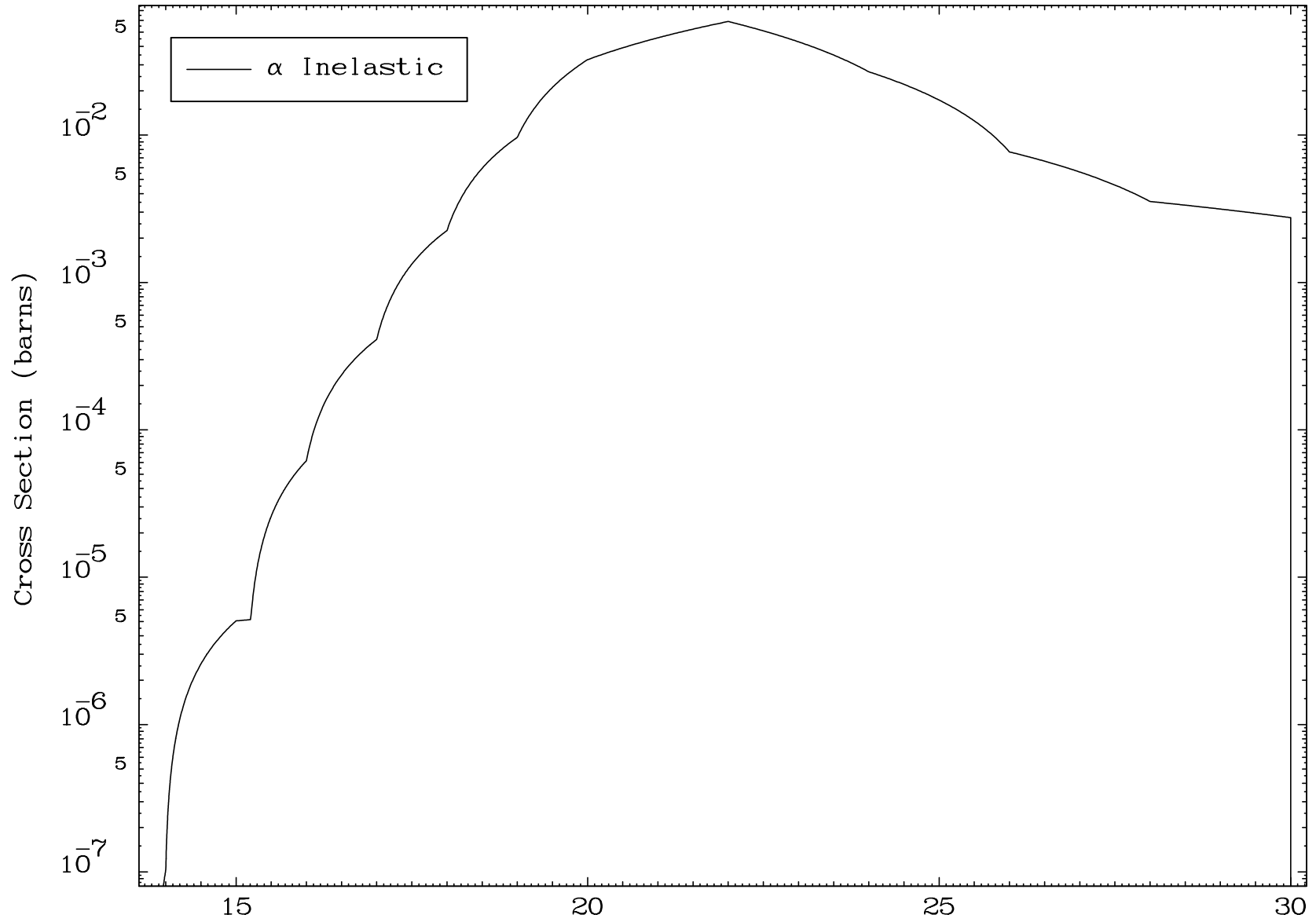


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(α, n') Level
0 Kelvin Cross Sections

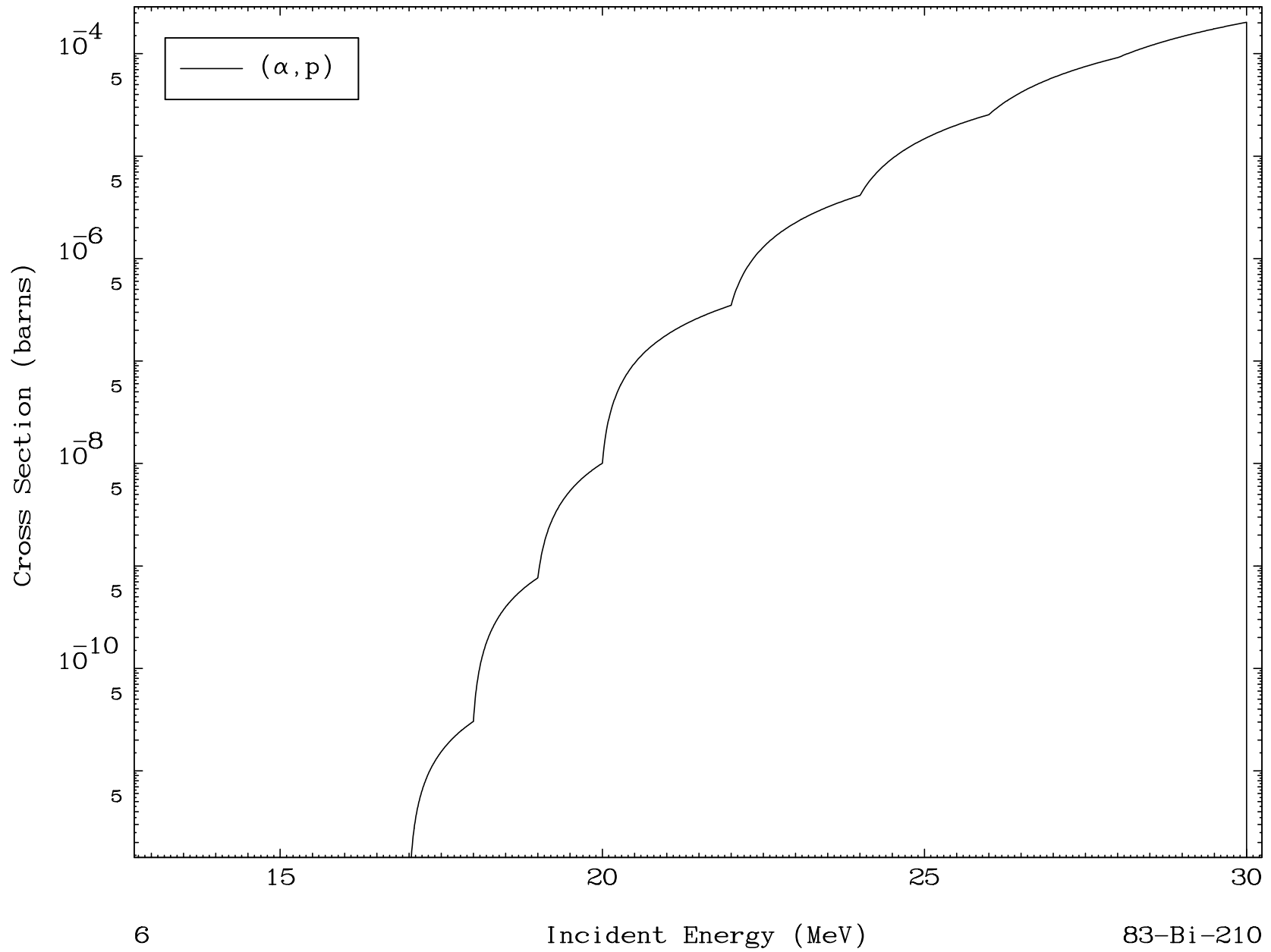
83-Bi-210

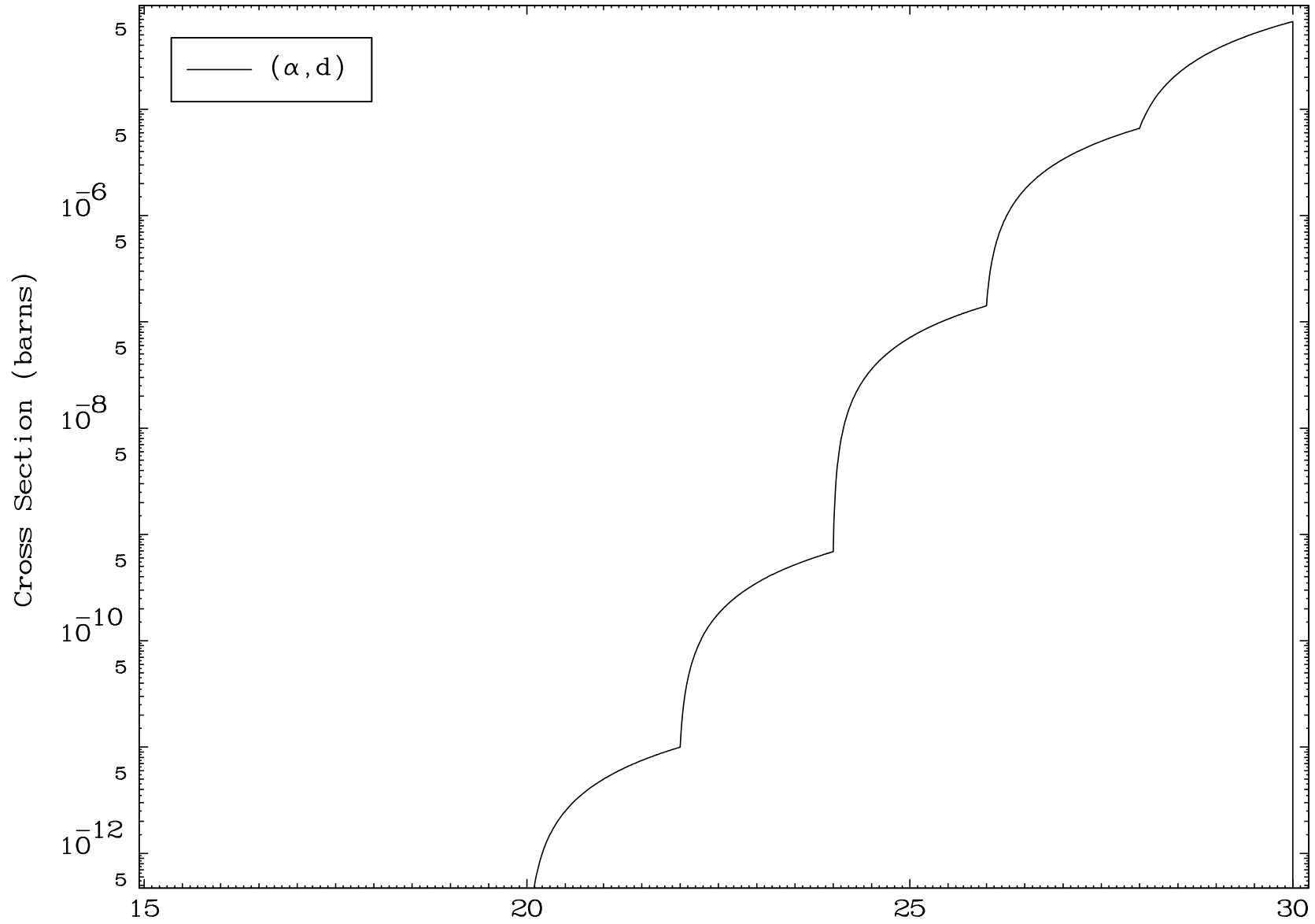


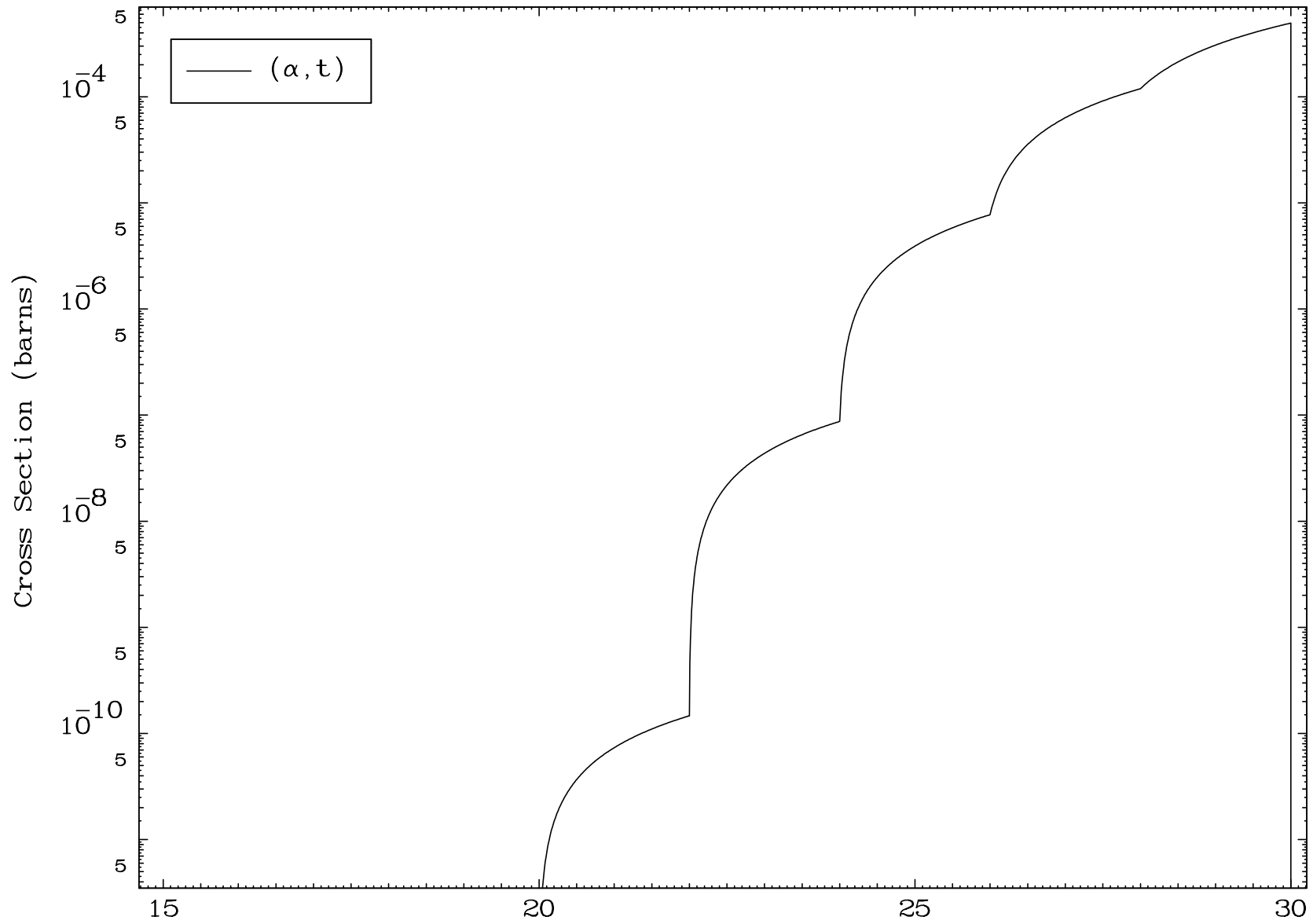
5

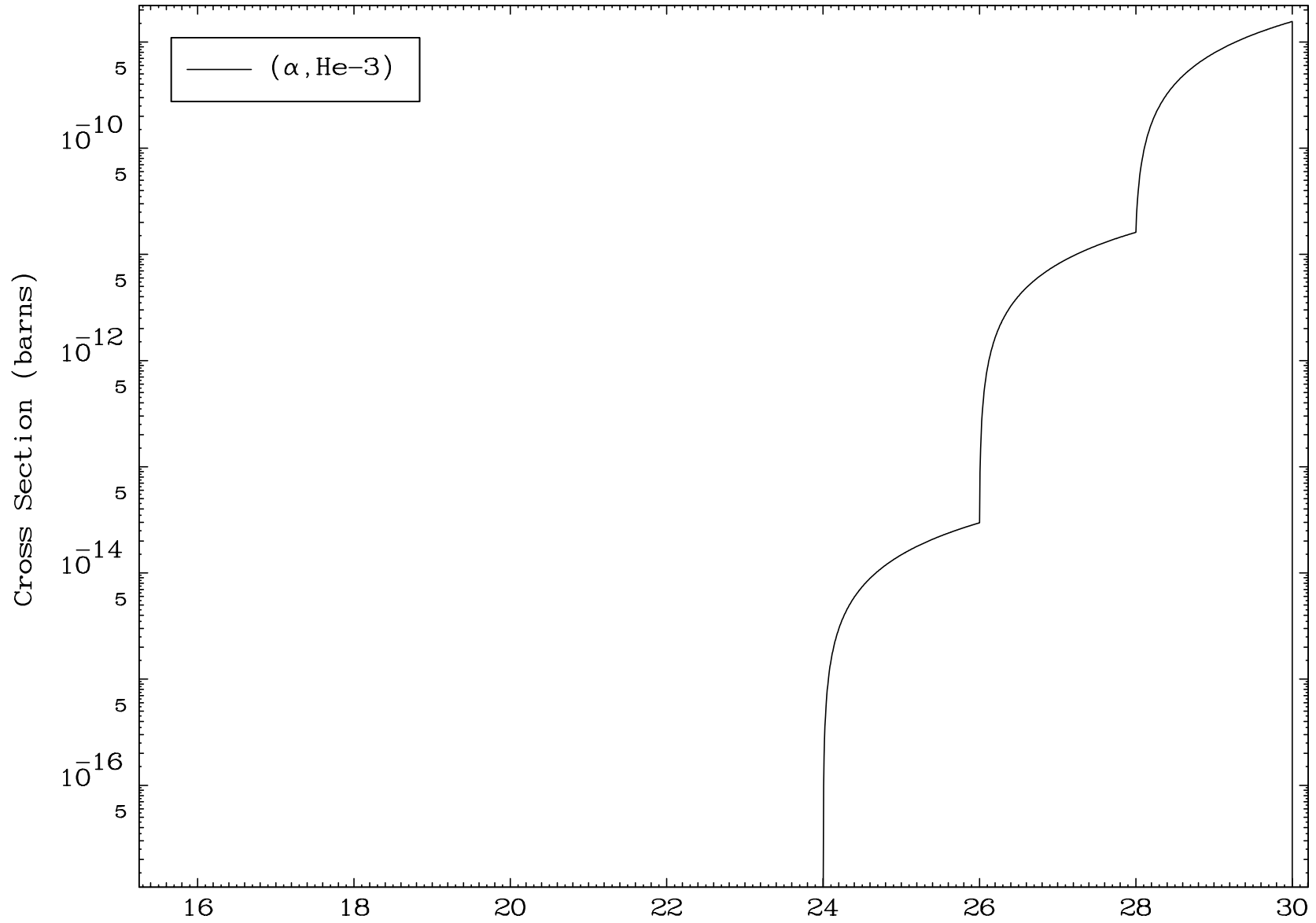
Incident Energy (MeV)

83-Bi-210





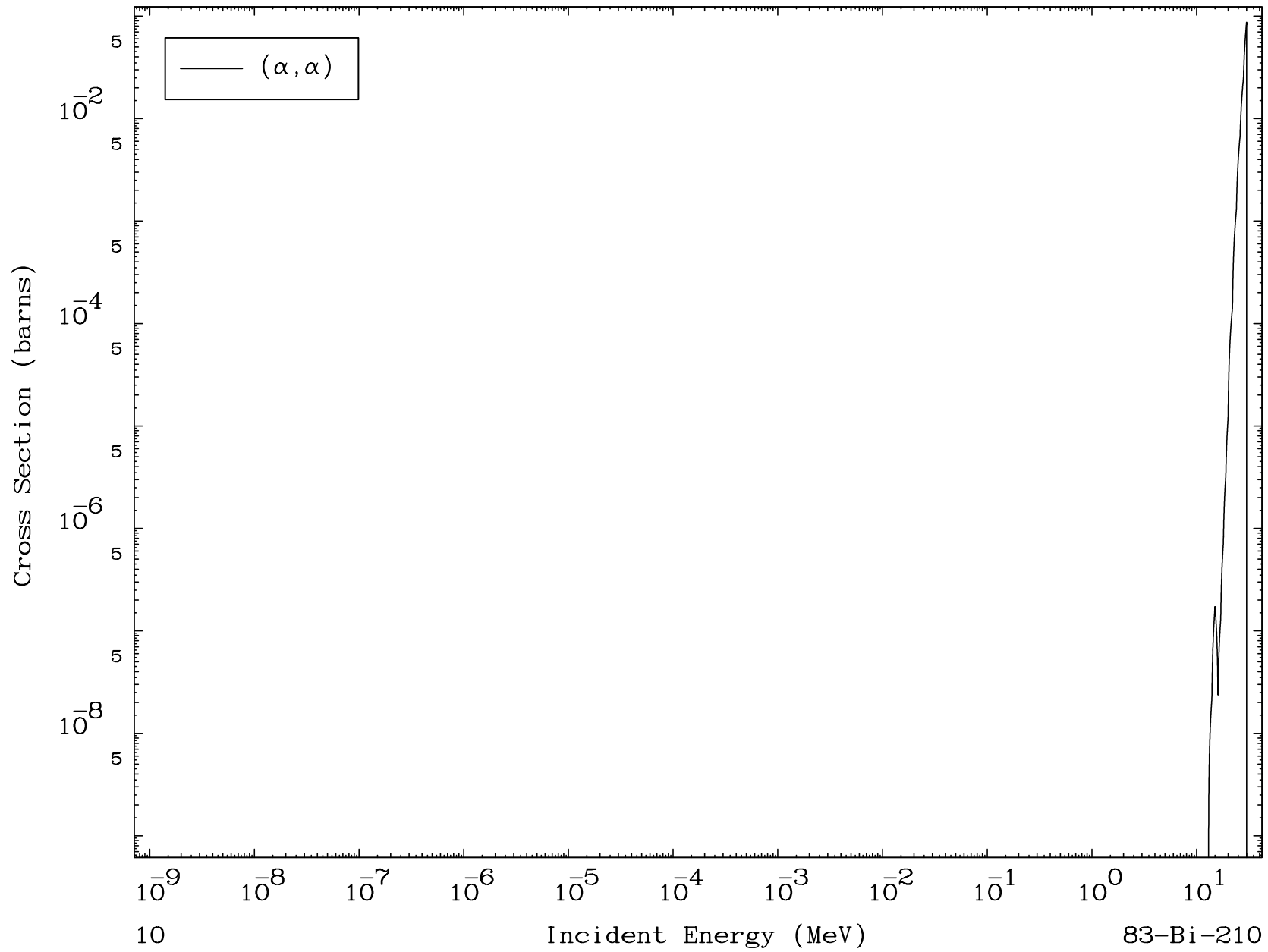




MAT 8329

(α, α) Levels
0 Kelvin Cross Sections

83-Bi-210

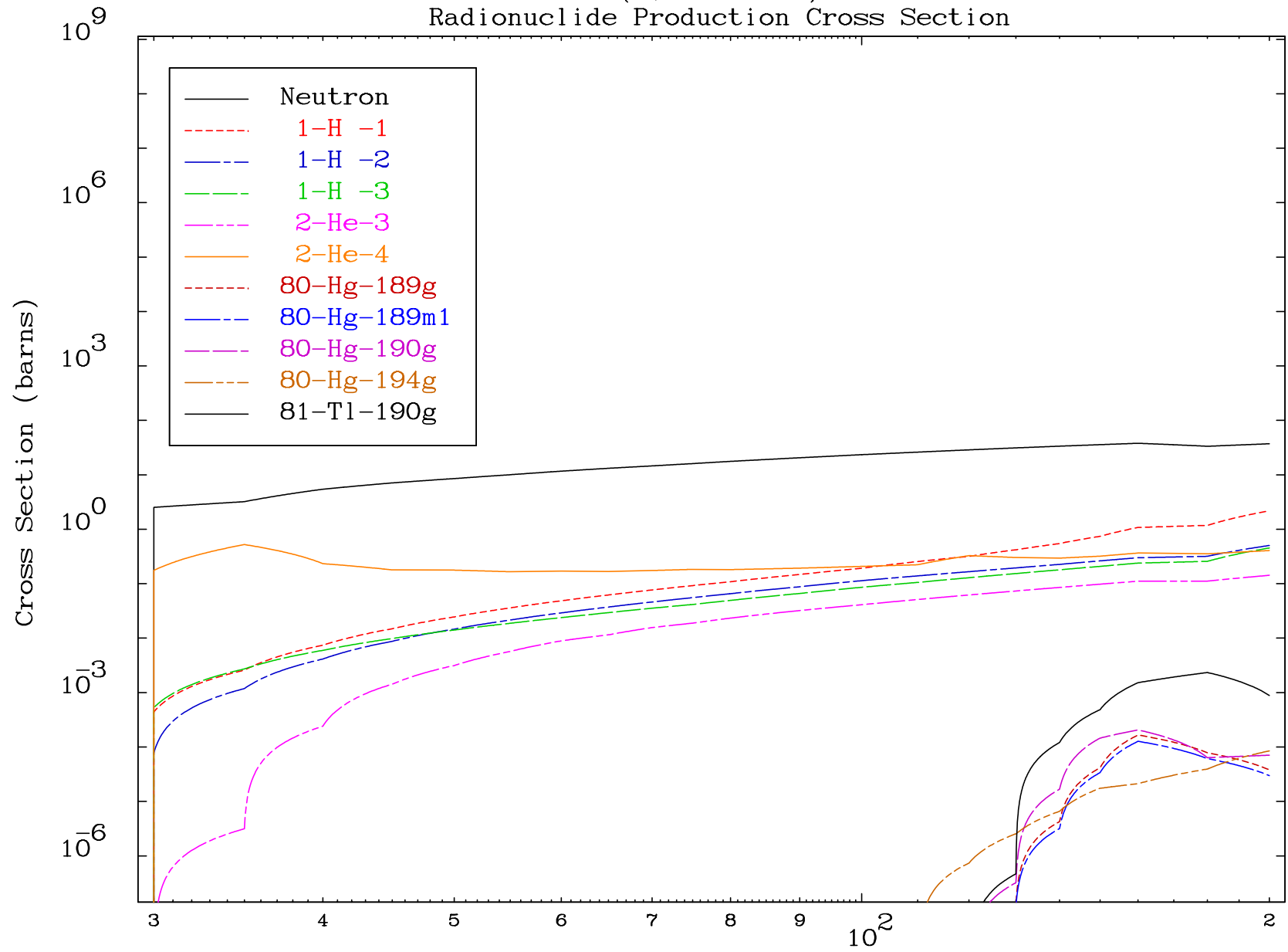


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(α , remainder)

83-Bi-210

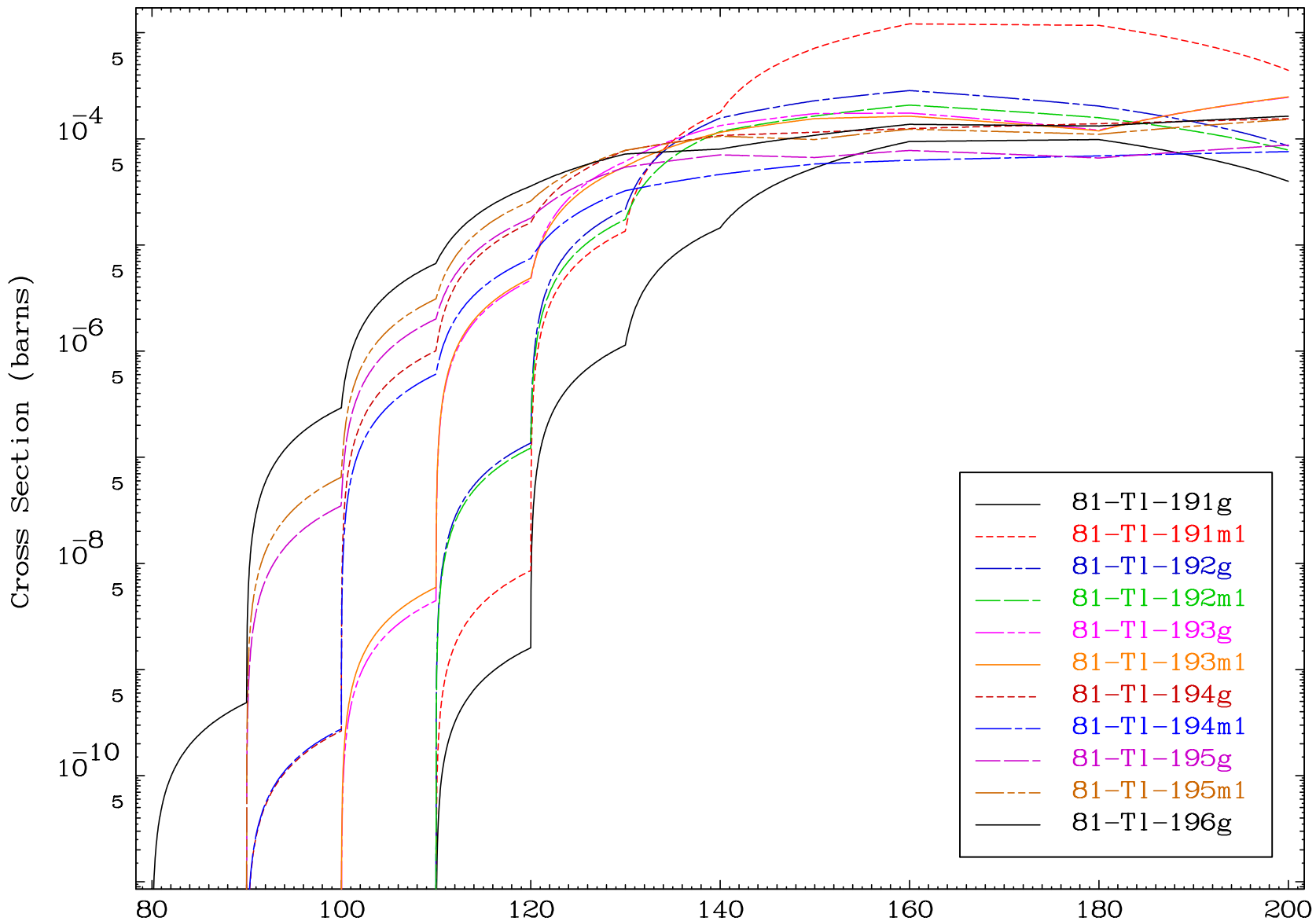
Radionuclide Production Cross Section

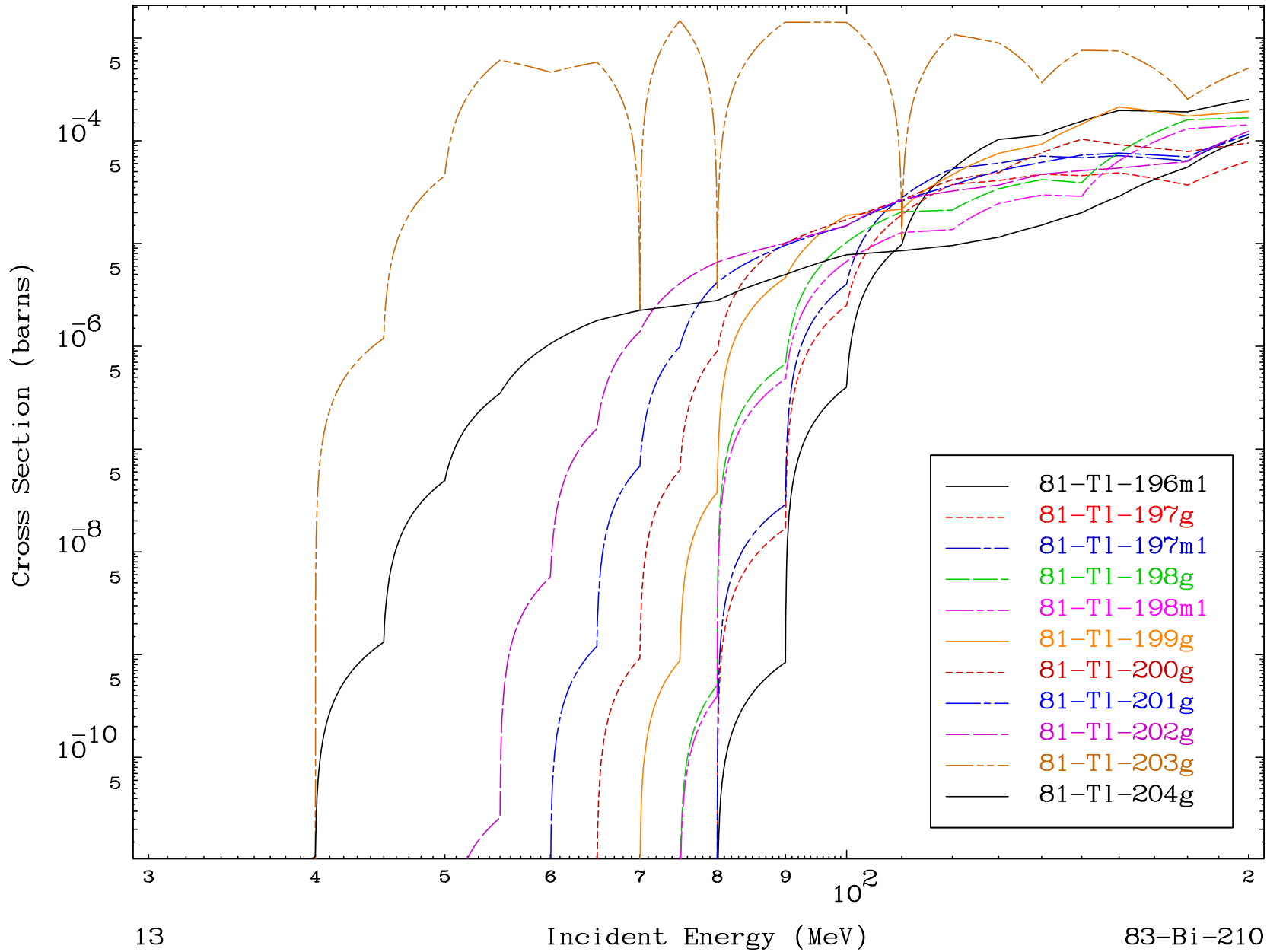


11

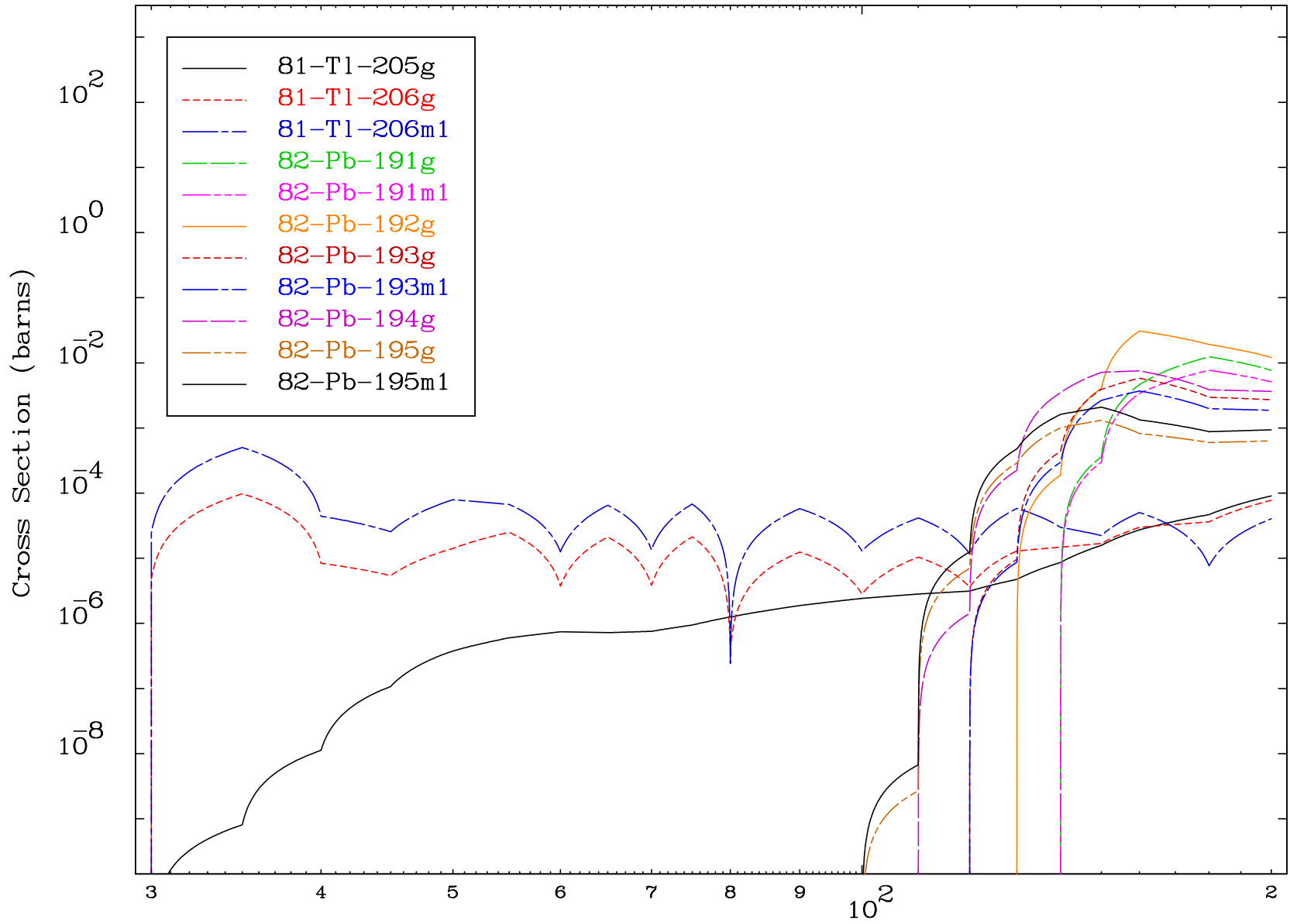
Incident Energy (MeV)

83-Bi-210

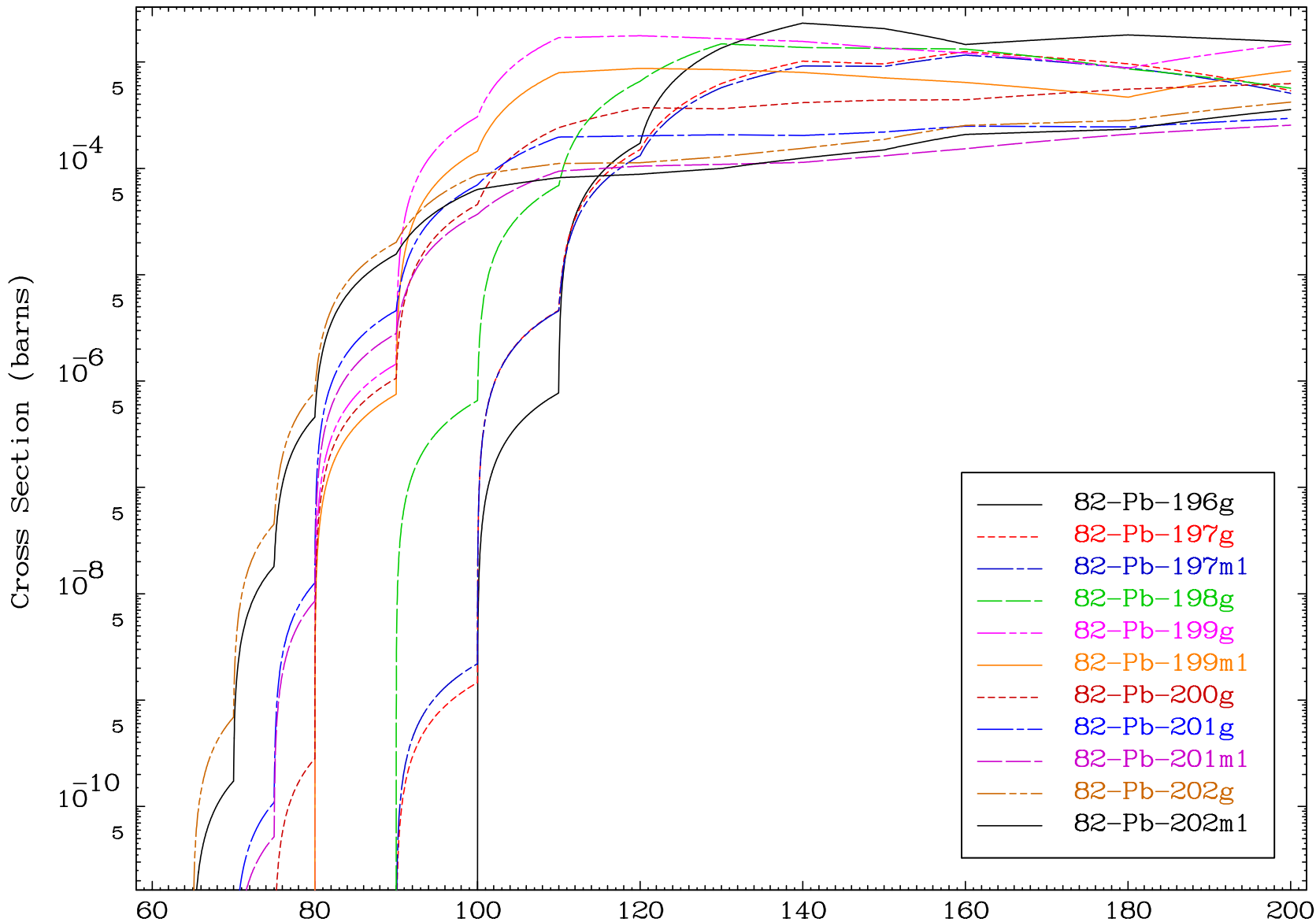




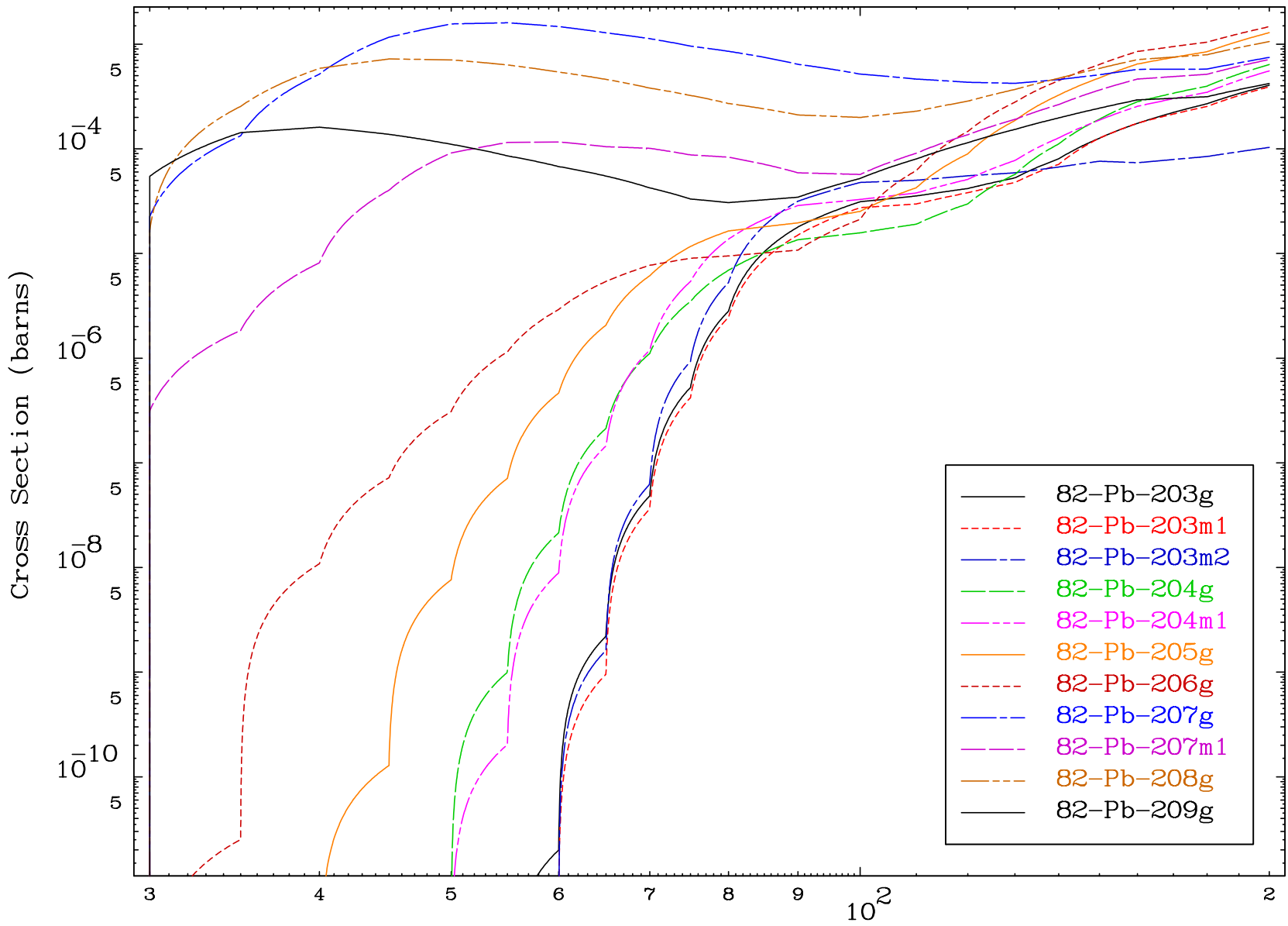
Radionuclide Production Cross Section

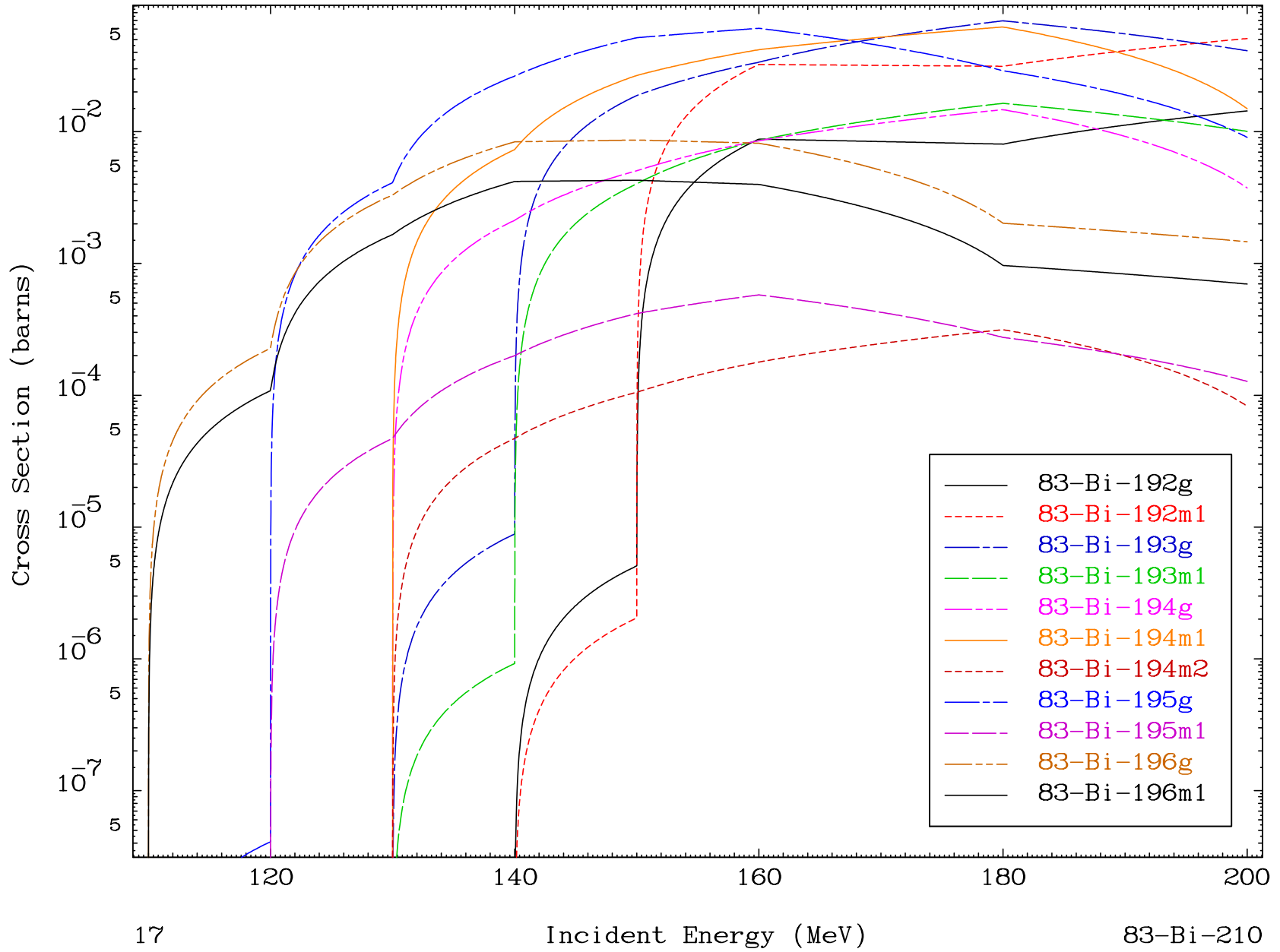


Radionuclide Production Cross Section

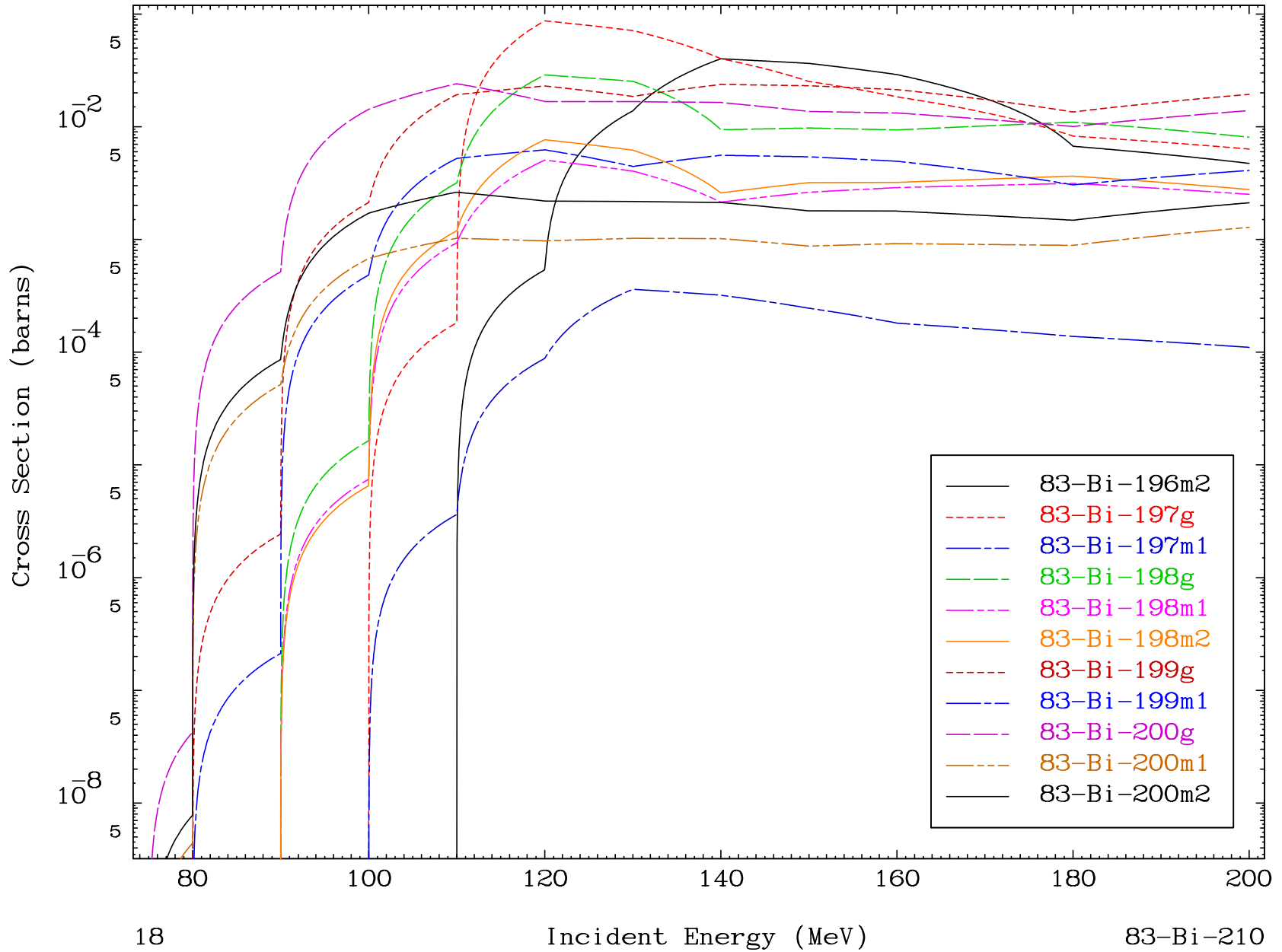


Radionuclide Production Cross Section

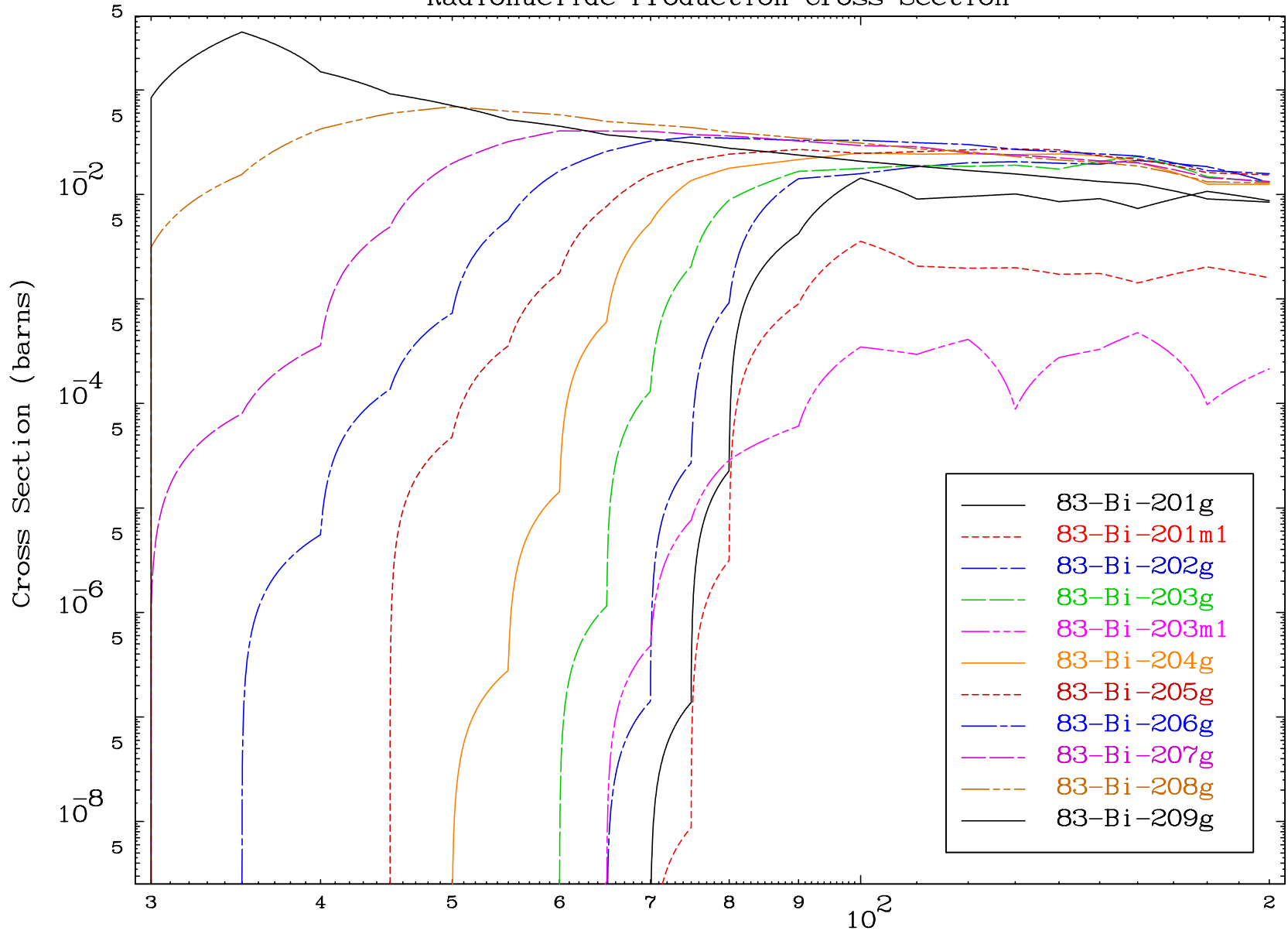




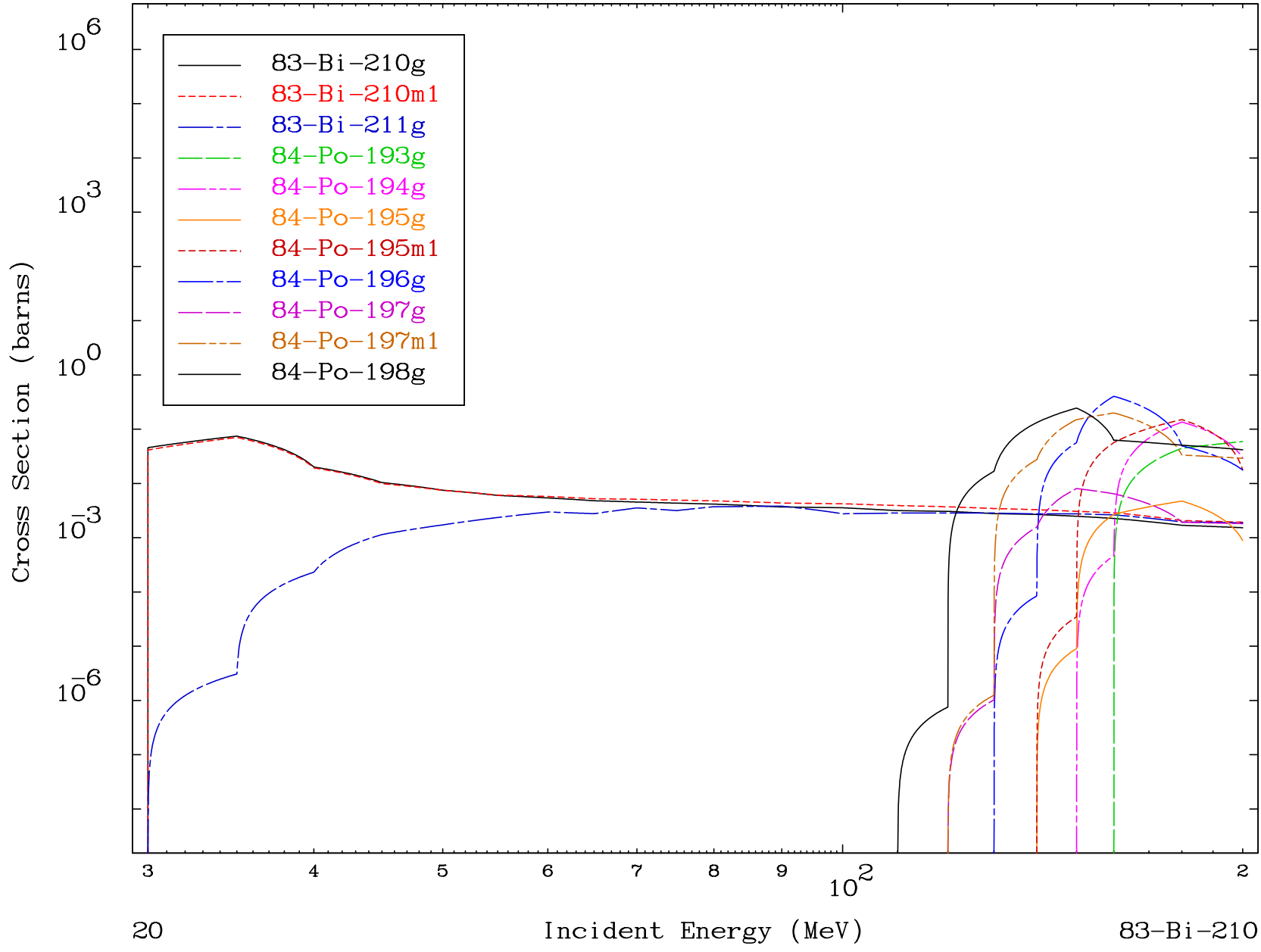
Radionuclide Production Cross Section

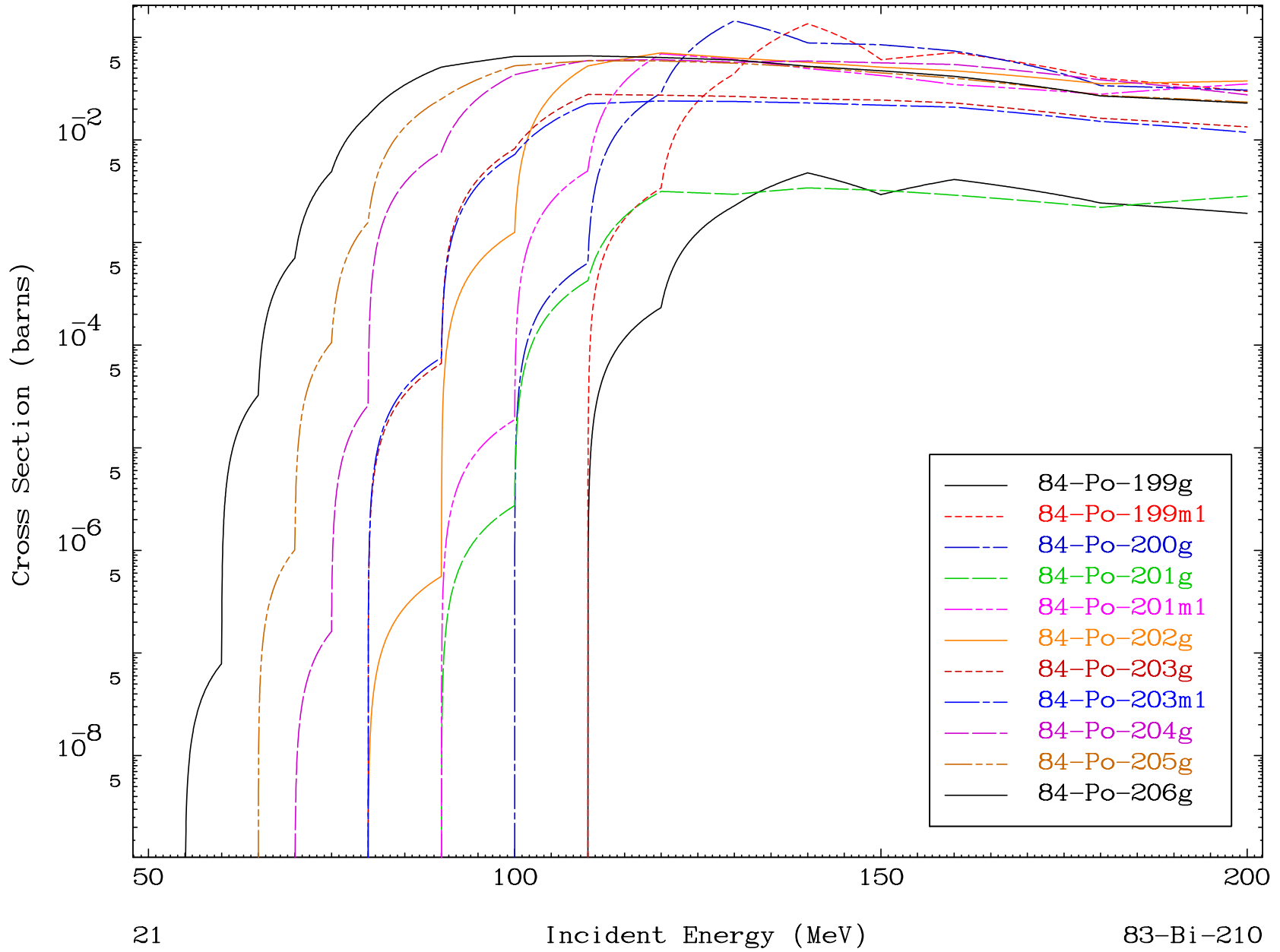


Radionuclide Production Cross Section

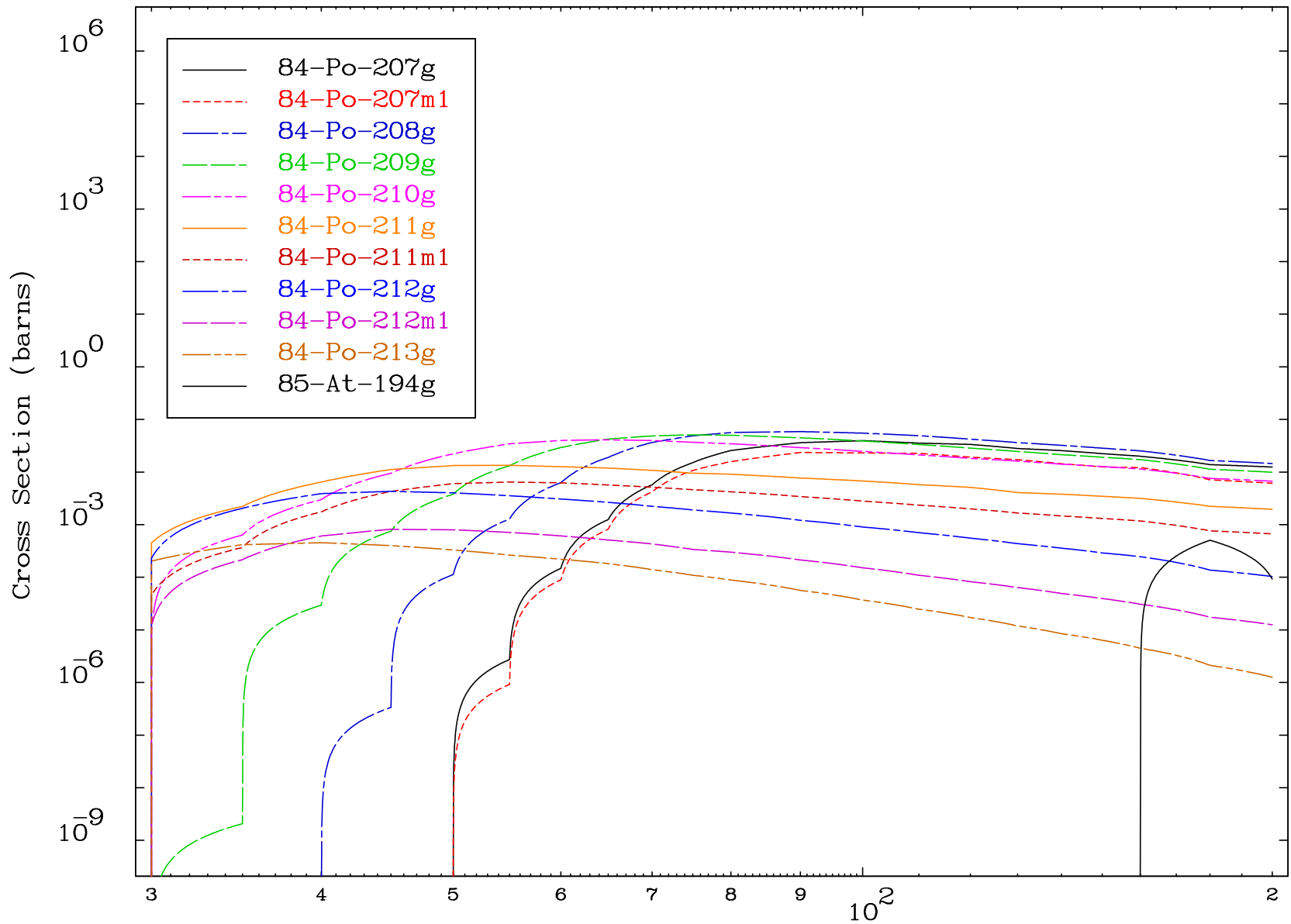


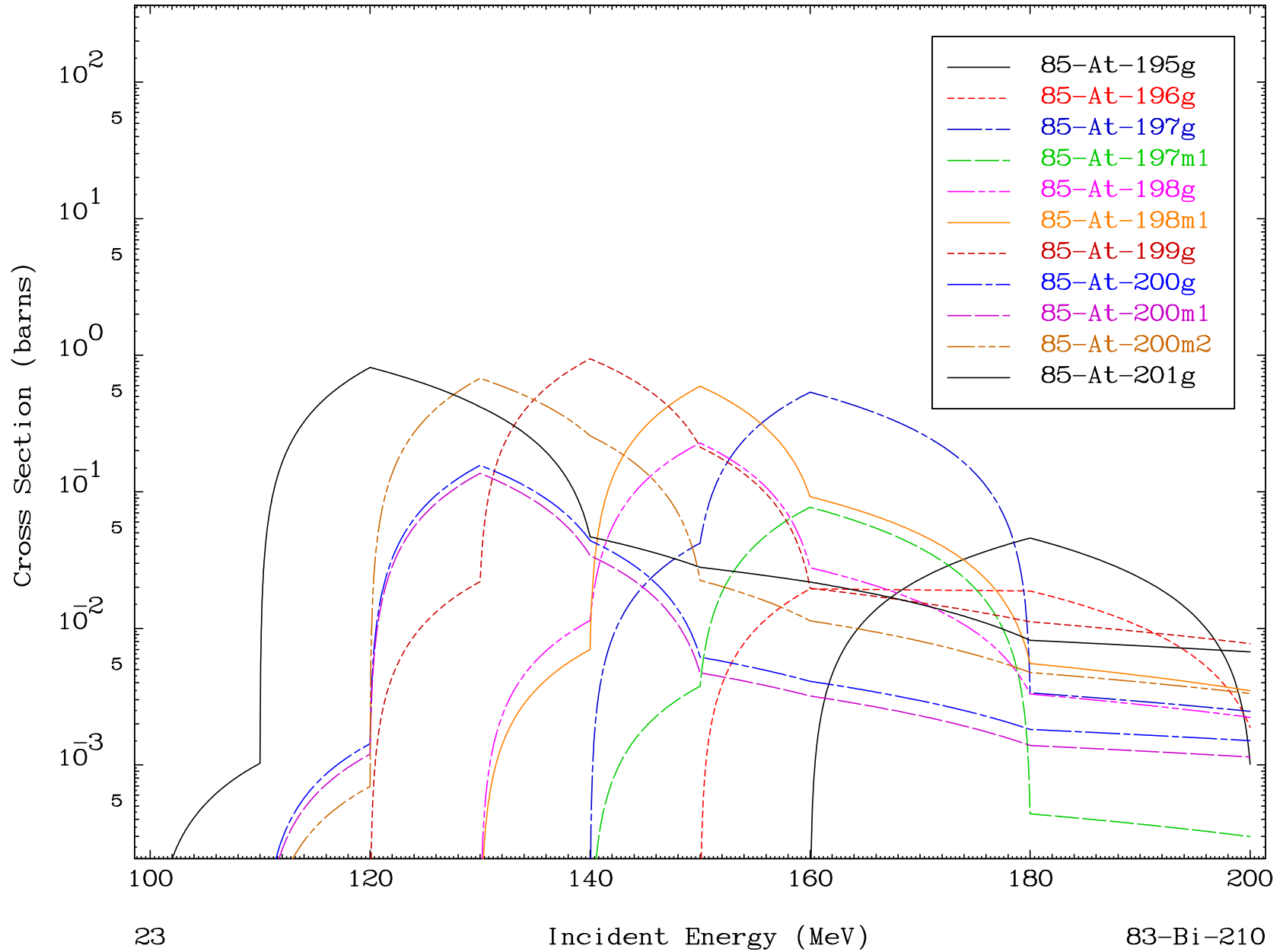
Radionuclide Production Cross Section

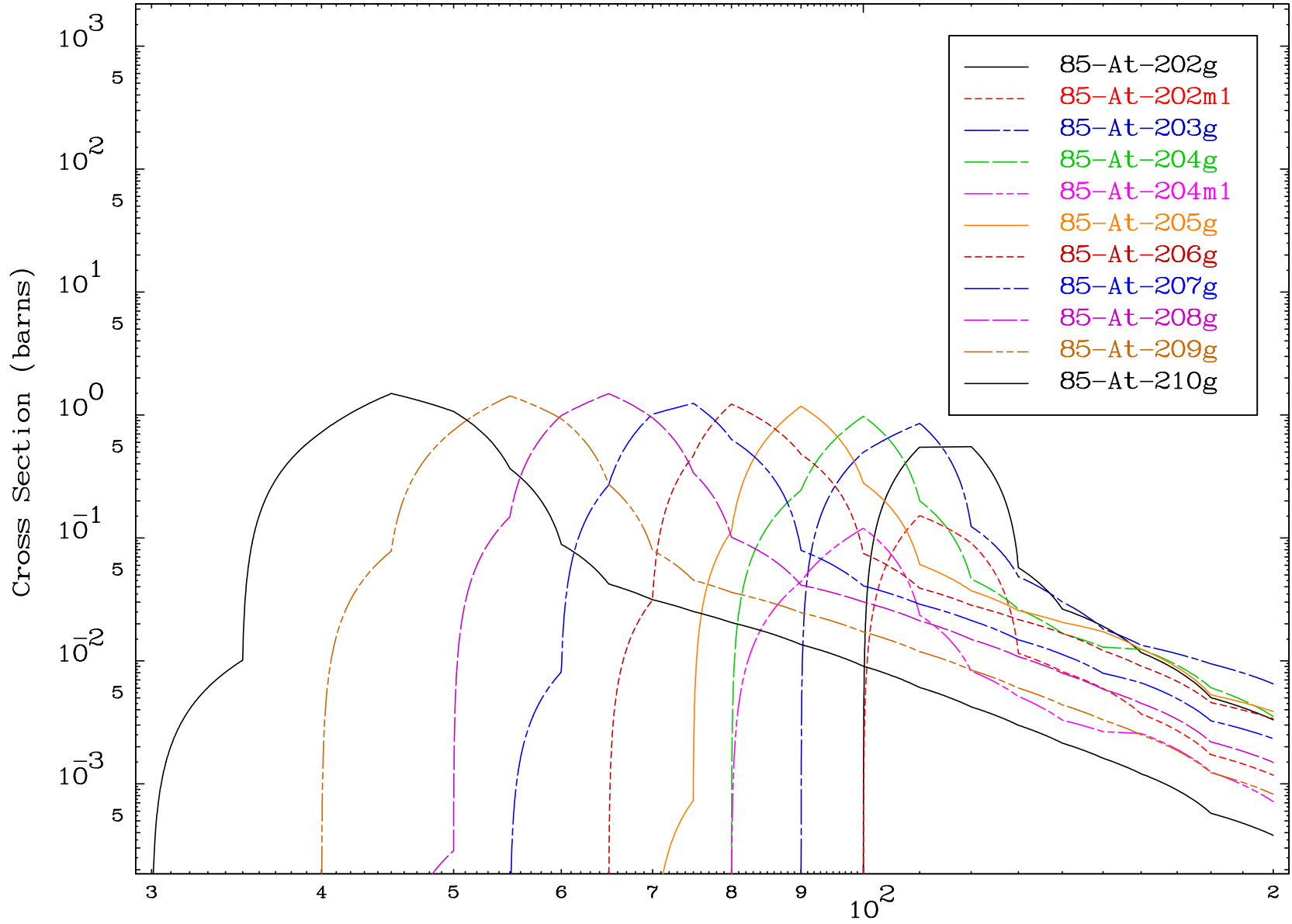




Radionuclide Production Cross Section





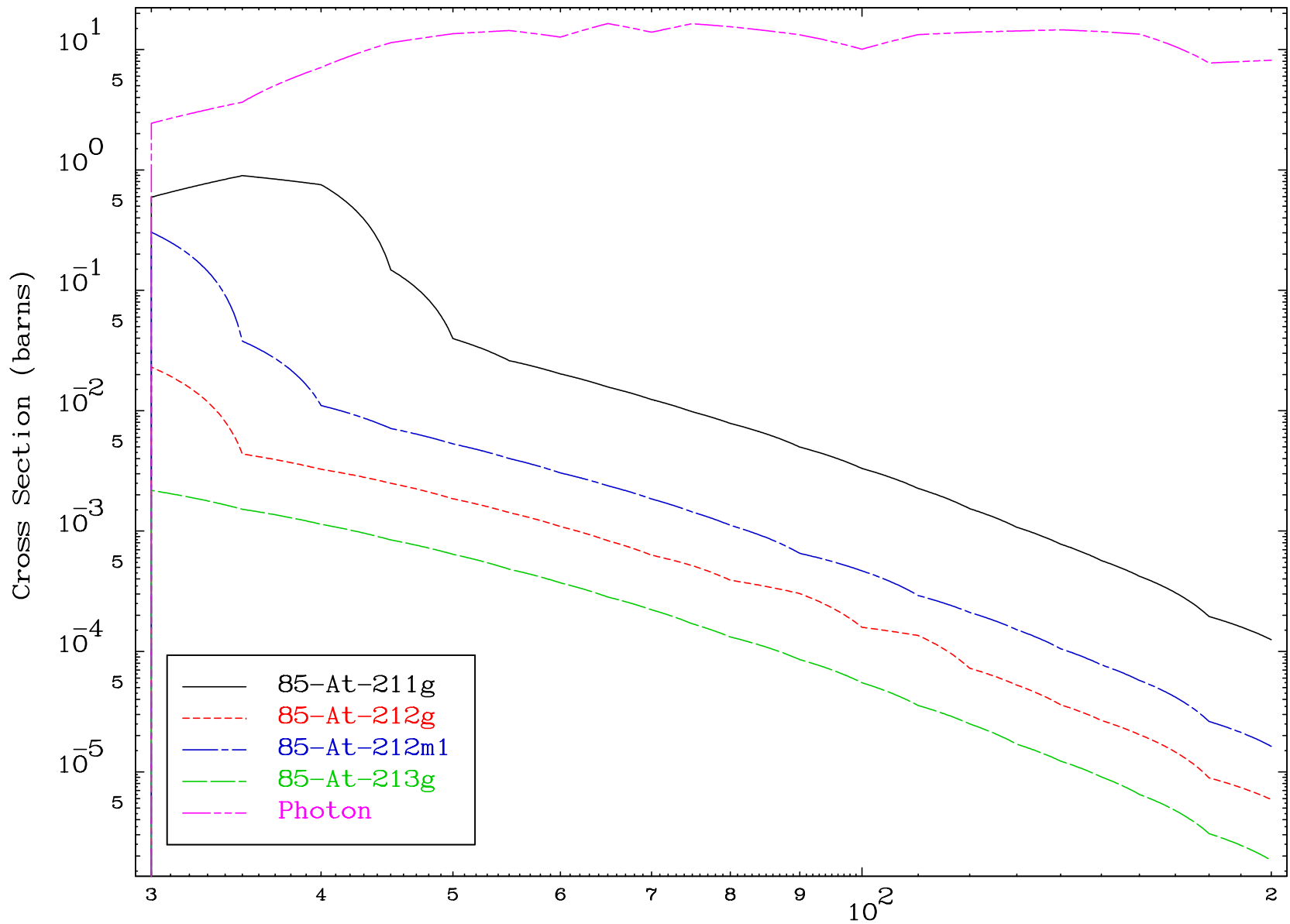


MAT 8329

(α , remainder)

83-Bi-210

Radionuclide Production Cross Section



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Incident Energy (MeV)

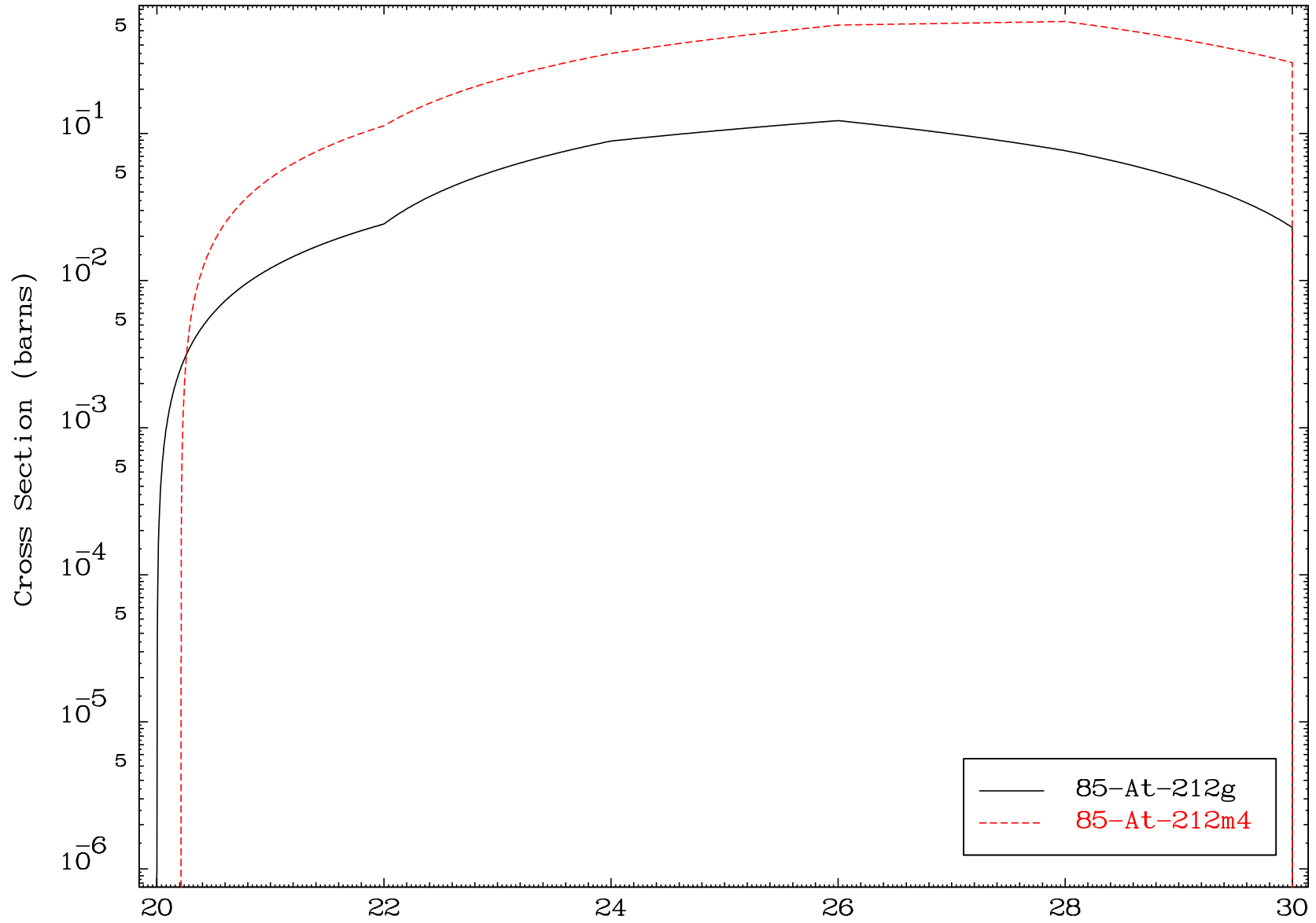
83-Bi-210

MAT 8329

($\alpha, 2n$)

83-Bi-210

Radionuclide Production Cross Section



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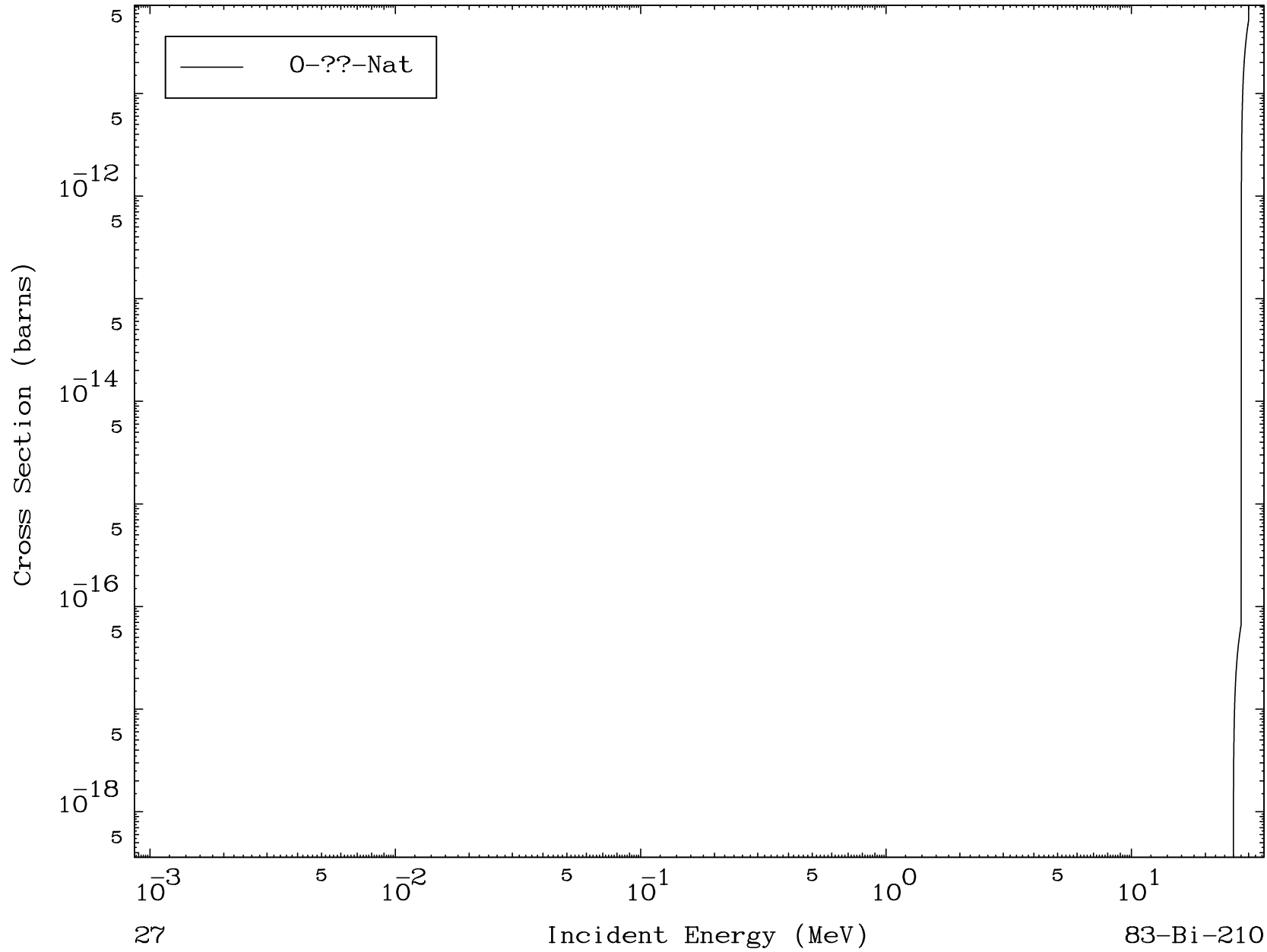
Incident Energy (MeV)

83-Bi-210

MAT 8329

α Fission
Radionuclide Production Cross Section

83-Bi-210



27

Incident Energy (MeV)

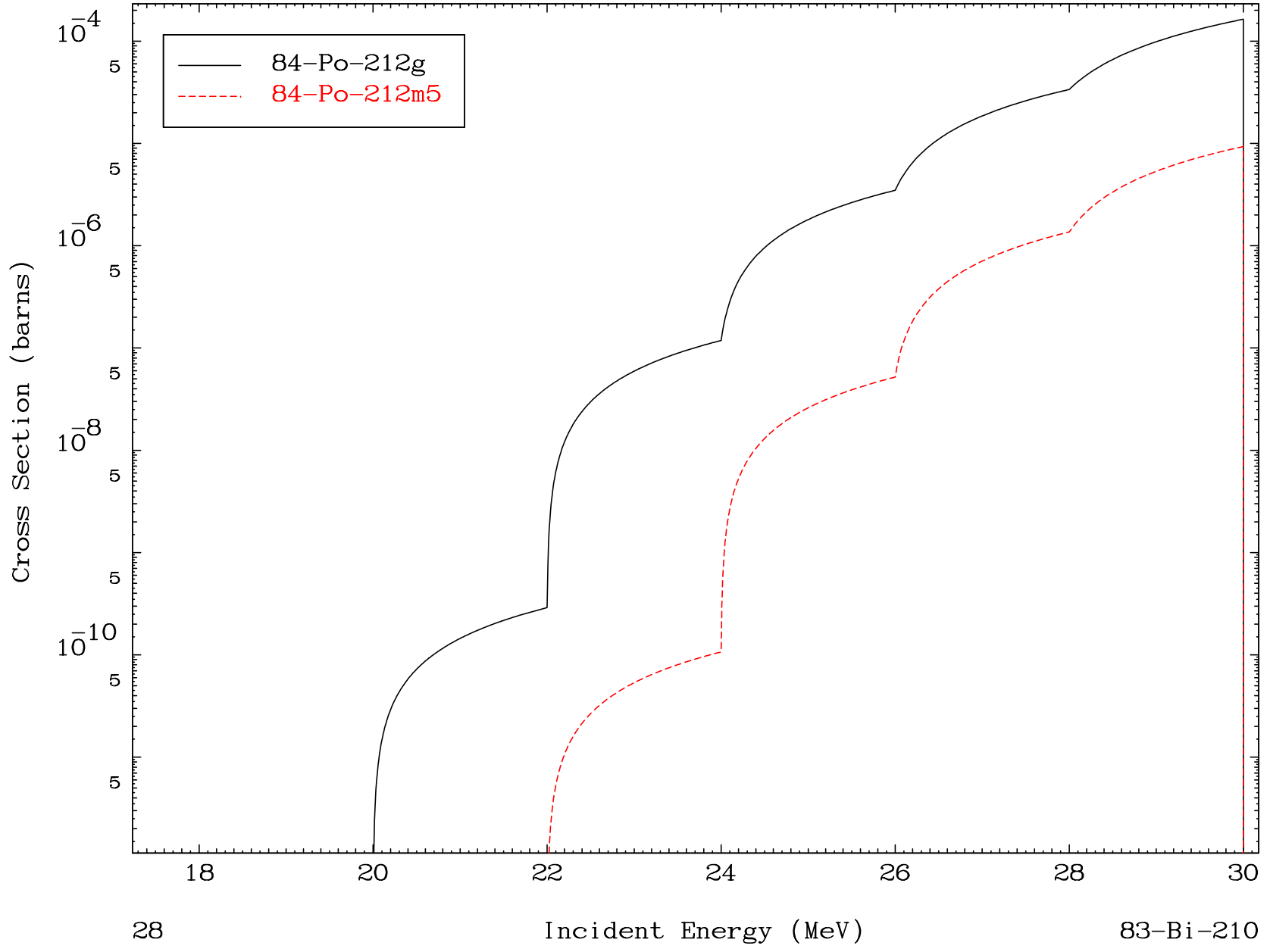
83-Bi-210

MAT 8329

(α, n') p

83-Bi-210

Radionuclide Production Cross Section



28

Incident Energy (MeV)

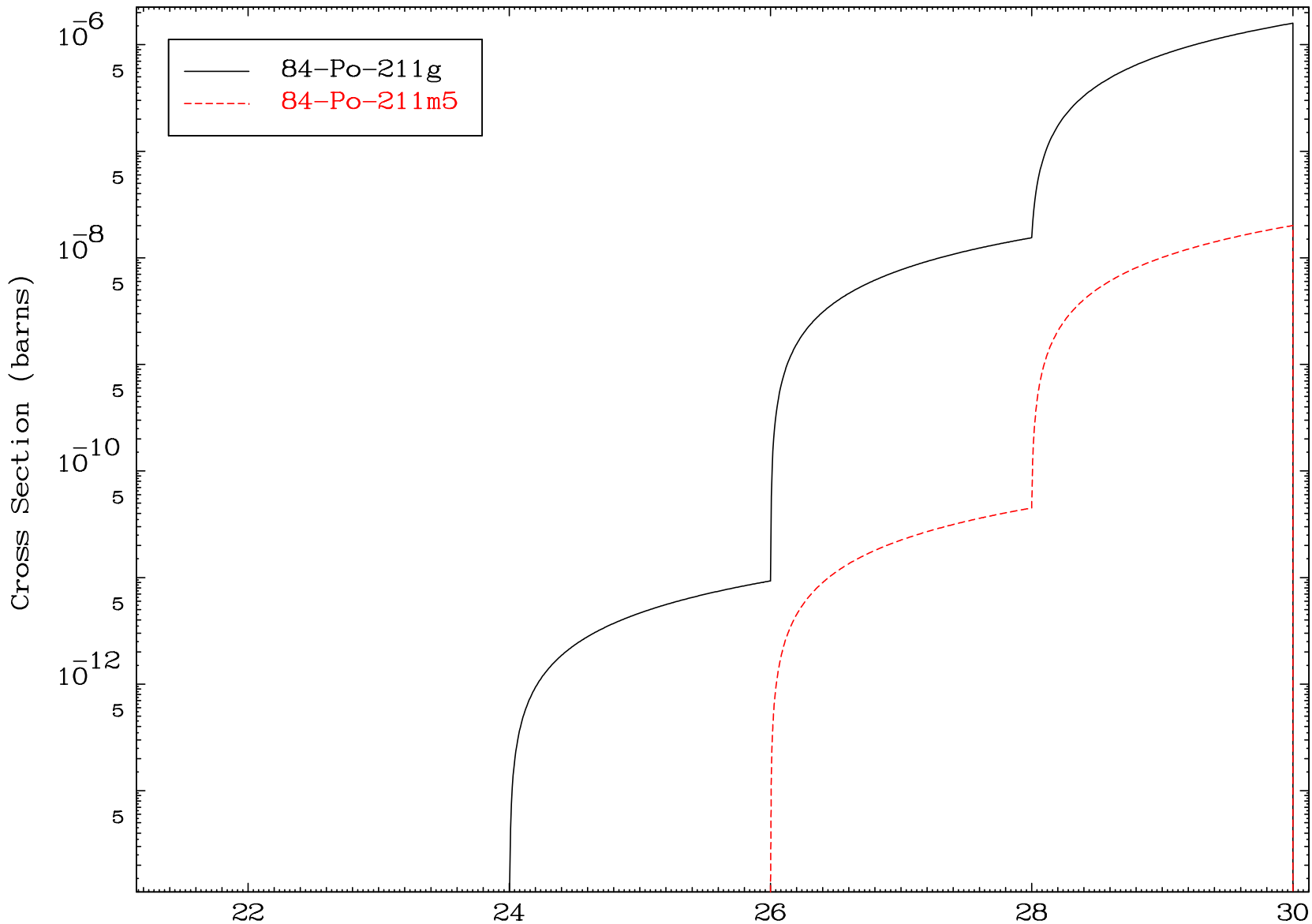
83-Bi-210

MAT 8329

(α, n') d

83-Bi-210

Radionuclide Production Cross Section



29

Incident Energy (MeV)

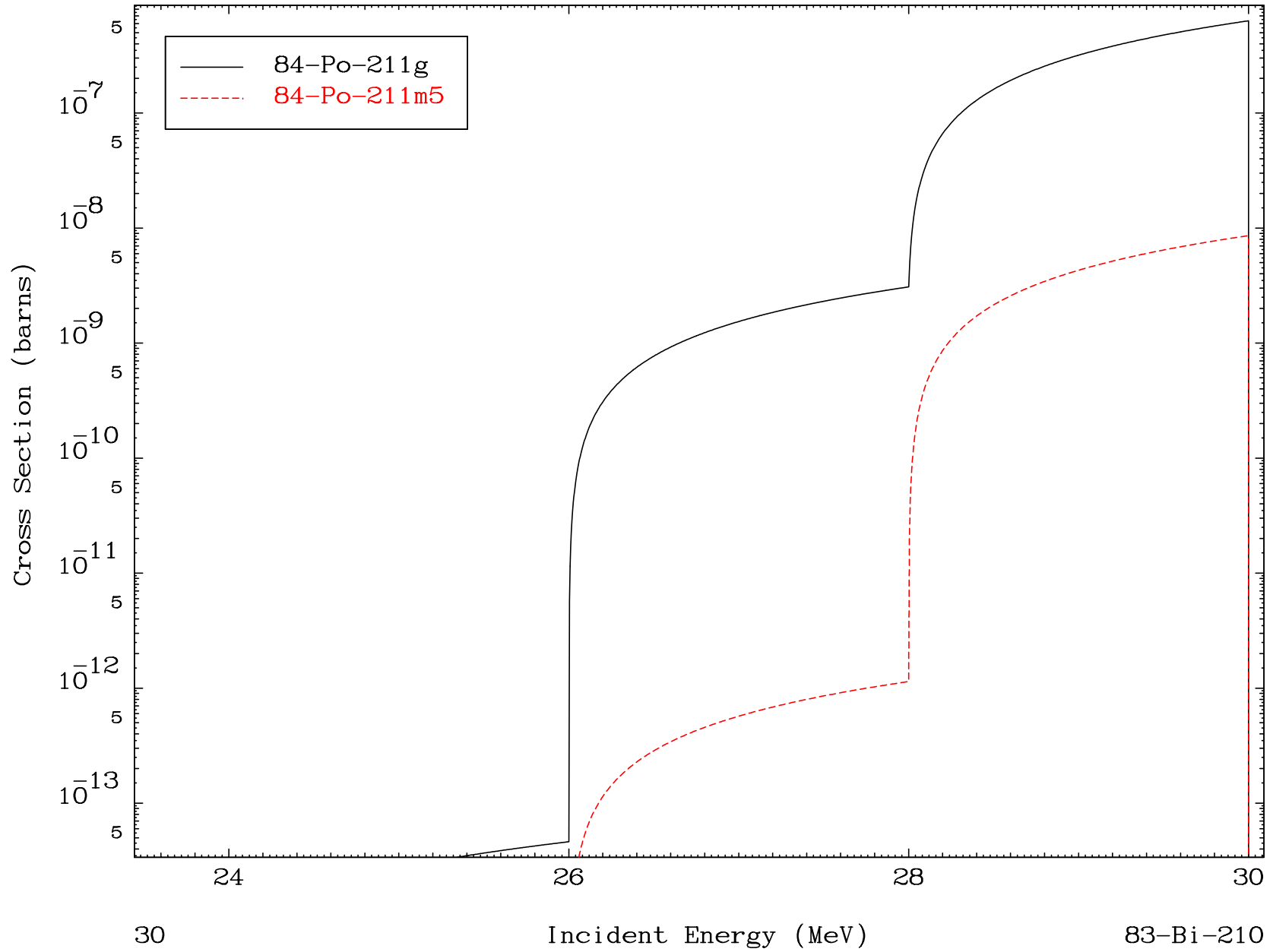
83-Bi-210

MAT 8329

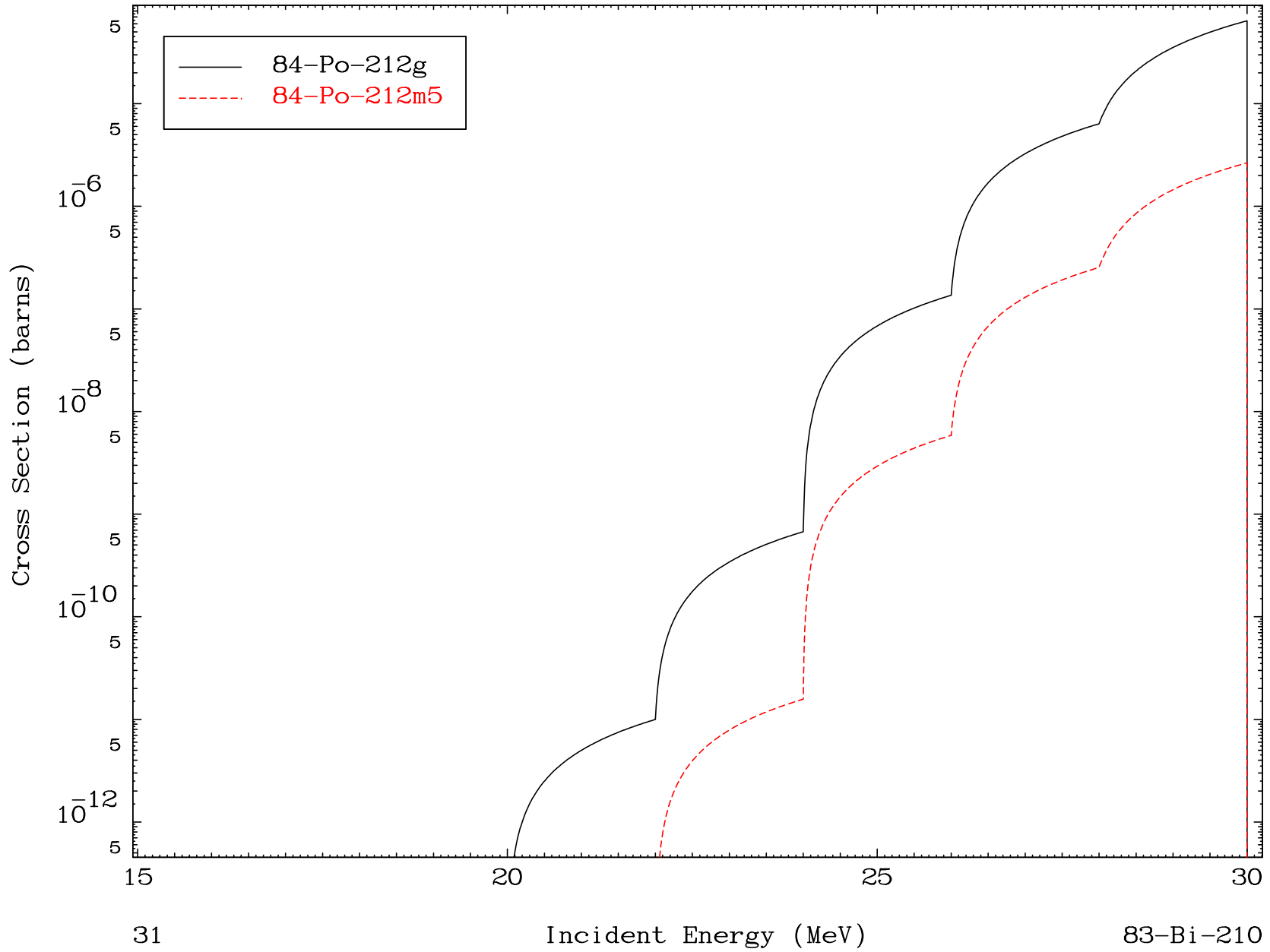
($\alpha, 2n$) p

83-Bi-210

Radionuclide Production Cross Section



Radionuclide Production Cross Section

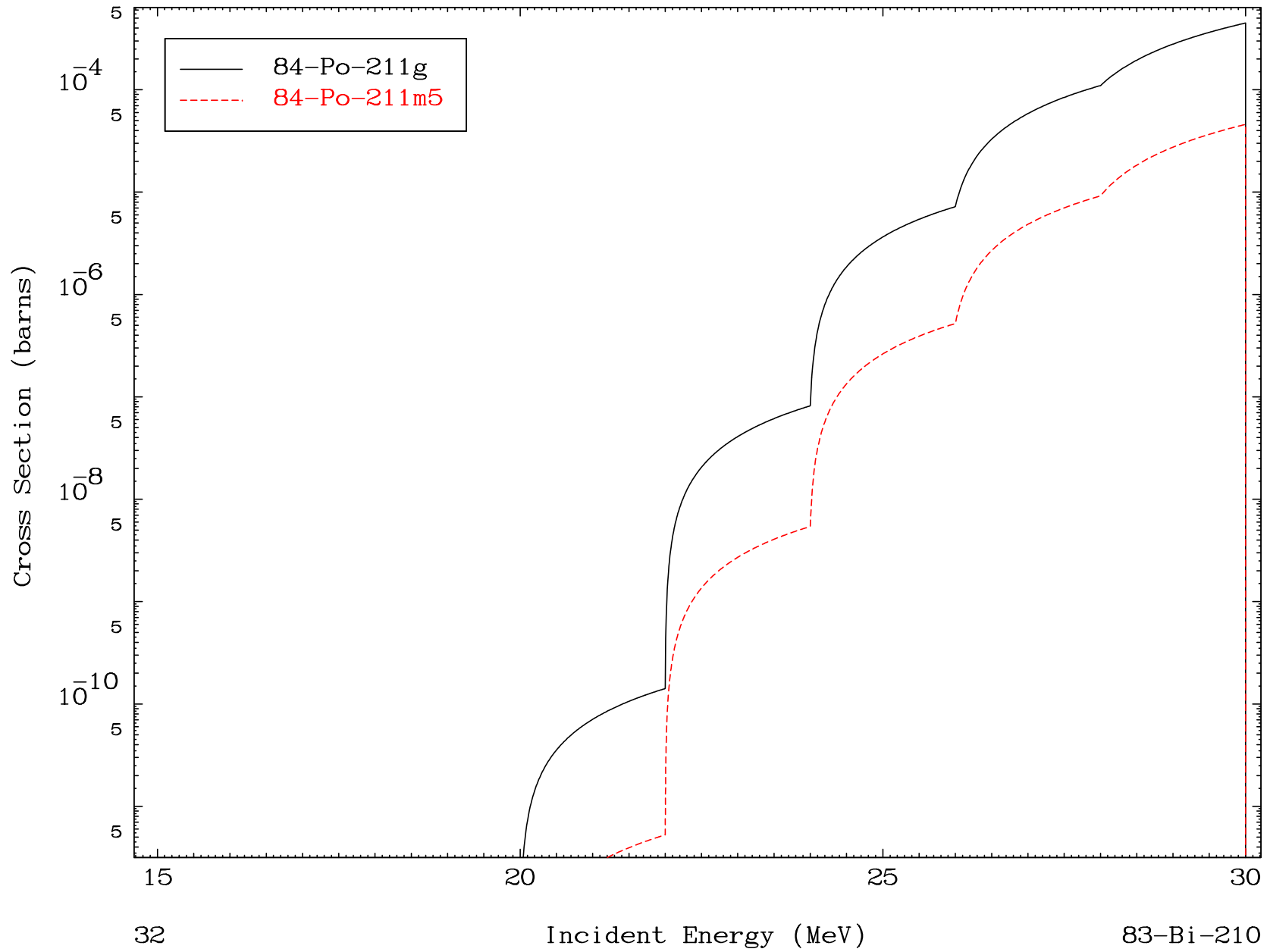


MAT 8329

(α, t)

83-Bi-210

Radionuclide Production Cross Section

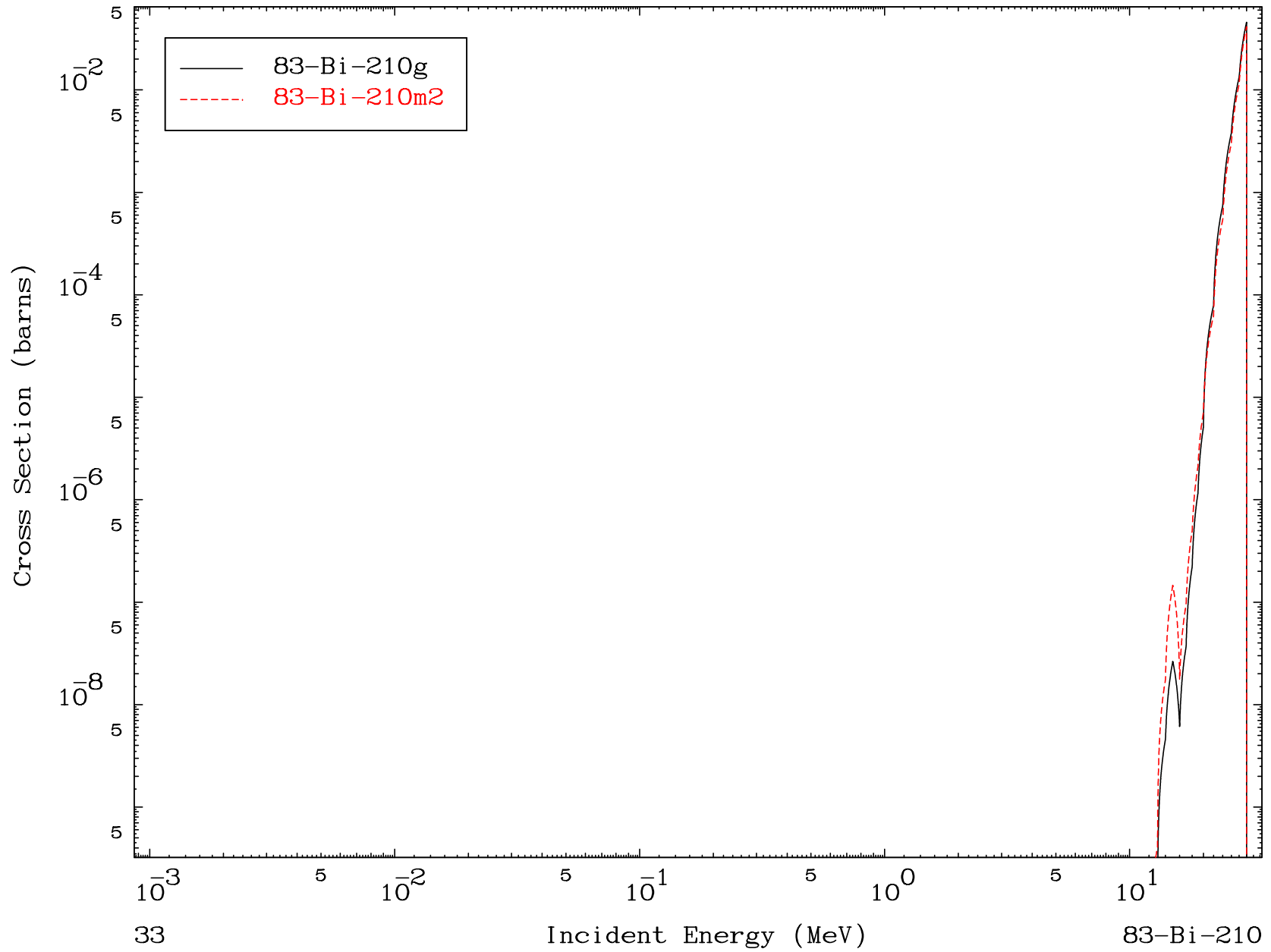


MAT 8329

(α, α)

83-Bi-210

Radionuclide Production Cross Section



Radionuclide Production Cross Section

