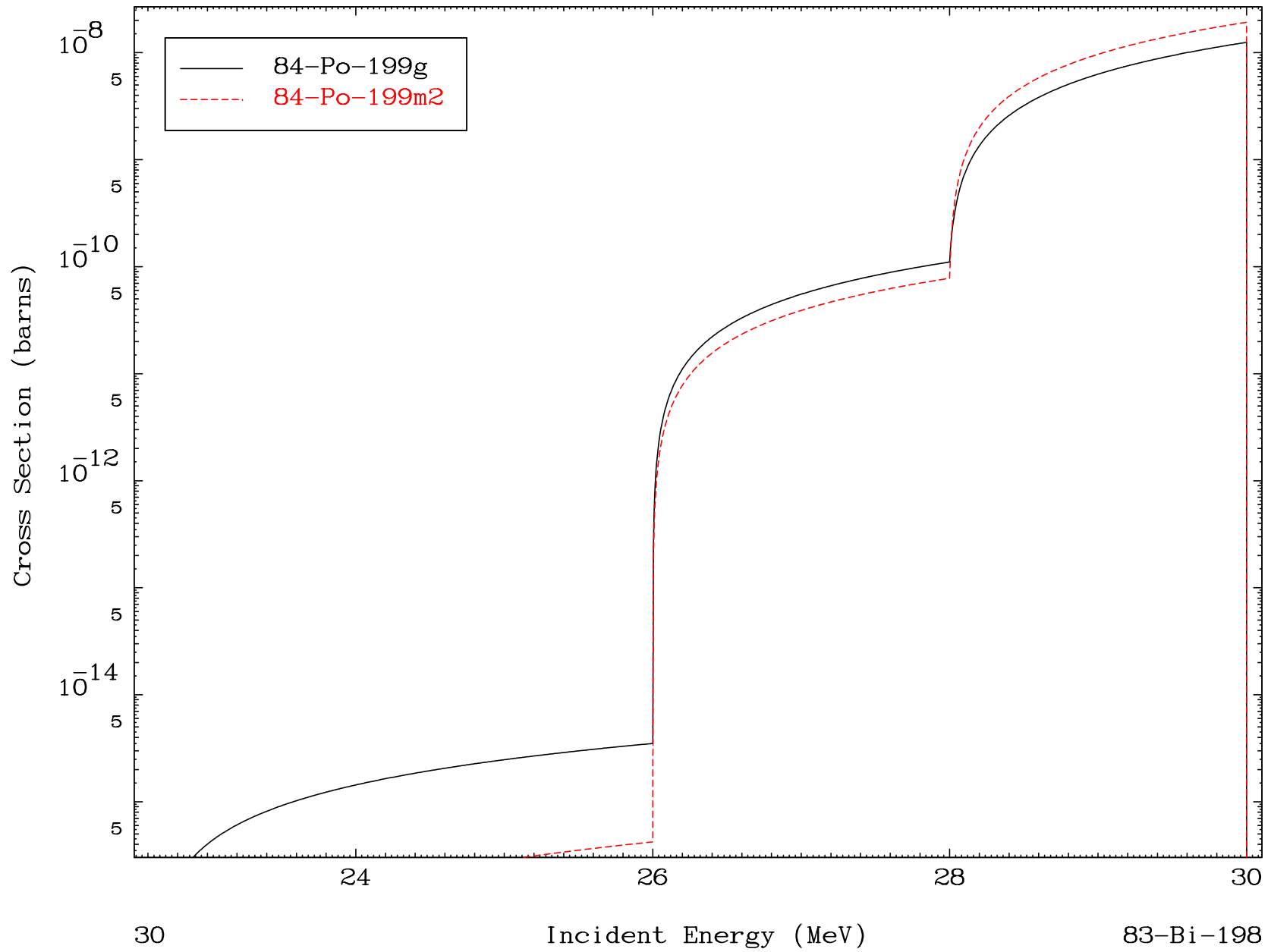


MAT 8293

( $\alpha, n'$ ) d

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Radionuclide Production Cross Section

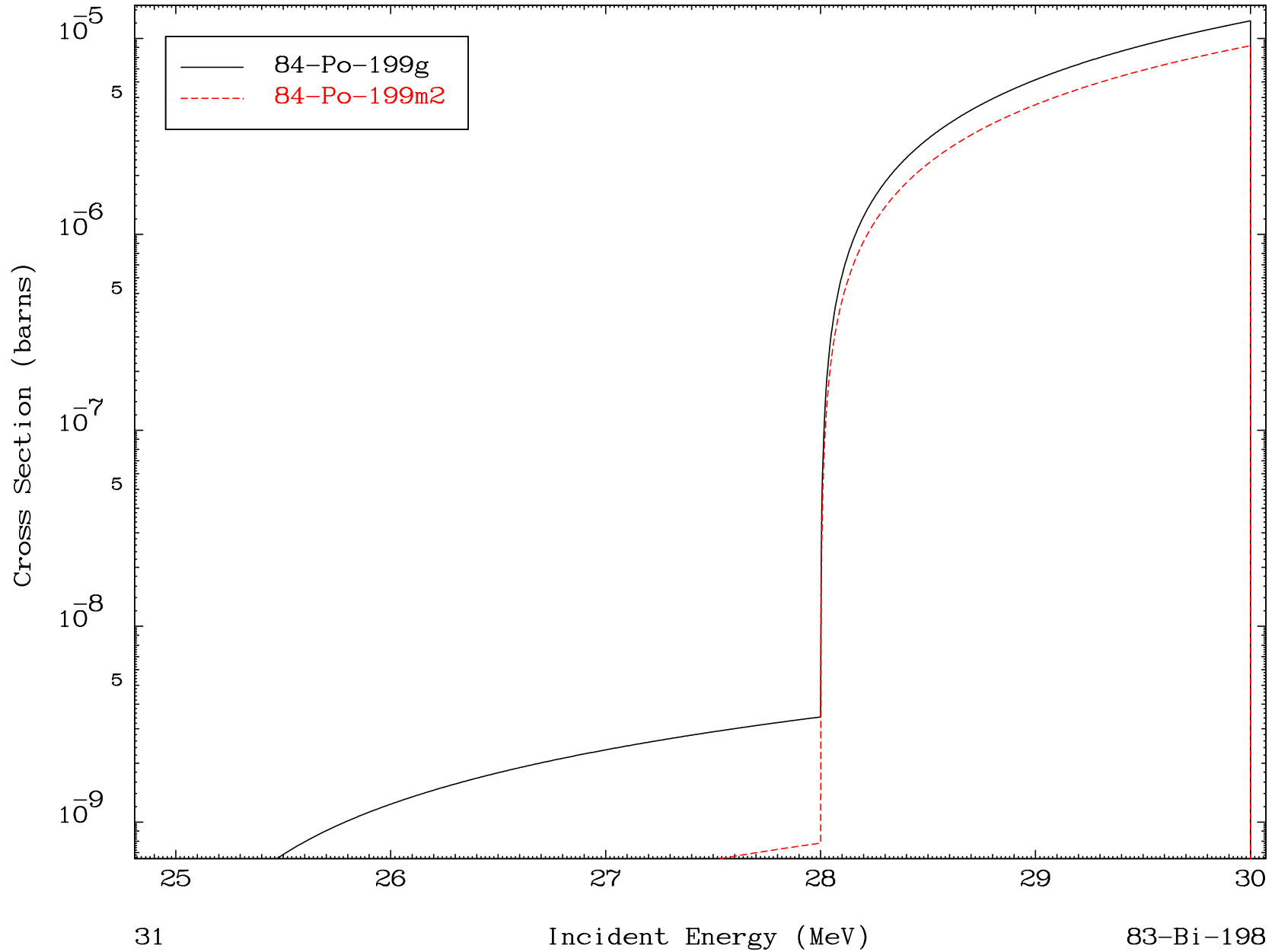


MAT 8293

( $\alpha, 2n$ ) p

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Radionuclide Production Cross Section

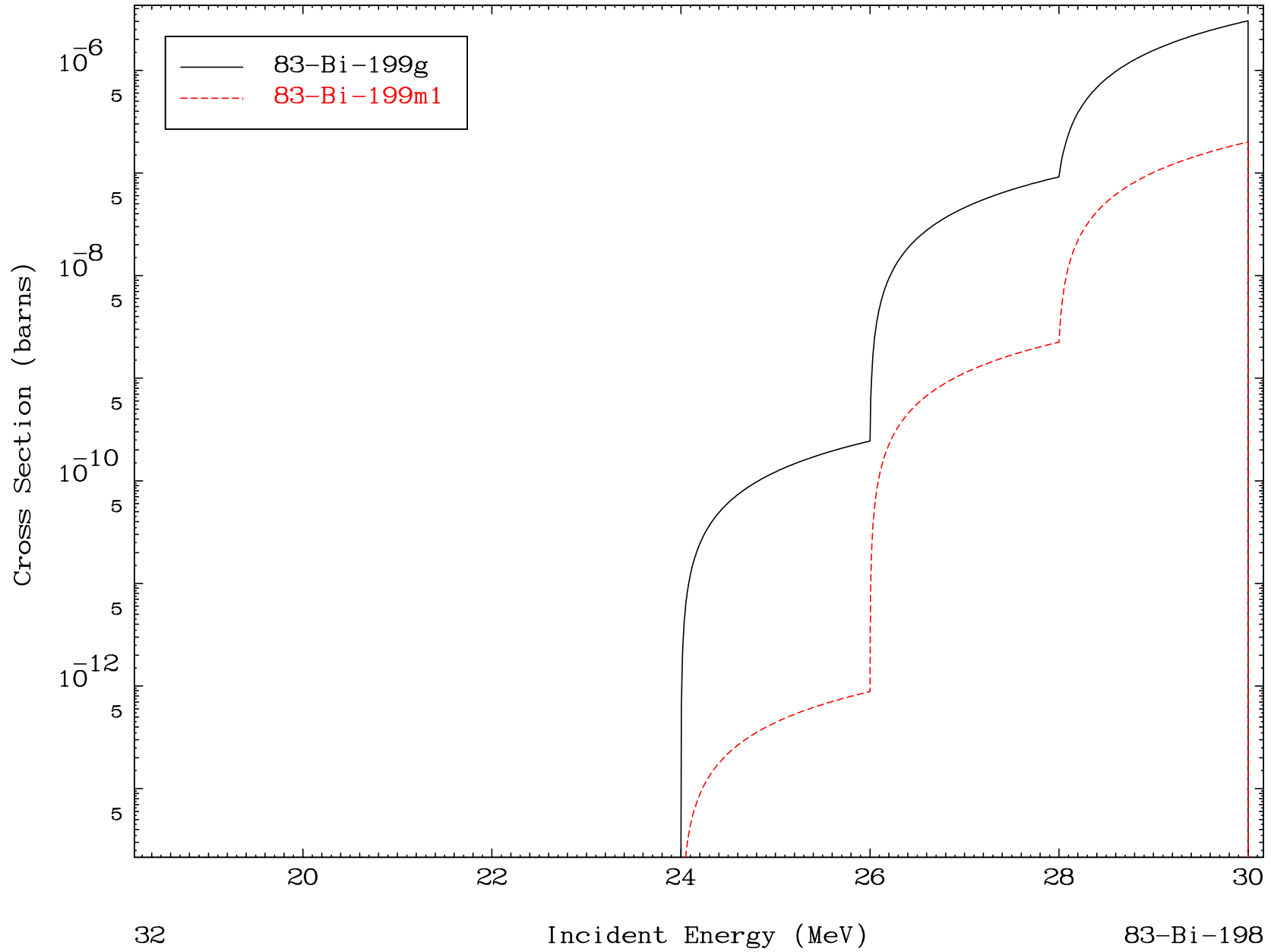


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Incident Energy (MeV)

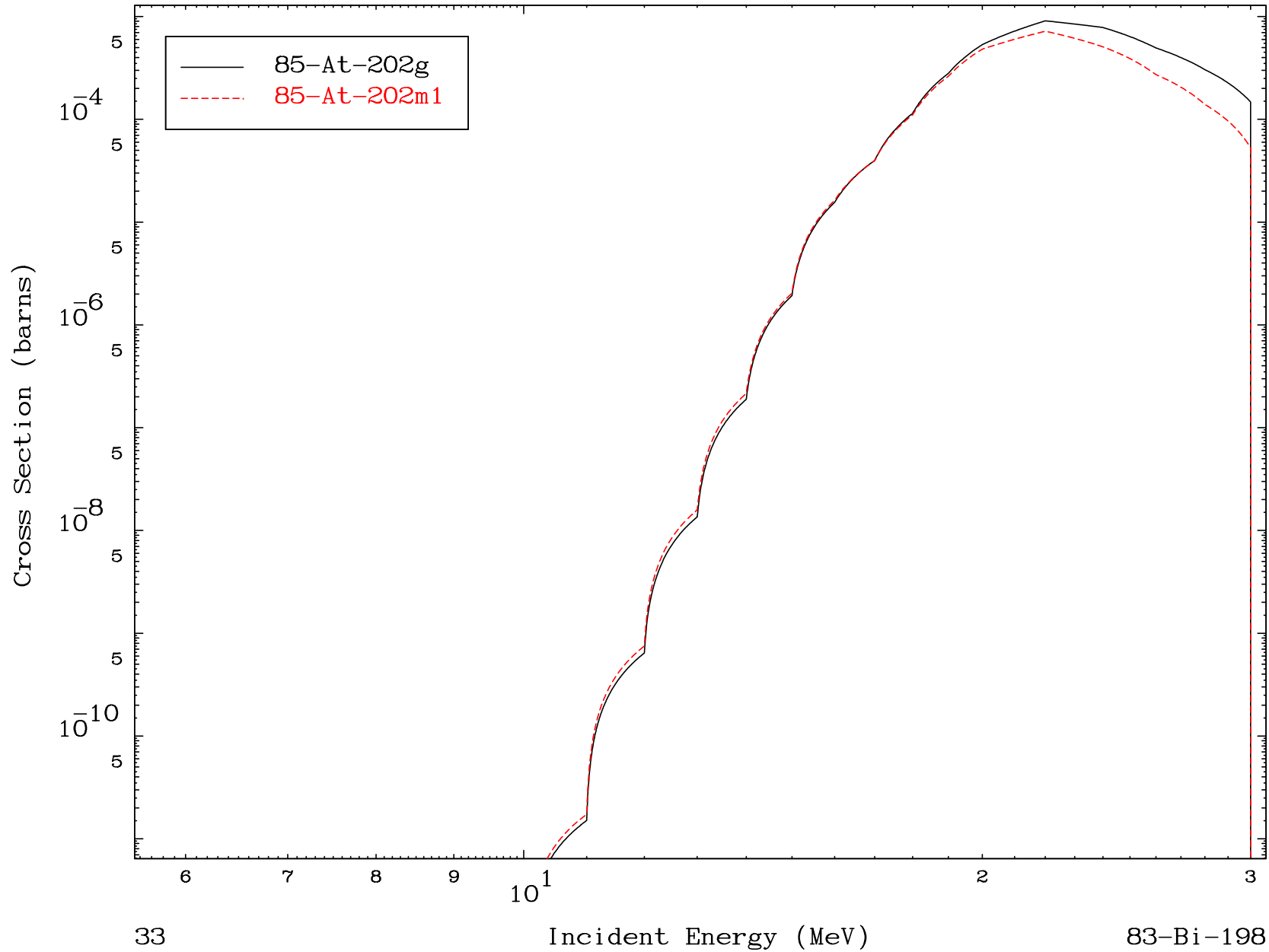
83-Bi-198

Radionuclide Production Cross Section





Radionuclide Production Cross Section

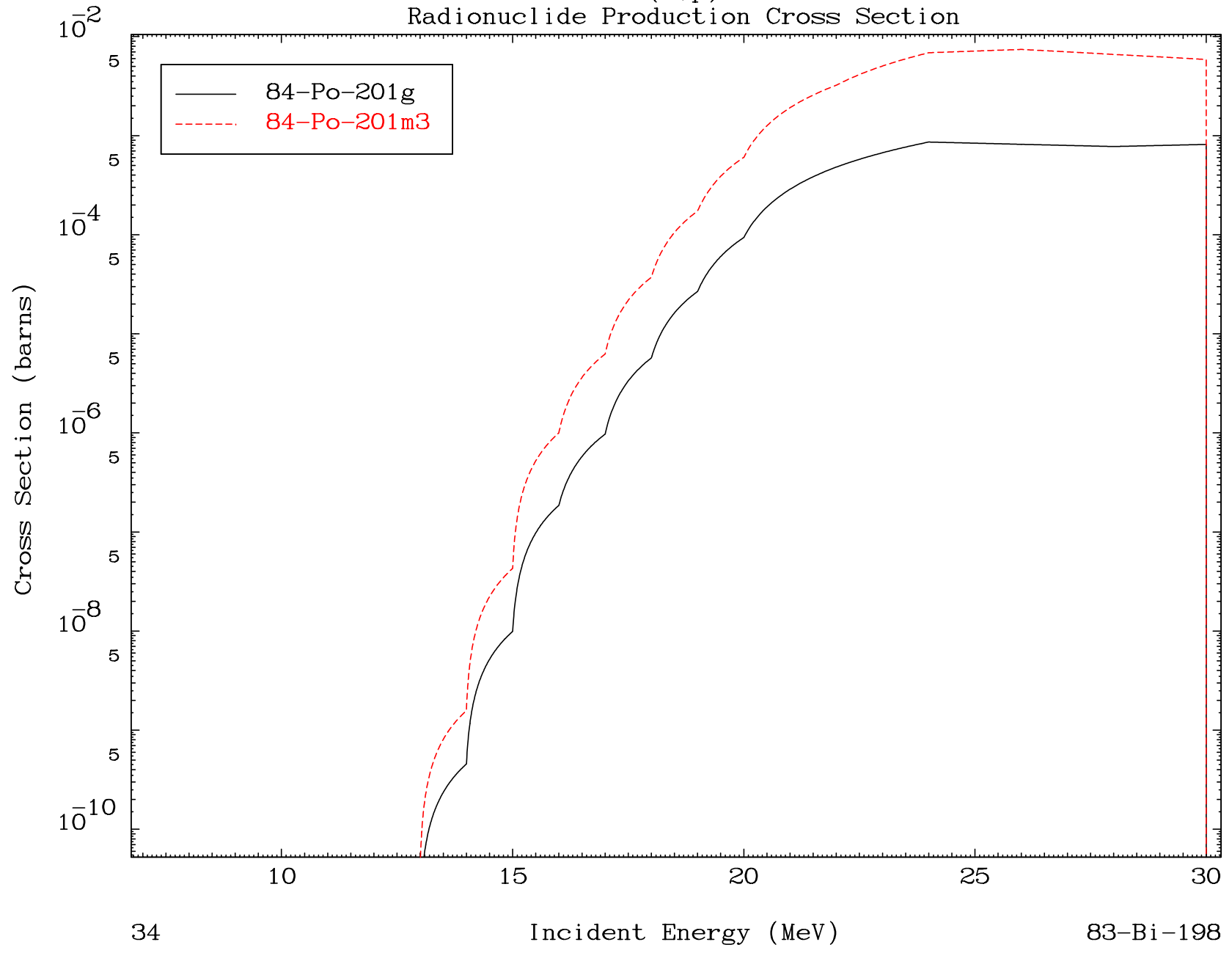


MAT 8293

( $\alpha, p$ )

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### Radionuclide Production Cross Section



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Incident Energy (MeV)

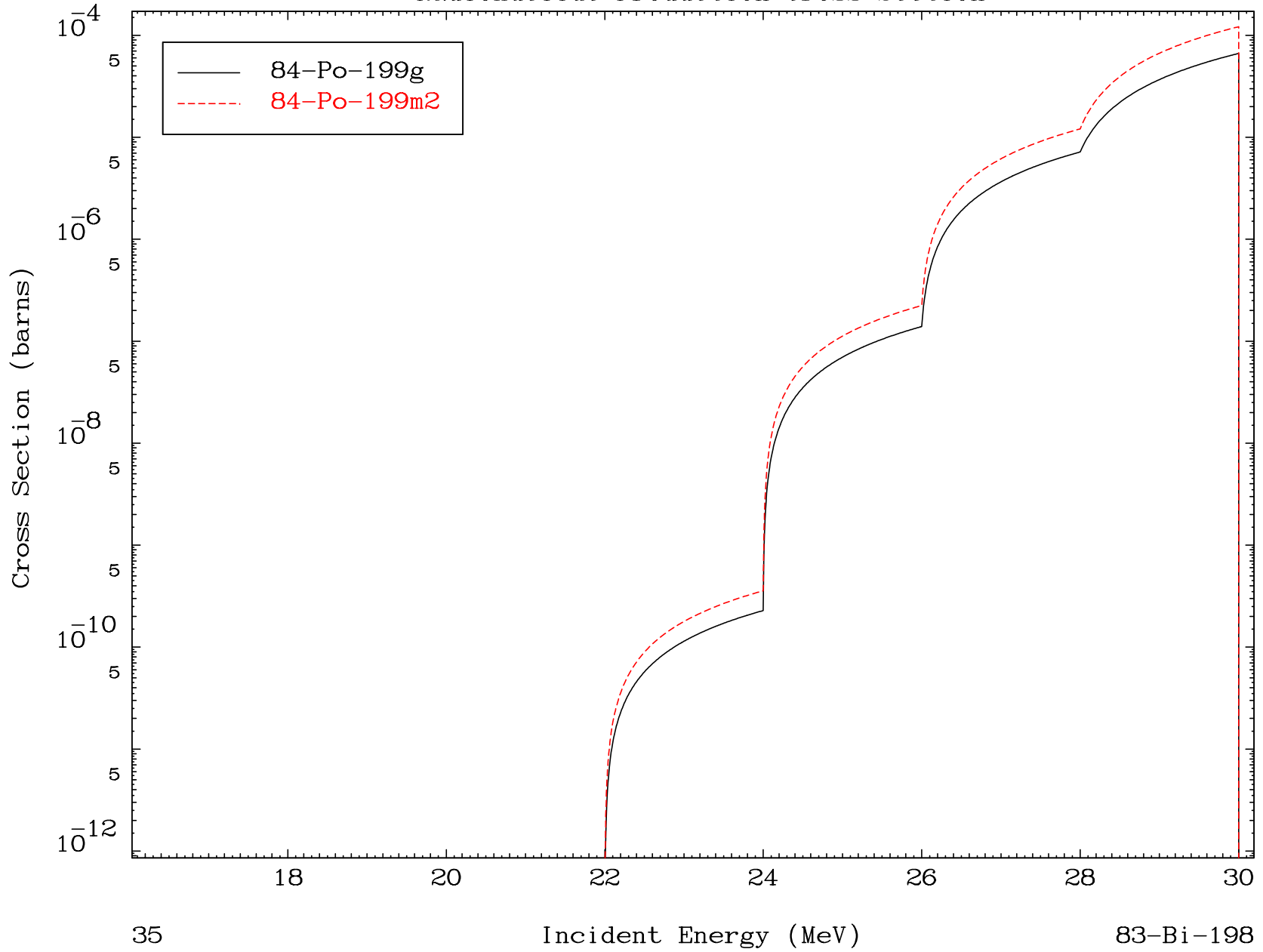
83-Bi-198

MAT 8293

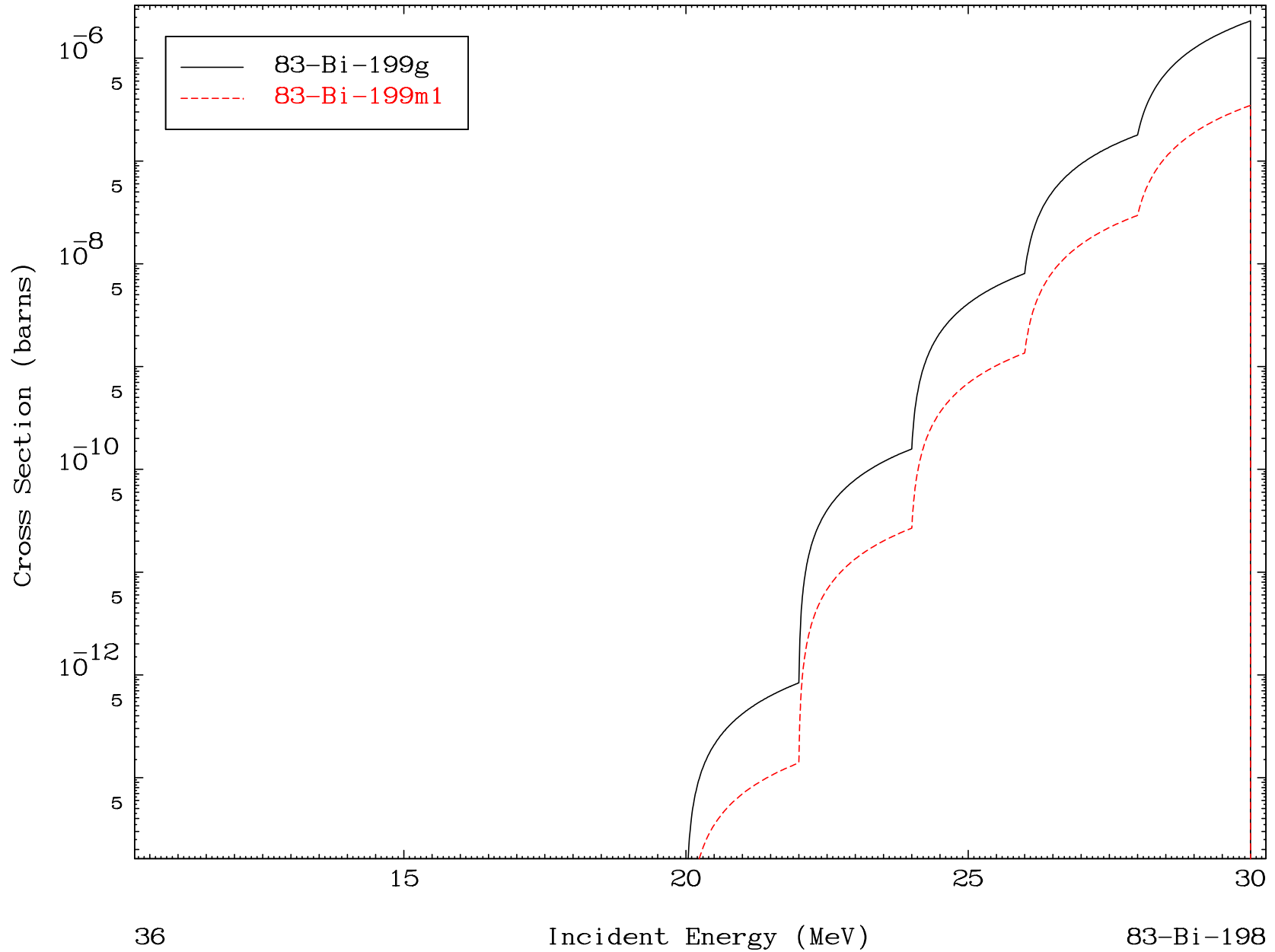
( $\alpha, t$ )

83-Bi-198

Radionuclide Production Cross Section



Radionuclide Production Cross Section

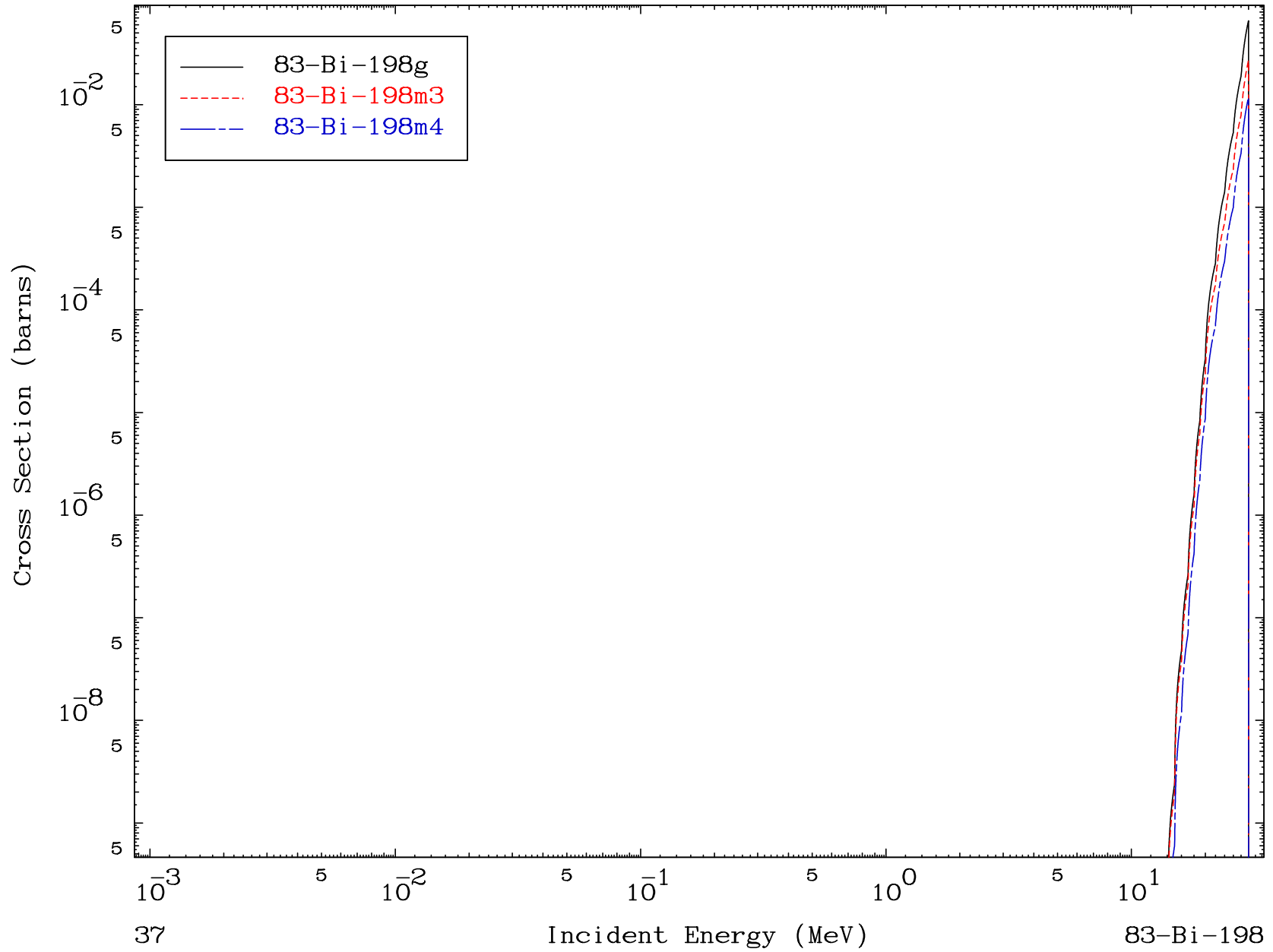


MAT 8293

( $\alpha, \alpha$ )

83-Bi-198

Radionuclide Production Cross Section

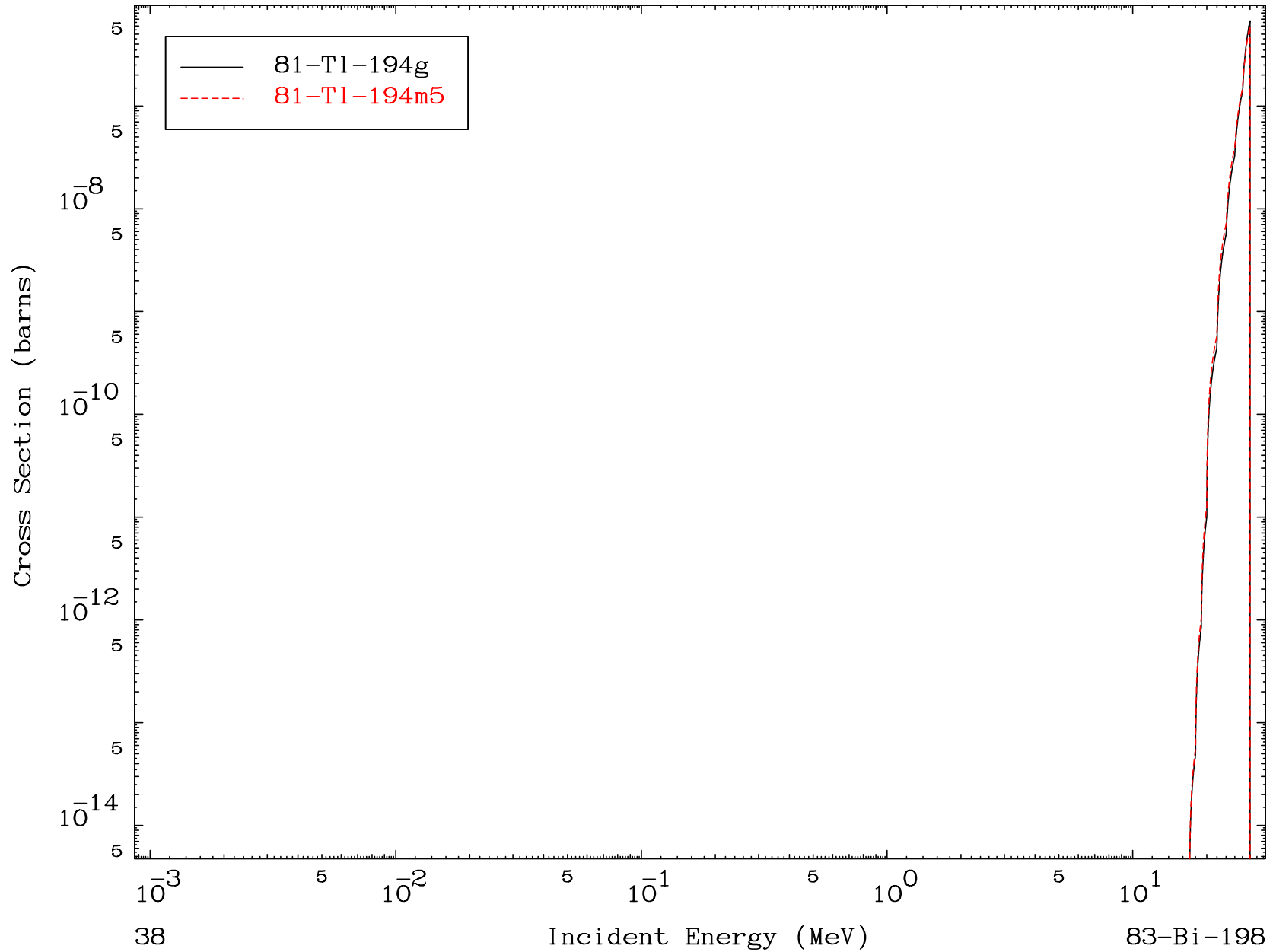


MAT 8293

( $\alpha, 2\alpha$ )

83-Bi-198

Radionuclide Production Cross Section

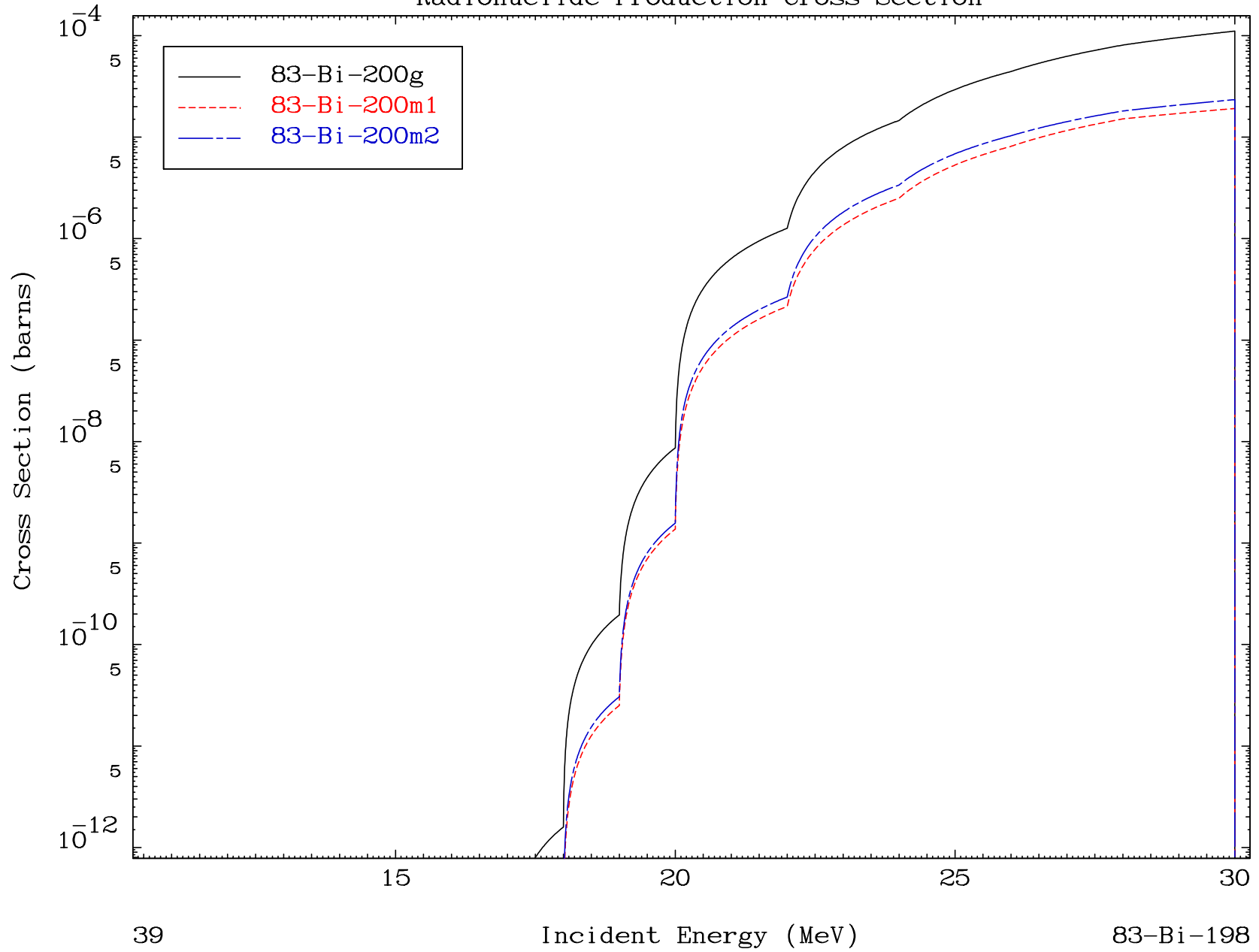


MAT 8293

( $\alpha, 2p$ )

83-Bi-198

Radionuclide Production Cross Section

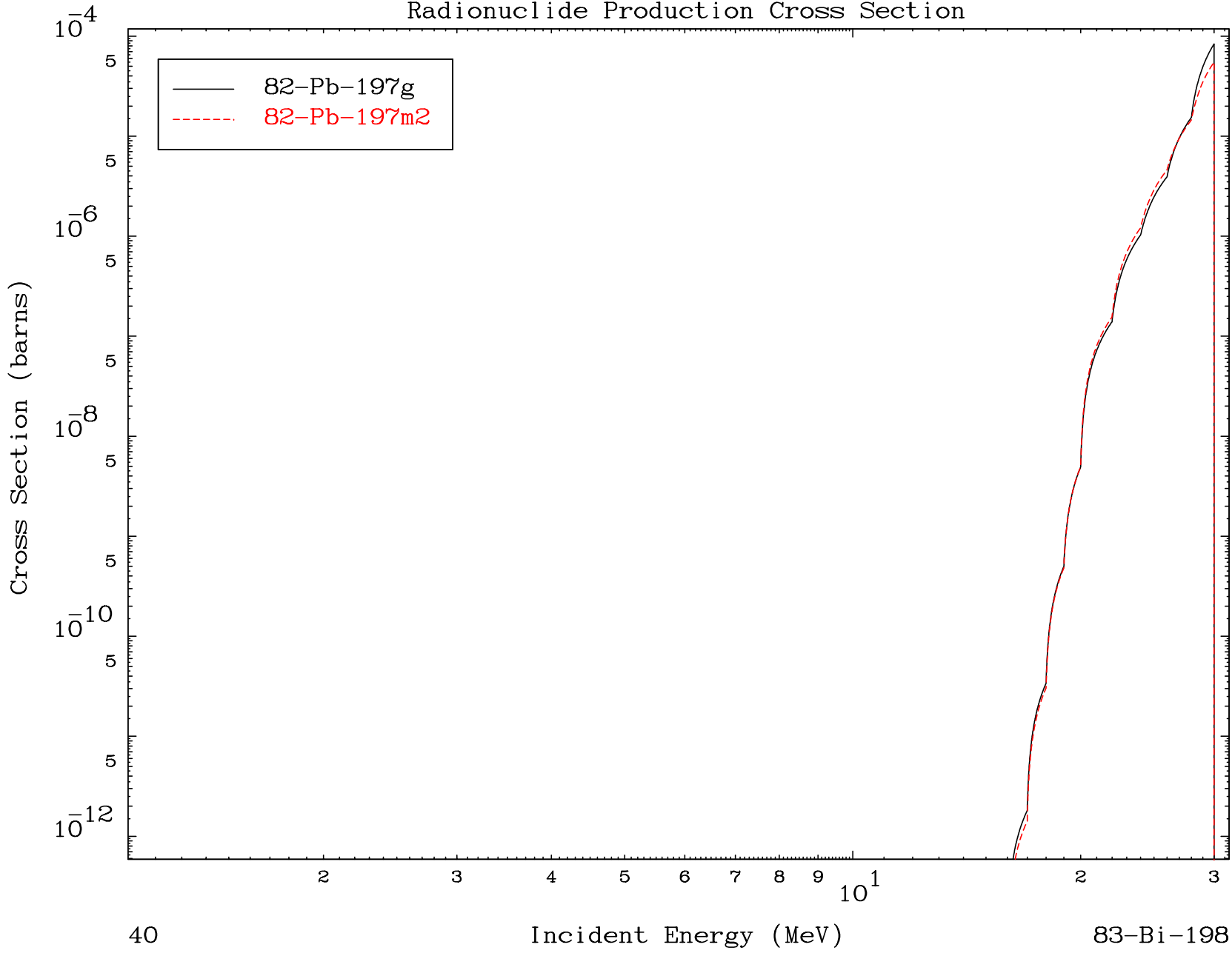


MAT 8293

( $\alpha, p$ )  $\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

83-Bi-198



MAT 8293

( $\alpha, p$ ) d

83-Bi-198

Radionuclide Production Cross Section

