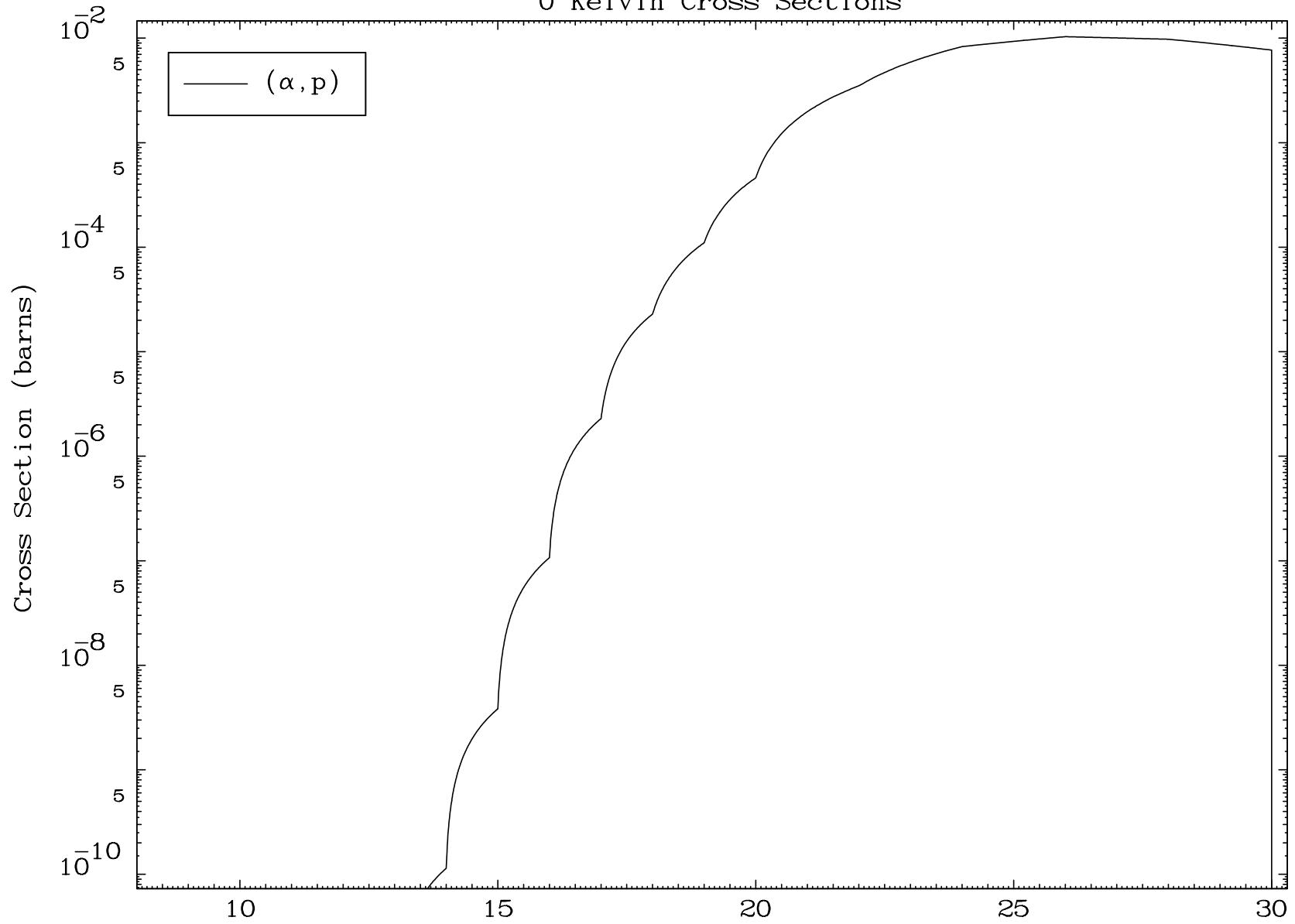


MAT 8710

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

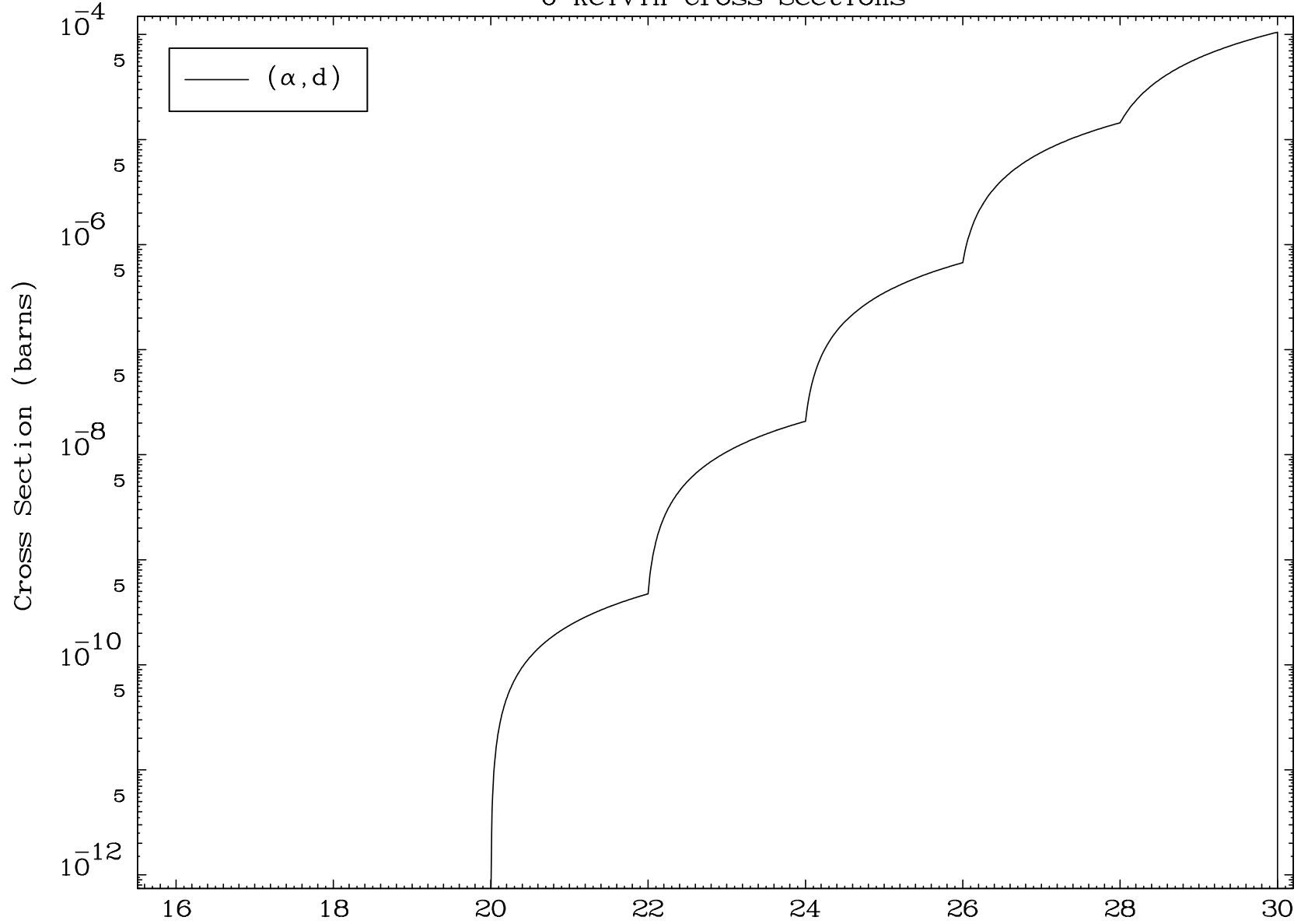
87-Fr-207

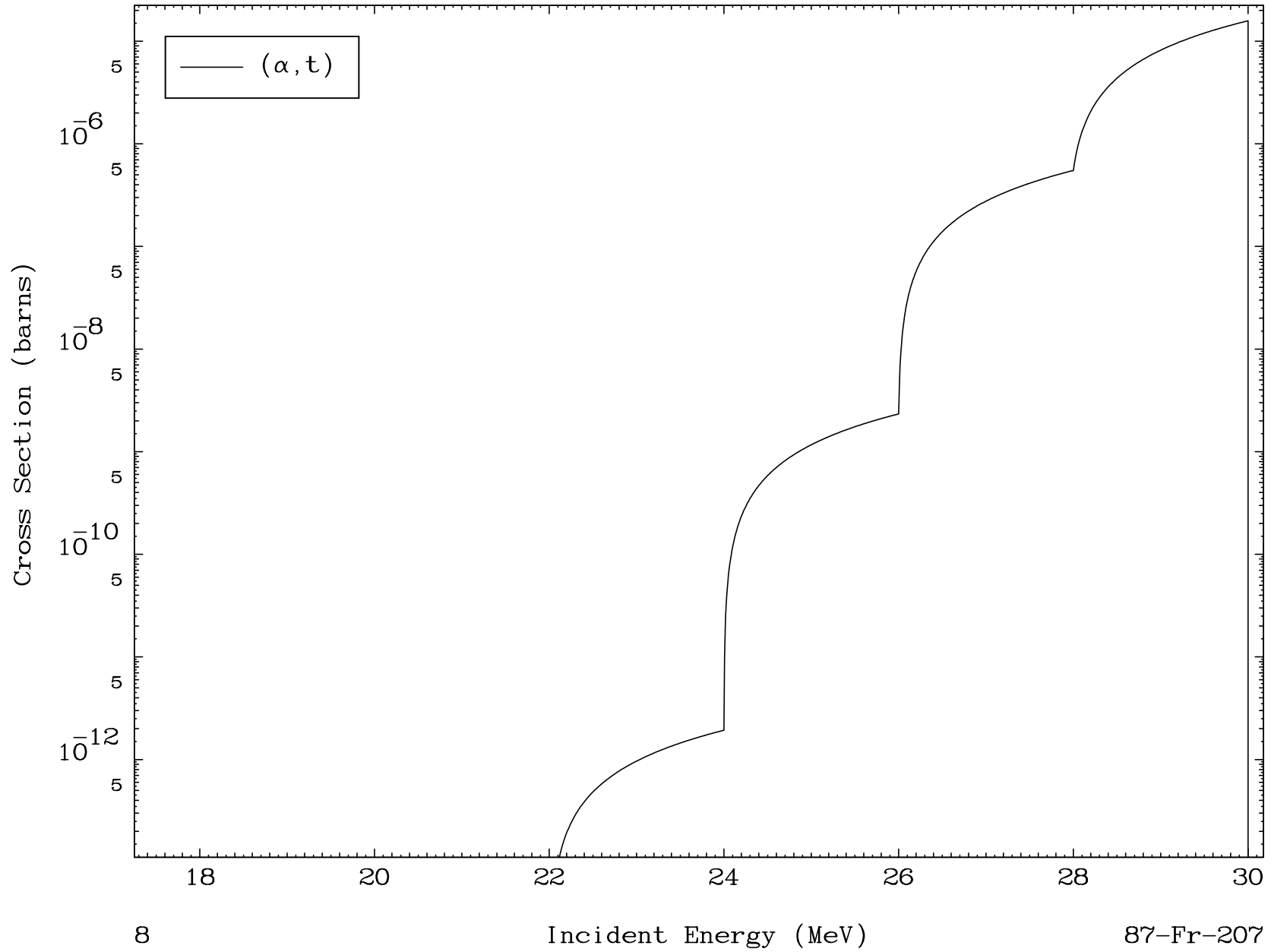


6

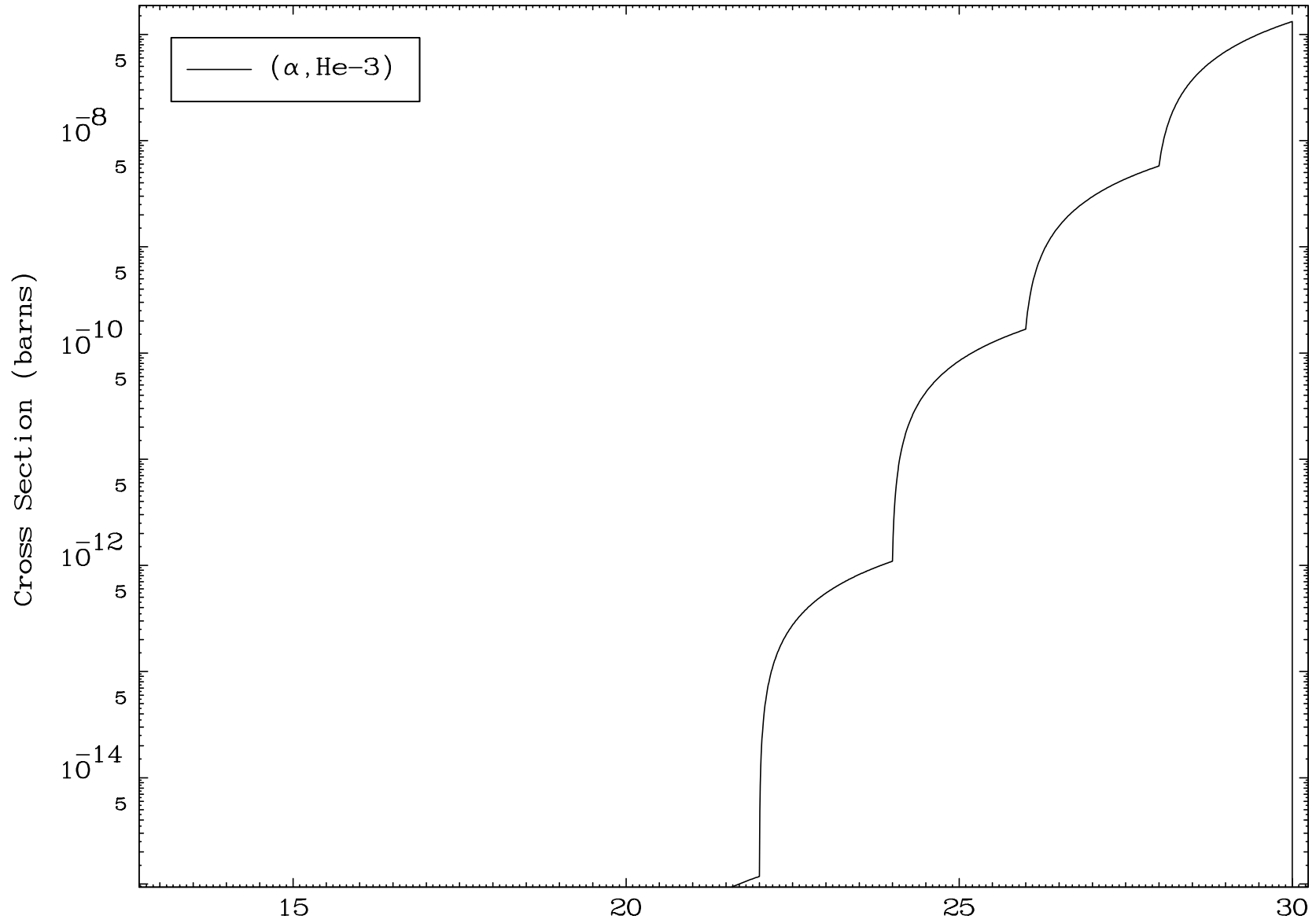
Incident Energy (MeV)

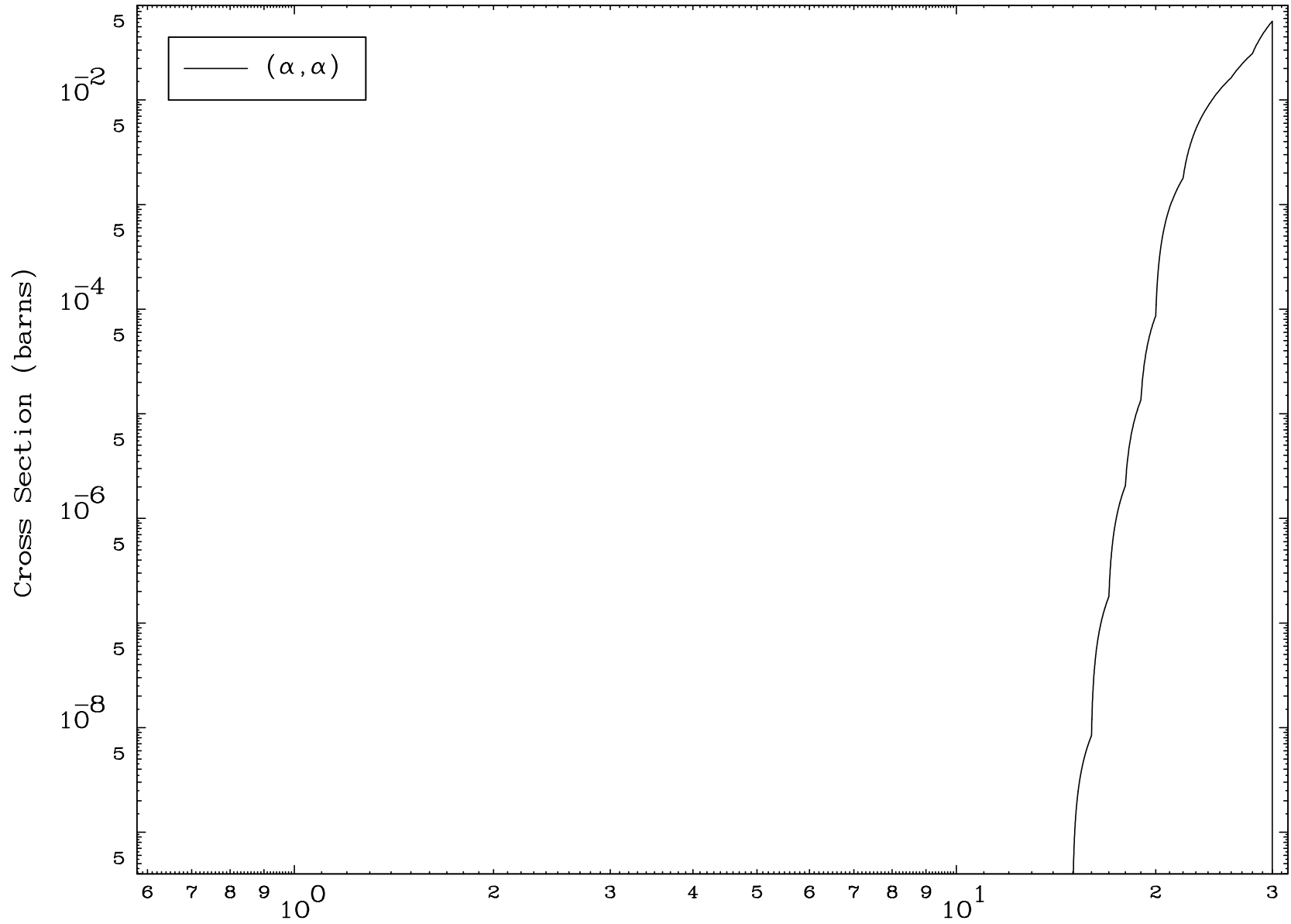
87-Fr-207



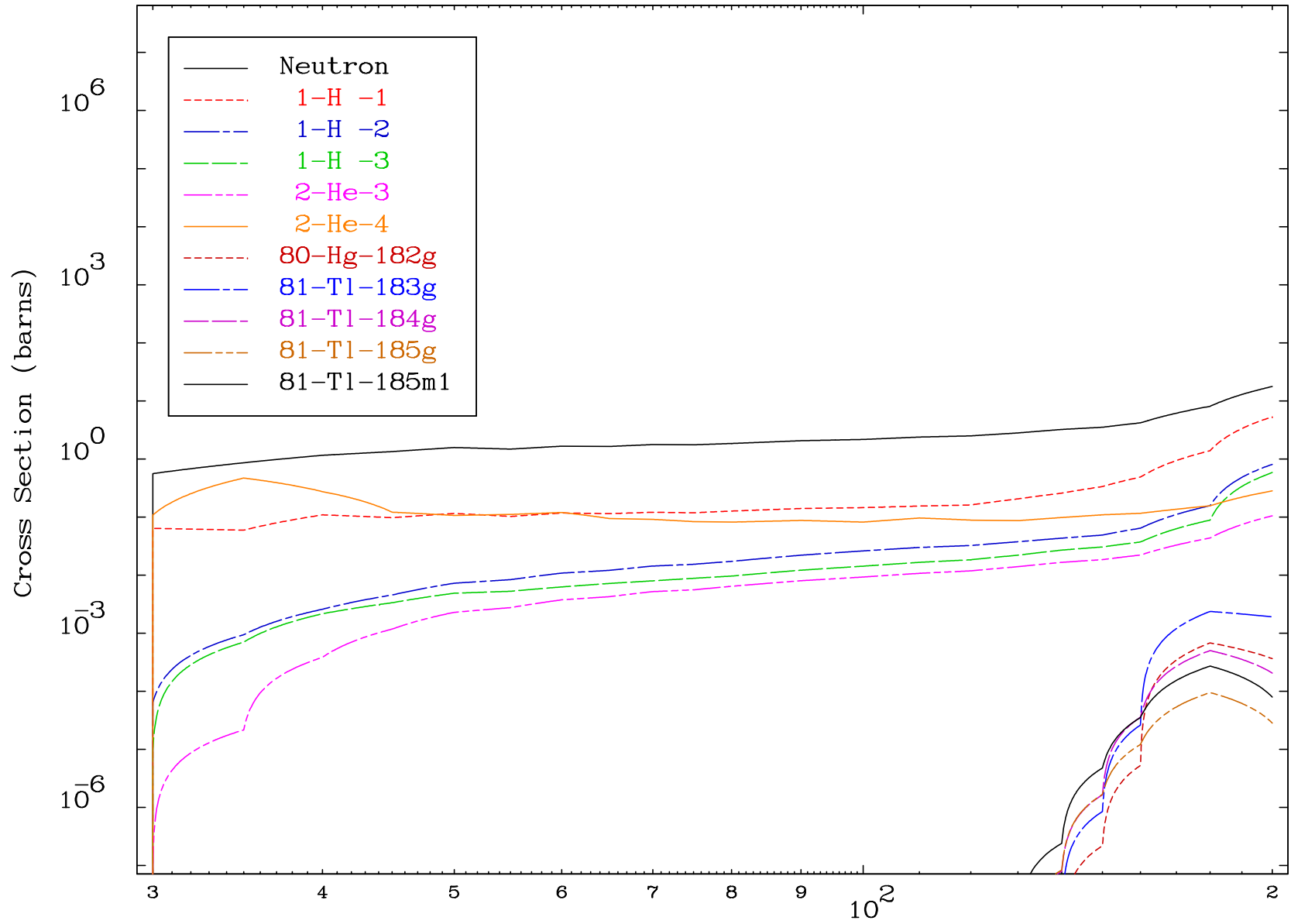




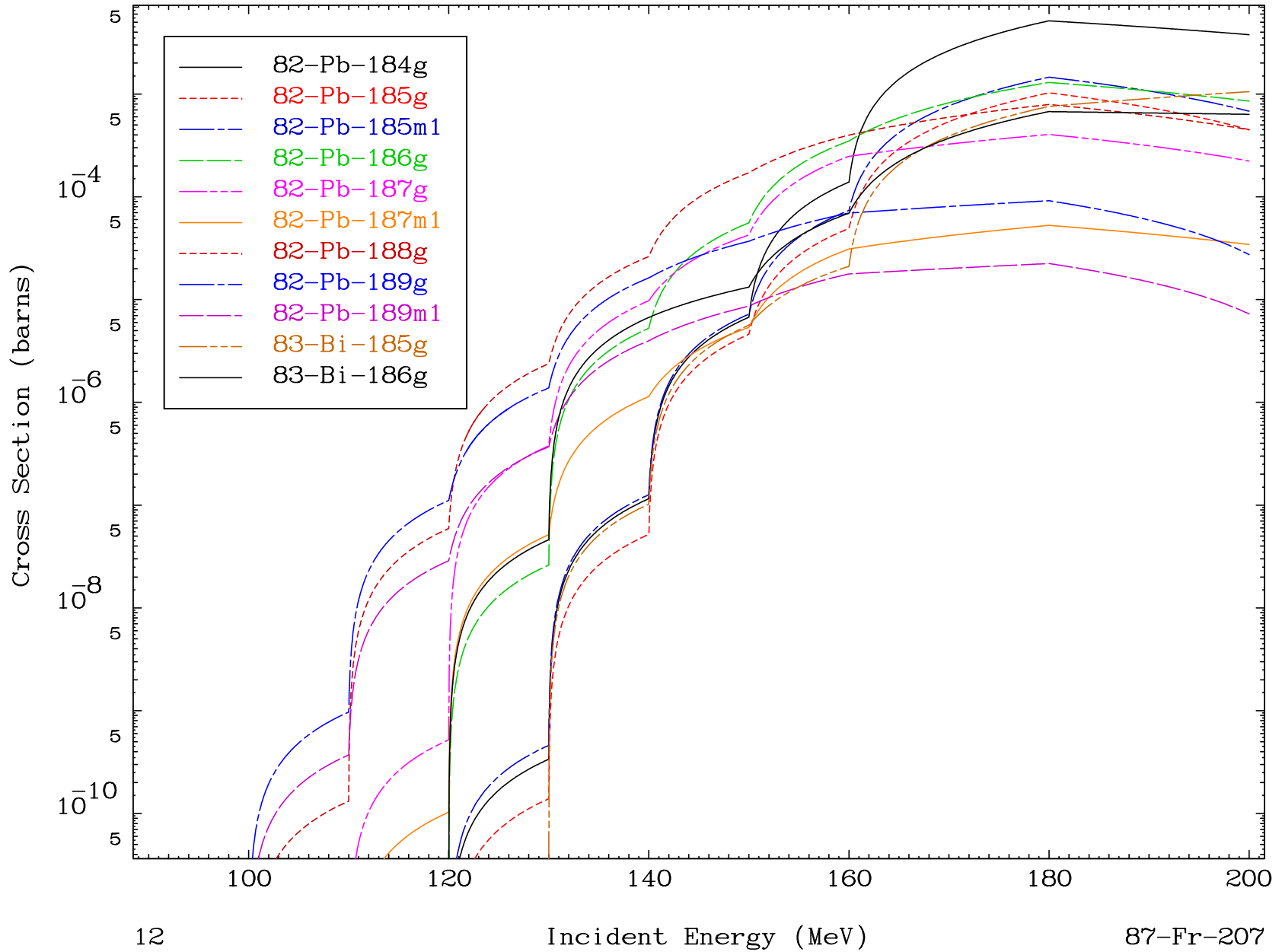




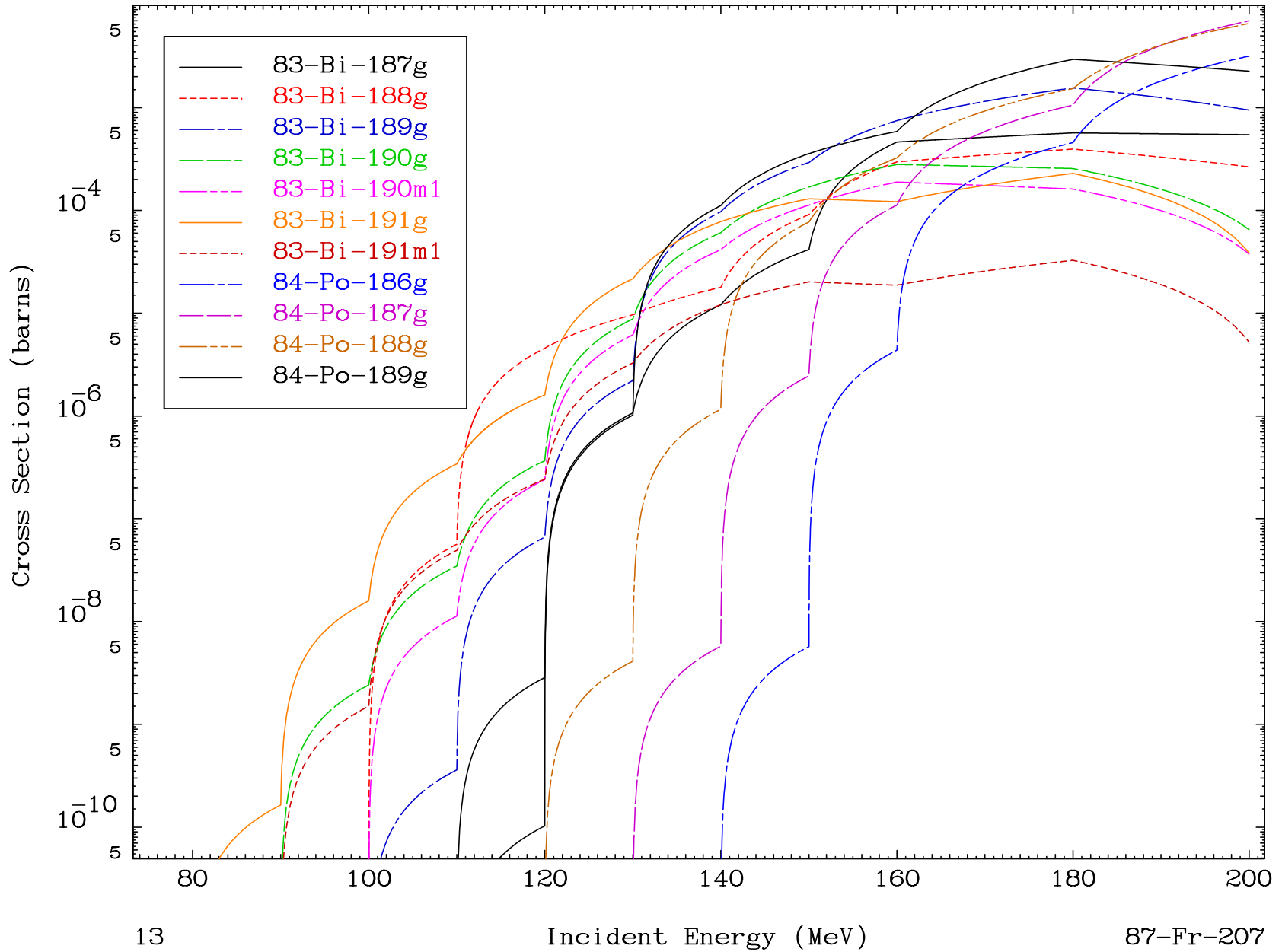
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

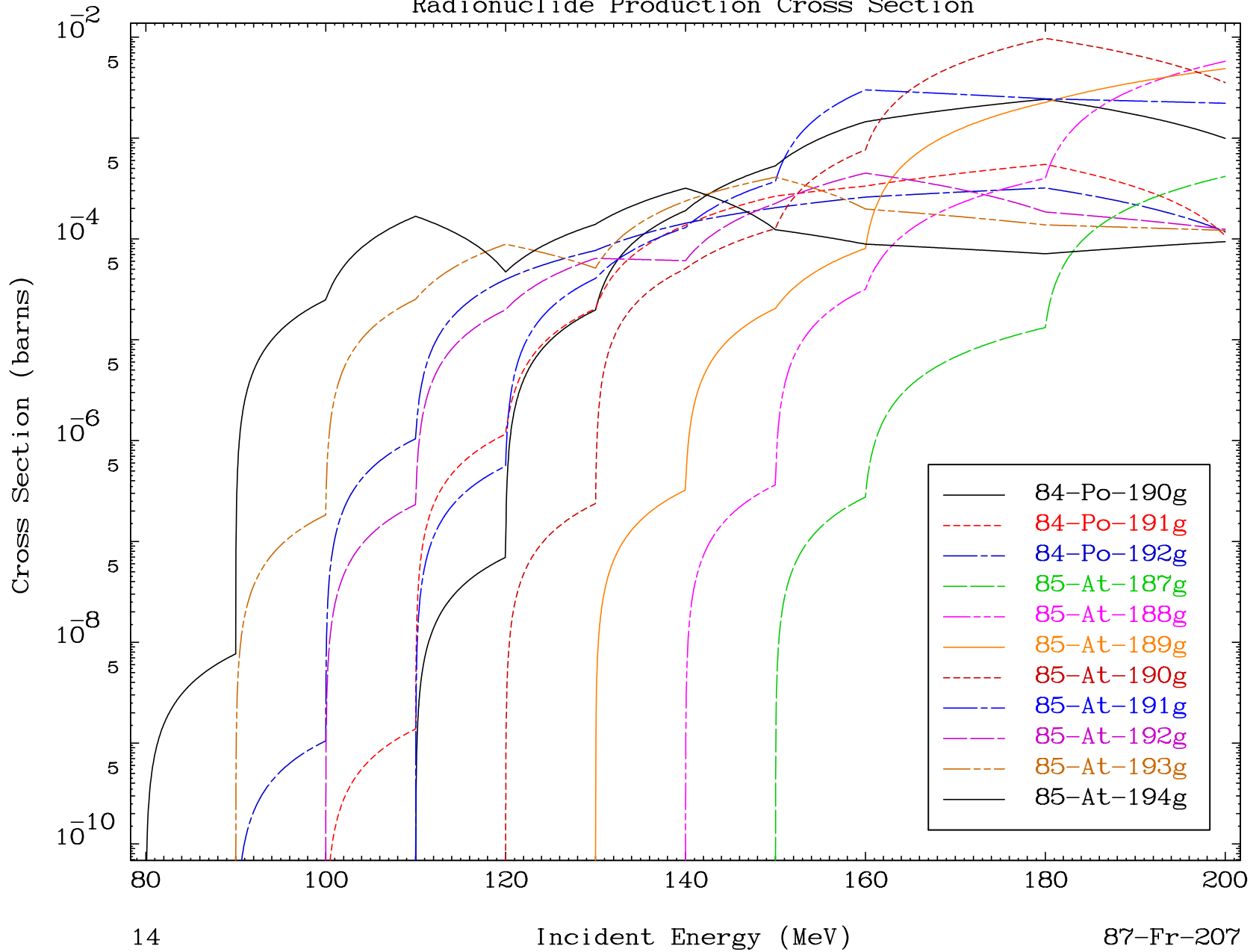


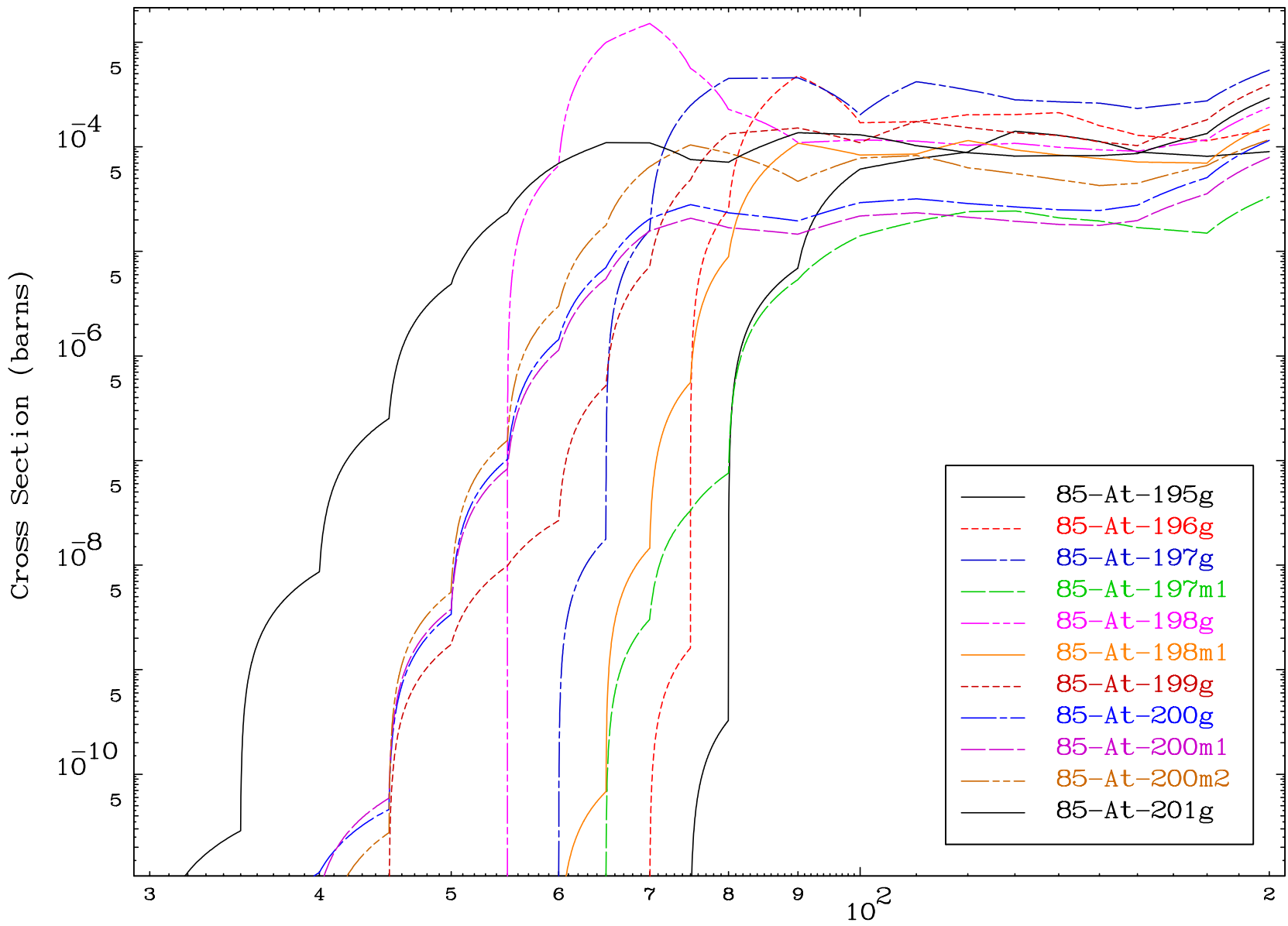
MAT 8710

( $\alpha$ , remainder)

87-Fr-207

### Radionuclide Production Cross Section



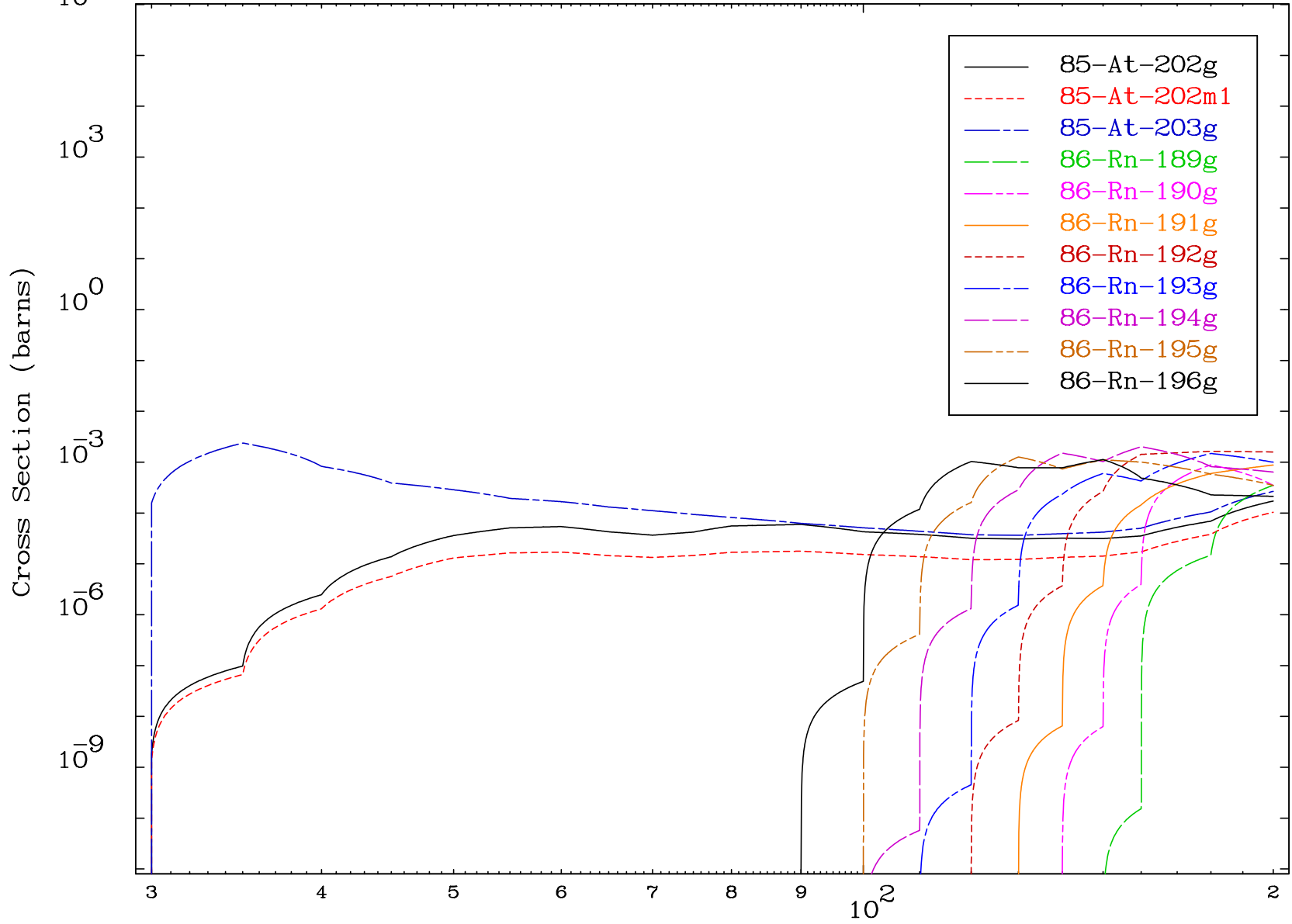


MAT 8710

( $\alpha$ , remainder)

87-Fr-207

### Radionuclide Production Cross Section

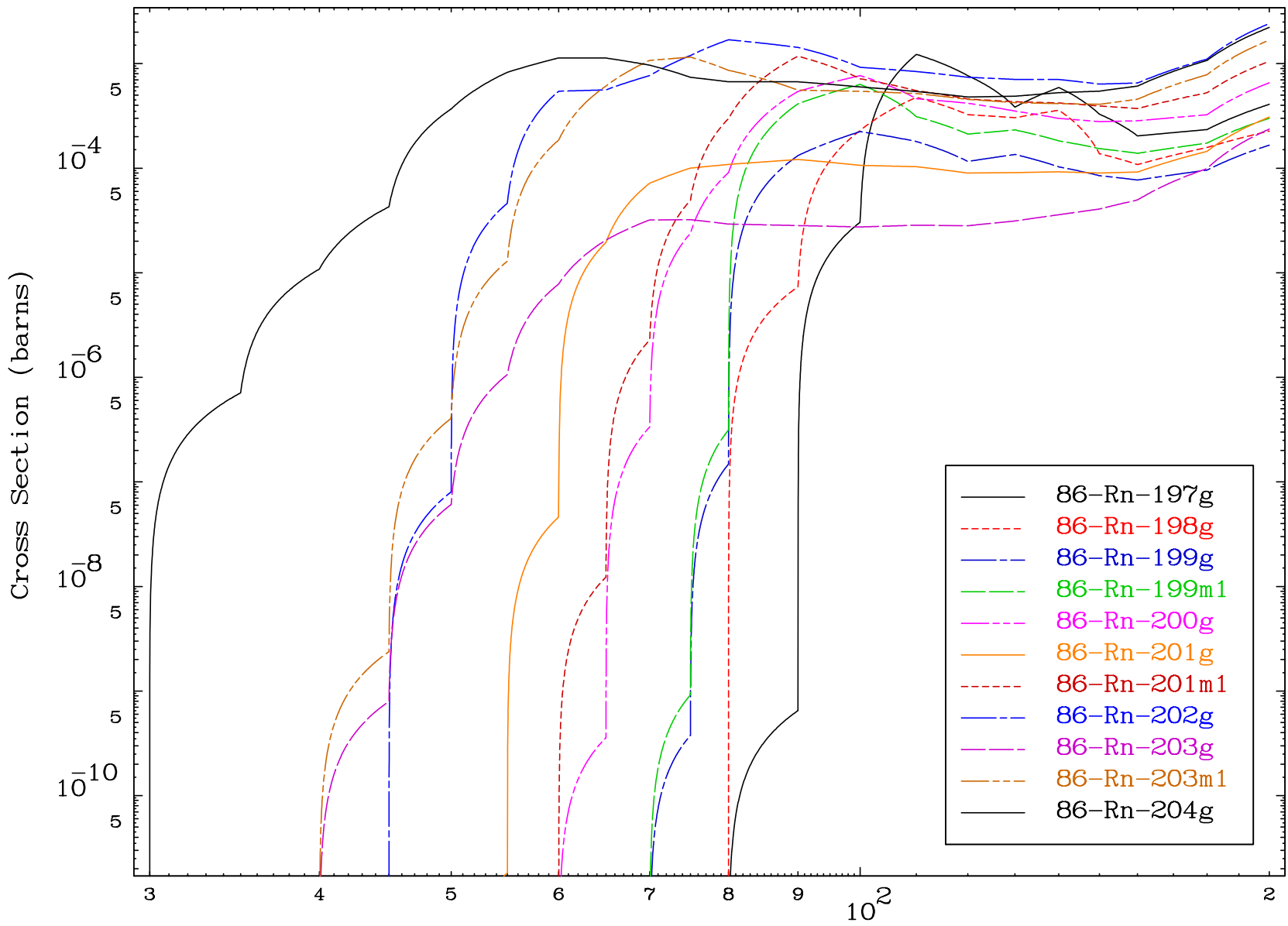


16

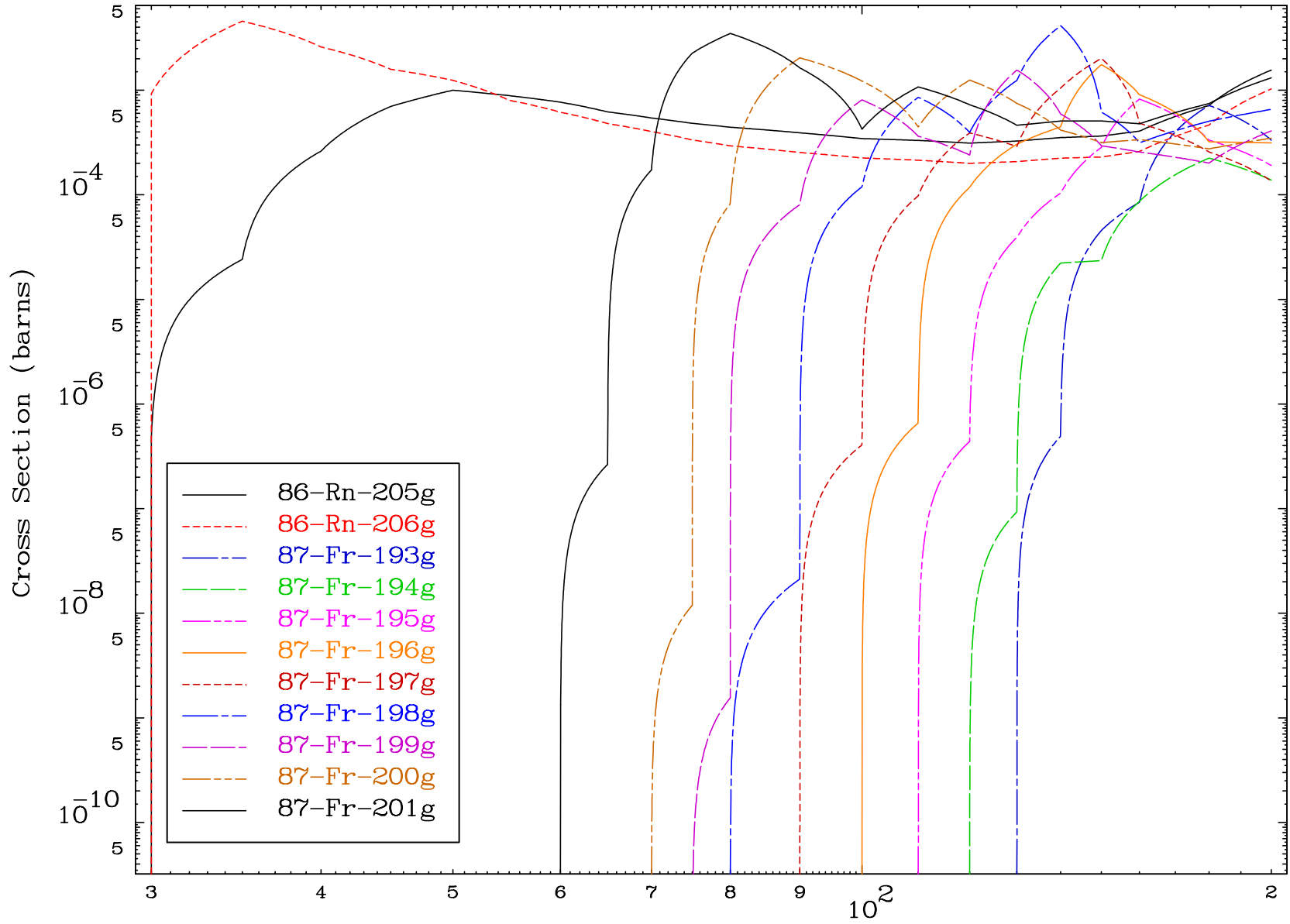
Incident Energy (MeV)

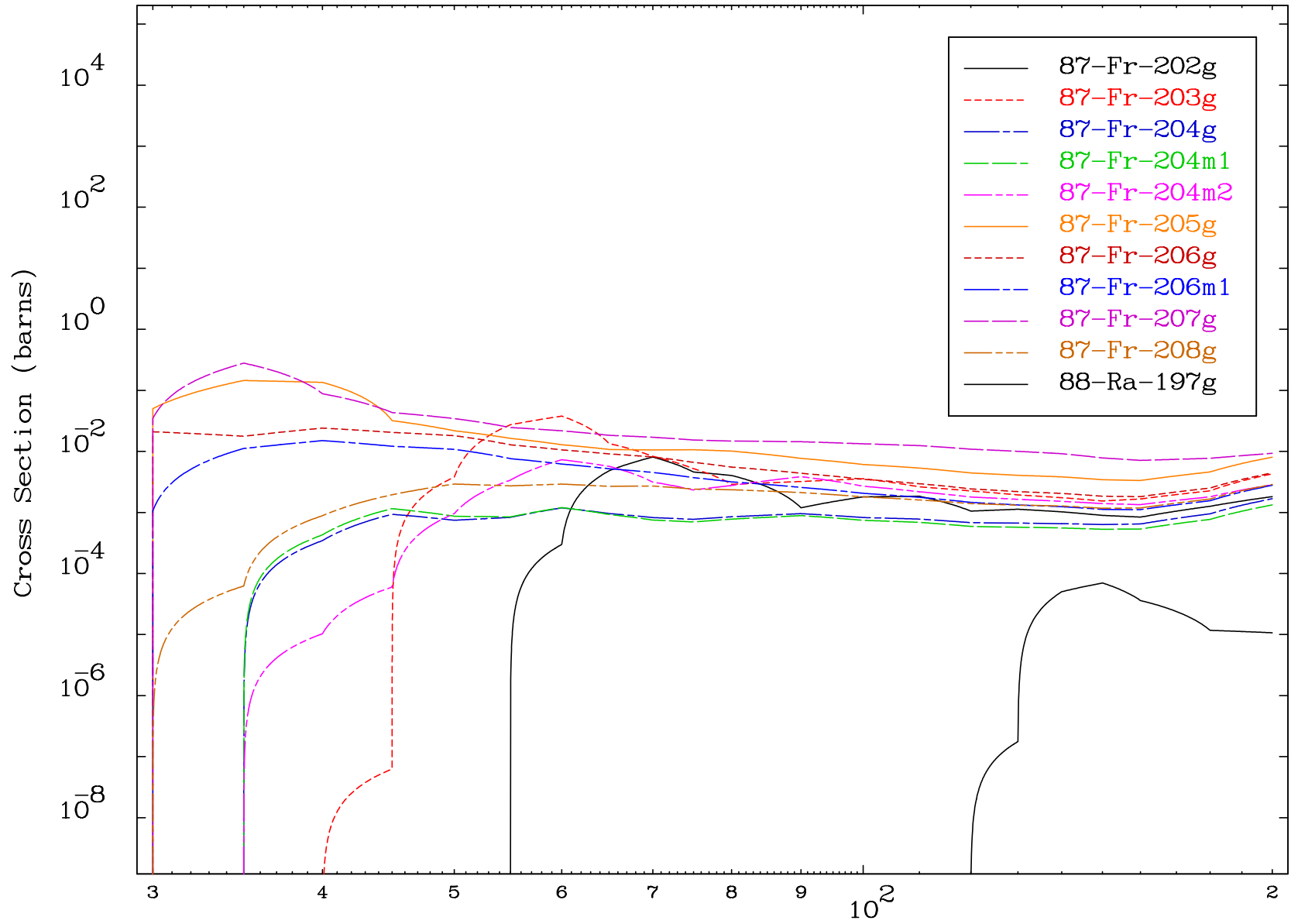
87-Fr-207

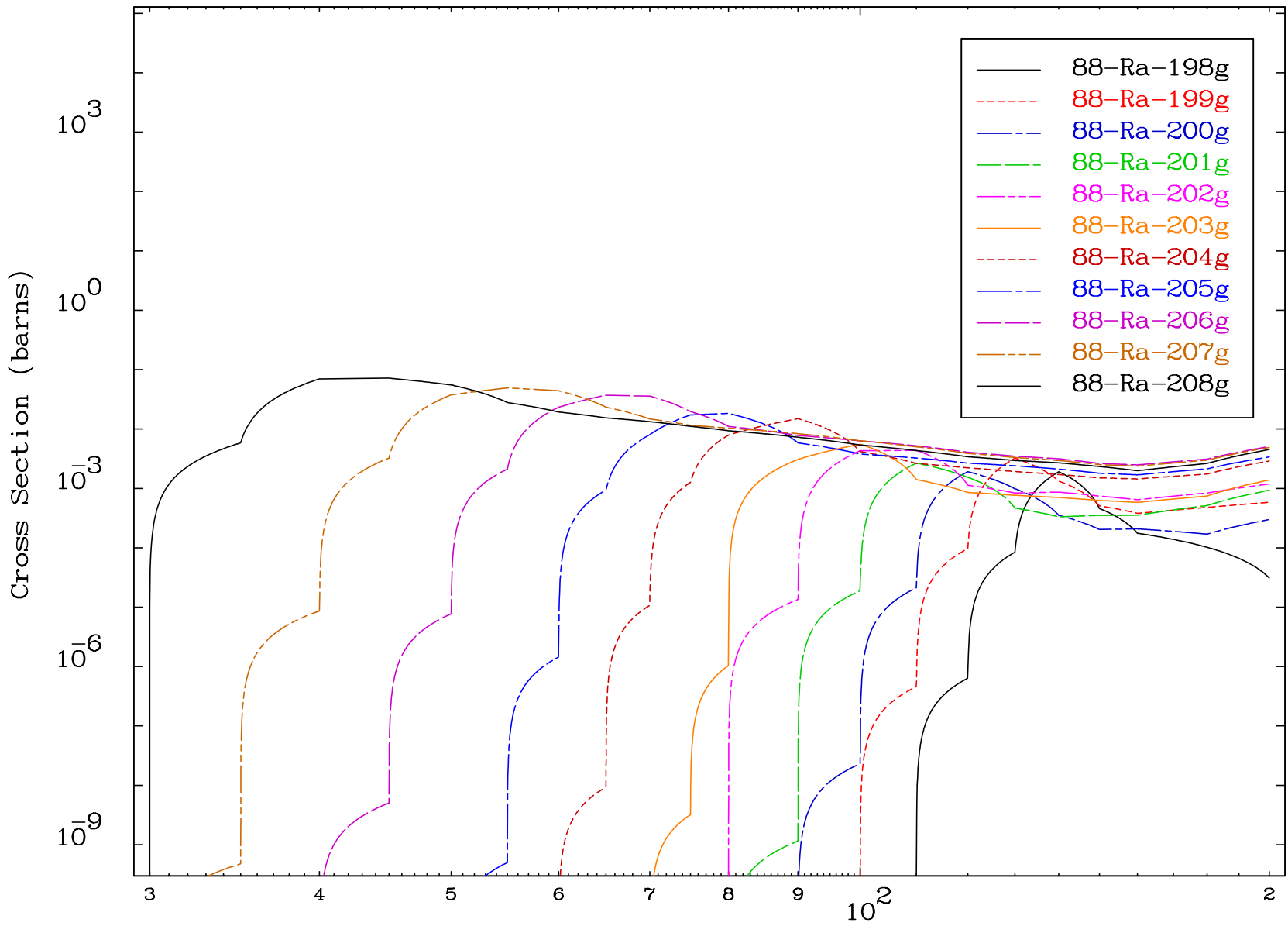


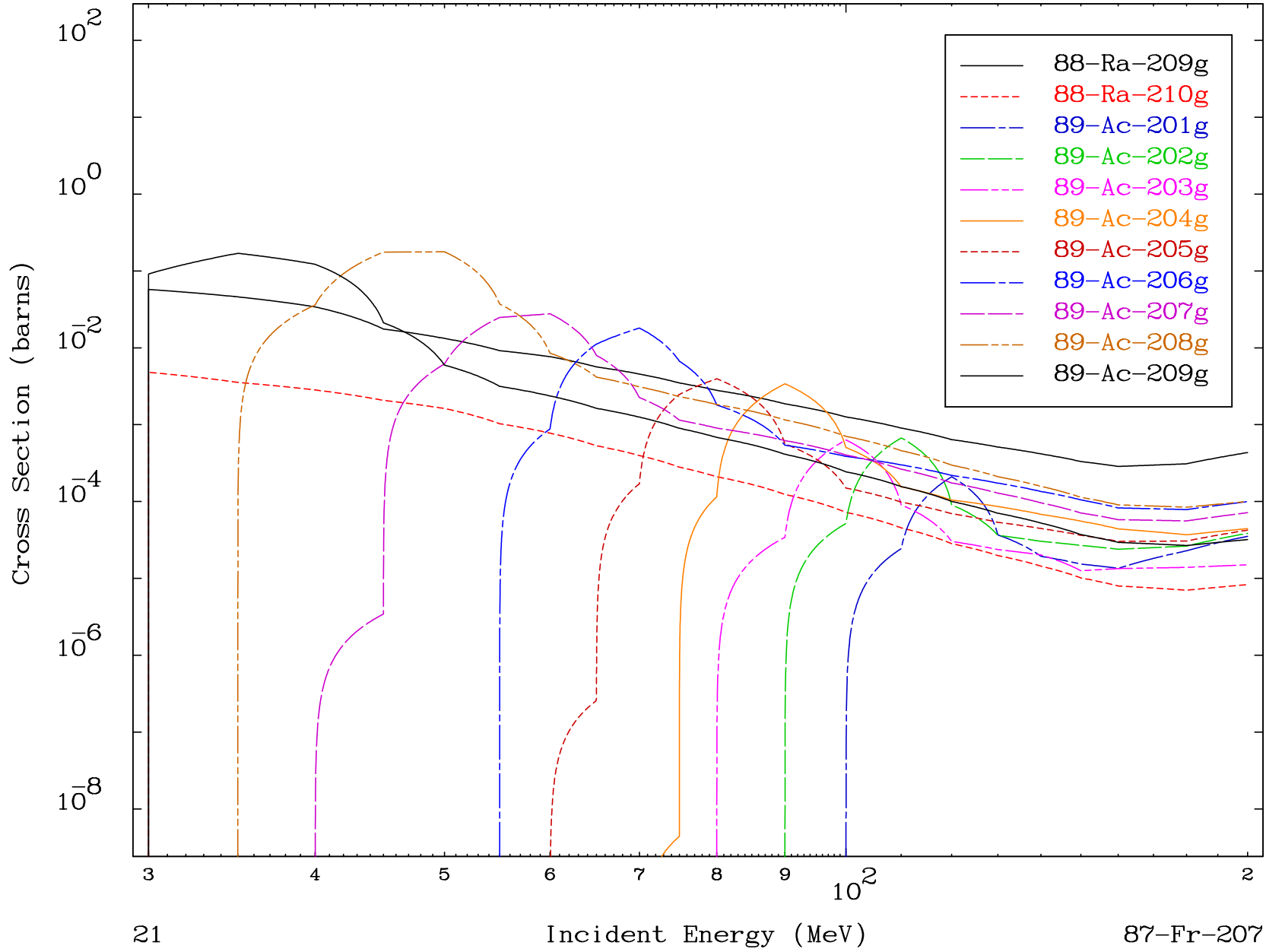


Radionuclide Production Cross Section







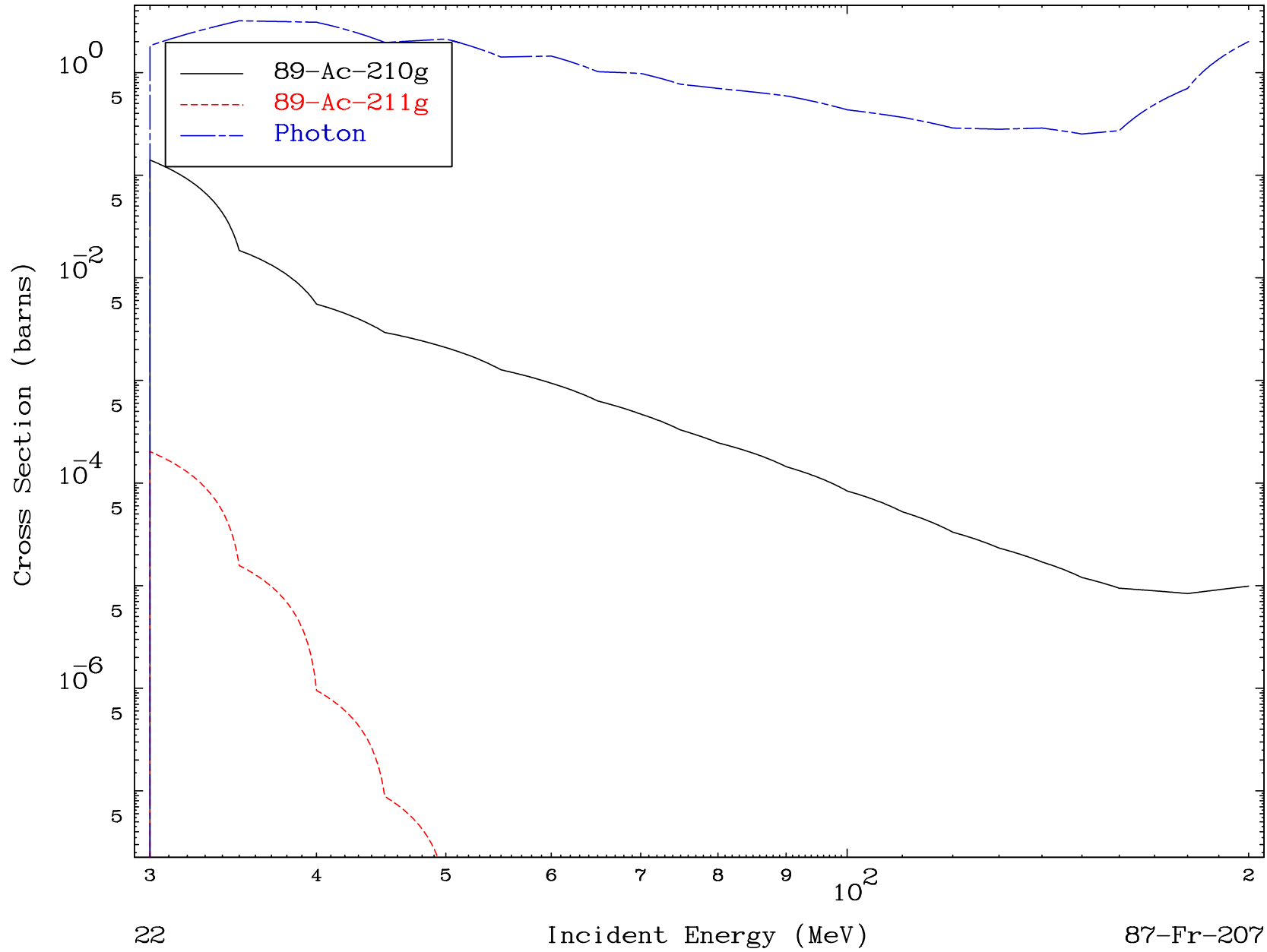


MAT 8710

( $\alpha$ , remainder)

87-Fr-207

### Radionuclide Production Cross Section



22

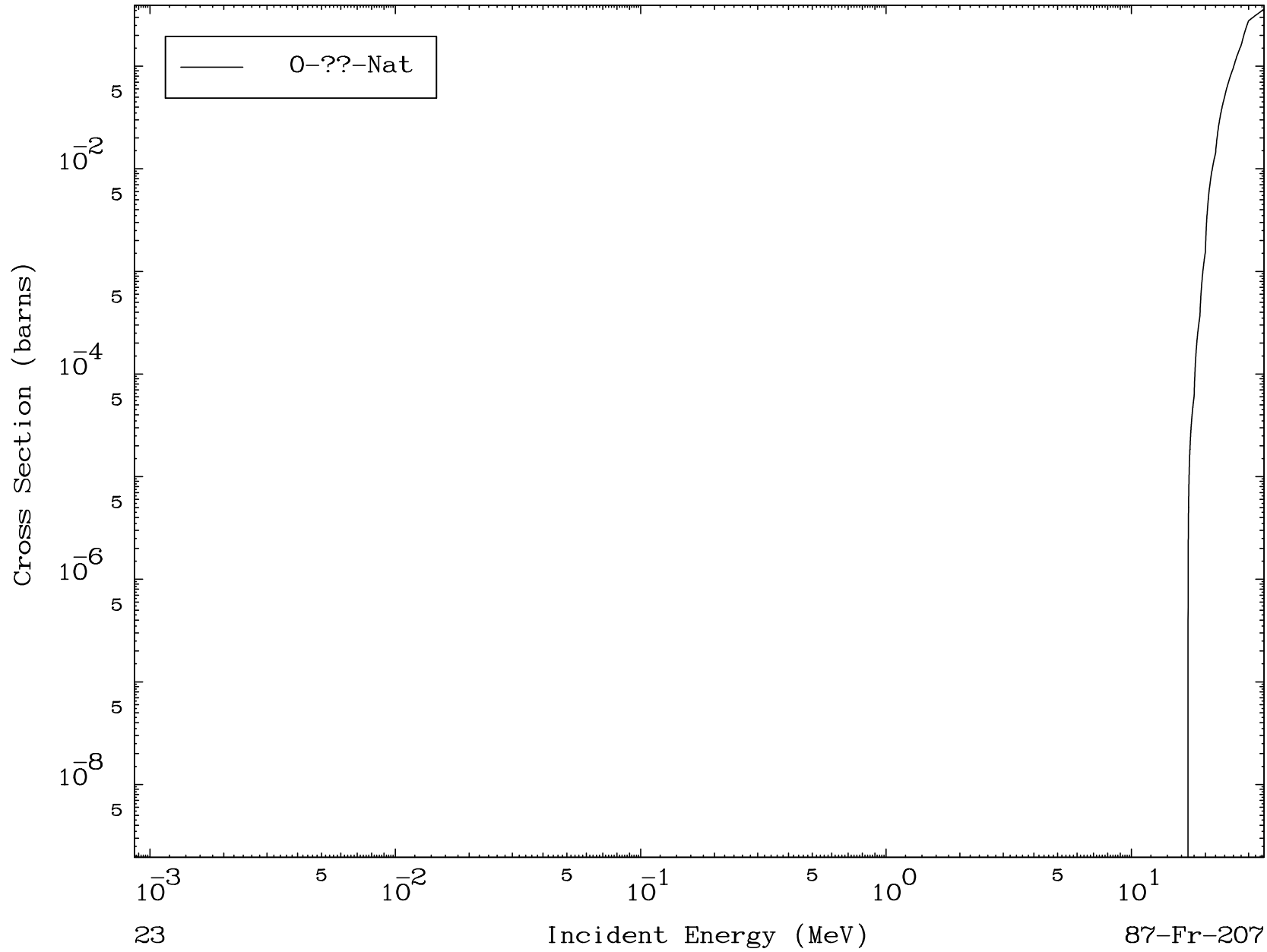
Incident Energy (MeV)

87-Fr-207

MAT 8710

$\alpha$  Fission  
Radionuclide Production Cross Section

87-Fr-207



23

Incident Energy (MeV)

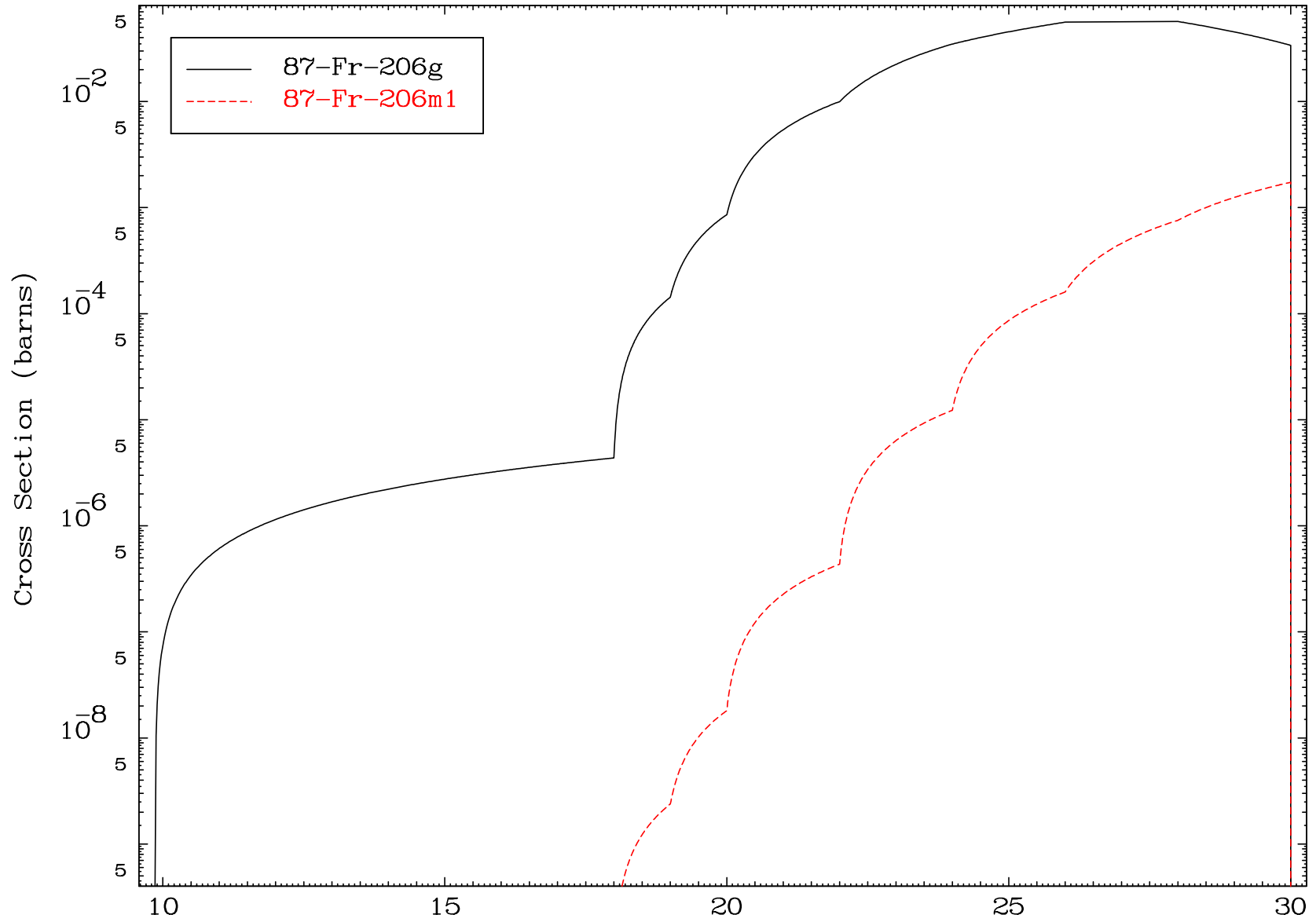
87-Fr-207

MAT 8710

( $\alpha, n'$ )  $\alpha$

87-Fr-207

Radionuclide Production Cross Section



24

Incident Energy (MeV)

87-Fr-207



Radionuclide Production Cross Section

