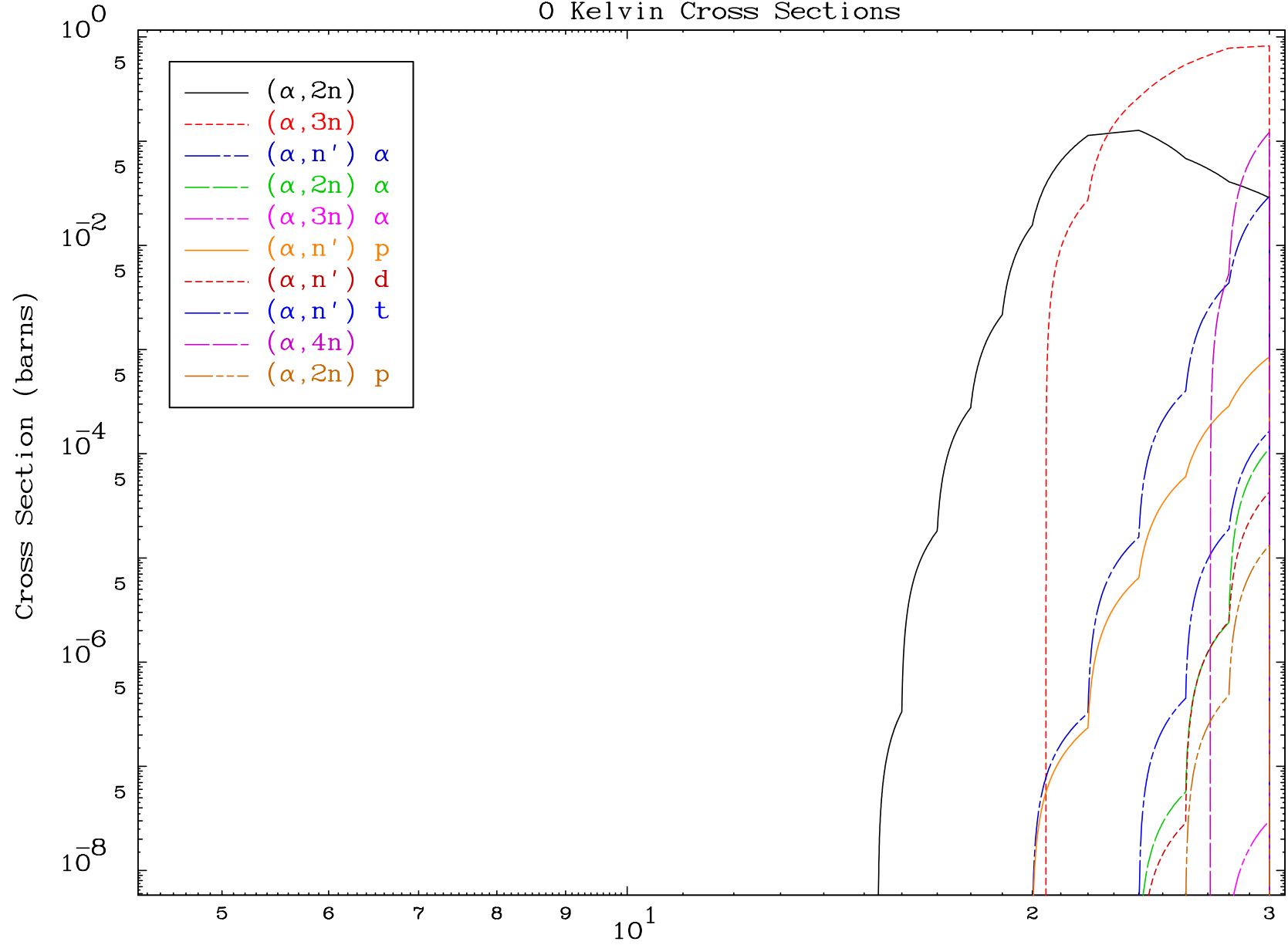


MAT 8767

α Neutron Production
0 Kelvin Cross Sections

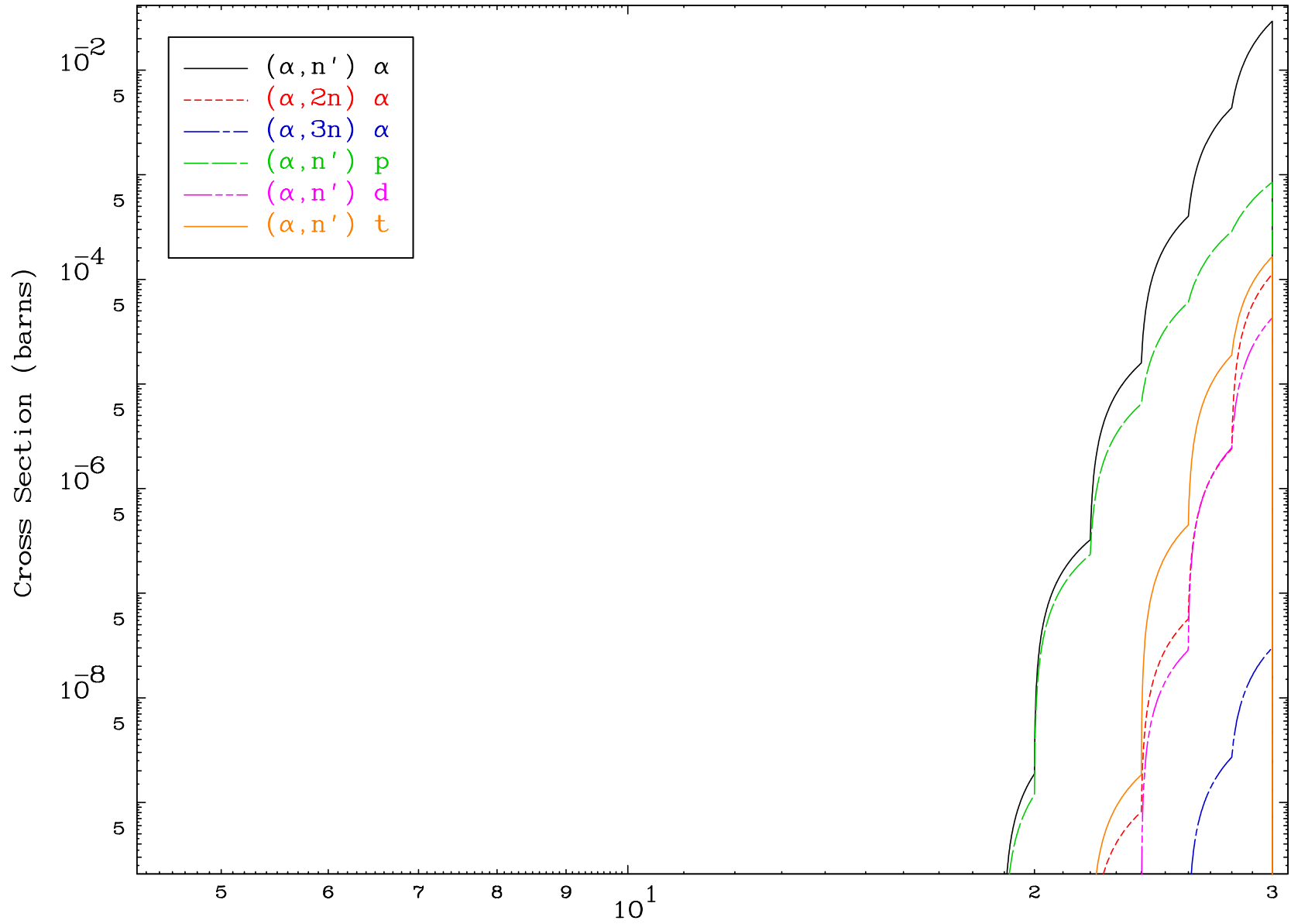
87-Fr-226

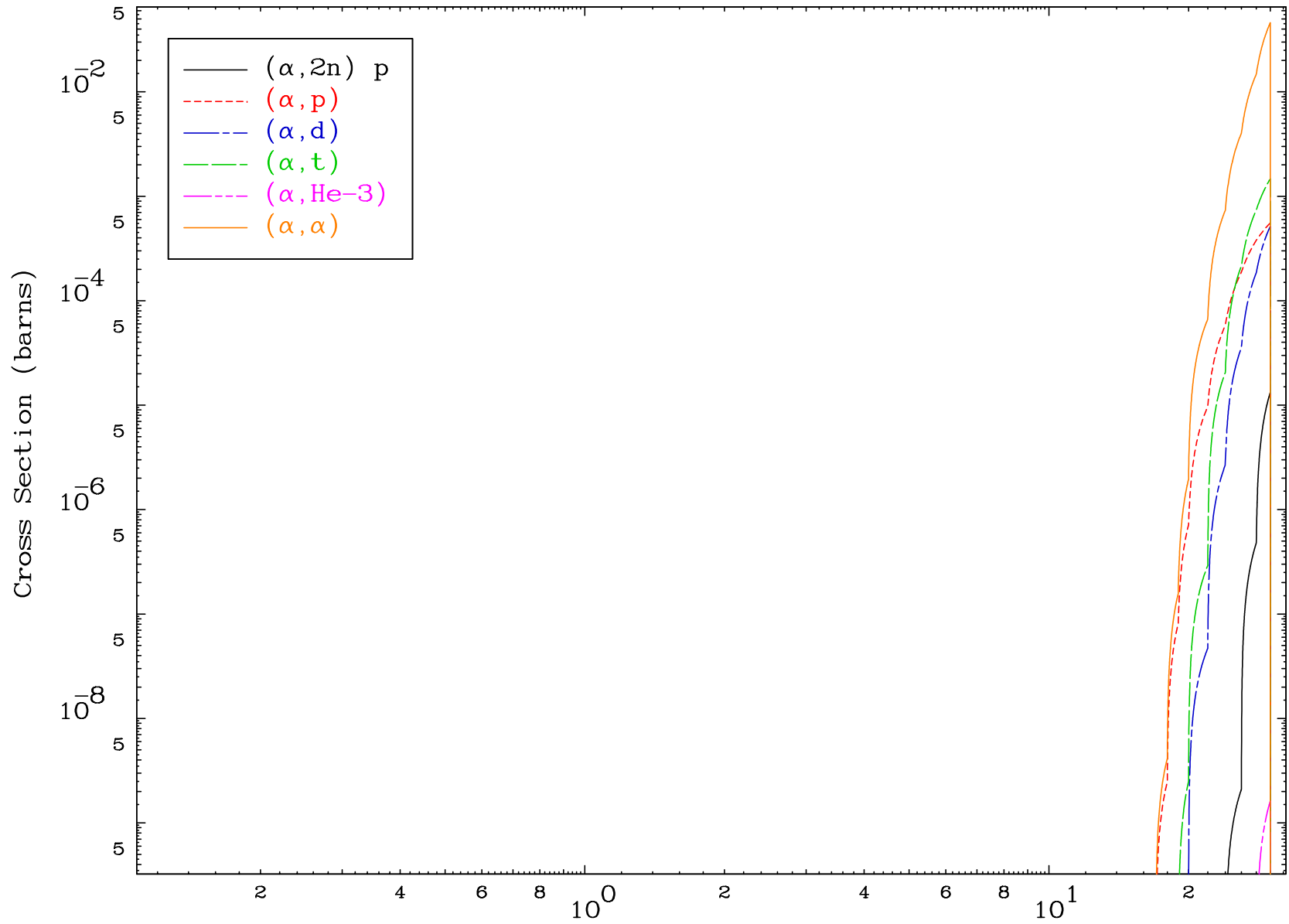


2

Incident Energy (MeV)

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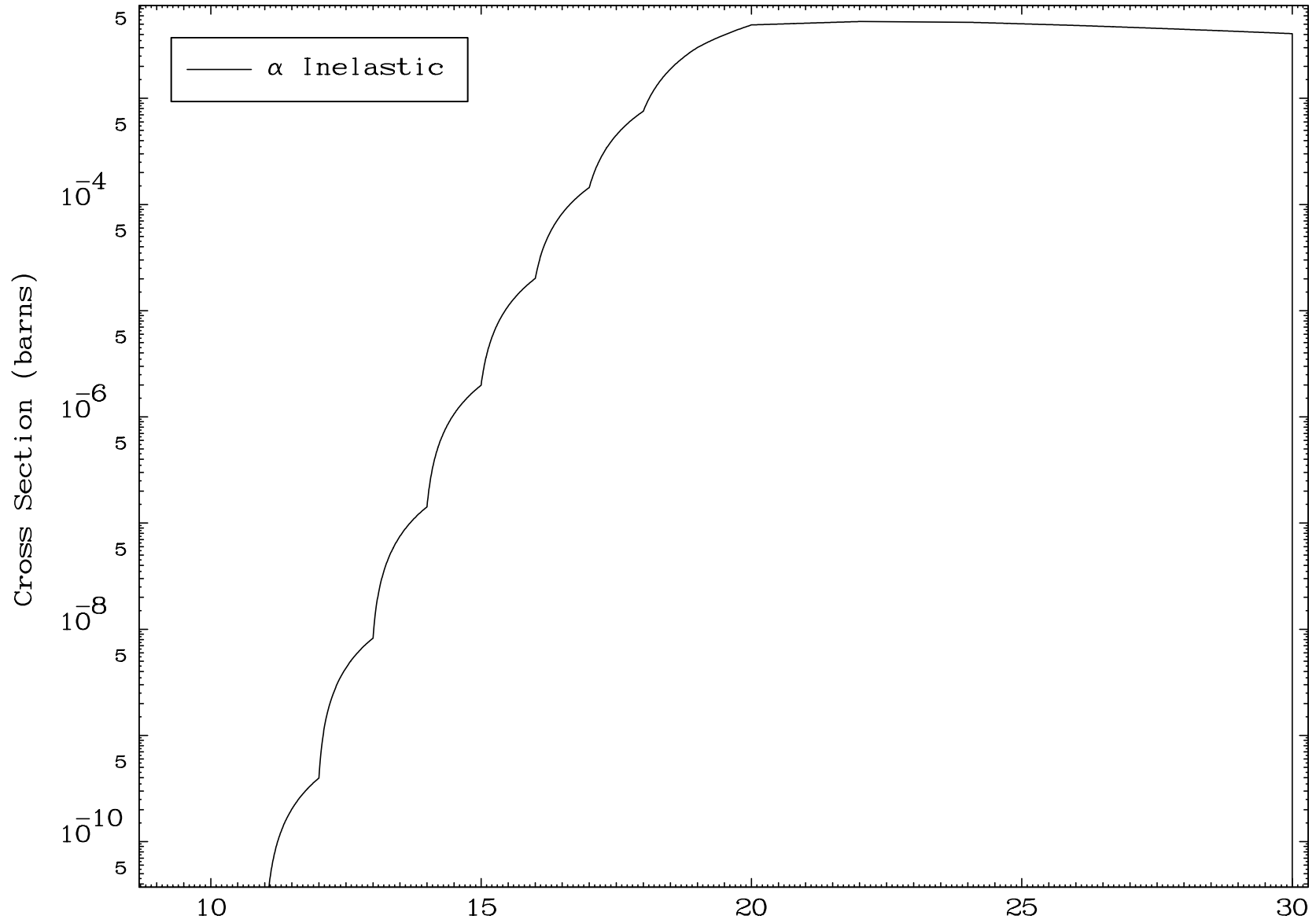




MAT 8767

(α, n') Level
0 Kelvin Cross Sections

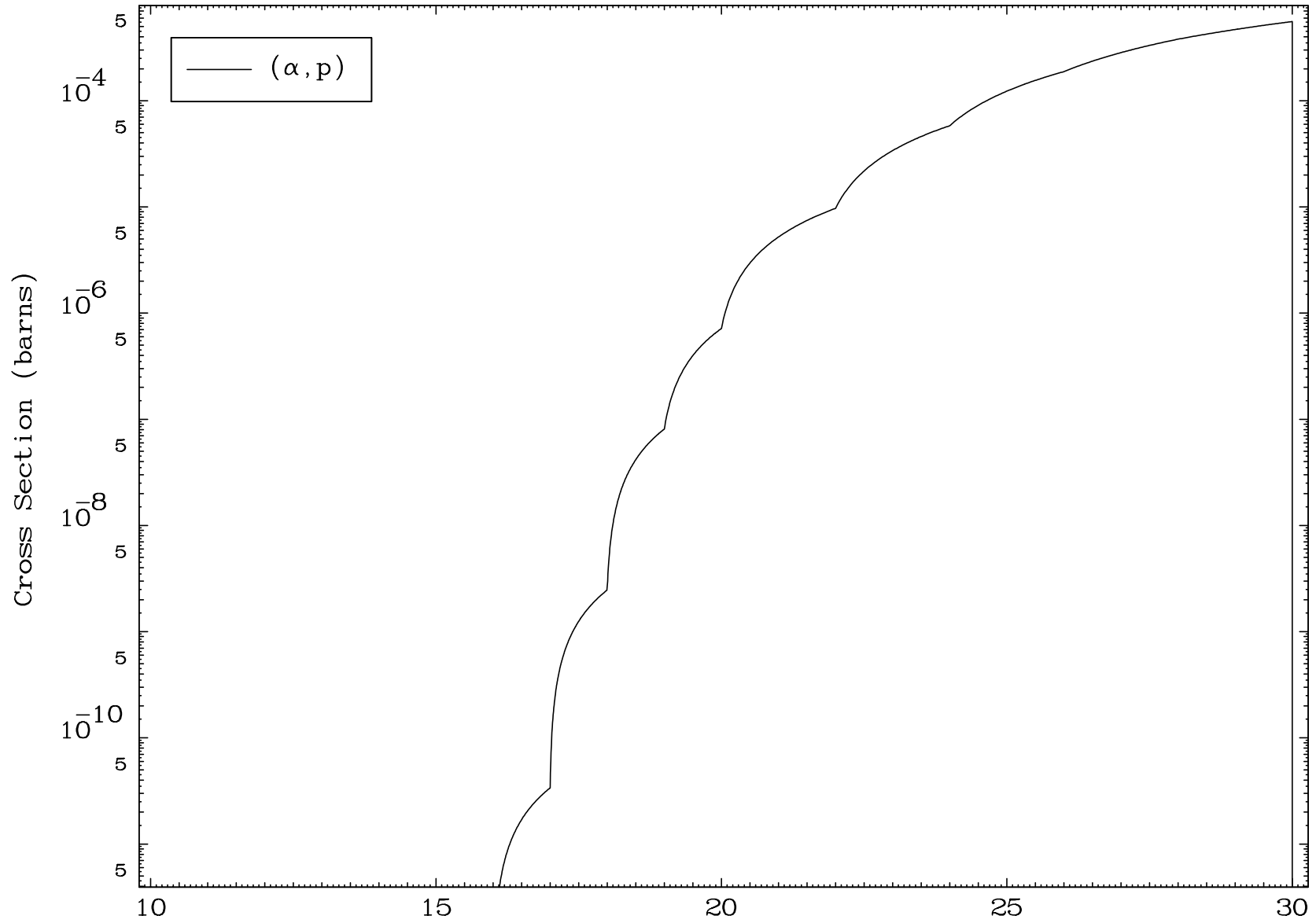
87-Fr-226

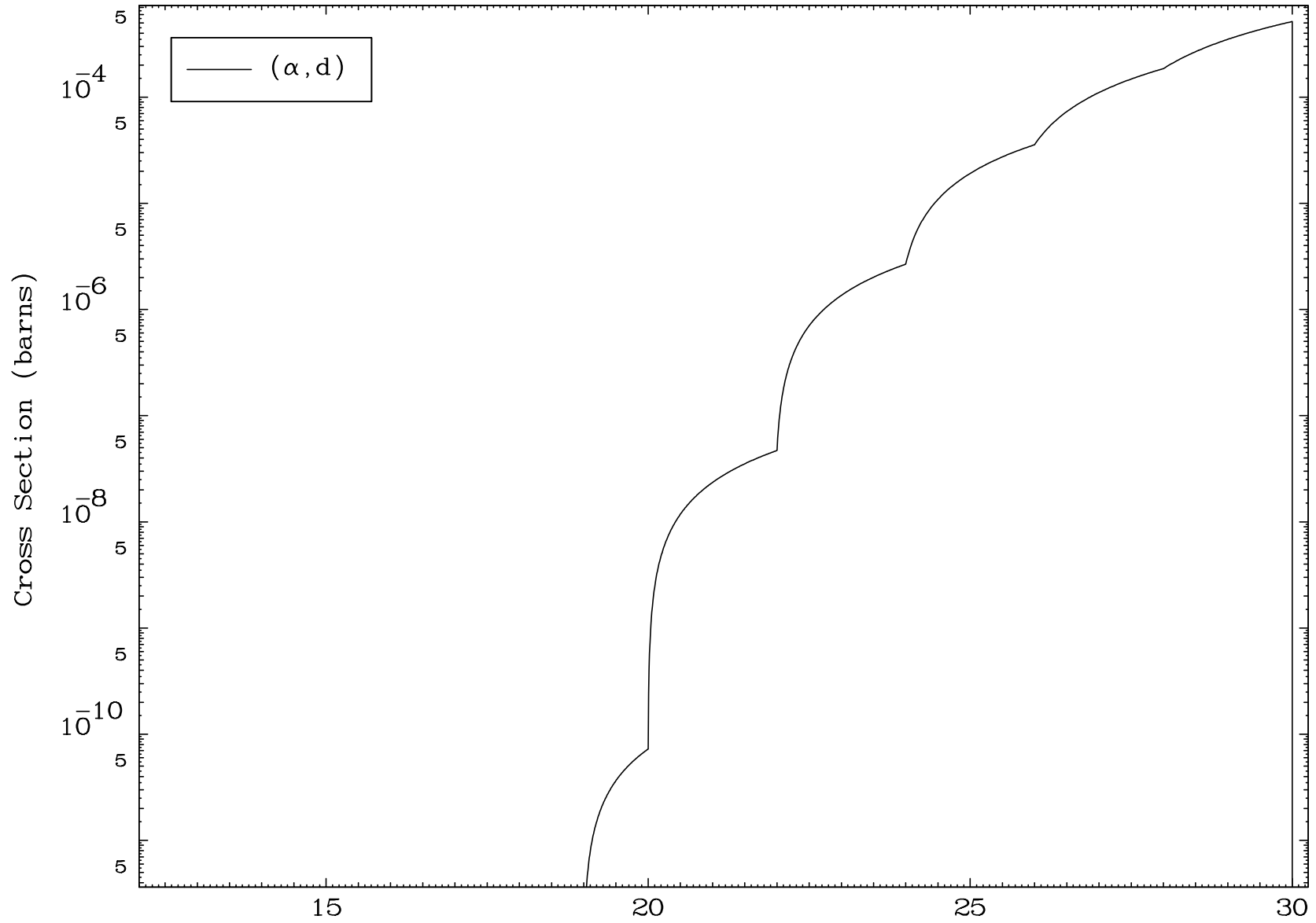


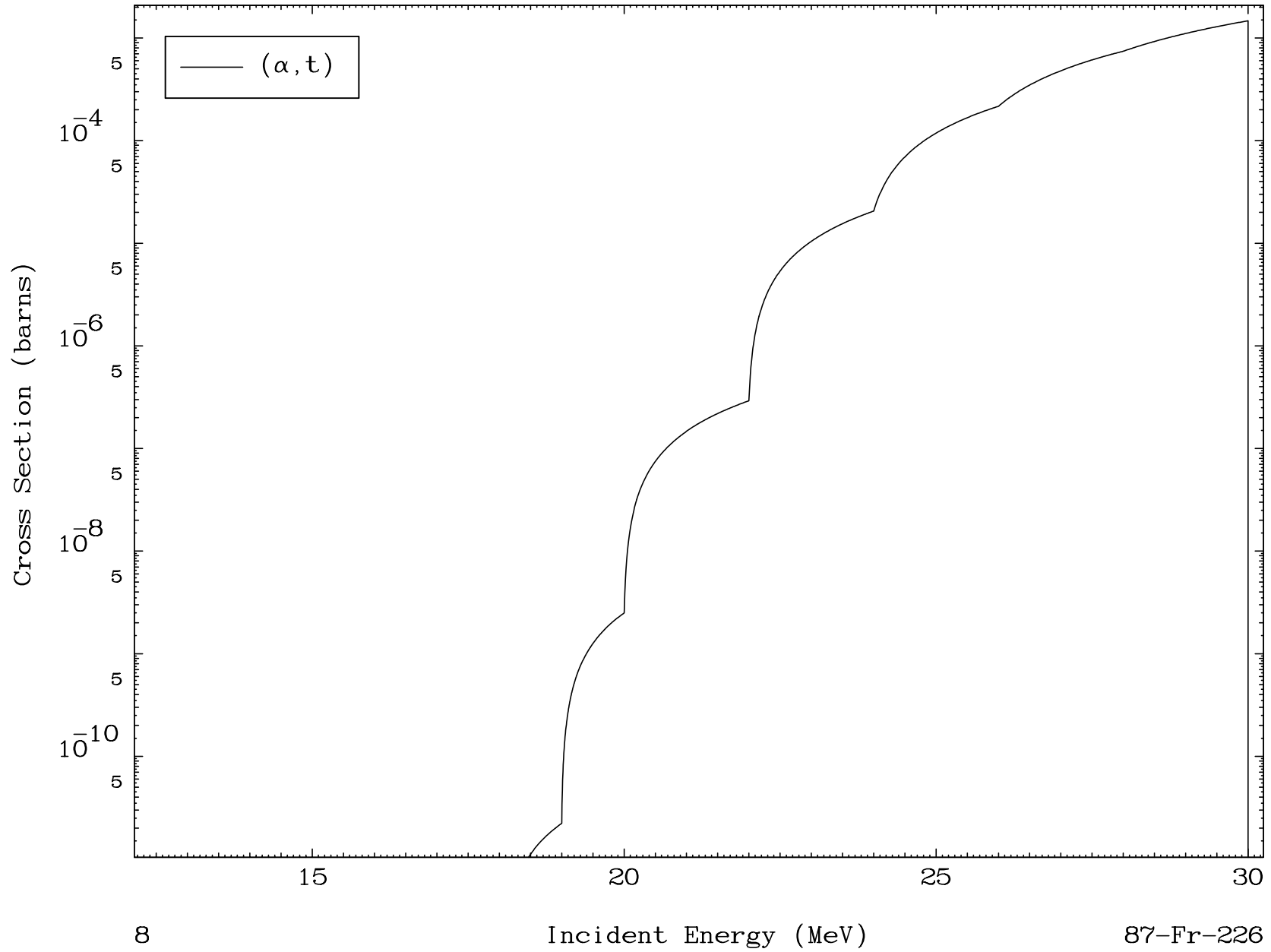
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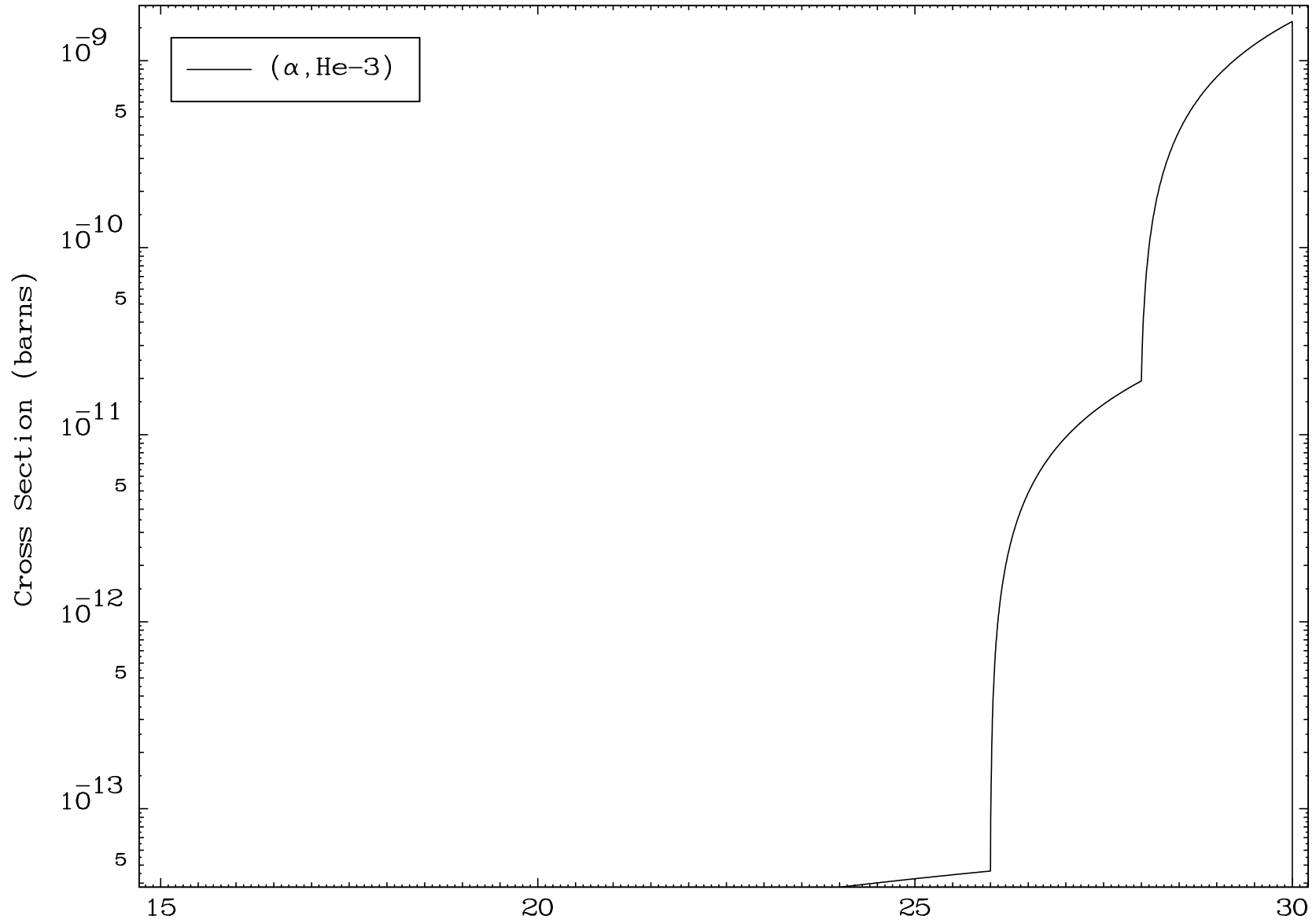
Incident Energy (MeV)

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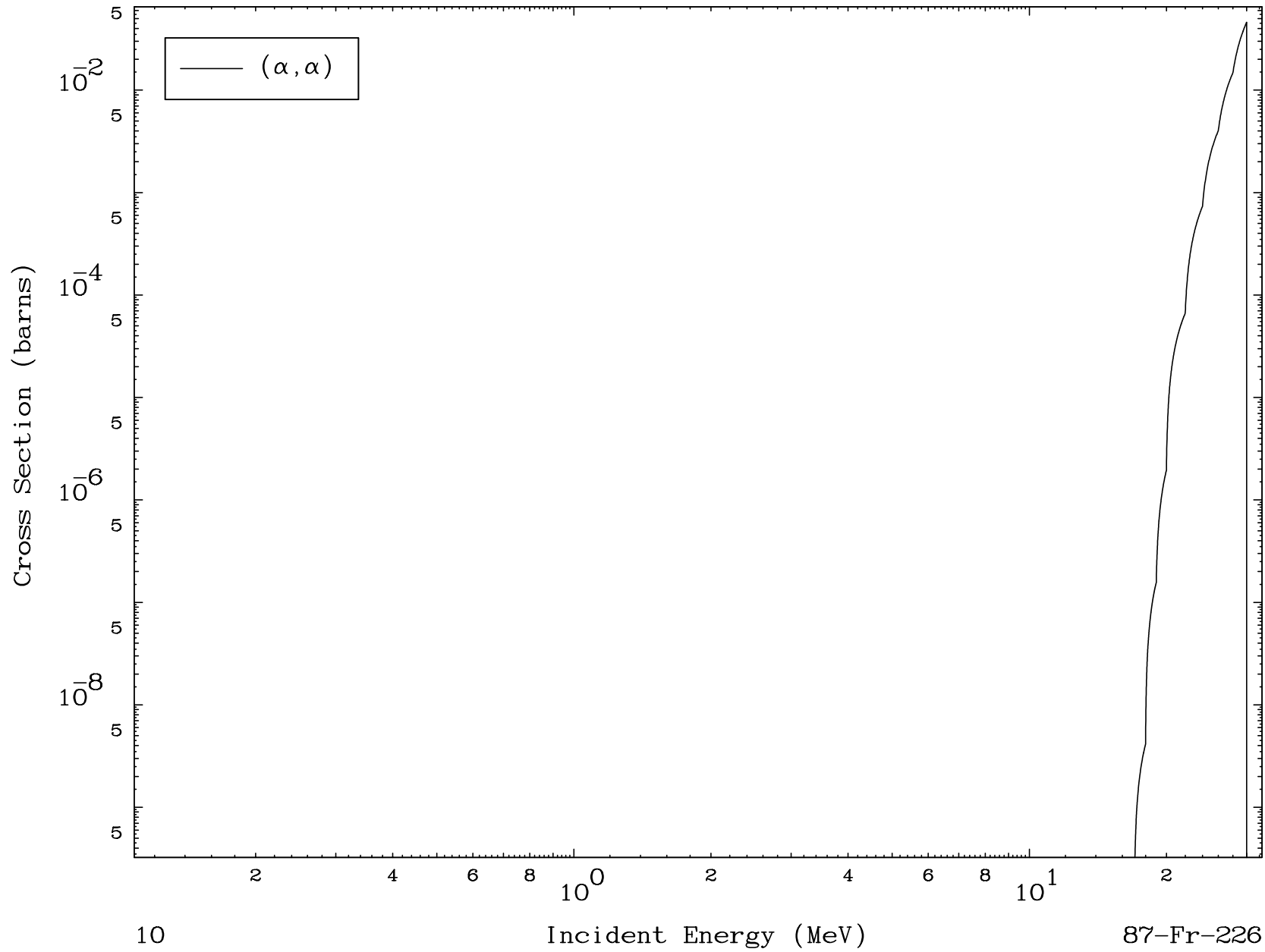




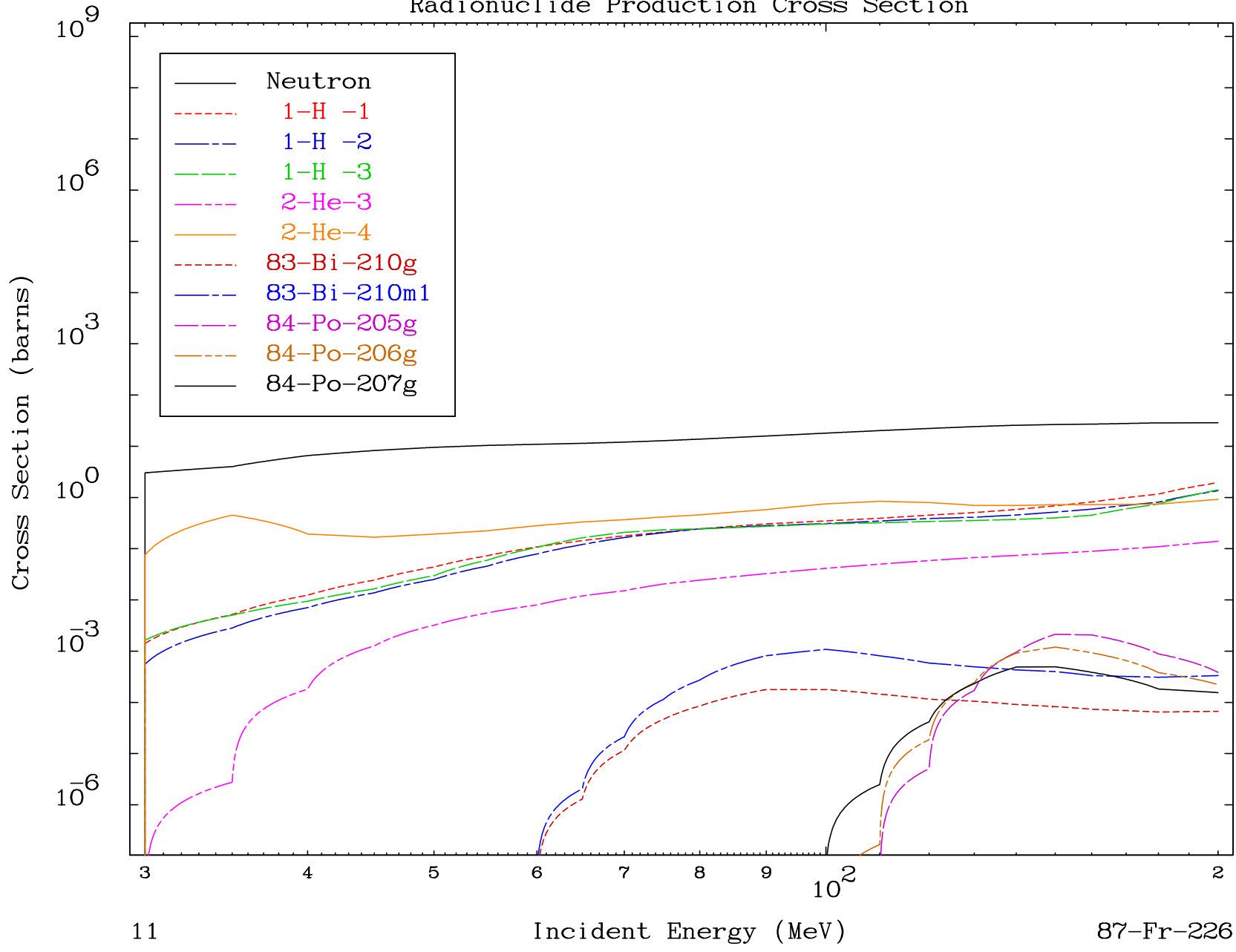
MAT 8767

(α, α) Levels
0 Kelvin Cross Sections

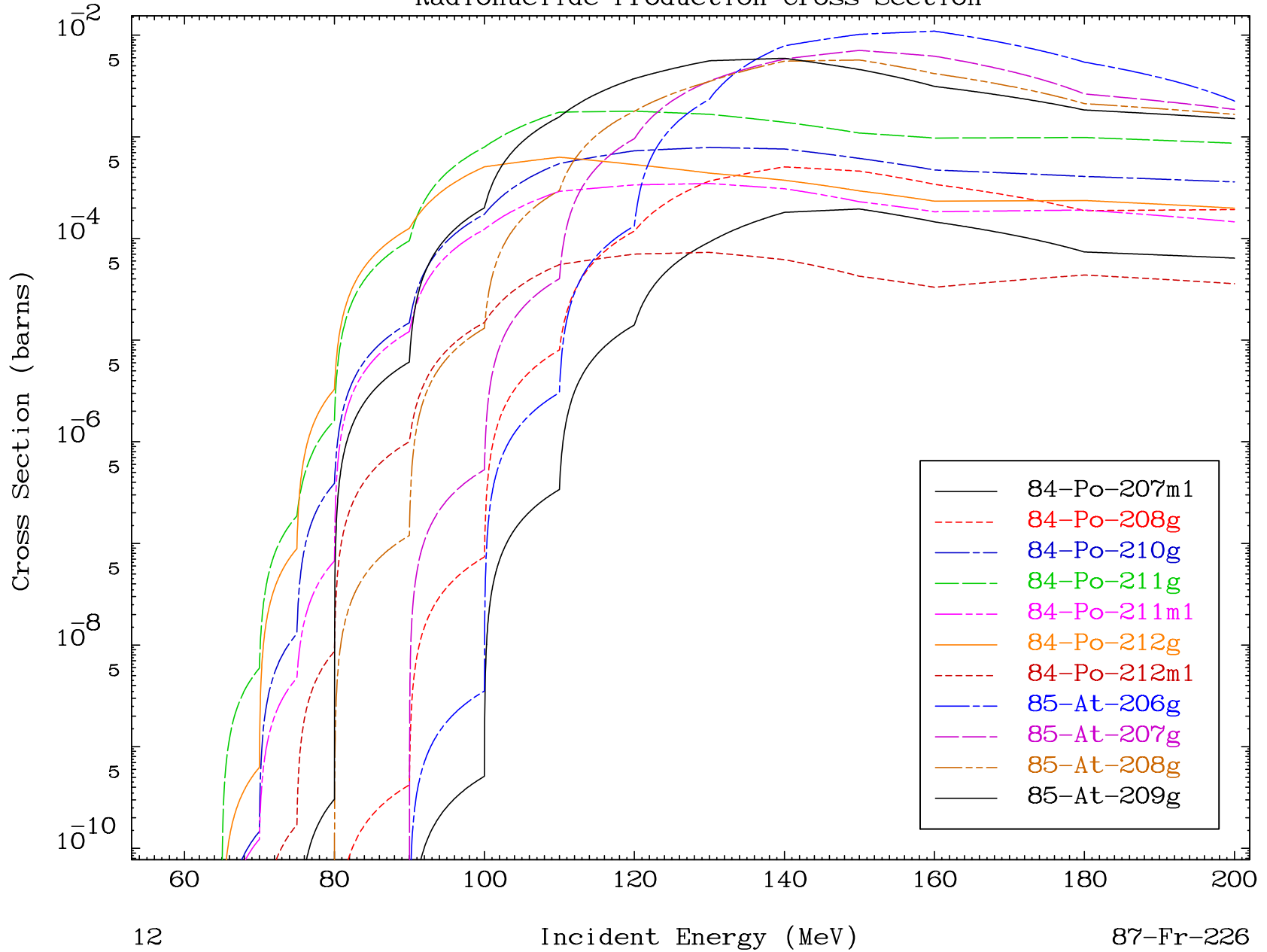
87-Fr-226



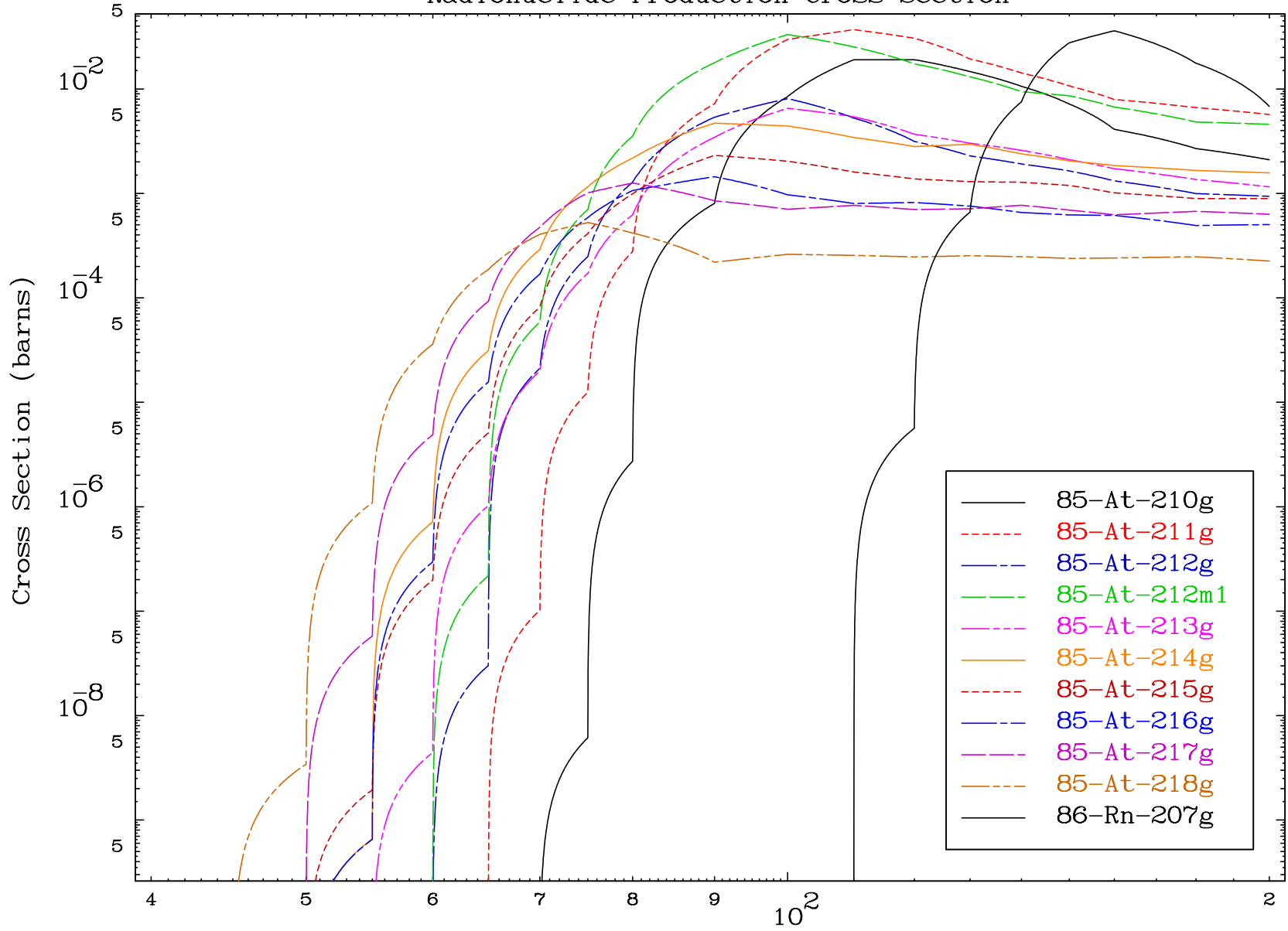
Radionuclide Production Cross Section



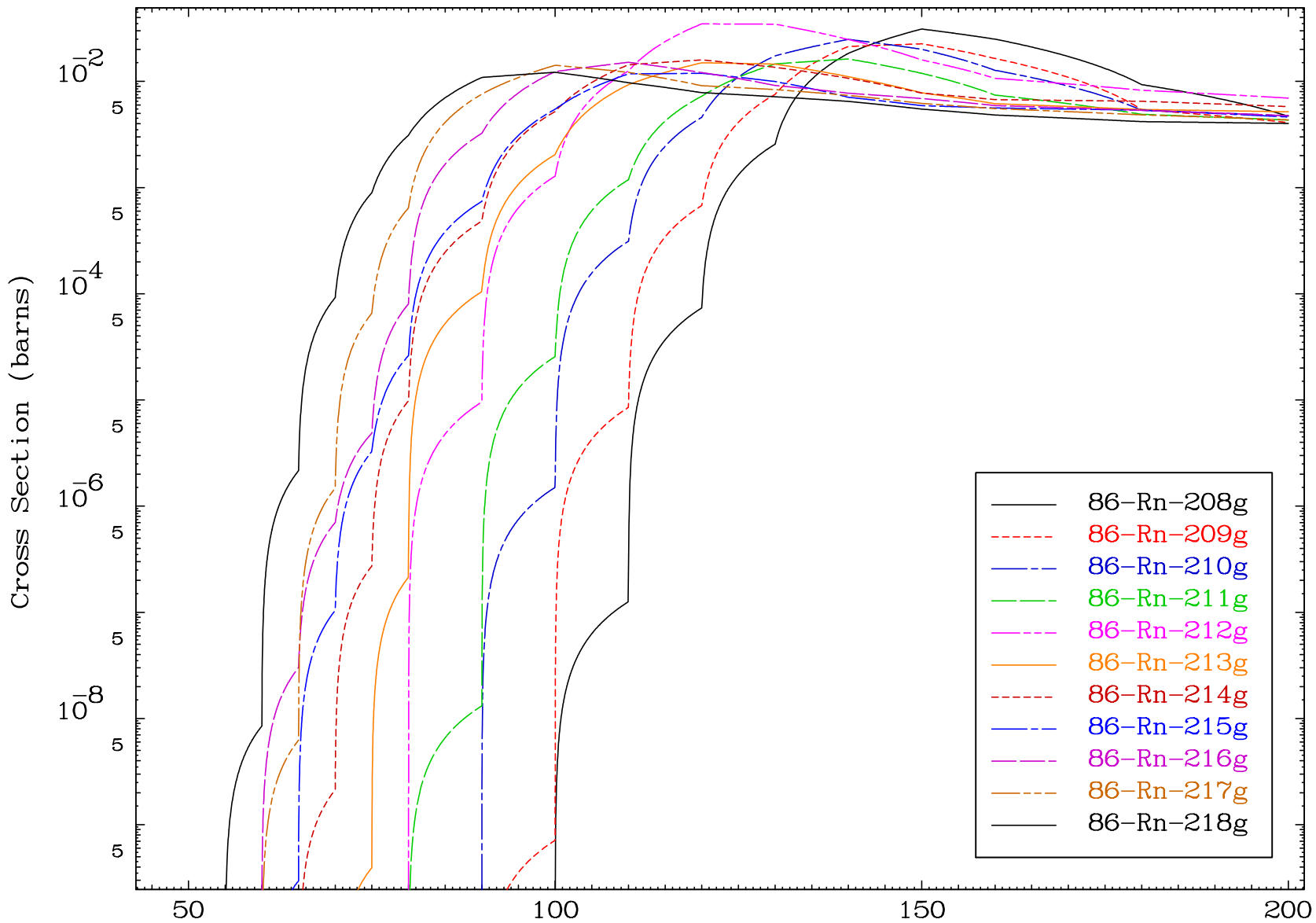
Radionuclide Production Cross Section



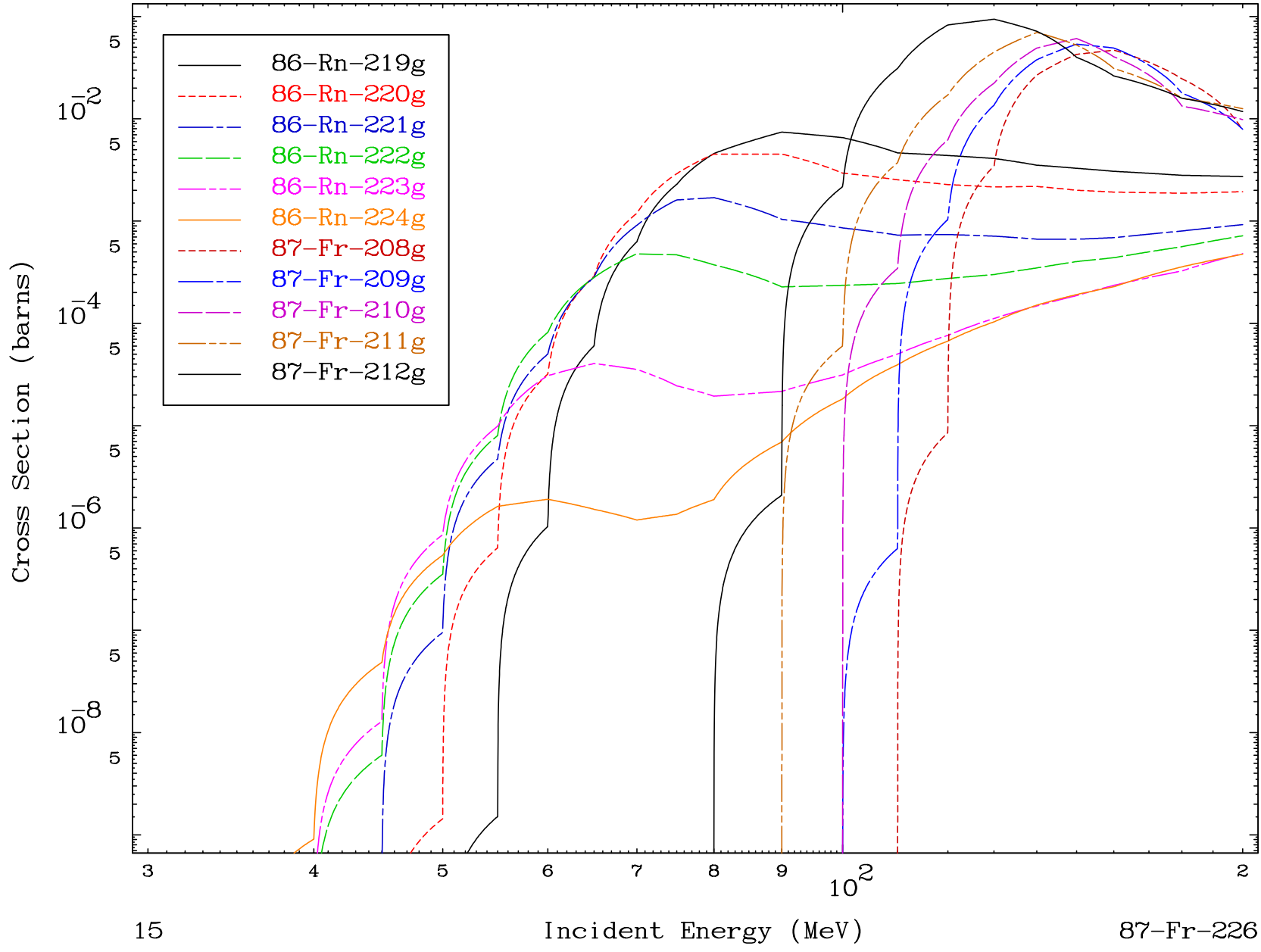
Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

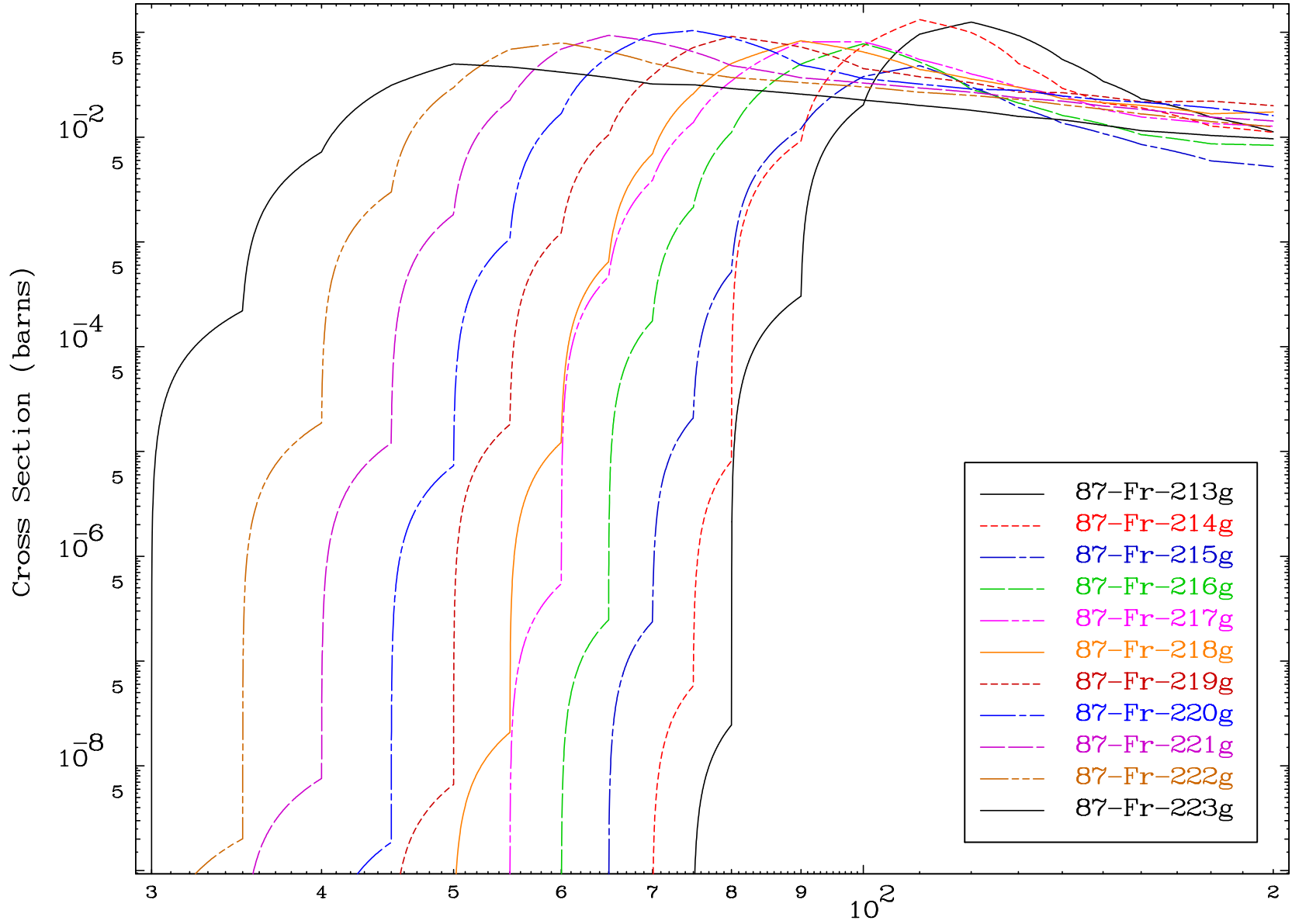


MAT 8767

(α , remainder)

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Radionuclide Production Cross Section

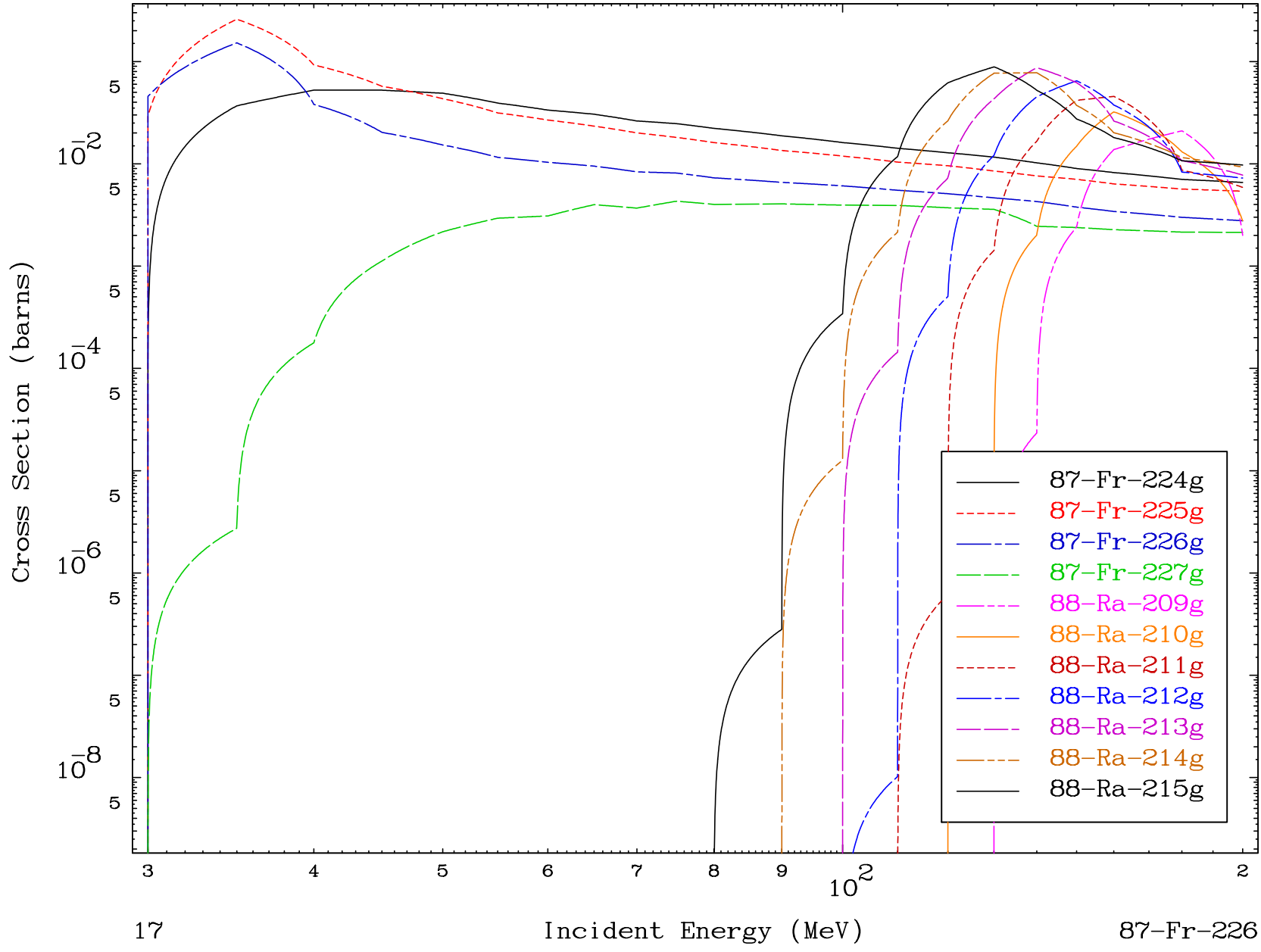


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Incident Energy (MeV)

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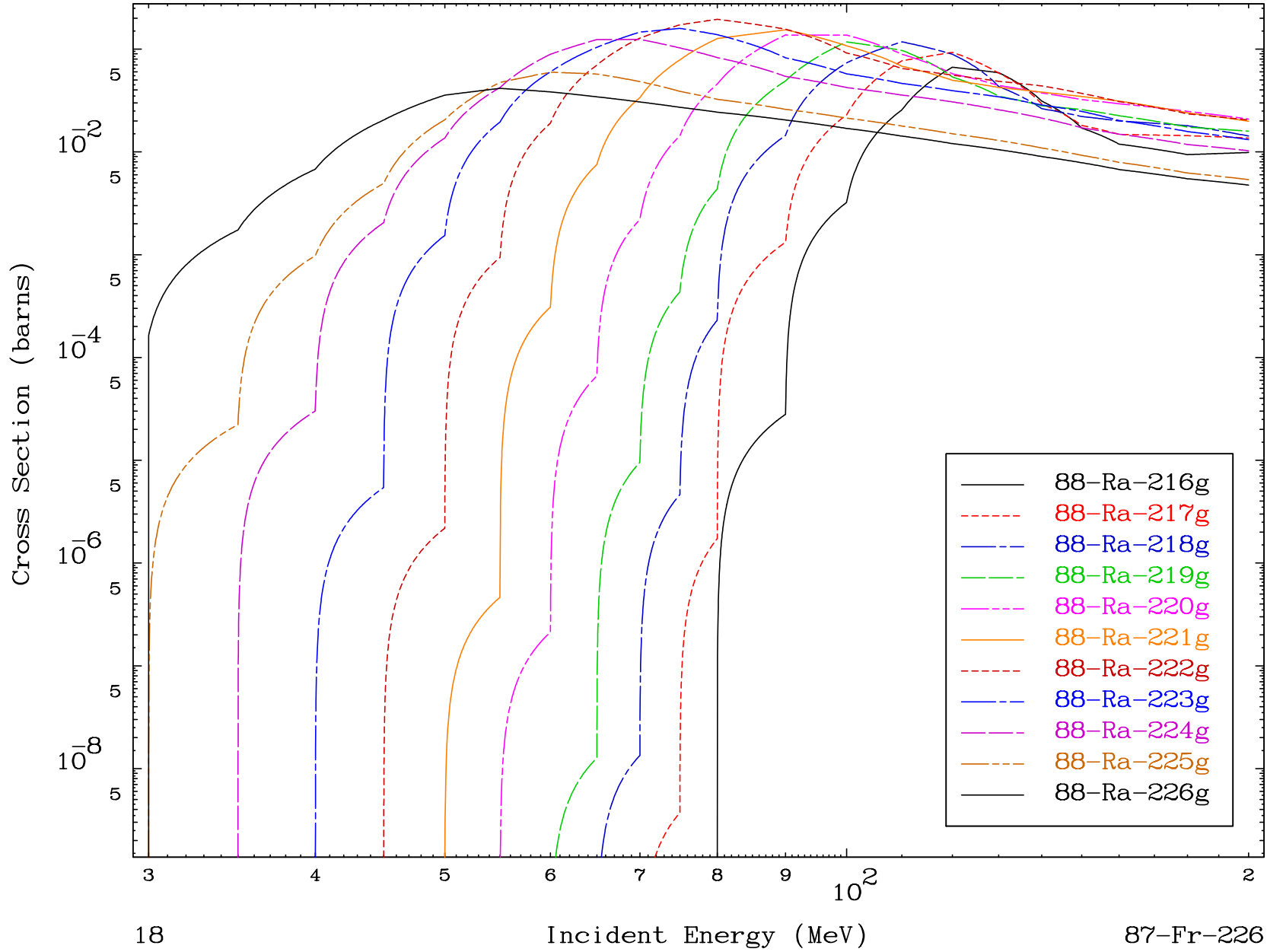
Radionuclide Production Cross Section



MAT 8767

(α , remainder)
Radionuclide Production Cross Section

87-Fr-226

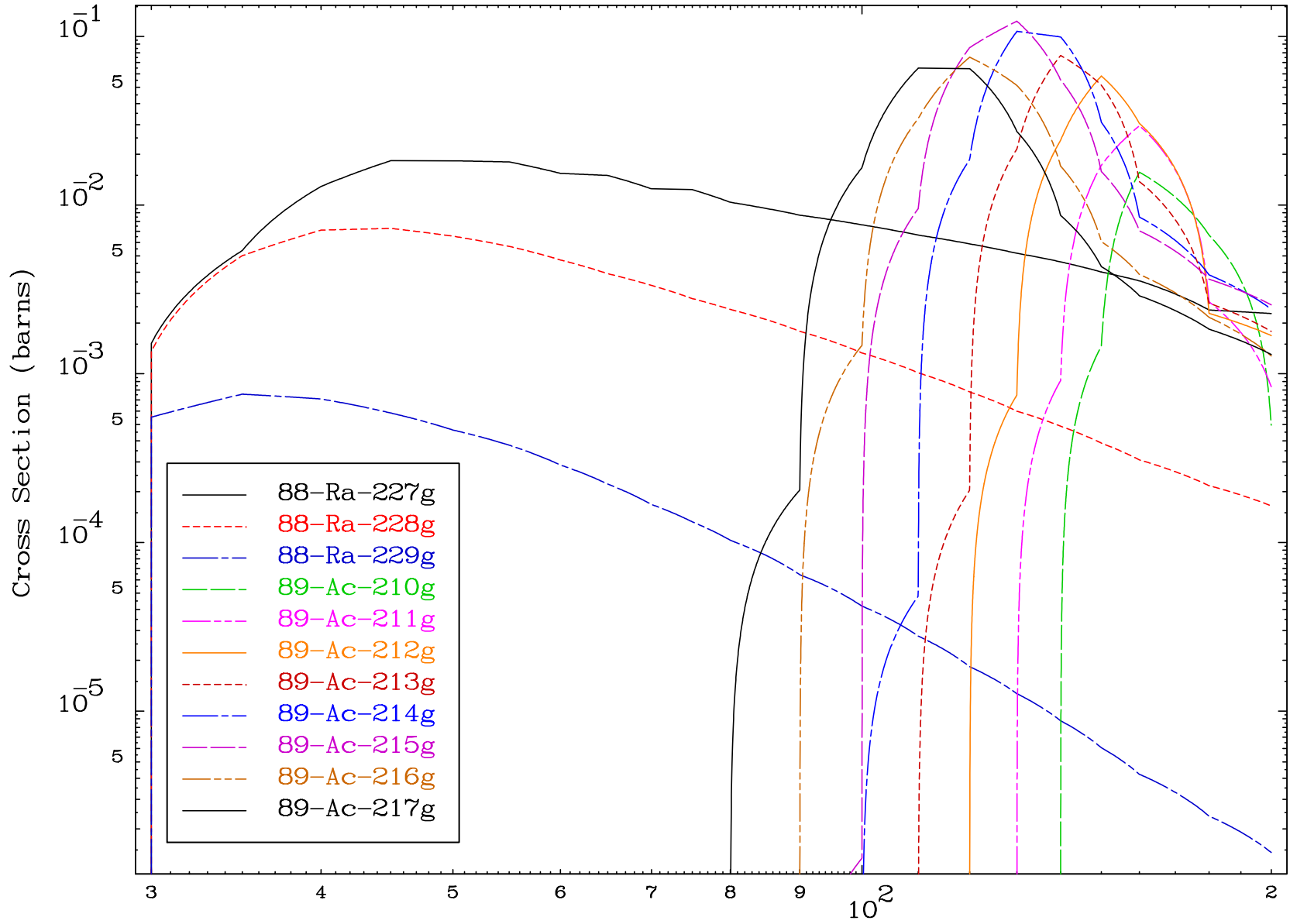


MAT 8767

(α , remainder)

87-Fr-226

Radionuclide Production Cross Section



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Incident Energy (MeV)

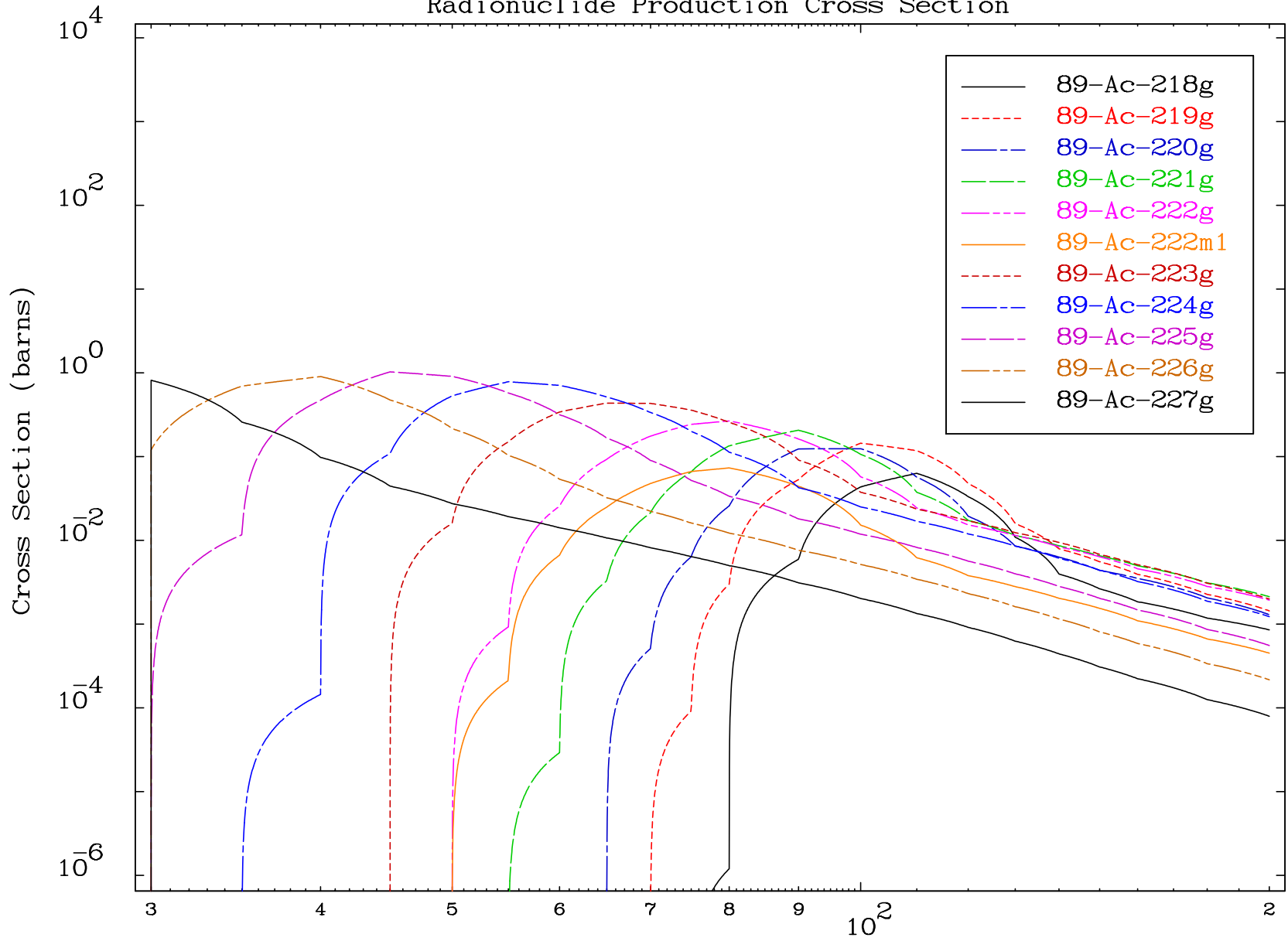
87-Fr-226

MAT 8767

(α , remainder)

87-Fr-226

Radionuclide Production Cross Section



20

Incident Energy (MeV)

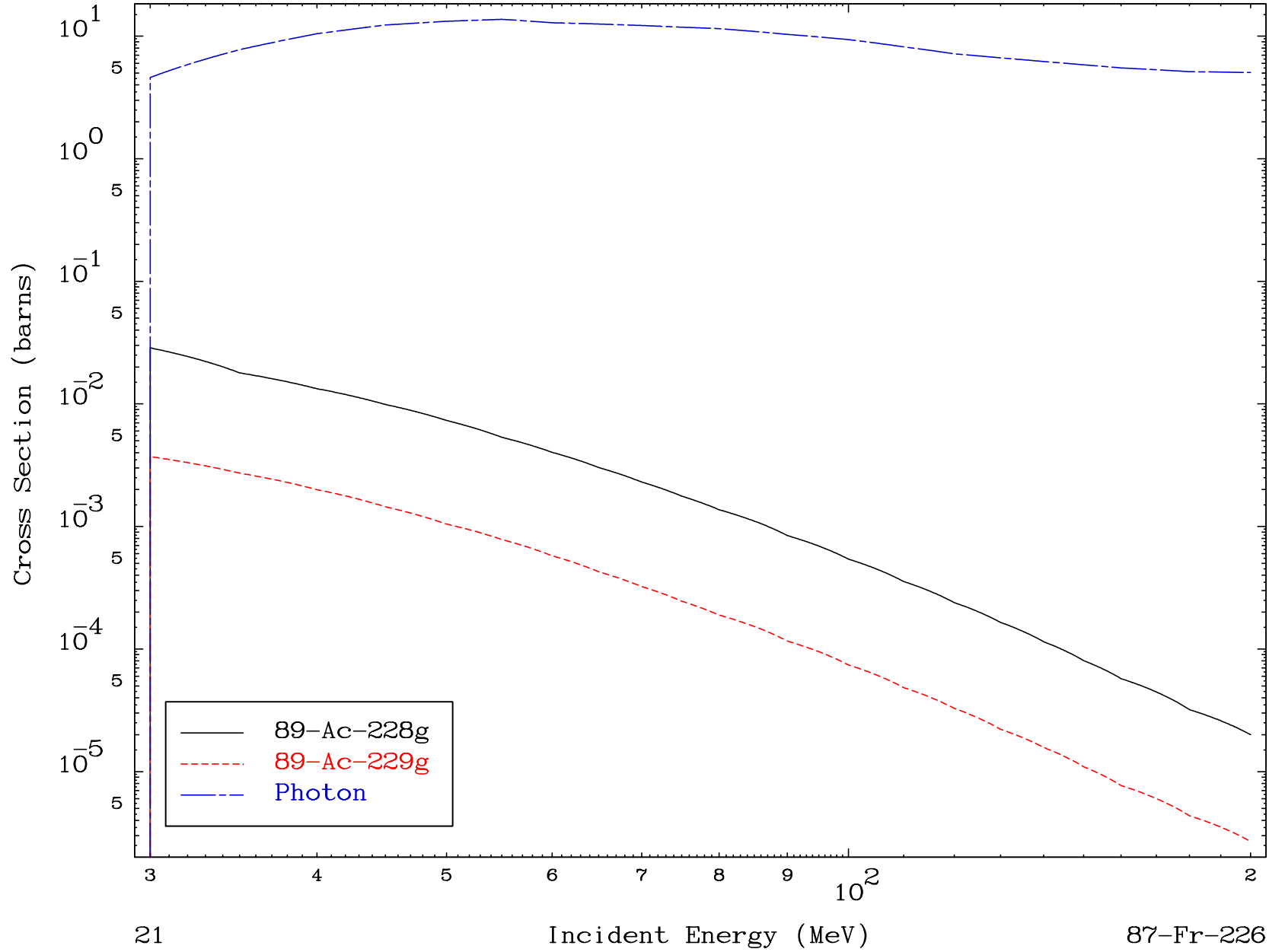
87-Fr-226

MAT 8767

(α , remainder)

87-Fr-226

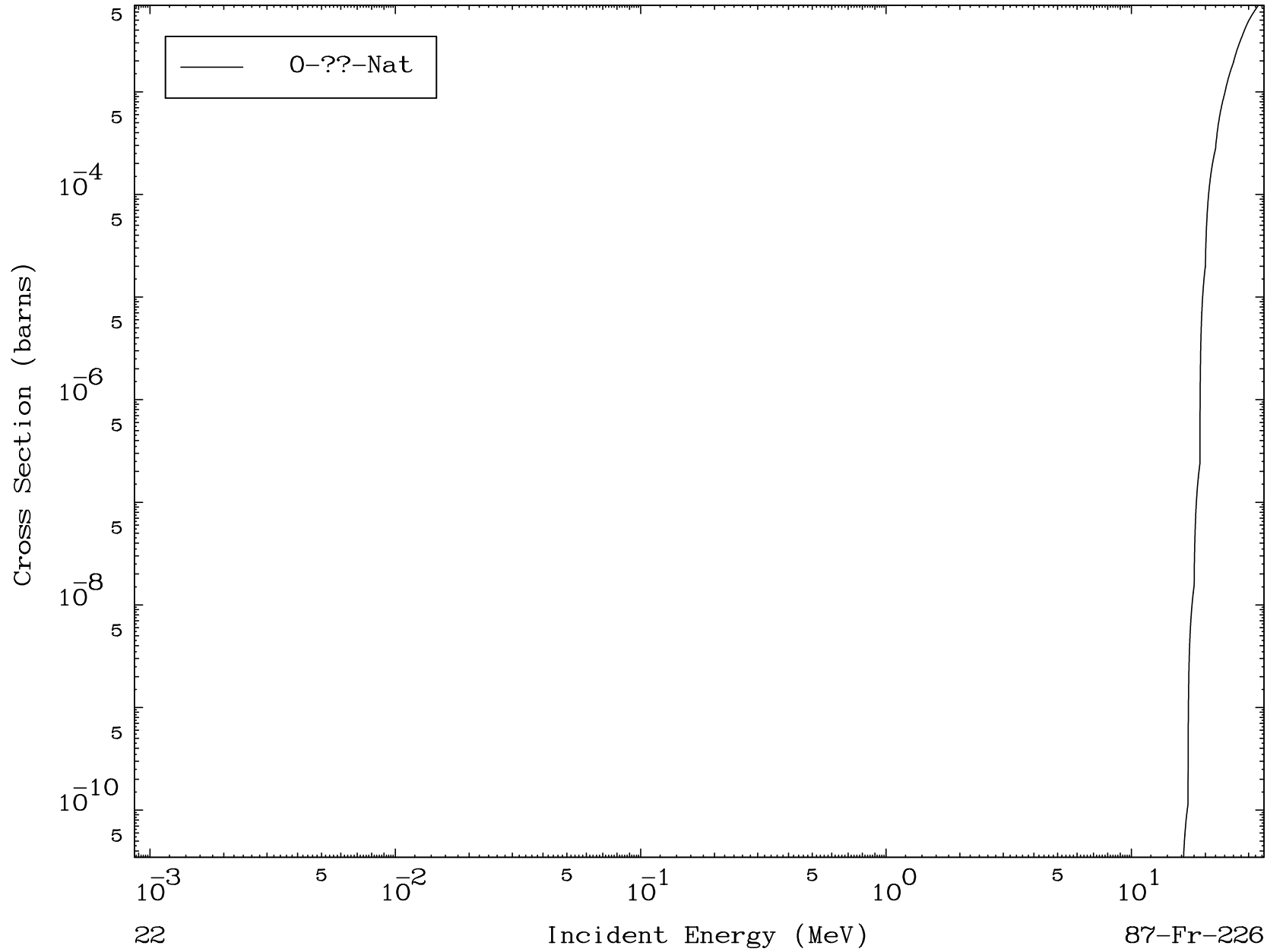
Radionuclide Production Cross Section



MAT 8767

α Fission
Radionuclide Production Cross Section

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Incident Energy (MeV)

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