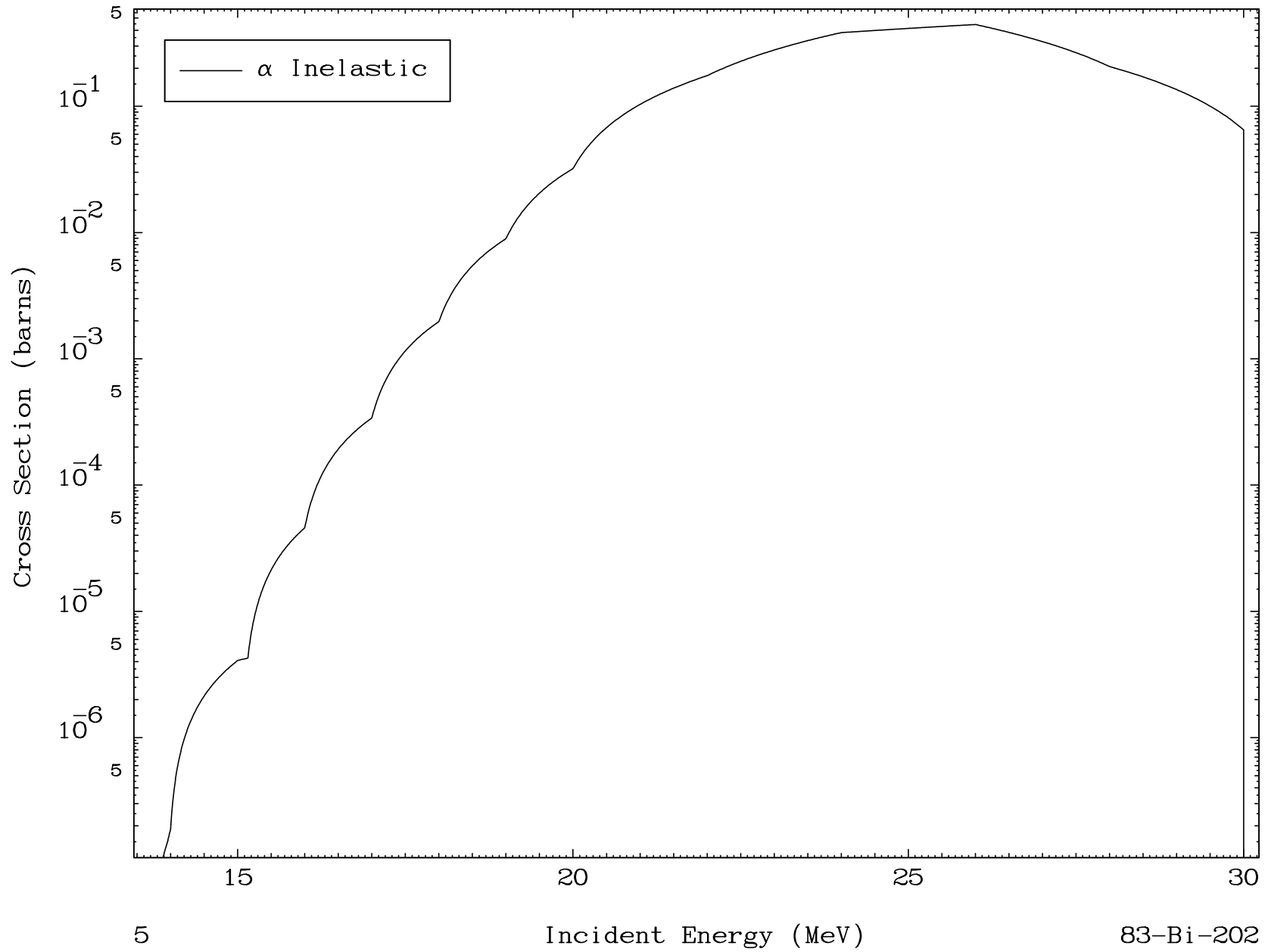


MAT 8304

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

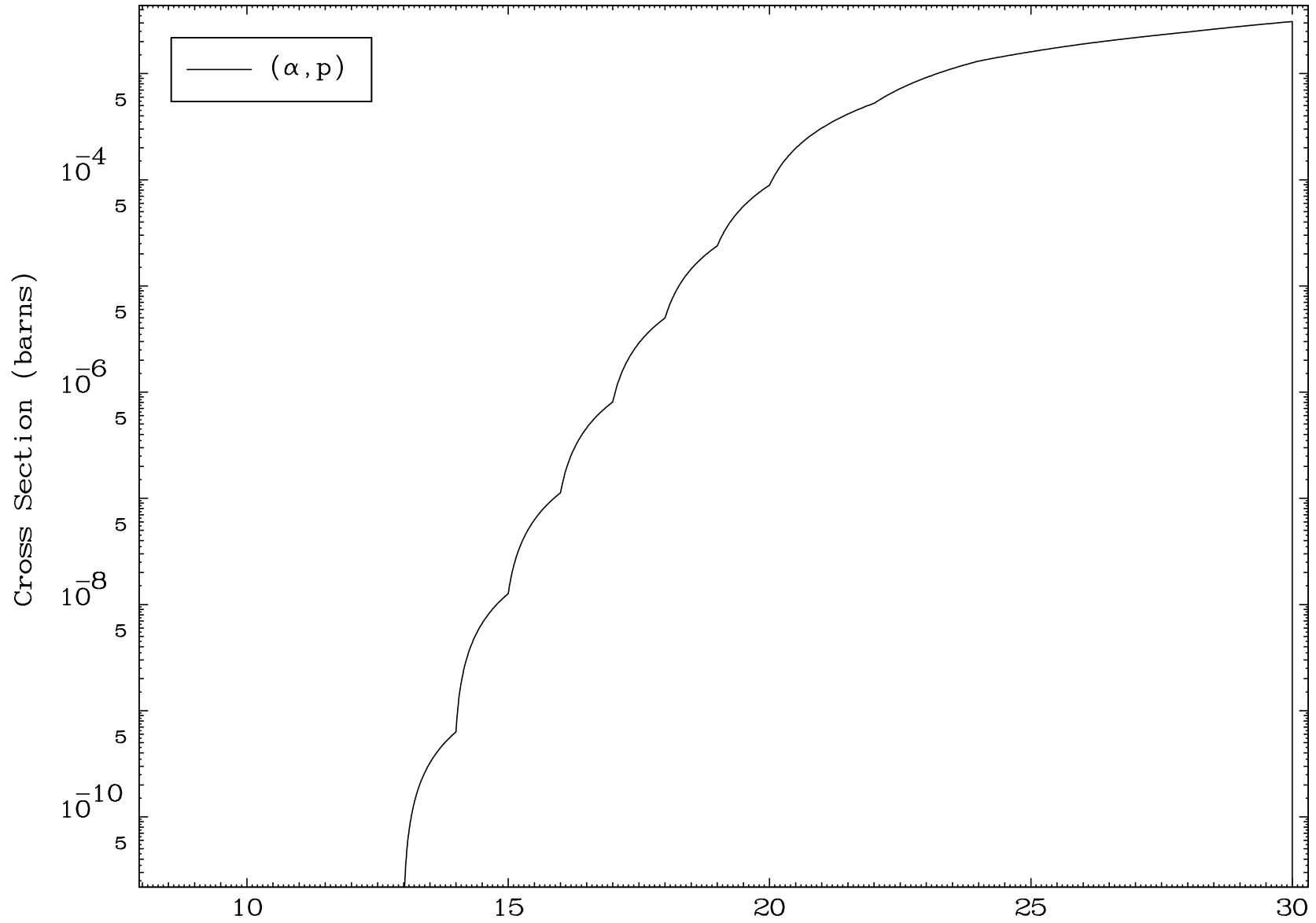
83-Bi-202

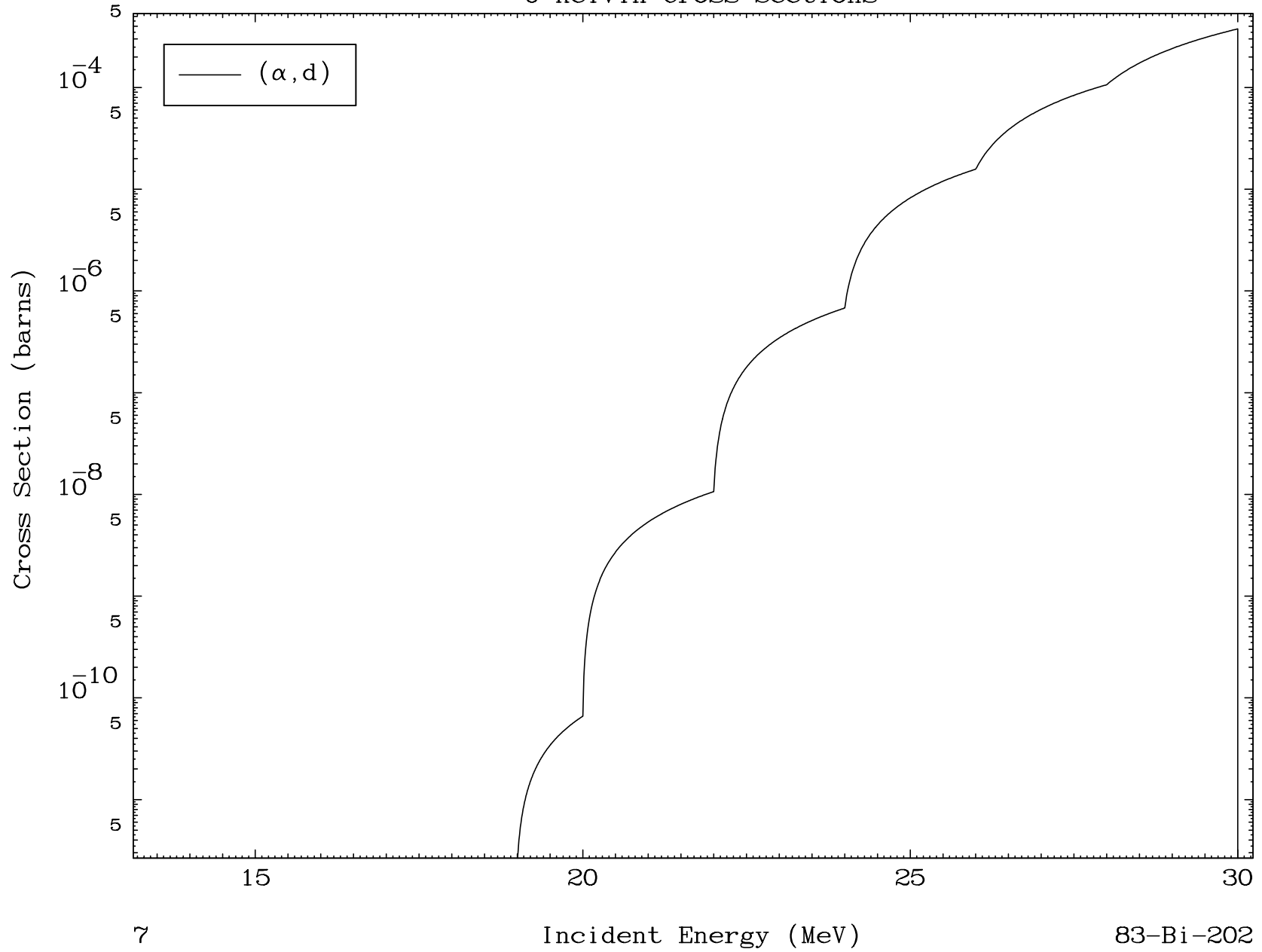


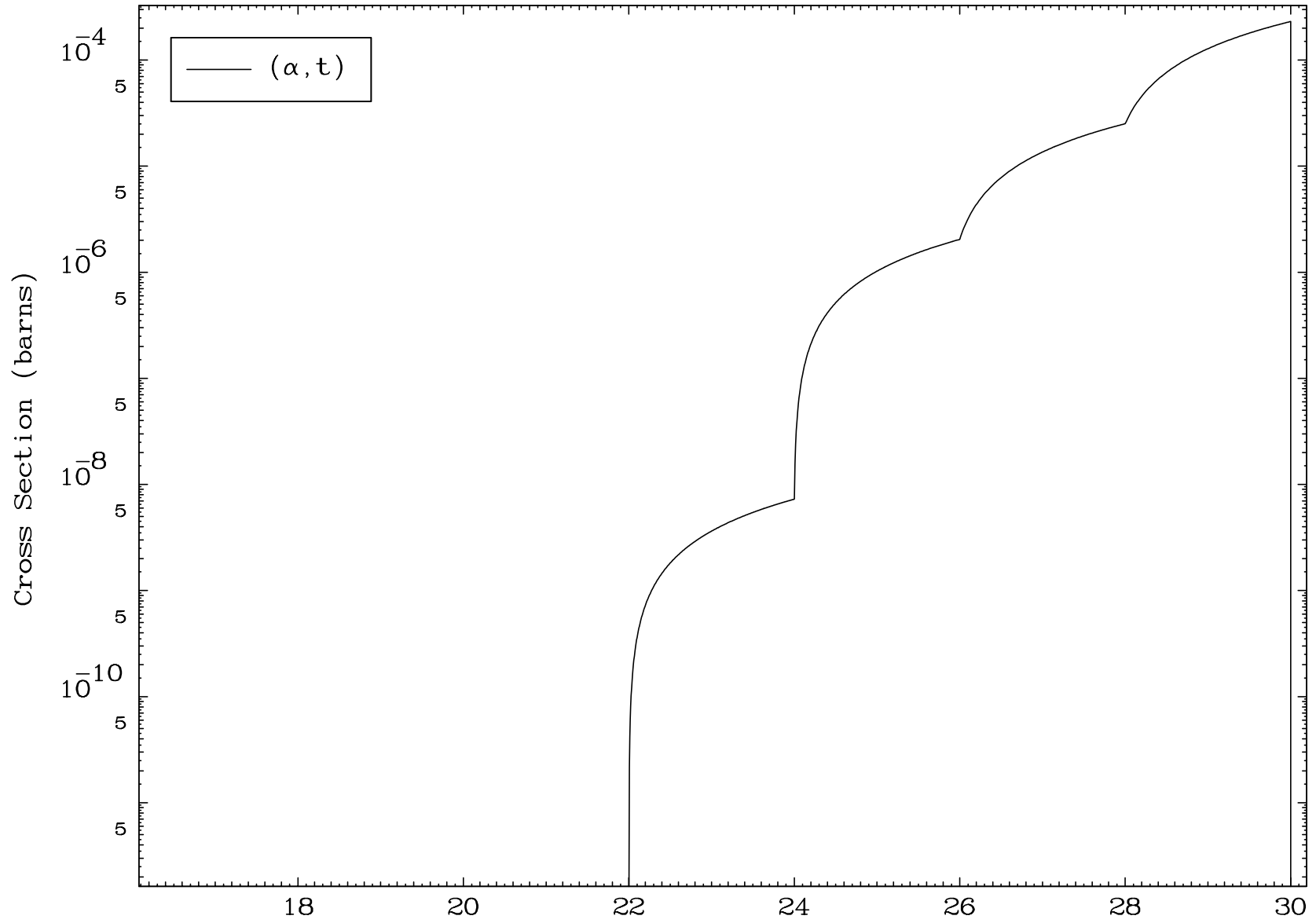
5

Incident Energy (MeV)

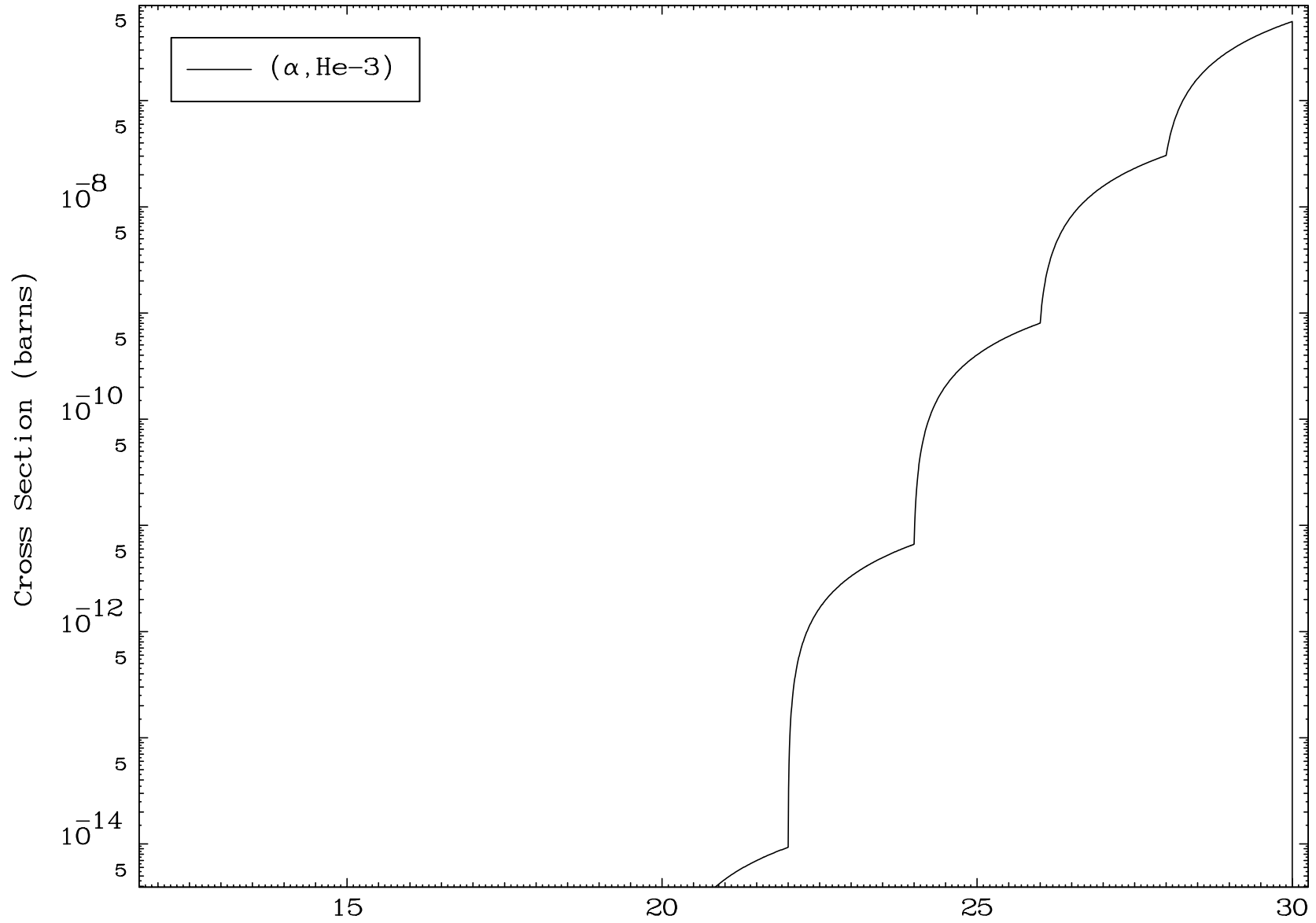
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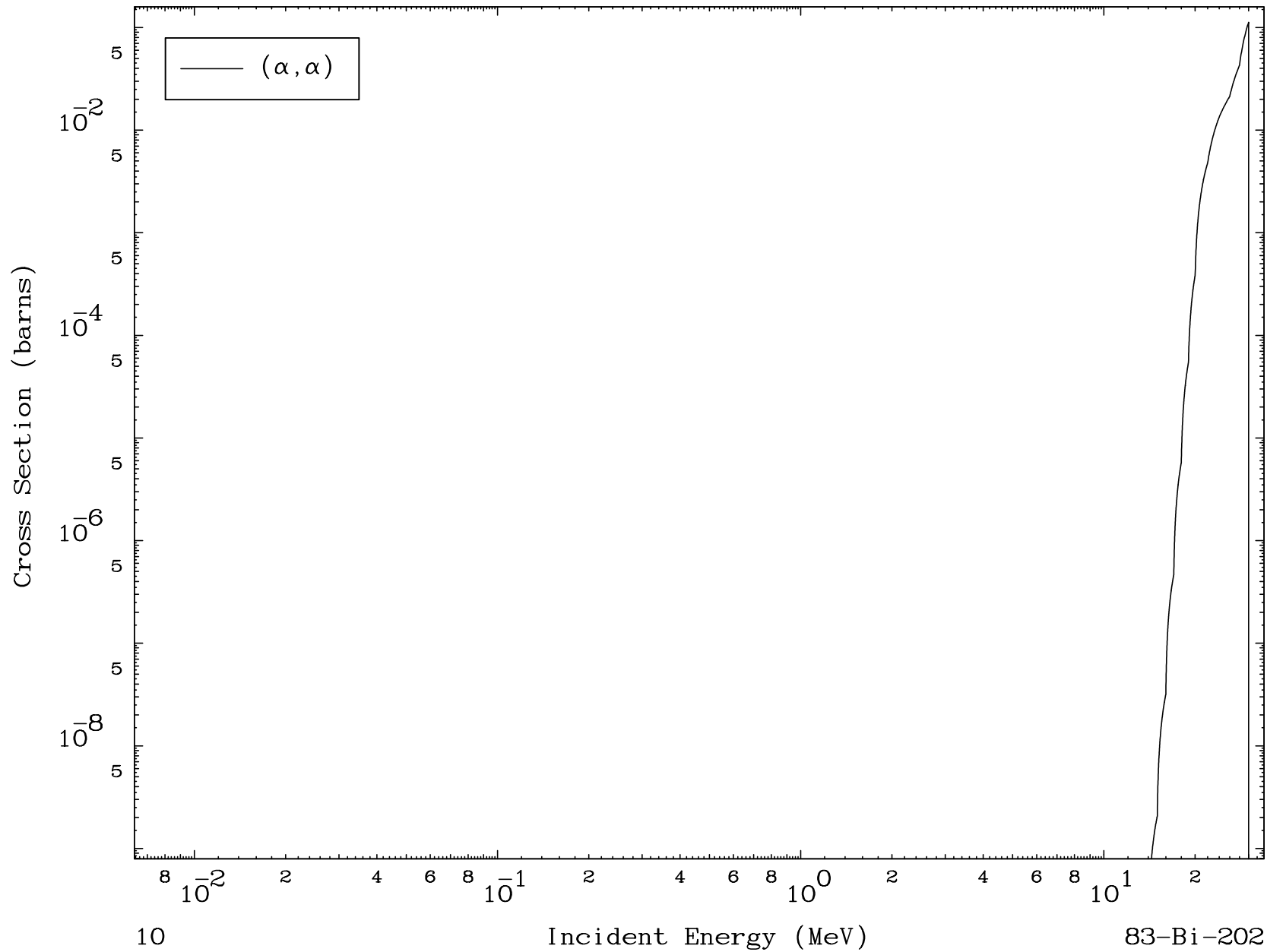




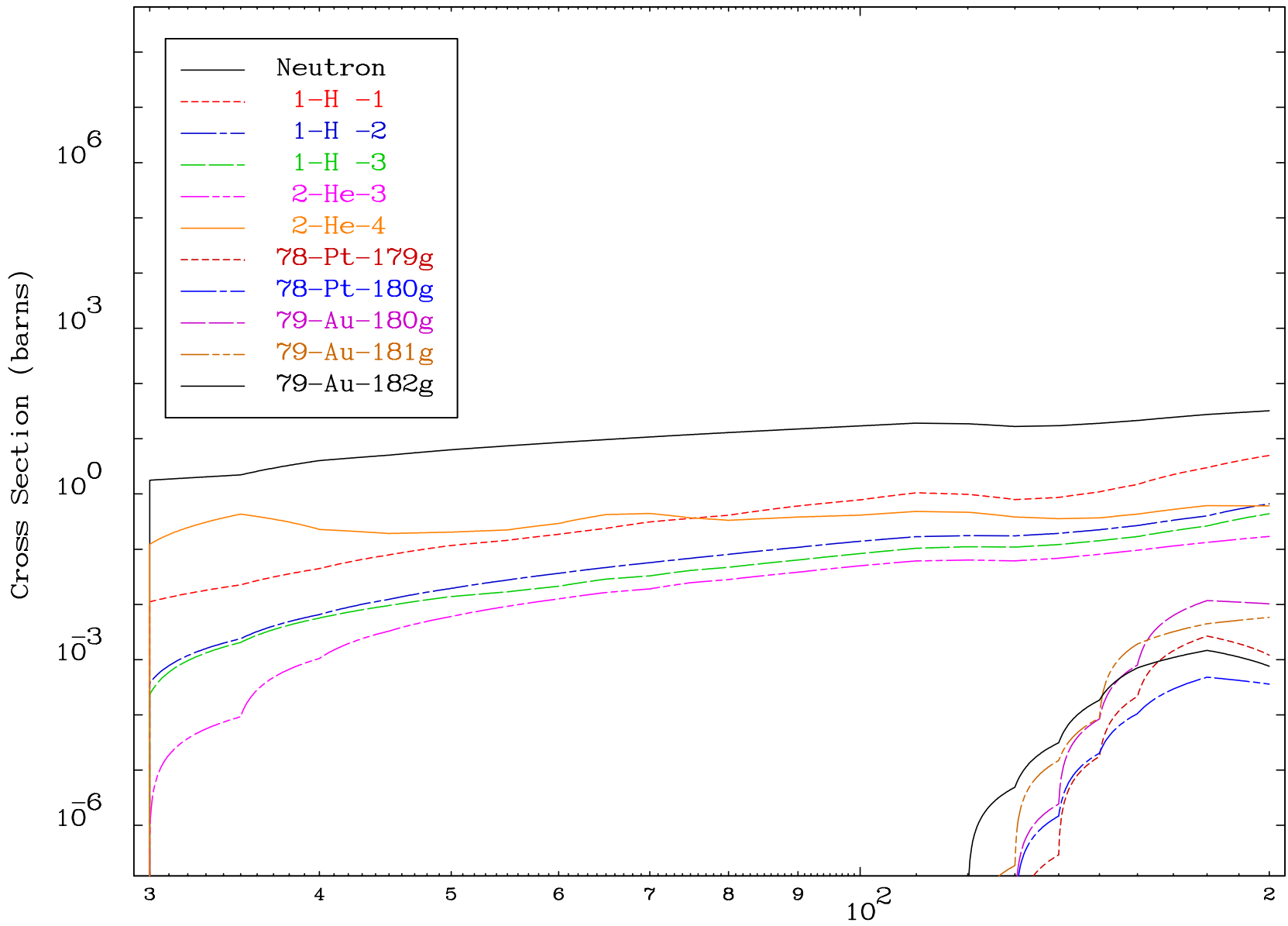


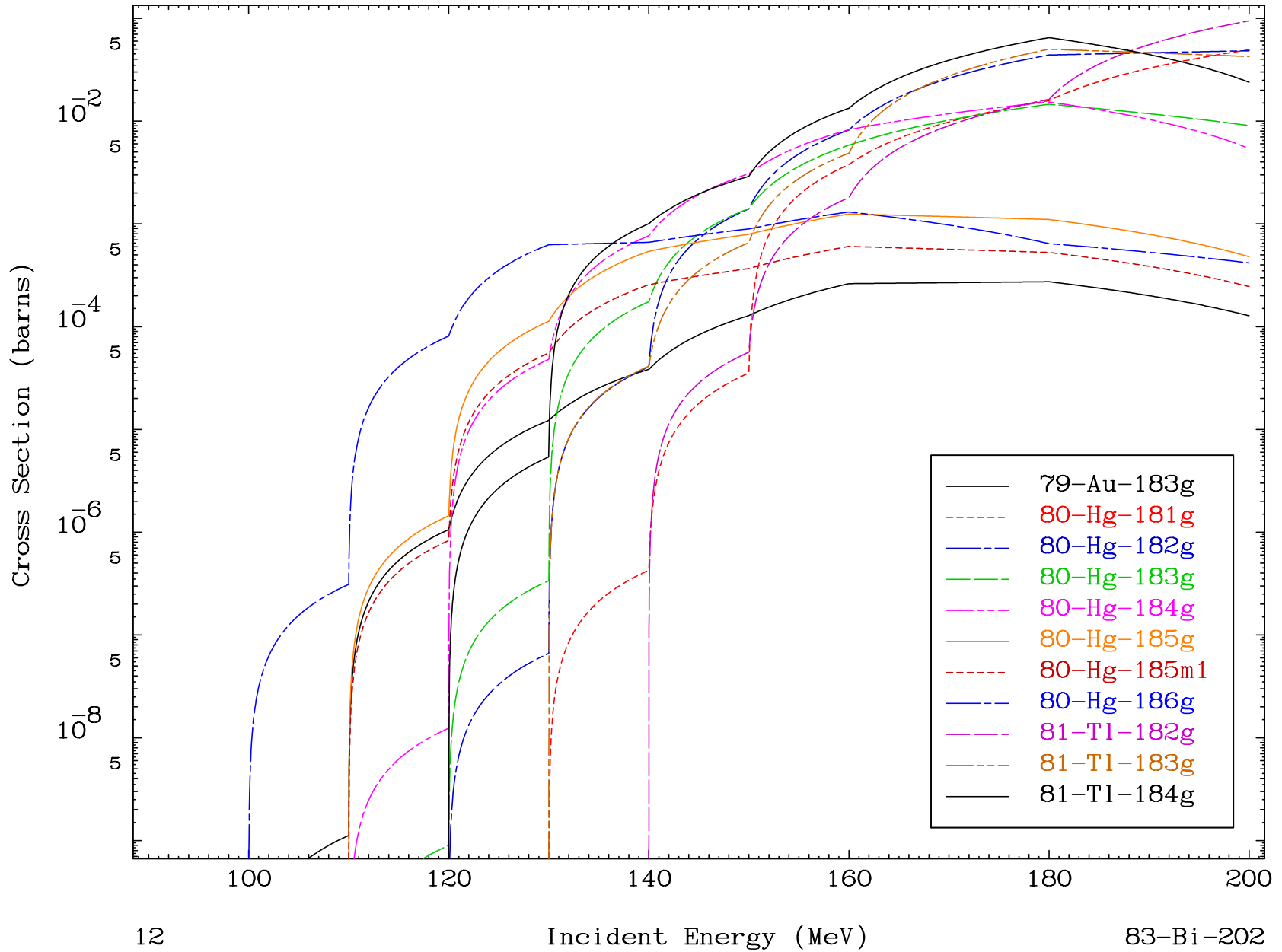


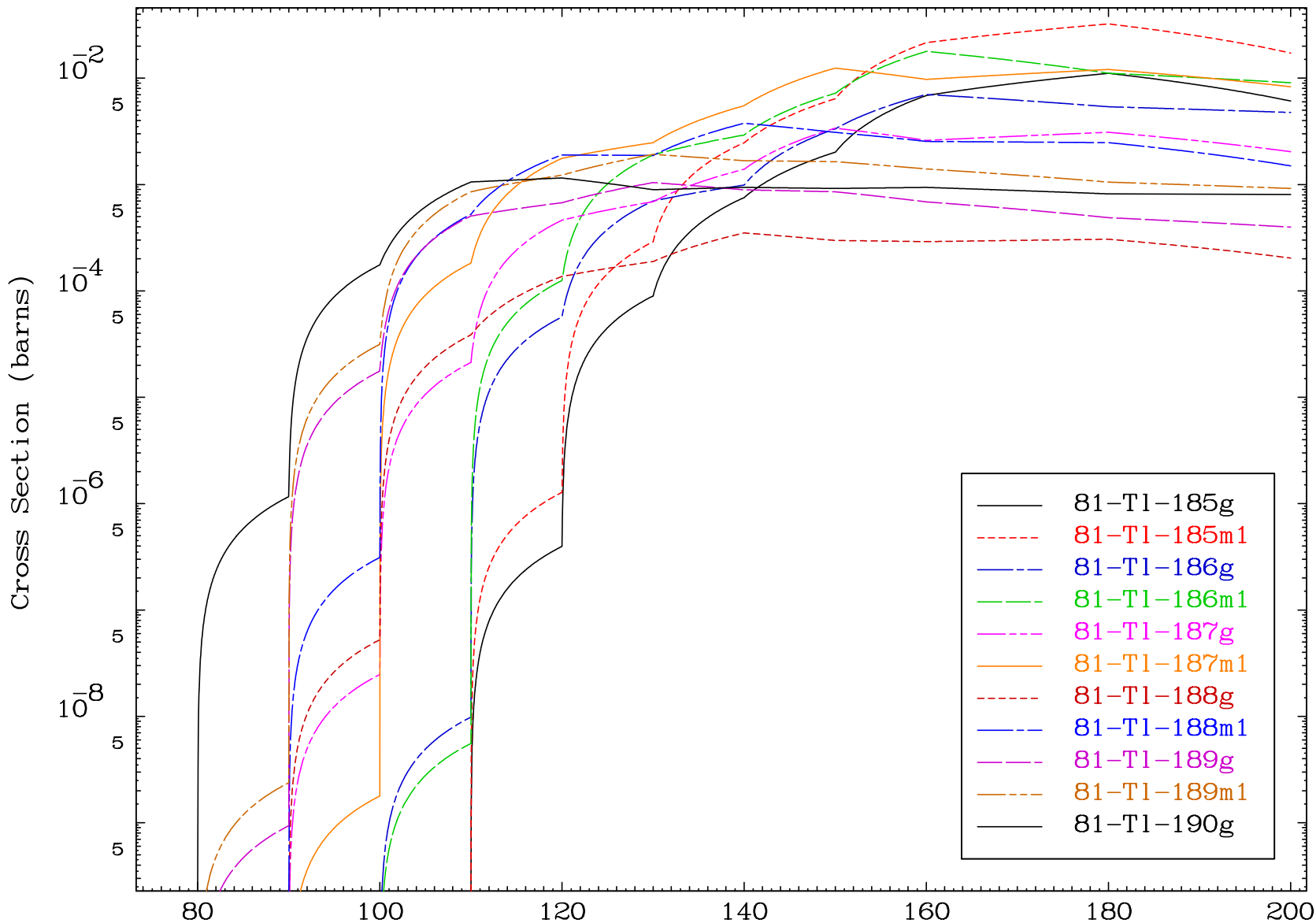


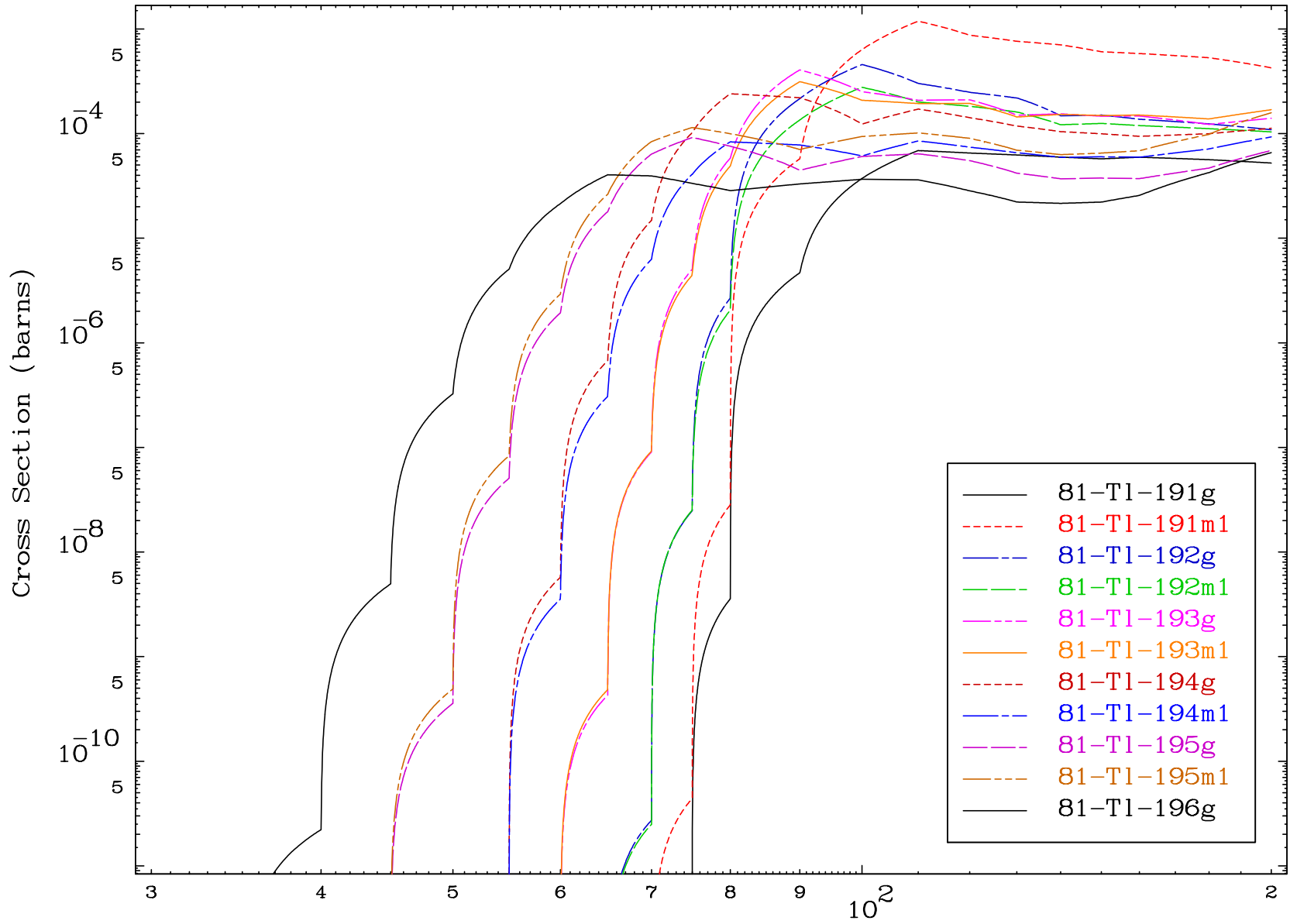


Radionuclide Production Cross Section

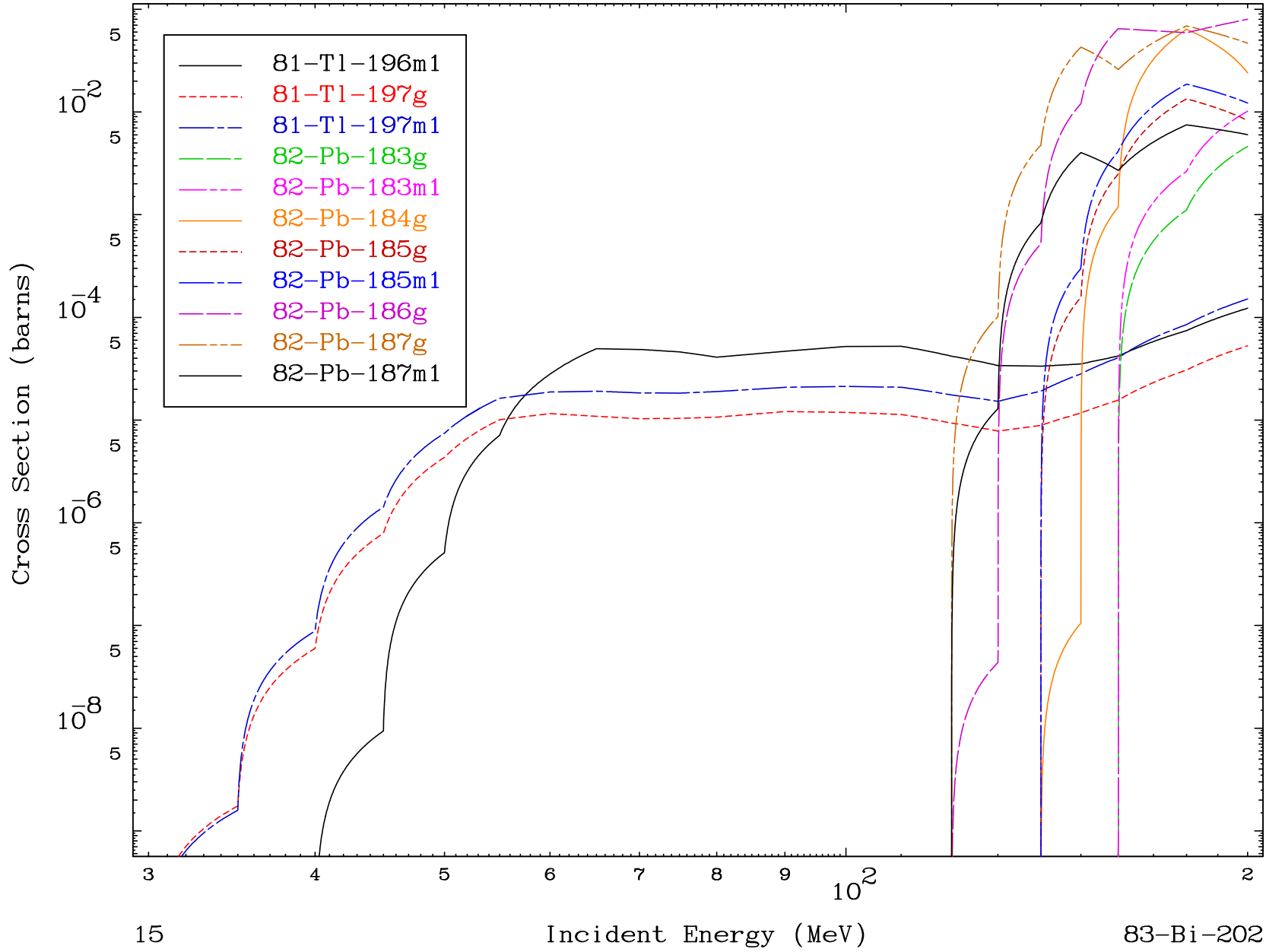


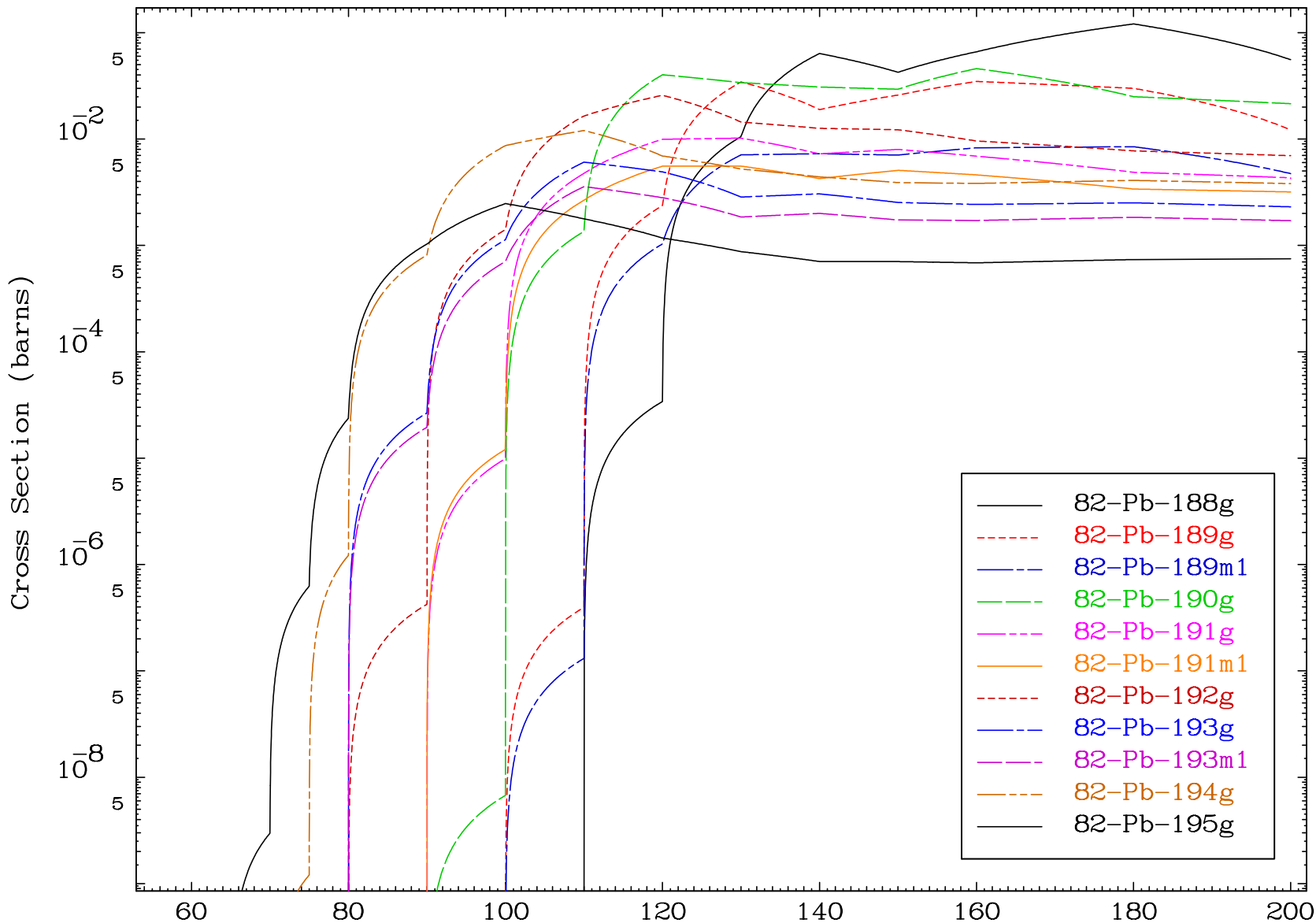




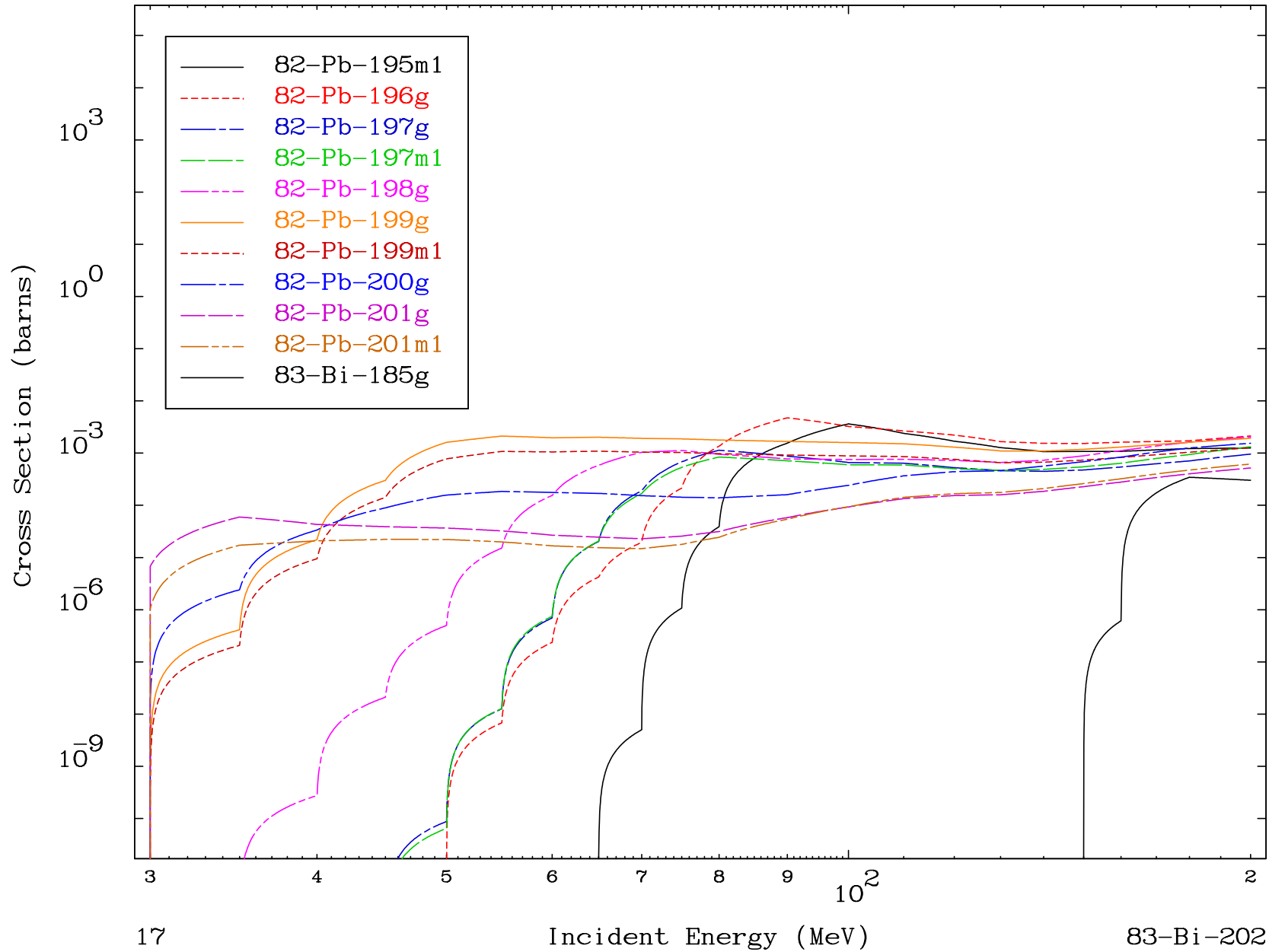


Radionuclide Production Cross Section

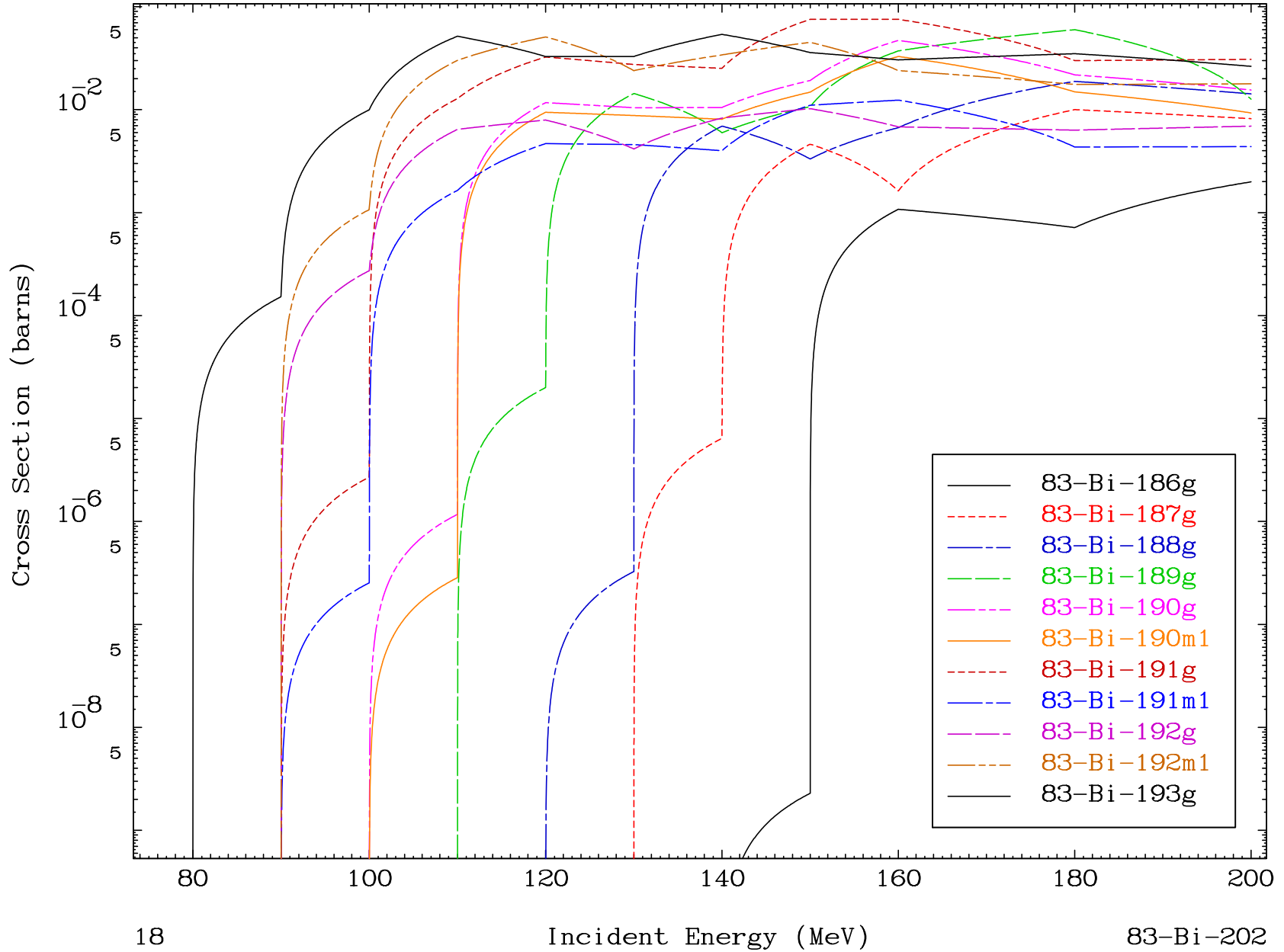


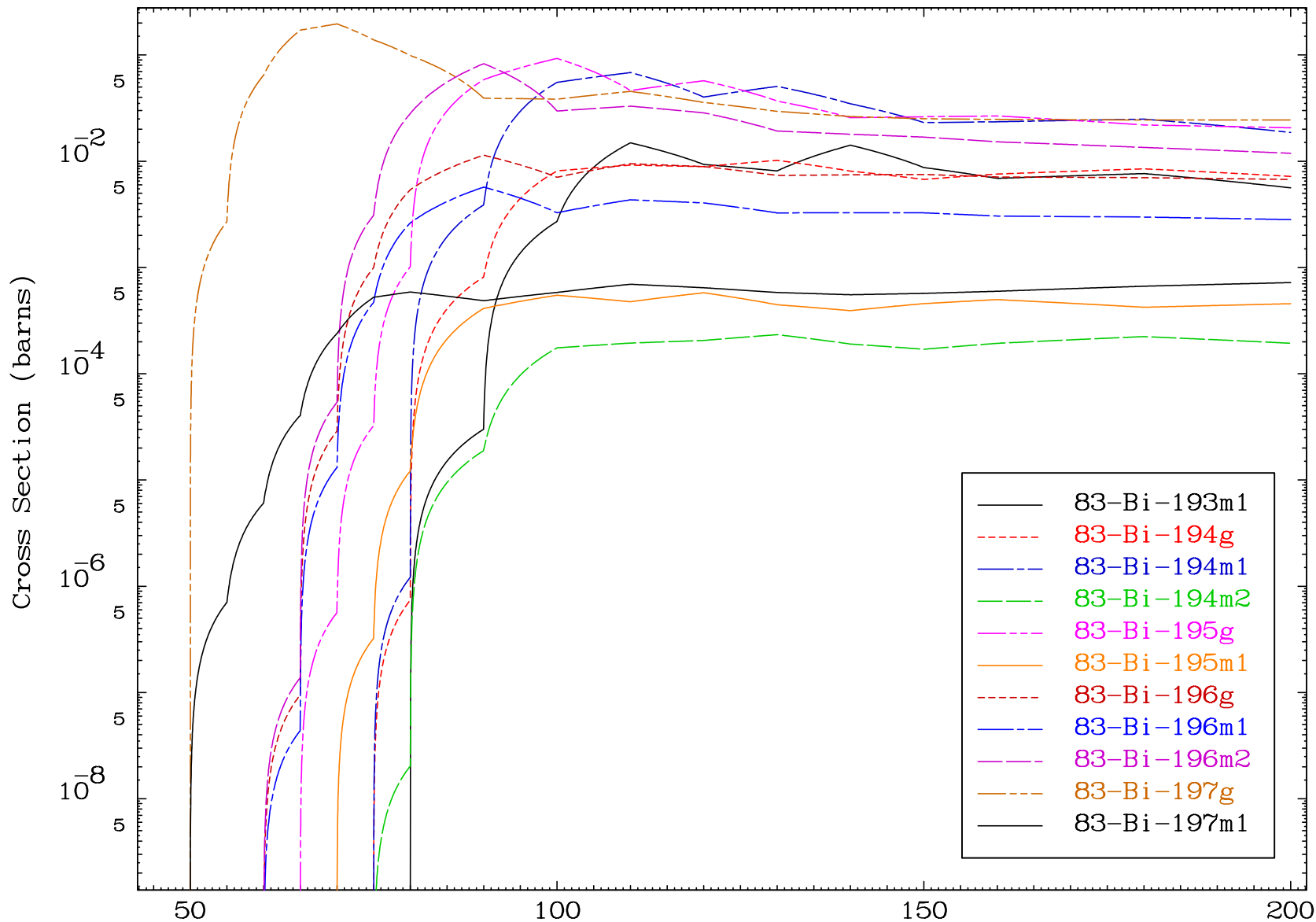


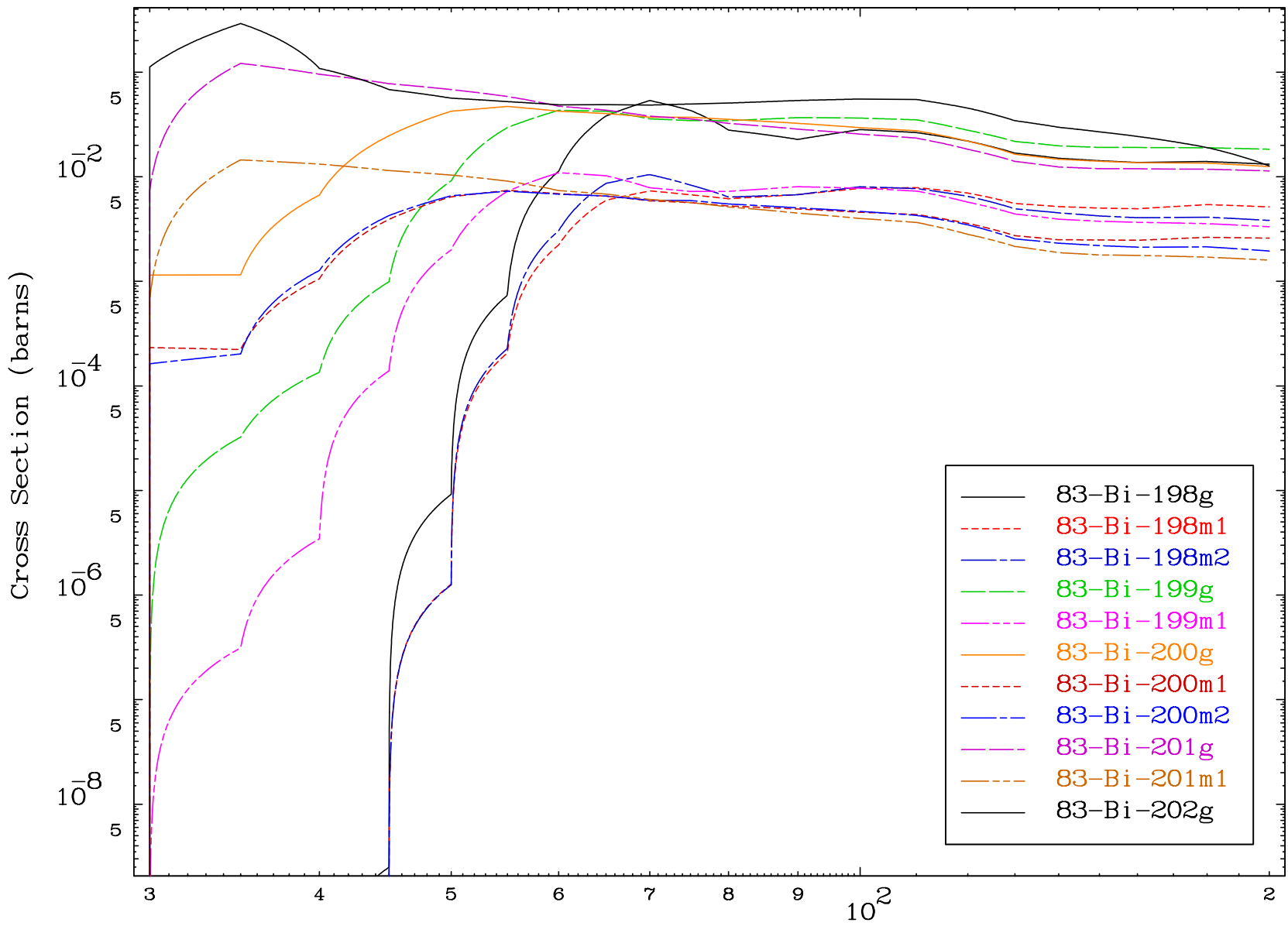




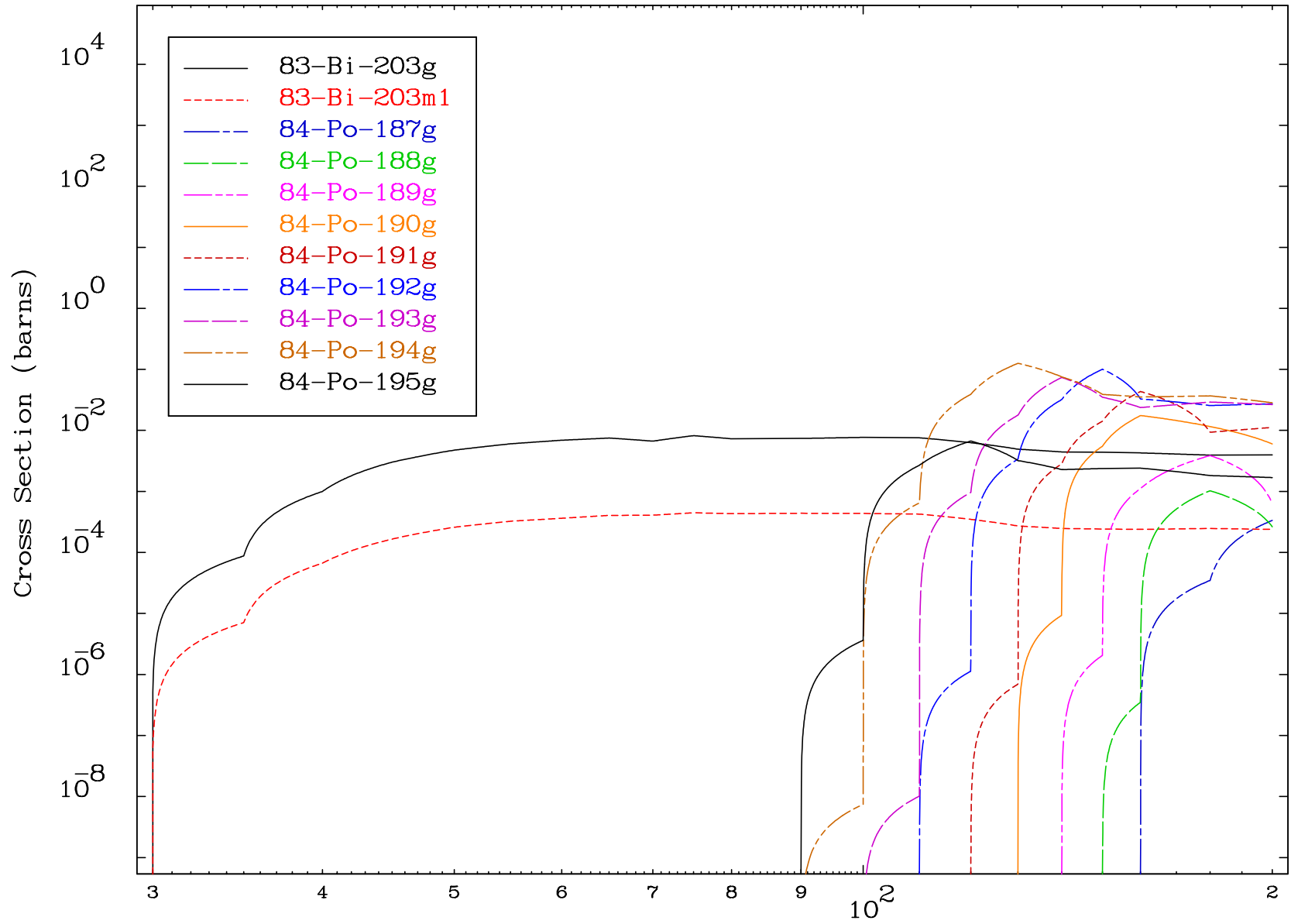
Radionuclide Production Cross Section



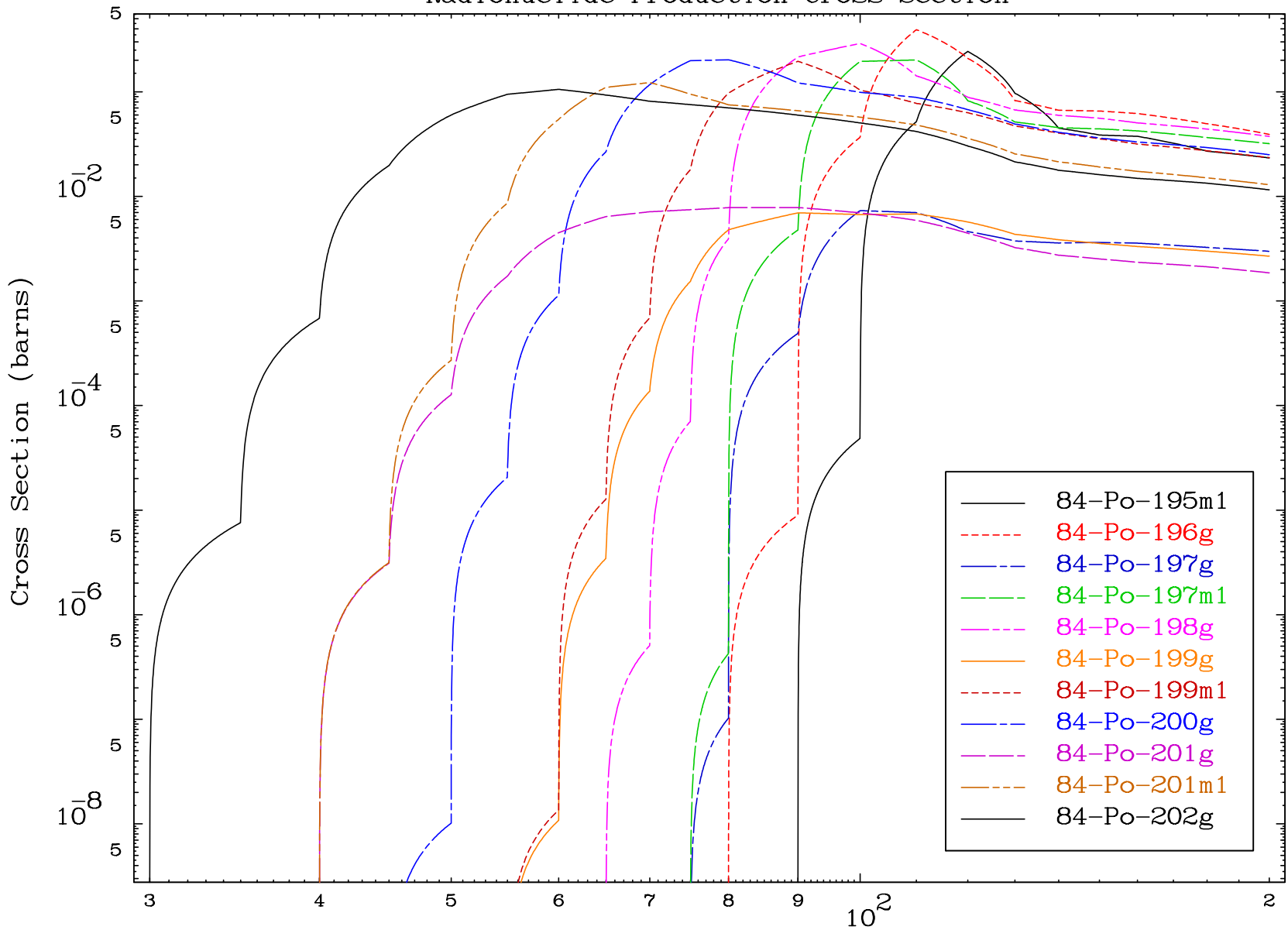




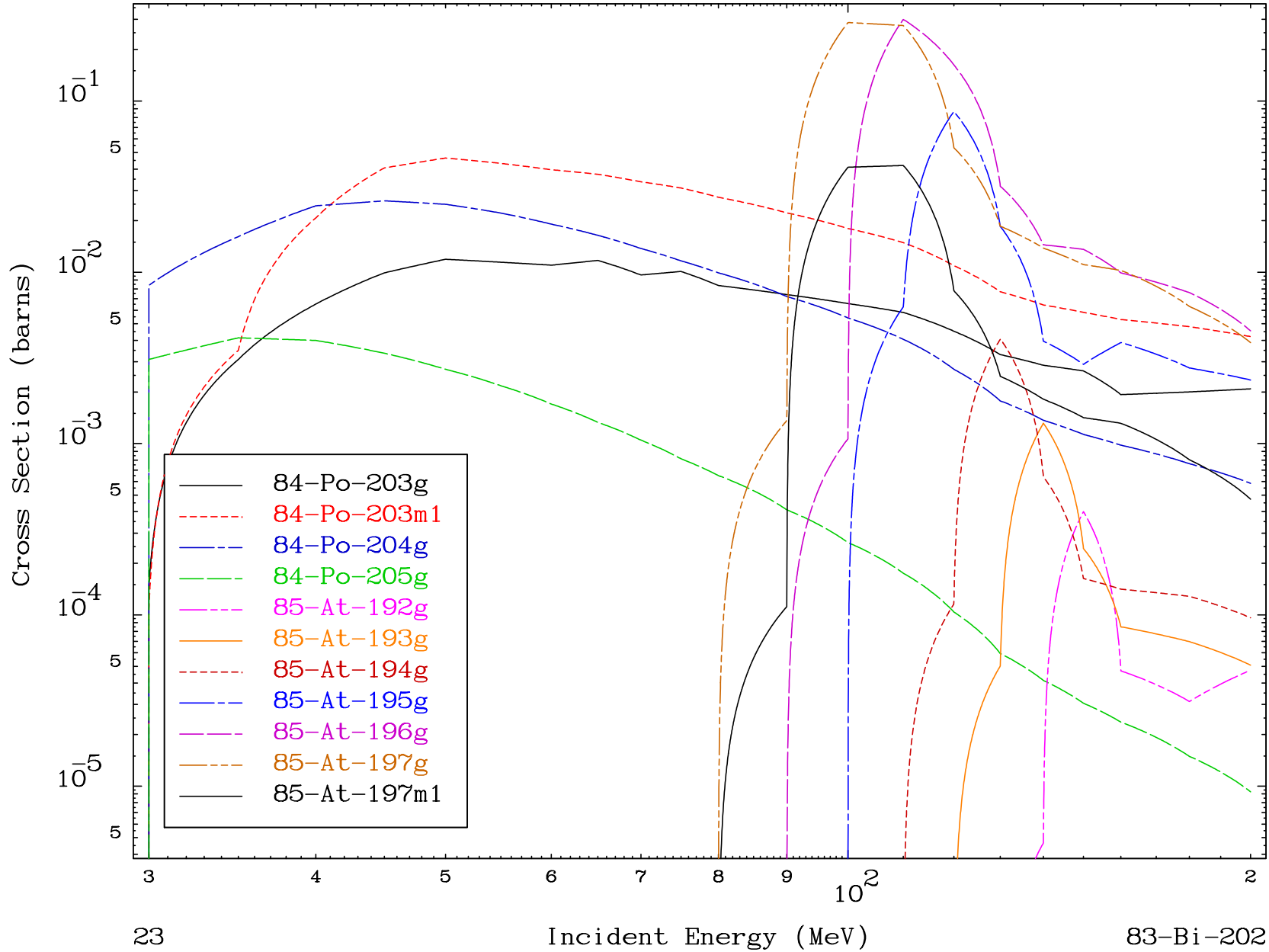
Radionuclide Production Cross Section

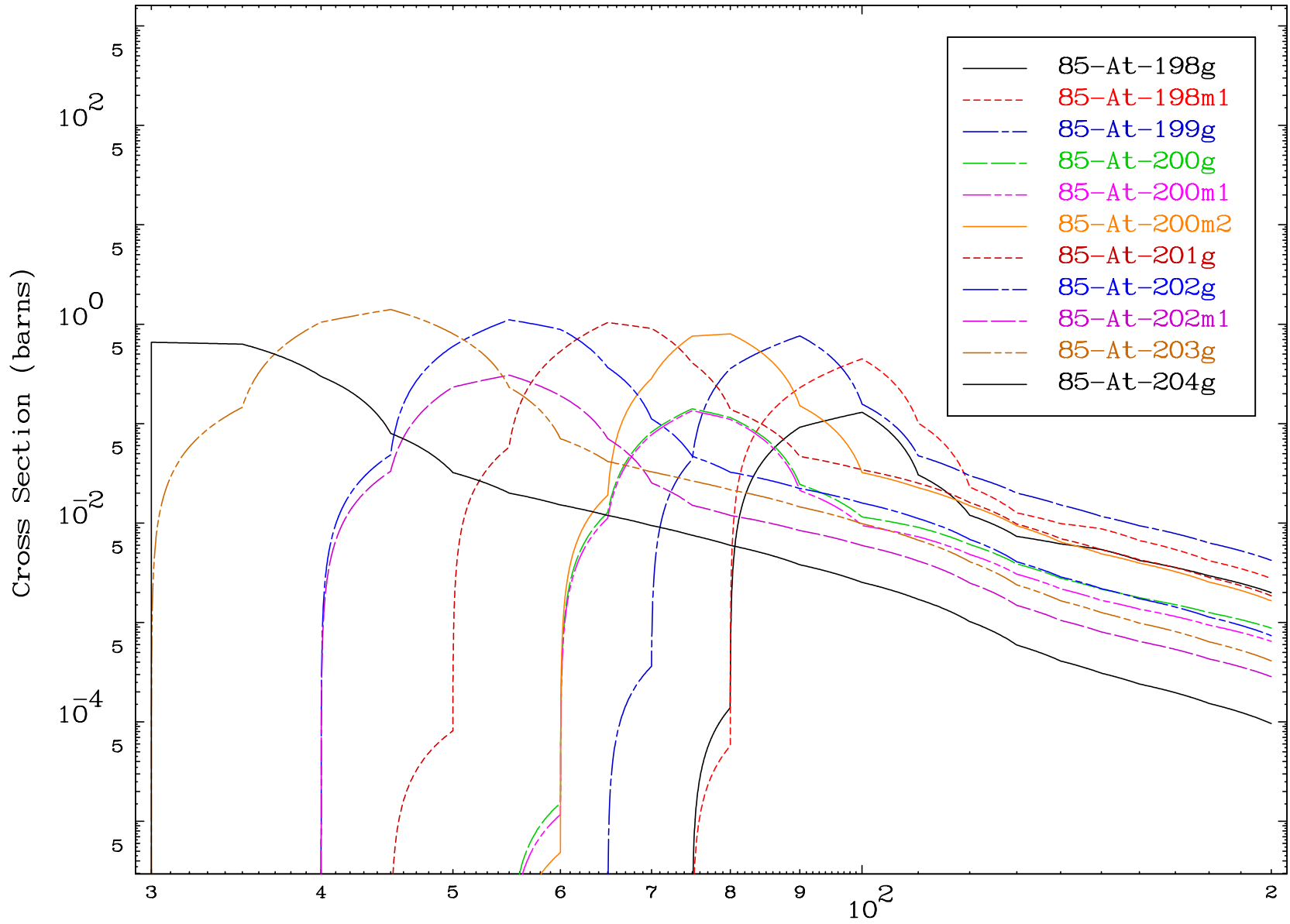


Radionuclide Production Cross Section

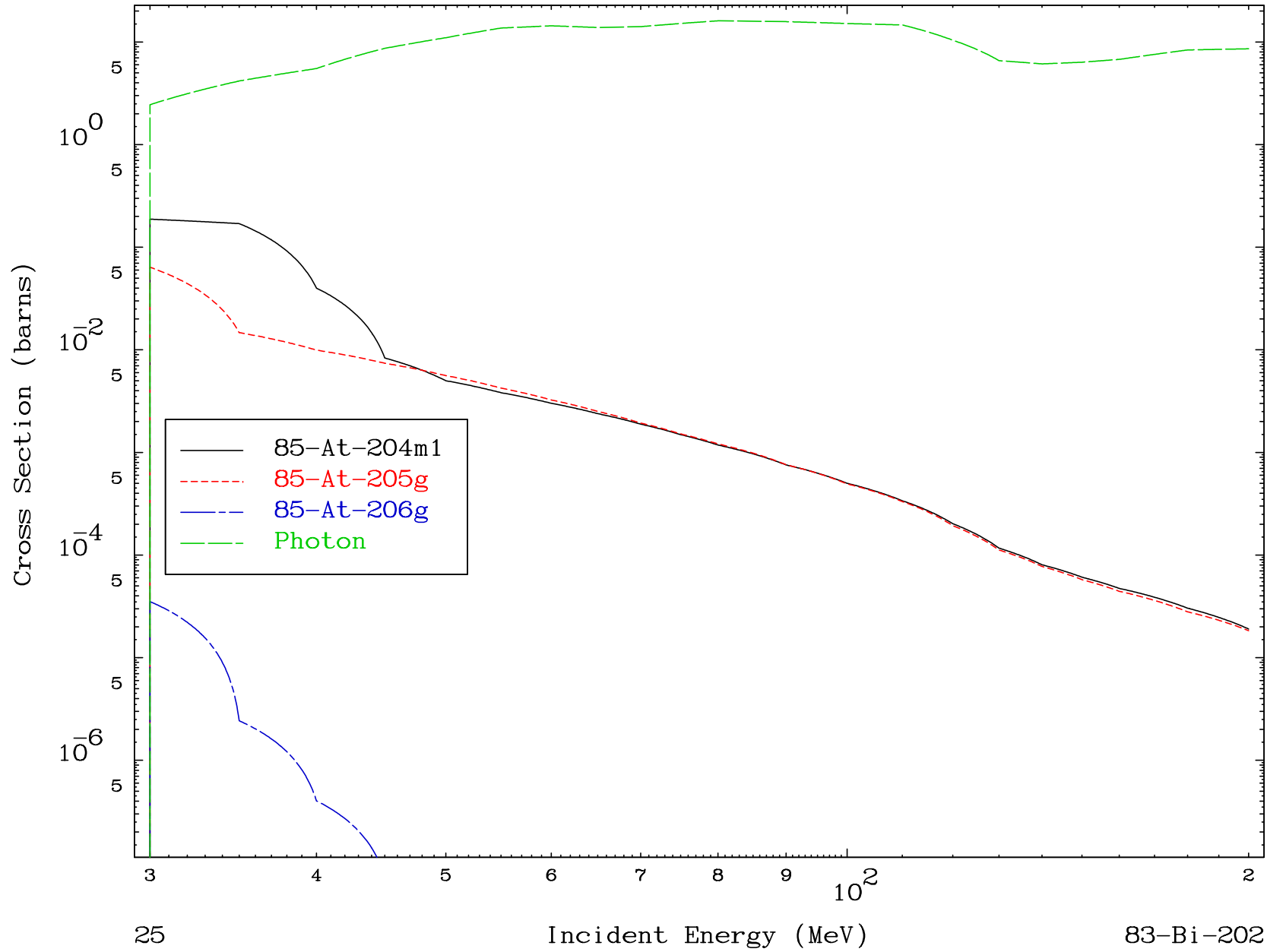


Radionuclide Production Cross Section







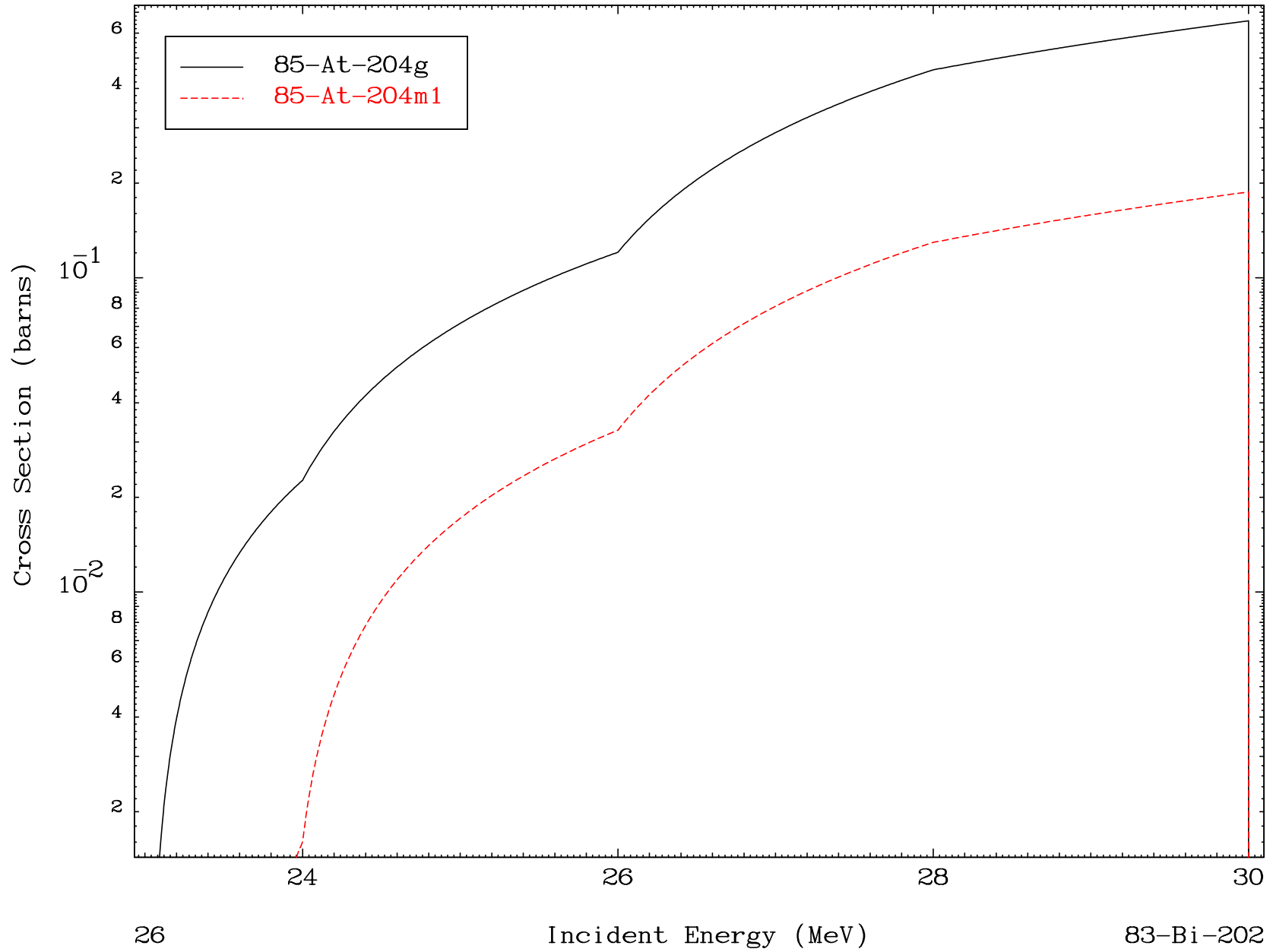


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( $\alpha, 2n$ )

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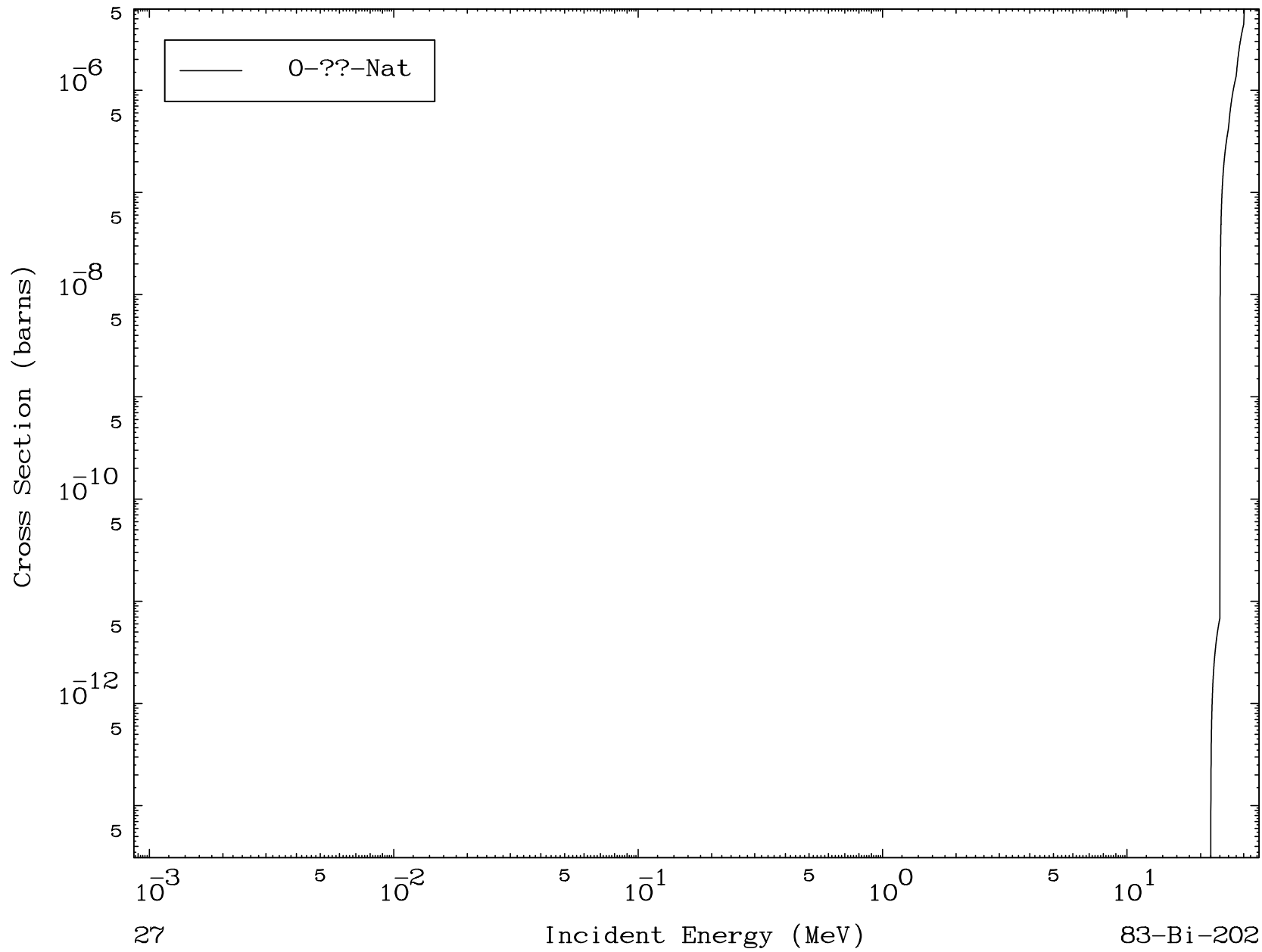
Radionuclide Production Cross Section



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$\alpha$  Fission  
Radionuclide Production Cross Section

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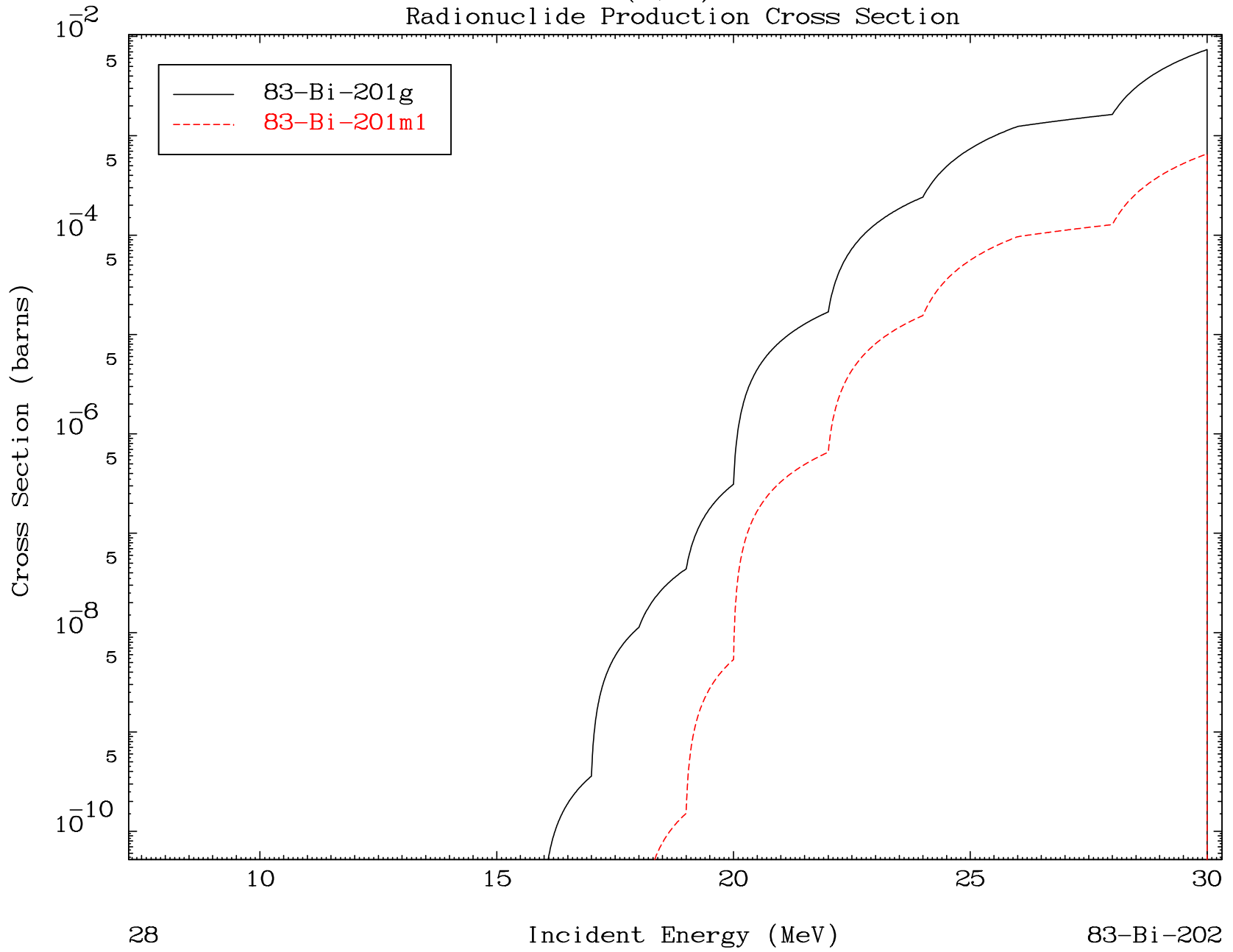


MAT 8304

( $\alpha, n'$ )  $\alpha$

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Radionuclide Production Cross Section

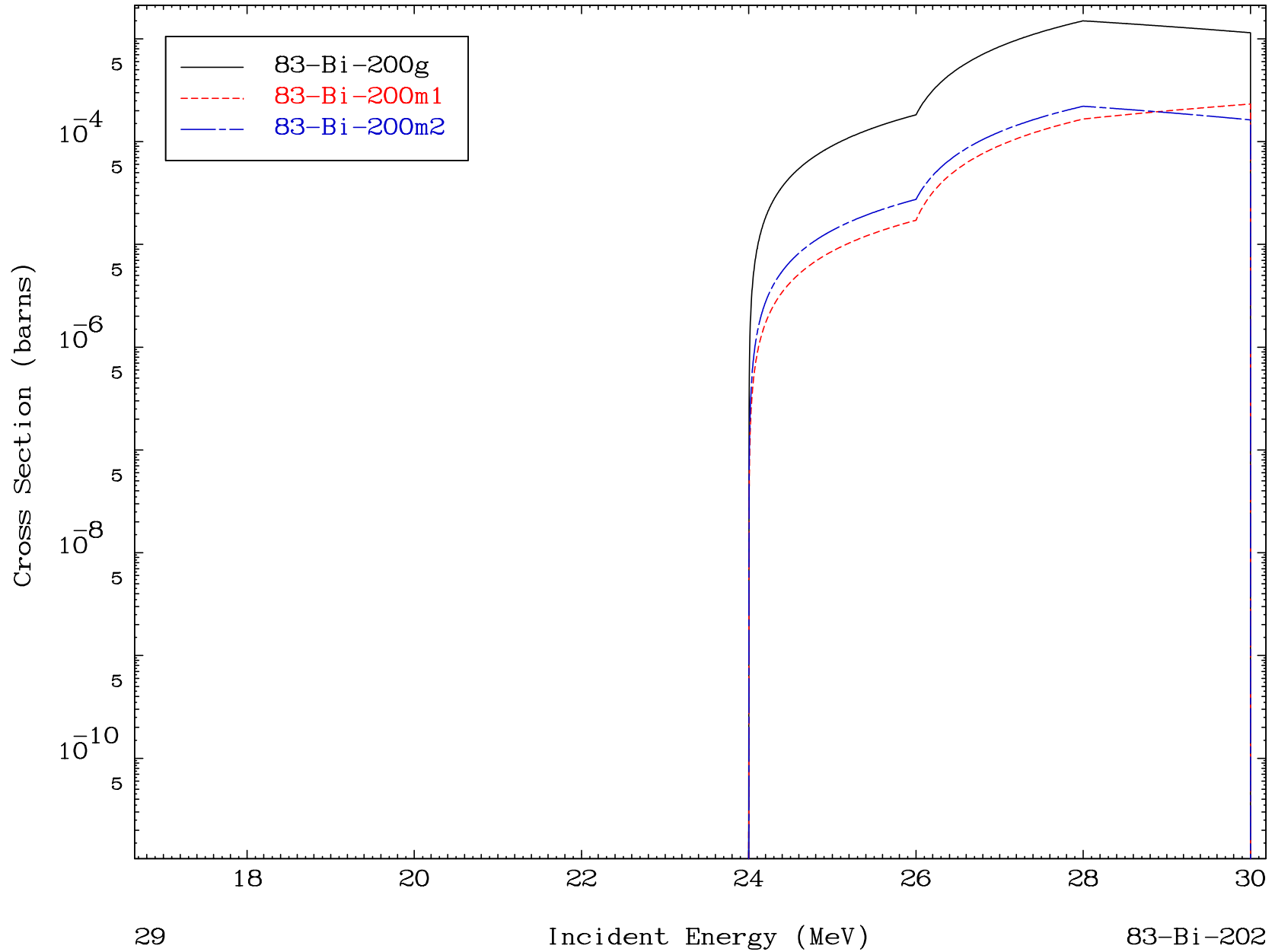


MAT 8304

( $\alpha, 2n$ )  $\alpha$

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Radionuclide Production Cross Section

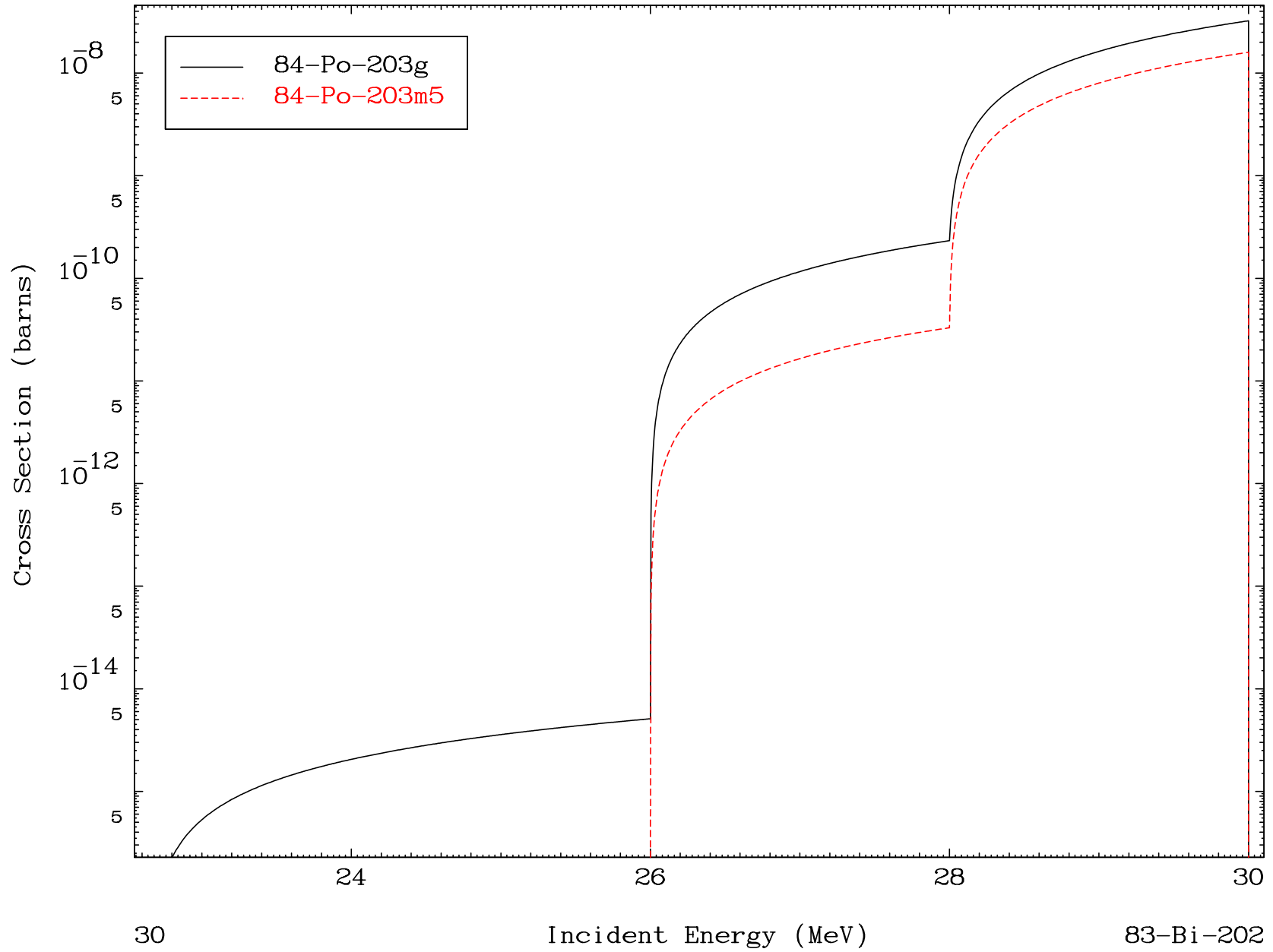


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( $\alpha, n'$ ) d

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Radionuclide Production Cross Section

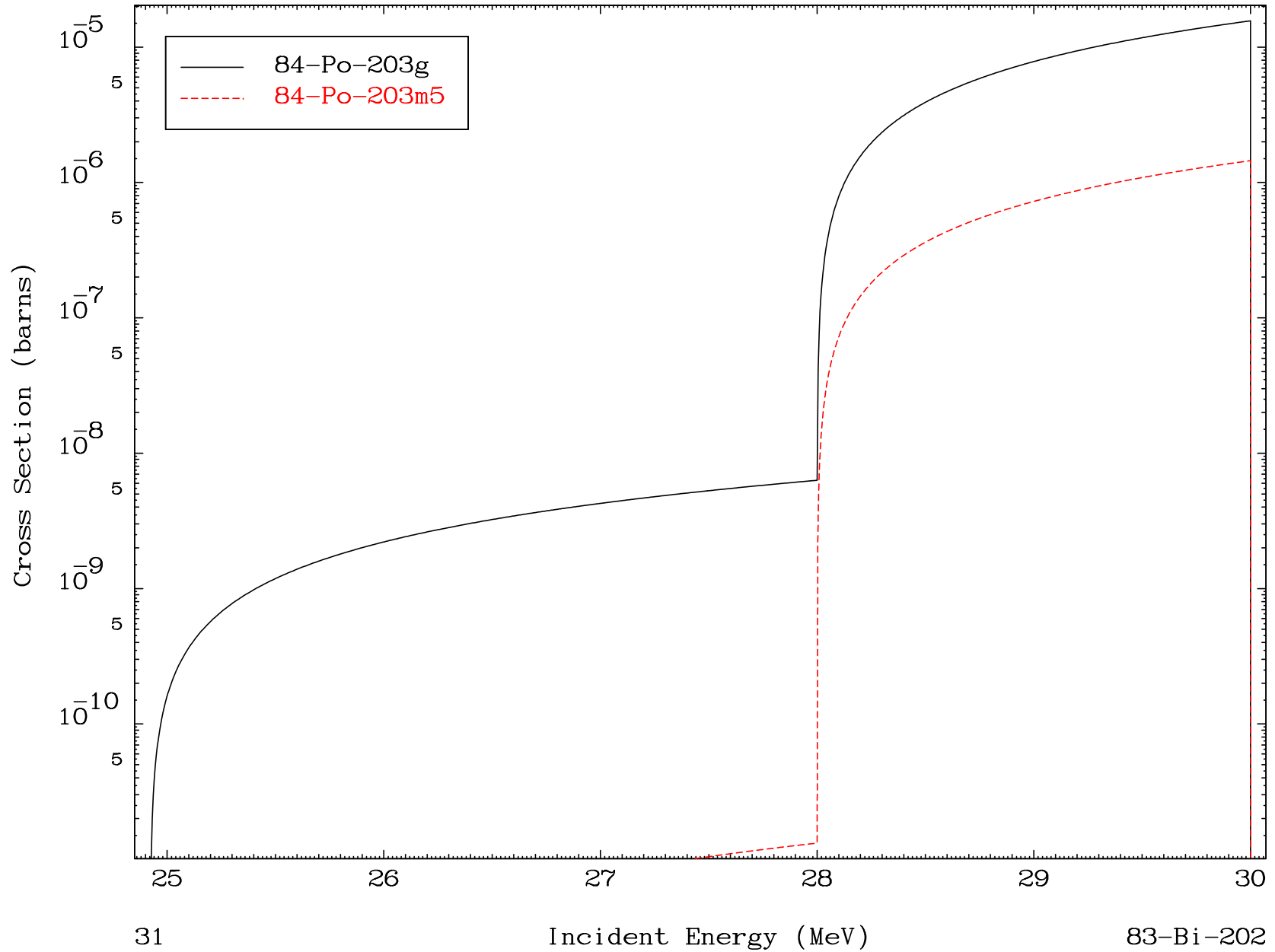


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( $\alpha, 2n$ ) p

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Radionuclide Production Cross Section

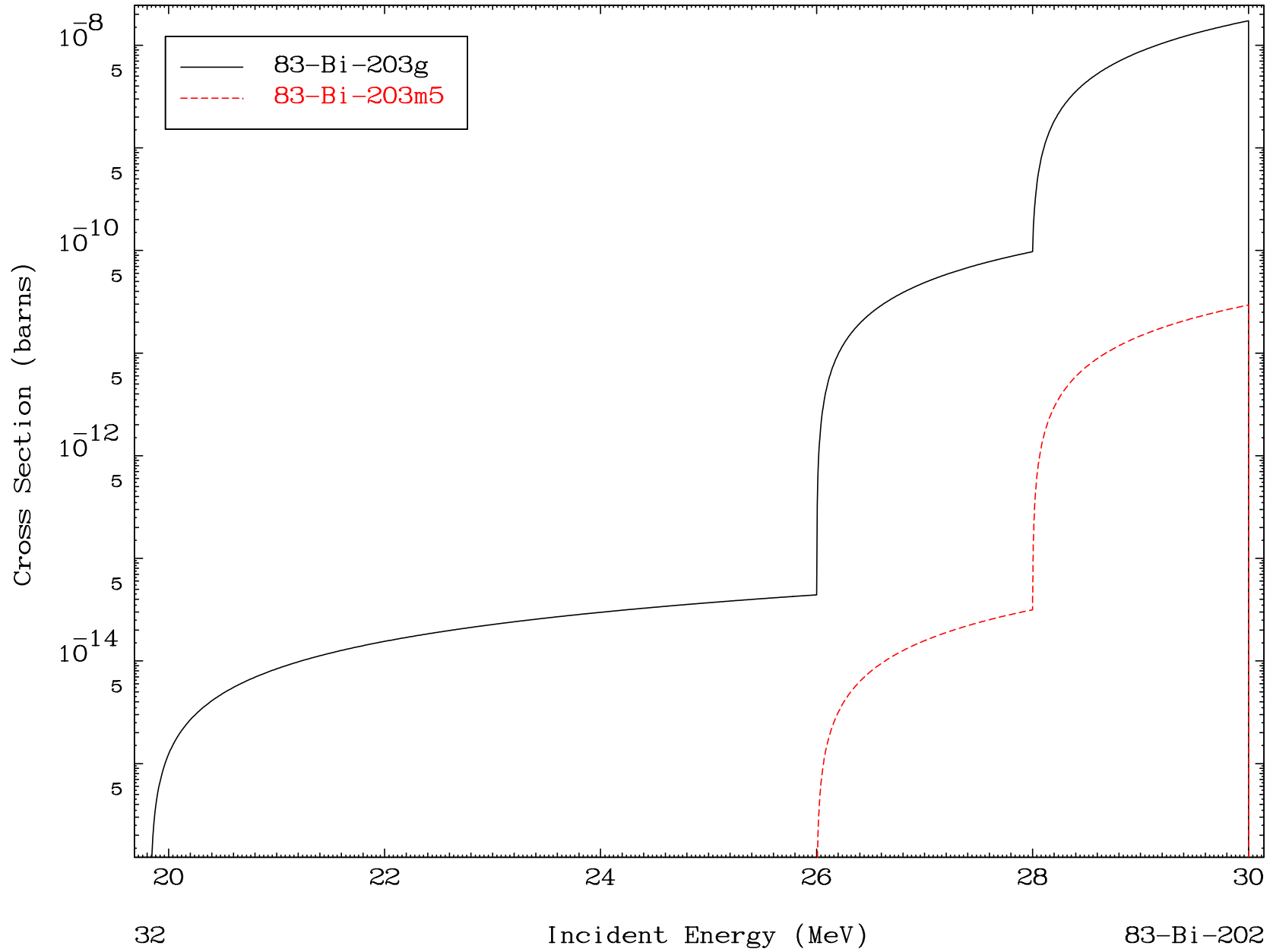


MAT 8304

( $\alpha, 2n$ ) p

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Radionuclide Production Cross Section



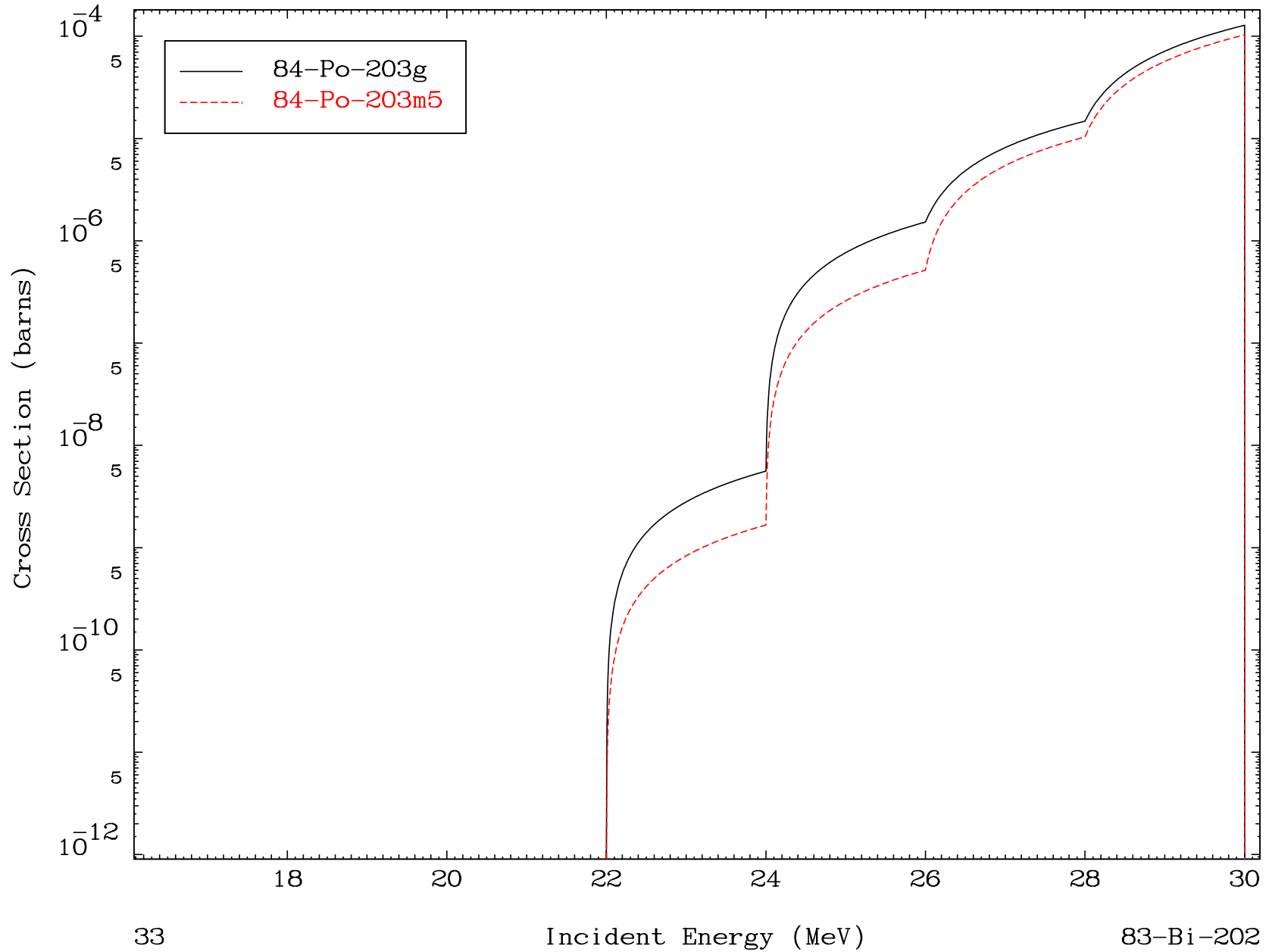


MAT 8304

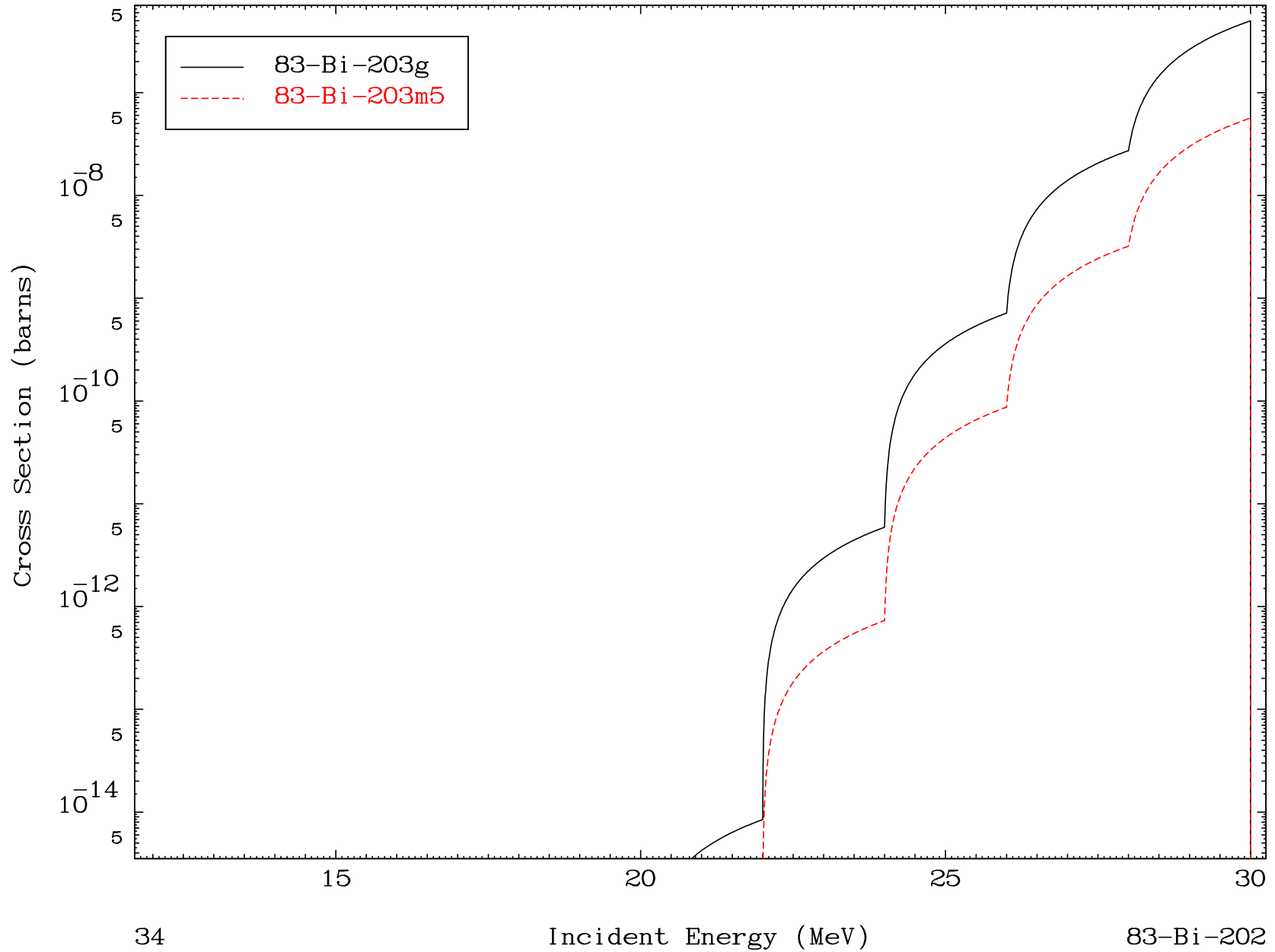
( $\alpha, t$ )

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### Radionuclide Production Cross Section



Radionuclide Production Cross Section

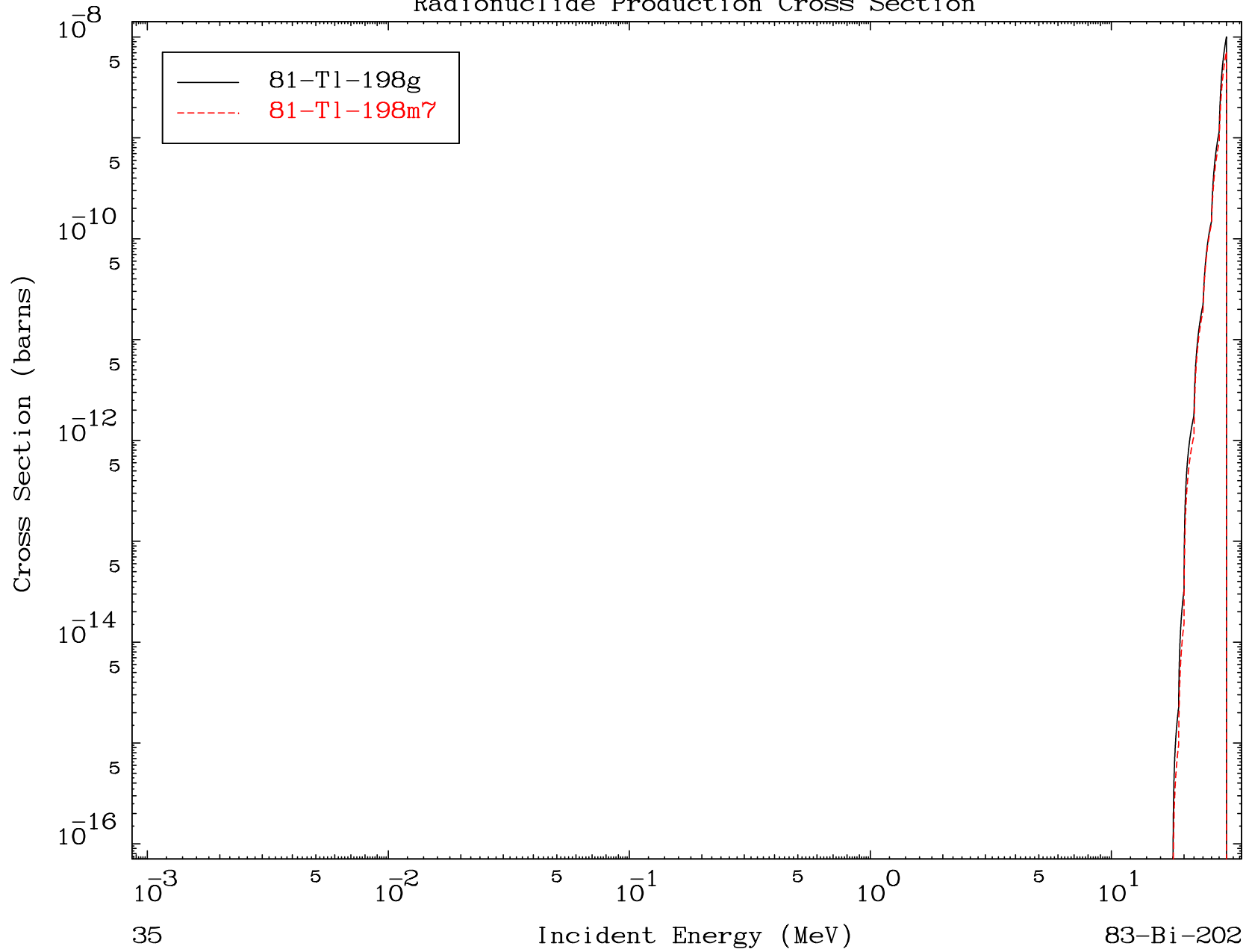


MAT 8304

( $\alpha, 2\alpha$ )

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

