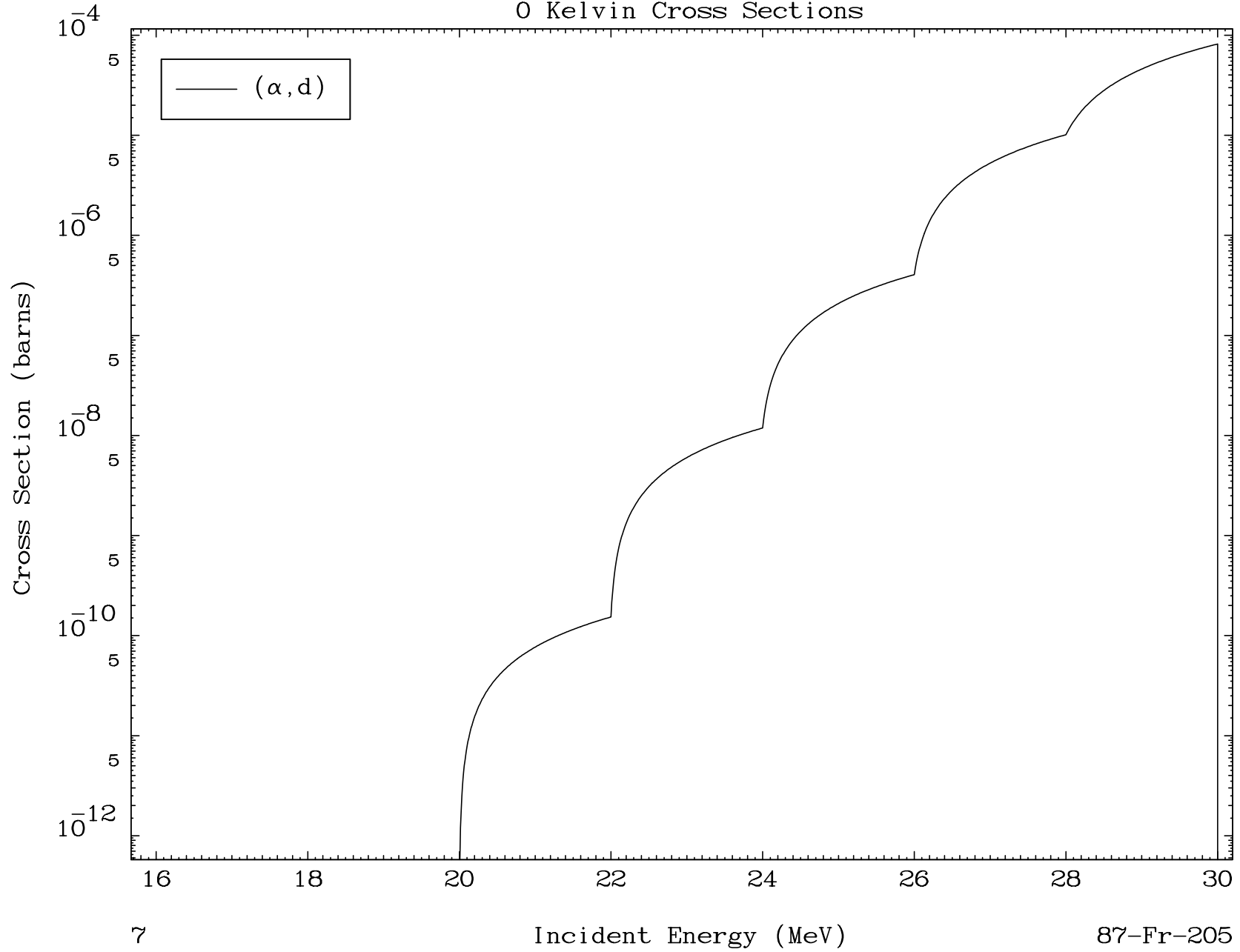
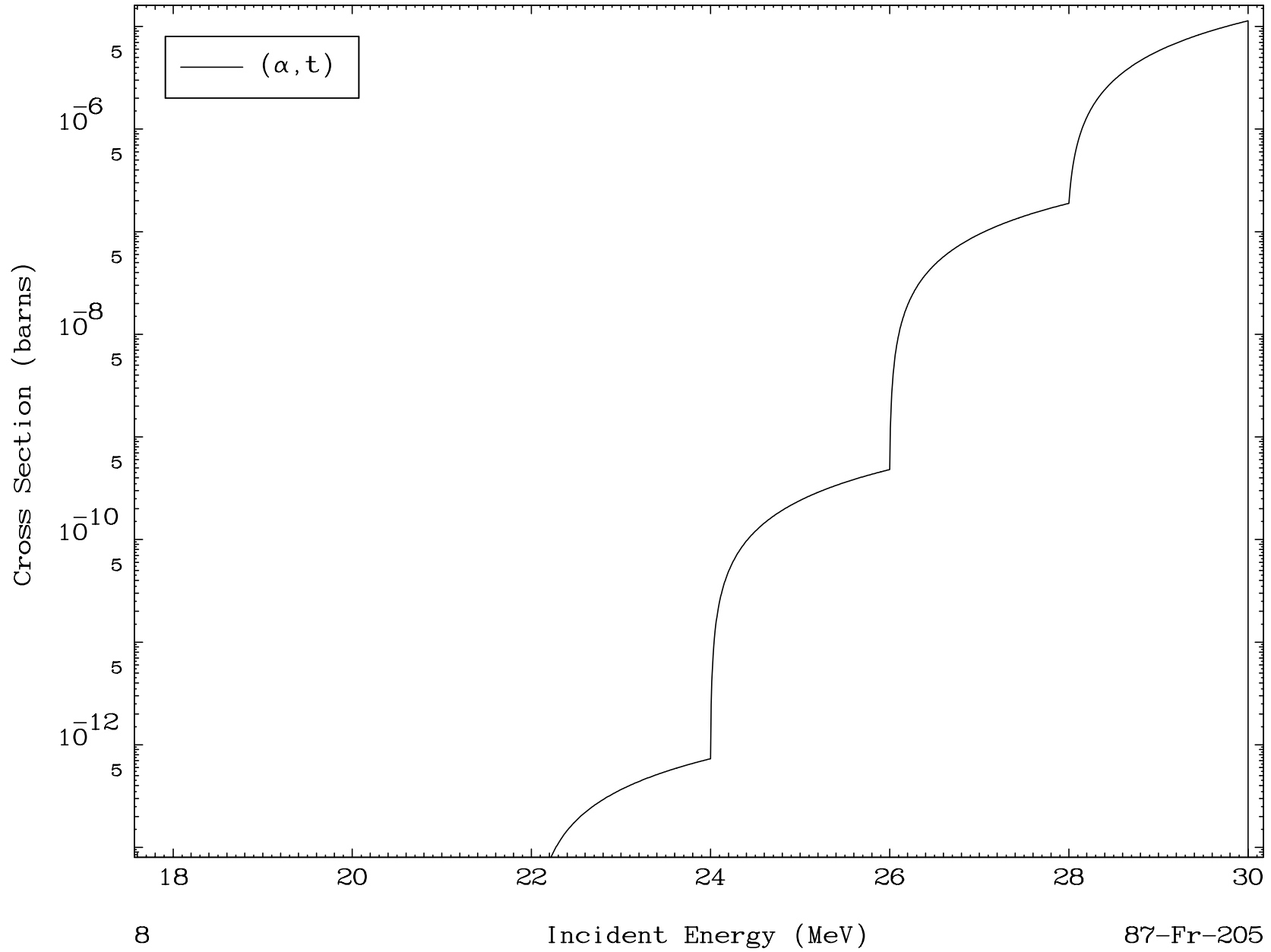


MAT 8704

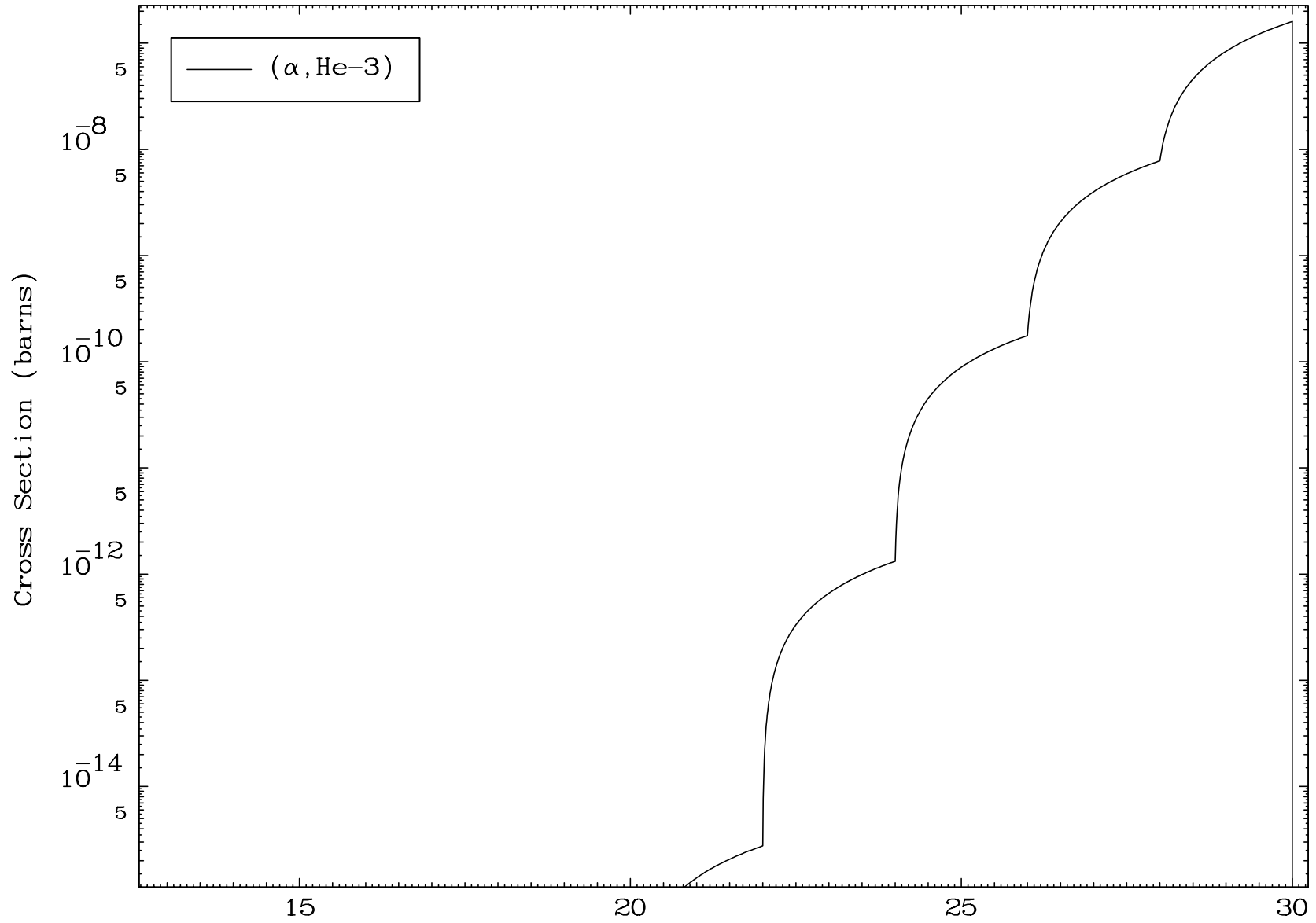
( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

87-Fr-205





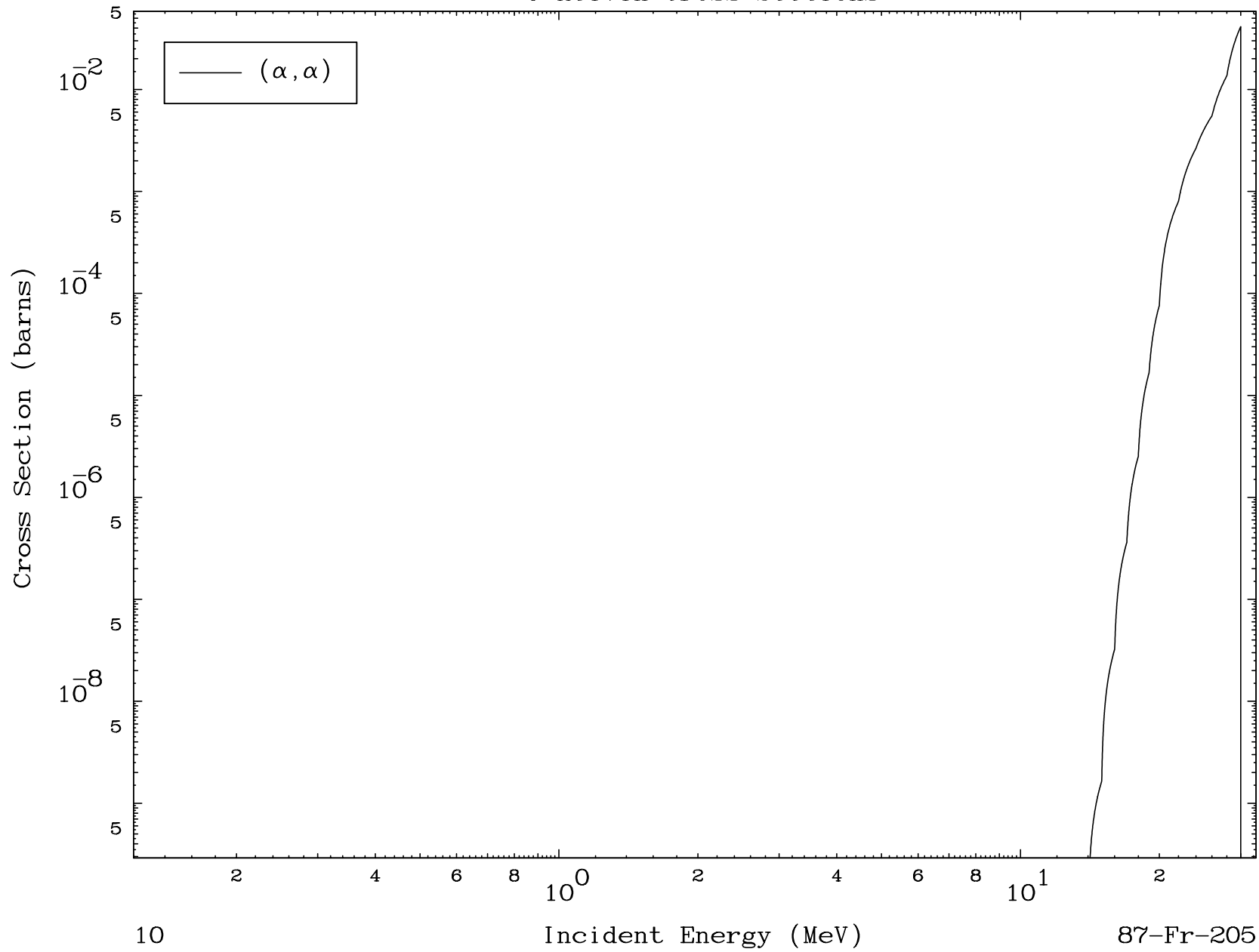




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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

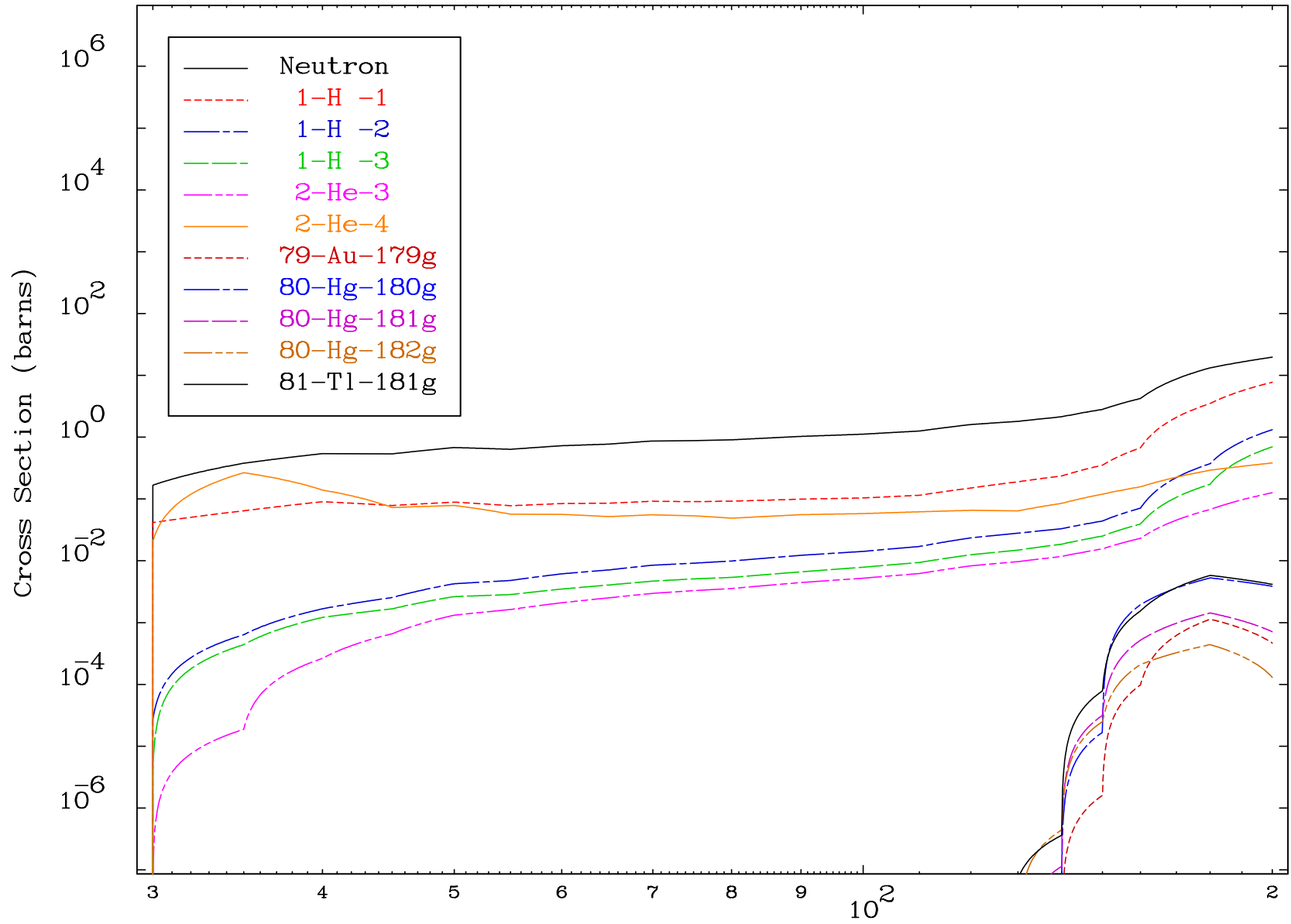
87-Fr-205



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Incident Energy (MeV)

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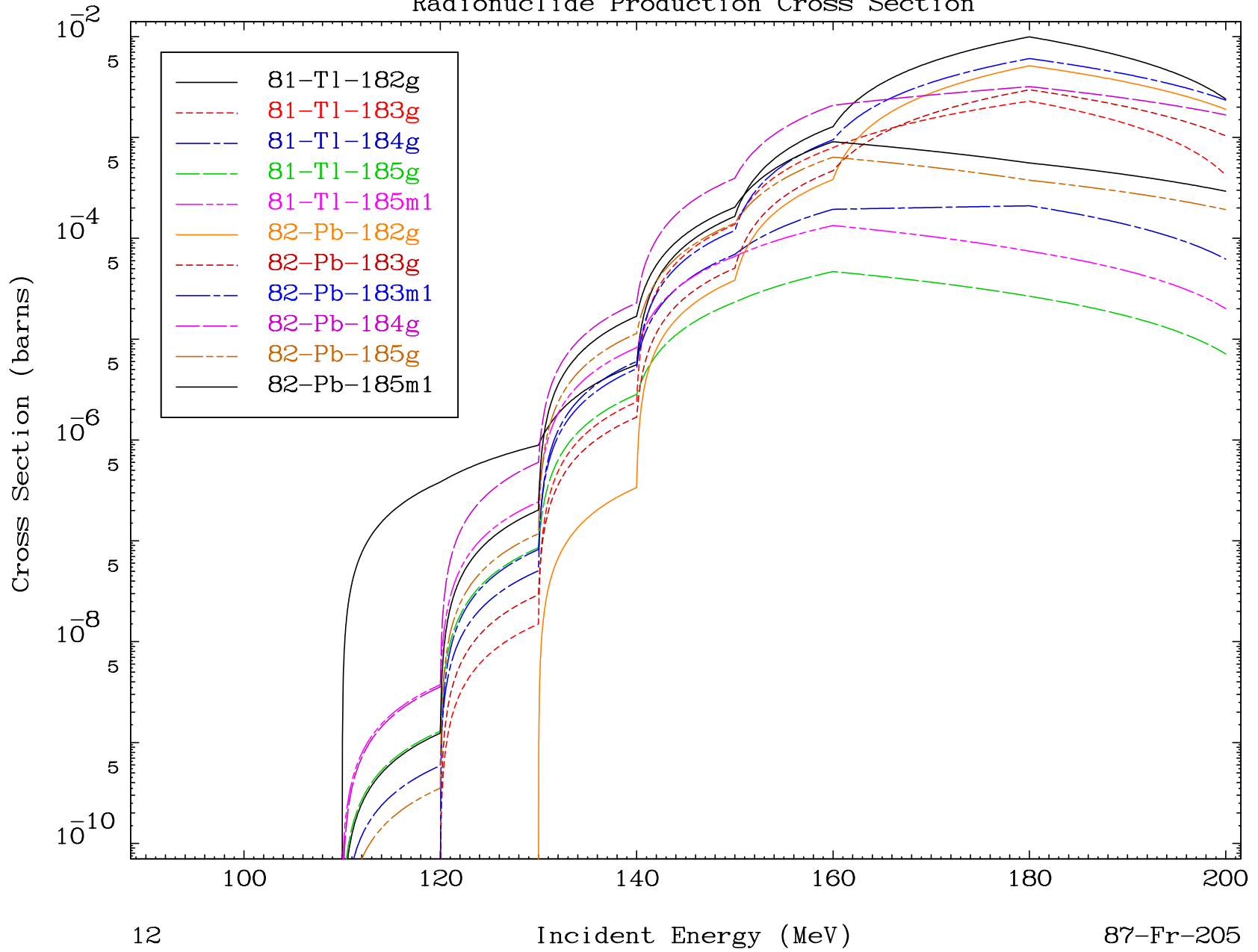


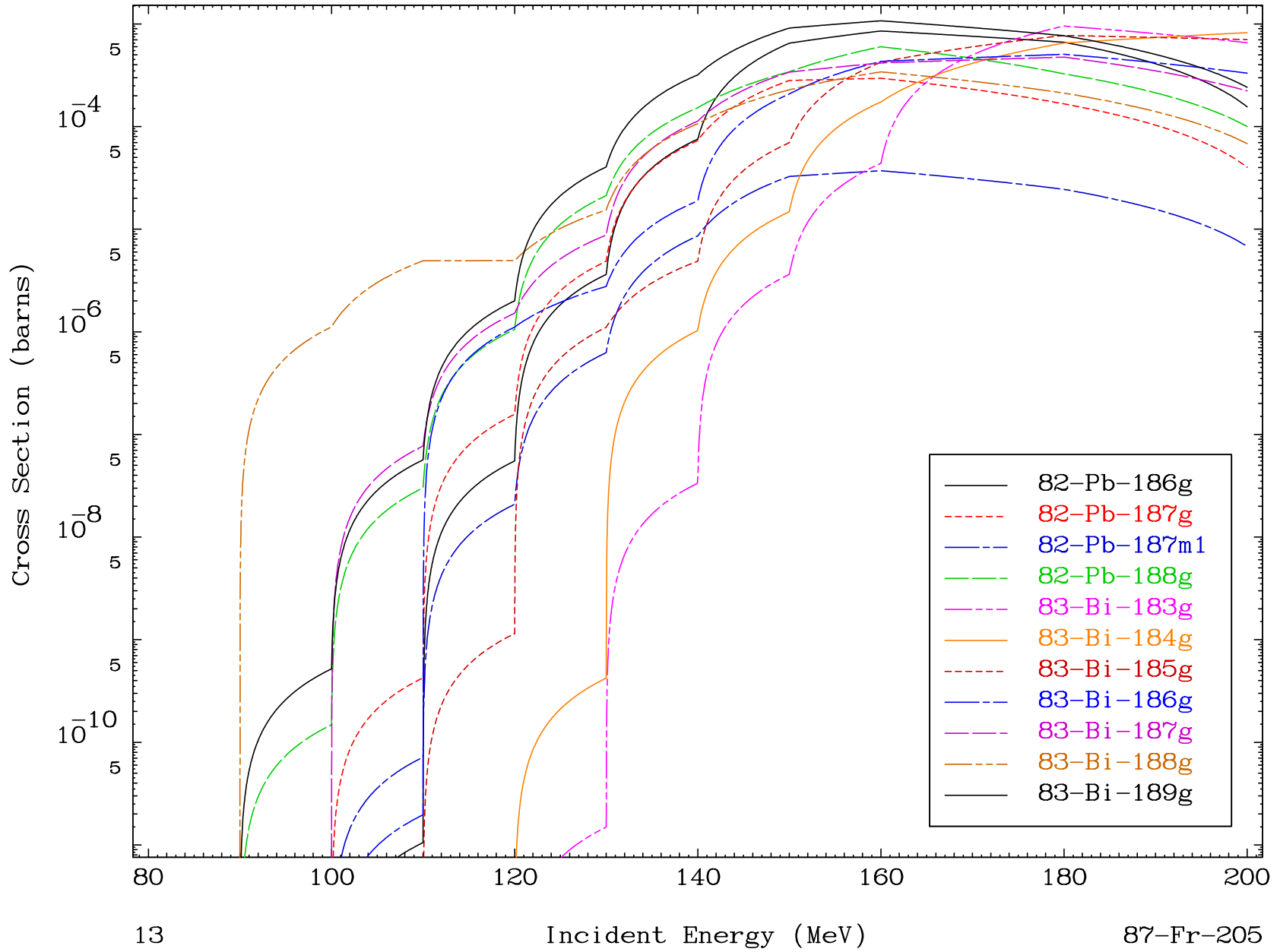
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( $\alpha$ , remainder)

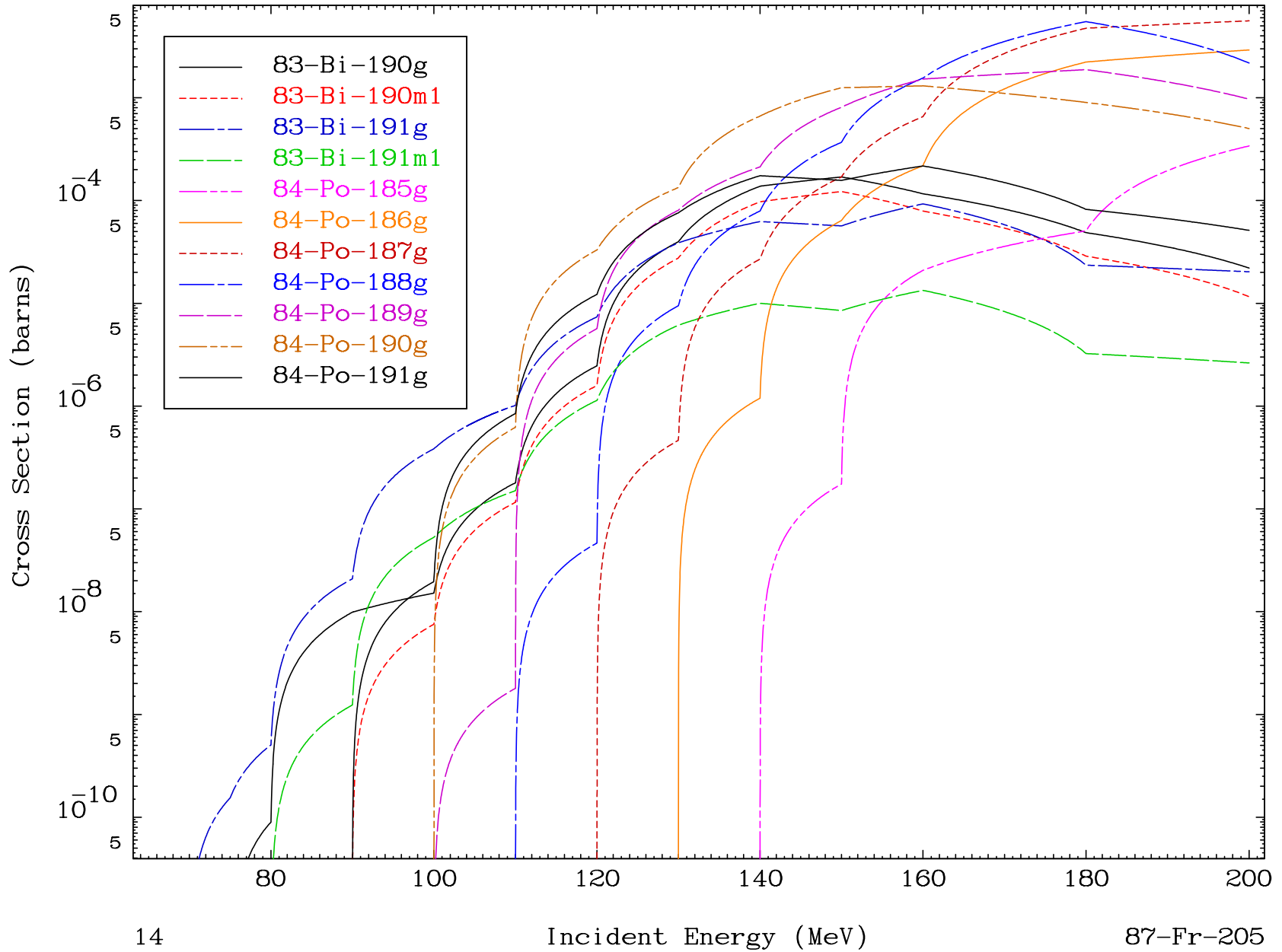
87-Fr-205

Radionuclide Production Cross Section





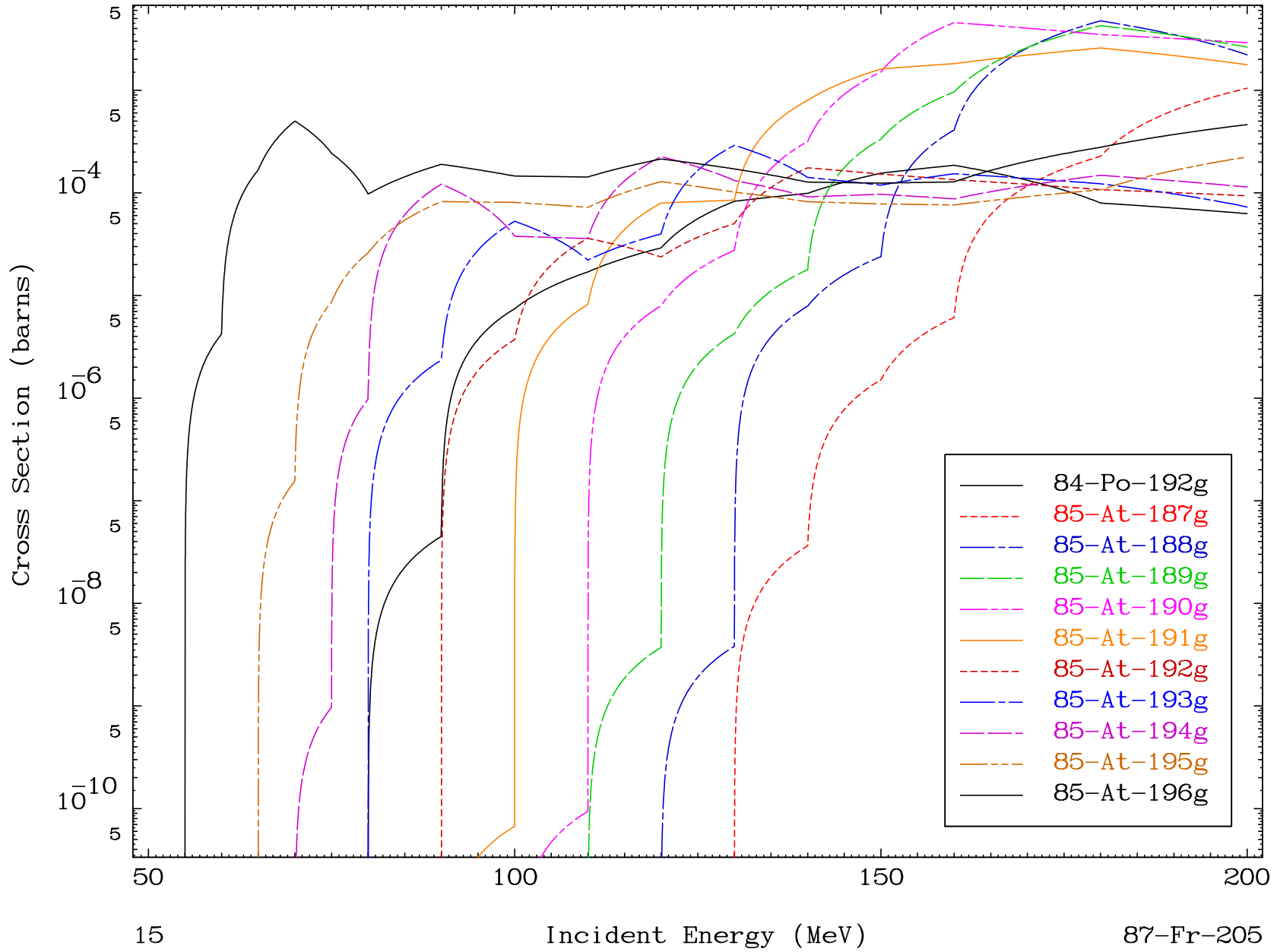
Radionuclide Production Cross Section



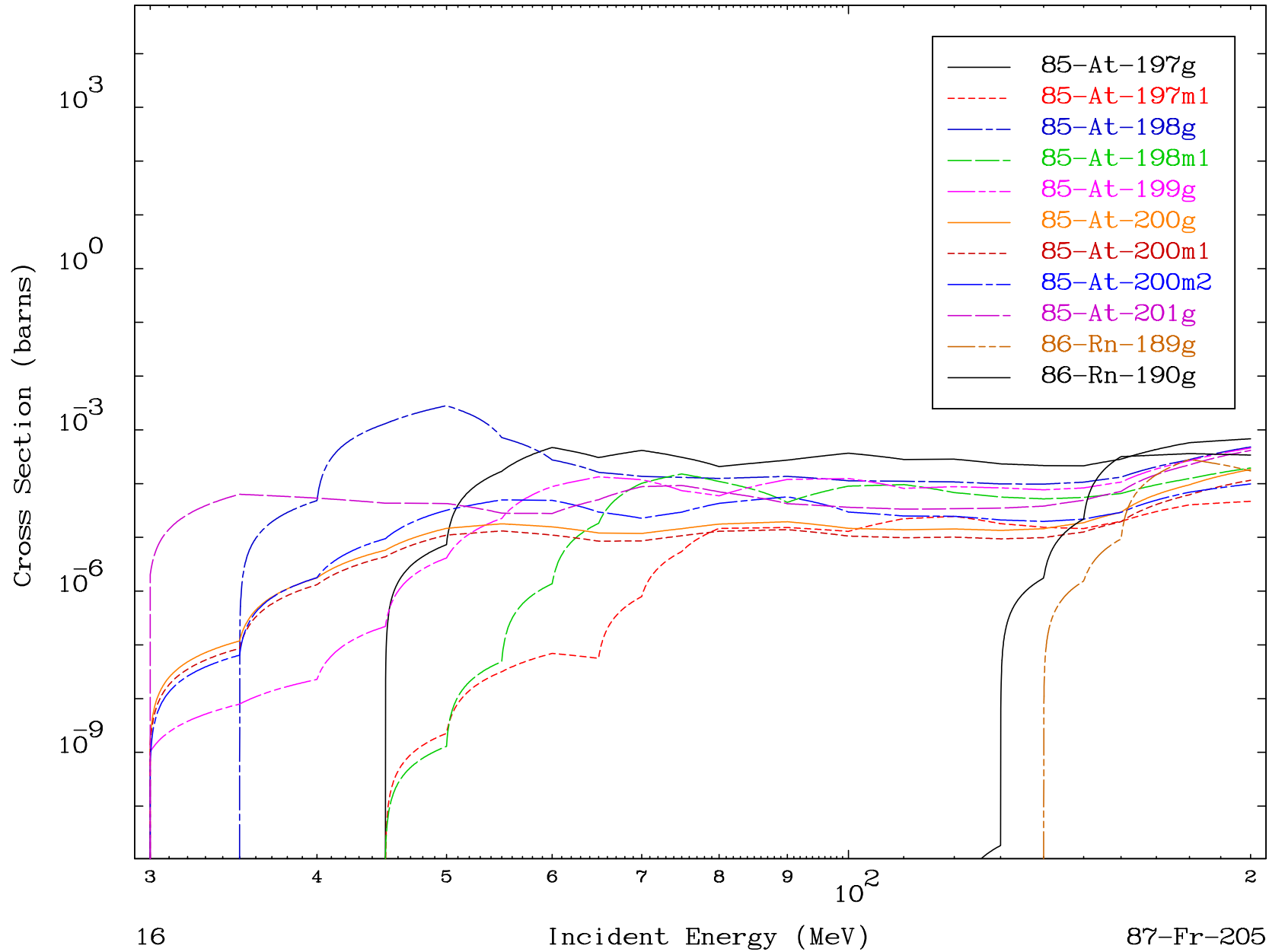
MAT 8704

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

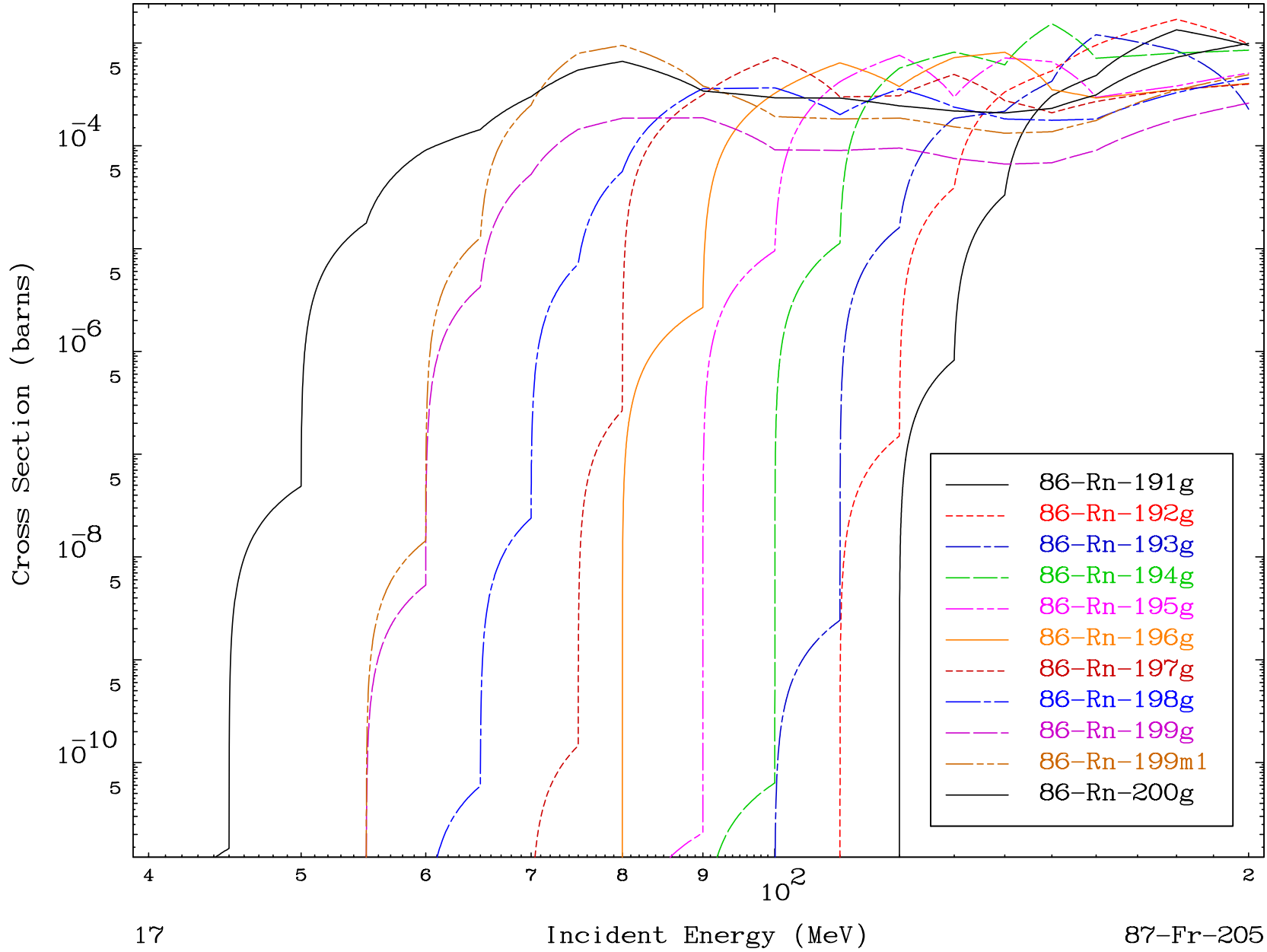
87-Fr-205

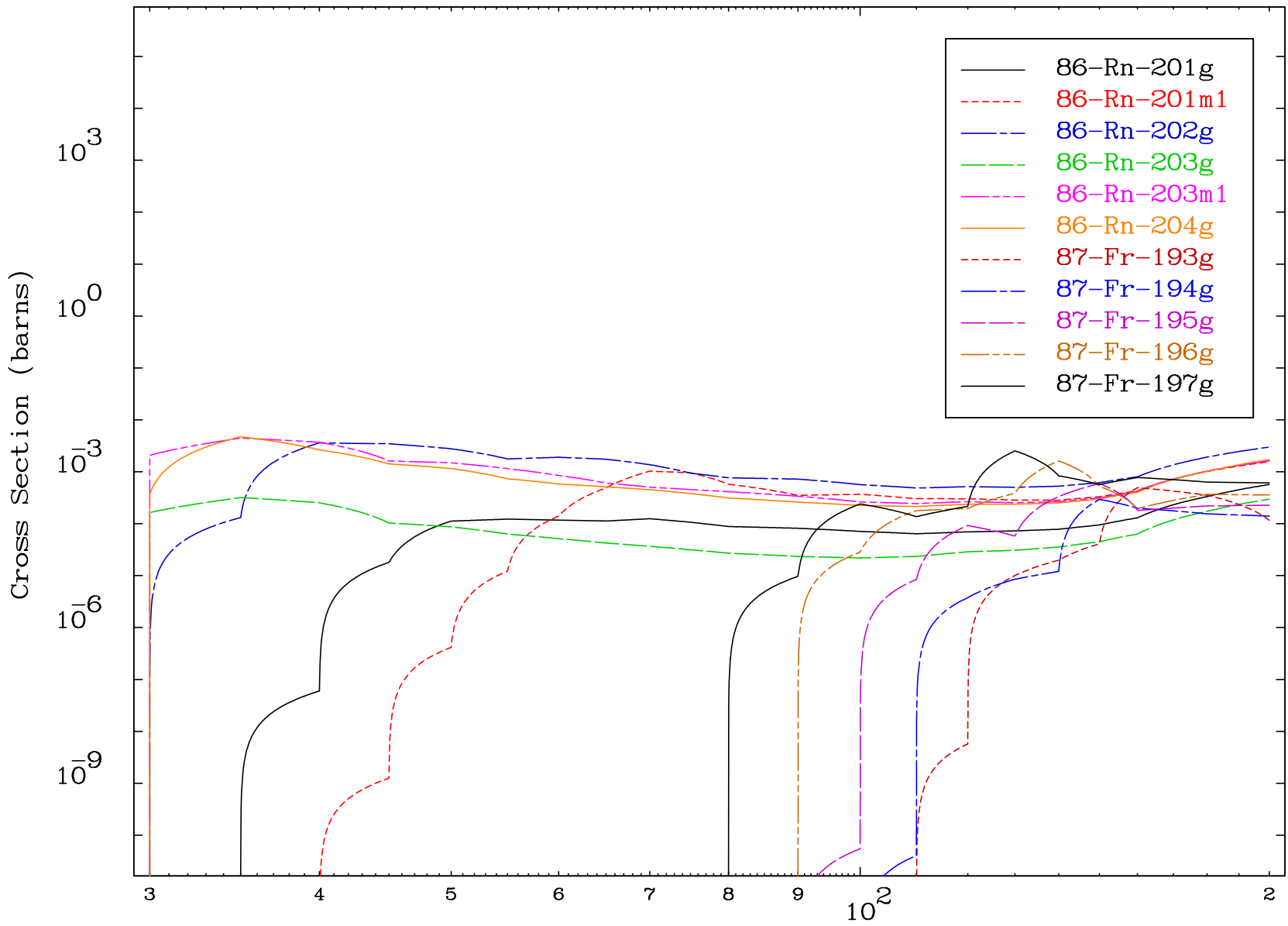


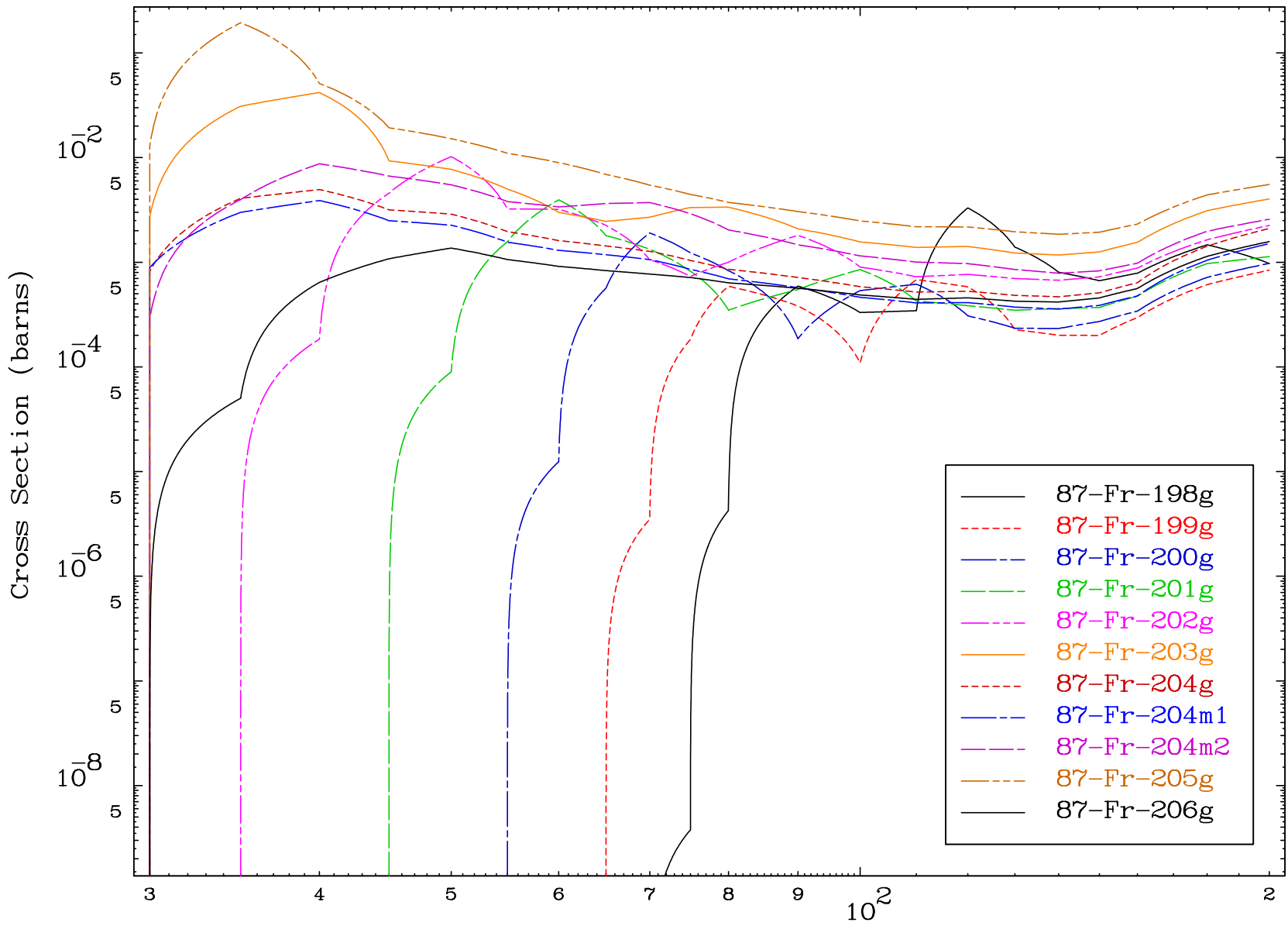
Radionuclide Production Cross Section









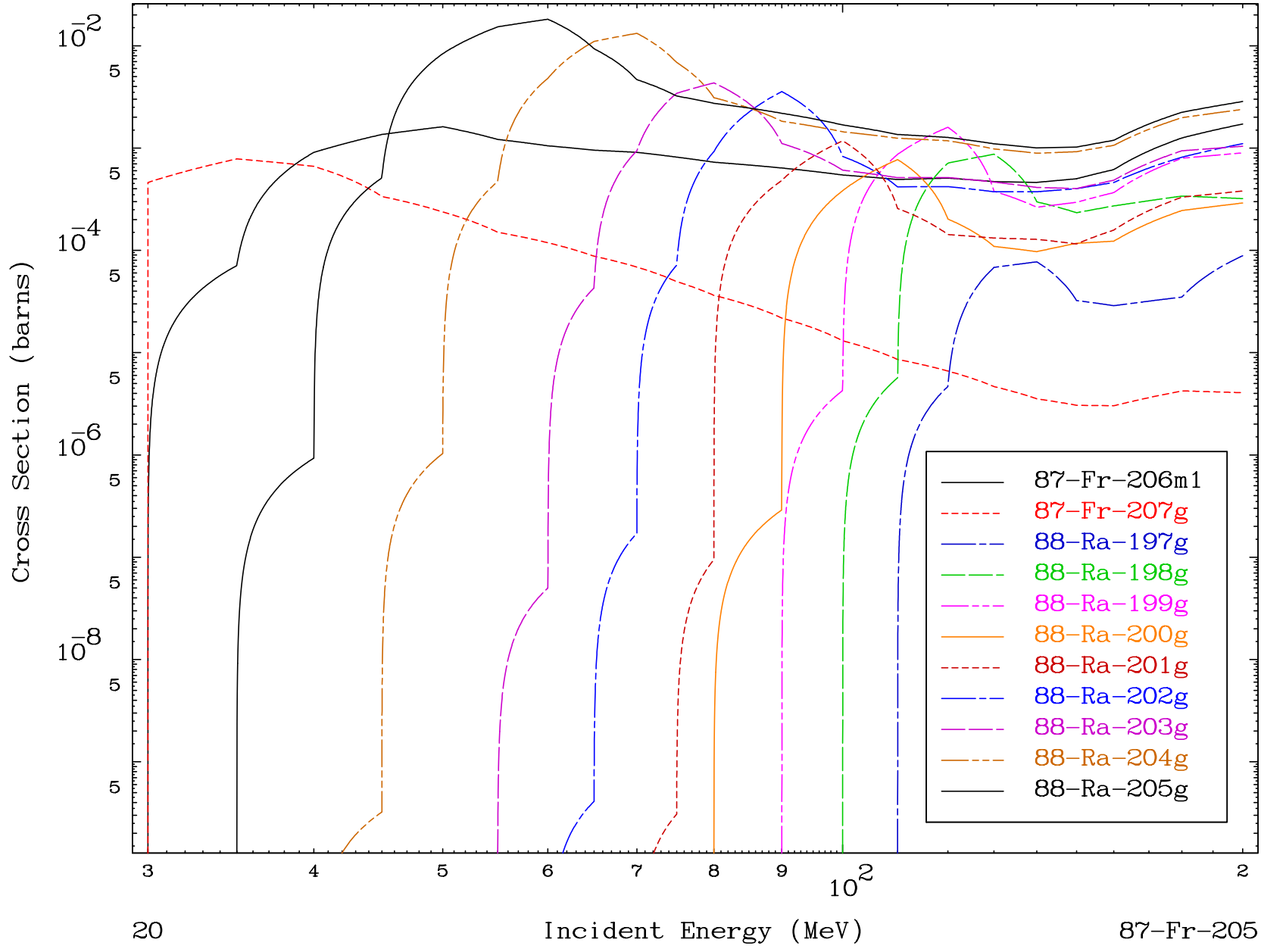


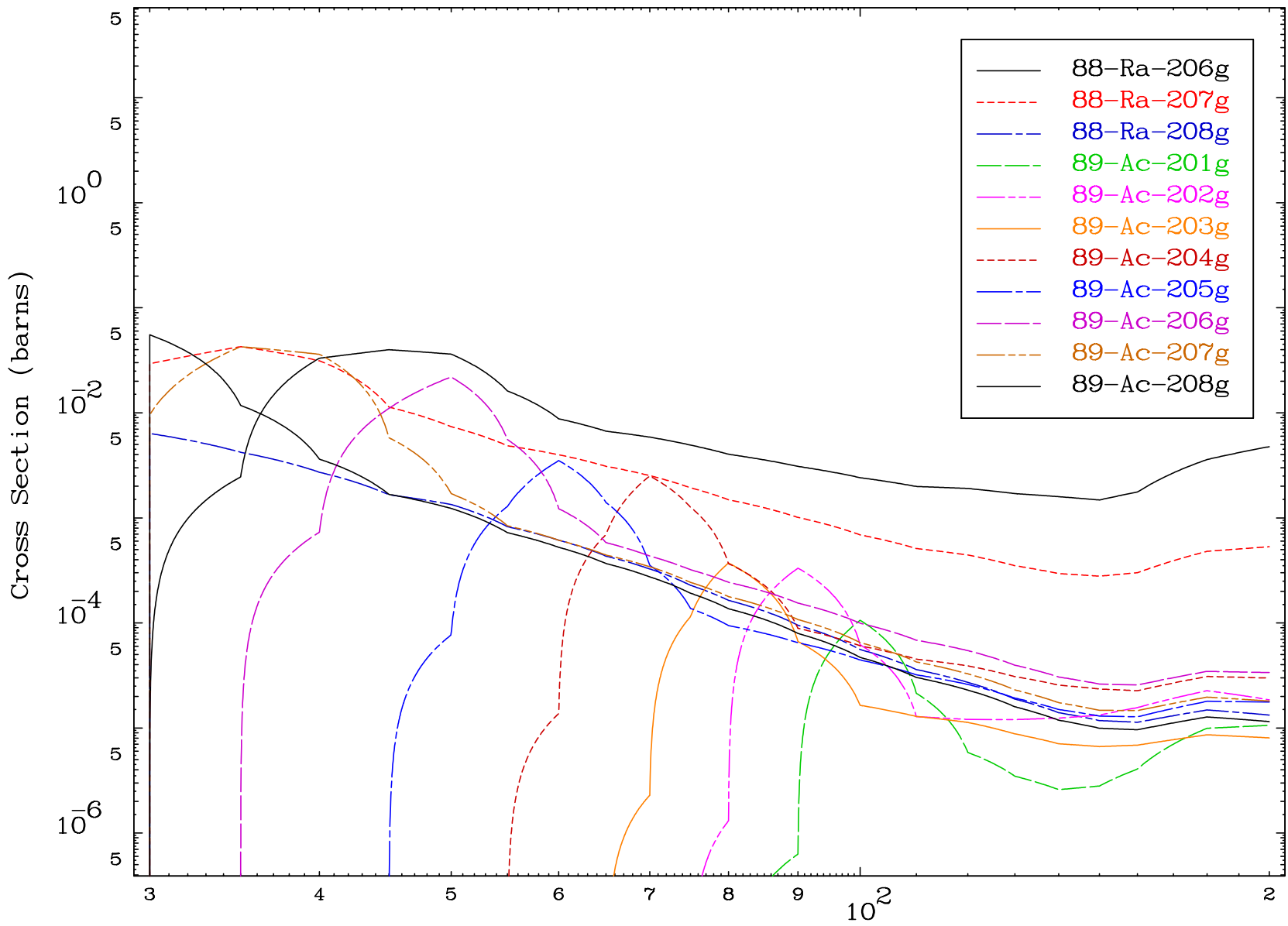
MAT 8704

( $\alpha$ , remainder)

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### Radionuclide Production Cross Section



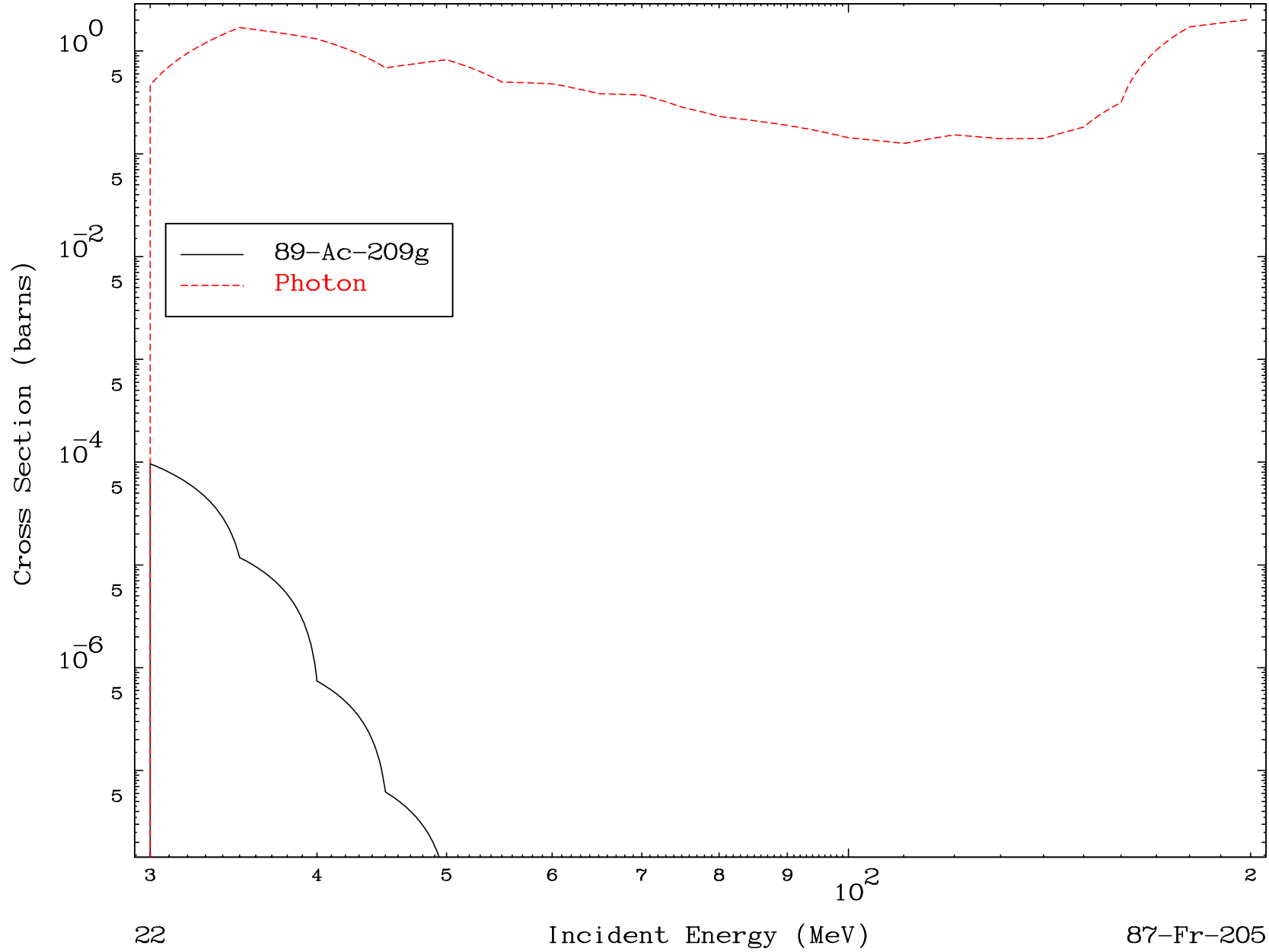


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( $\alpha$ , remainder)

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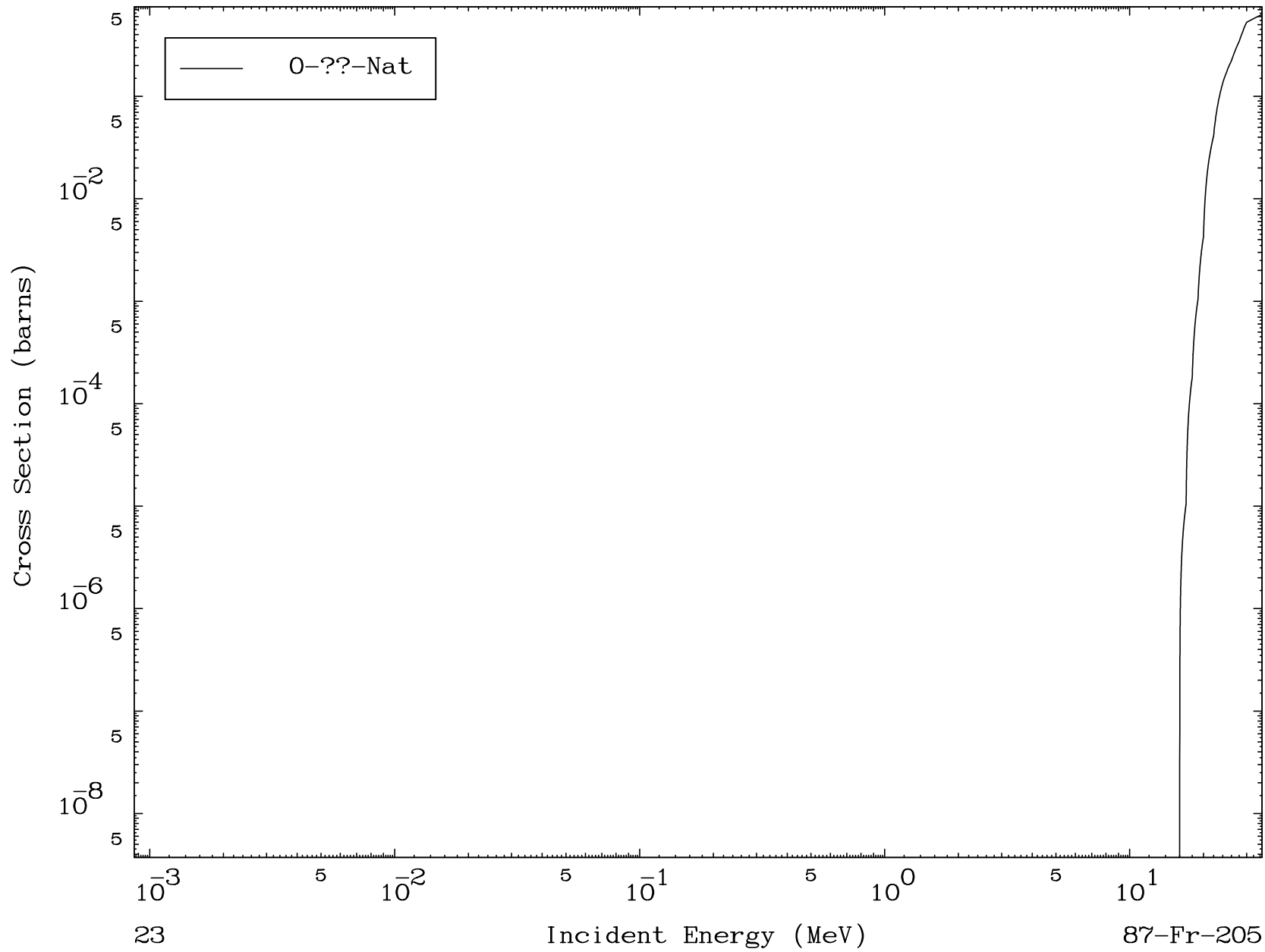
Radionuclide Production Cross Section



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$\alpha$  Fission  
Radionuclide Production Cross Section

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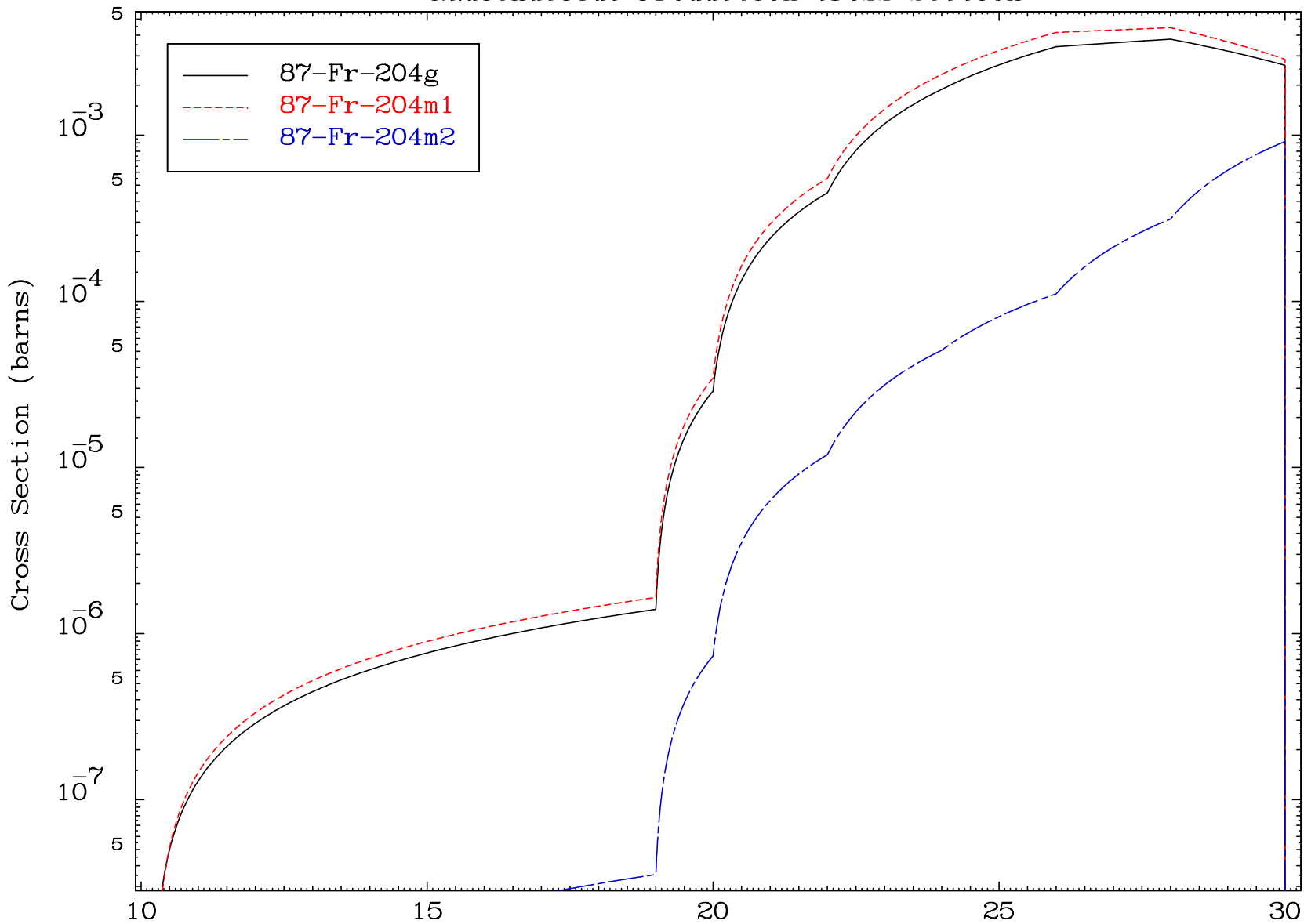


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( $\alpha, n'$ )  $\alpha$

87-Fr-205

Radionuclide Production Cross Section



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Incident Energy (MeV)

87-Fr-205

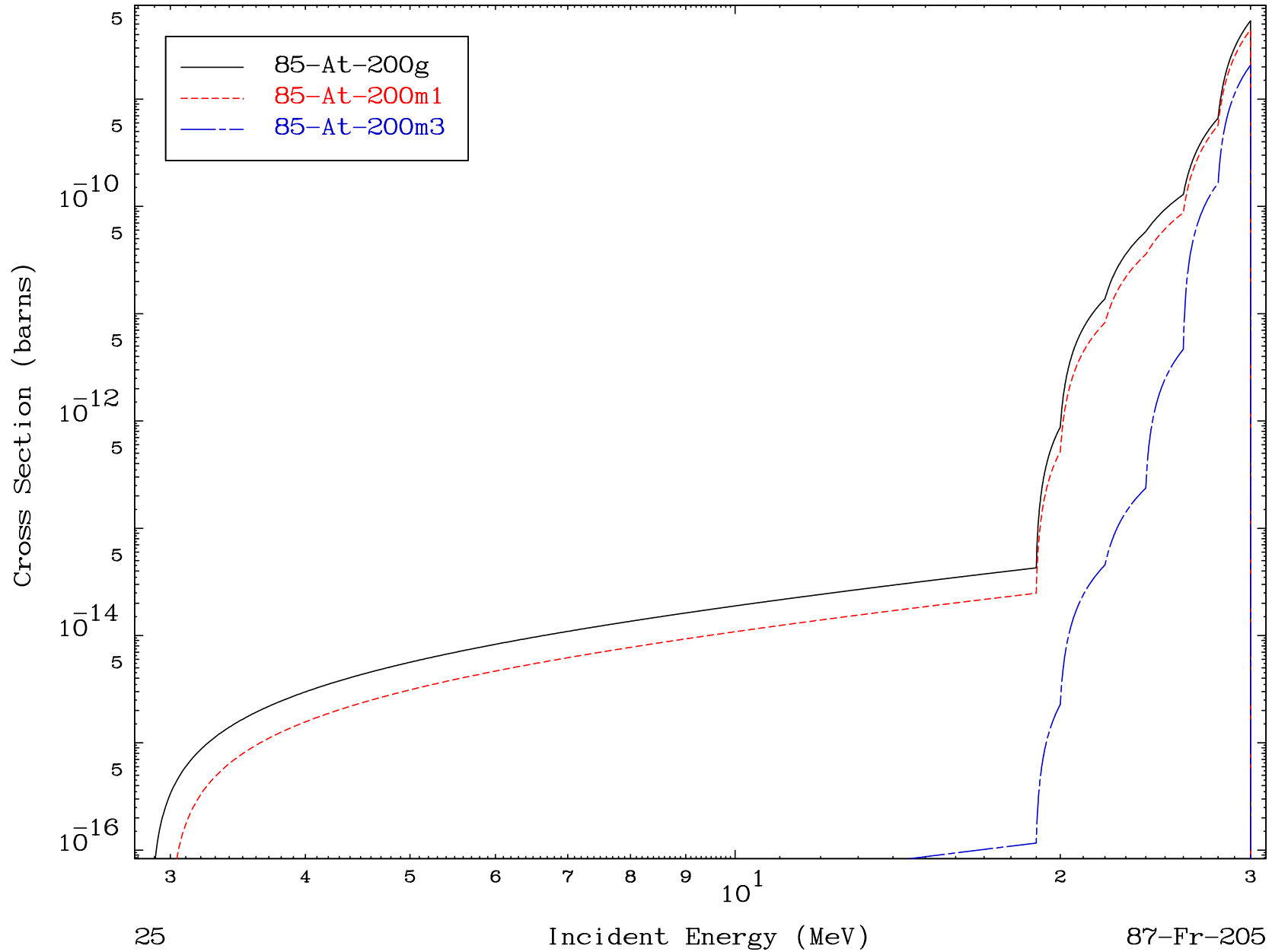


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( $\alpha, n'$ )  $2\alpha$

87-Fr-205

Radionuclide Production Cross Section

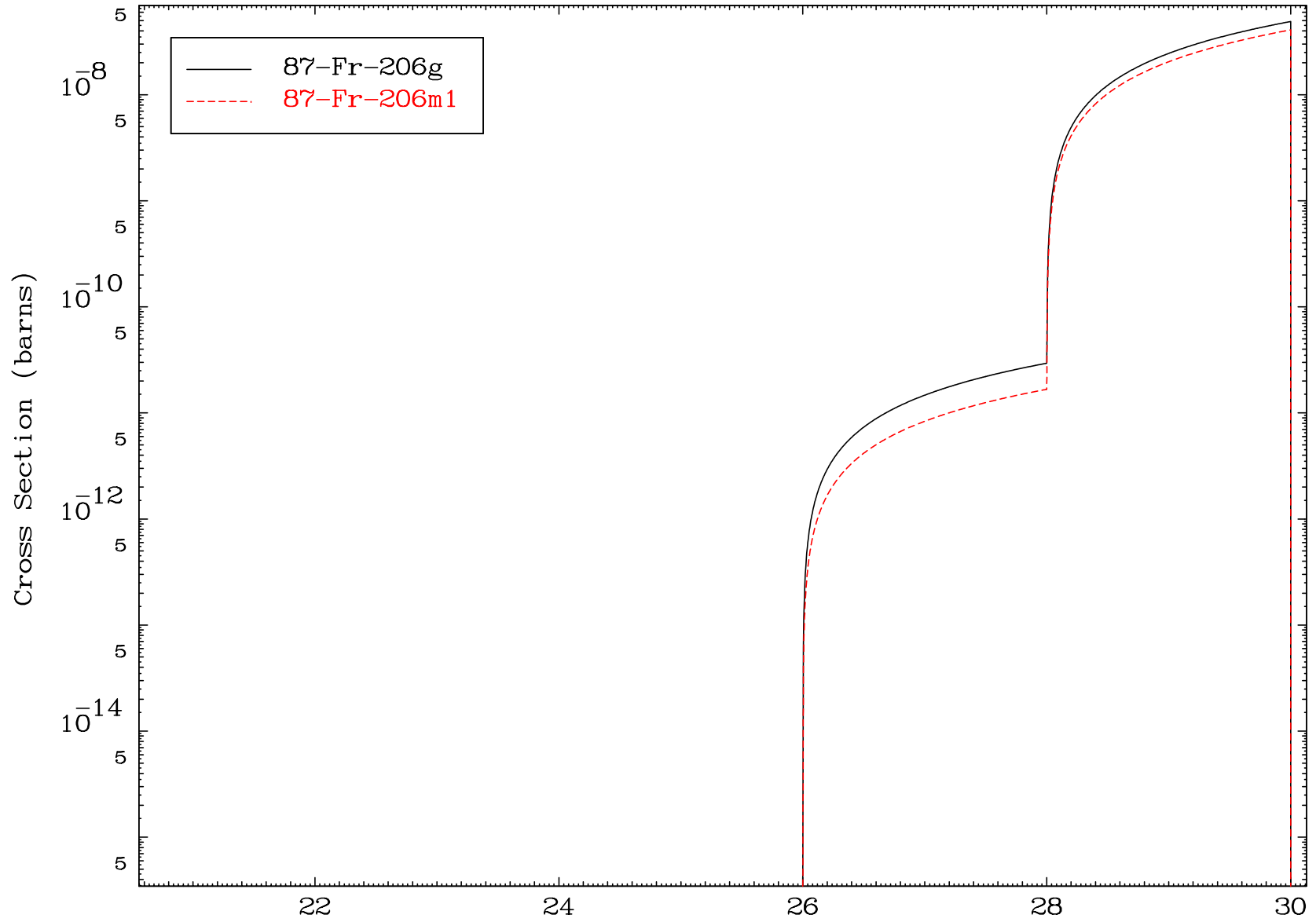


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( $\alpha, 2n$ ) p

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Radionuclide Production Cross Section



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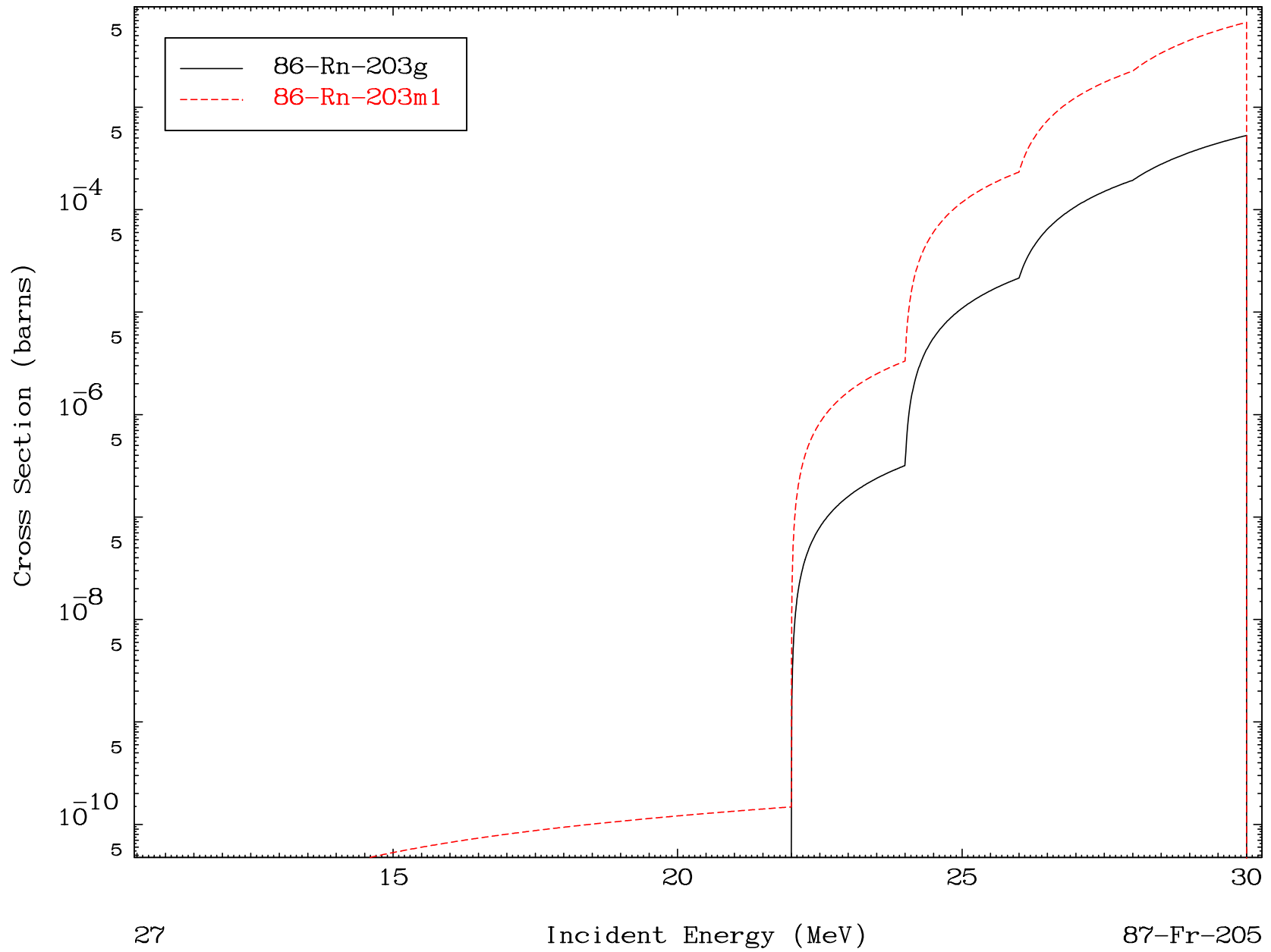
Incident Energy (MeV)

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( $\alpha, n'$ ) p  $\alpha$   
Radionuclide Production Cross Section

87-Fr-205

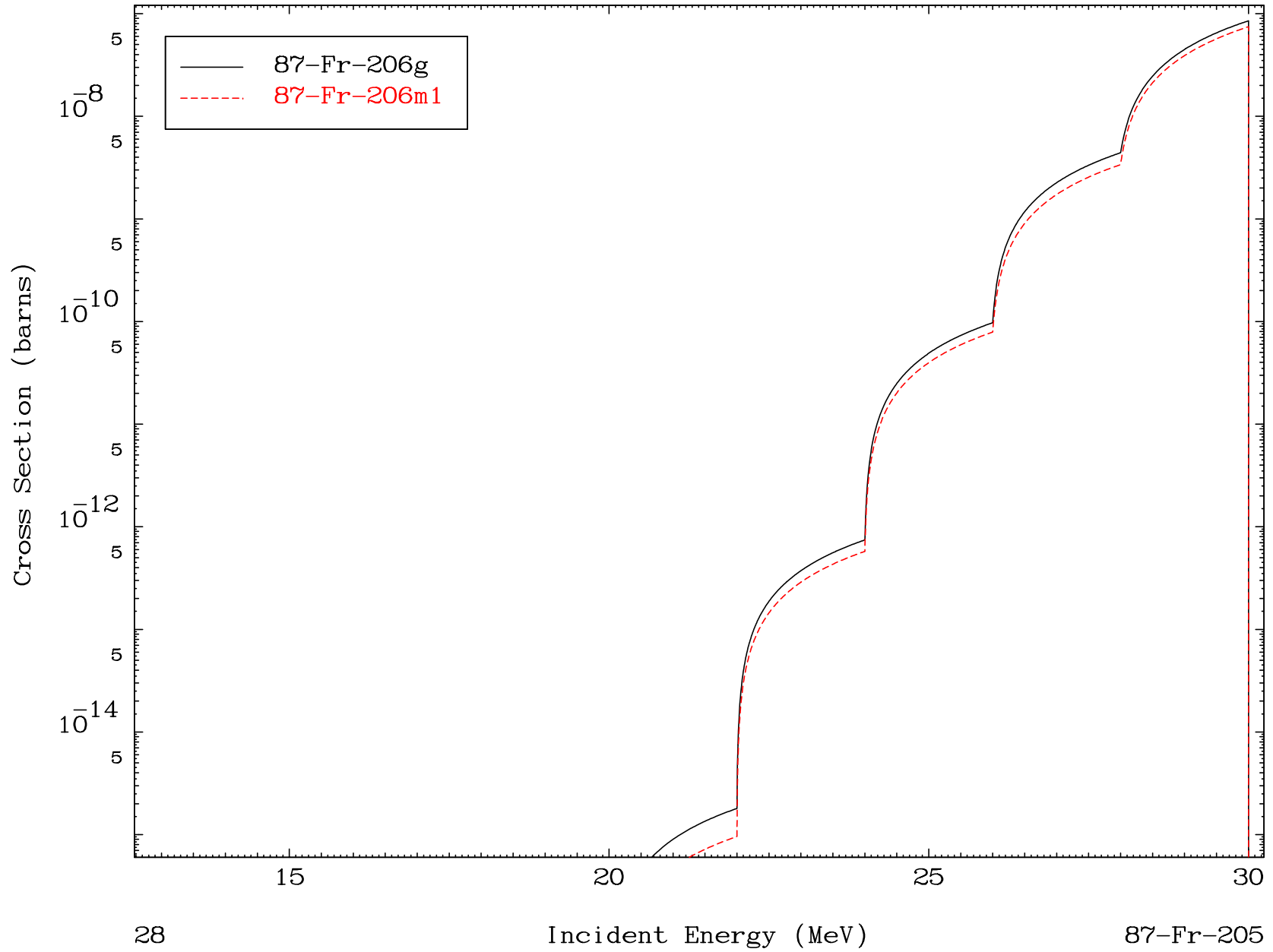


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( $\alpha, \text{He-3}$ )

87-Fr-205

Radionuclide Production Cross Section

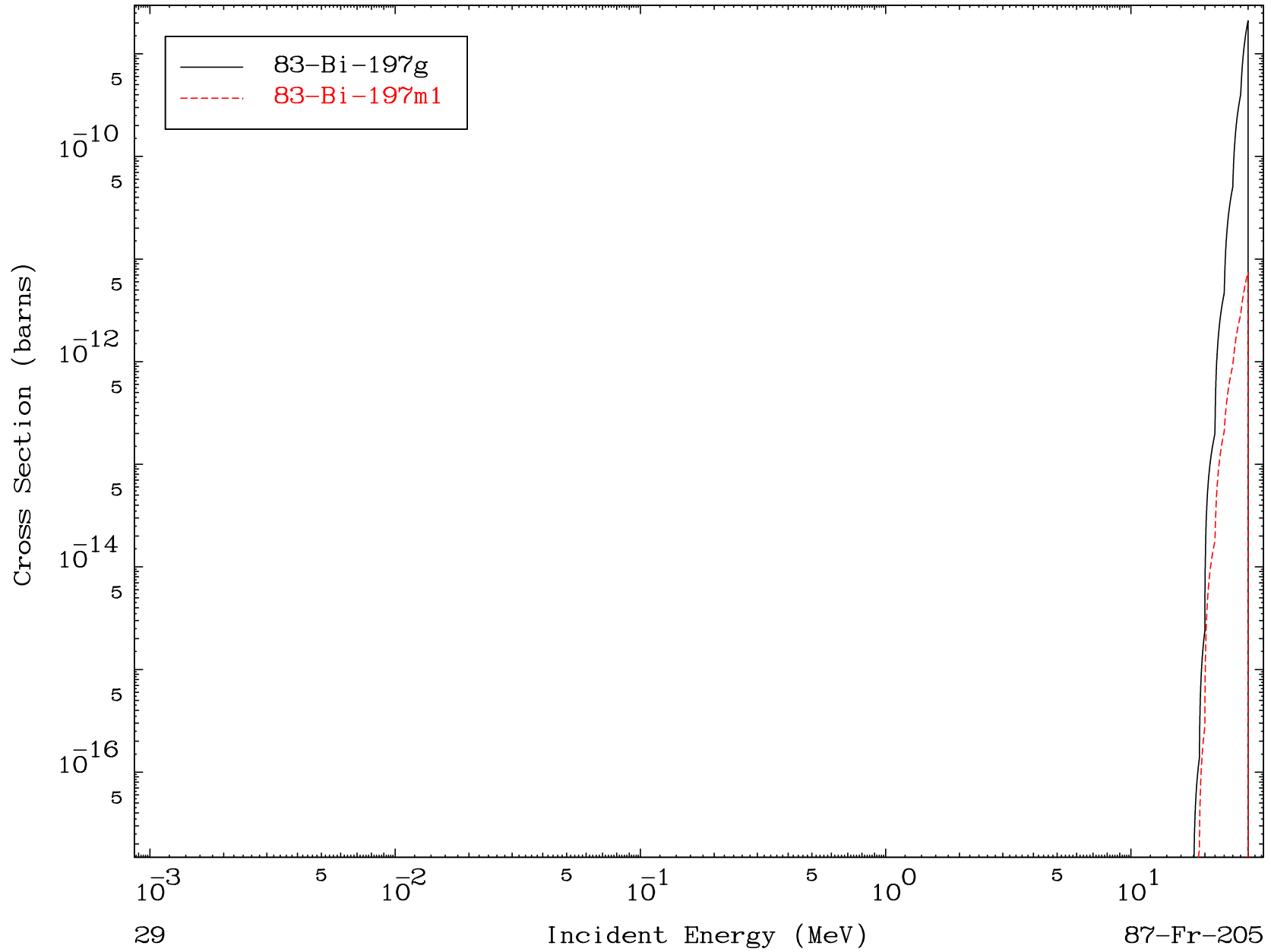


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( $\alpha, 3\alpha$ )

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Radionuclide Production Cross Section



MAT 8704

( $\alpha, d$ )  $\alpha$

87-Fr-205

Radionuclide Production Cross Section

