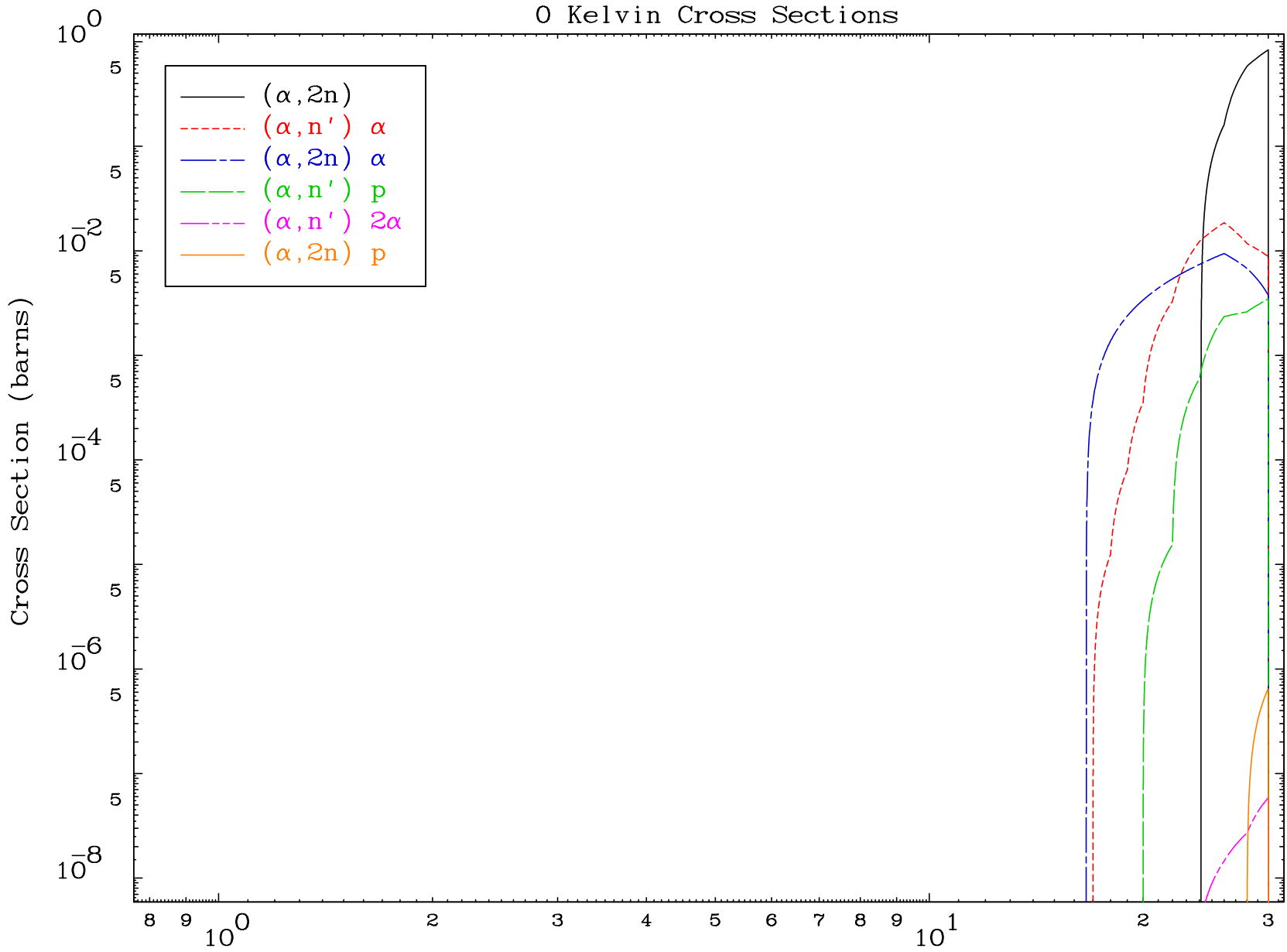


MAT 8725

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

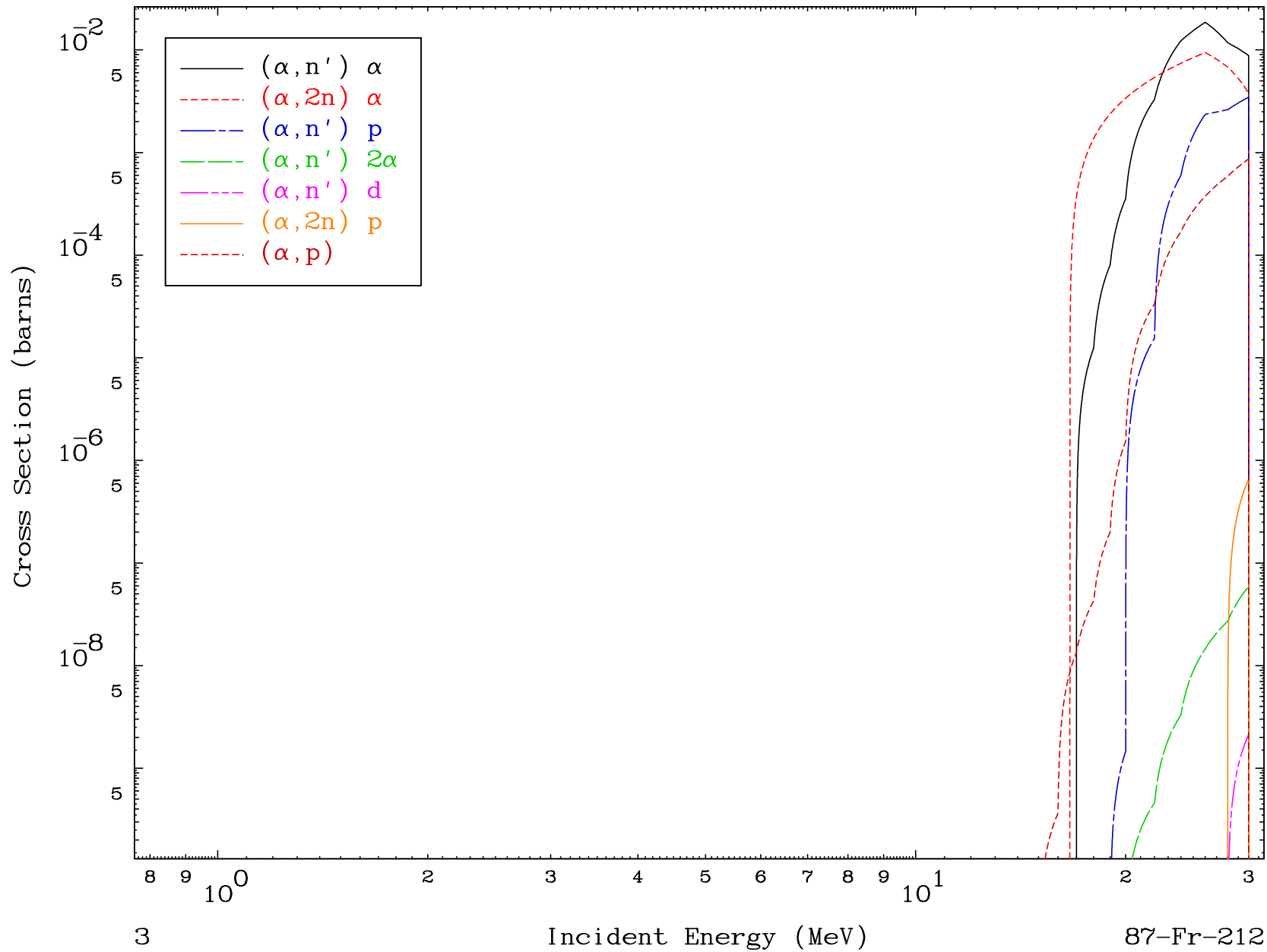
87-Fr-212

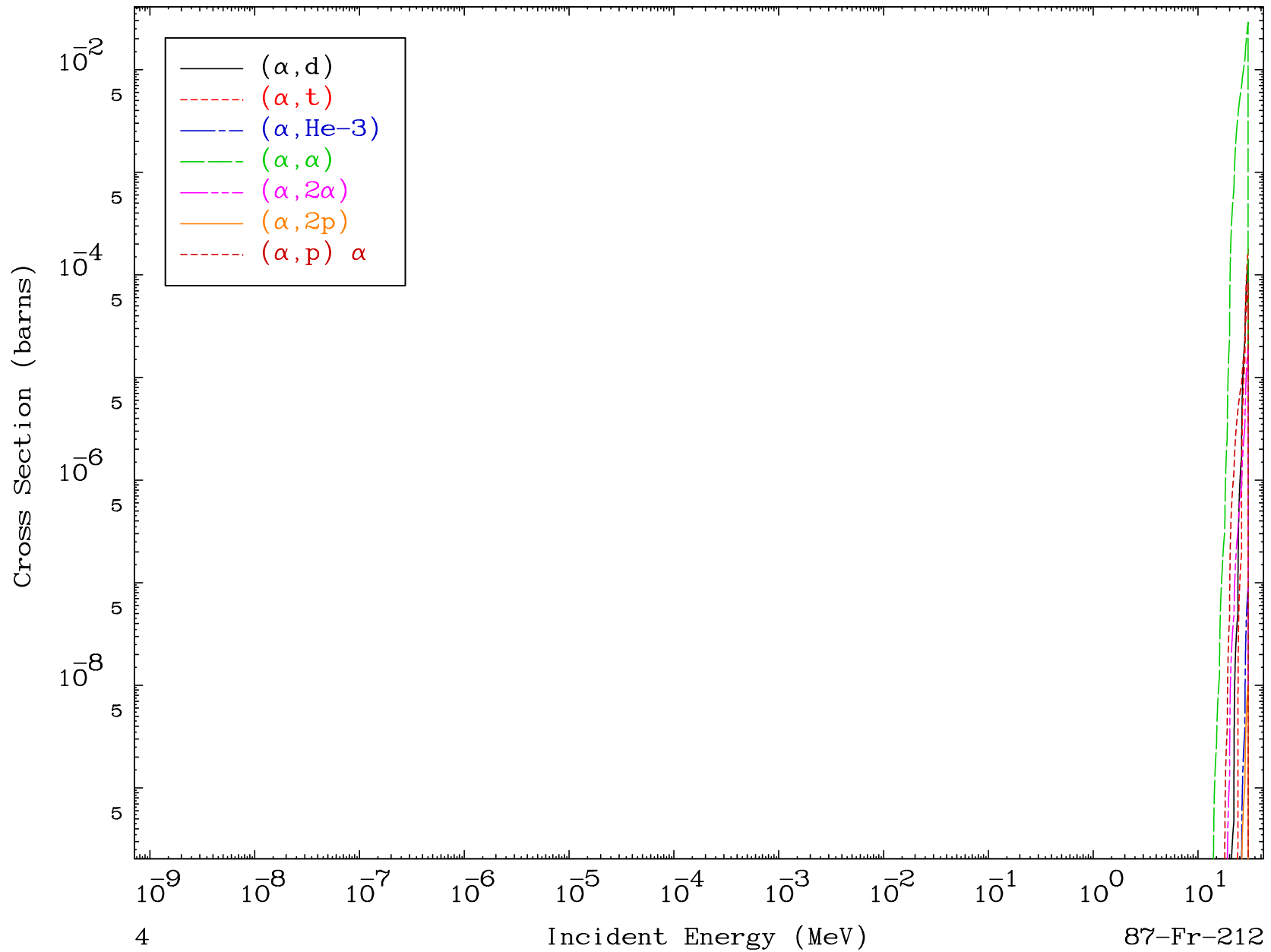


2

Incident Energy (MeV)

87-Fr-212

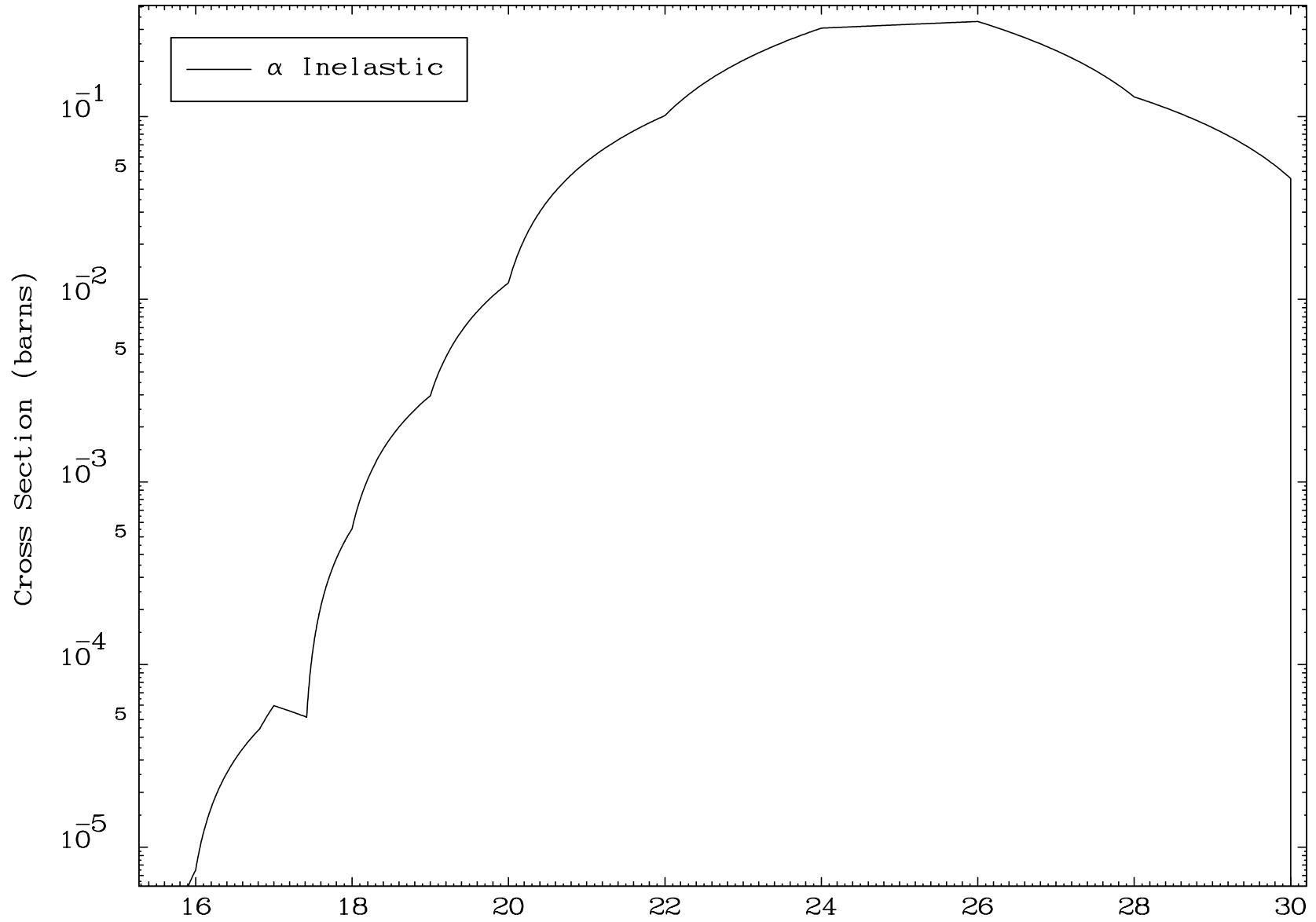




MAT 8725

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

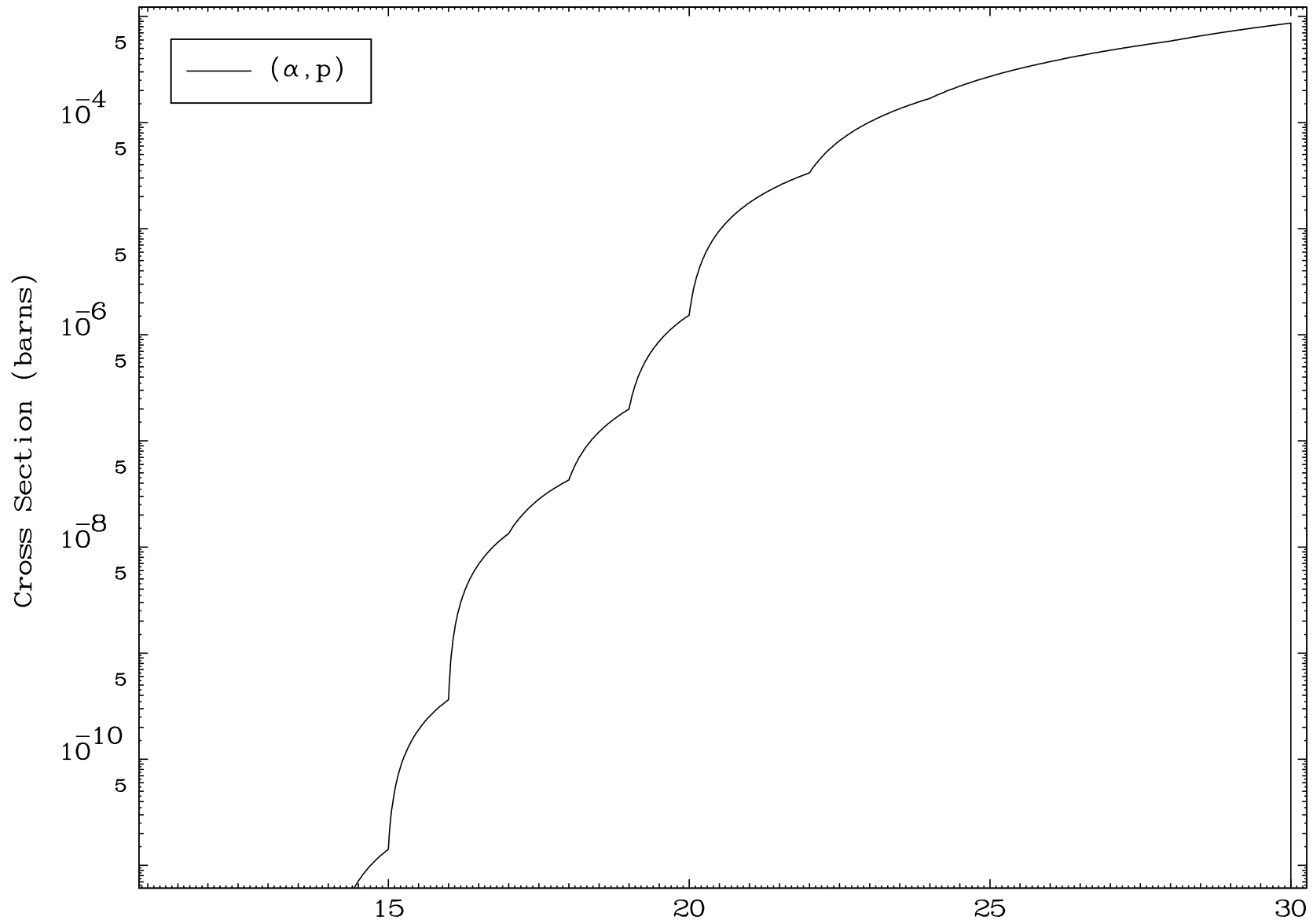
87-Fr-212

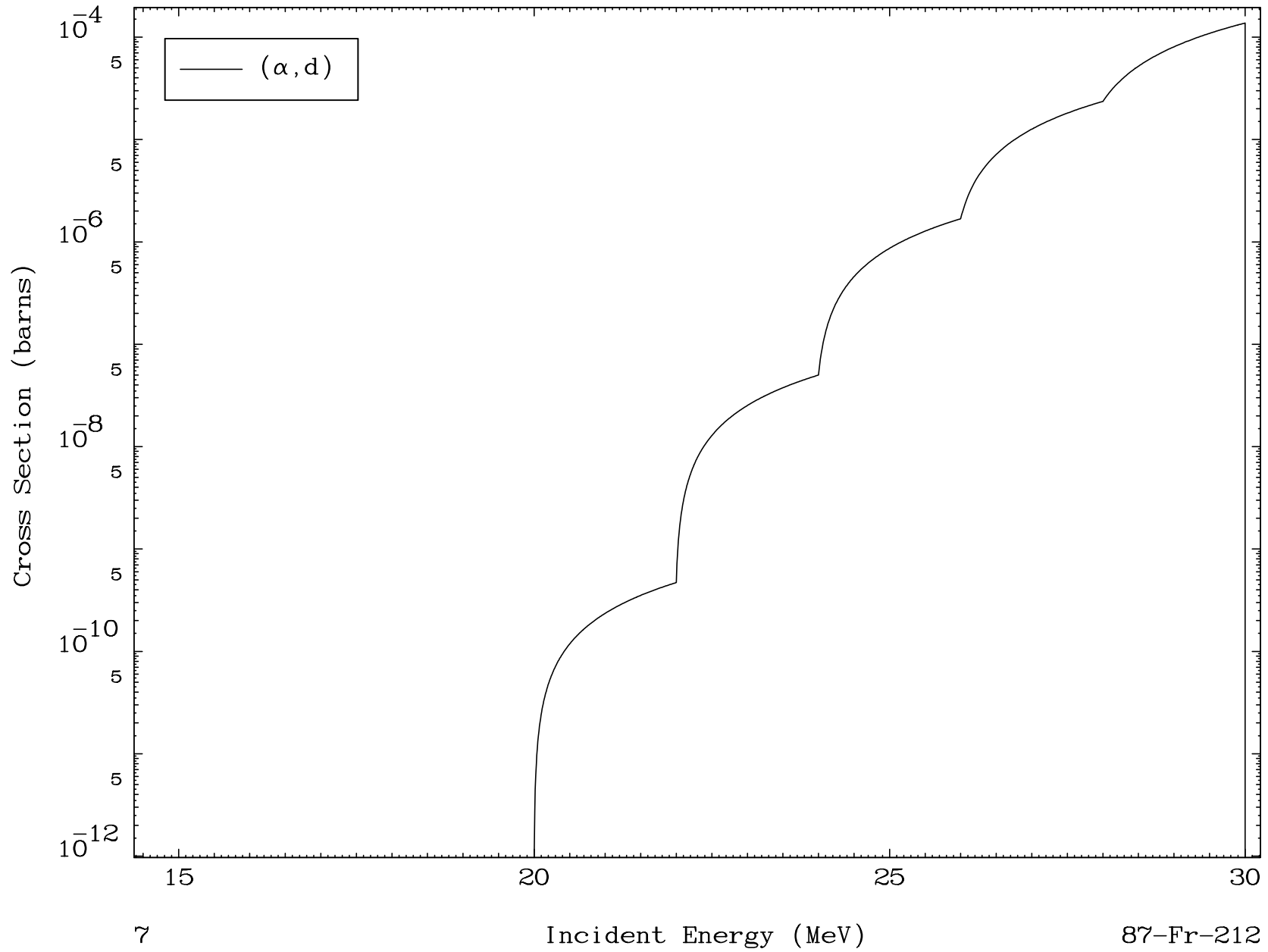


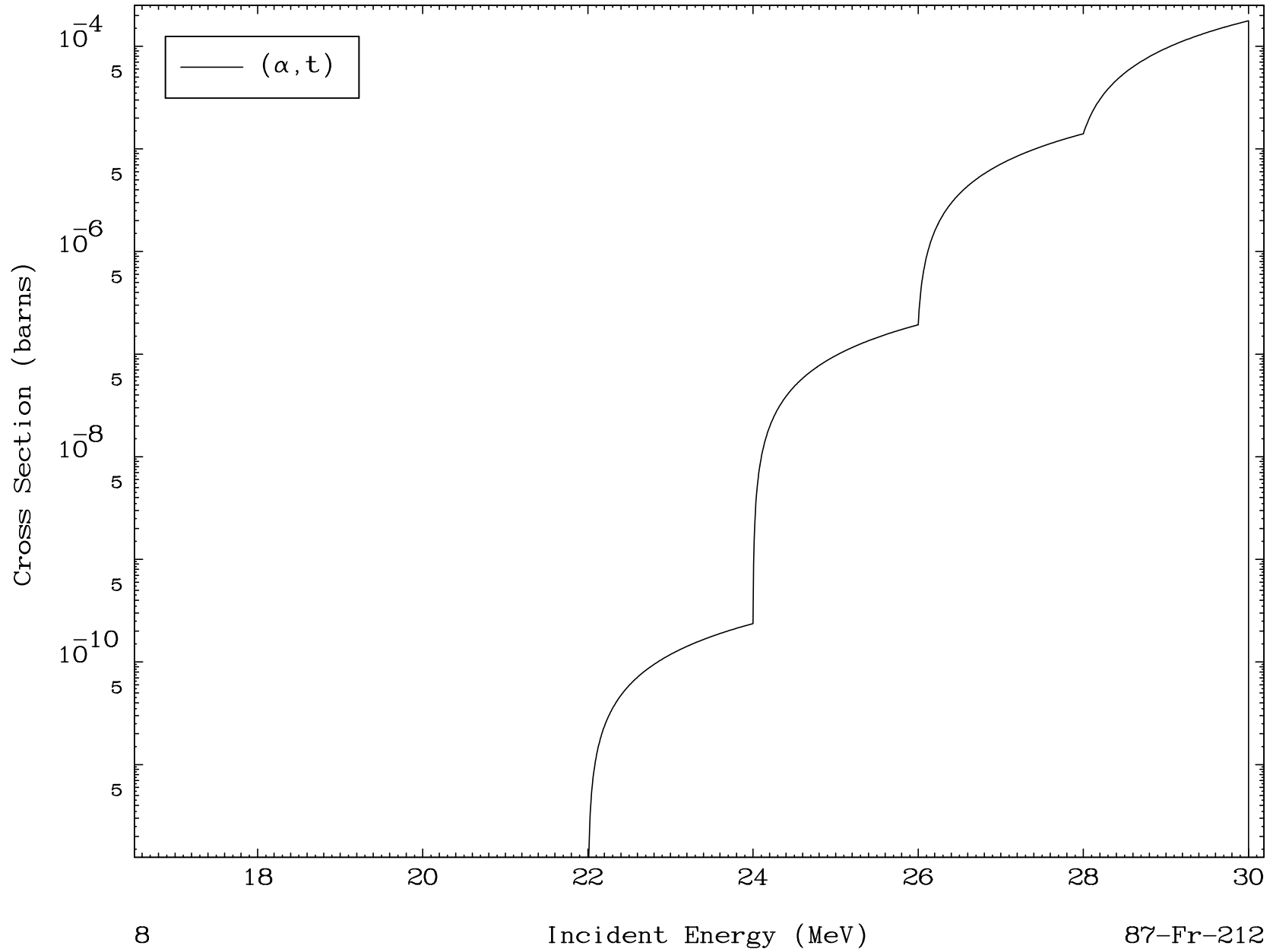
5

Incident Energy (MeV)

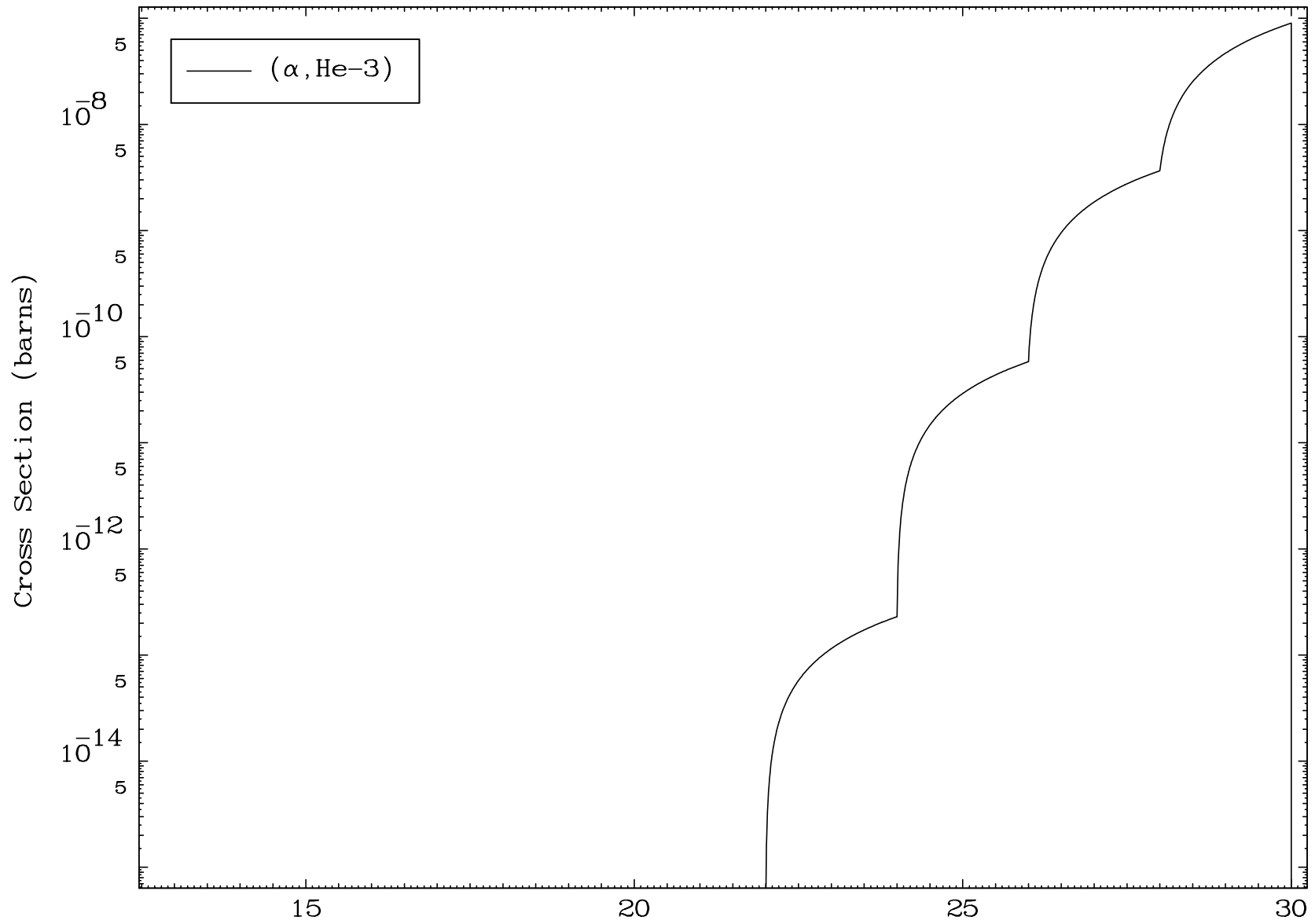
87-Fr-212







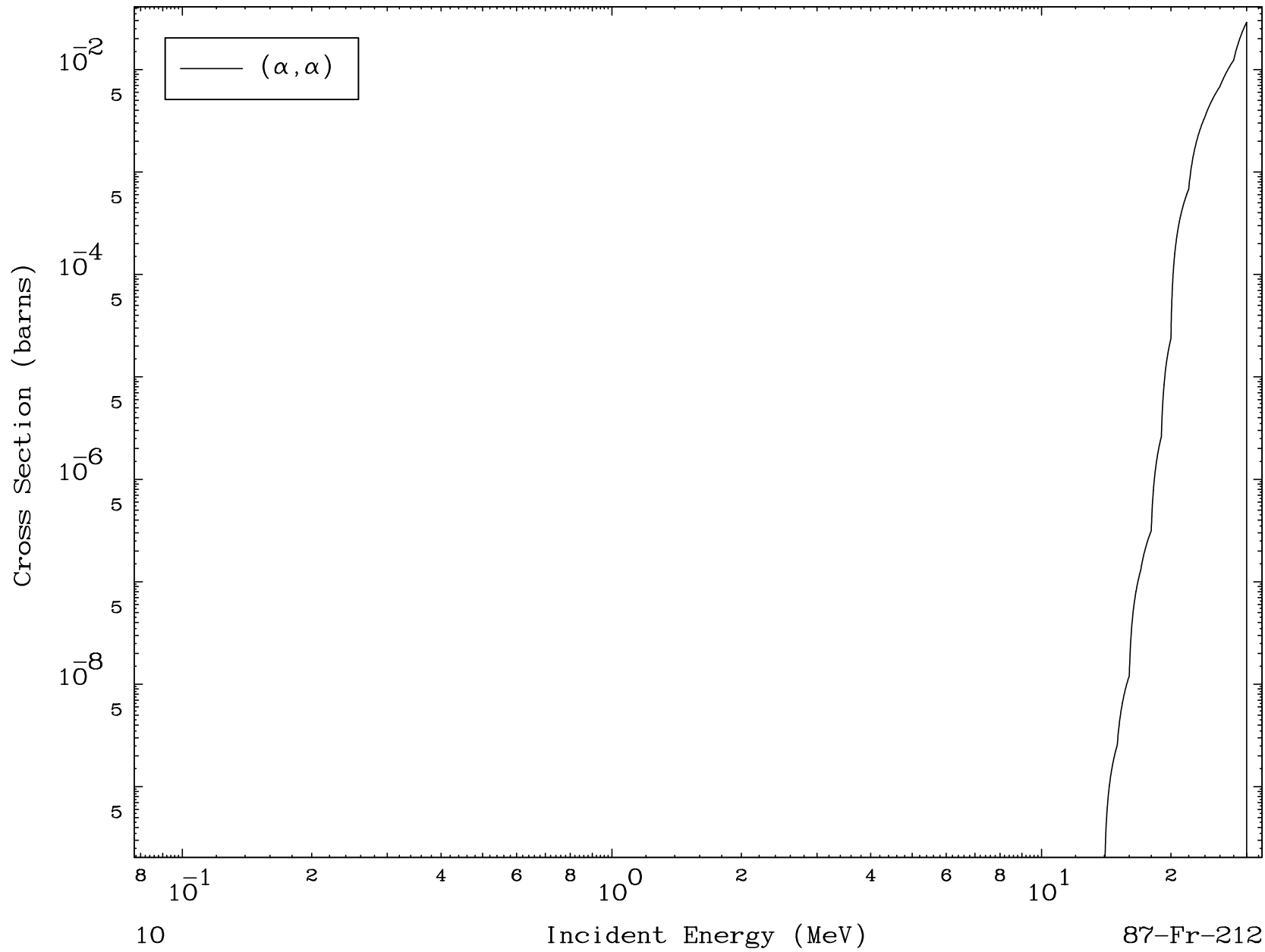




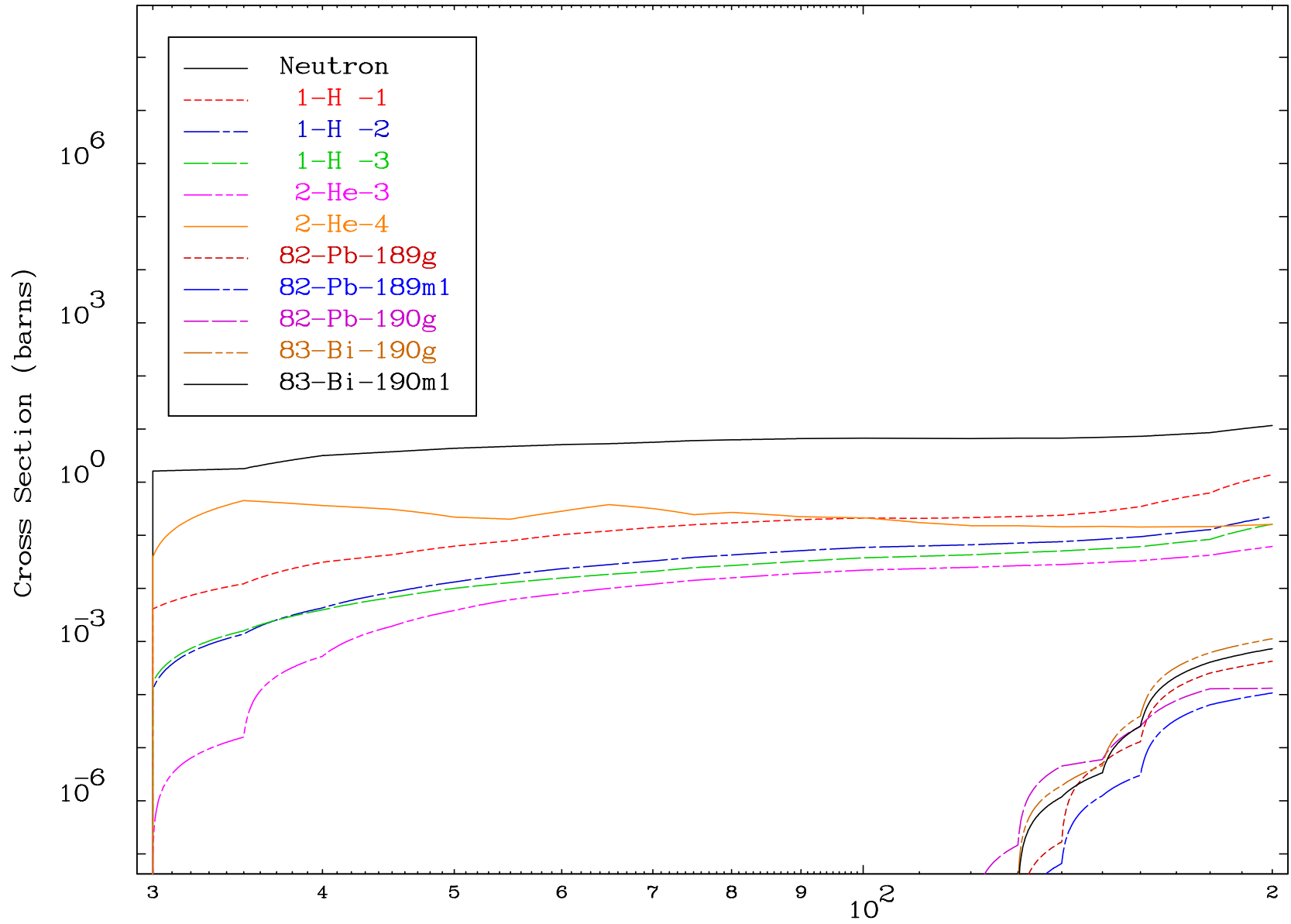
MAT 8725

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-212



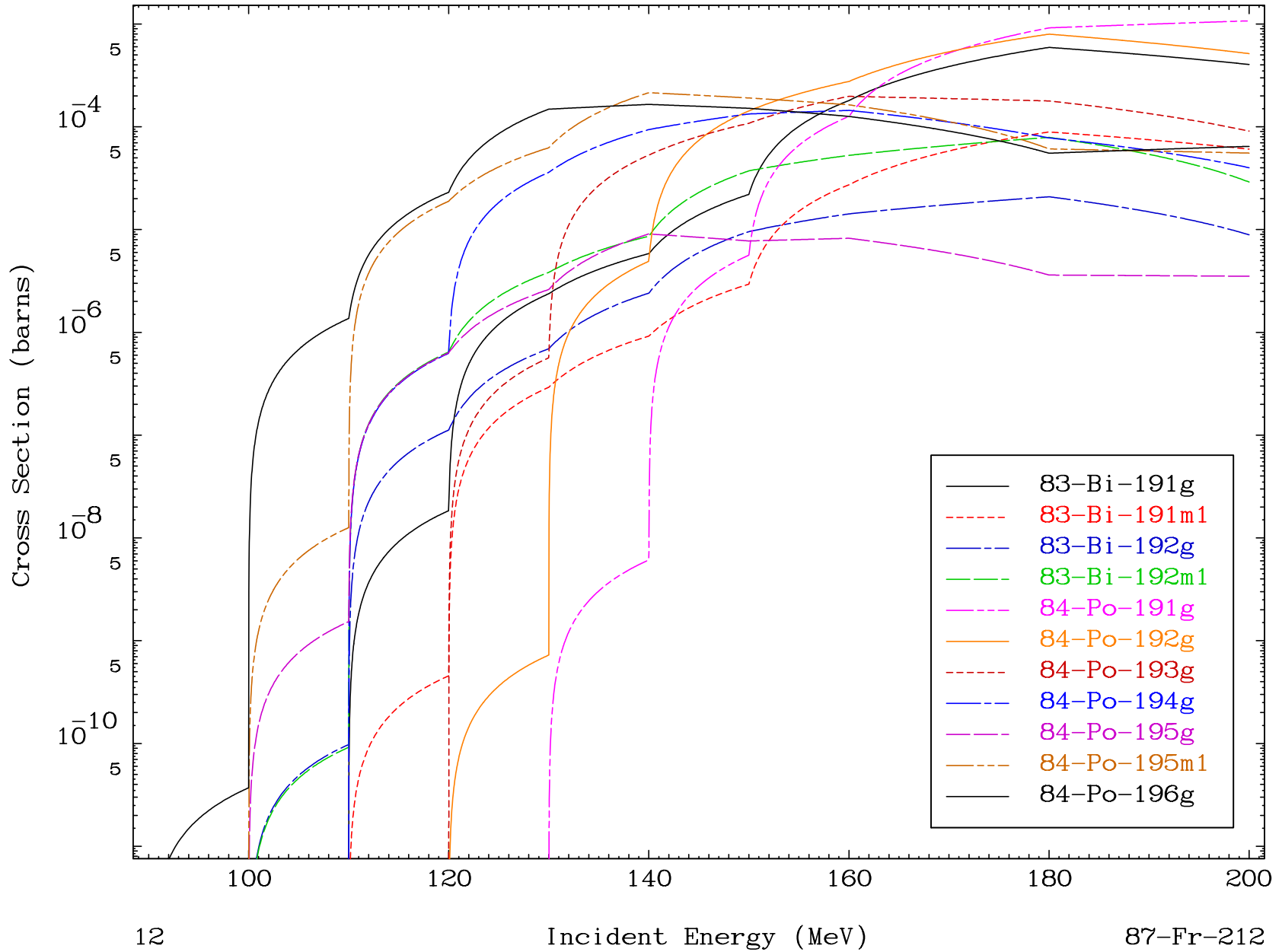
Radionuclide Production Cross Section



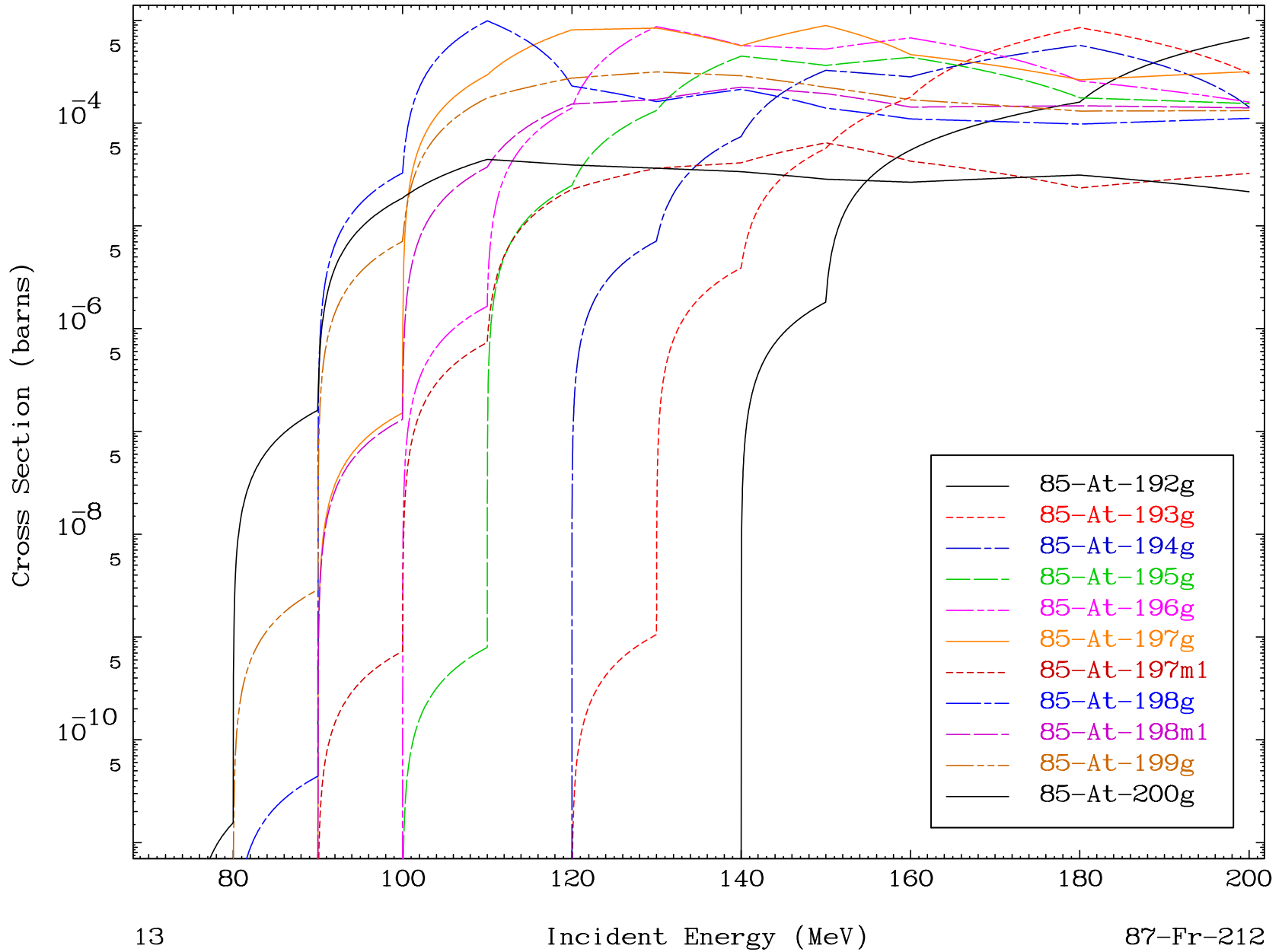
MAT 8725

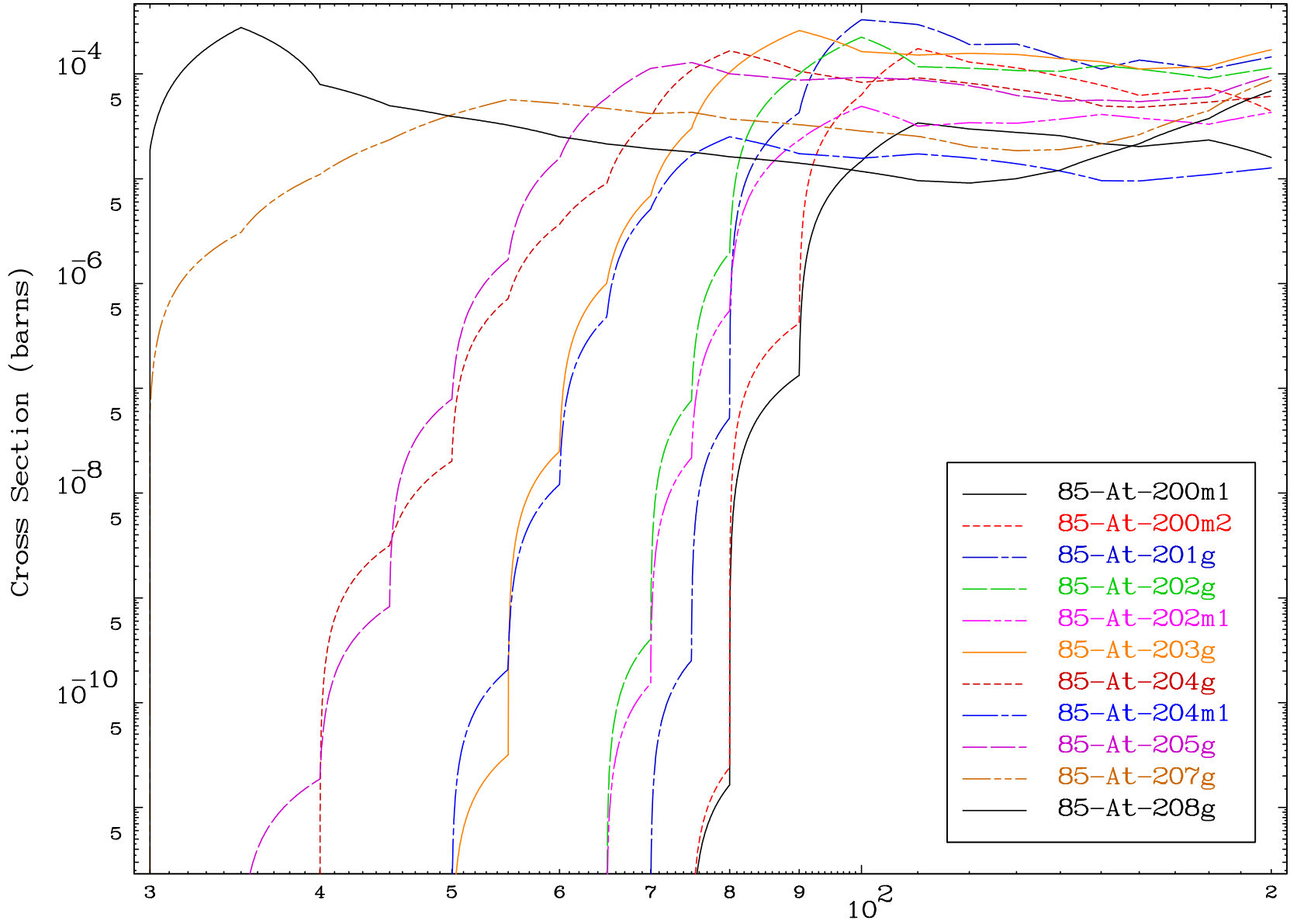
( $\alpha$ , remainder)  
Radionuclide Production Cross Section

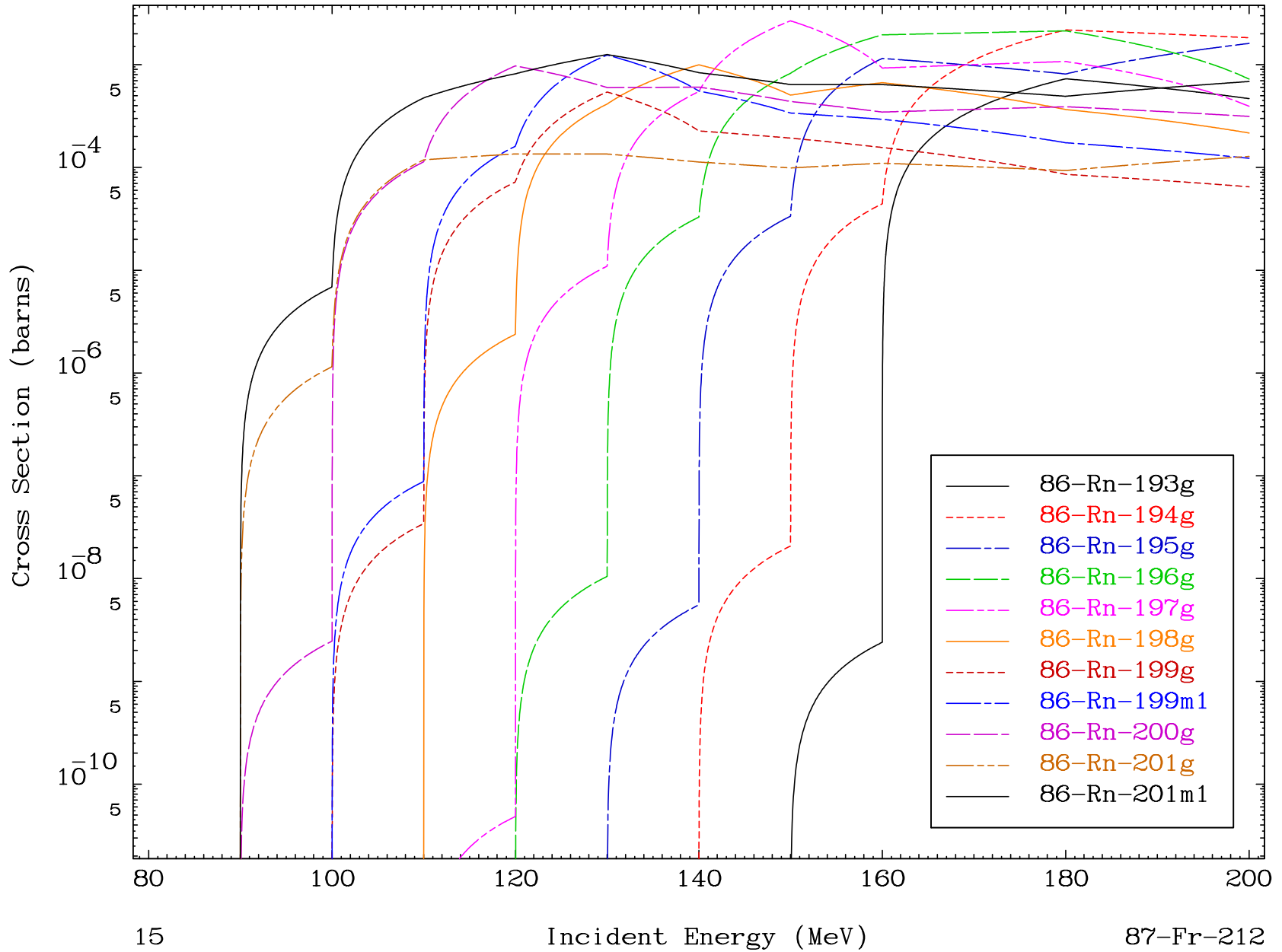
87-Fr-212



Radionuclide Production Cross Section





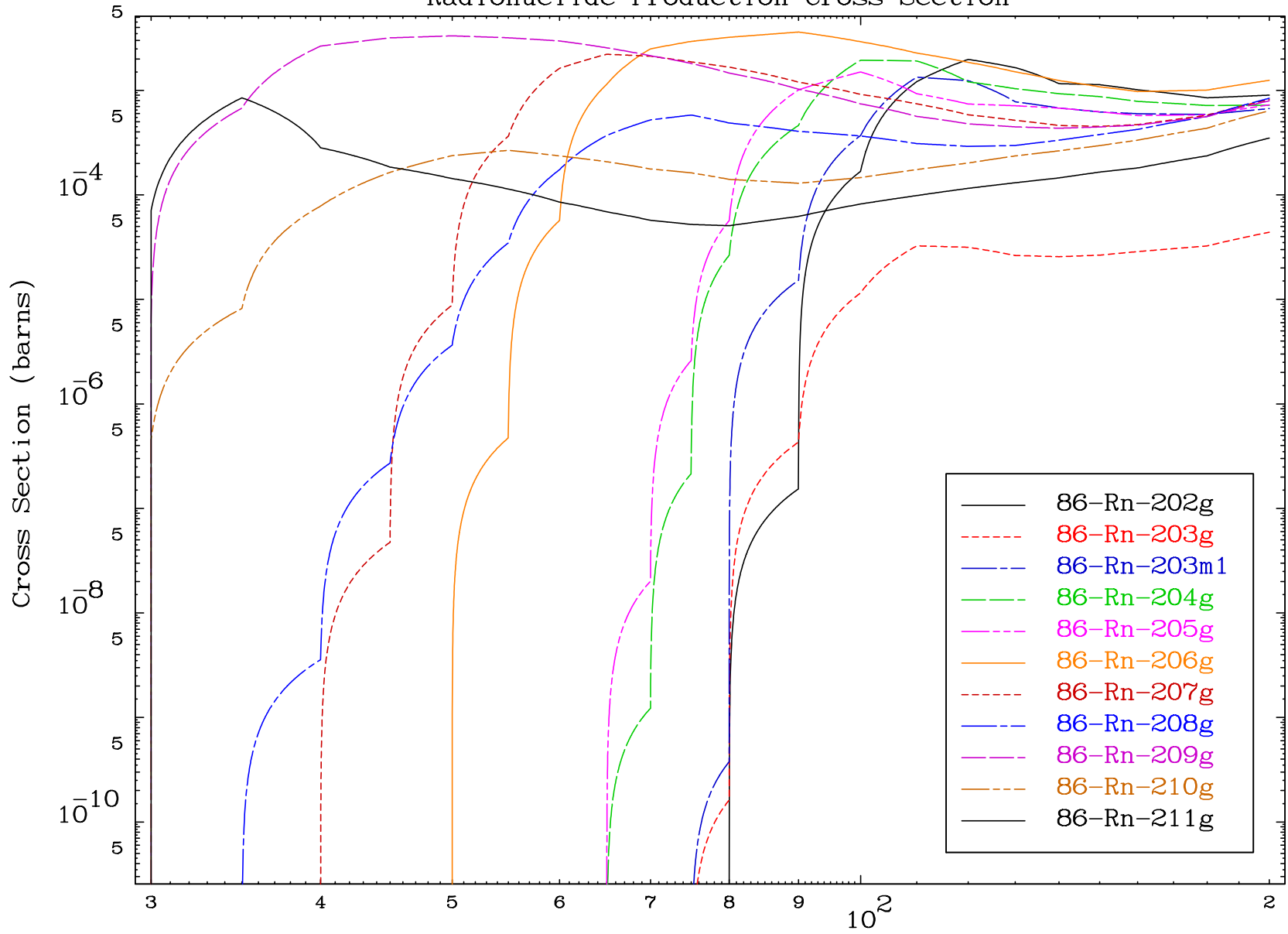


MAT 8725

( $\alpha$ , remainder)

87-Fr-212

### Radionuclide Production Cross Section



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Incident Energy (MeV)

87-Fr-212

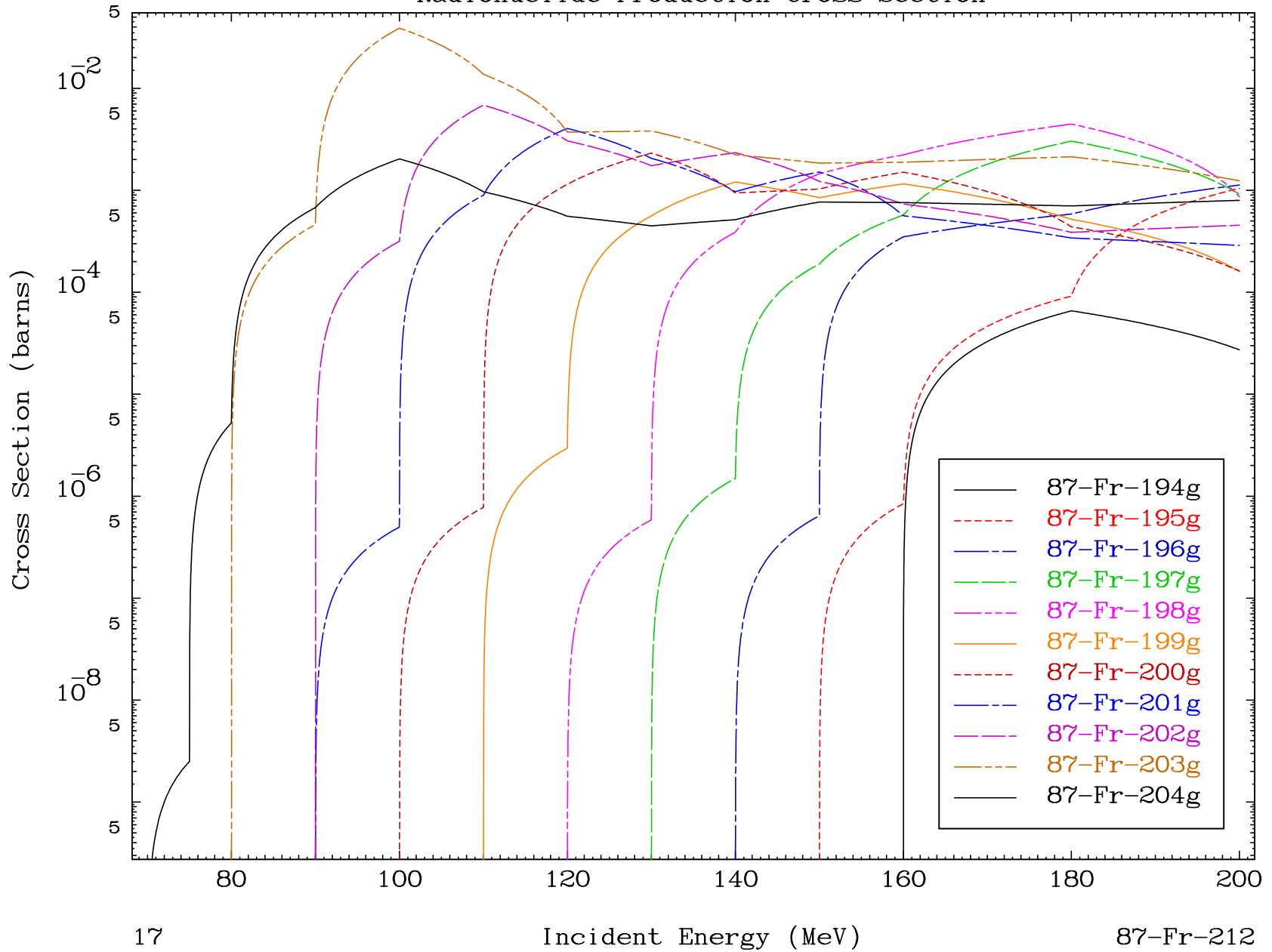


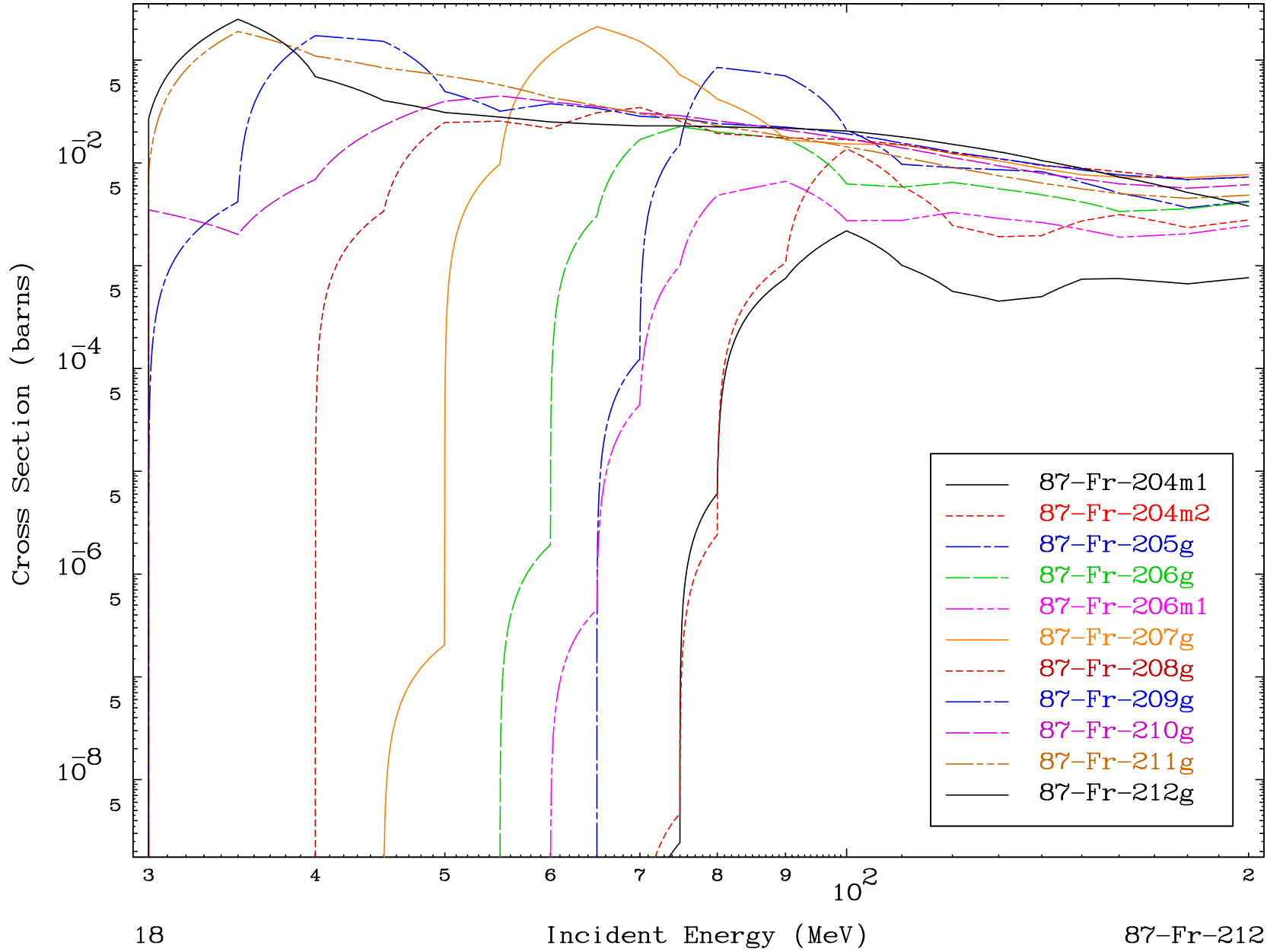
MAT 8725

( $\alpha$ , remainder)

87-Fr-212

### Radionuclide Production Cross Section



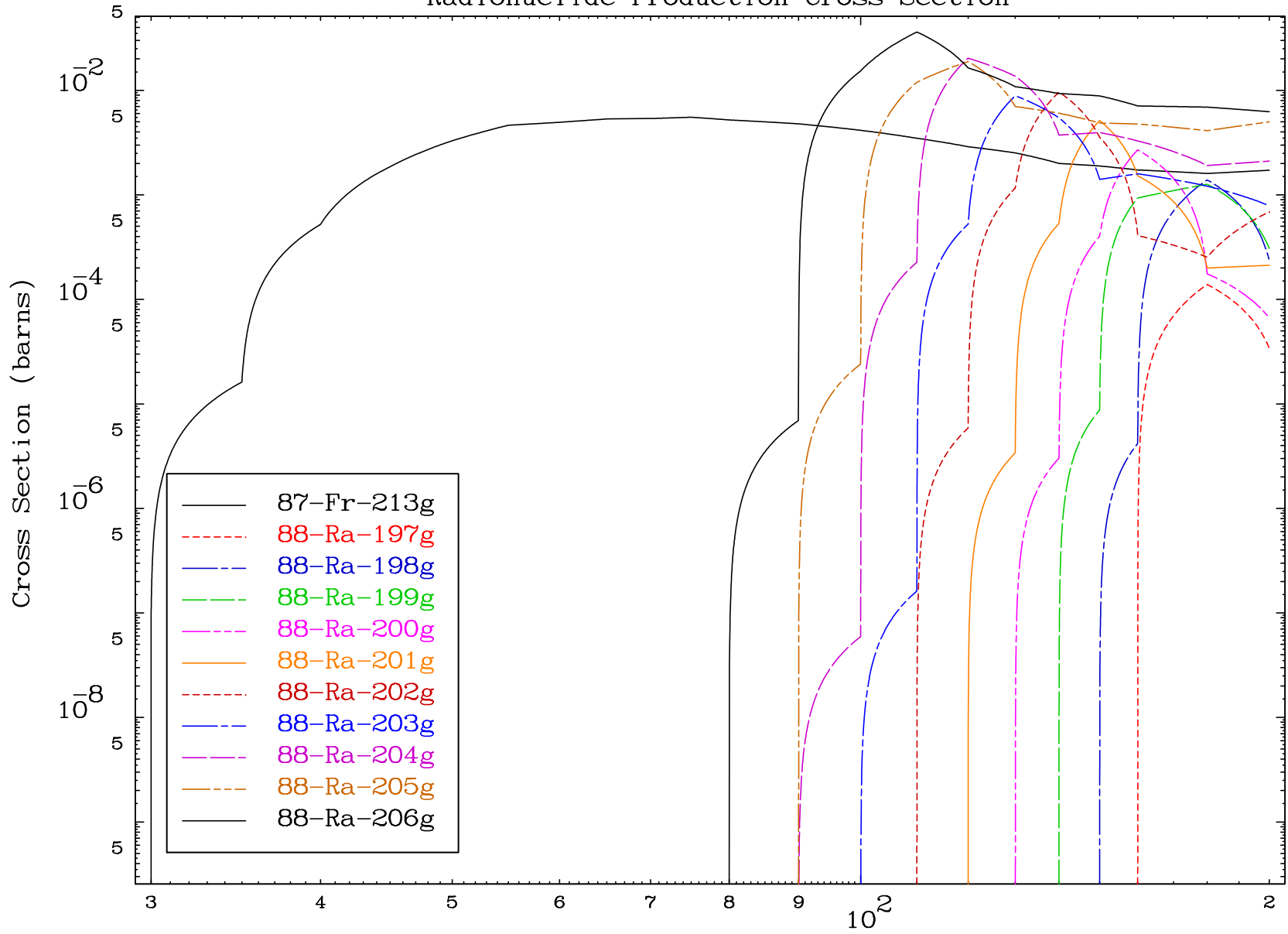


MAT 8725

( $\alpha$ , remainder)

87-Fr-212

### Radionuclide Production Cross Section



19

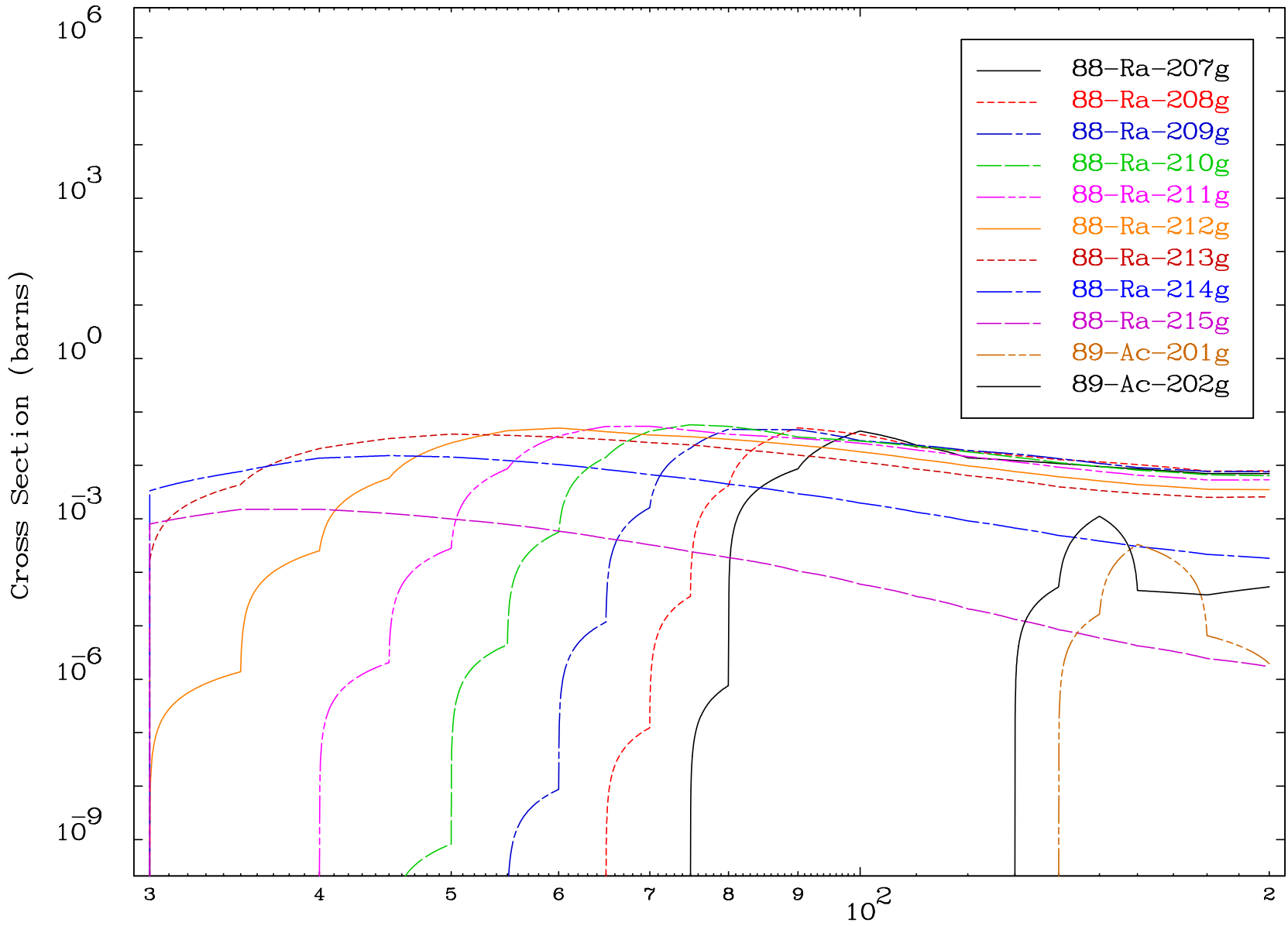
Incident Energy (MeV)

87-Fr-212

MAT 8725

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

87-Fr-212



20

Incident Energy (MeV)

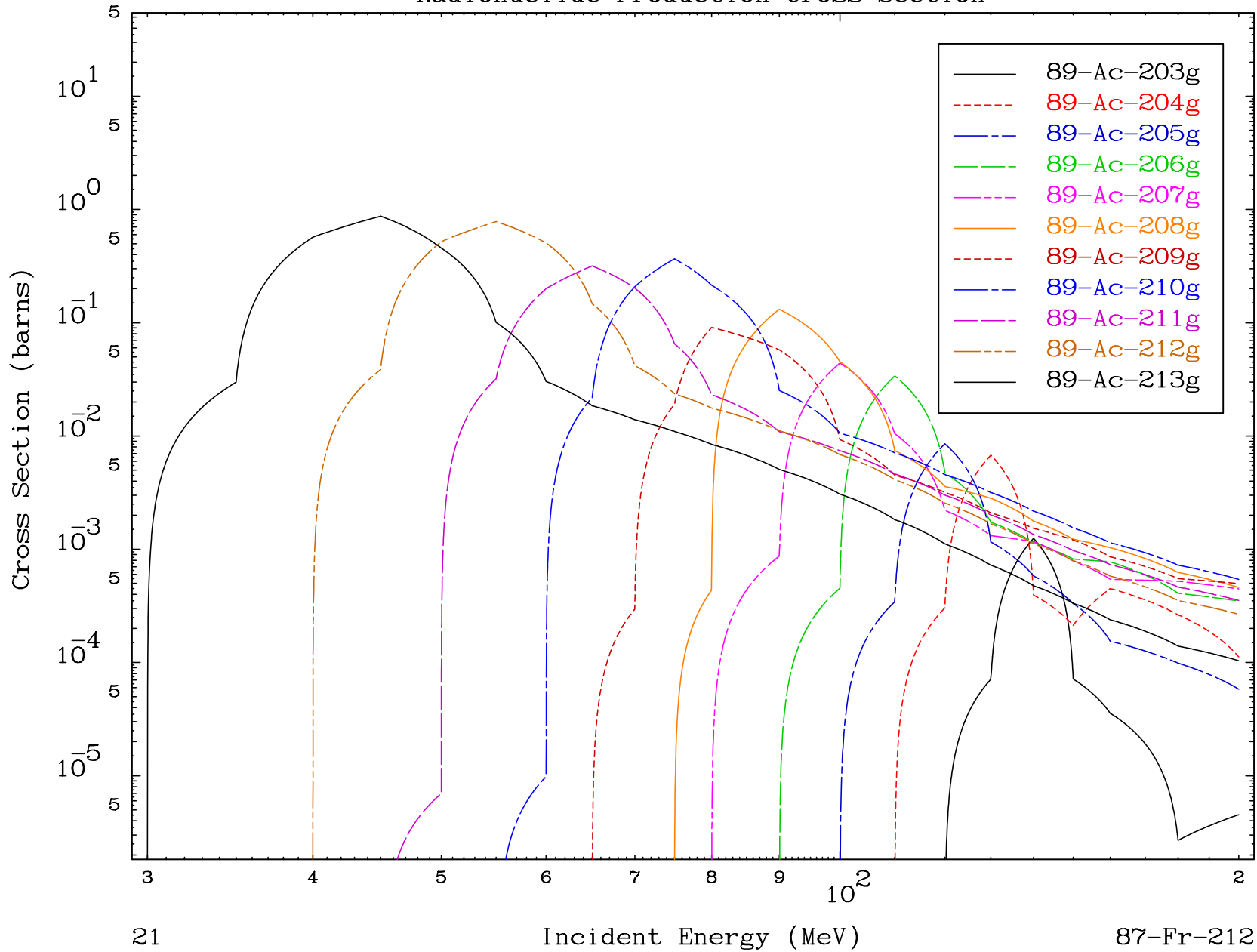
87-Fr-212

MAT 8725

( $\alpha$ , remainder)

87-Fr-212

### Radionuclide Production Cross Section

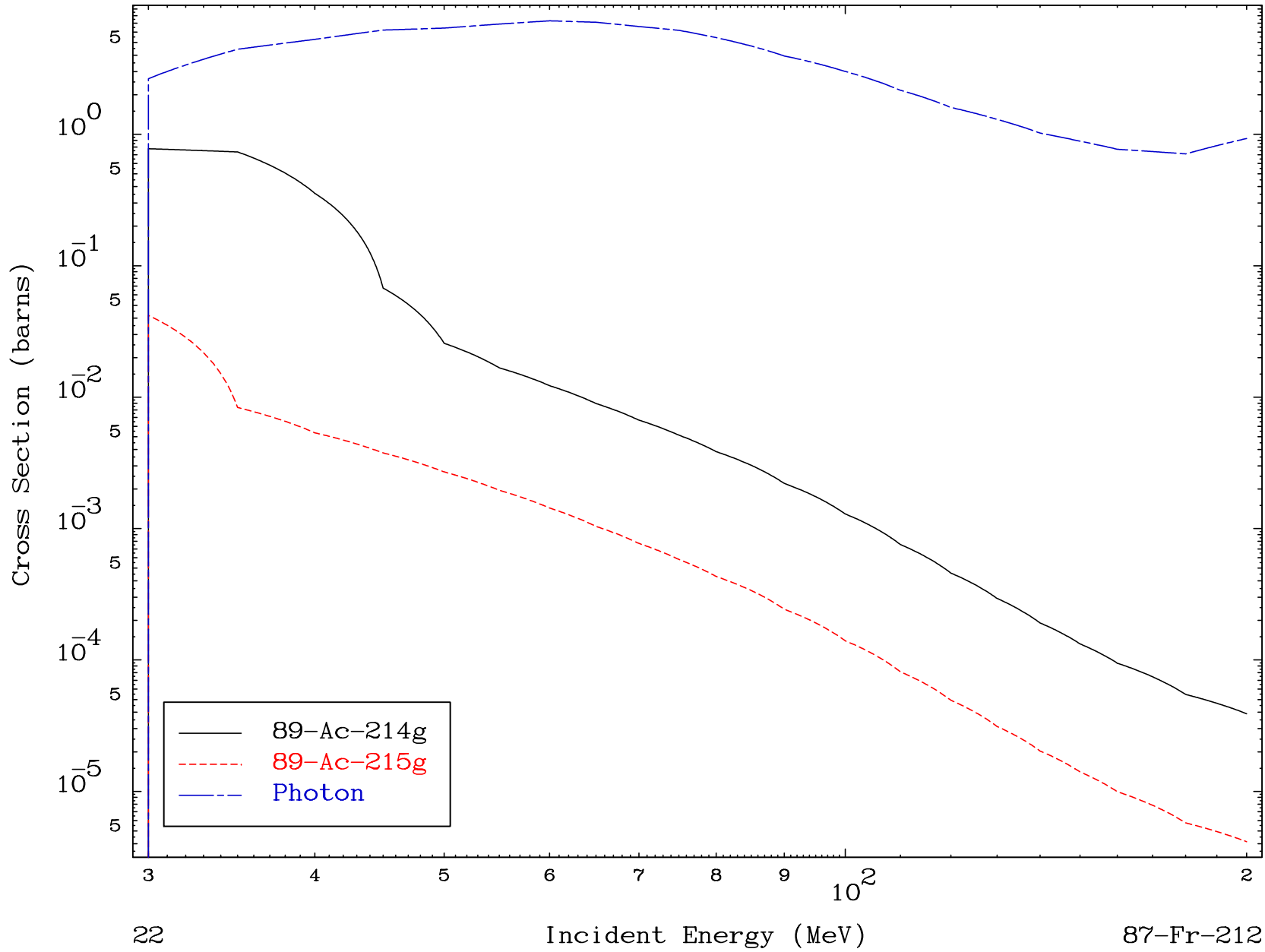


MAT 8725

( $\alpha$ , remainder)

87-Fr-212

Radionuclide Production Cross Section



MAT 8725

$\alpha$  Fission  
Radionuclide Production Cross Section

87-Fr-212

