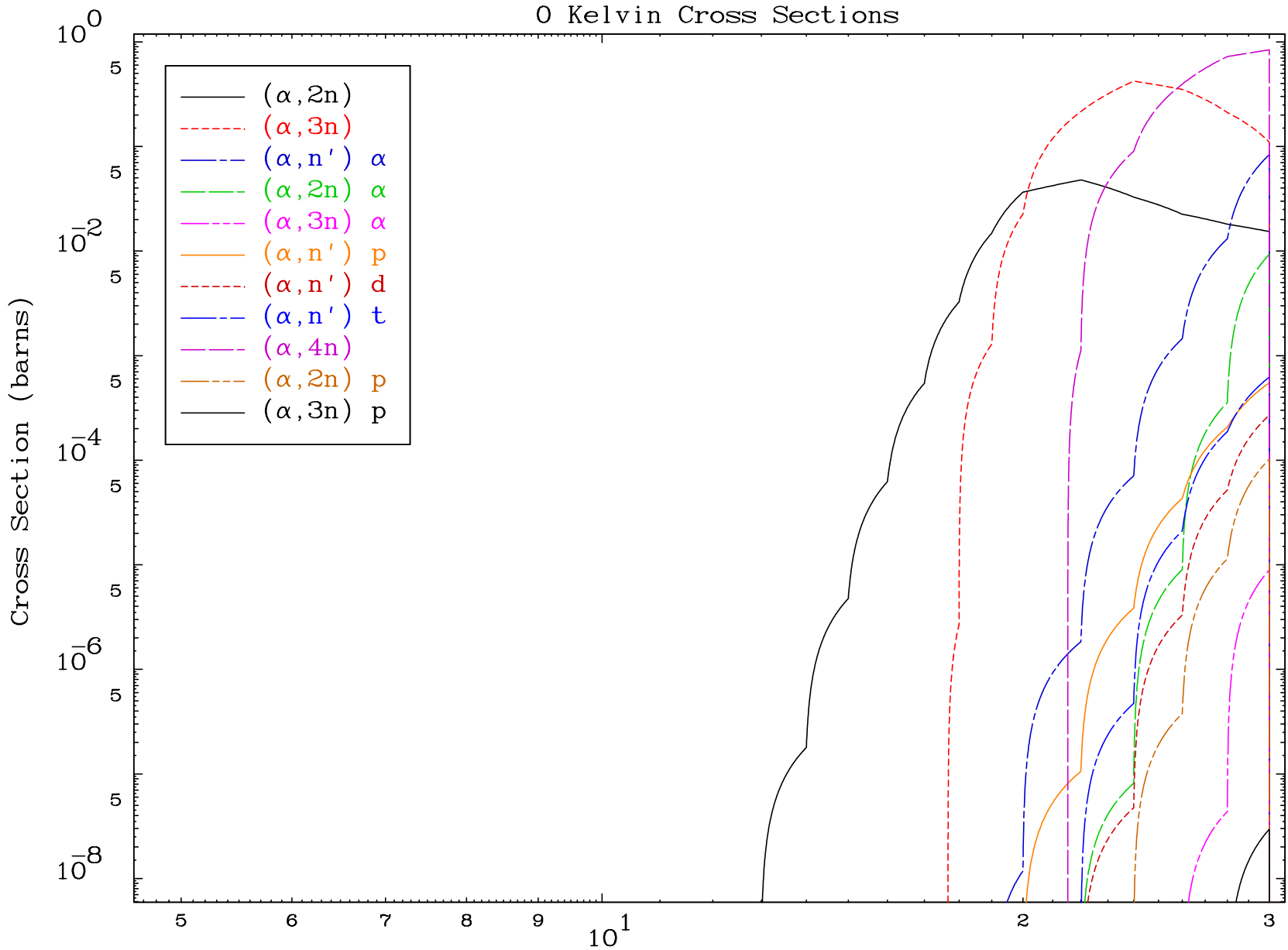


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α Neutron Production
0 Kelvin Cross Sections

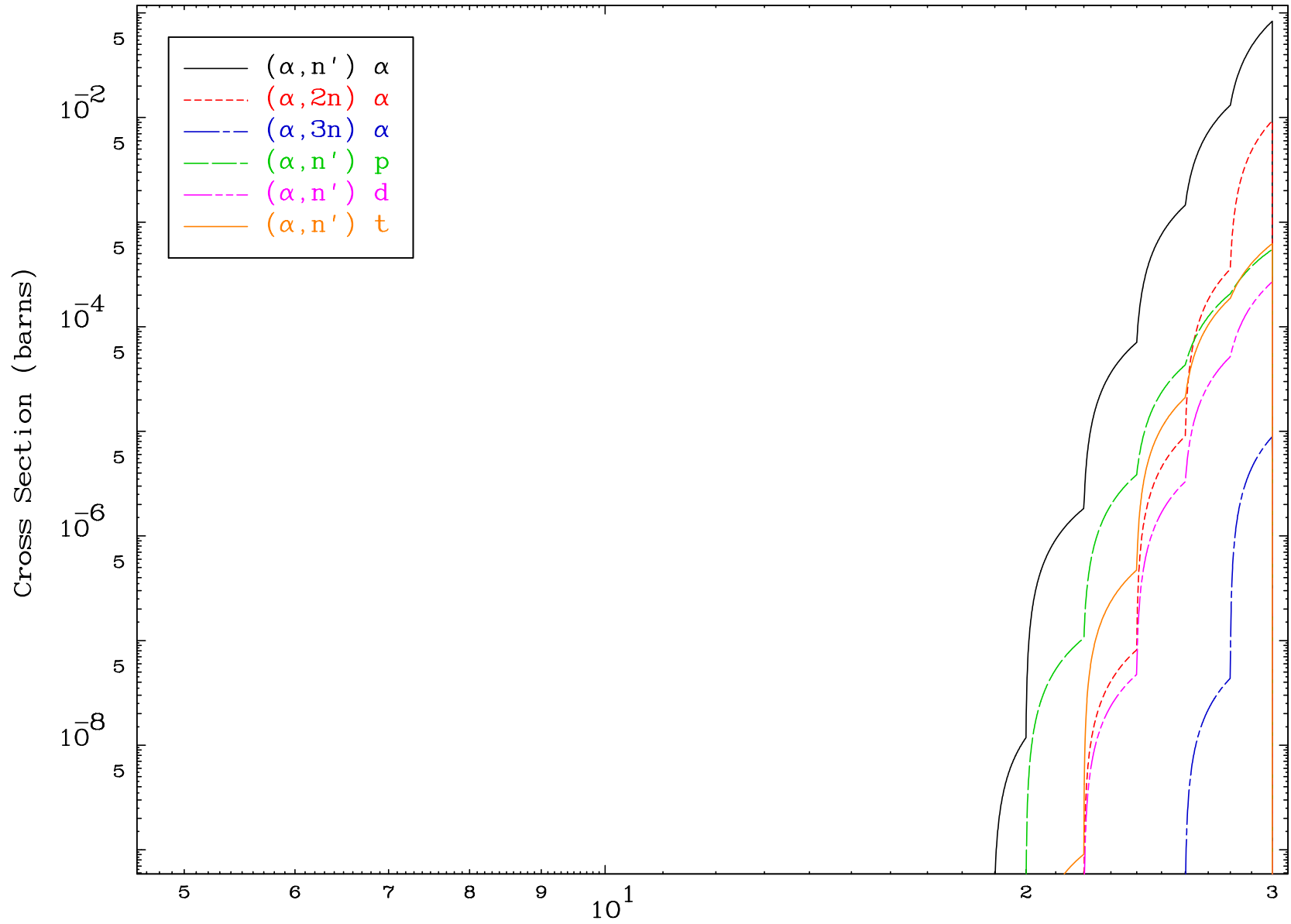
83-Bi-223

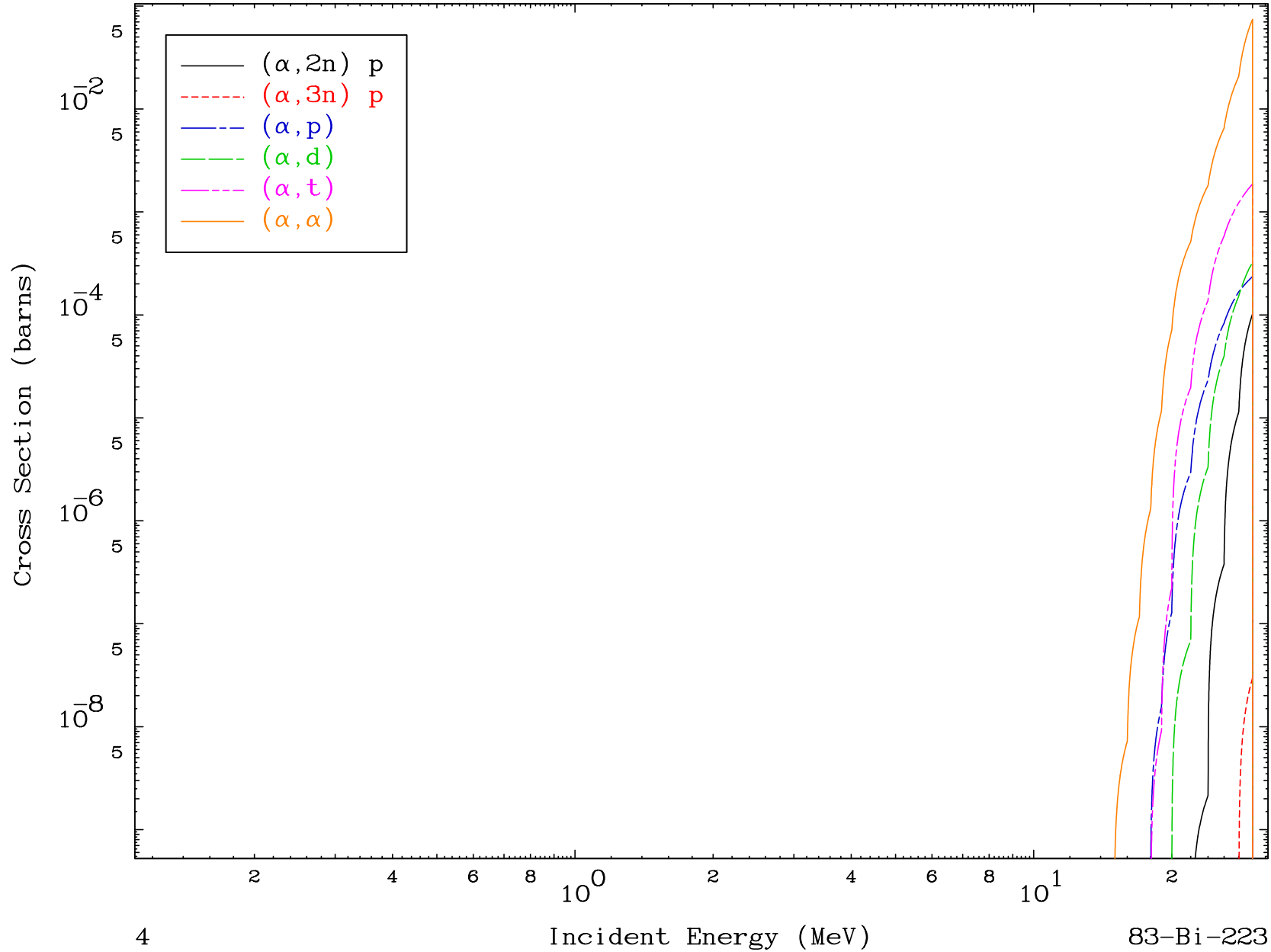


2

Incident Energy (MeV)

83-Bi-223

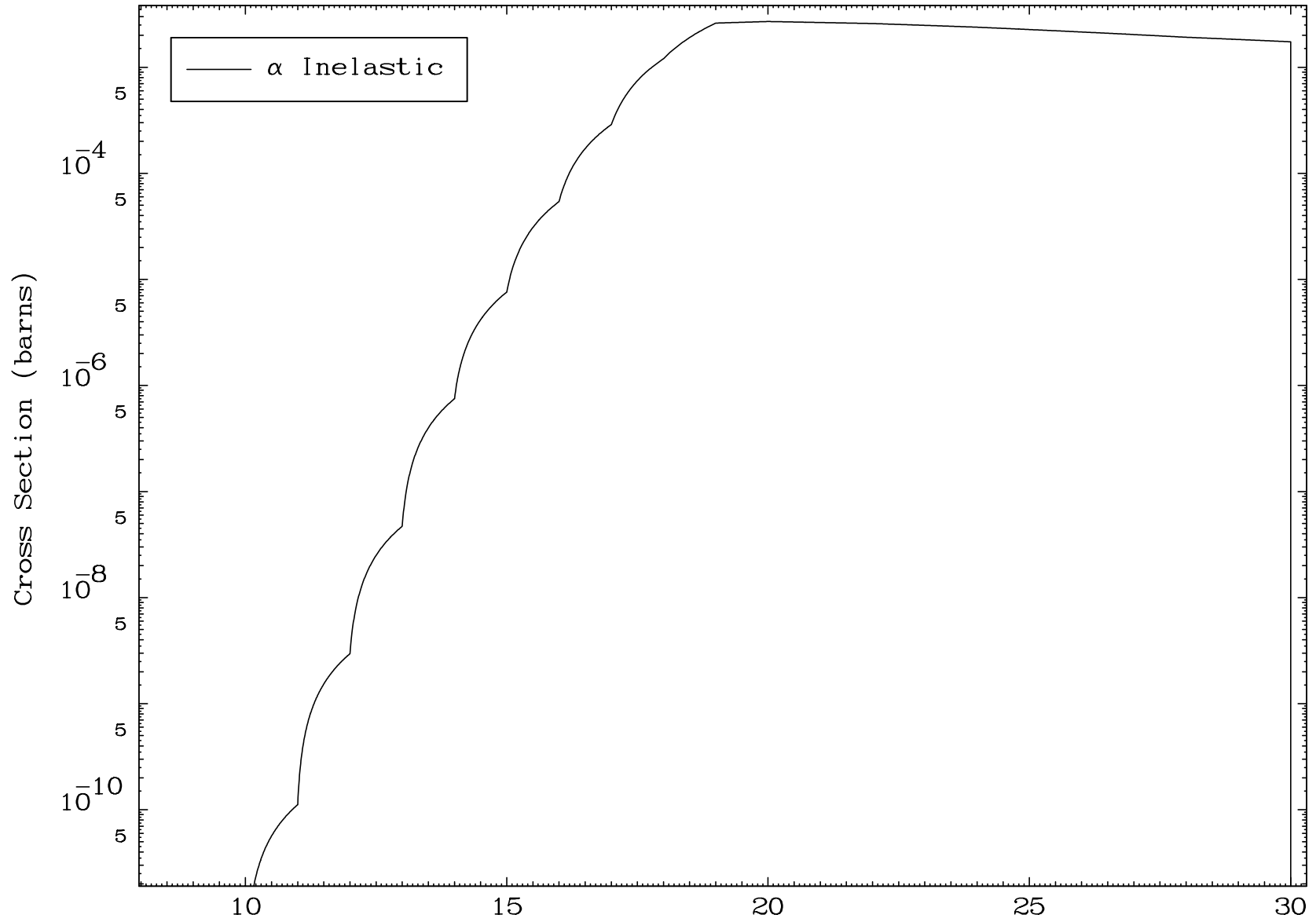




MAT 8367

(α, n') Level
0 Kelvin Cross Sections

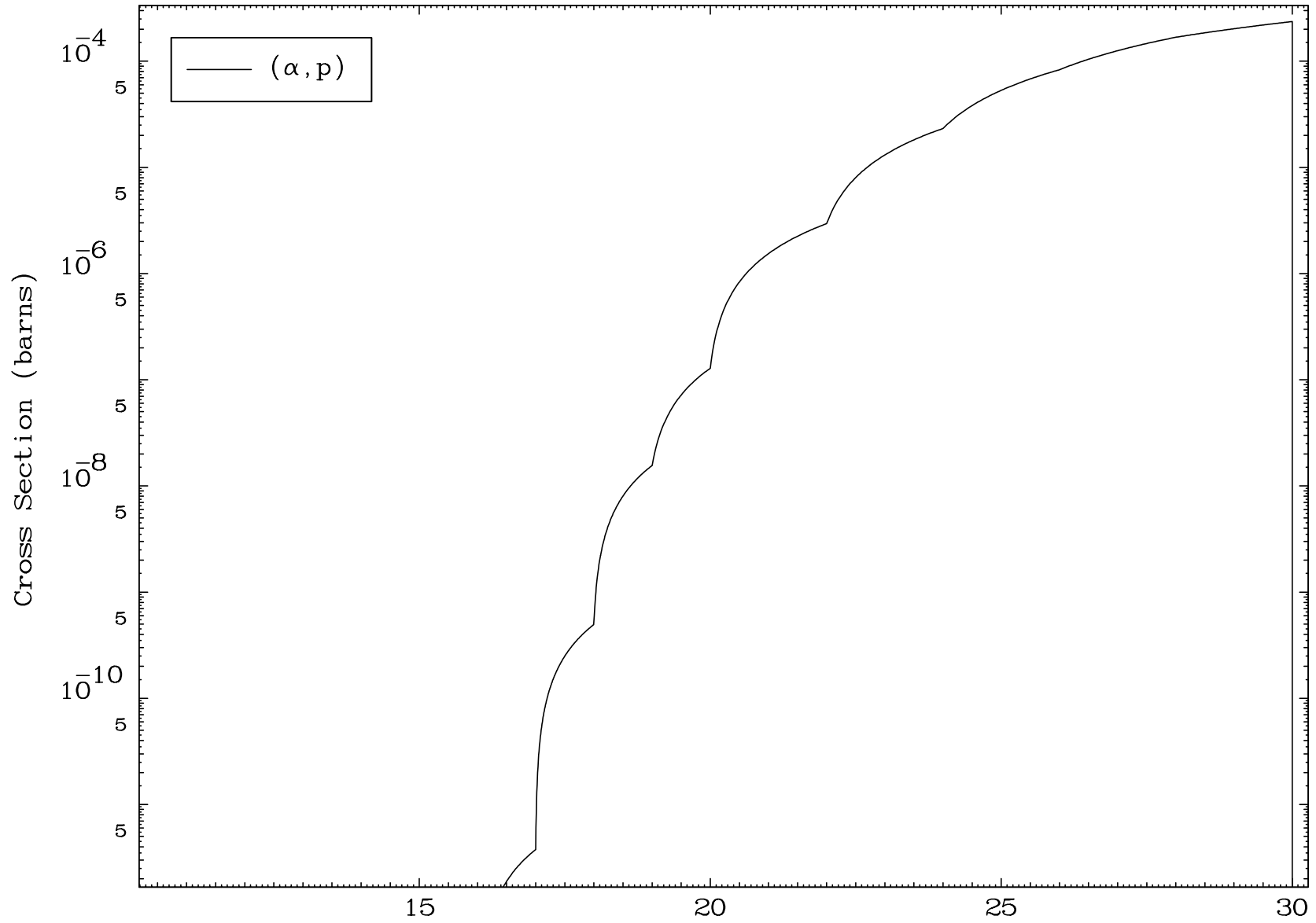
83-Bi-223

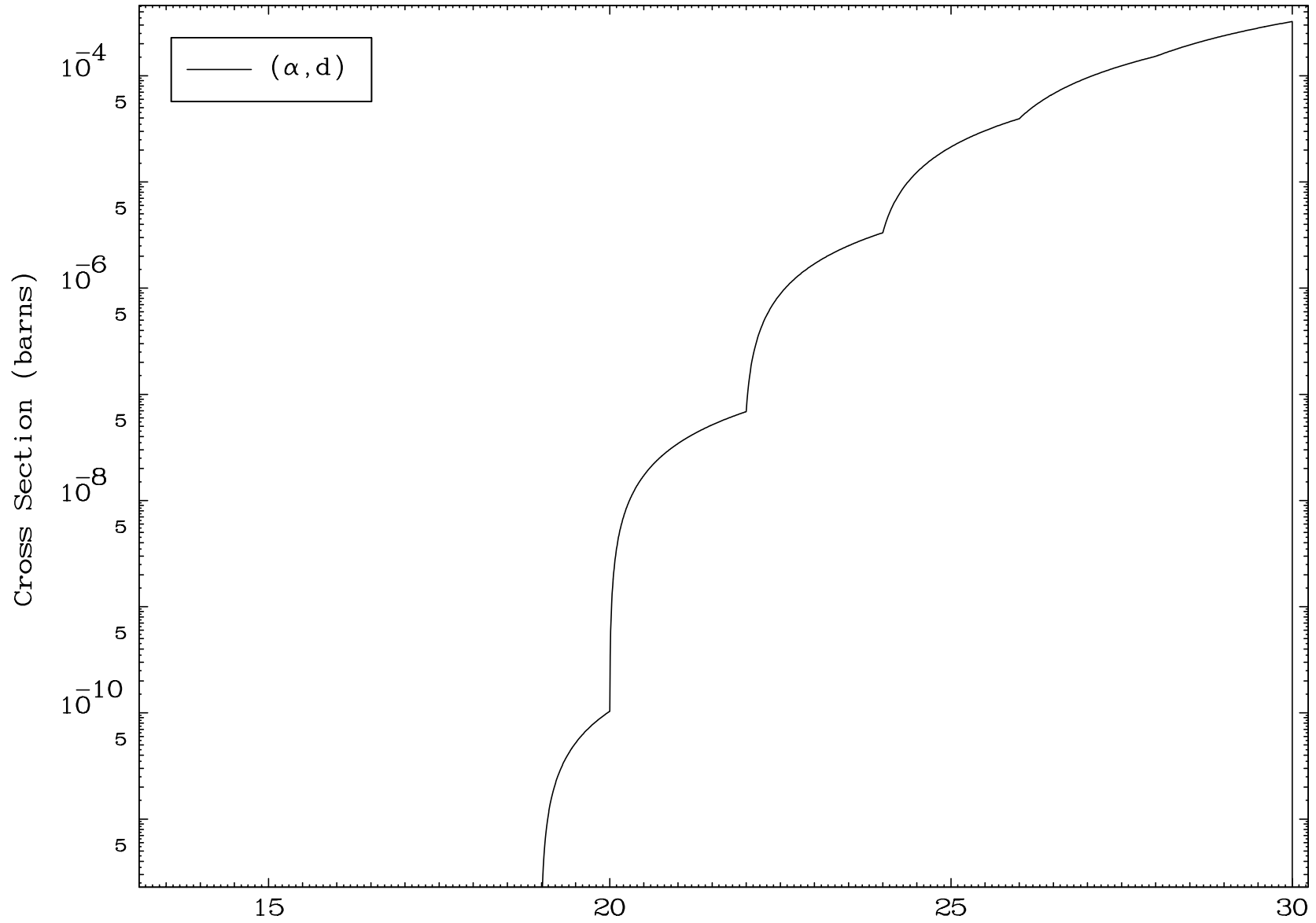


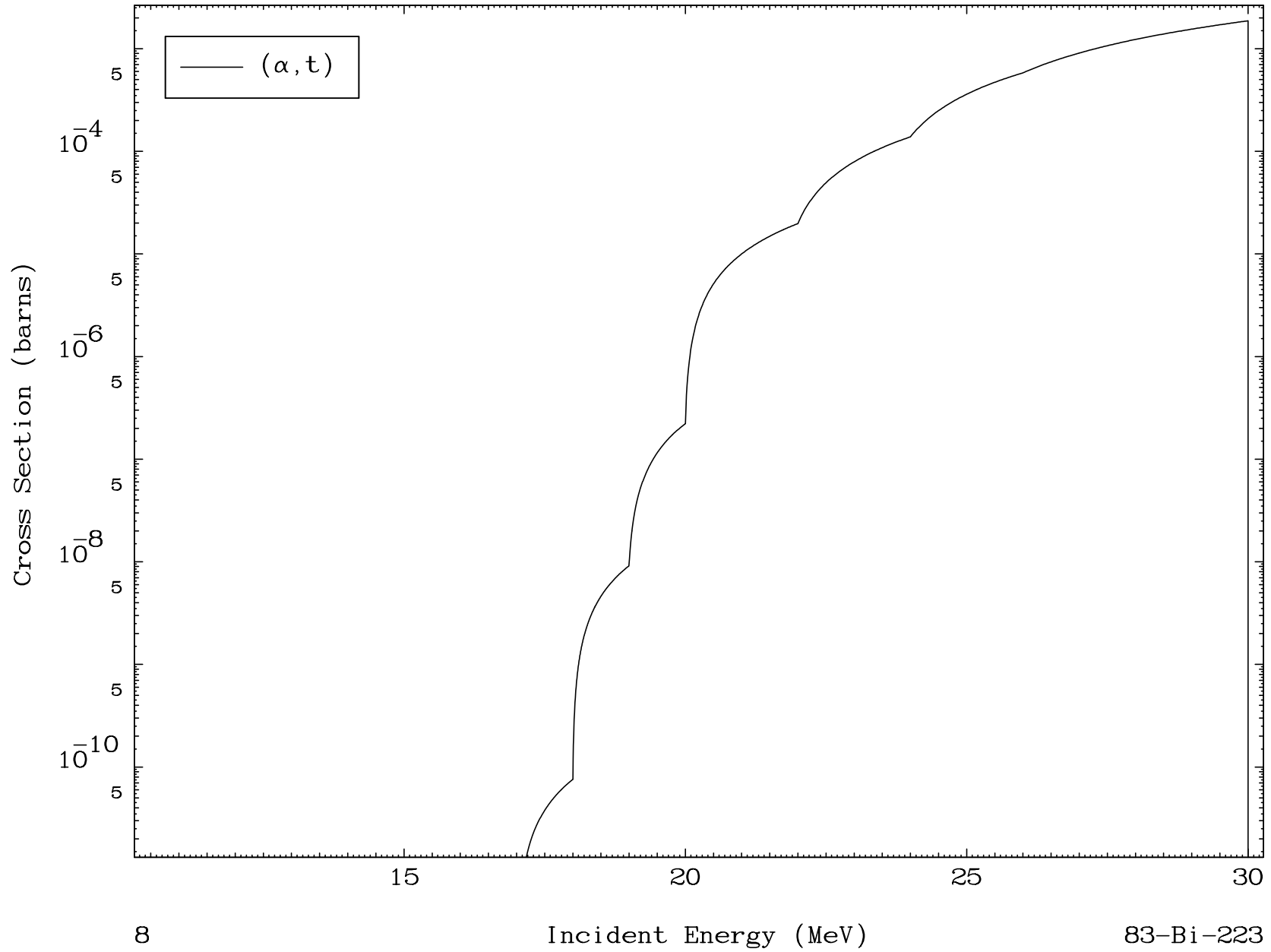
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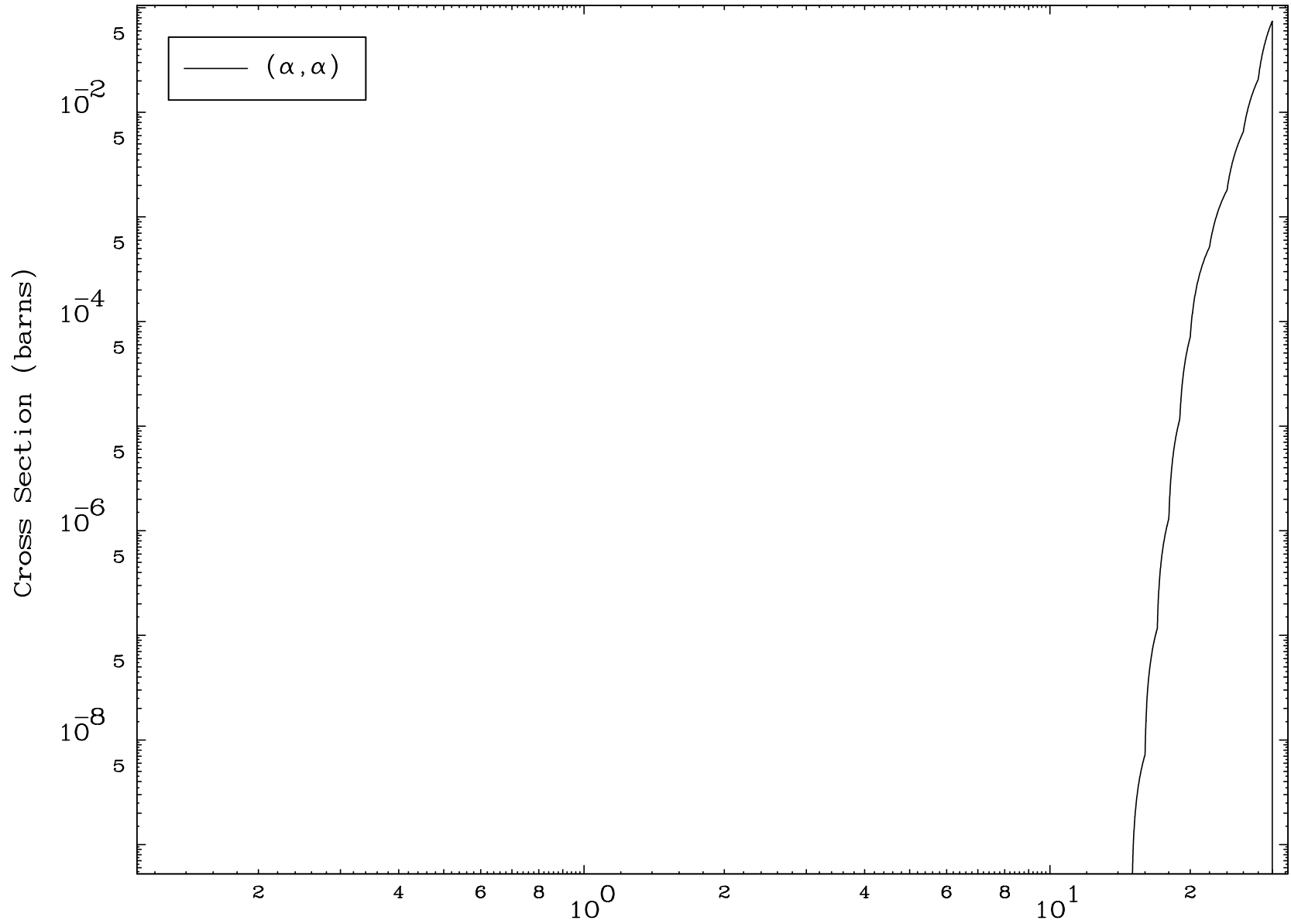
Incident Energy (MeV)

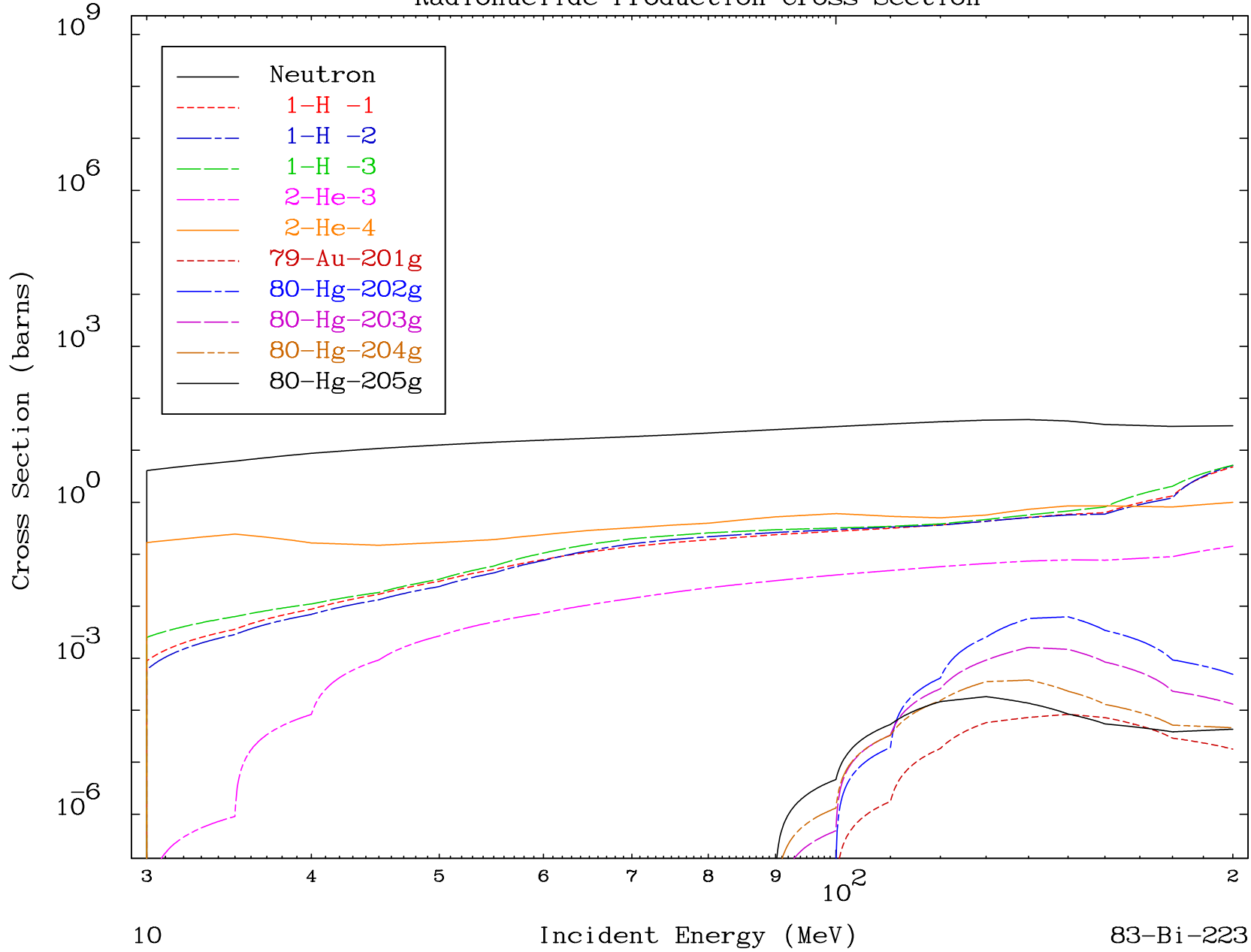
83-Bi-223









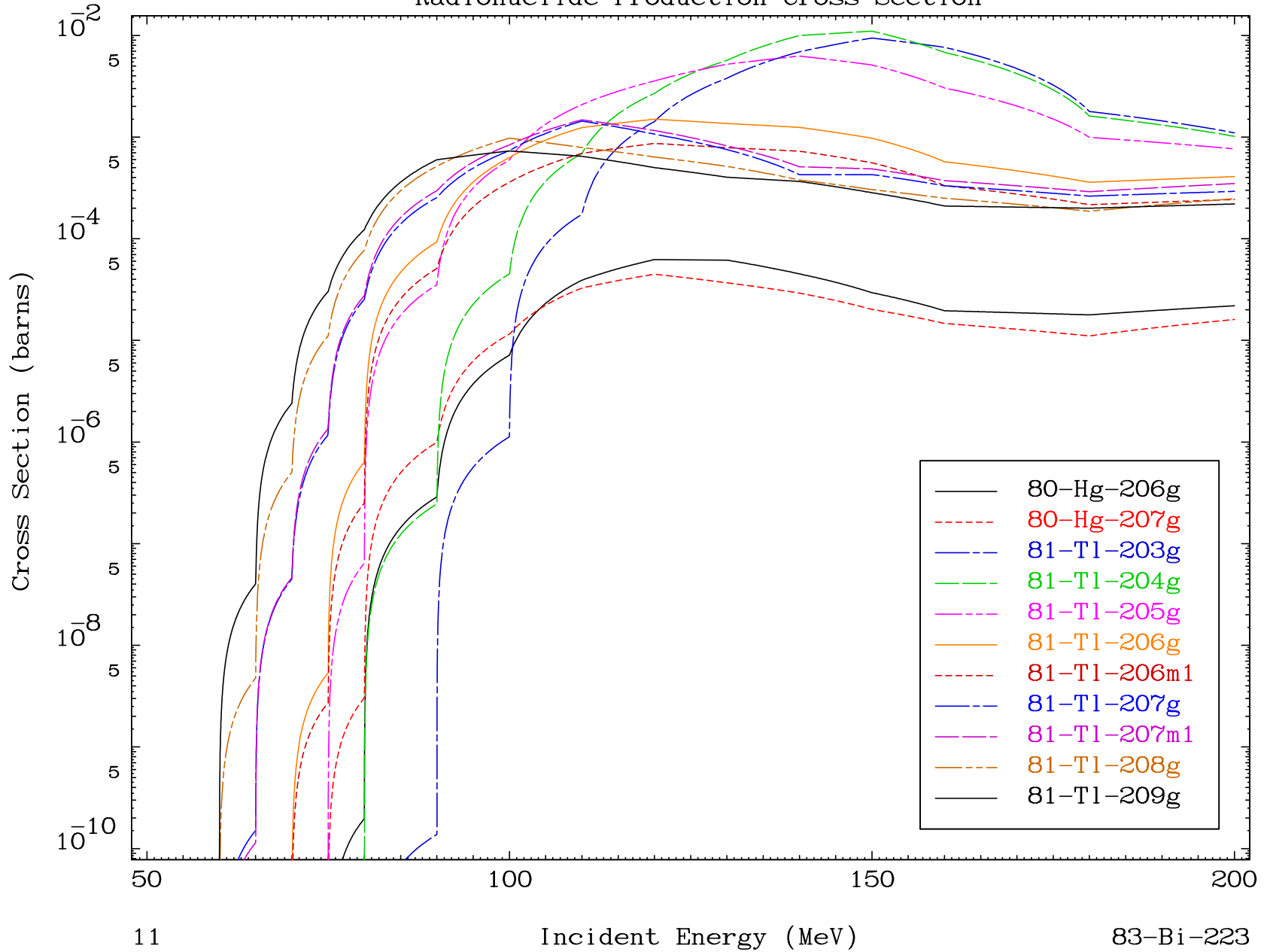


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(α , remainder)

83-Bi-223

Radionuclide Production Cross Section

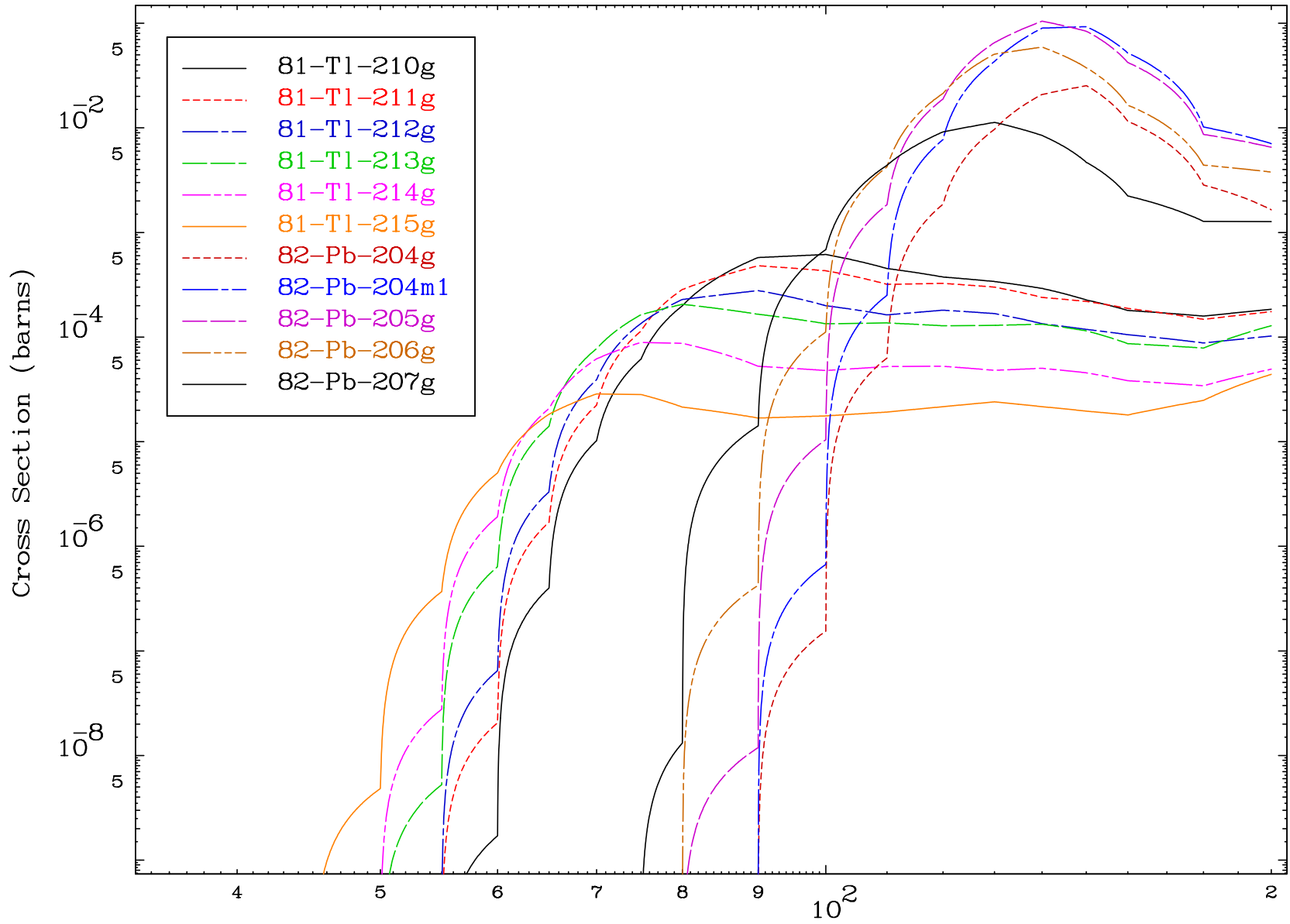


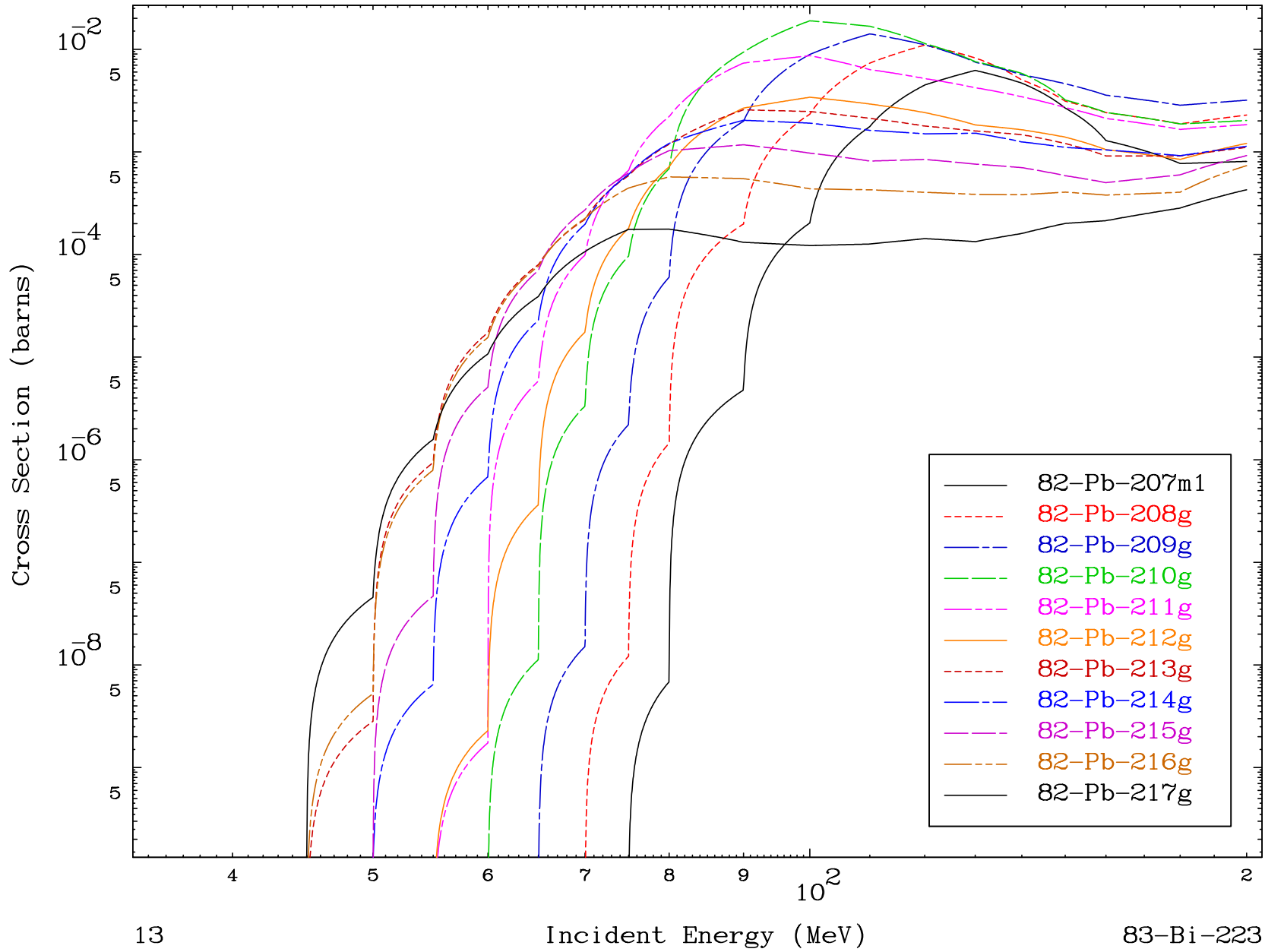
11

Incident Energy (MeV)

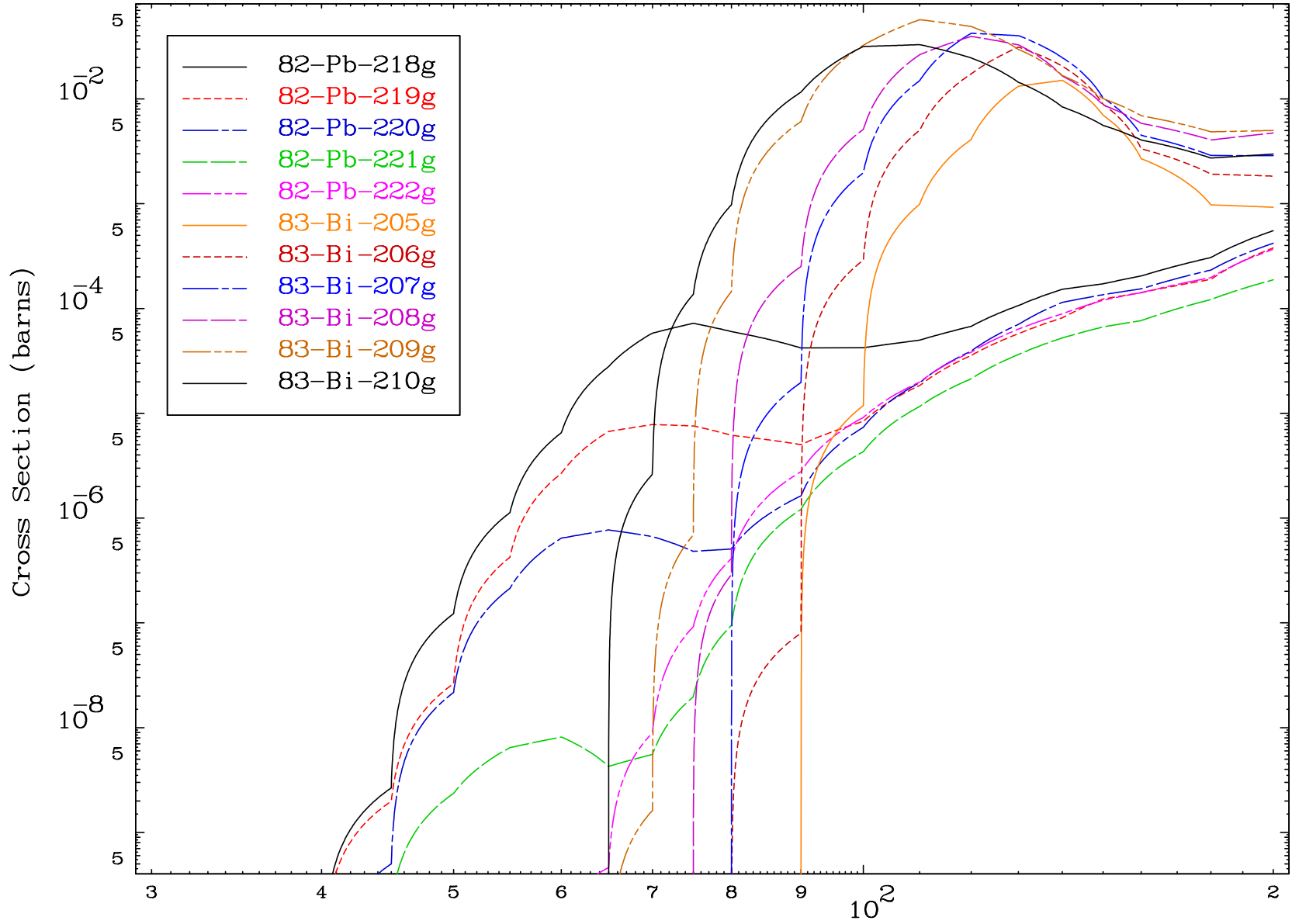
83-Bi-223

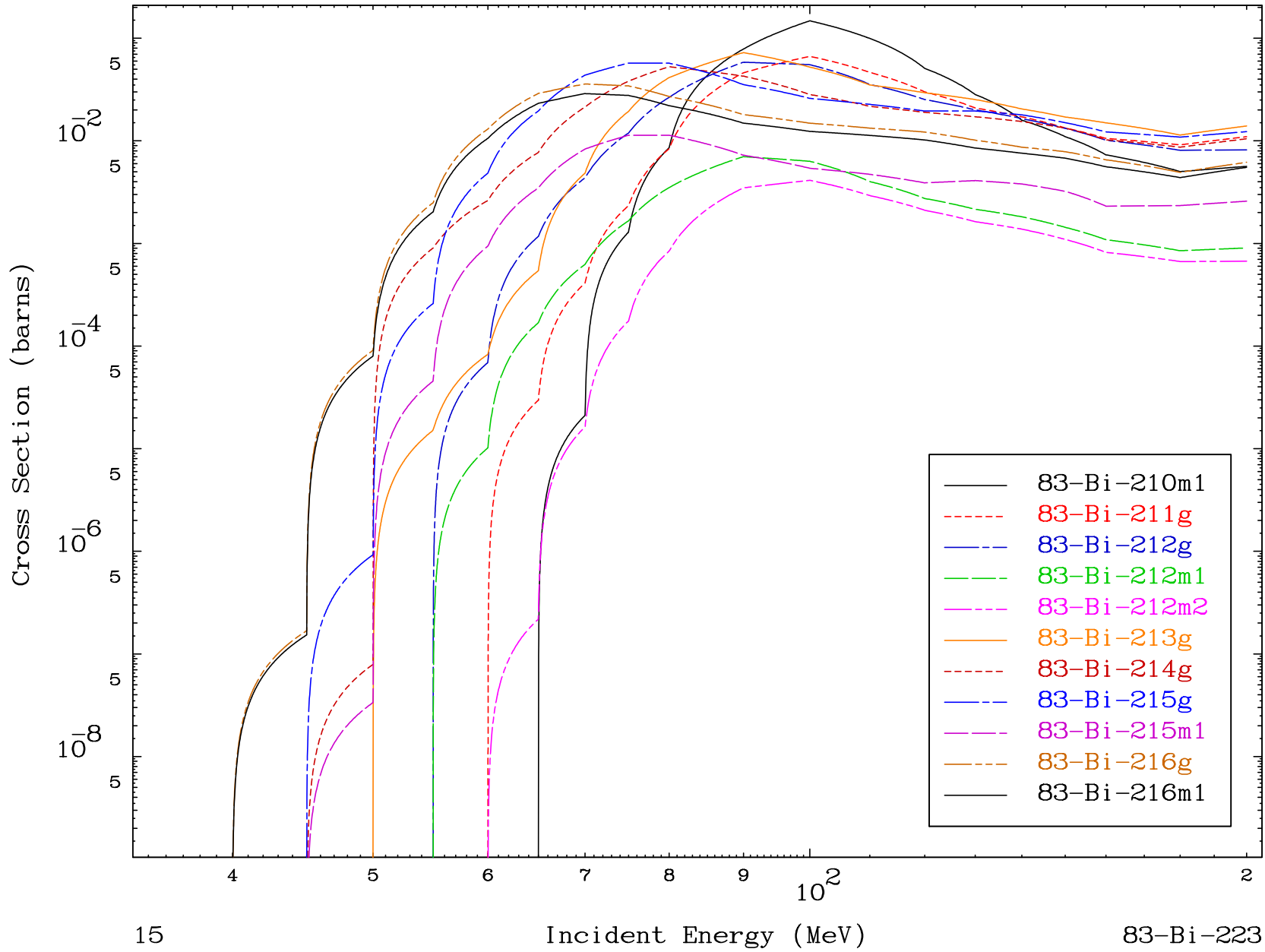
Radionuclide Production Cross Section



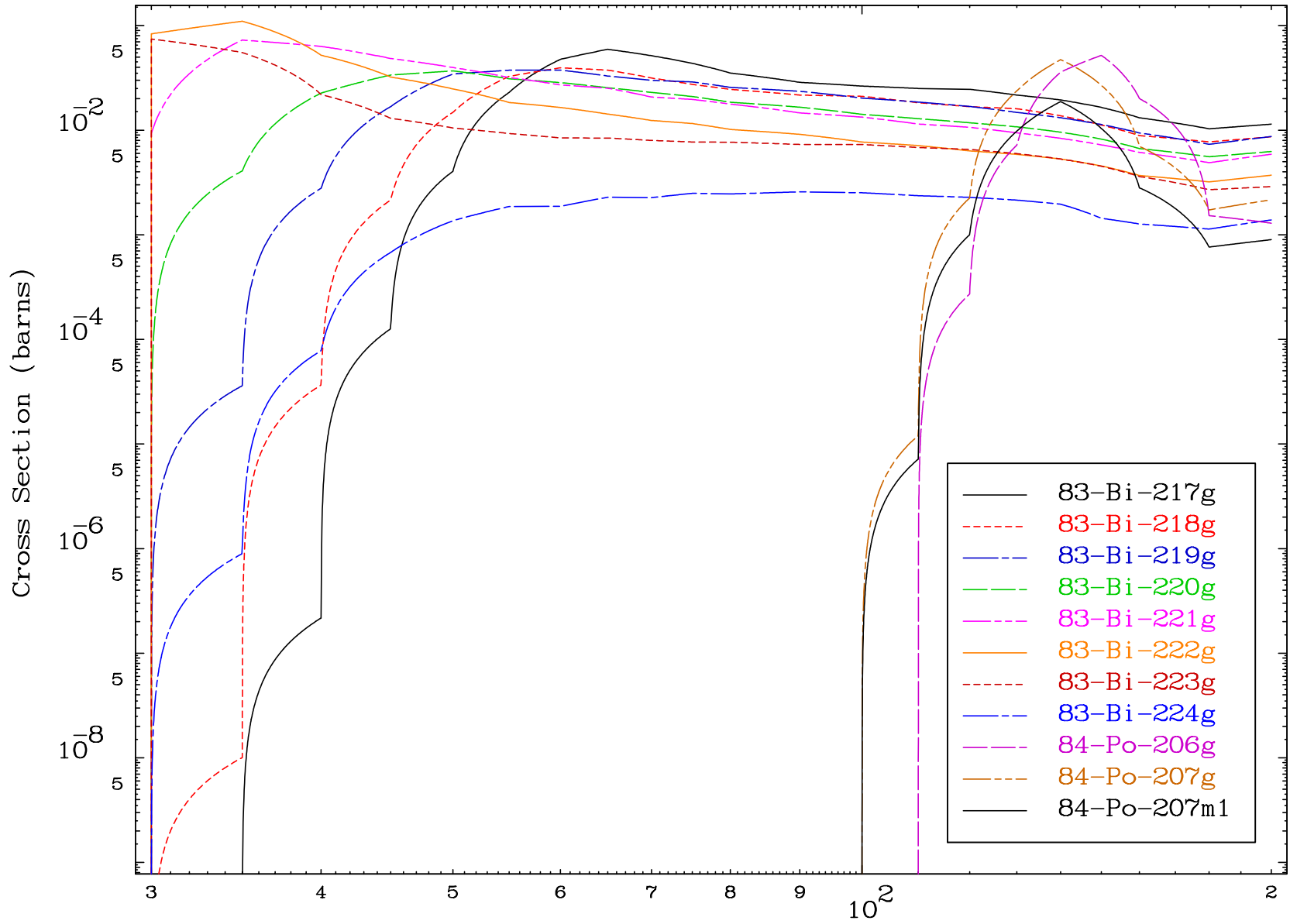


Radionuclide Production Cross Section

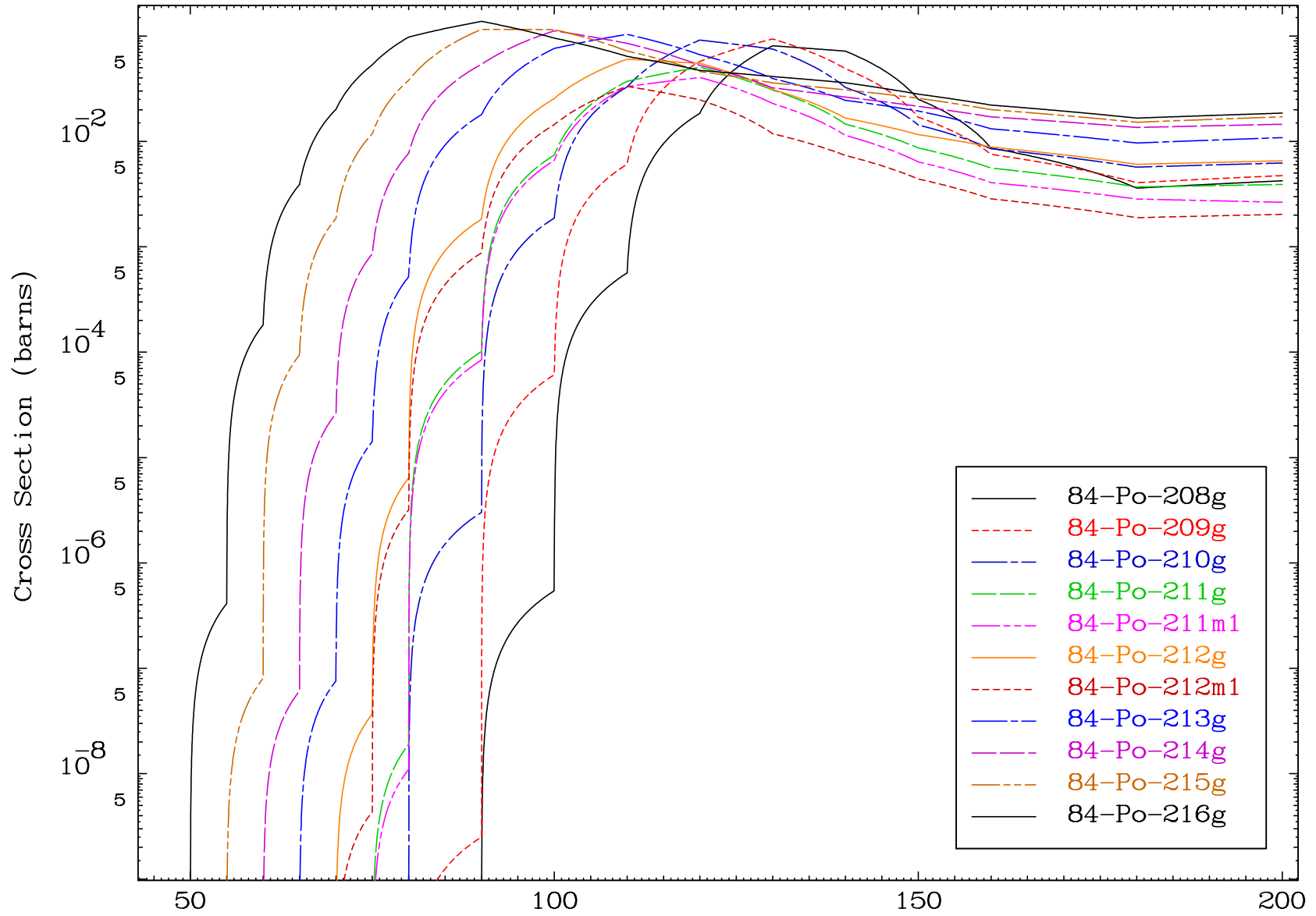




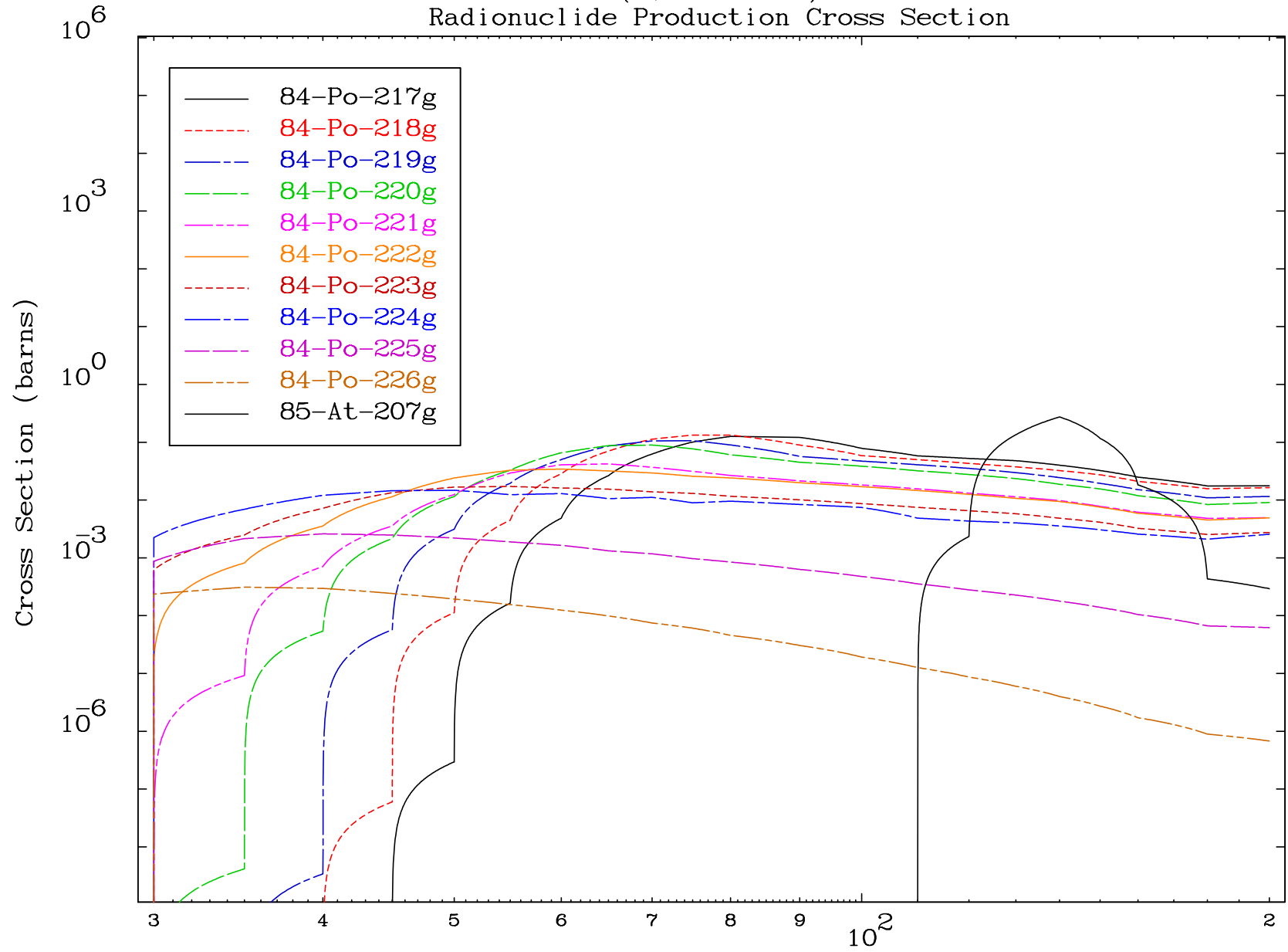
Radionuclide Production Cross Section



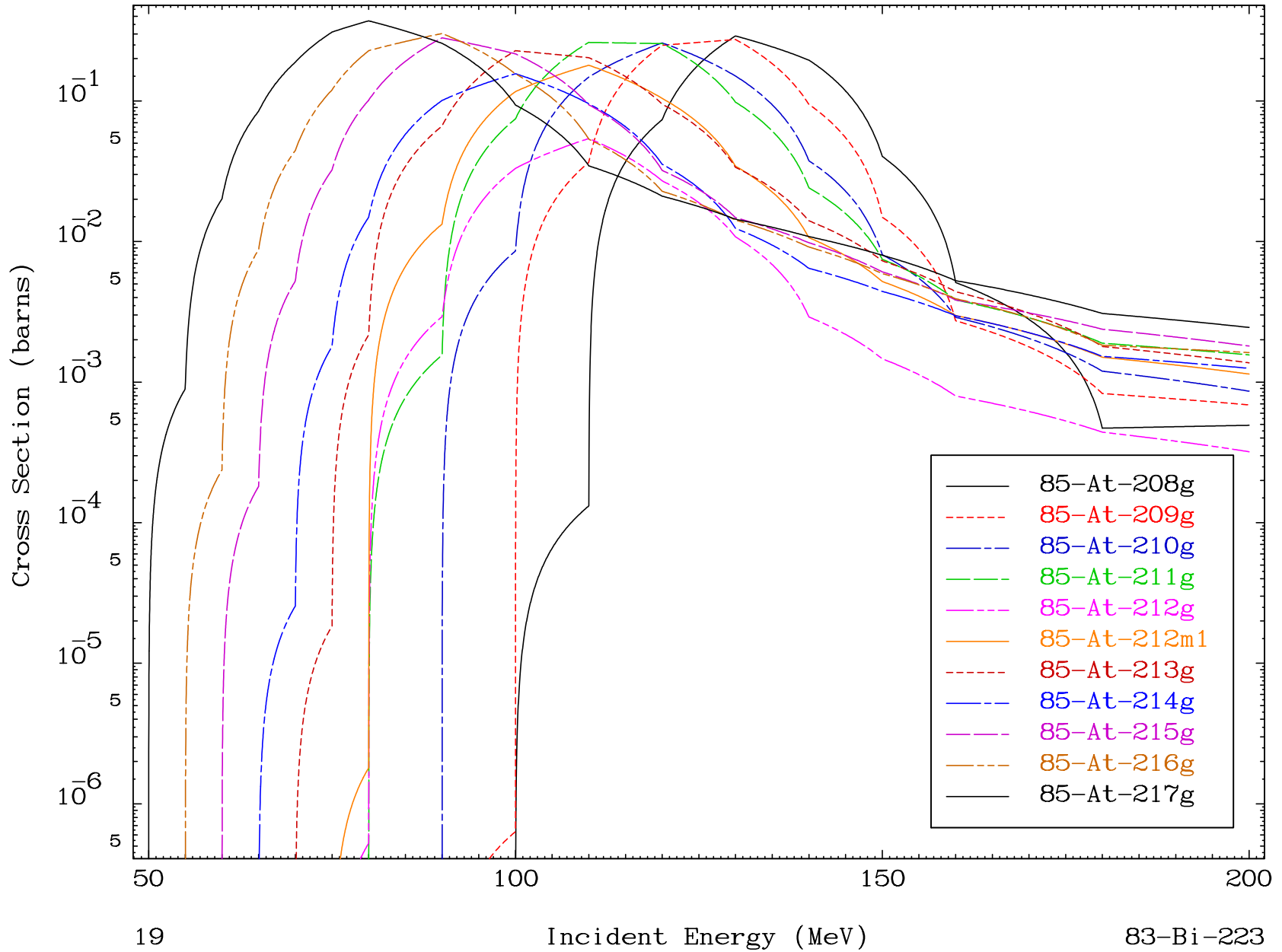
Radionuclide Production Cross Section

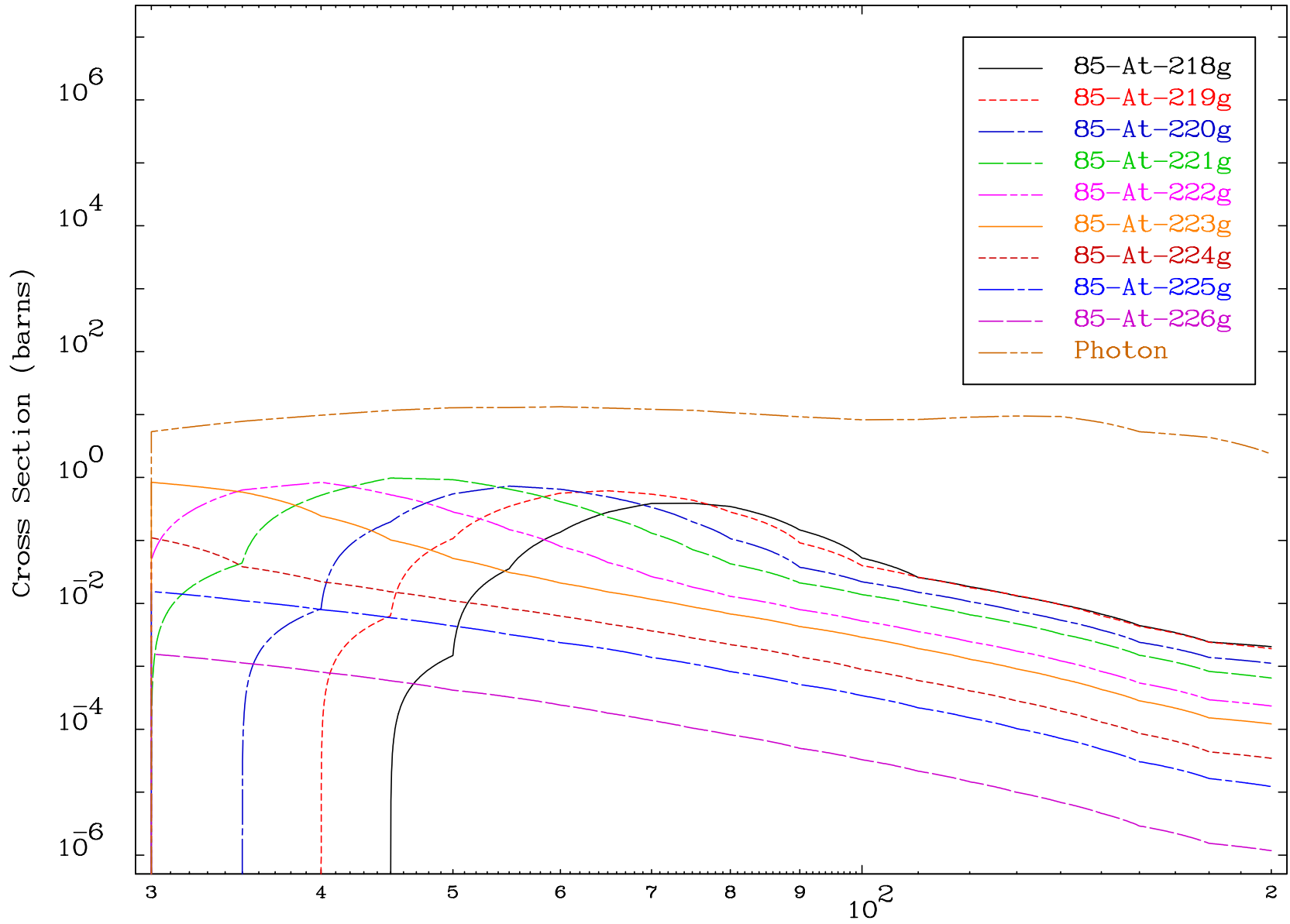


Radionuclide Production Cross Section



Radionuclide Production Cross Section





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α Fission
Radionuclide Production Cross Section

83-Bi-223

