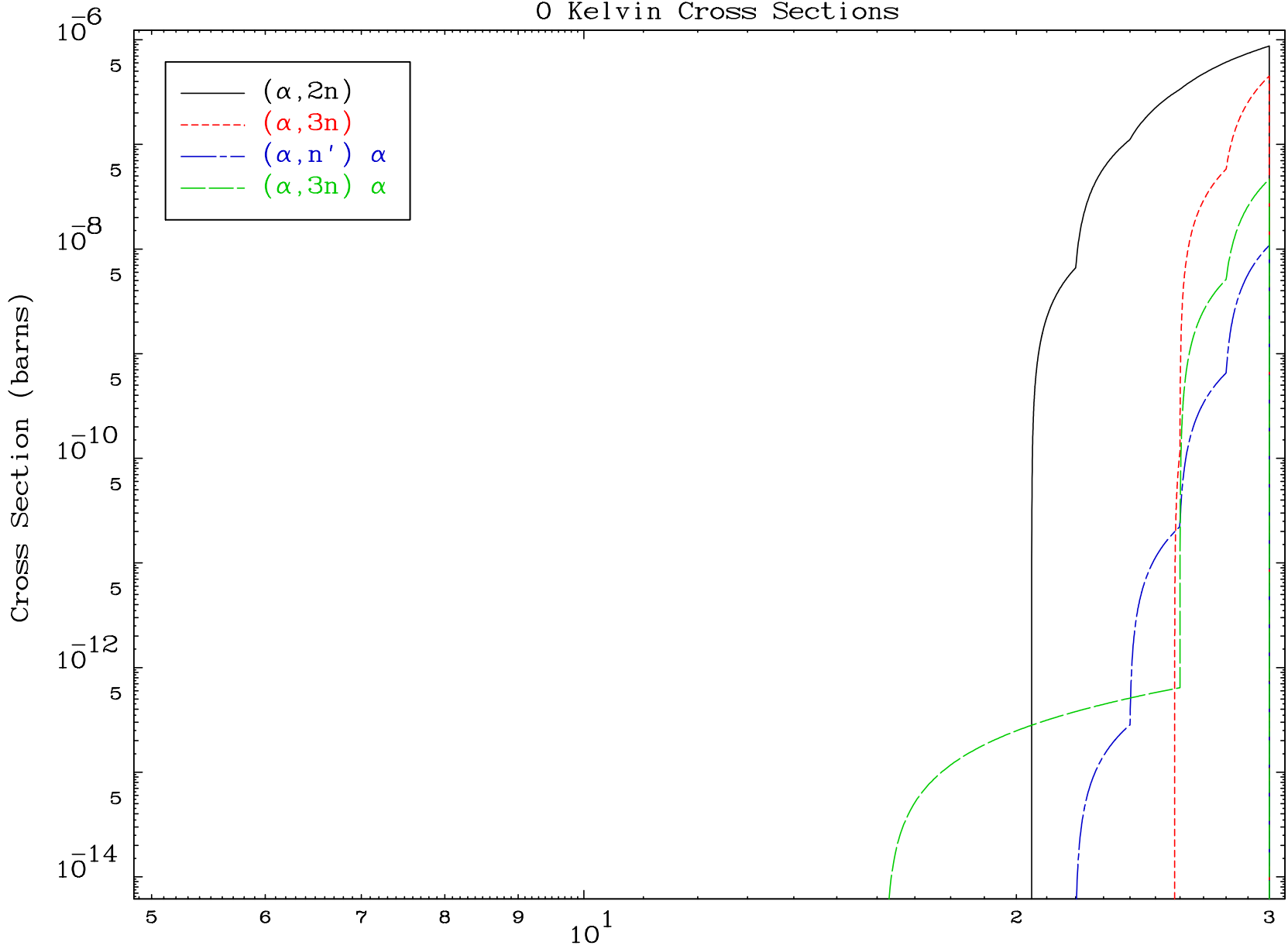


MAT 570

α Neutron Production
0 Kelvin Cross Sections

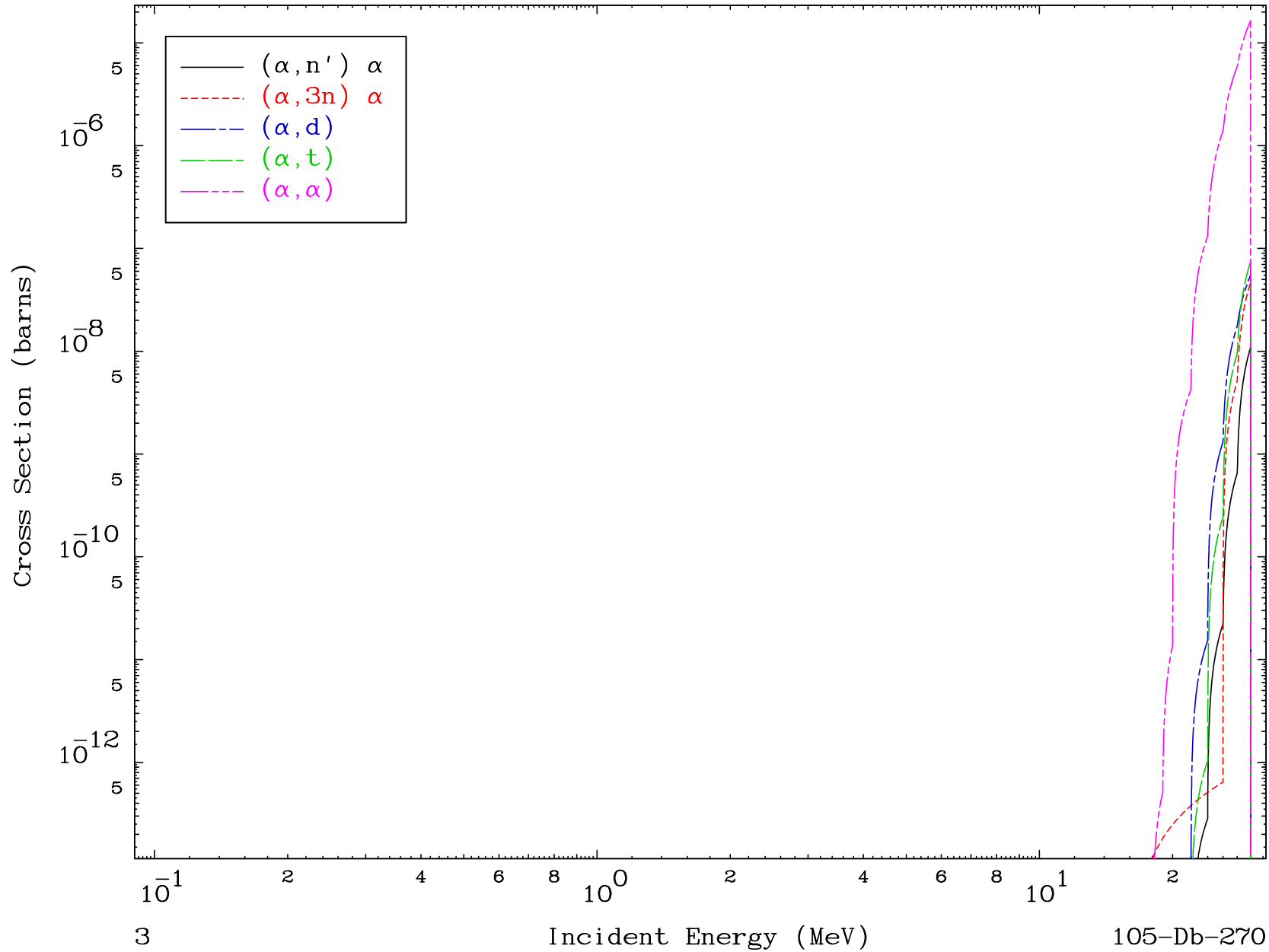
105-Db-270

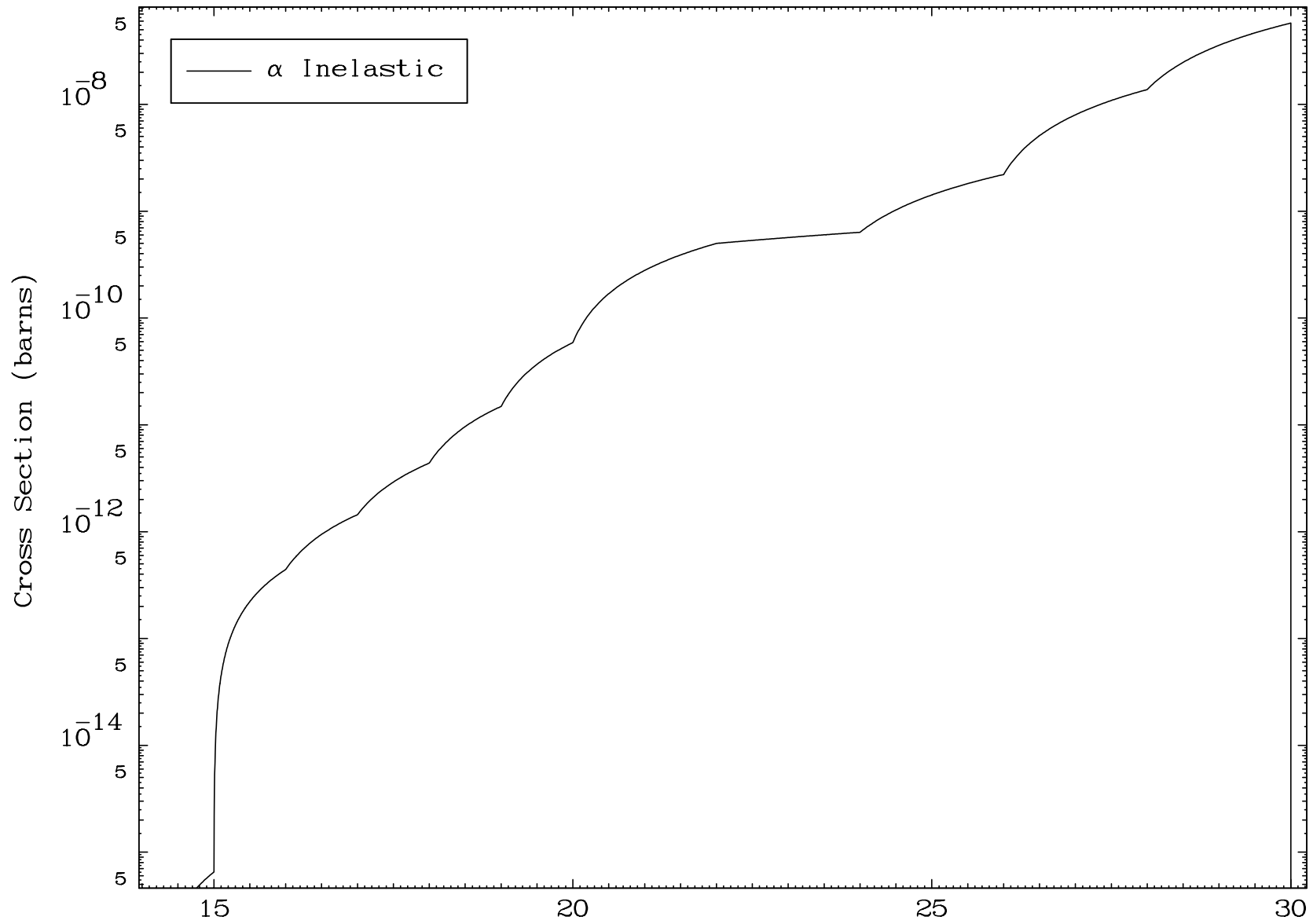


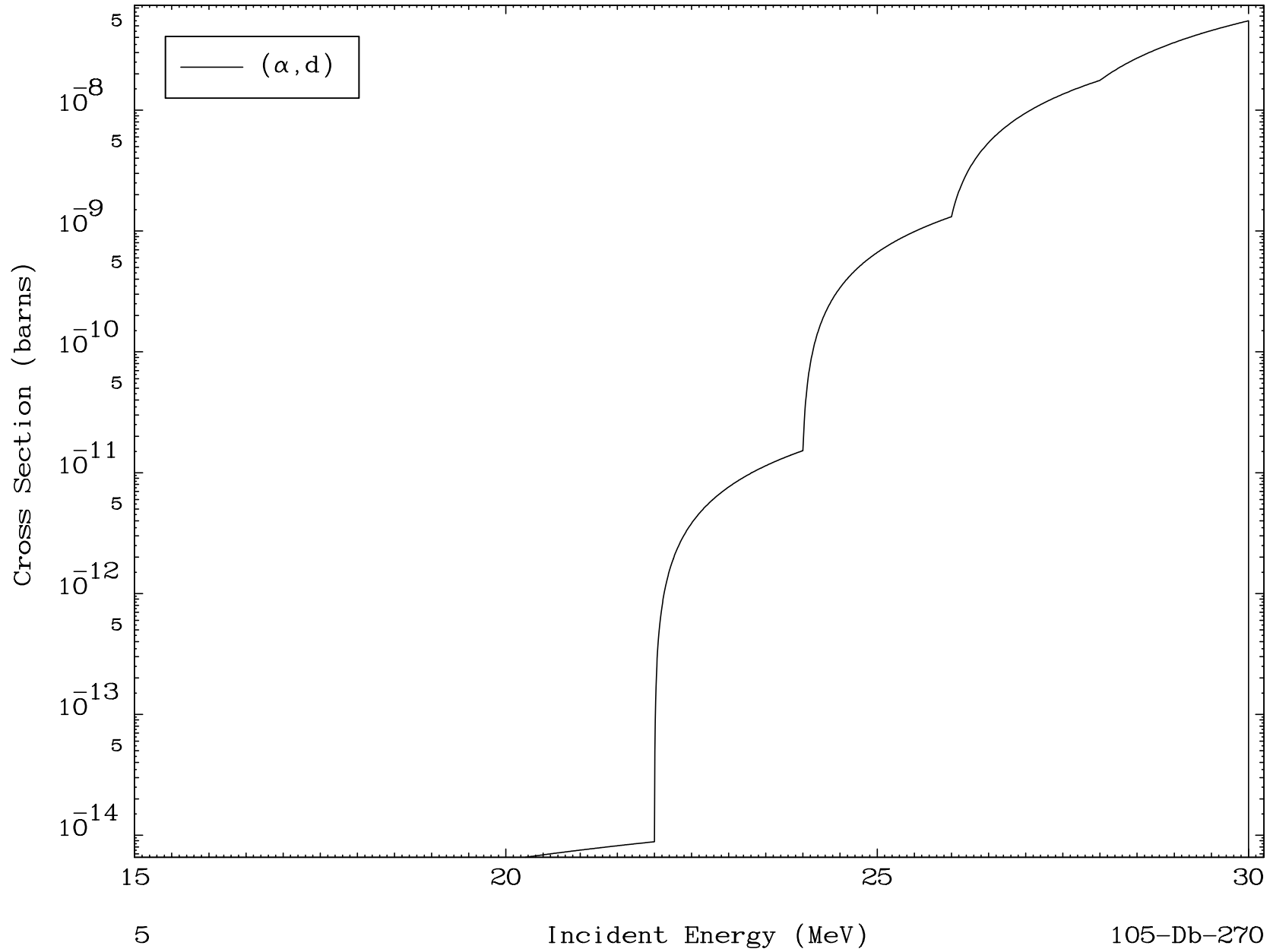
2

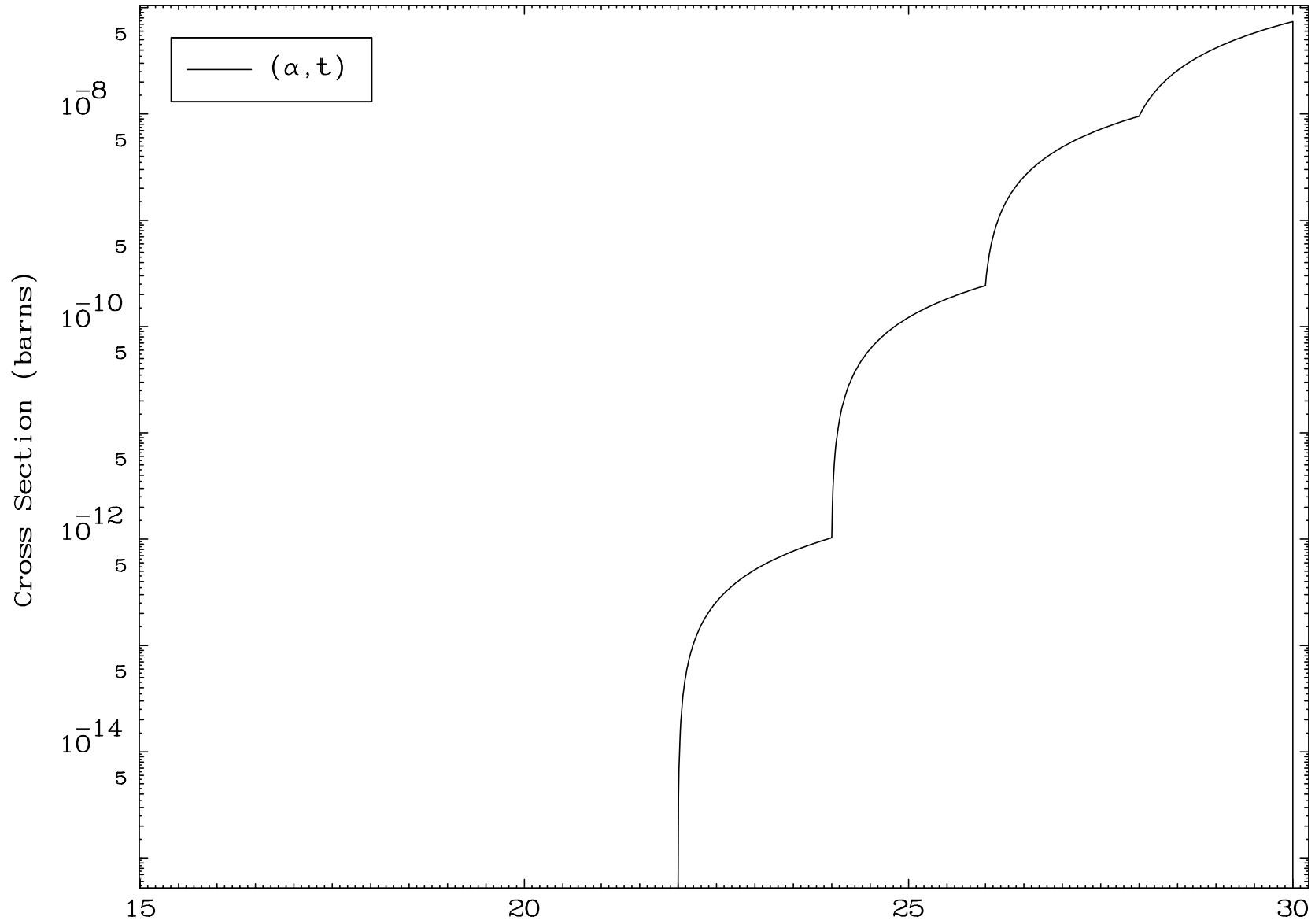
Incident Energy (MeV)

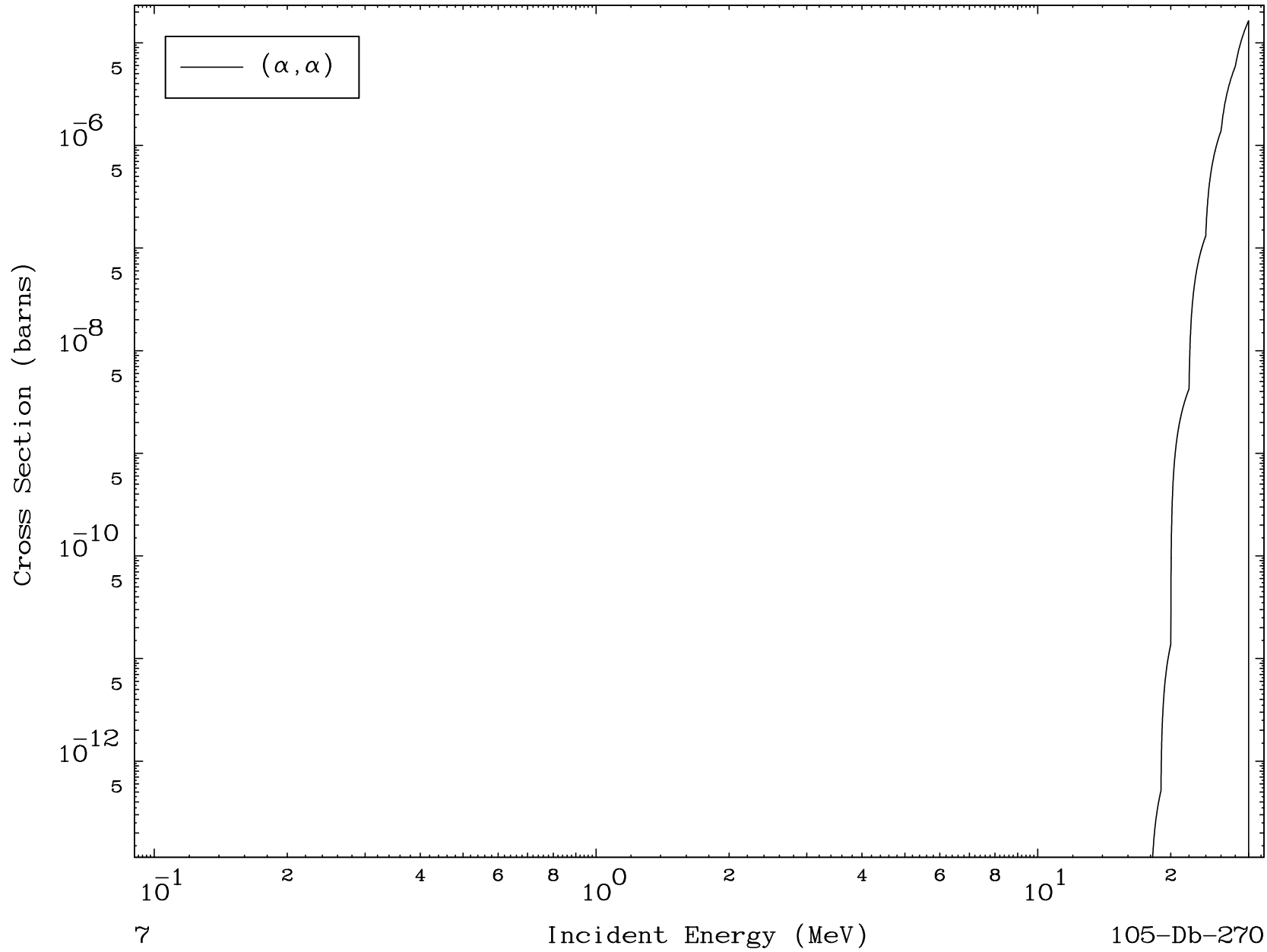
105-Db-270



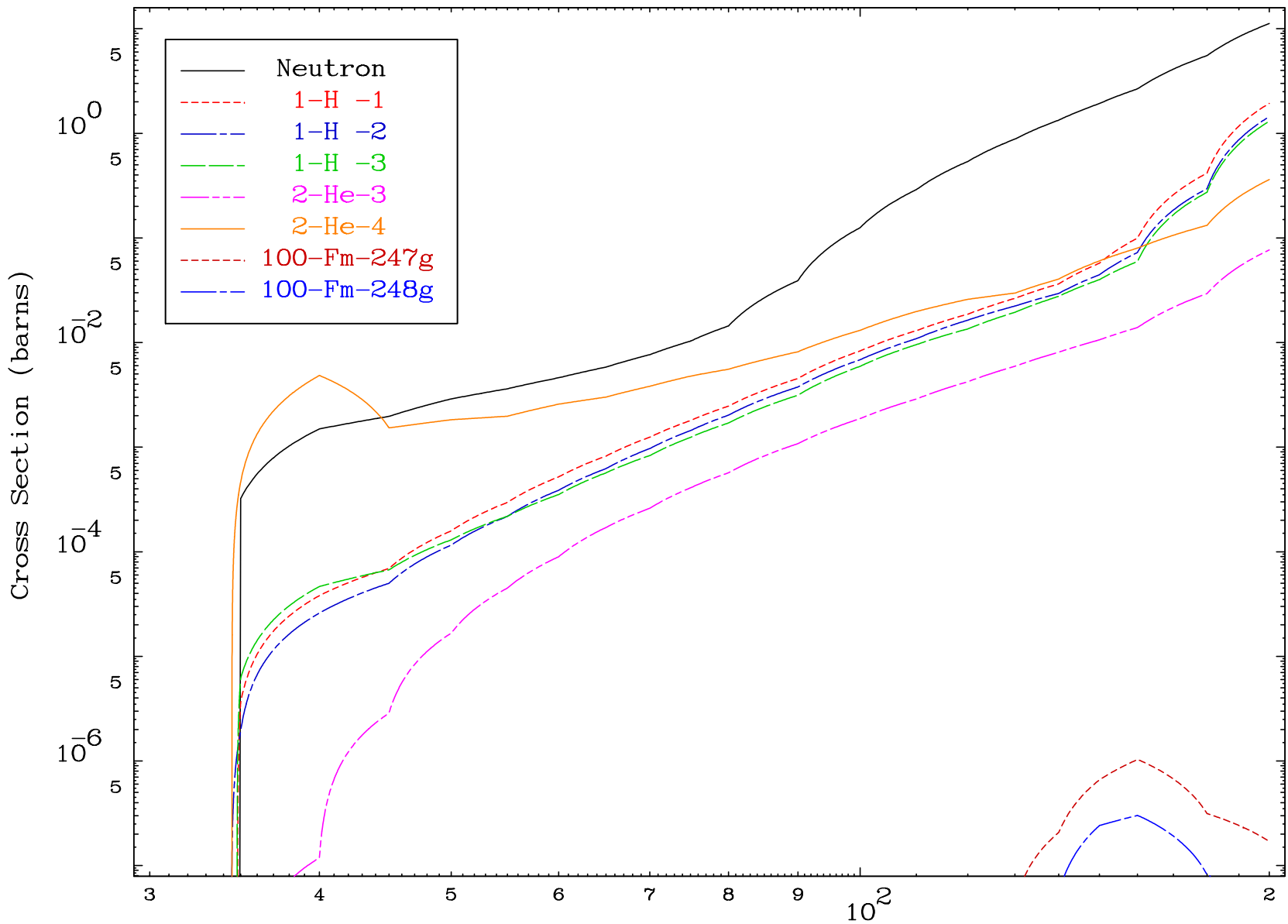




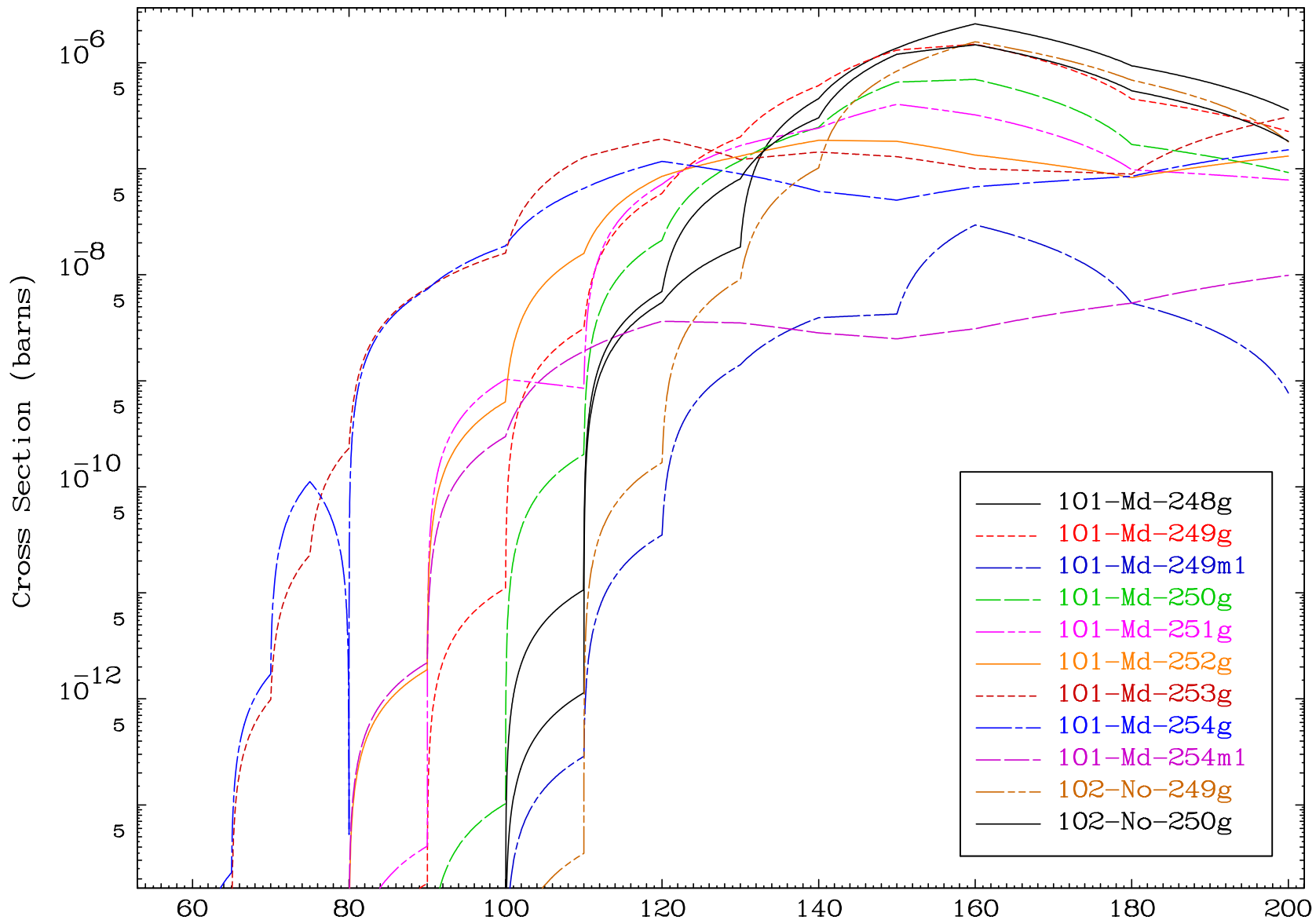




Radionuclide Production Cross Section



Radionuclide Production Cross Section

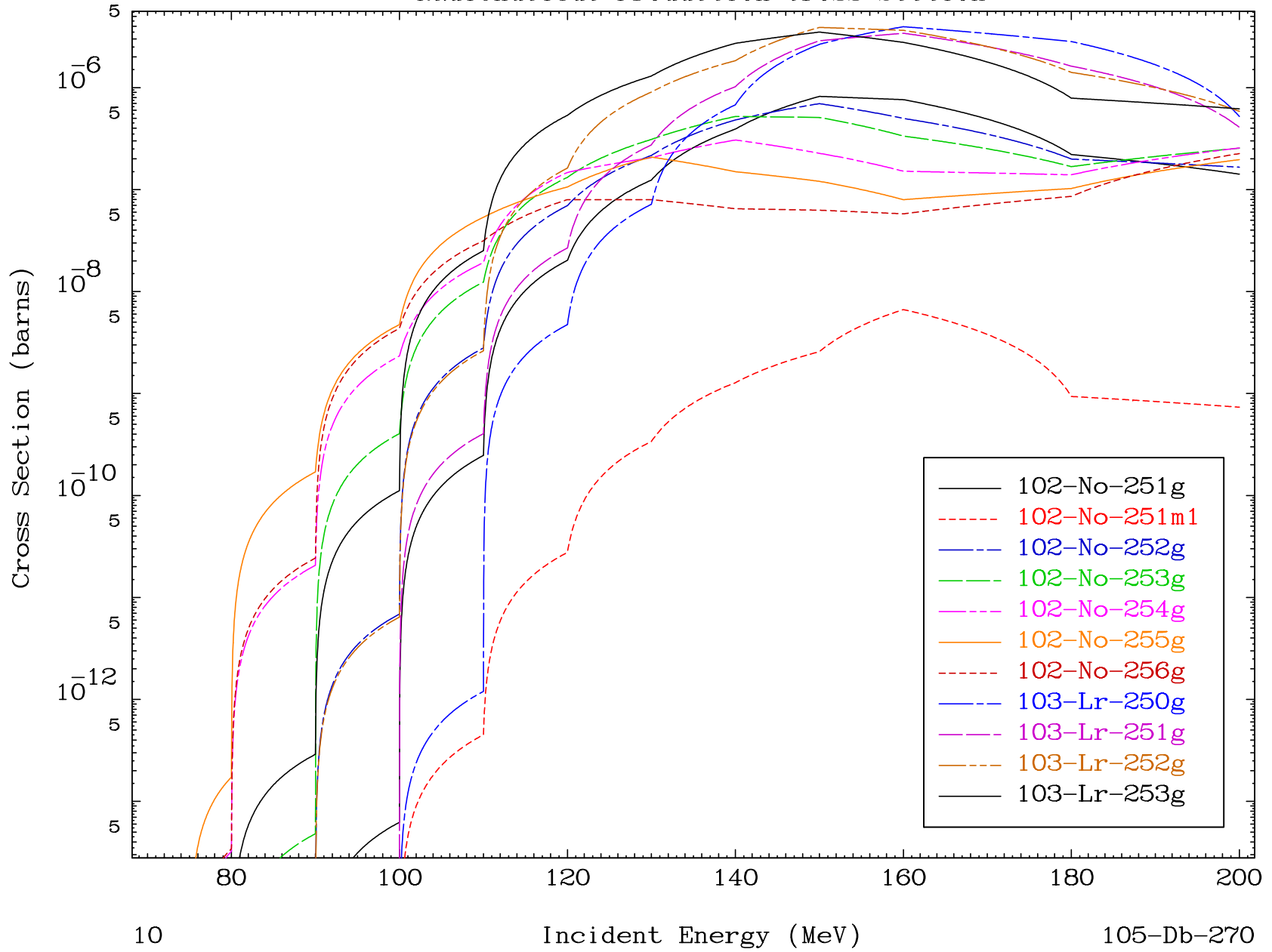


MAT 570

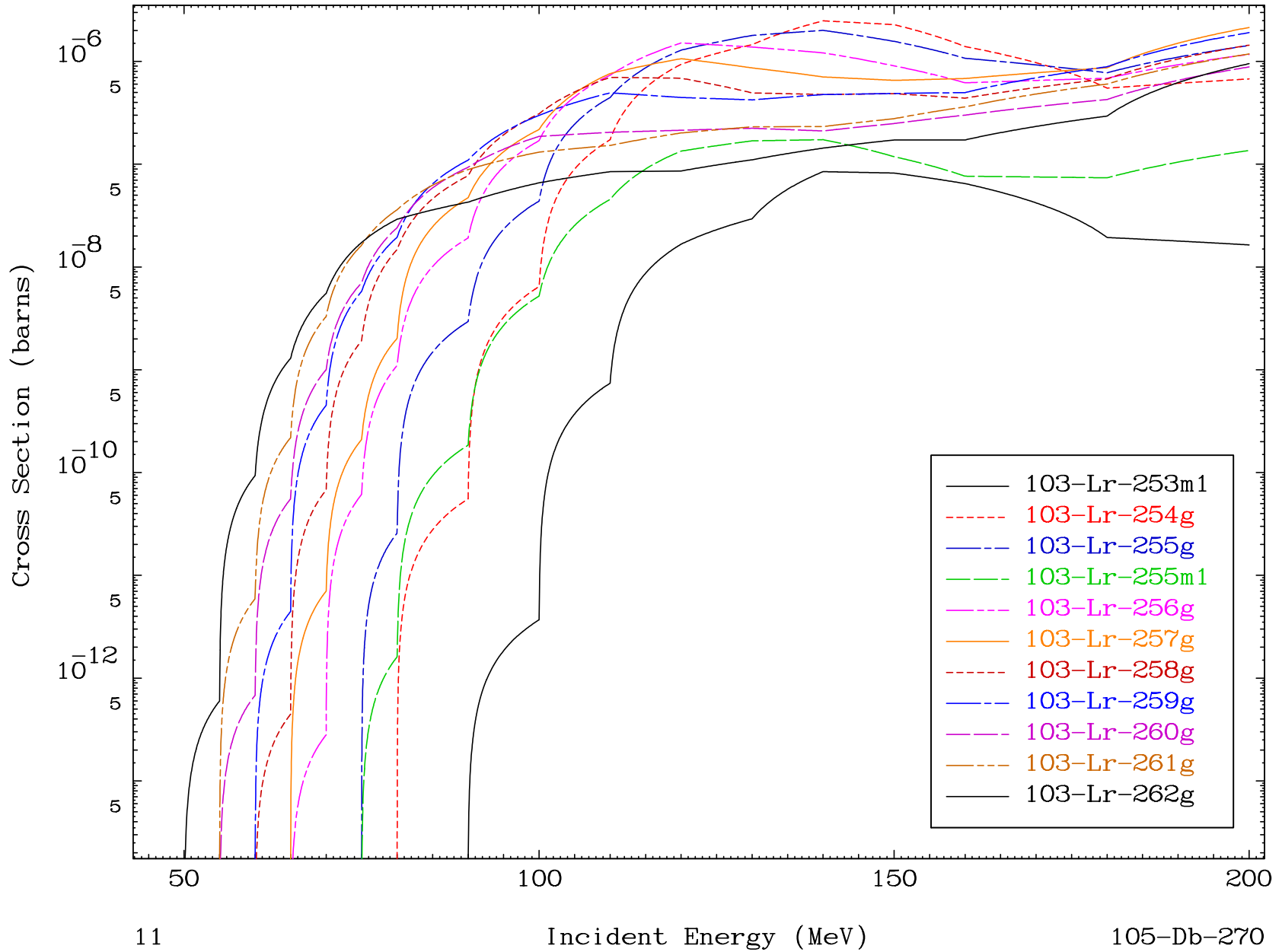
(α , remainder)

105-Db-270

Radionuclide Production Cross Section



Radionuclide Production Cross Section

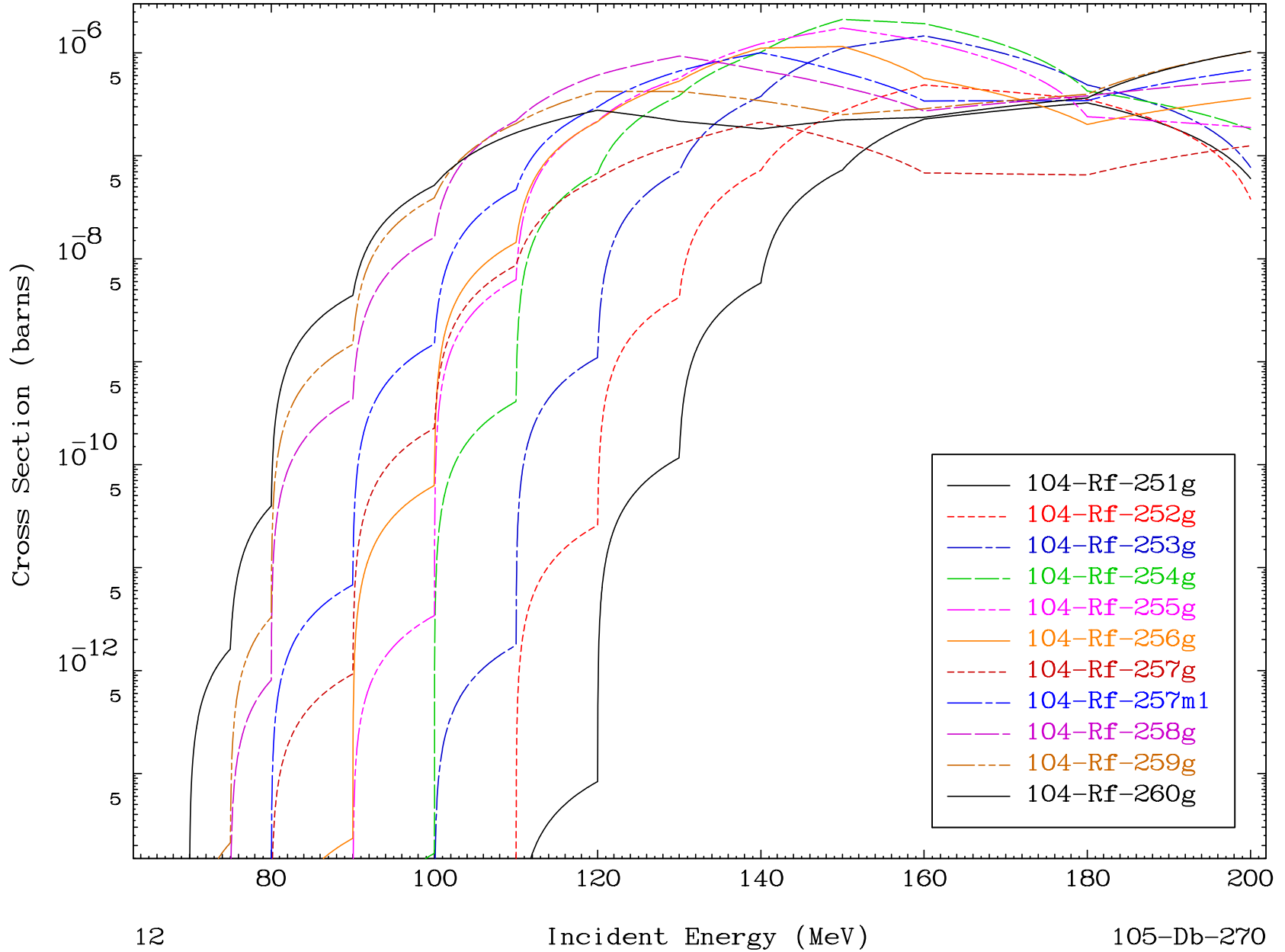


MAT 570

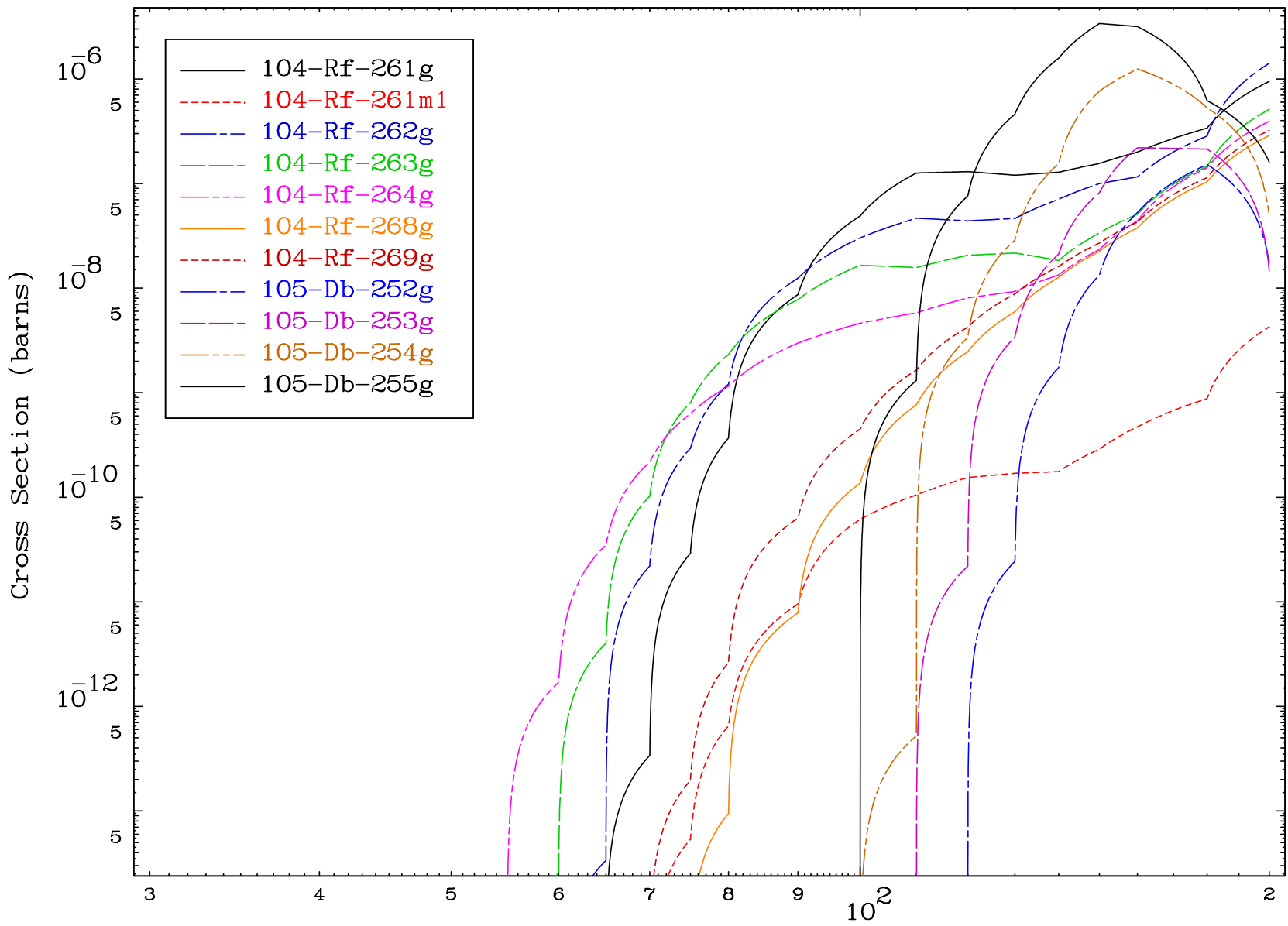
(α , remainder)

105-Db-270

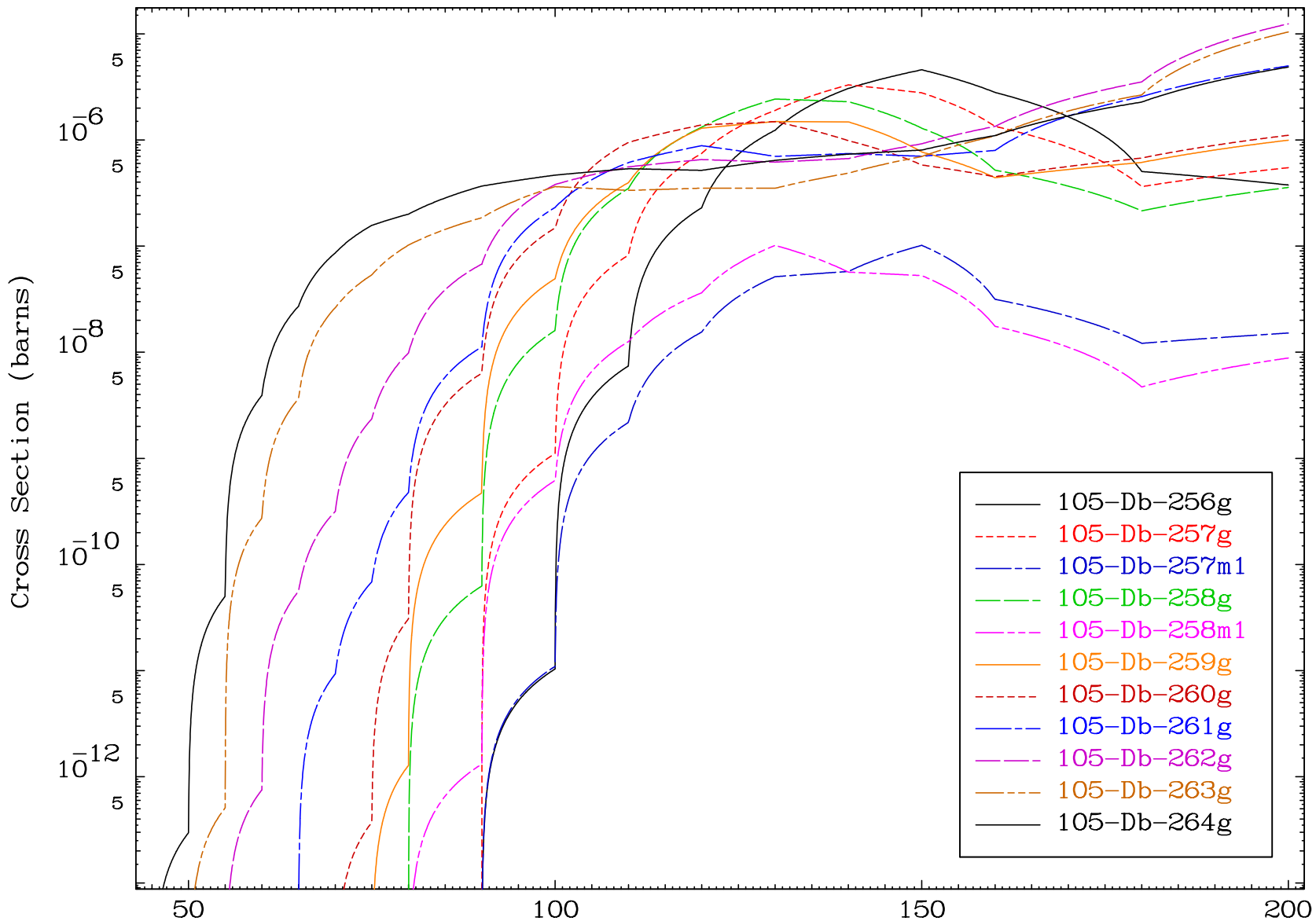
Radionuclide Production Cross Section



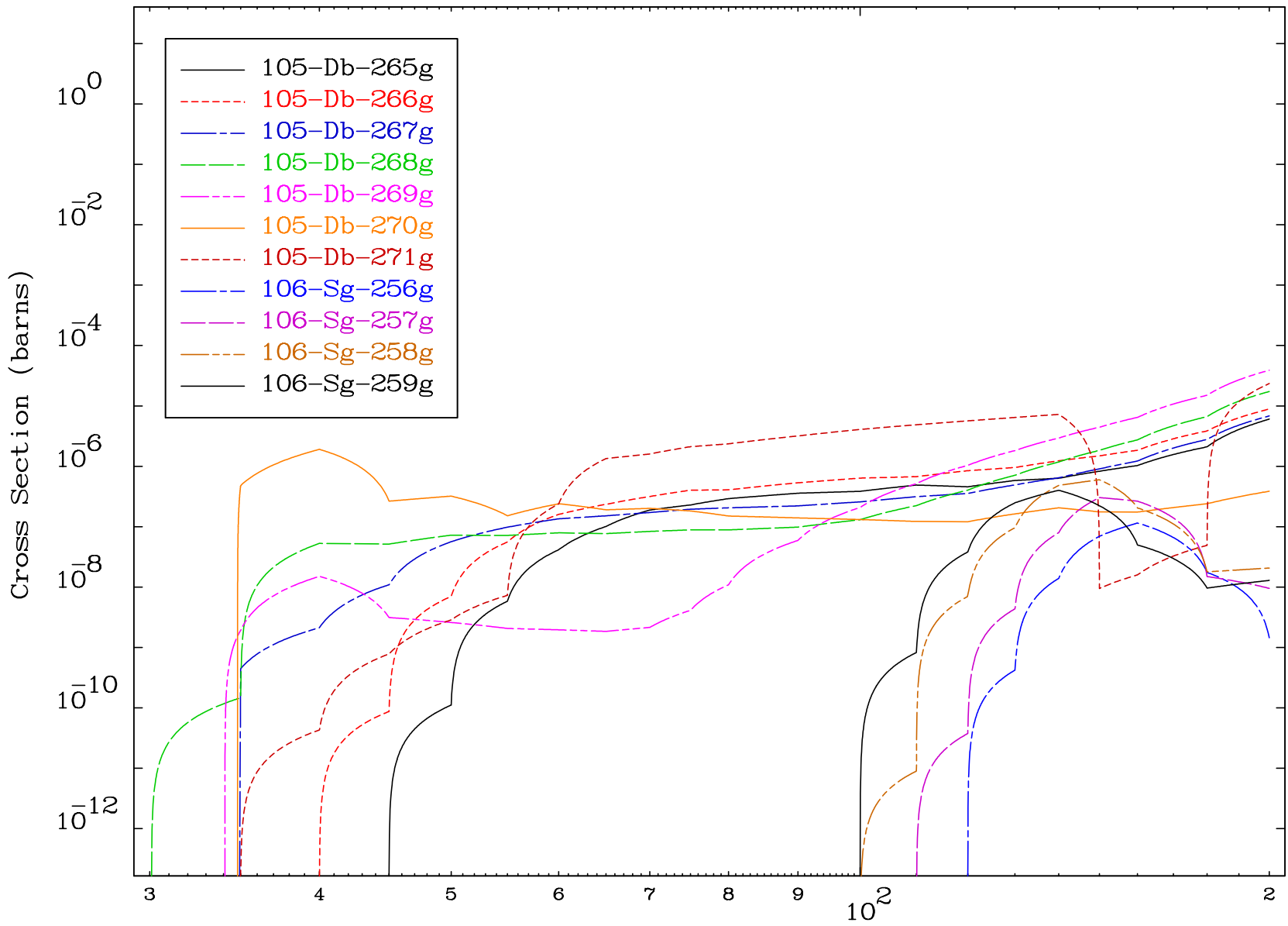
Radionuclide Production Cross Section

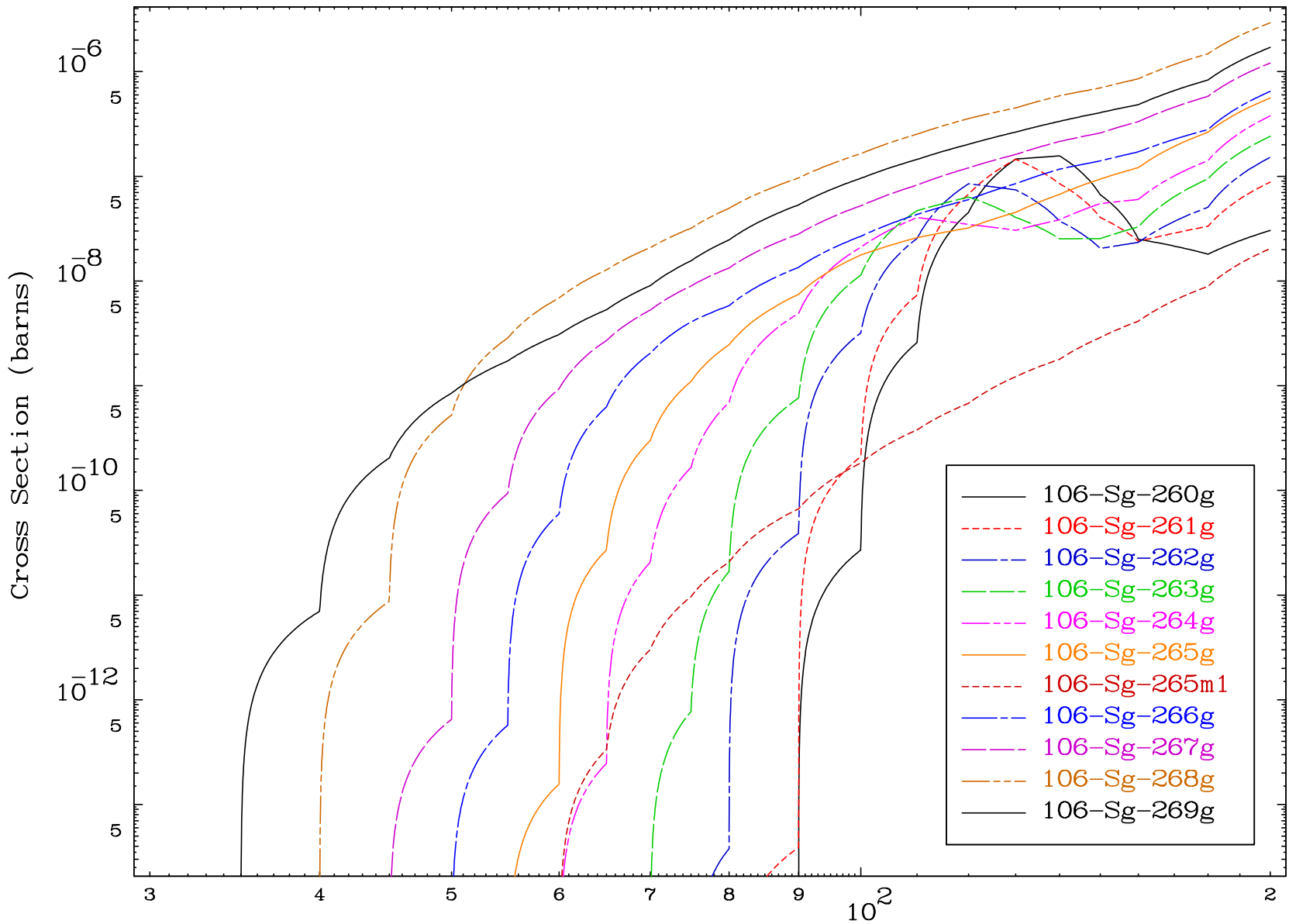


Radionuclide Production Cross Section

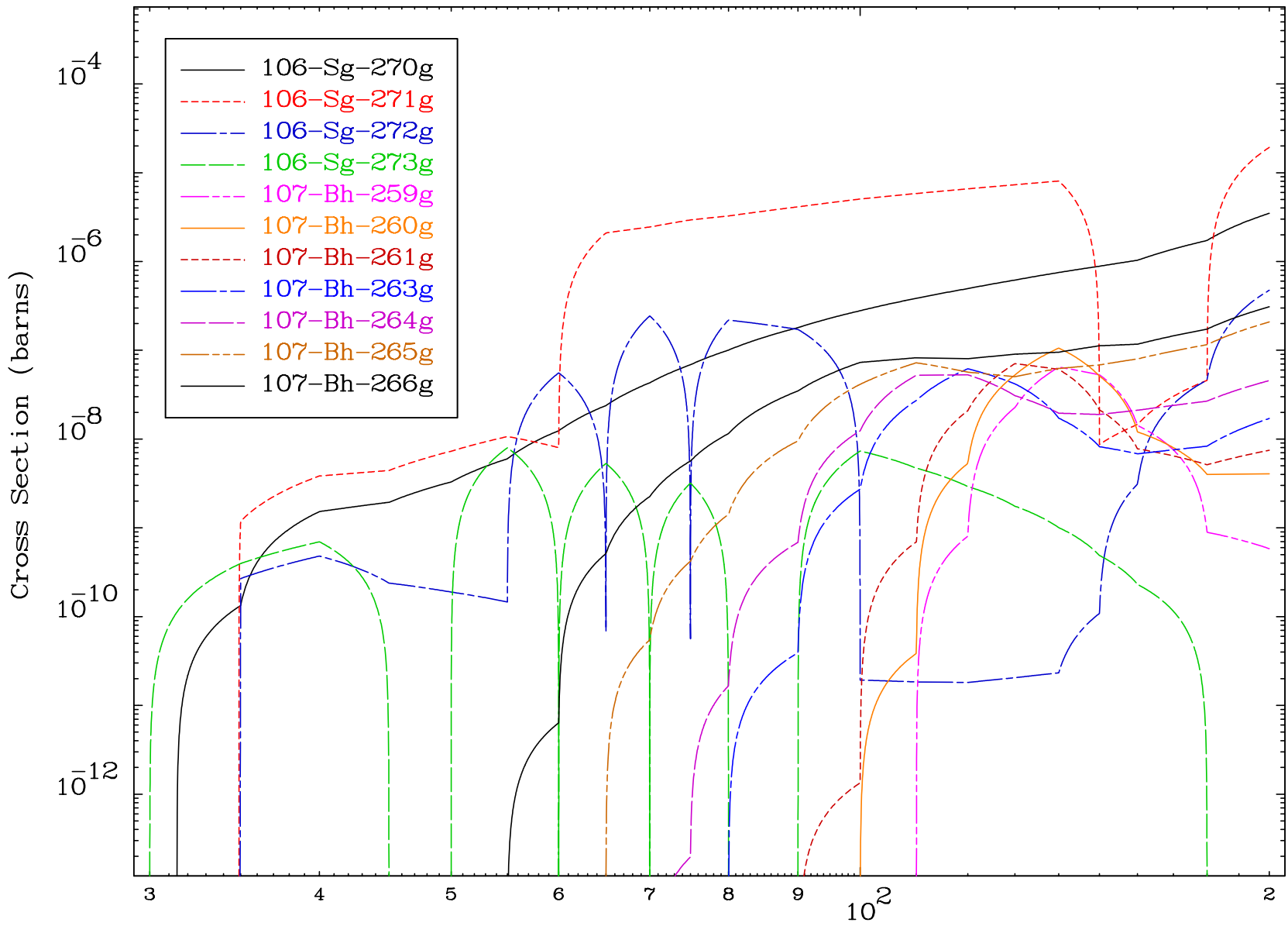


Radionuclide Production Cross Section





Radionuclide Production Cross Section



Radionuclide Production Cross Section

