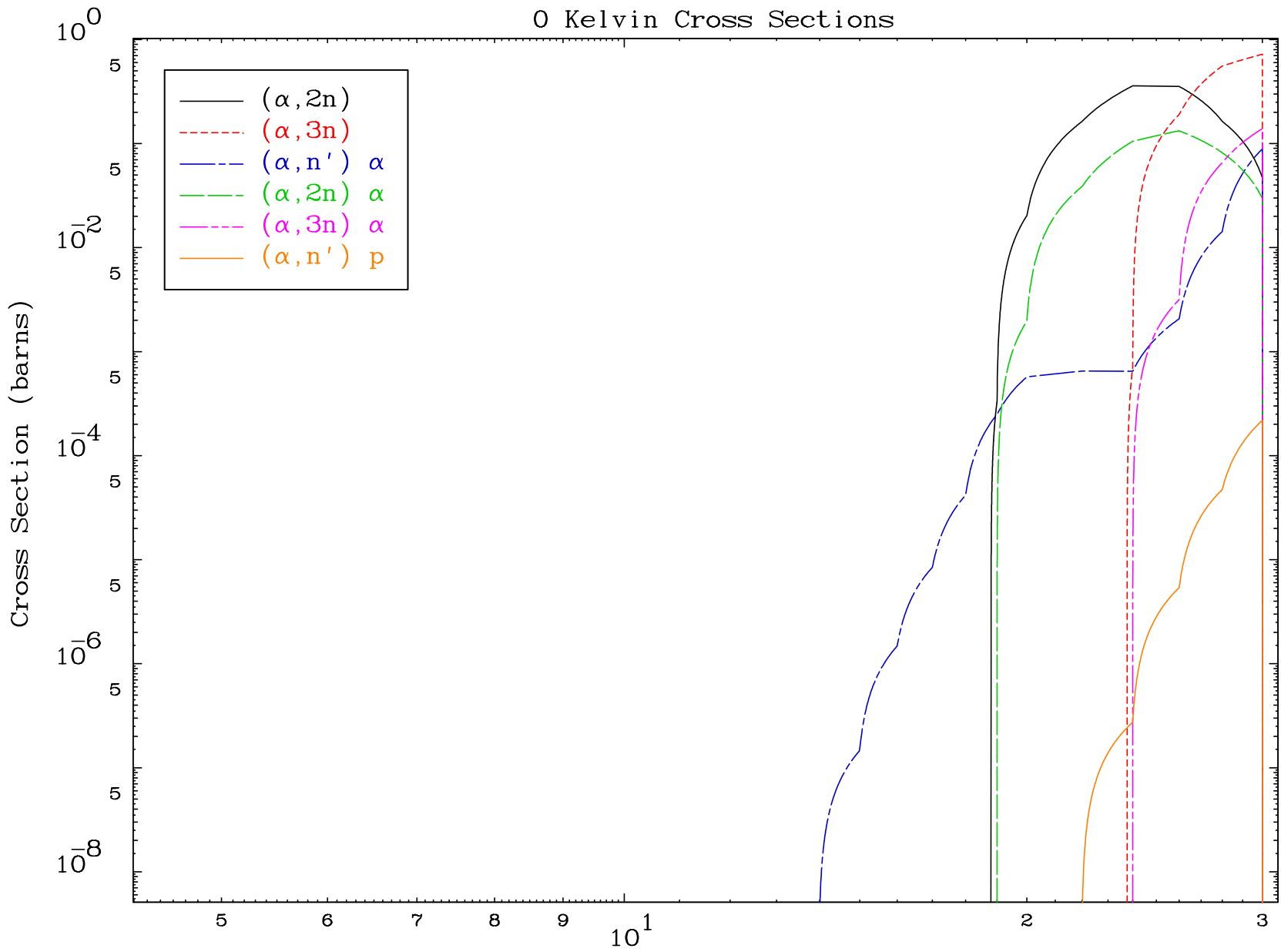


MAT 8334

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

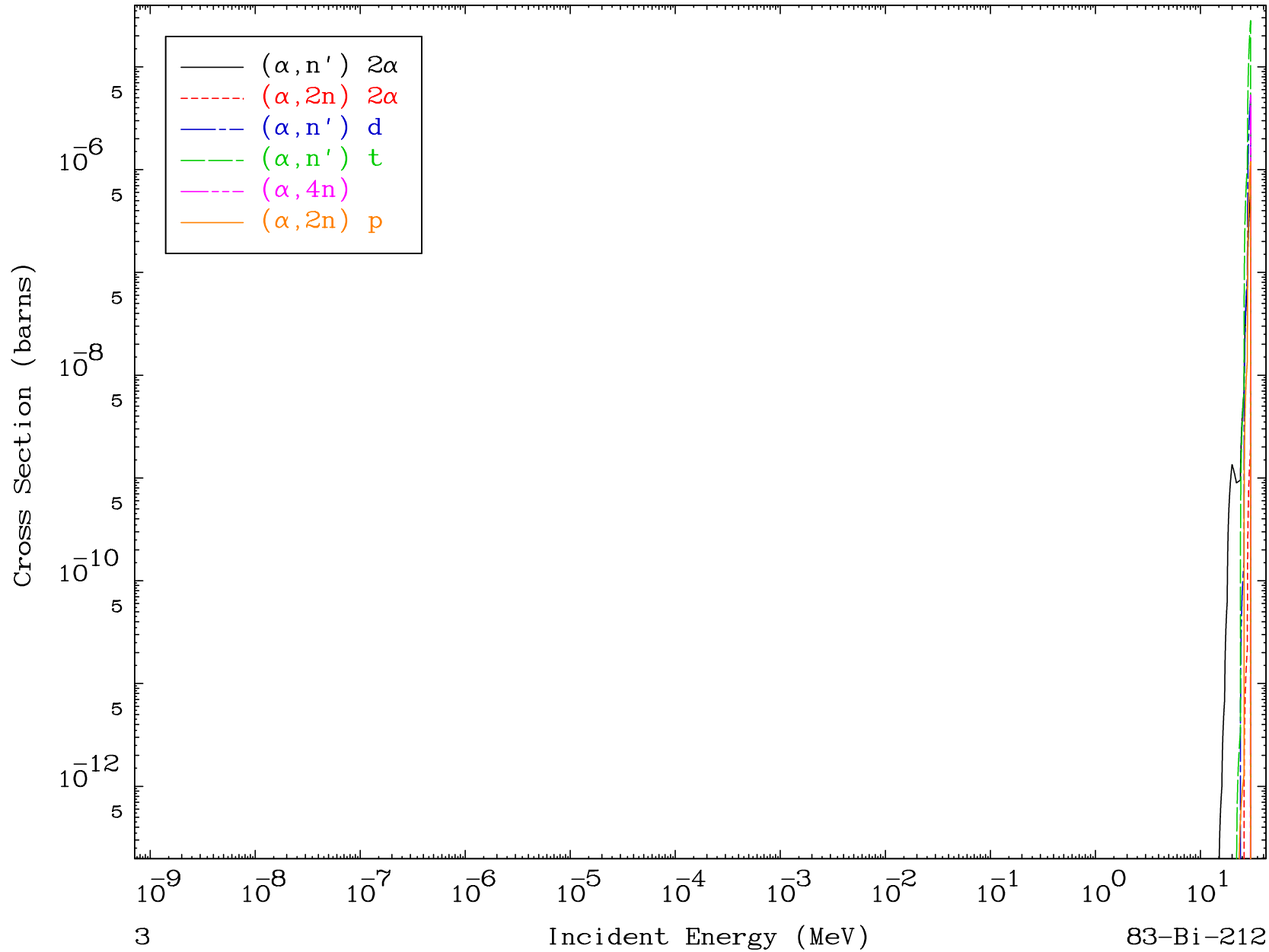
83-Bi-212

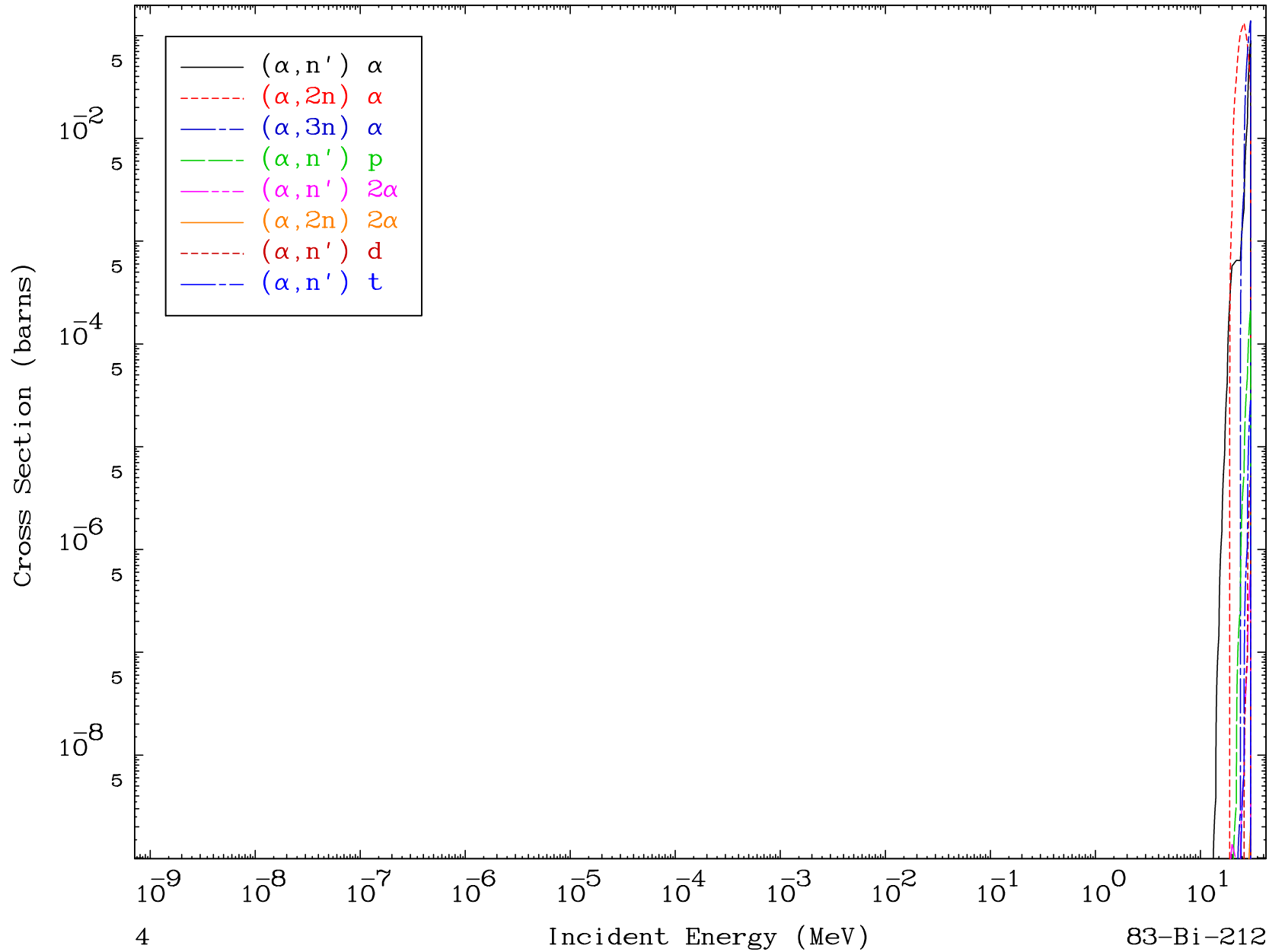


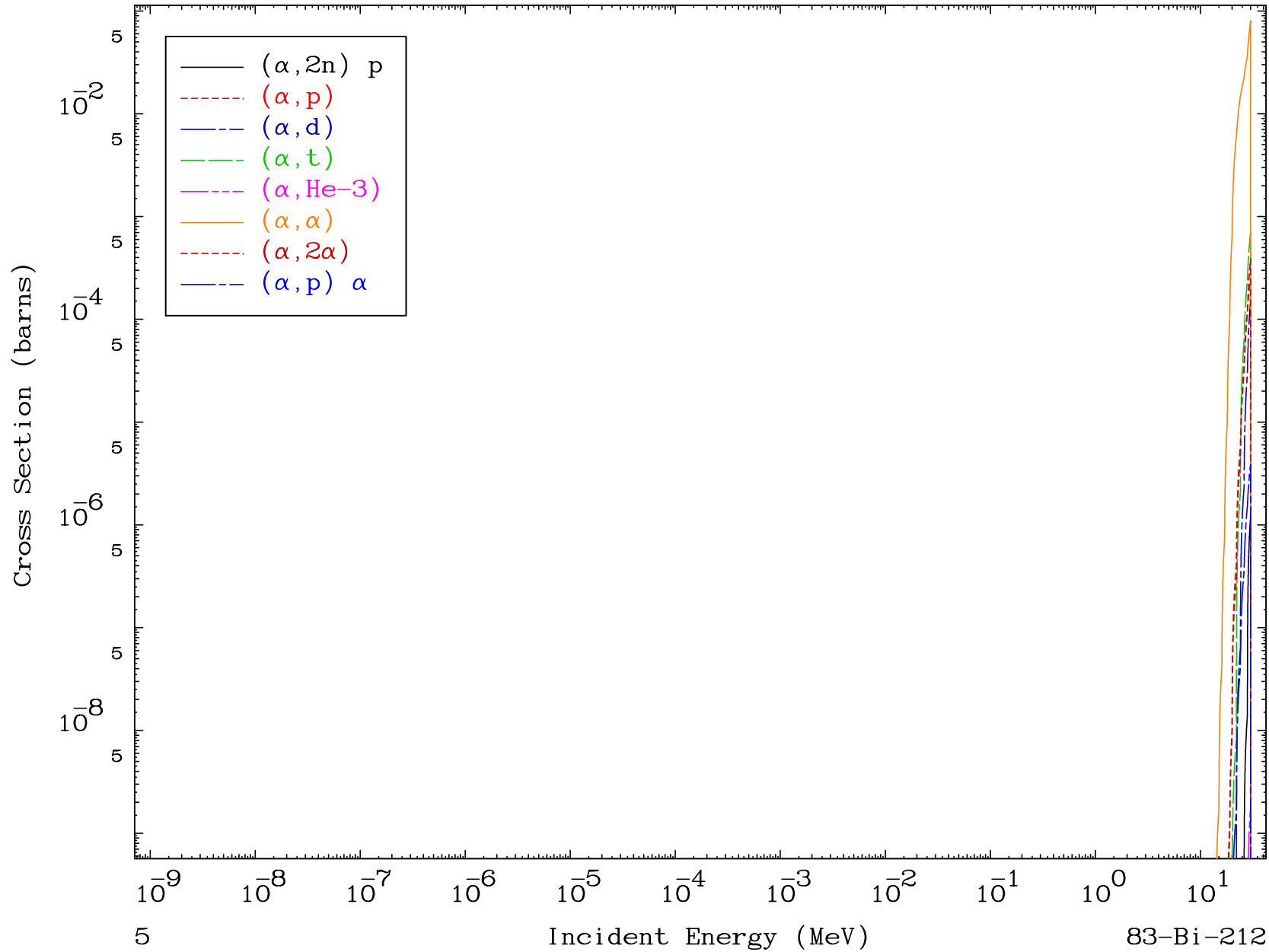
2

Incident Energy (MeV)

83-Bi-212



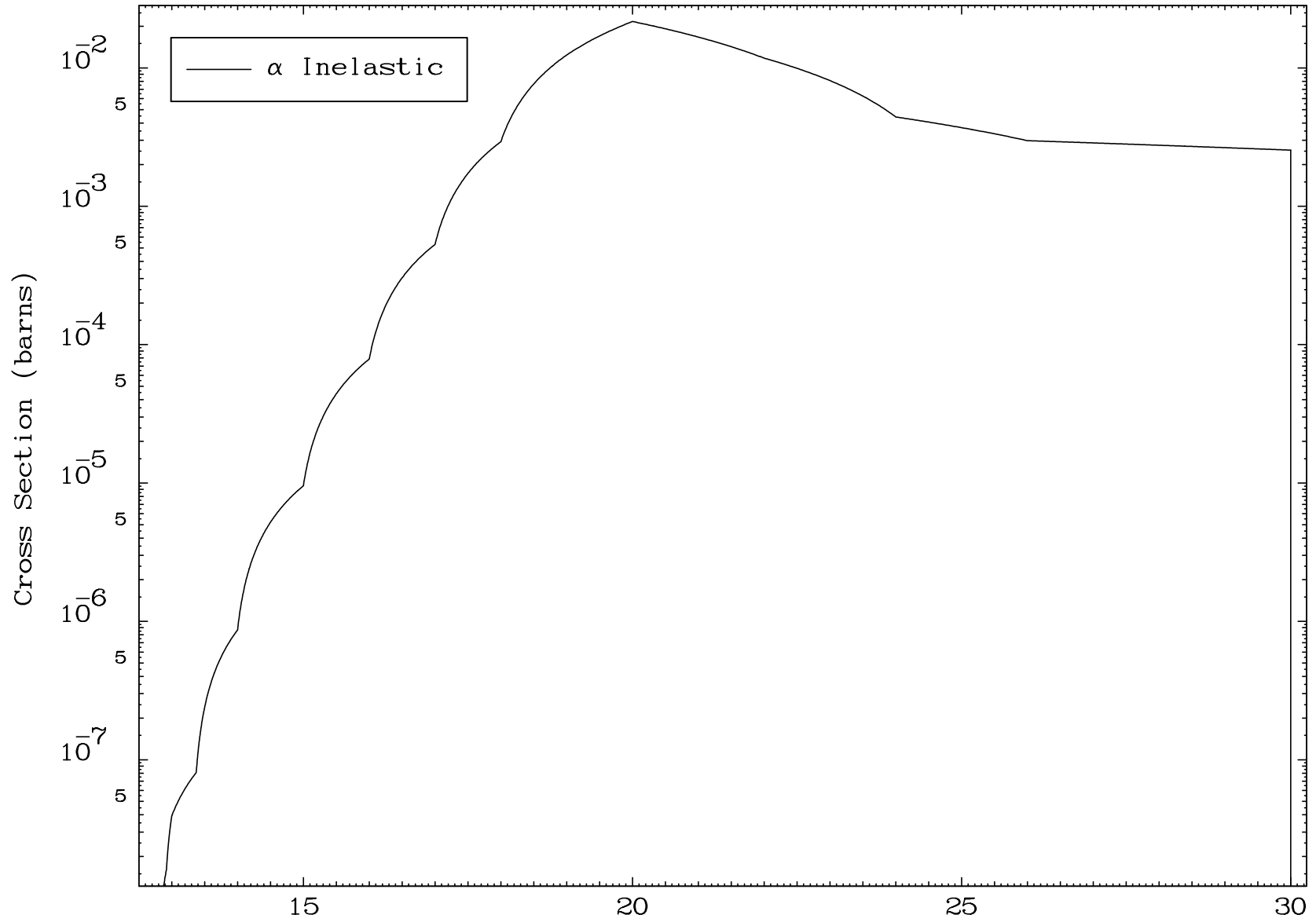




MAT 8334

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

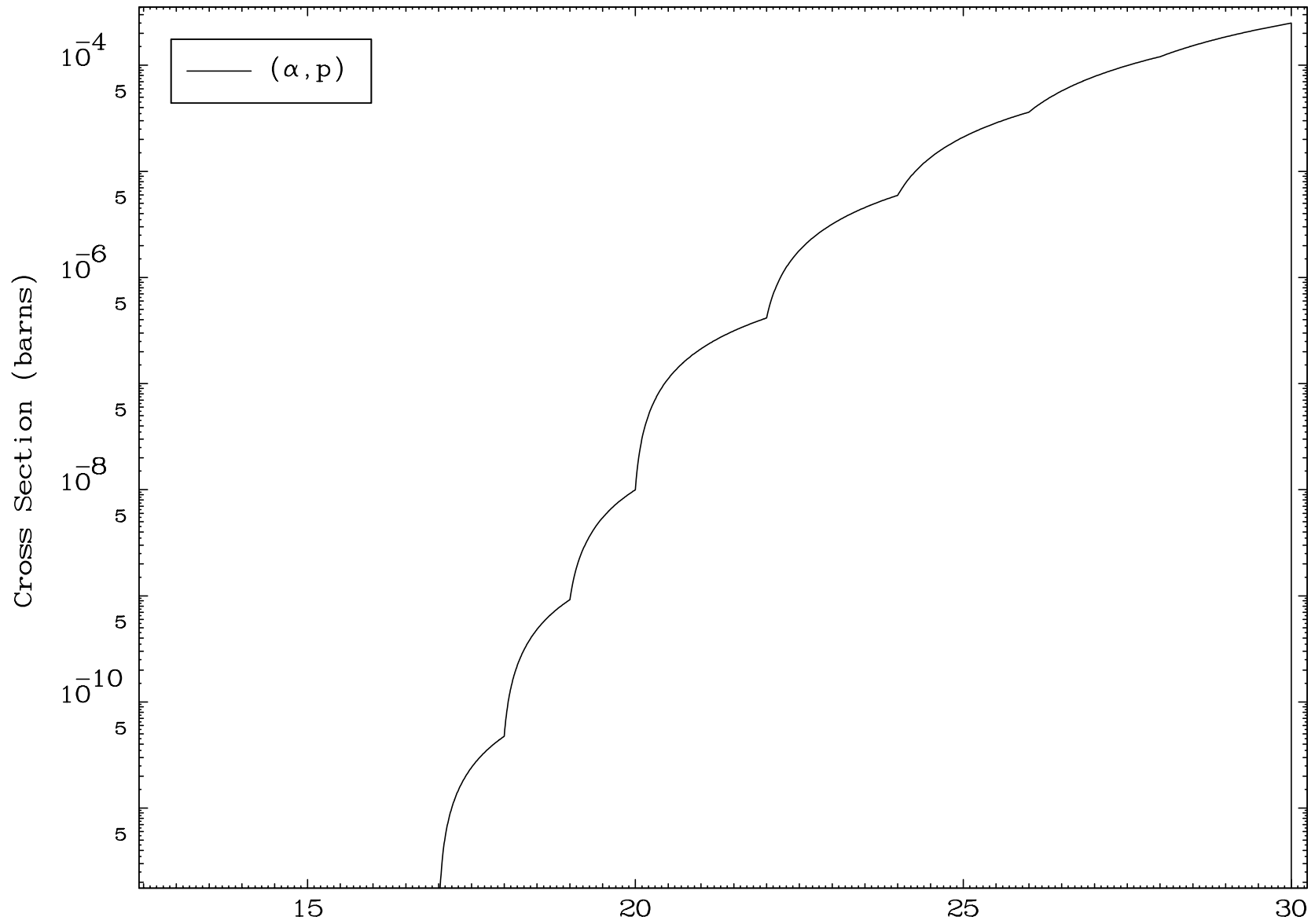
83-Bi-212

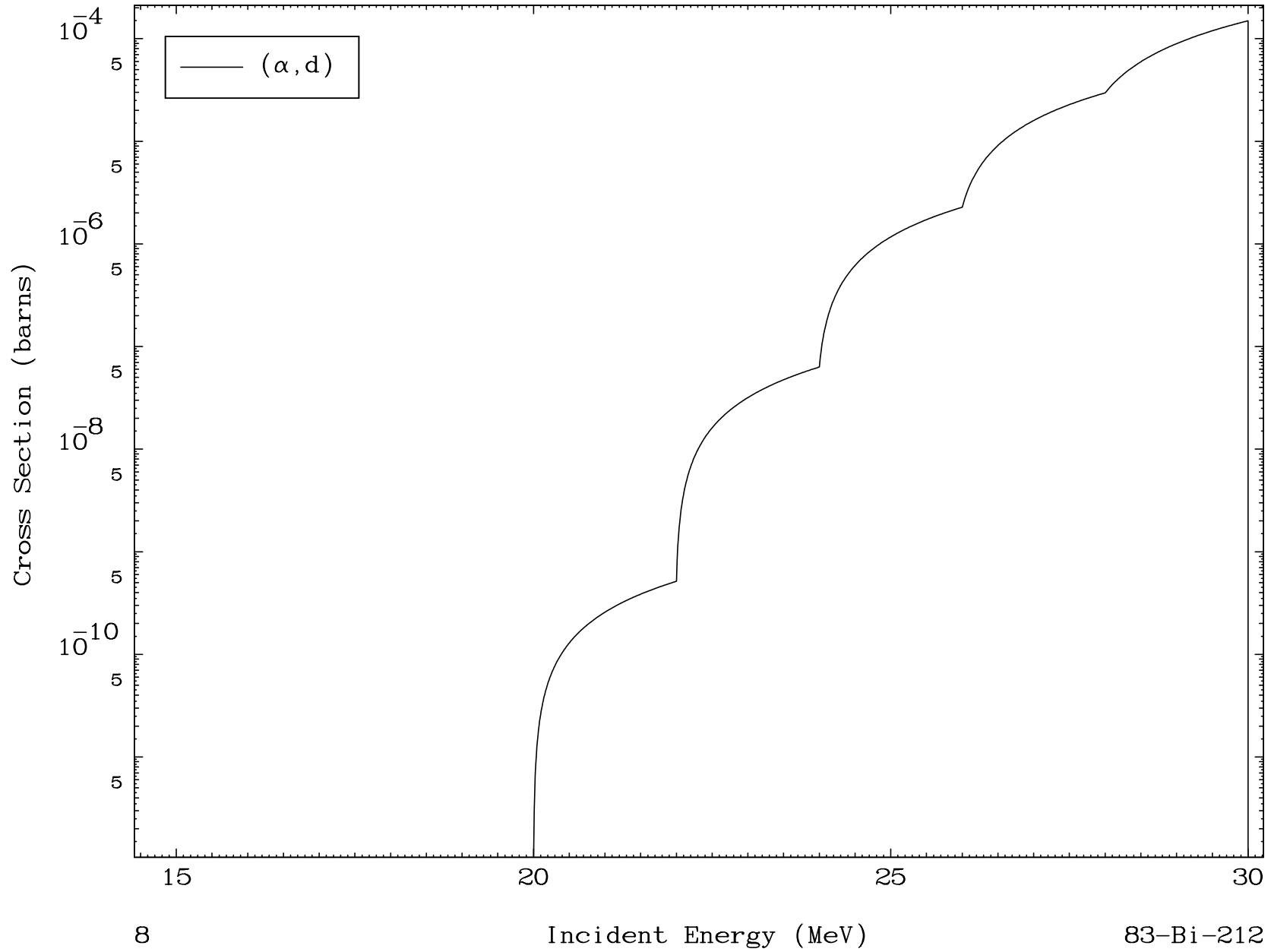


6

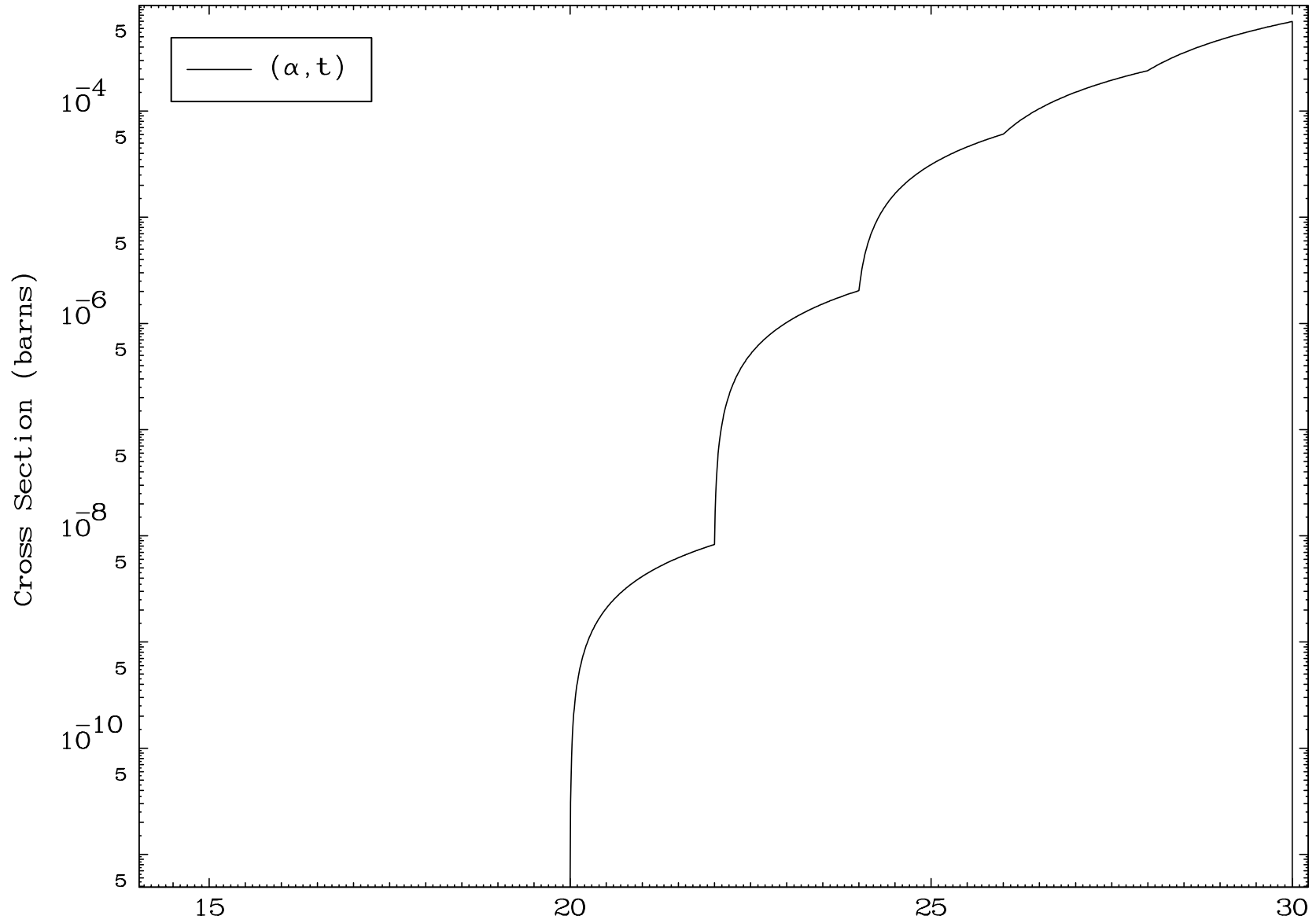
Incident Energy (MeV)

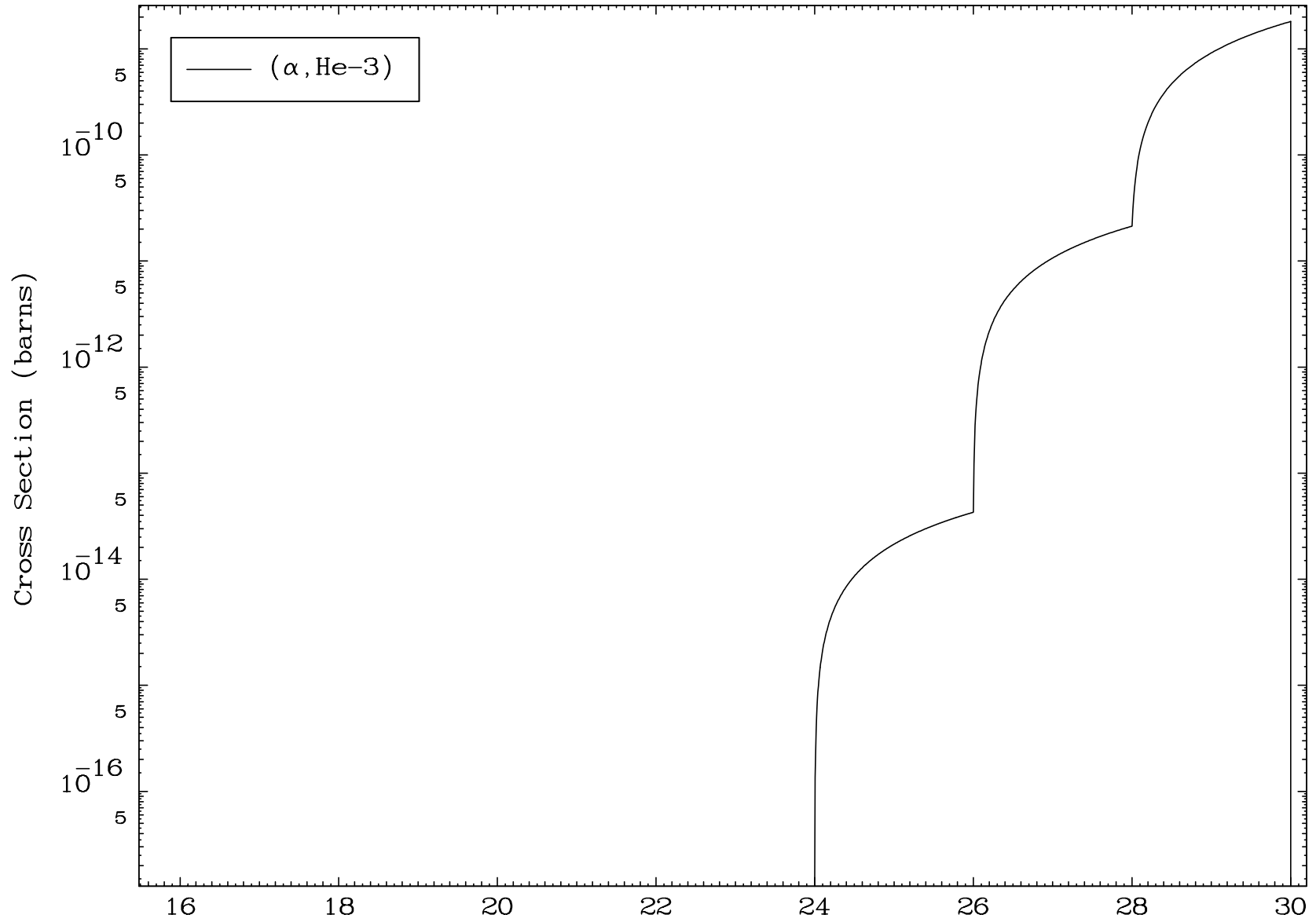
83-Bi-212

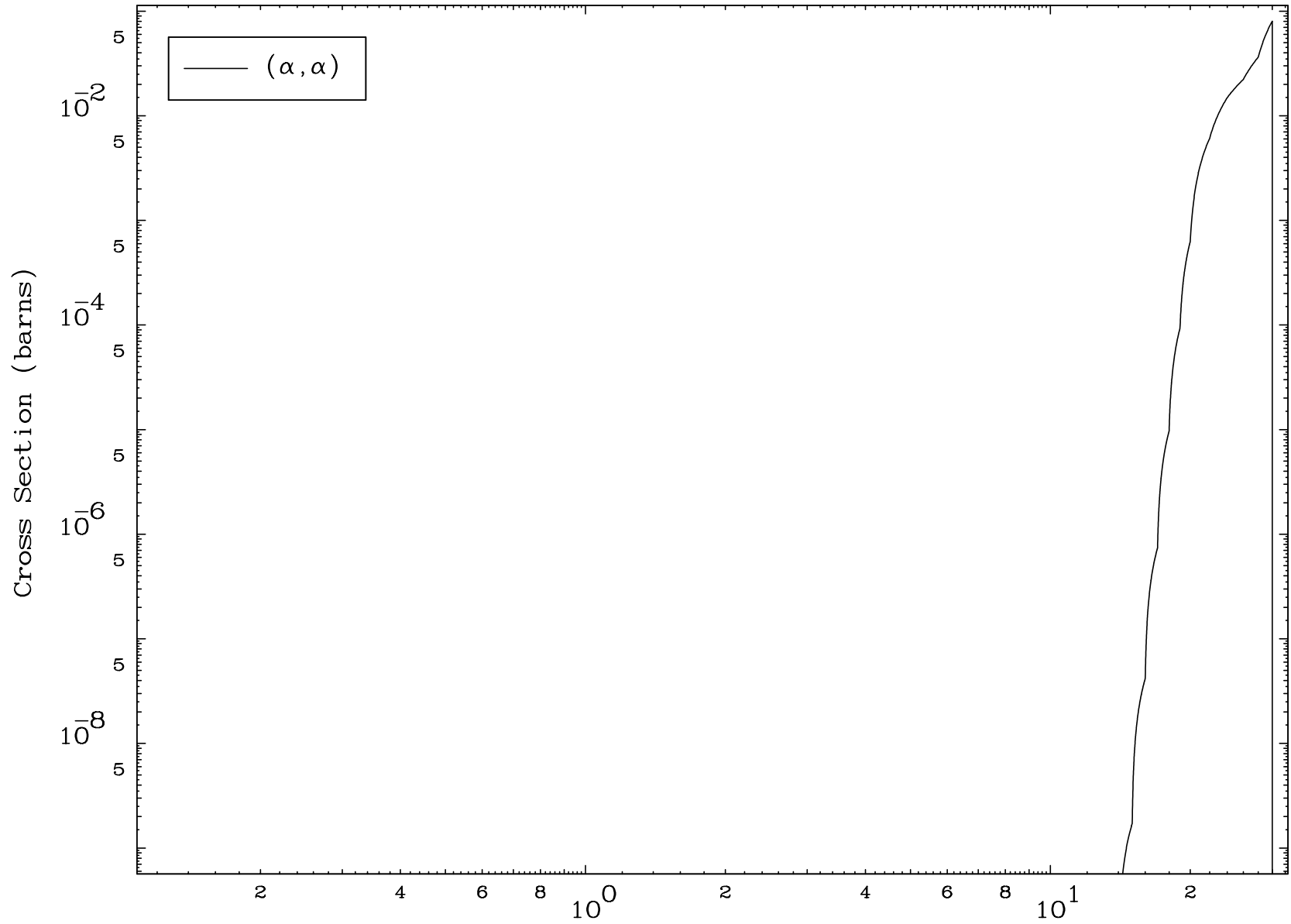




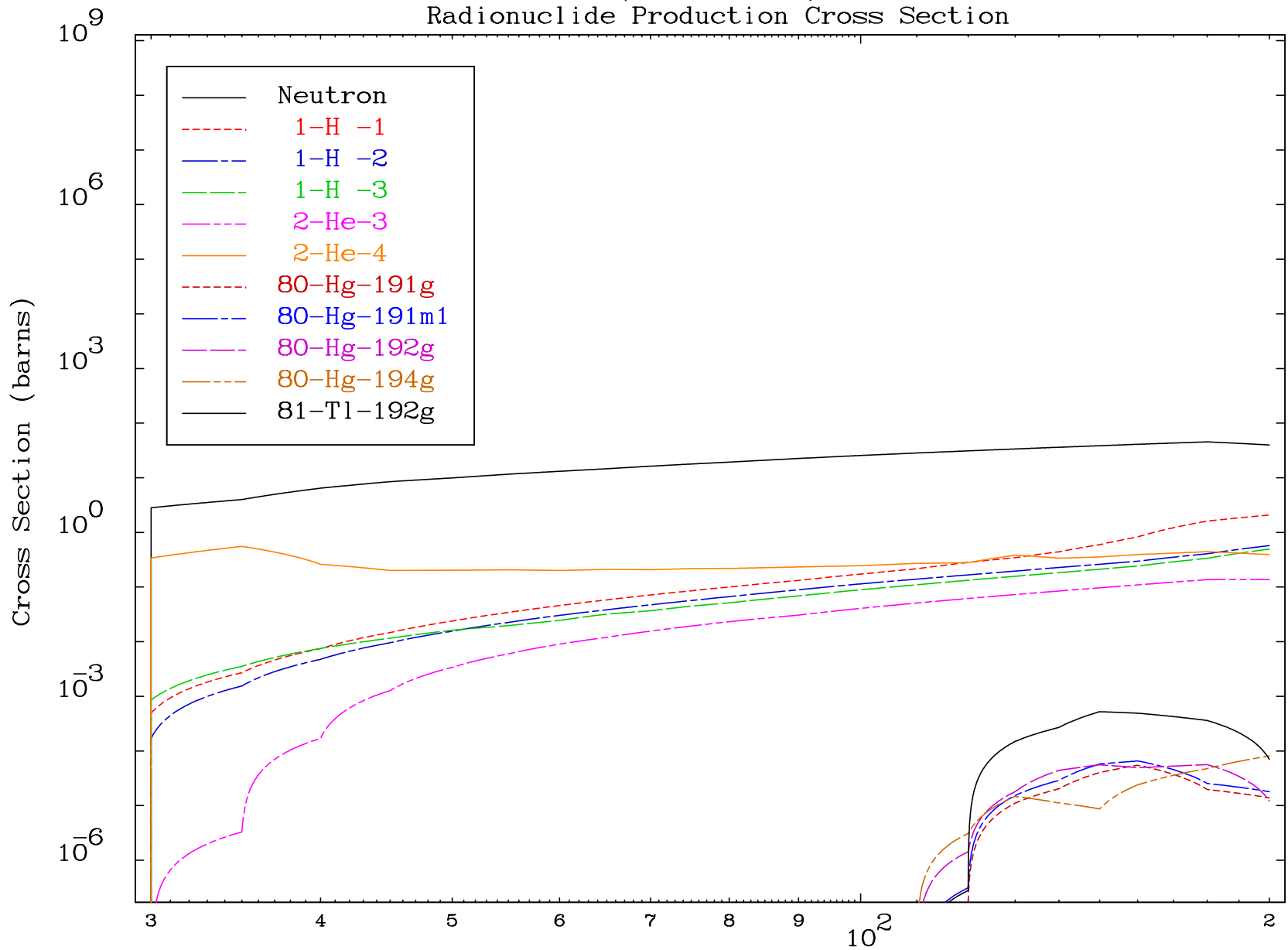




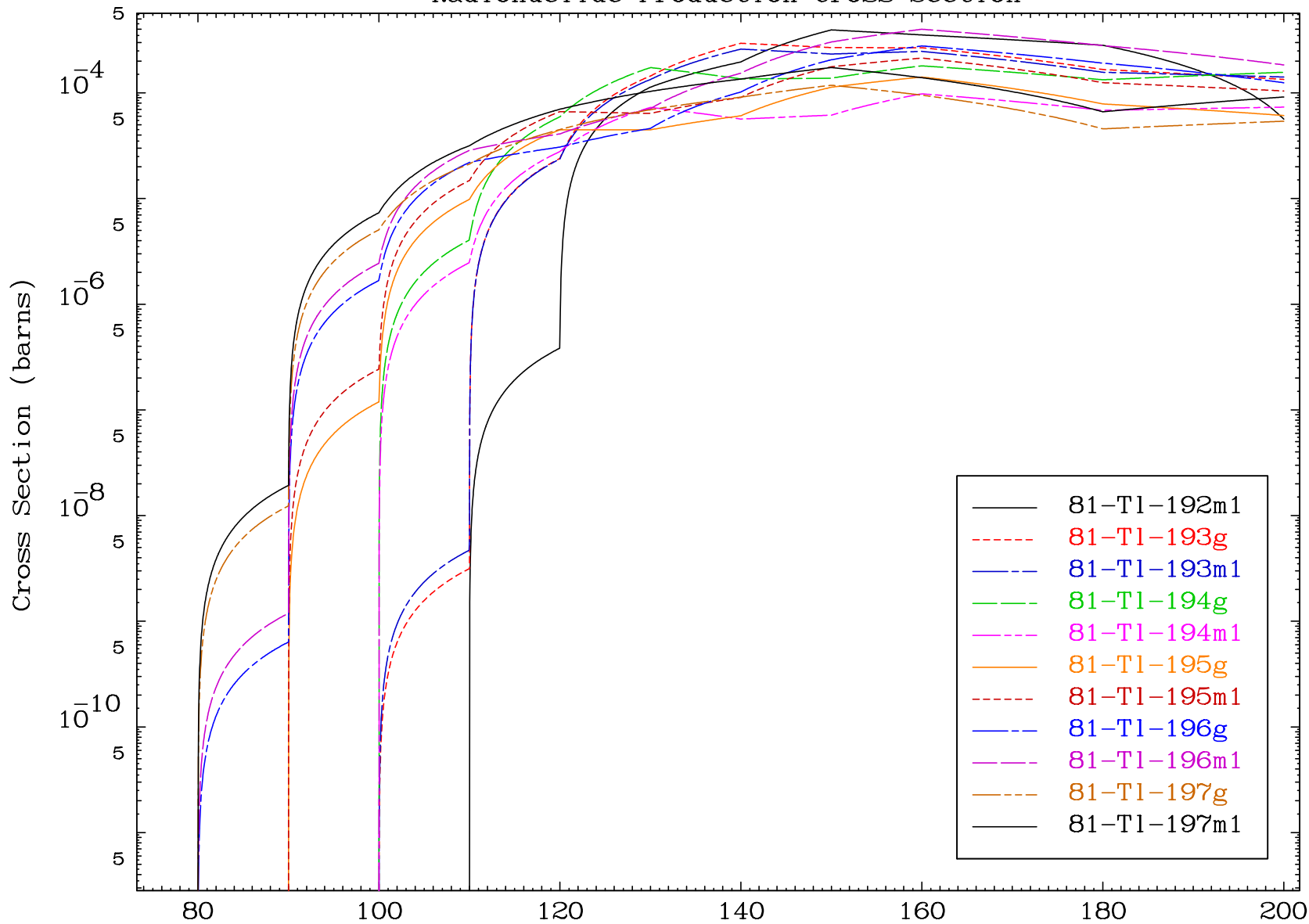


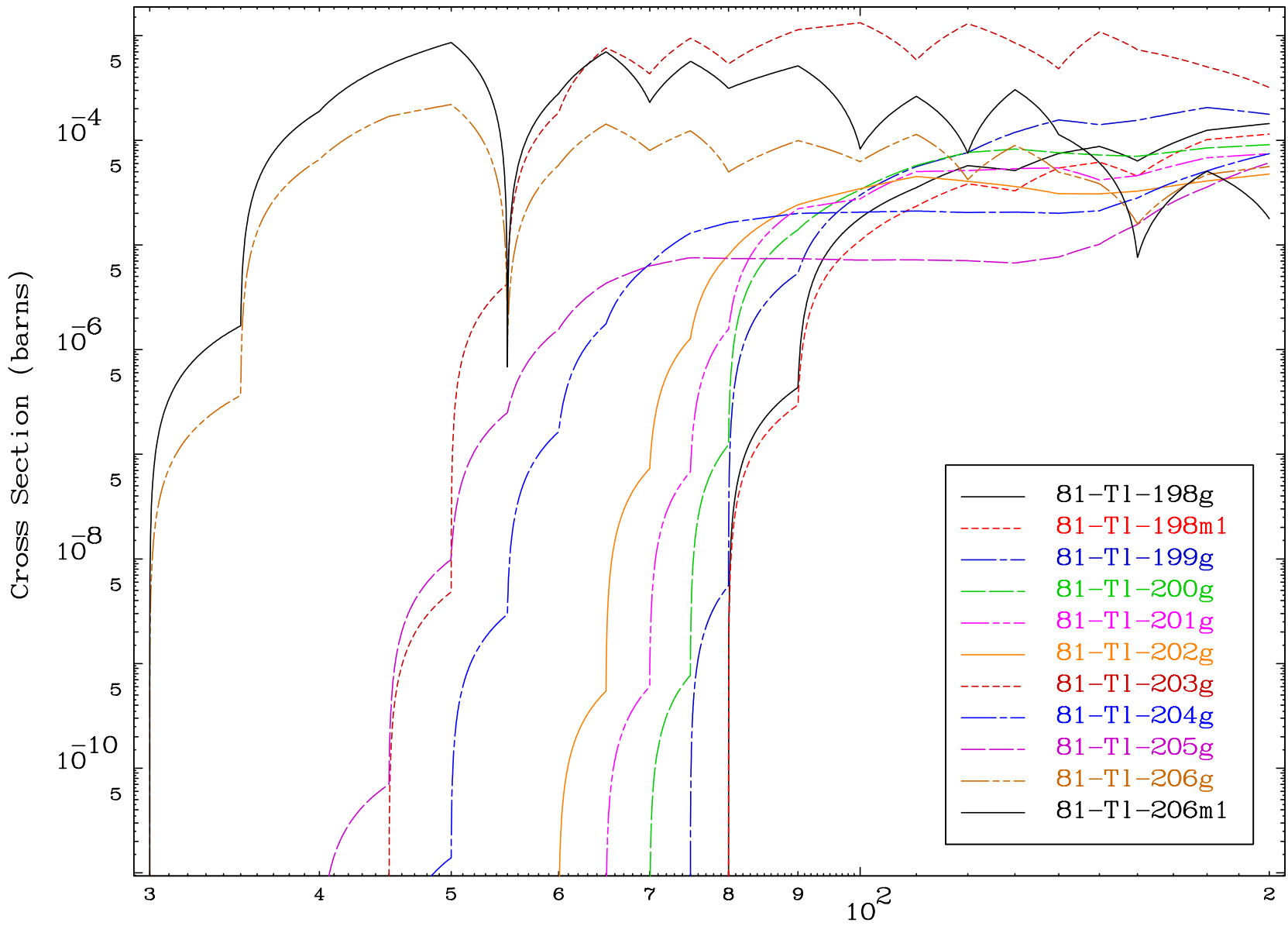


Radionuclide Production Cross Section

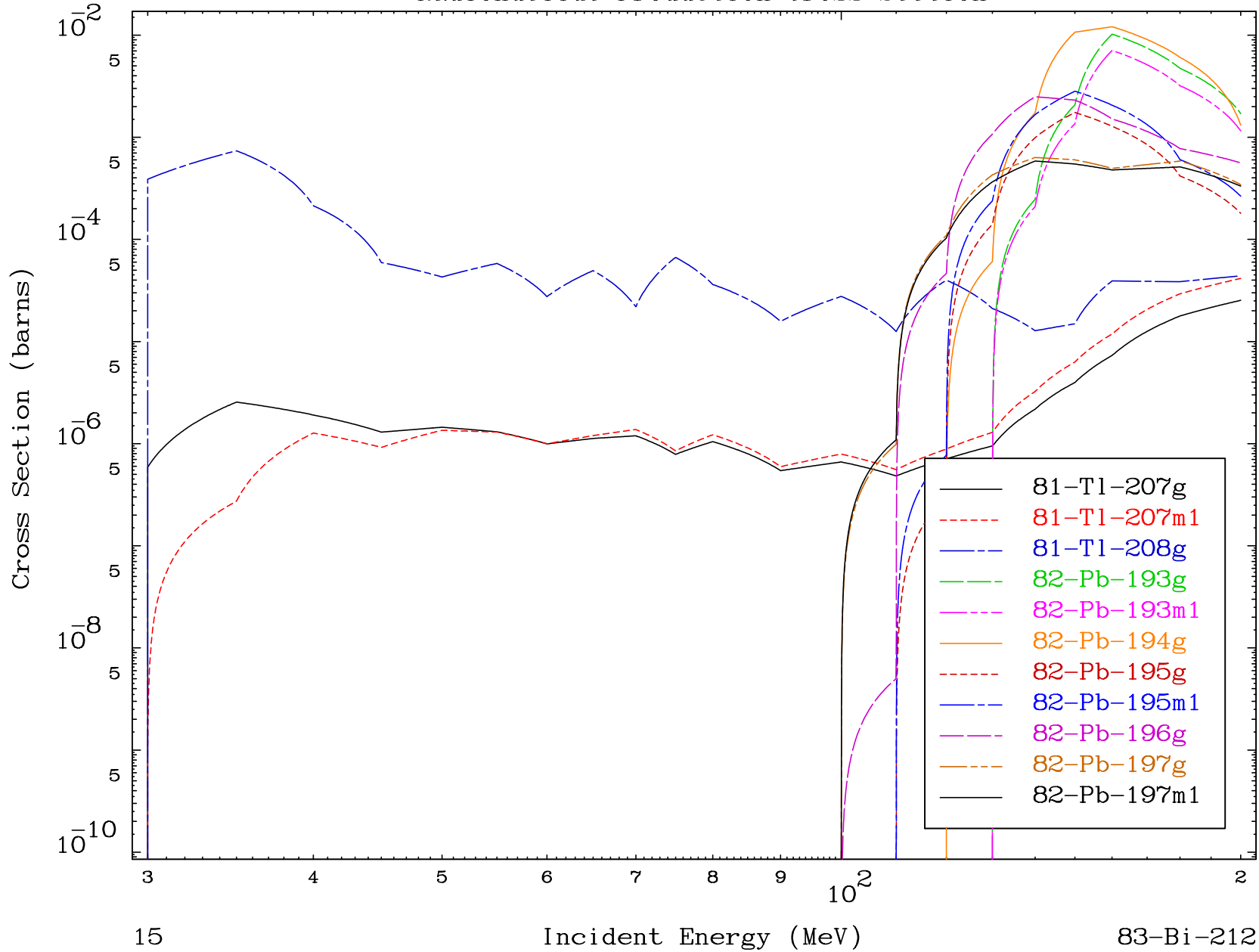


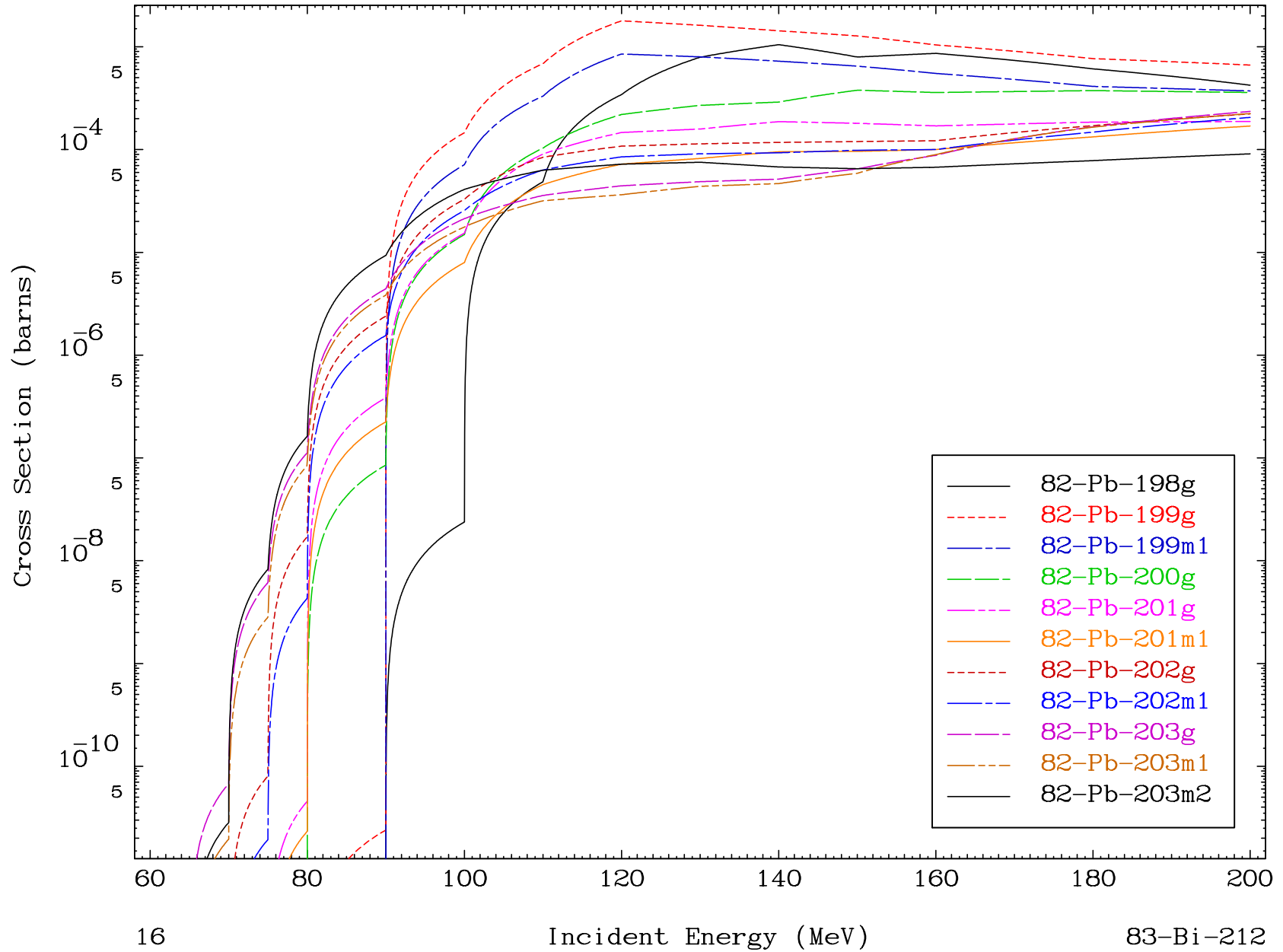
Radionuclide Production Cross Section



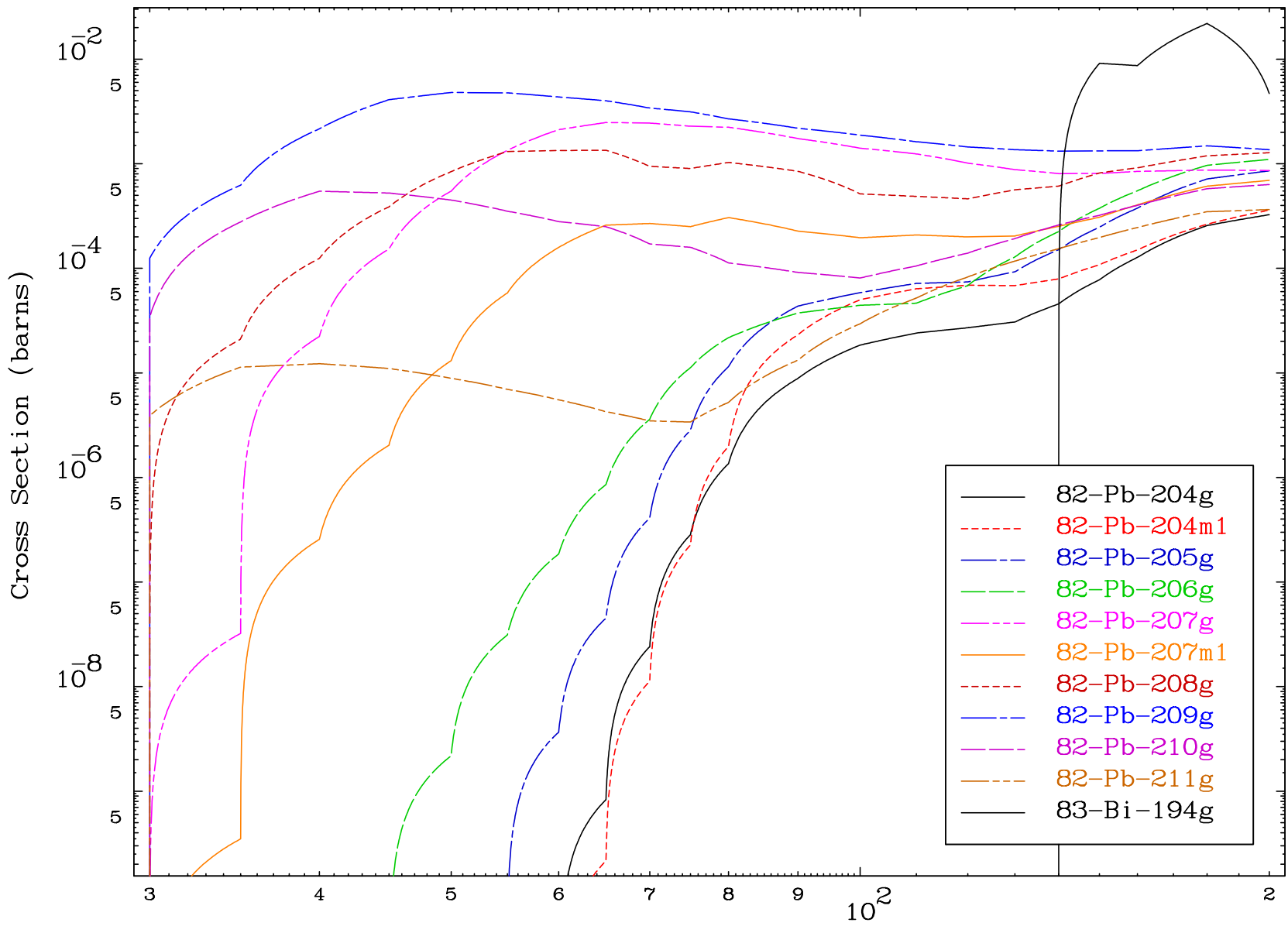


Radionuclide Production Cross Section







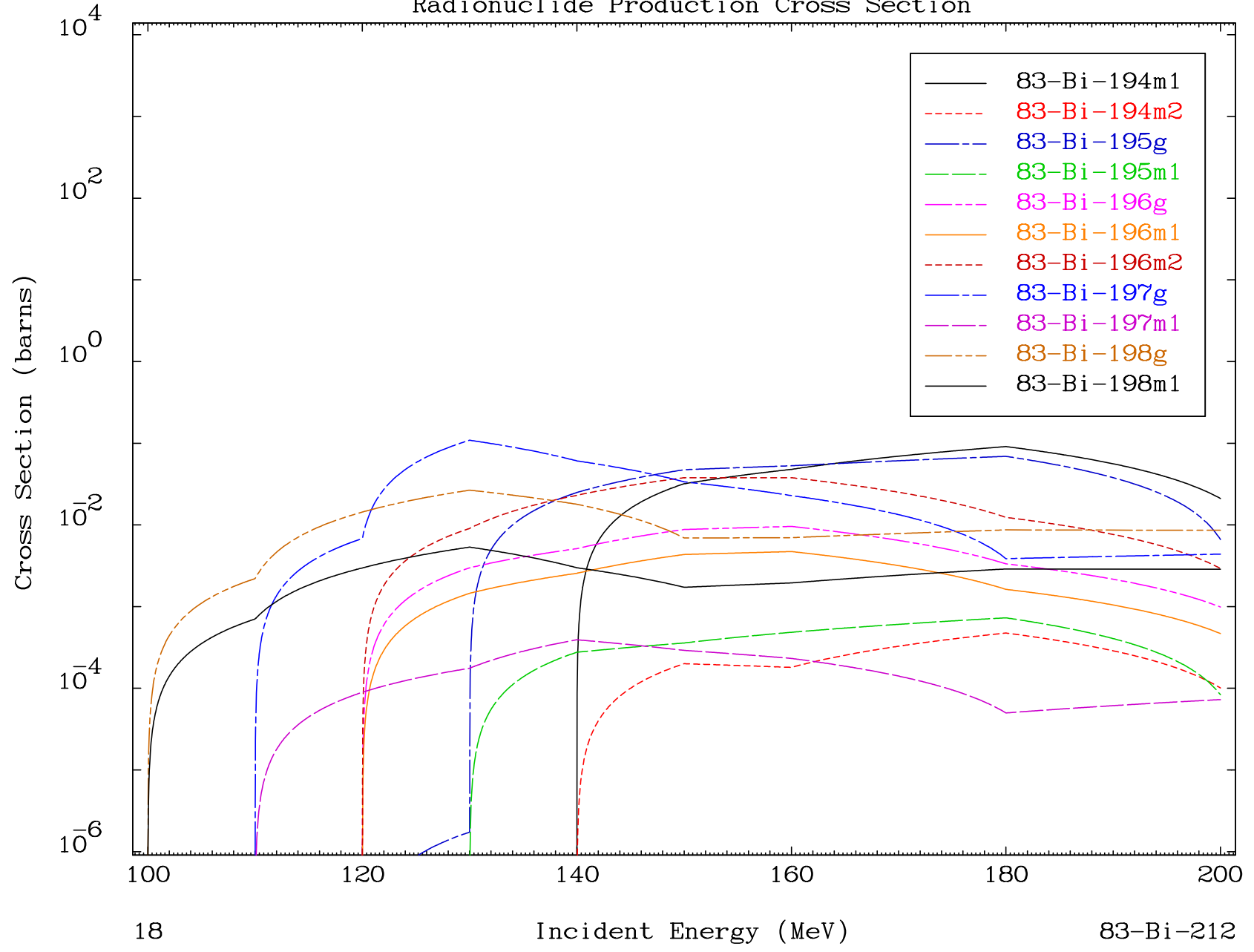


MAT 8334

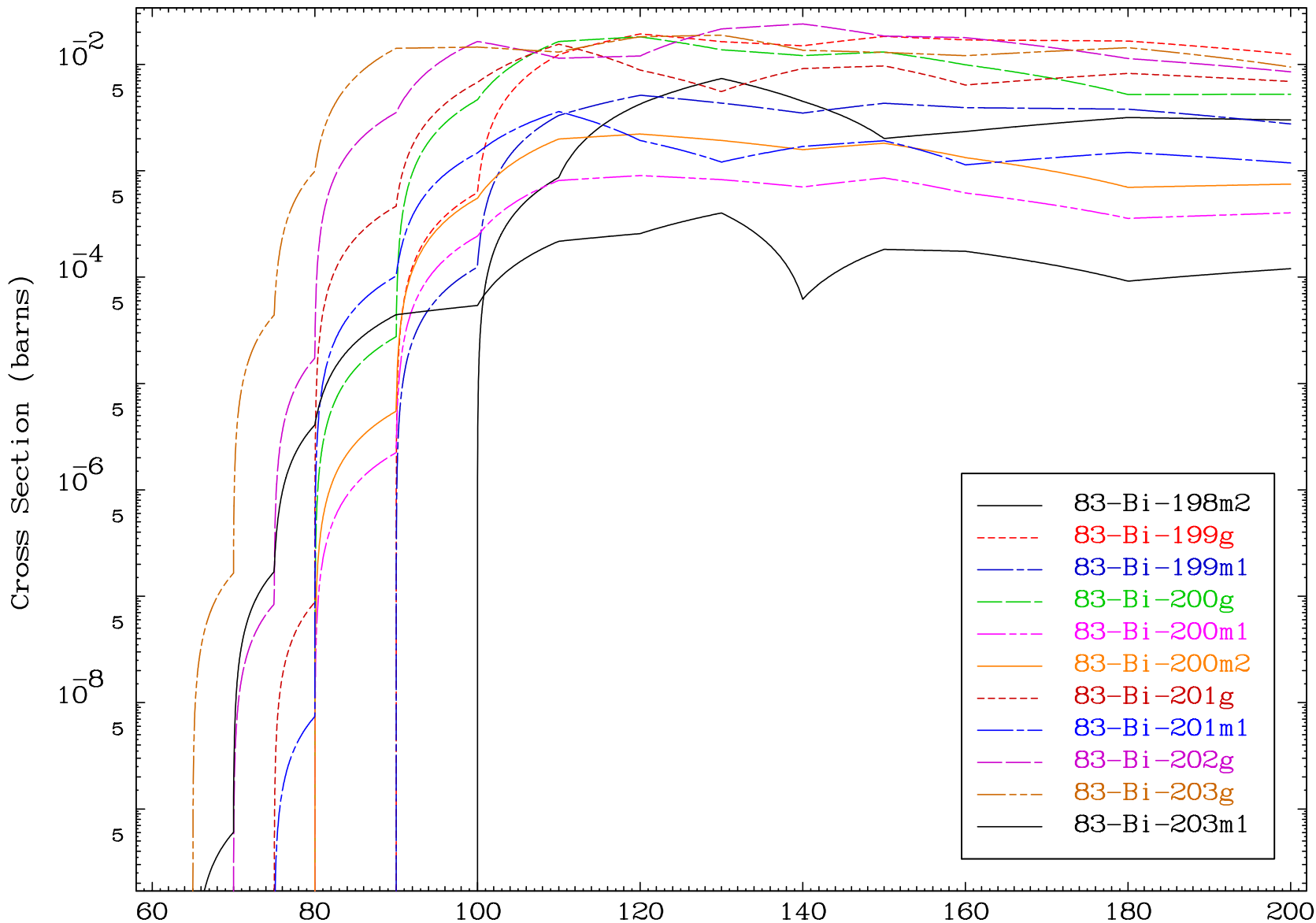
( $\alpha$ , remainder)

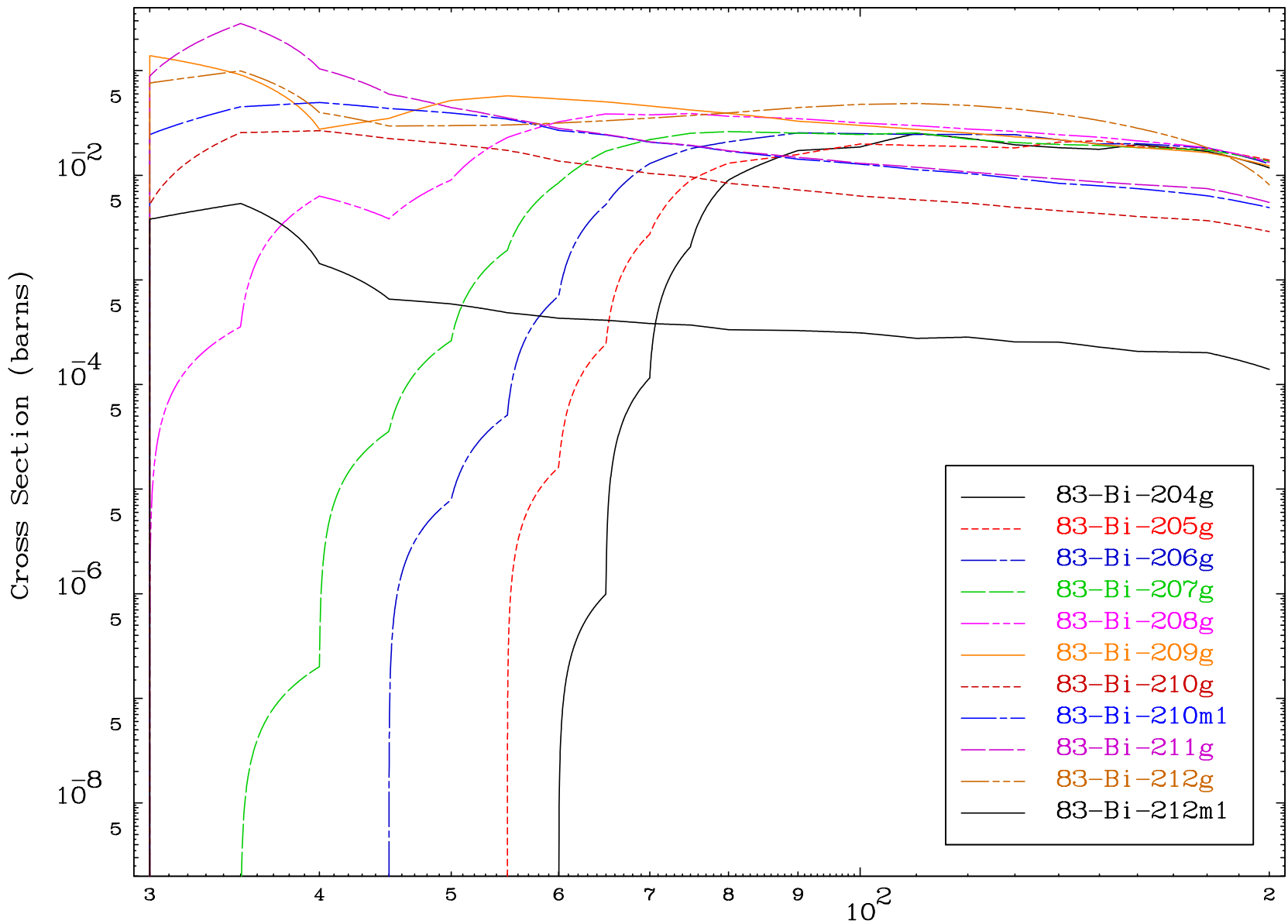
83-Bi-212

Radionuclide Production Cross Section

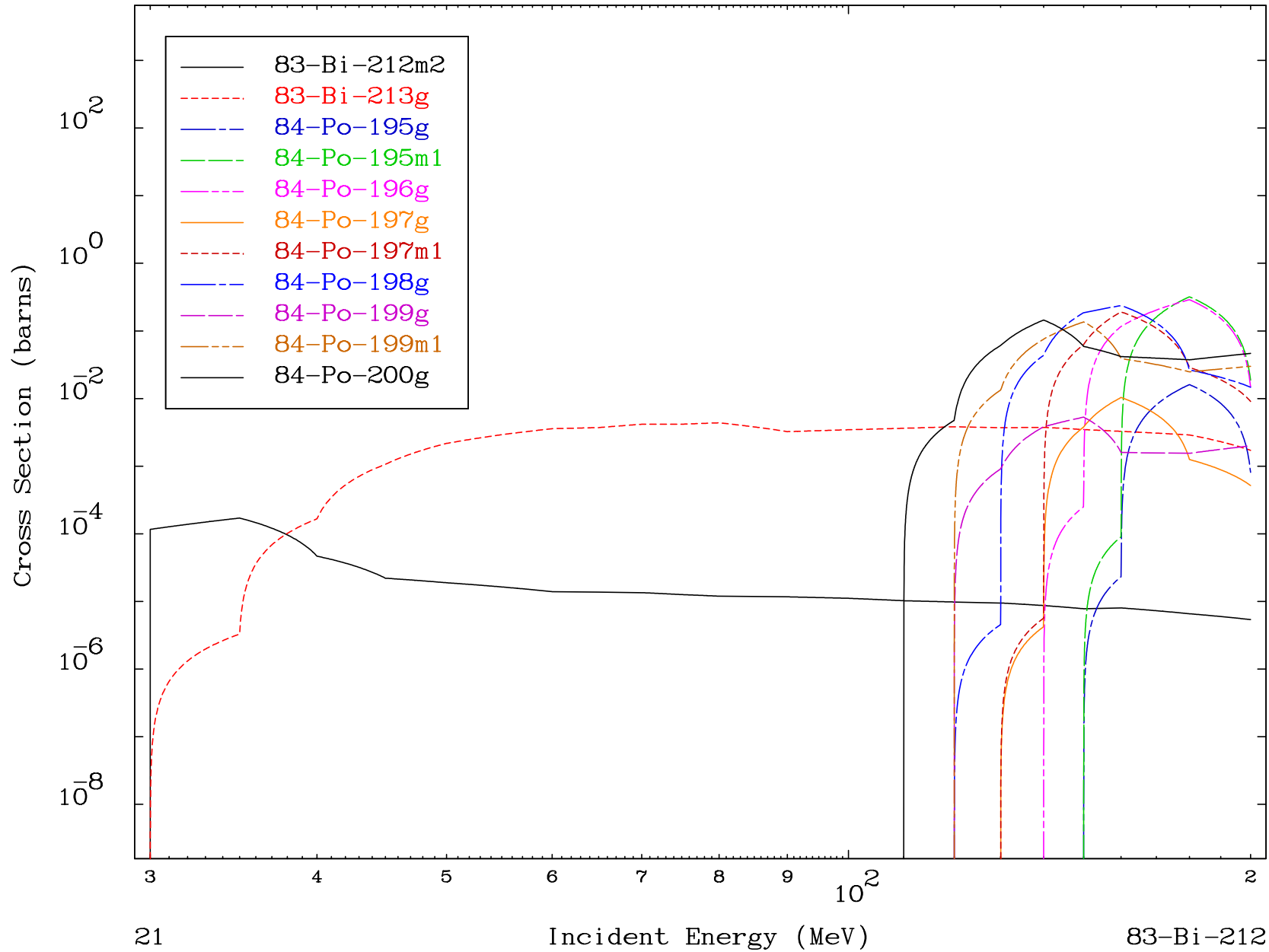


Radionuclide Production Cross Section

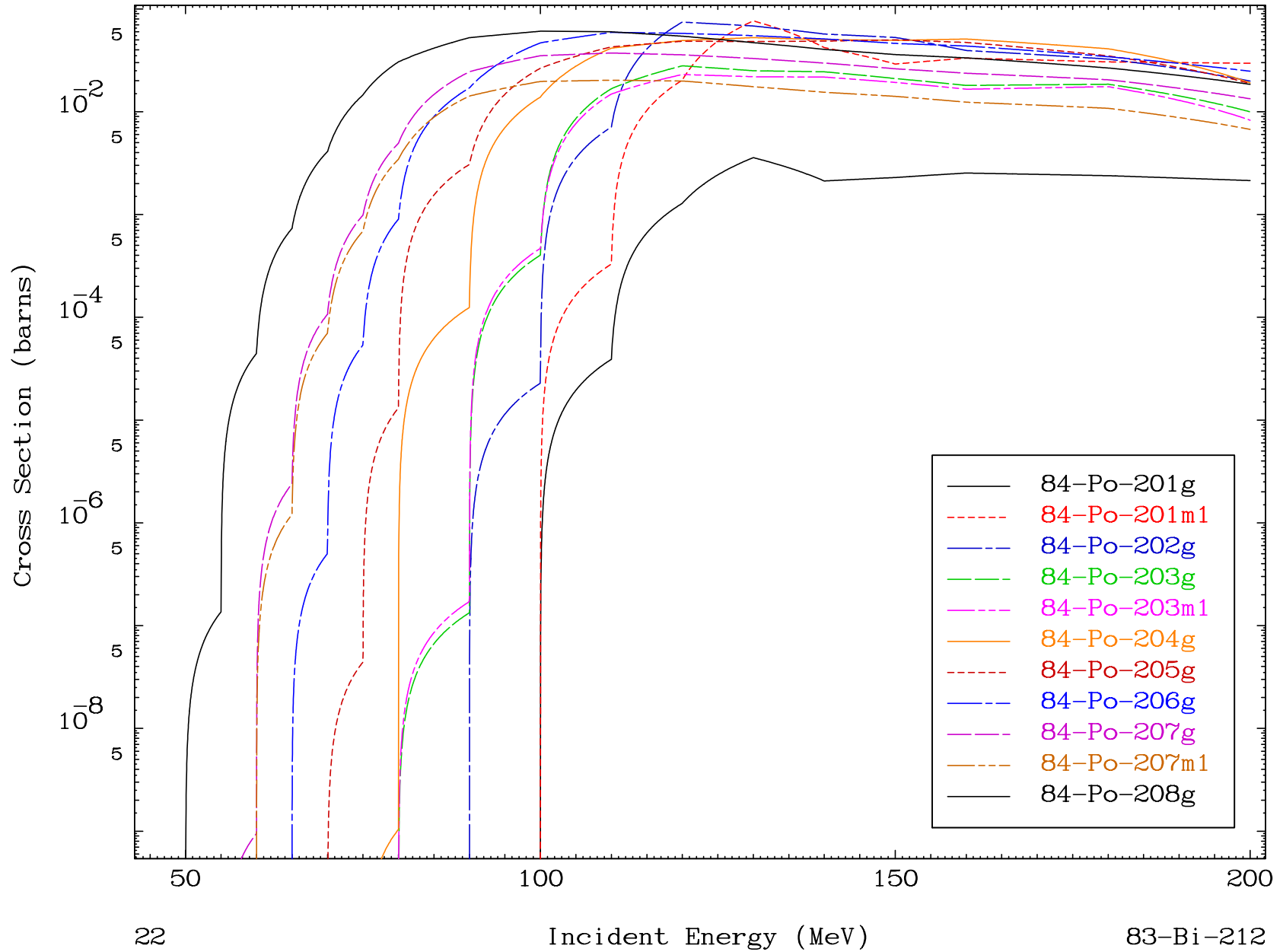




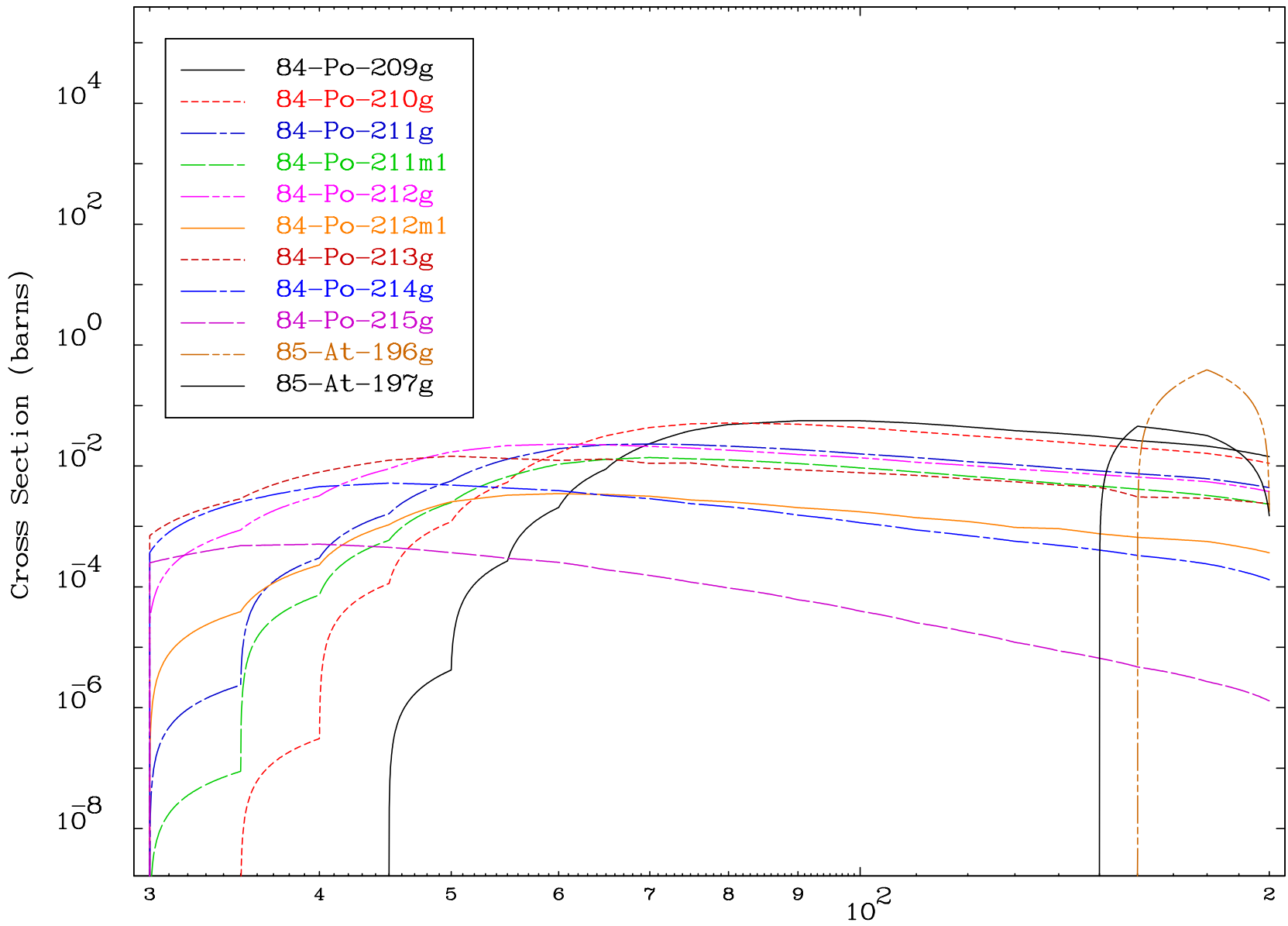
Radionuclide Production Cross Section

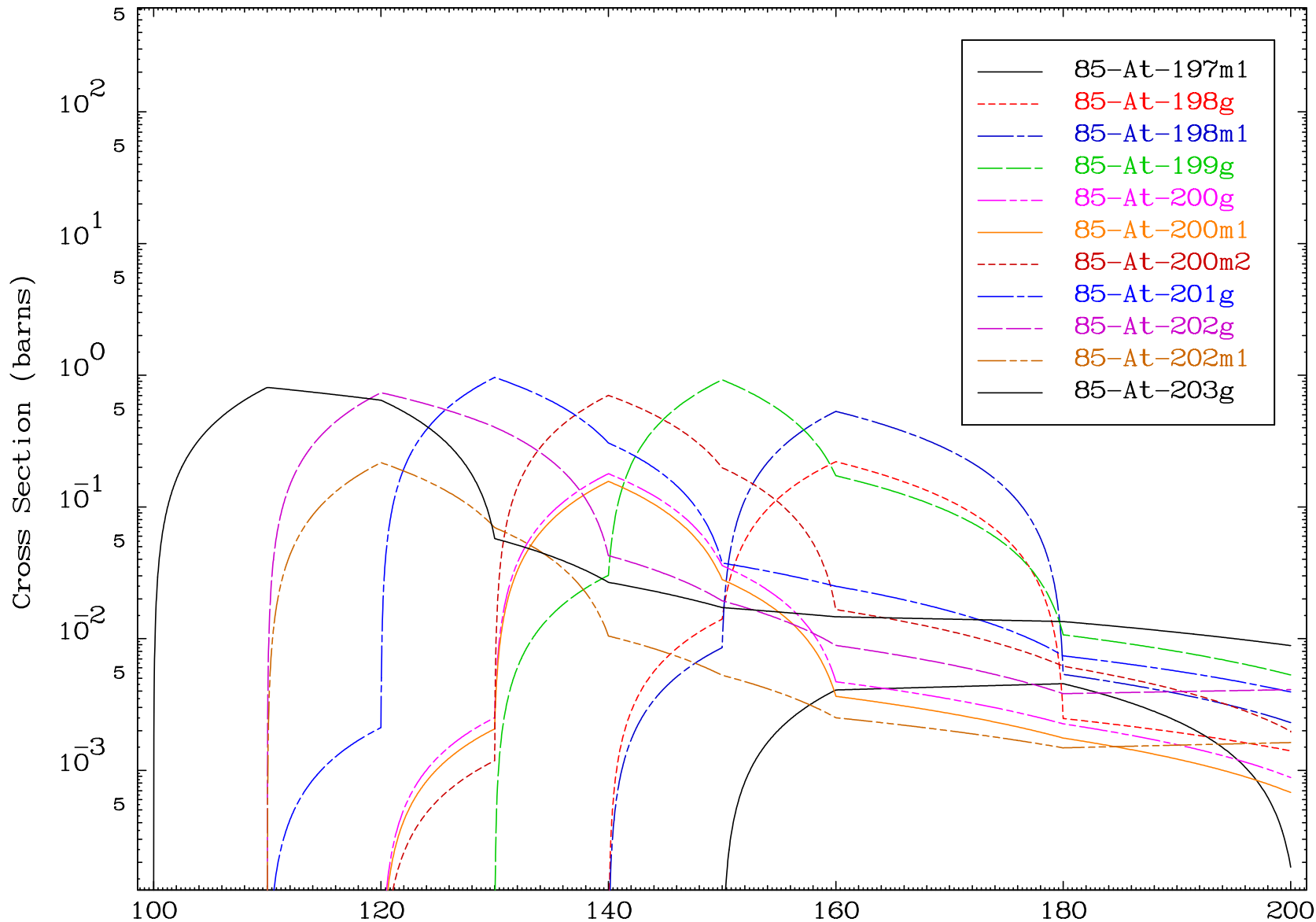


Radionuclide Production Cross Section

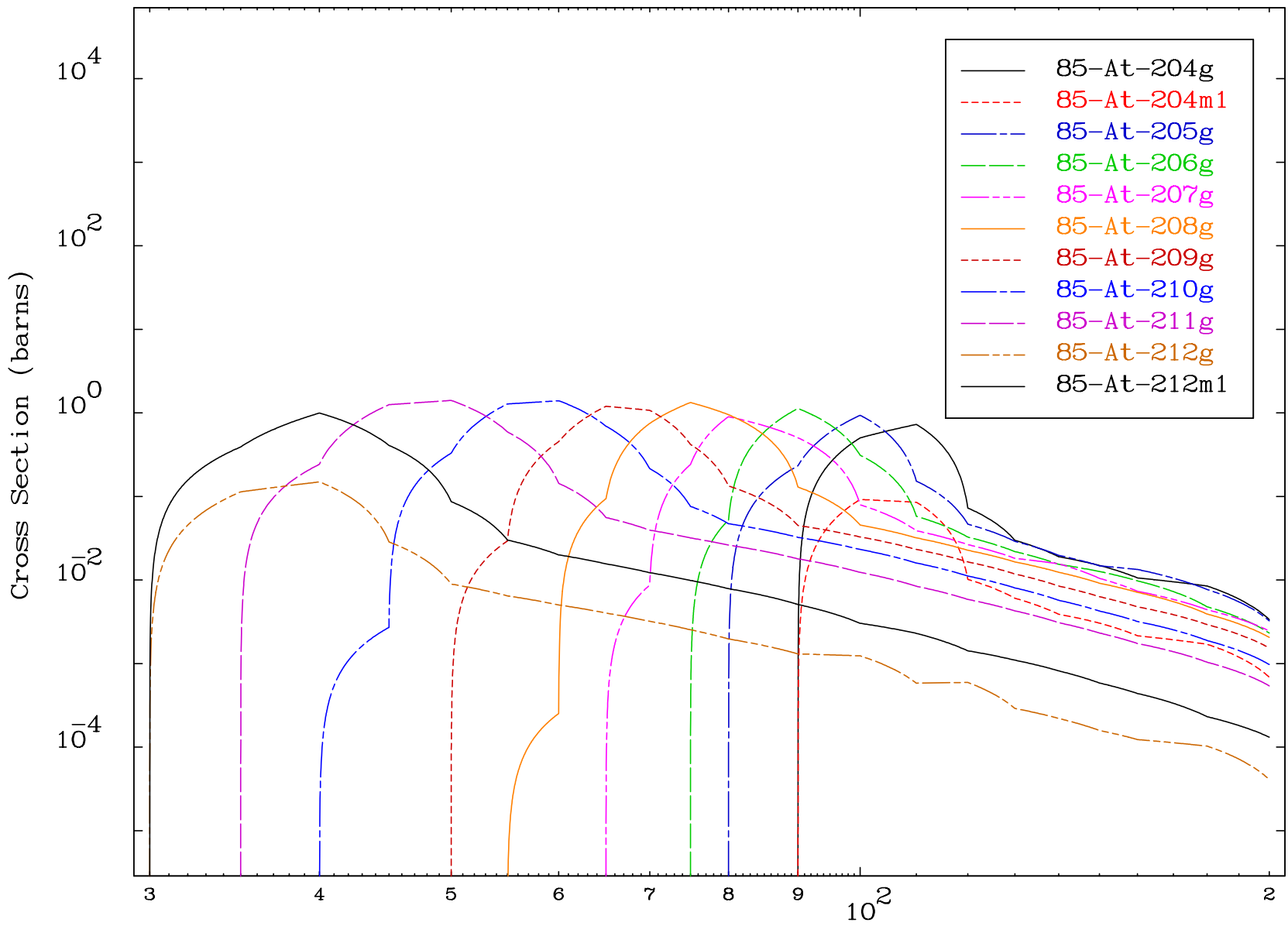


Radionuclide Production Cross Section







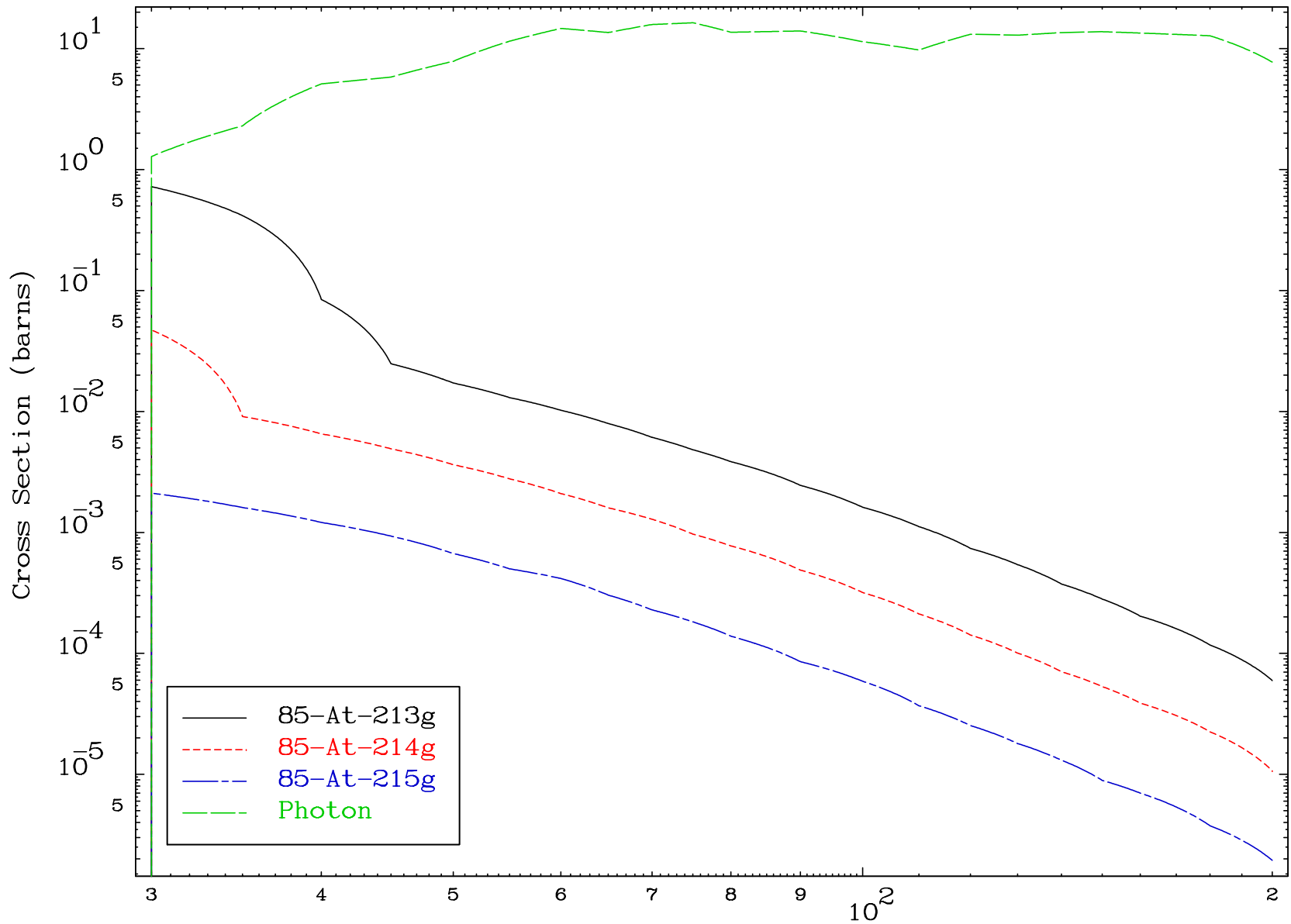


MAT 8334

( $\alpha$ , remainder)

83-Bi-212

Radionuclide Production Cross Section



26

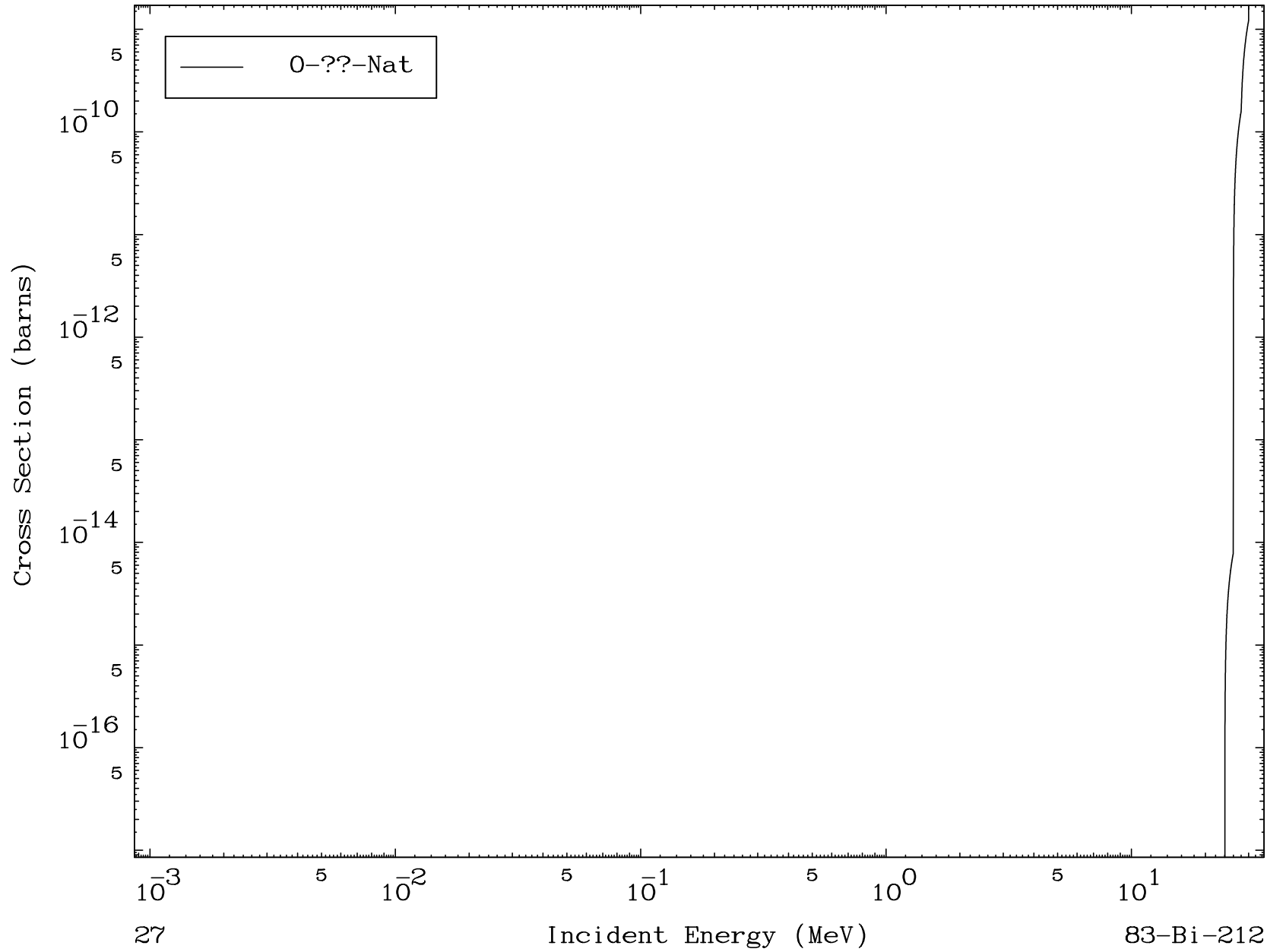
Incident Energy (MeV)

83-Bi-212

MAT 8334

$\alpha$  Fission  
Radionuclide Production Cross Section

83-Bi-212

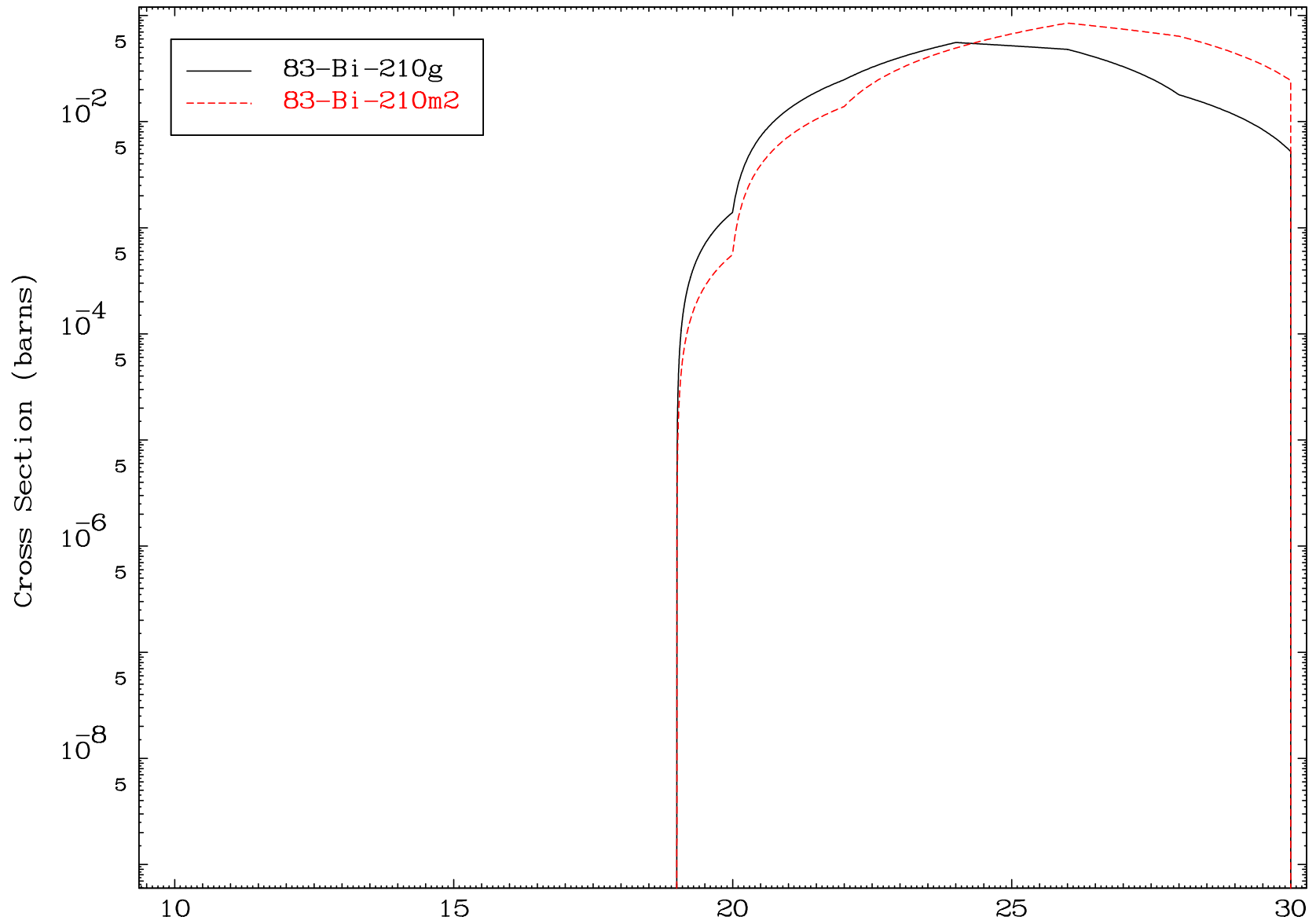


MAT 8334

( $\alpha, 2n$ )  $\alpha$

83-Bi-212

Radionuclide Production Cross Section



28

Incident Energy (MeV)

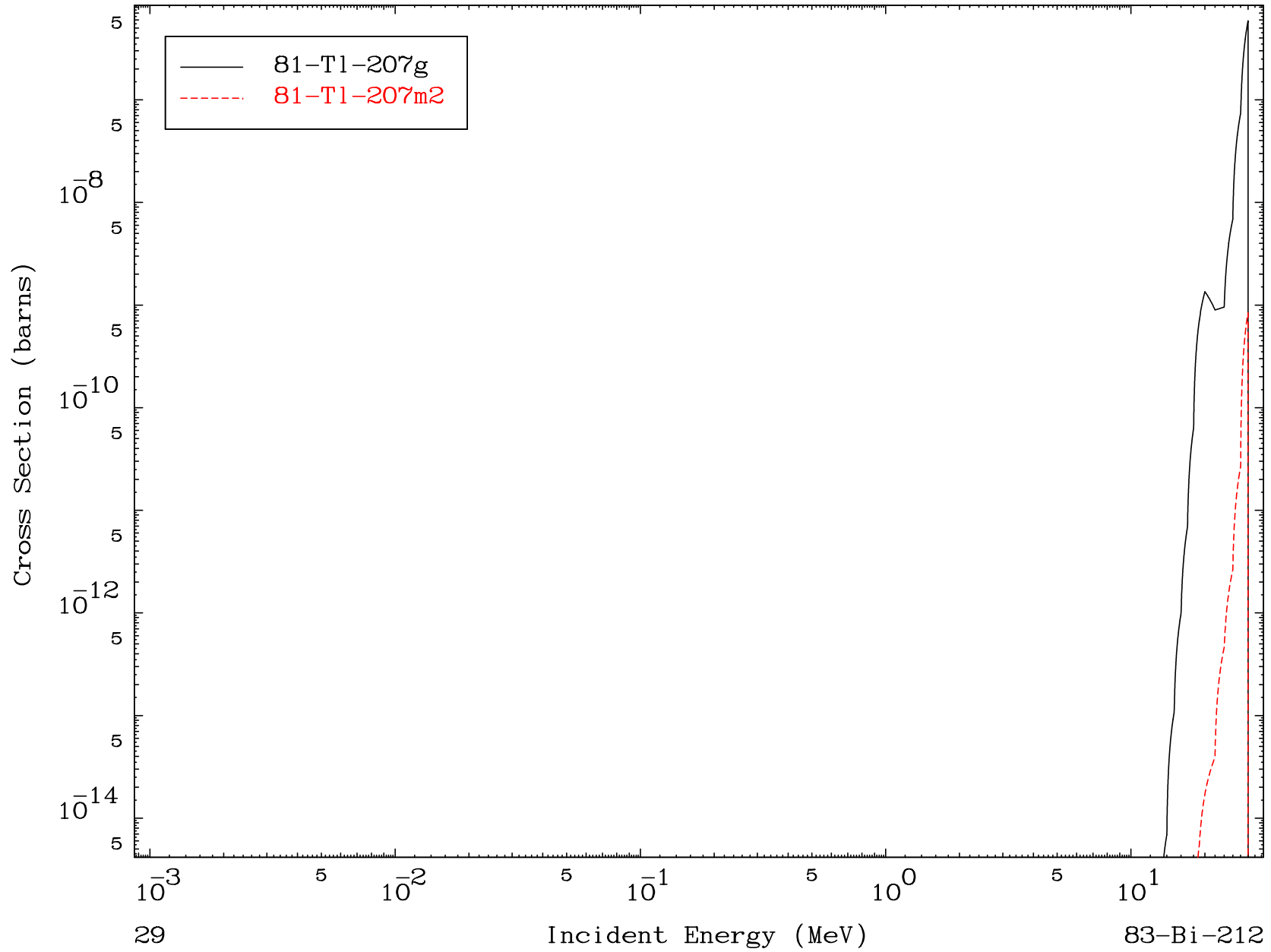
83-Bi-212

MAT 8334

( $\alpha, n'$ )  $2\alpha$

83-Bi-212

Radionuclide Production Cross Section

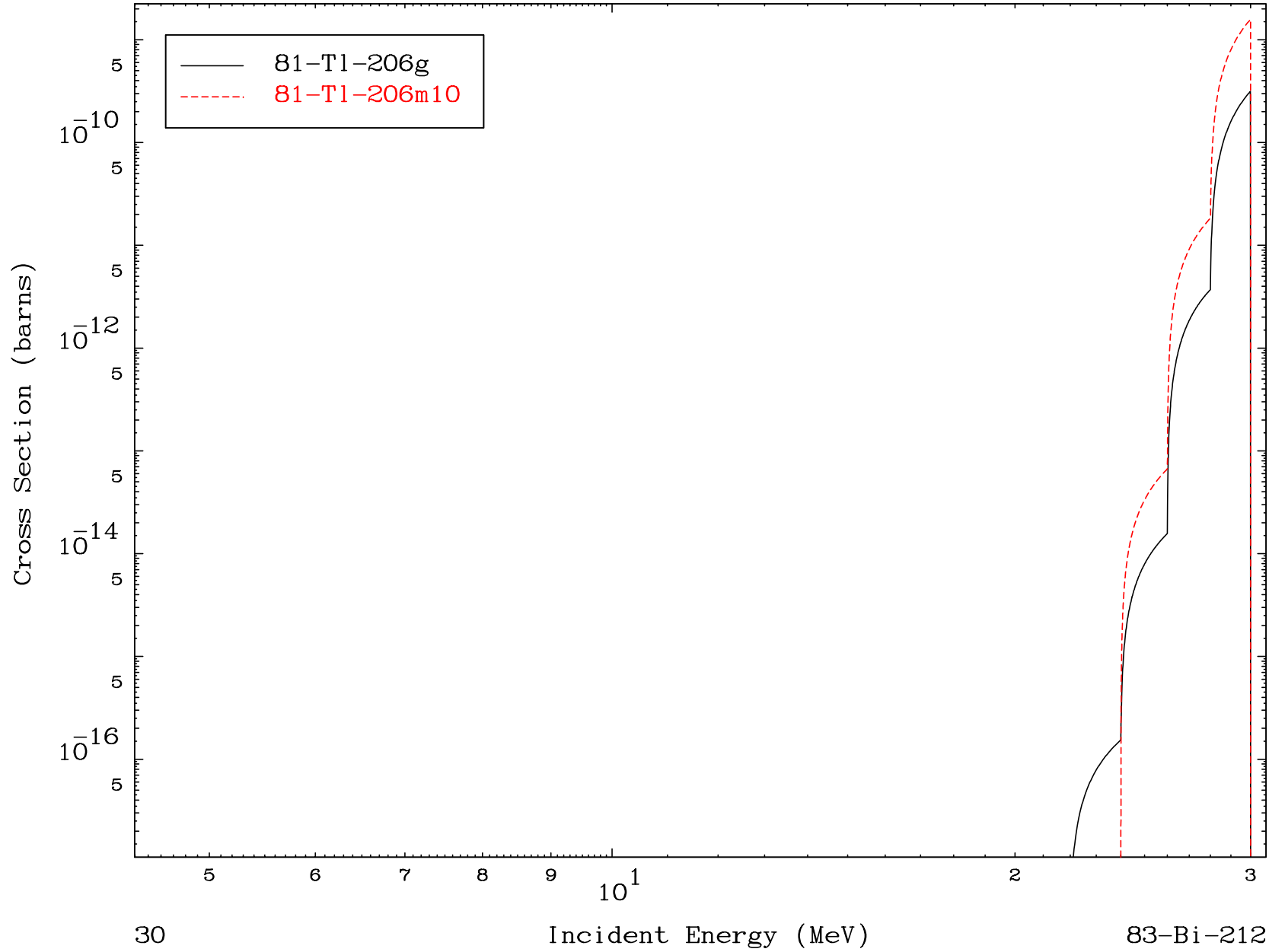


MAT 8334

( $\alpha, 2n$ )  $2\alpha$

83-Bi-212

Radionuclide Production Cross Section

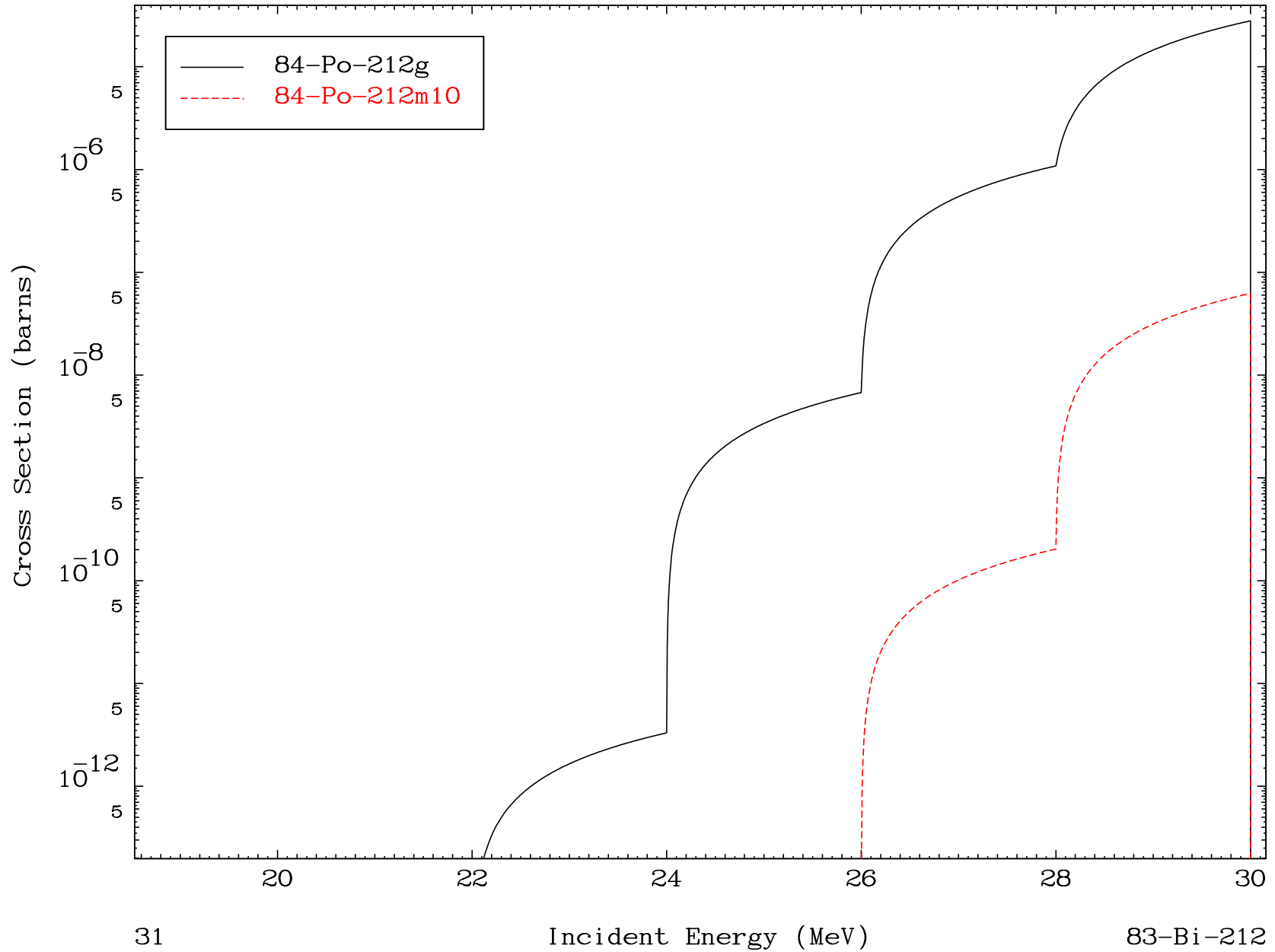


MAT 8334

( $\alpha, n'$ ) t

83-Bi-212

Radionuclide Production Cross Section



Radionuclide Production Cross Section

