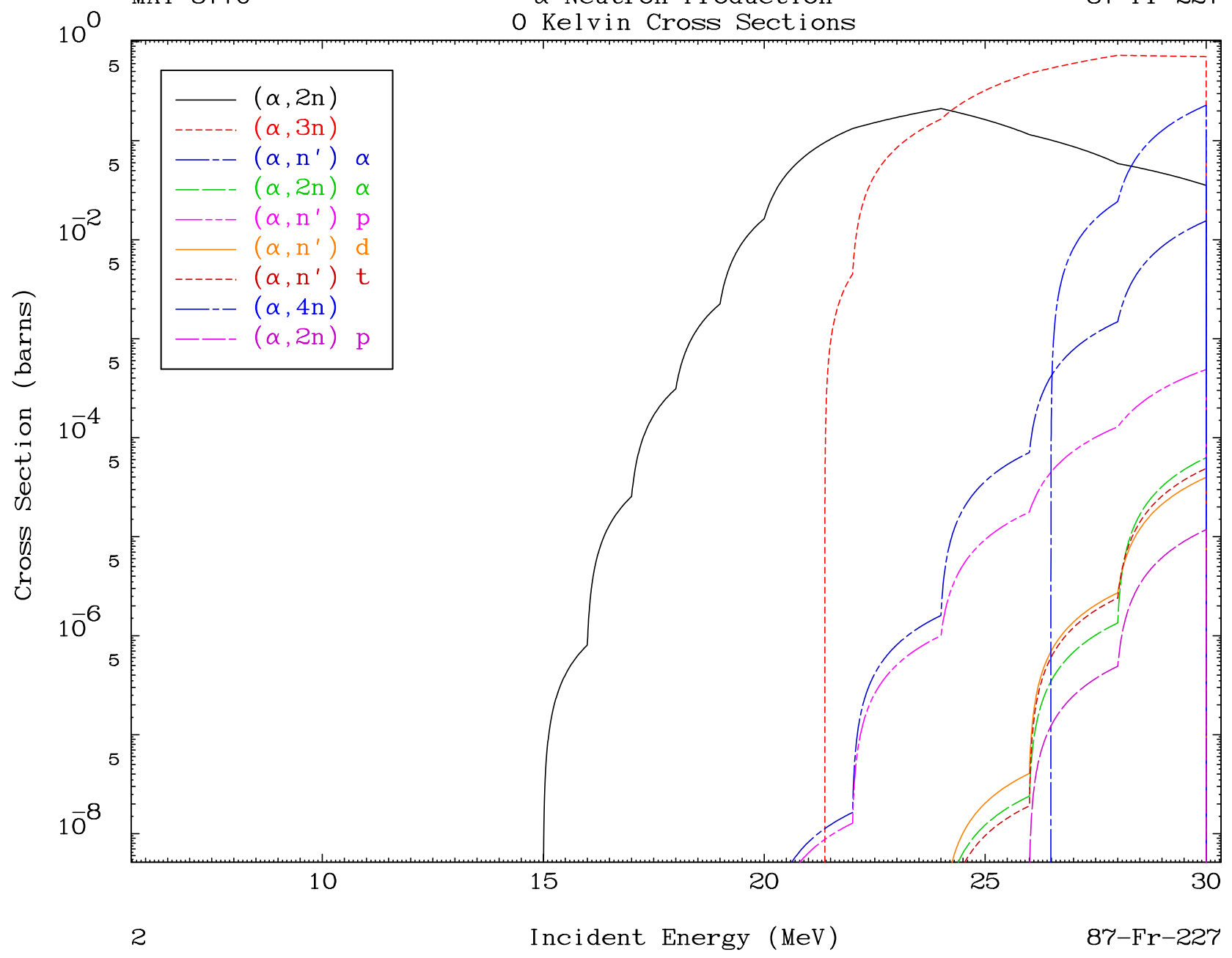
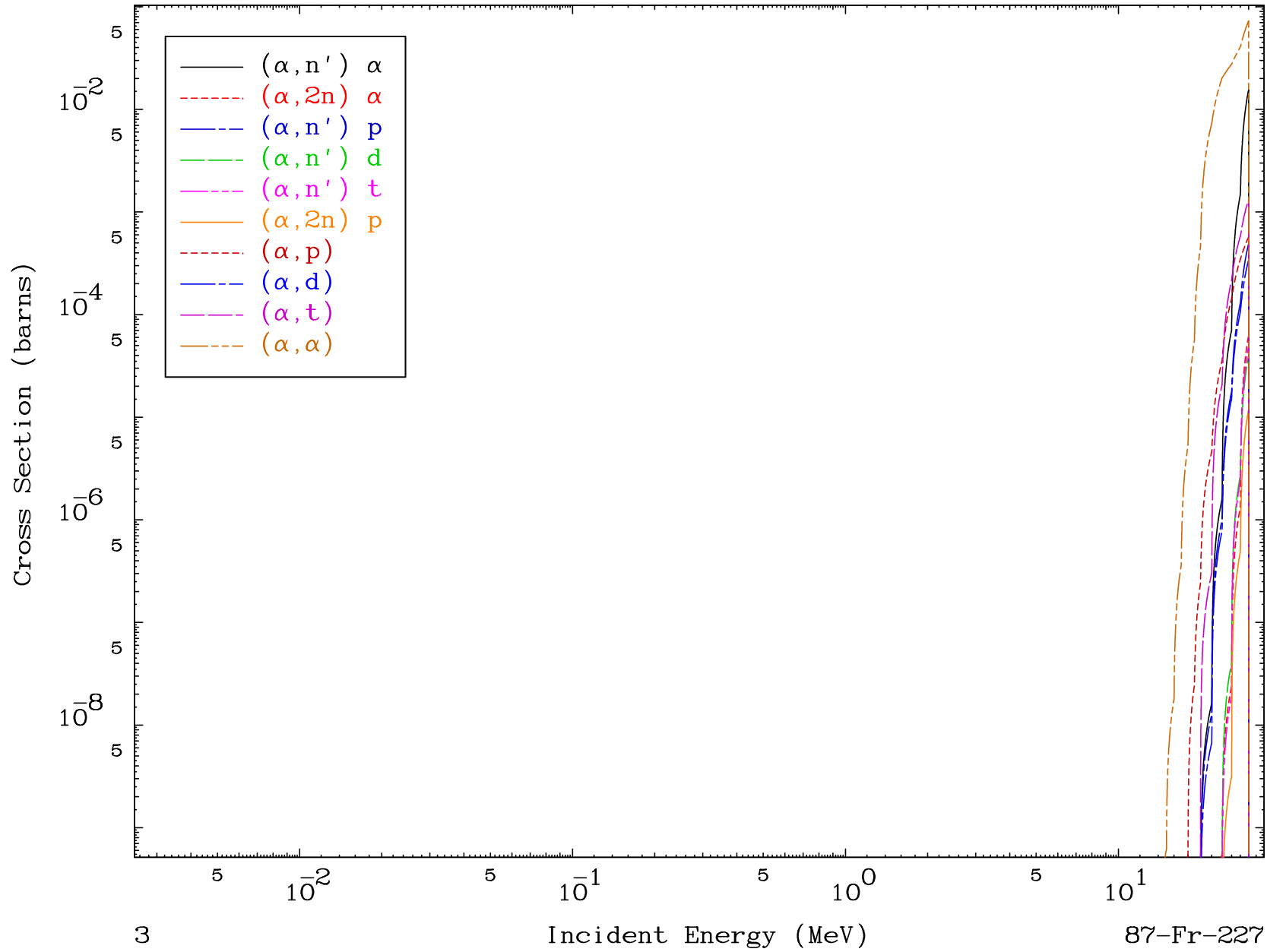


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$\alpha$  Neutron Production  
0 Kelvin Cross Sections

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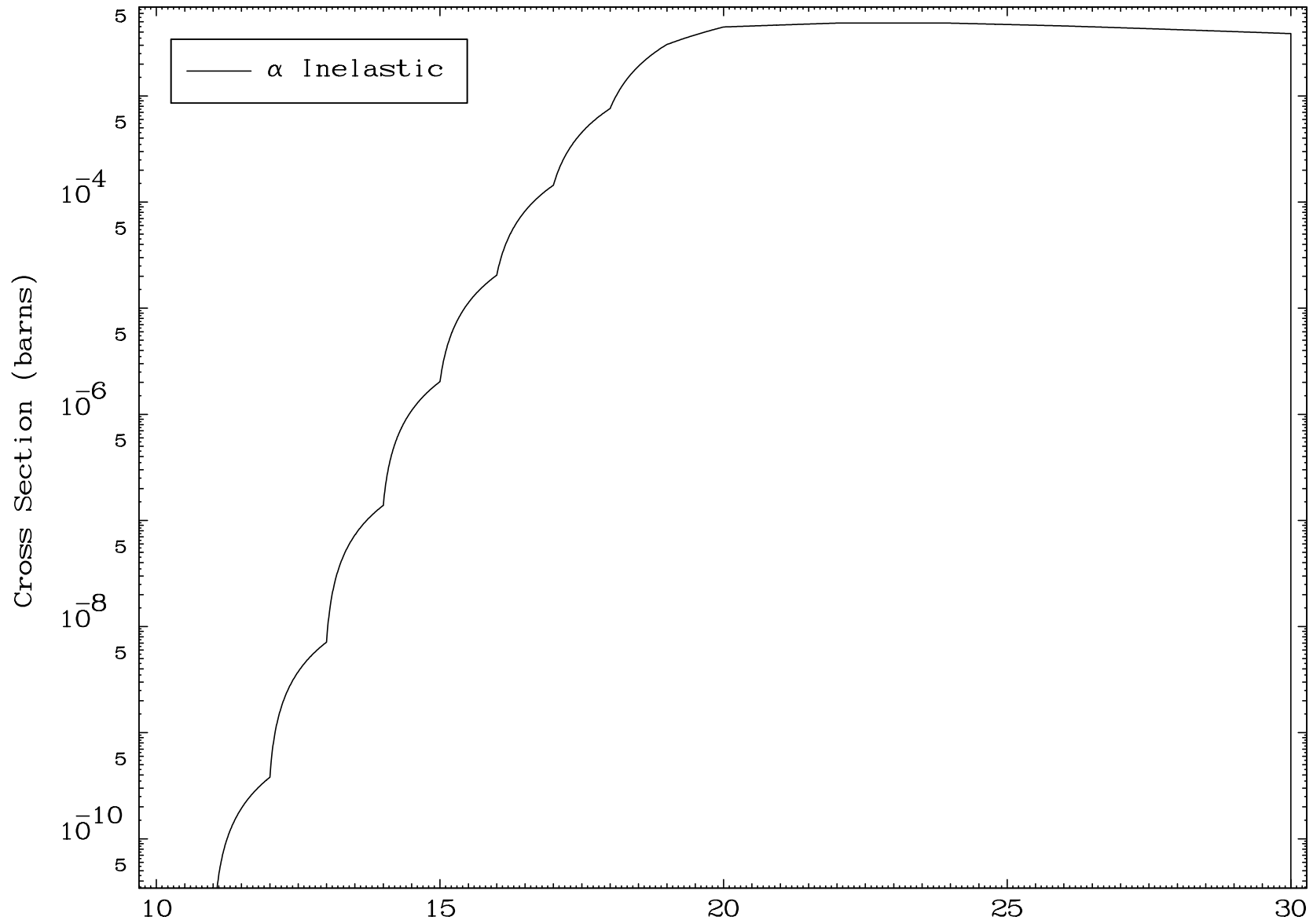




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( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

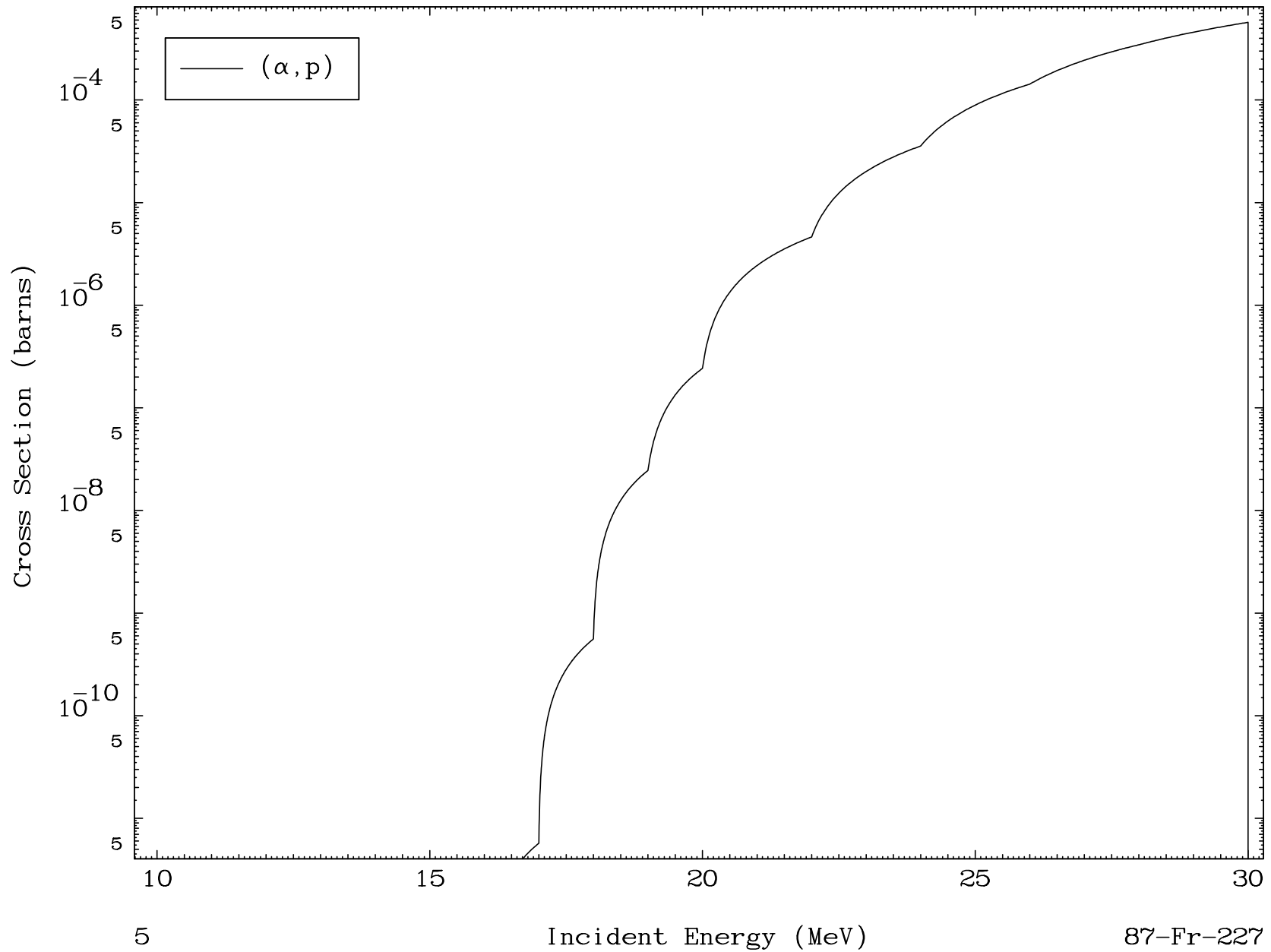
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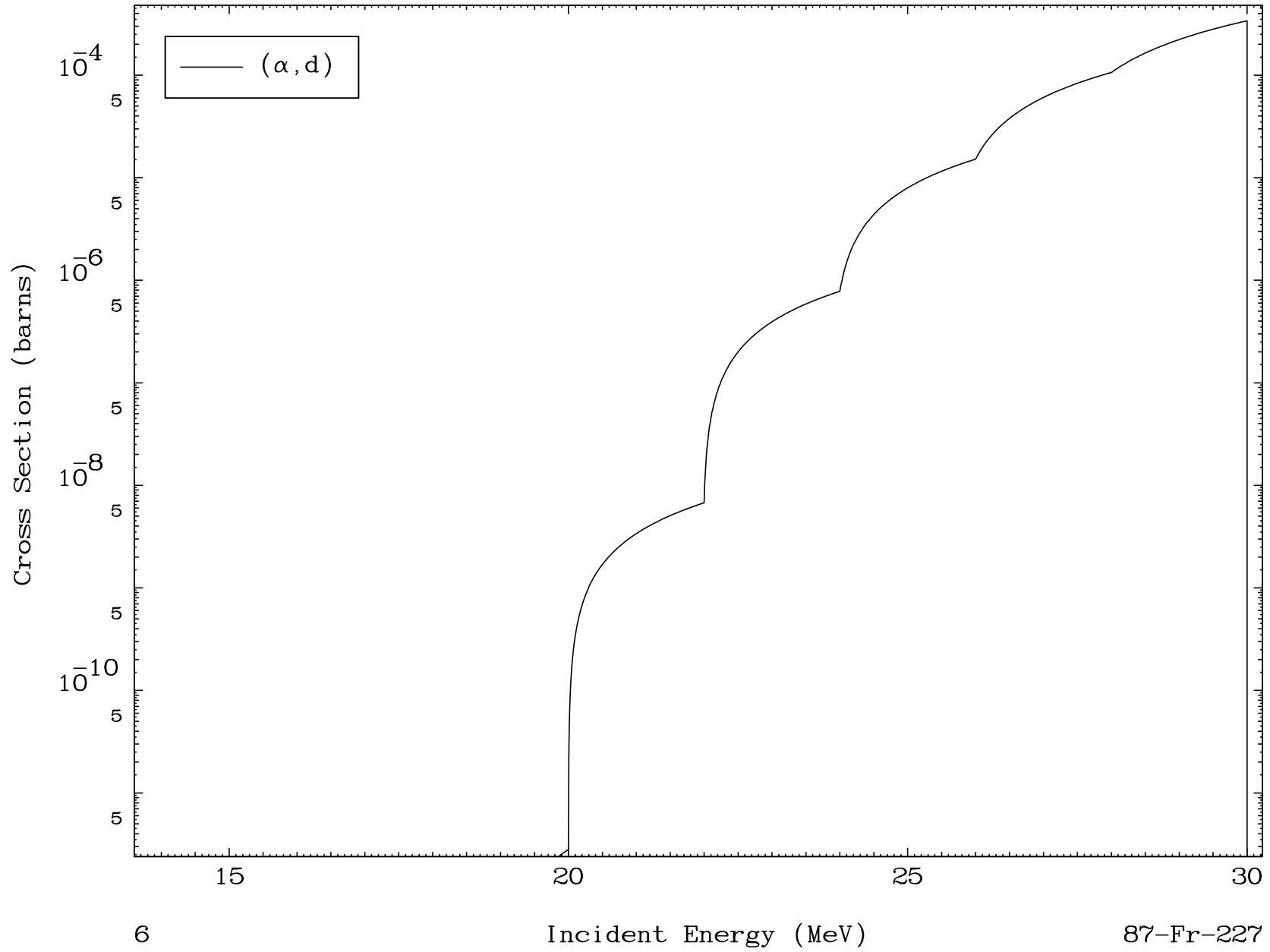


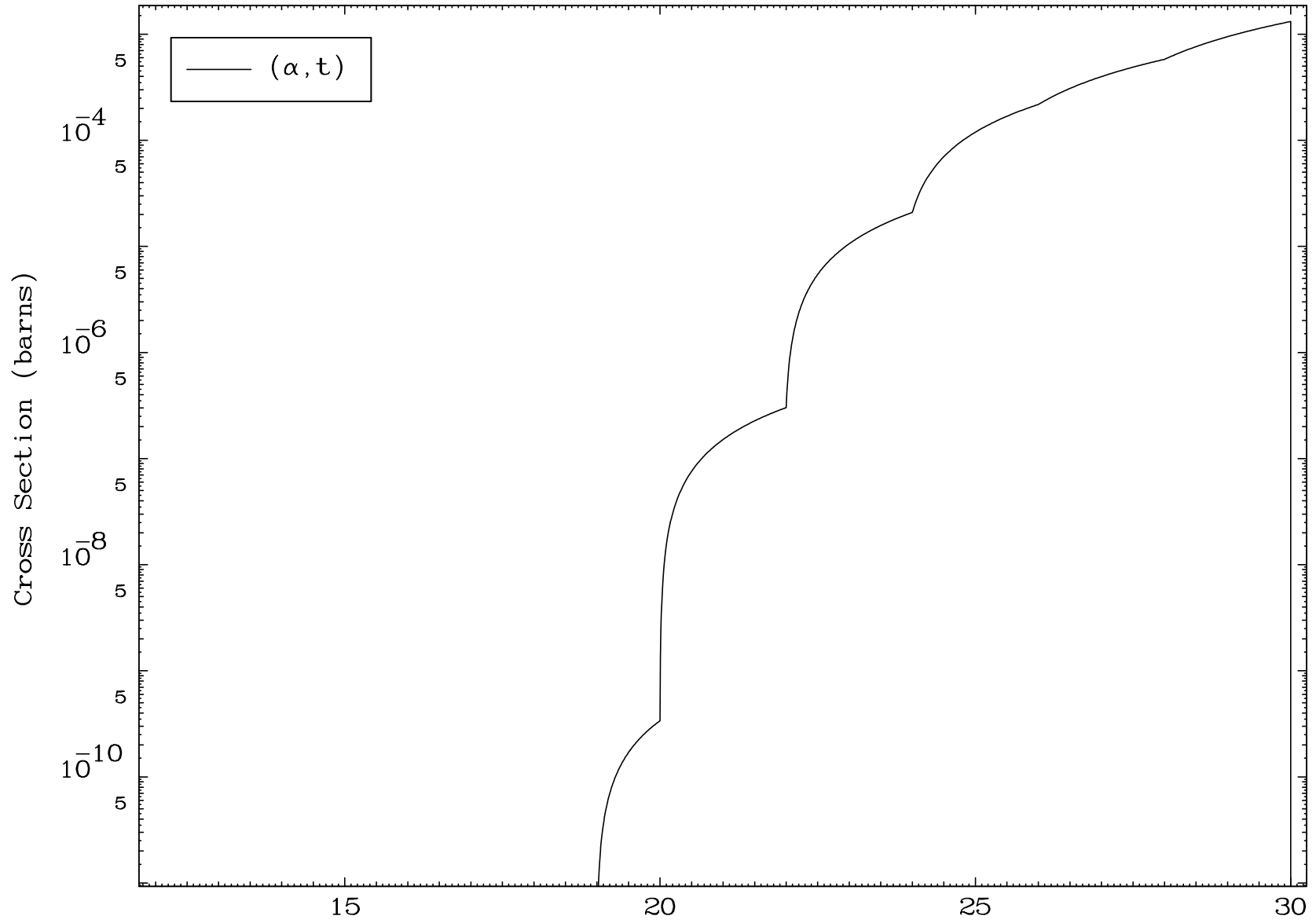
4

Incident Energy (MeV)

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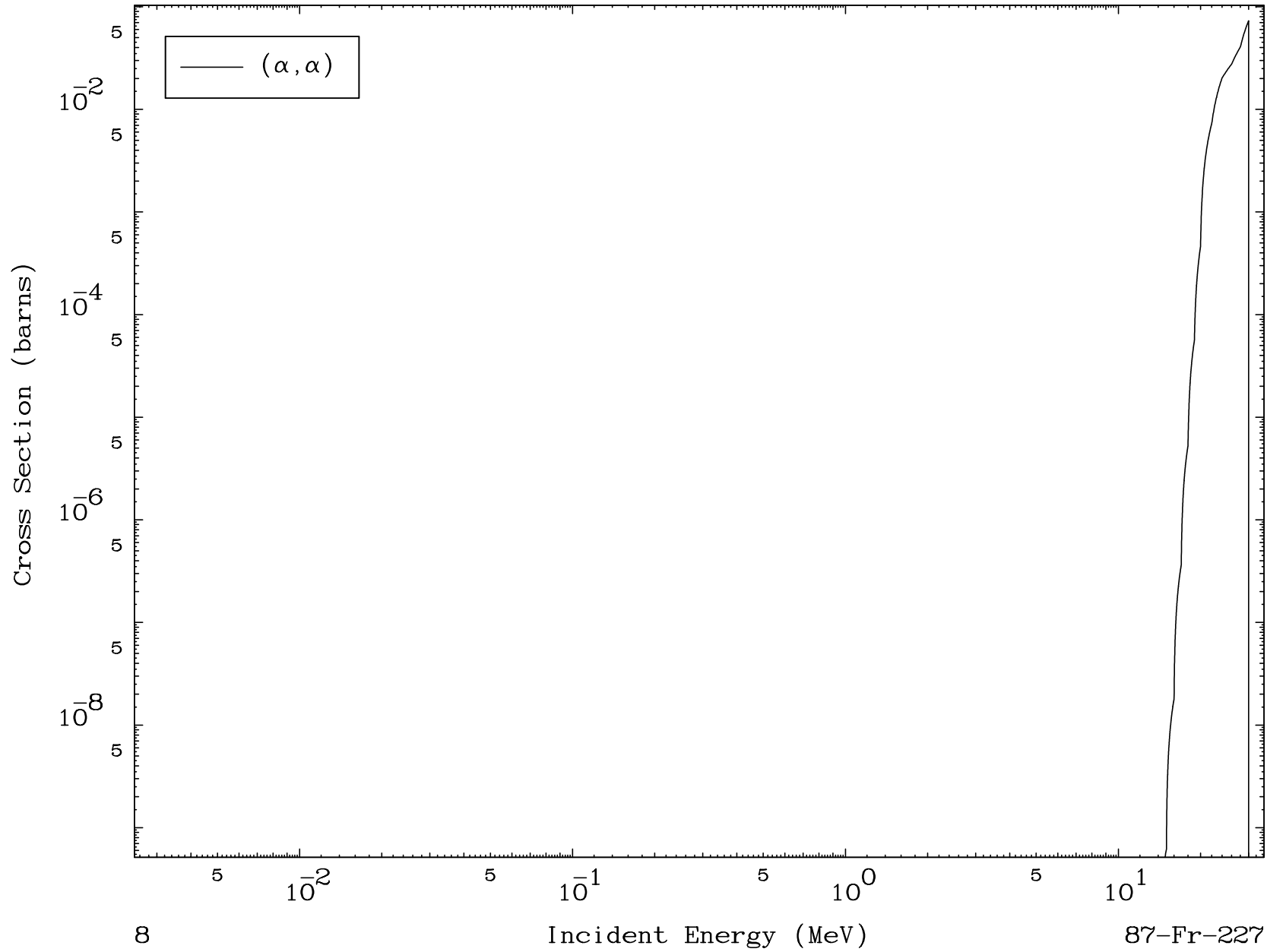




MAT 8770

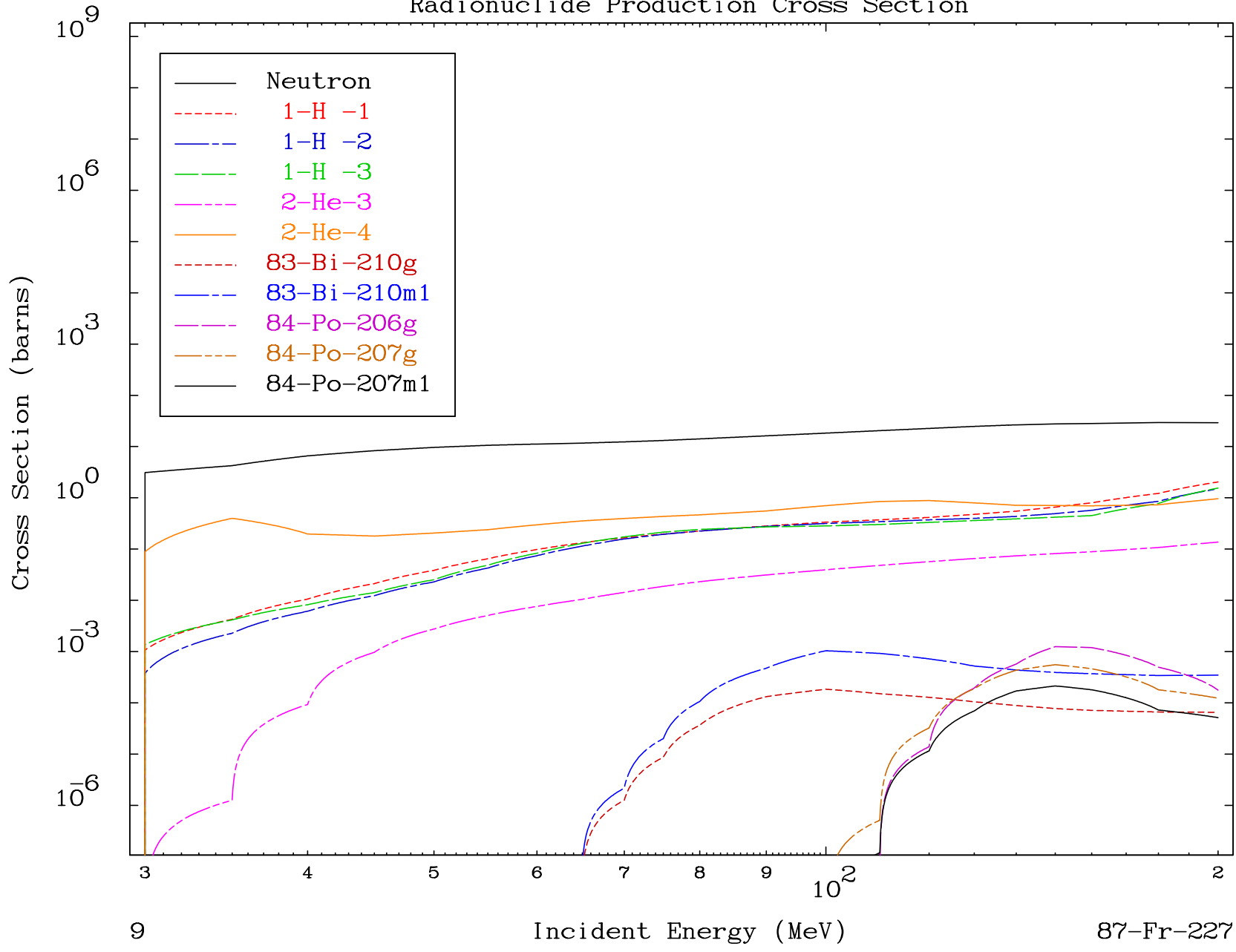
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-227

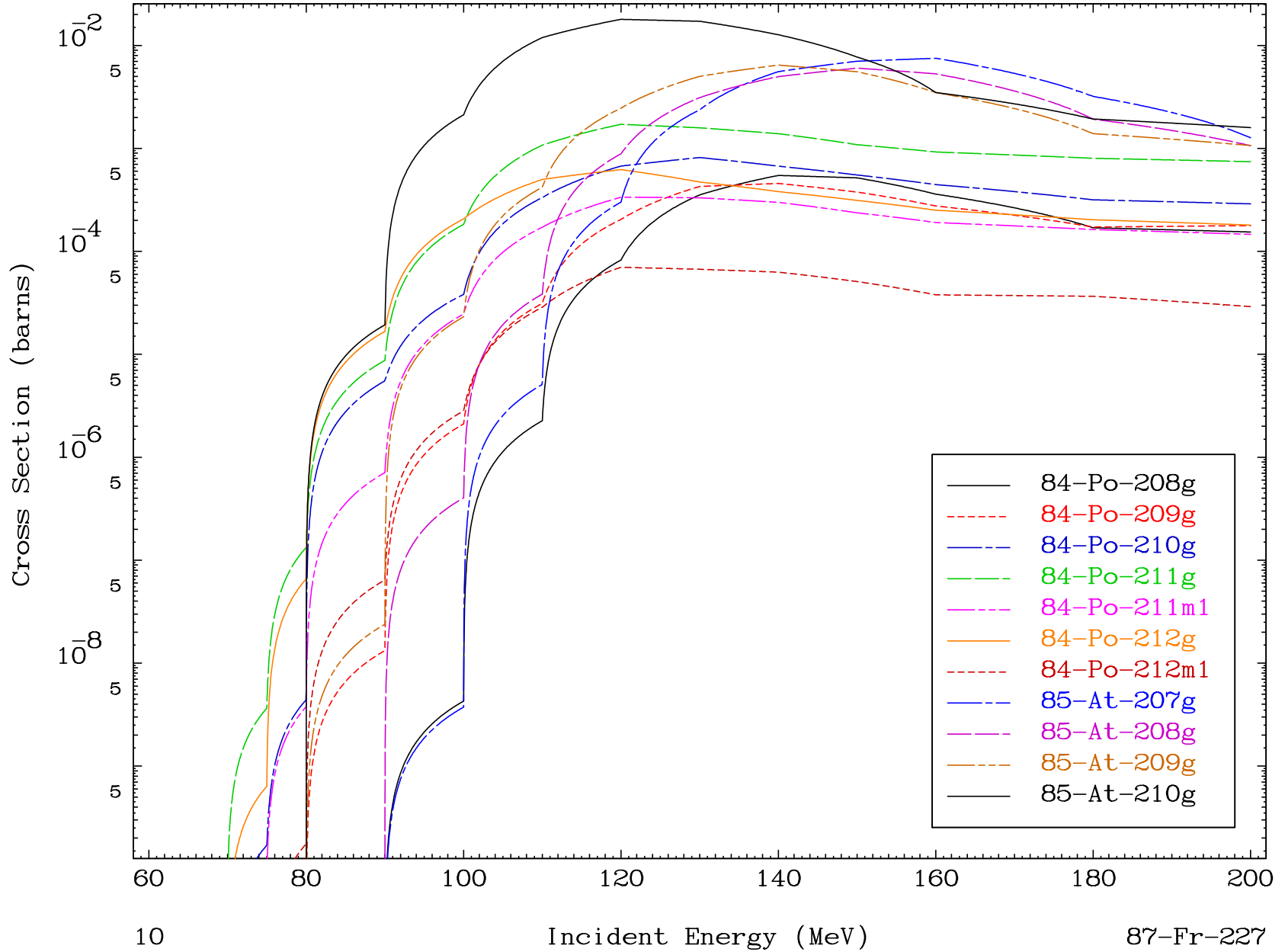


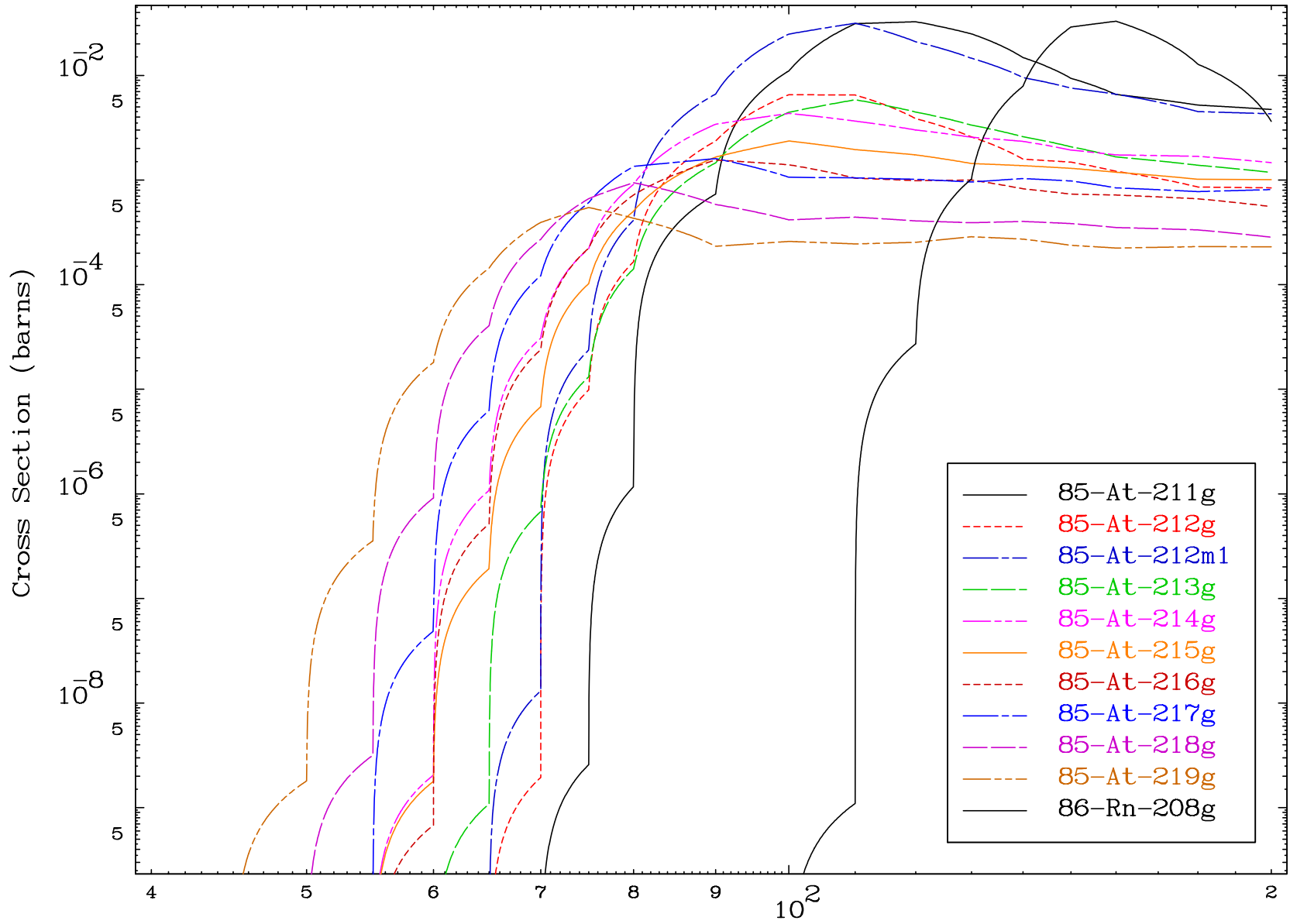


Radionuclide Production Cross Section



Radionuclide Production Cross Section



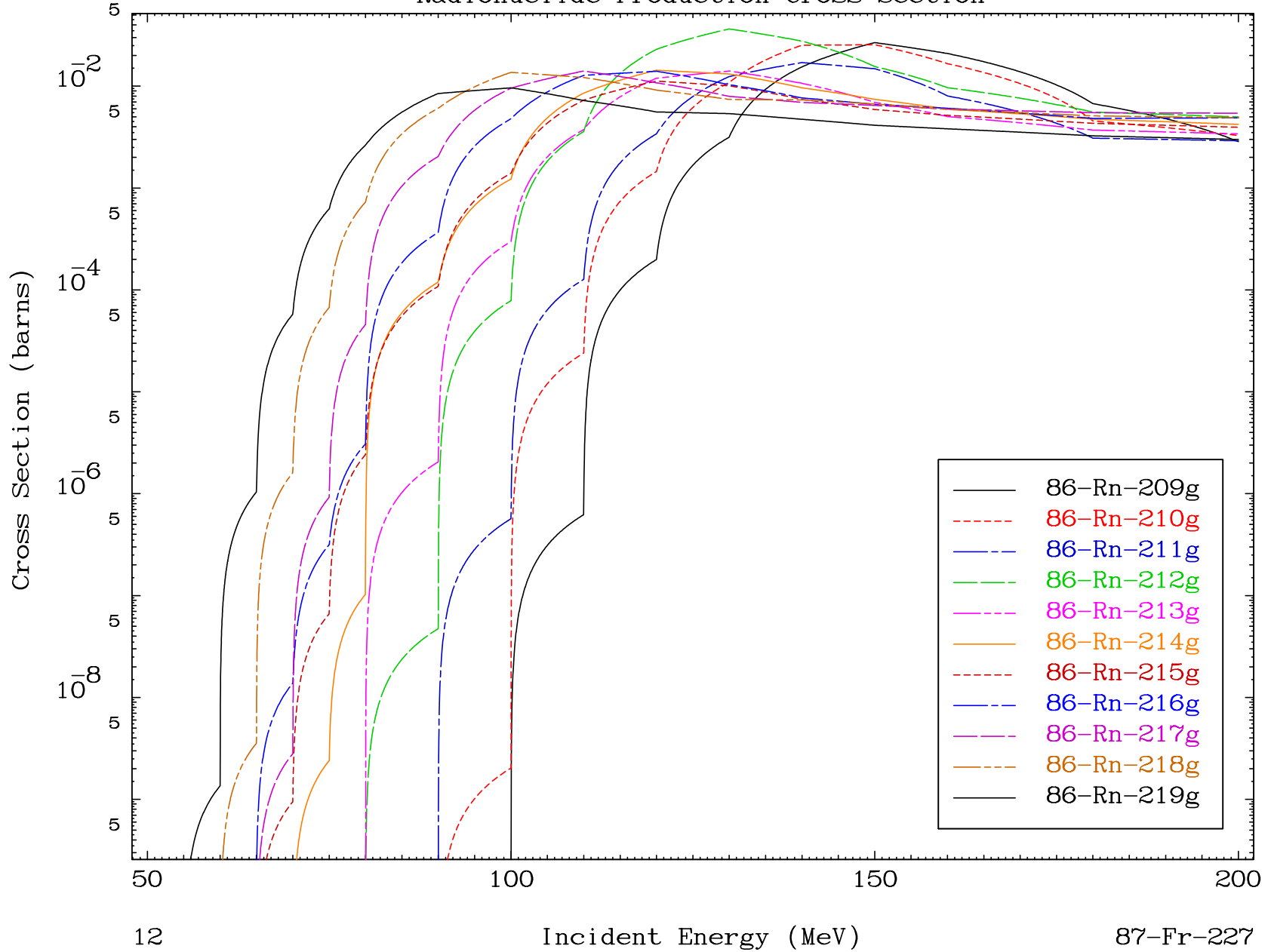


MAT 8770

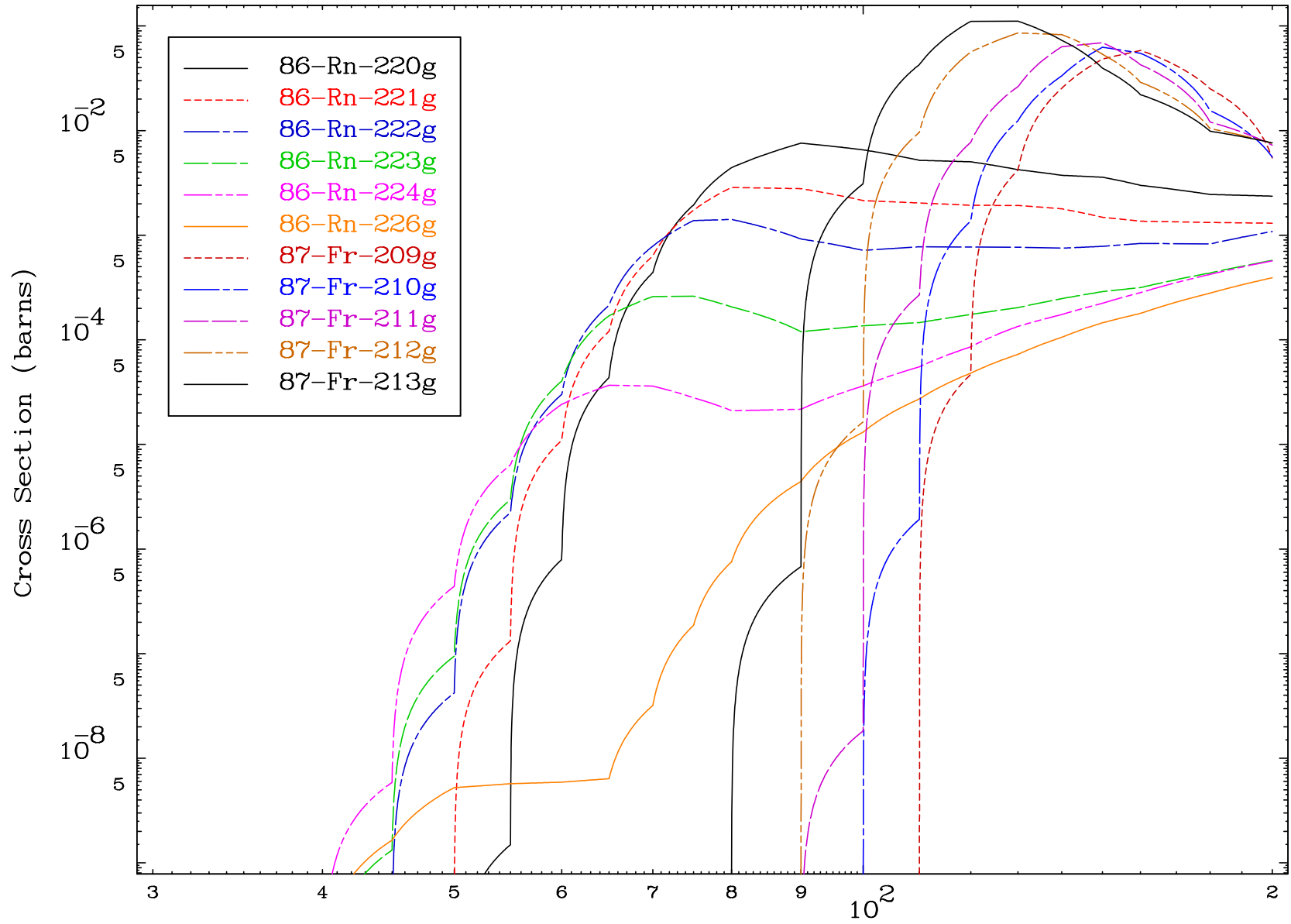
( $\alpha$ , remainder)

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### Radionuclide Production Cross Section



Radionuclide Production Cross Section

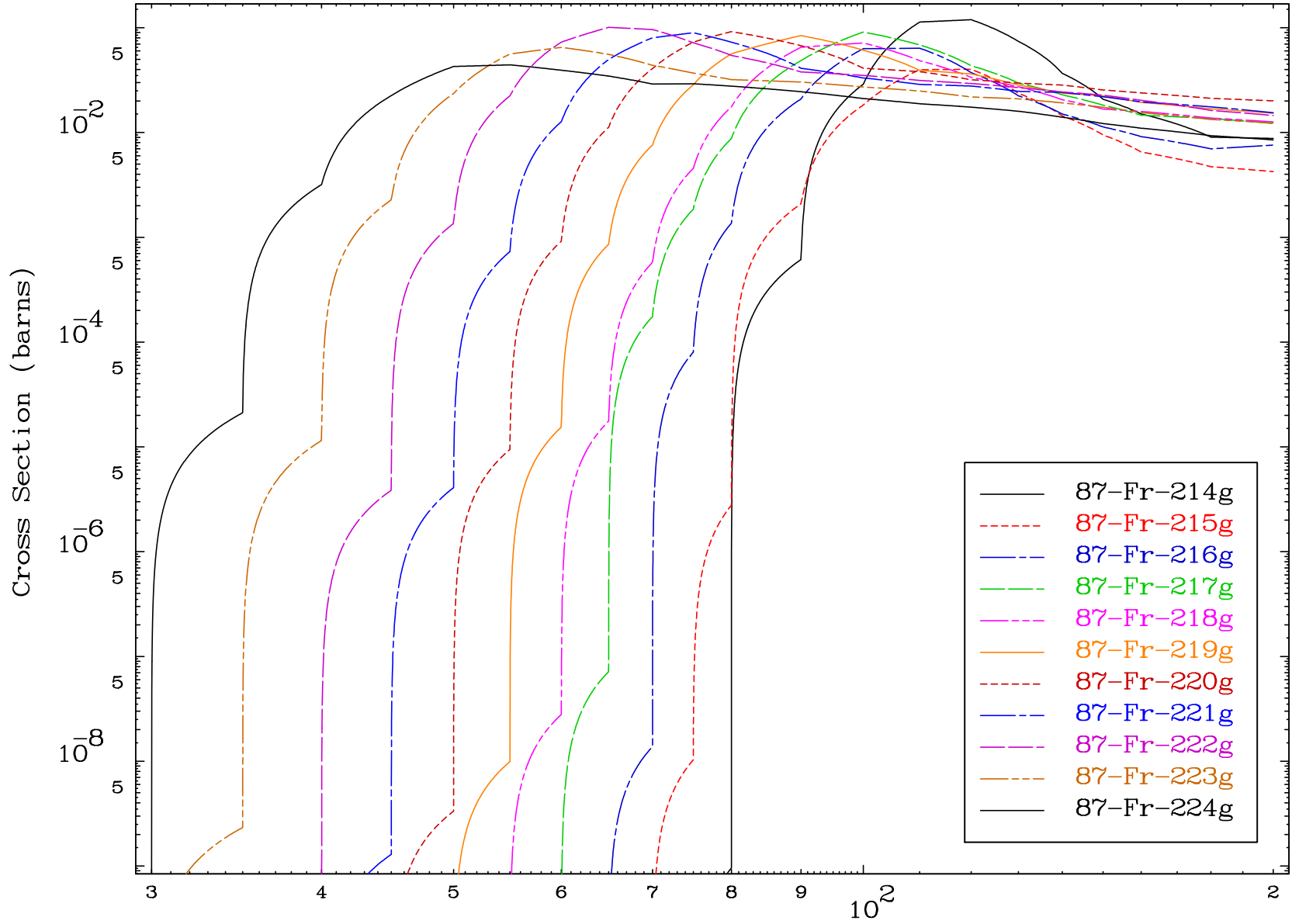


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( $\alpha$ , remainder)

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### Radionuclide Production Cross Section



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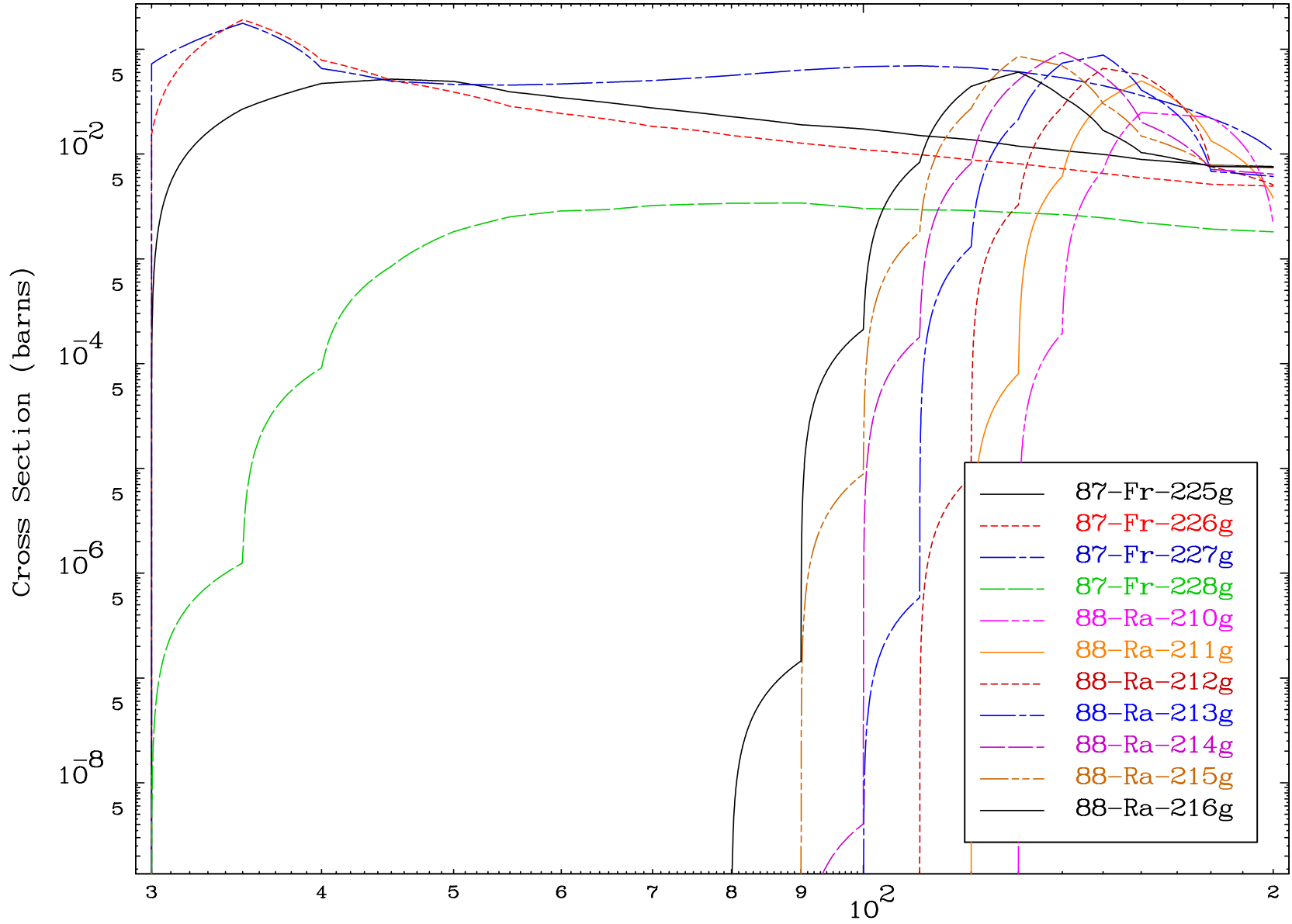
Incident Energy (MeV)

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( $\alpha$ , remainder)  
Radionuclide Production Cross Section

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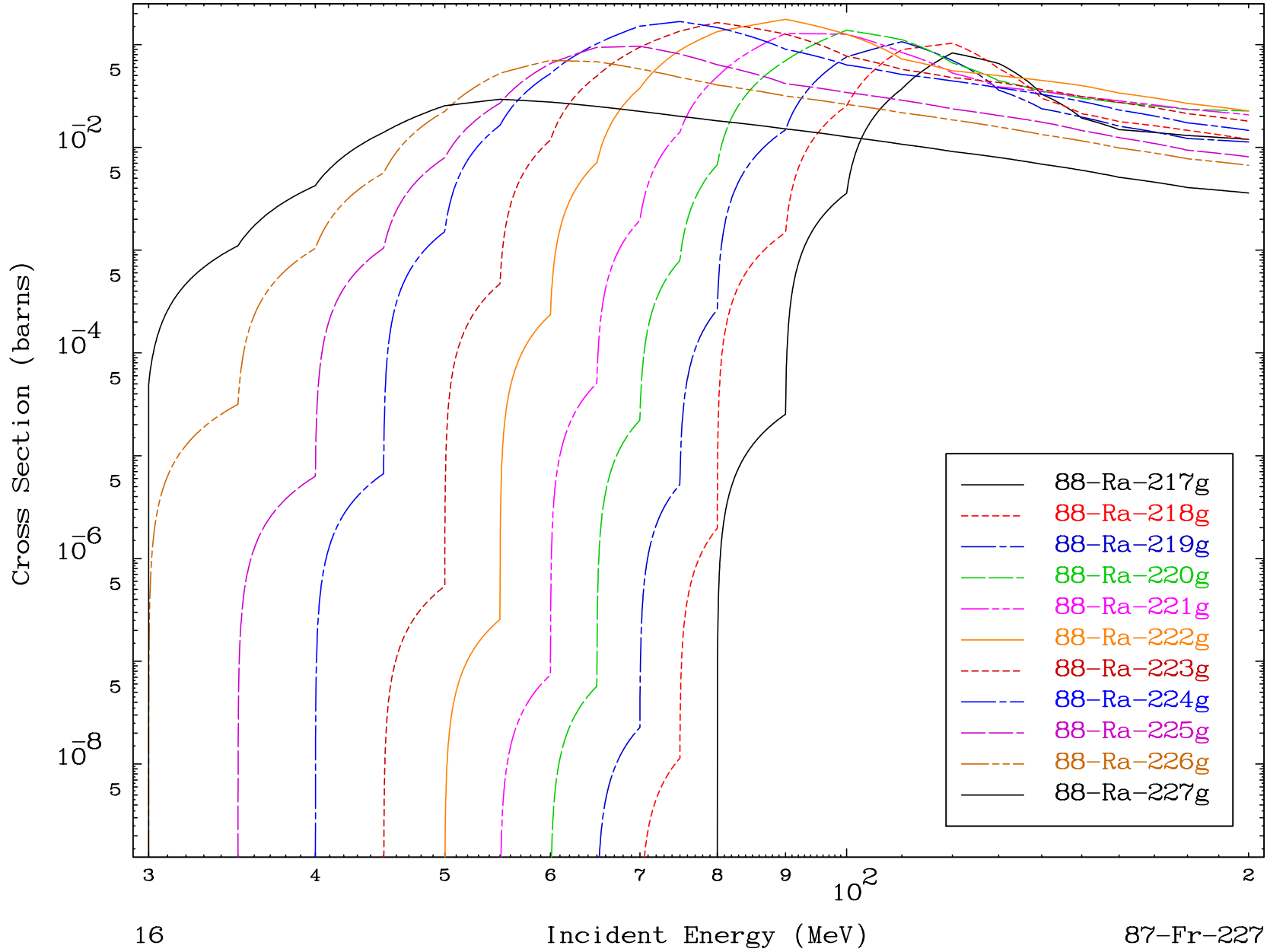
Incident Energy (MeV)

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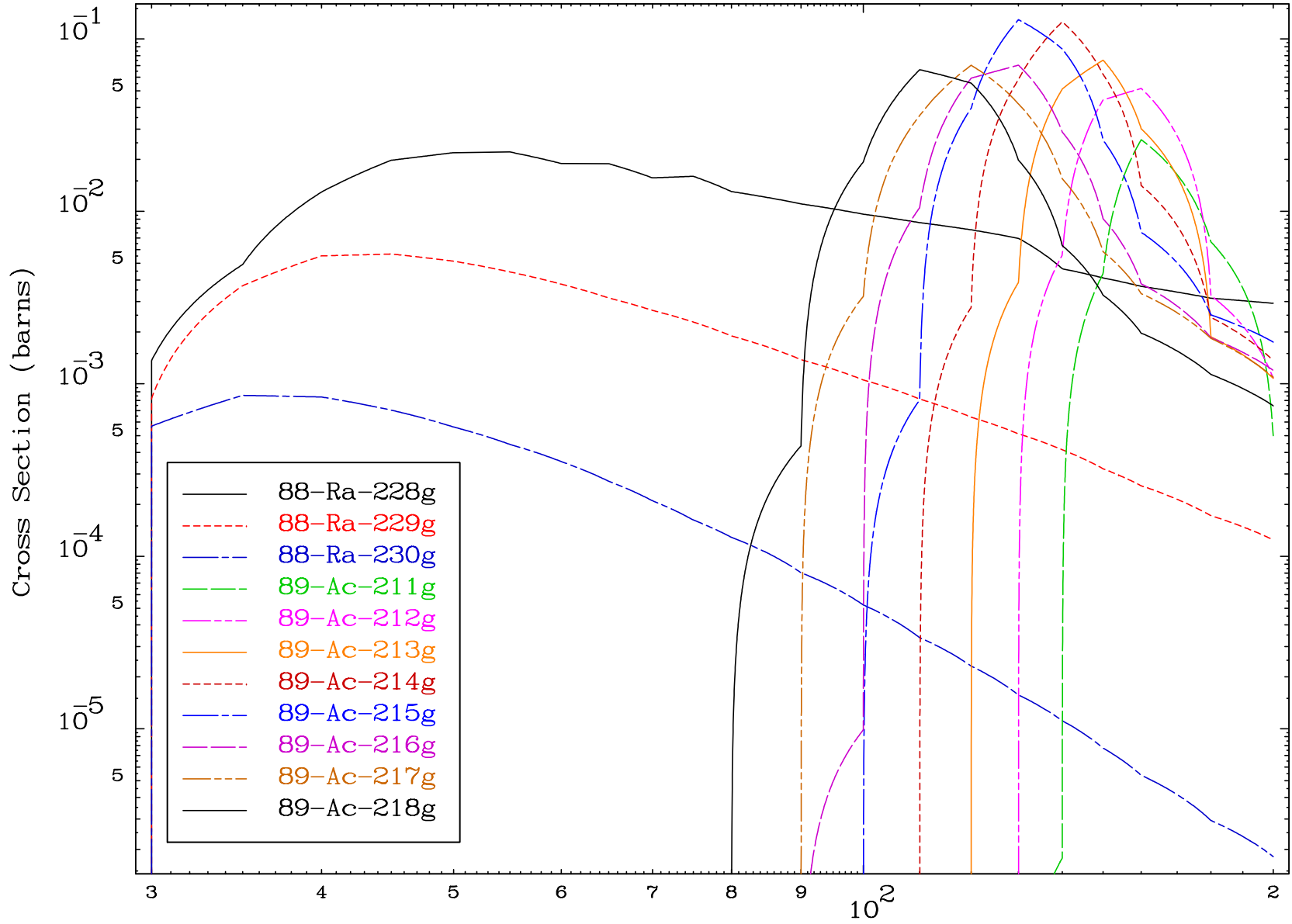
MAT 8770

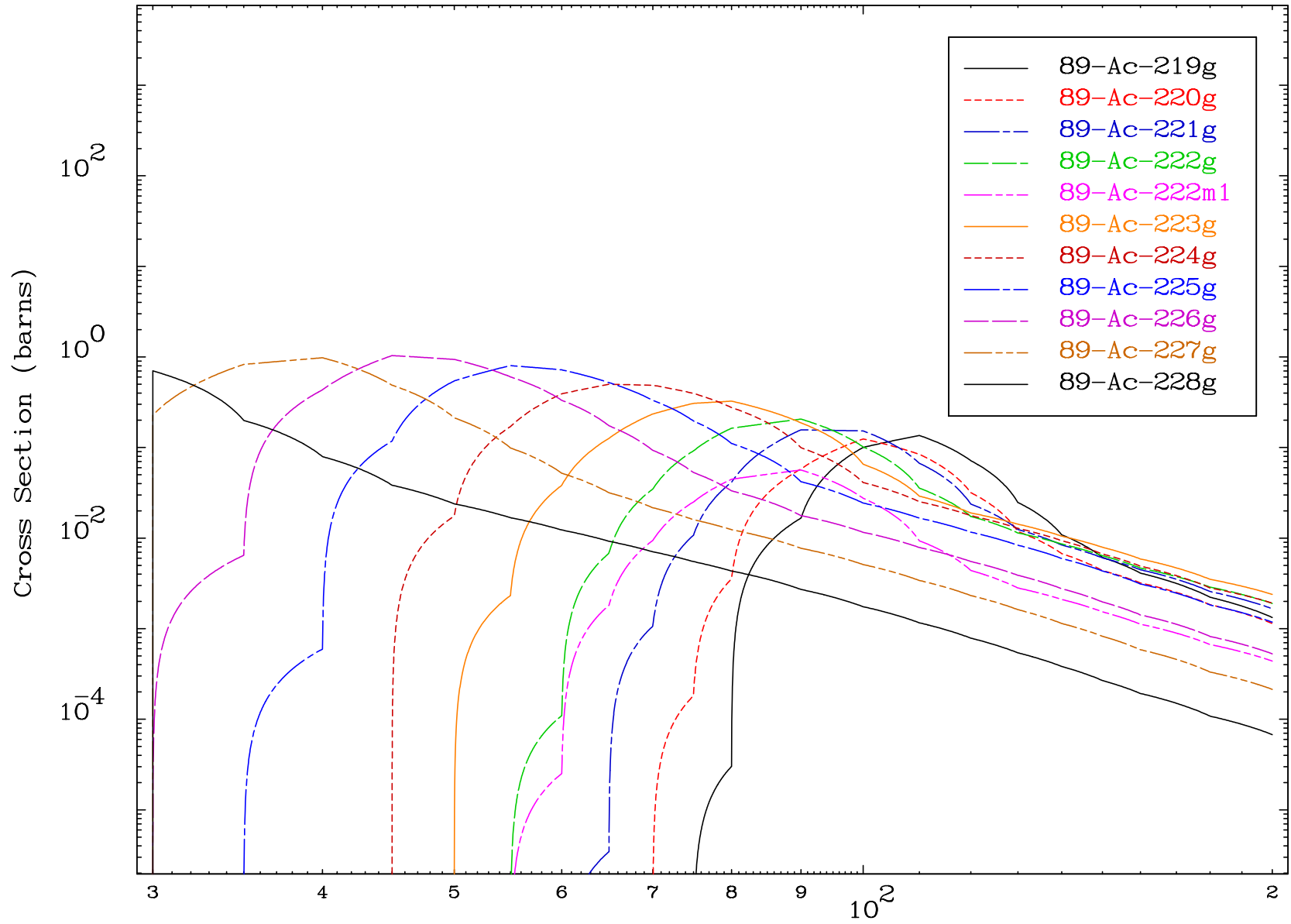
( $\alpha$ , remainder)  
Radionuclide Production Cross Section

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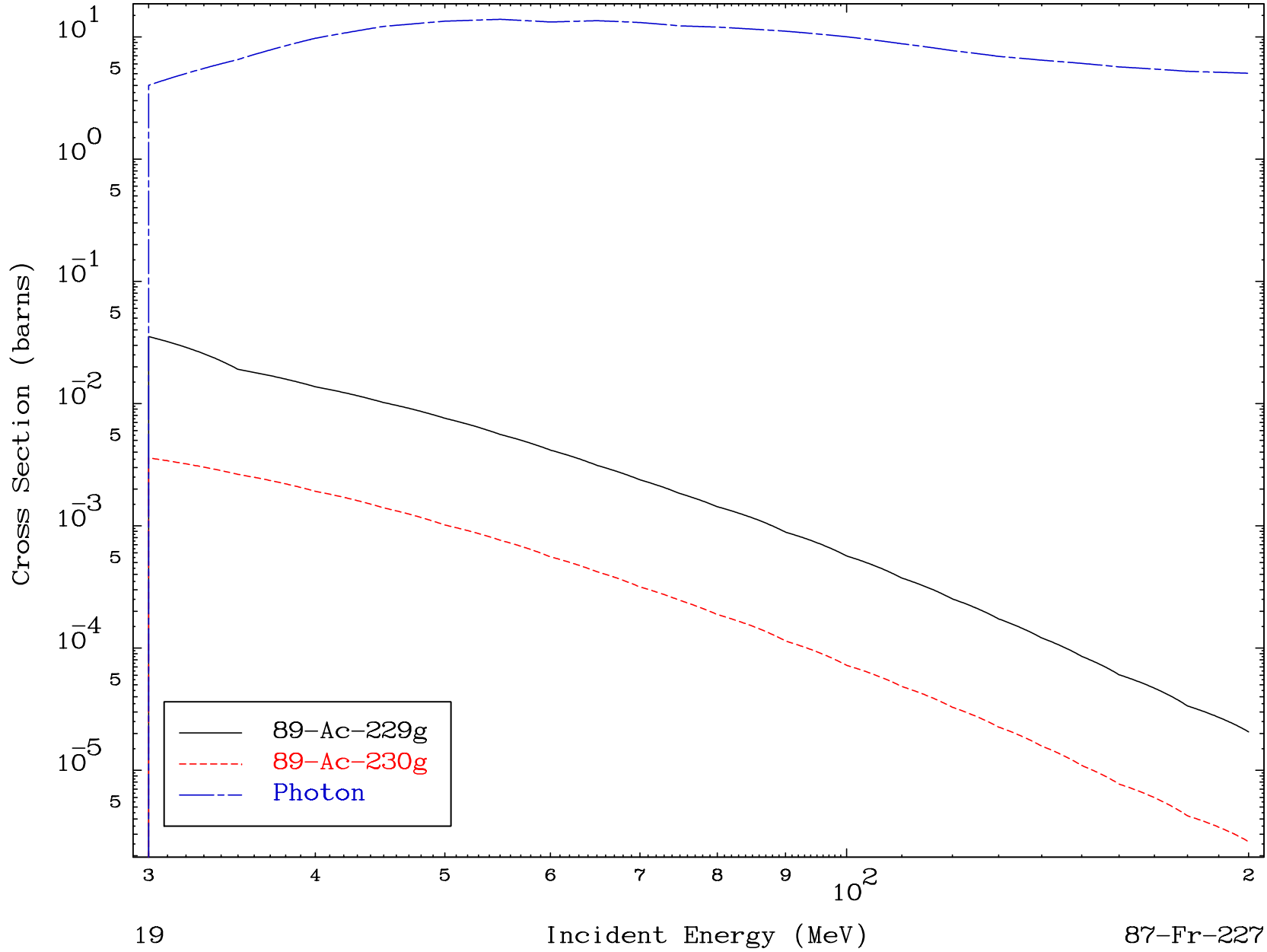


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( $\alpha$ , remainder)

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### Radionuclide Production Cross Section



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$\alpha$  Fission  
Radionuclide Production Cross Section

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