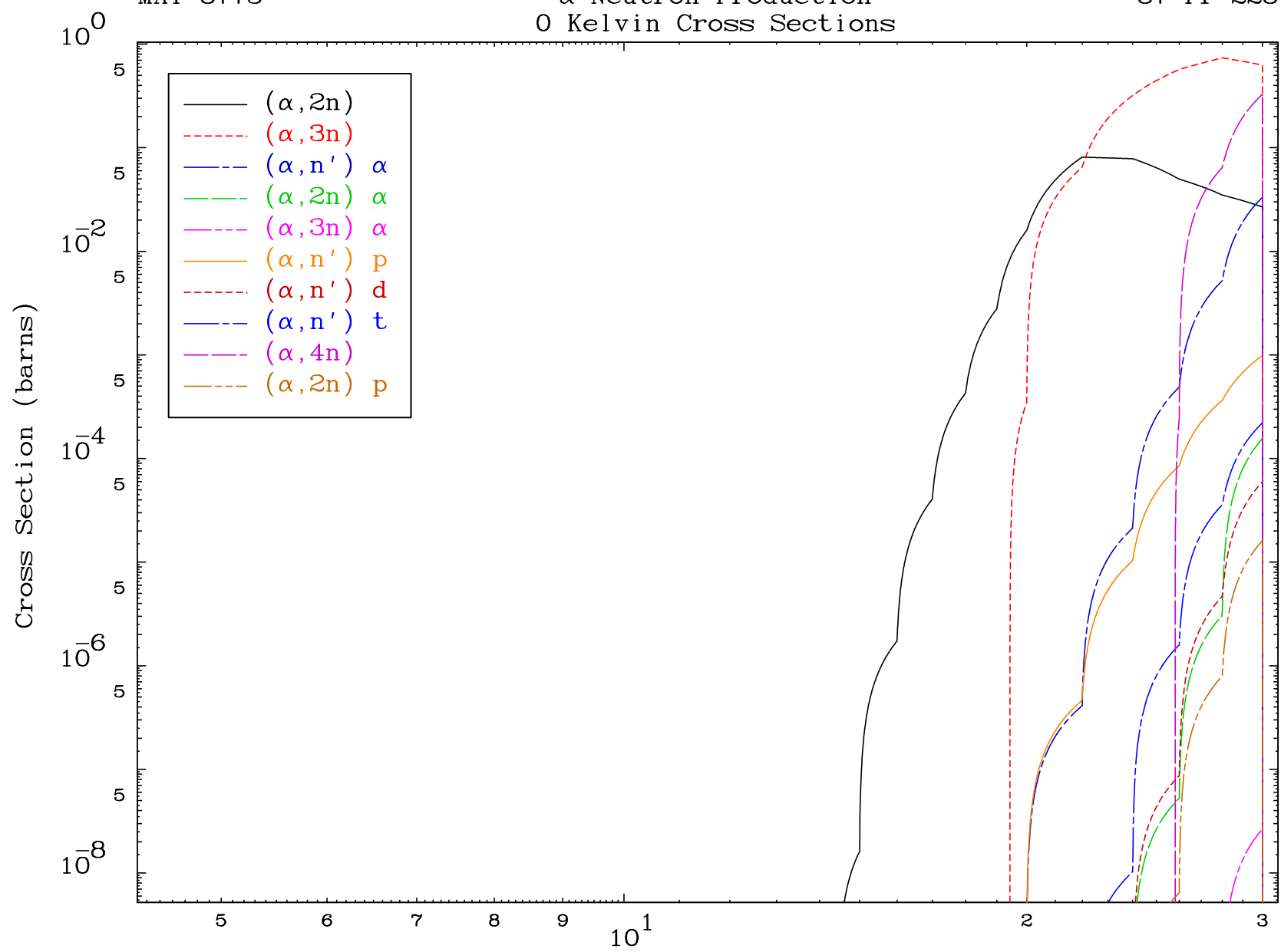


MAT 8773

α Neutron Production
0 Kelvin Cross Sections

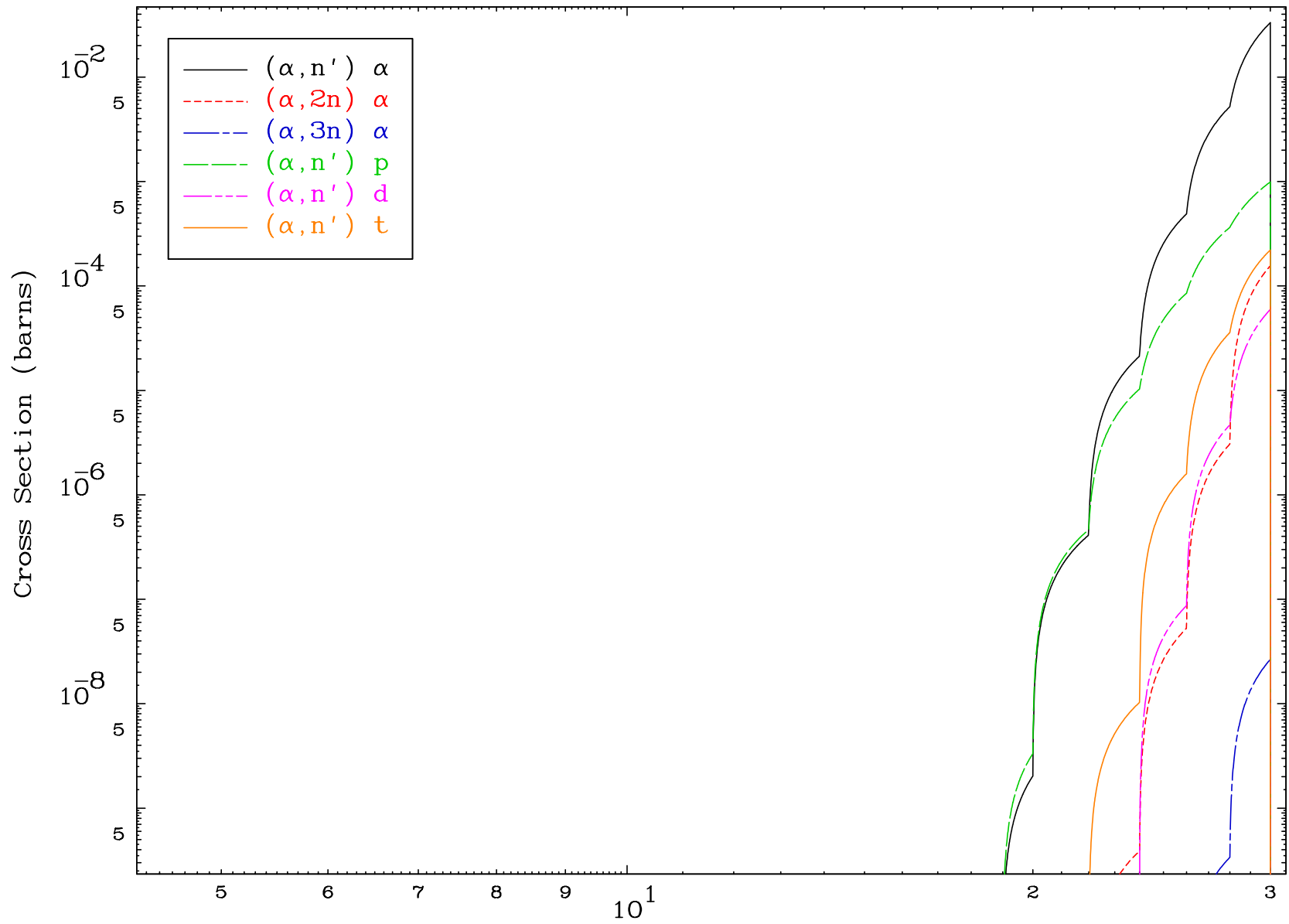
87-Fr-228

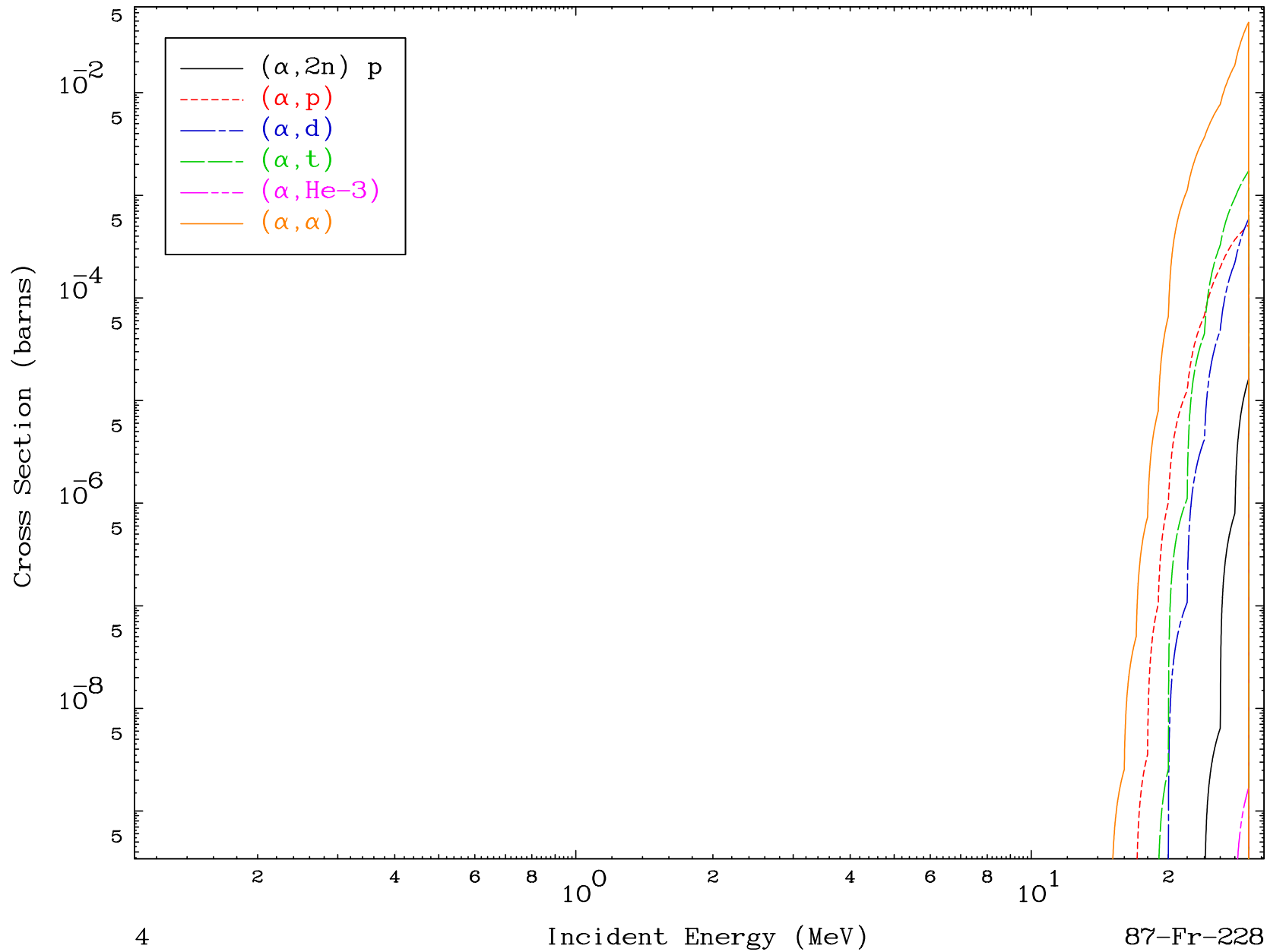


2

Incident Energy (MeV)

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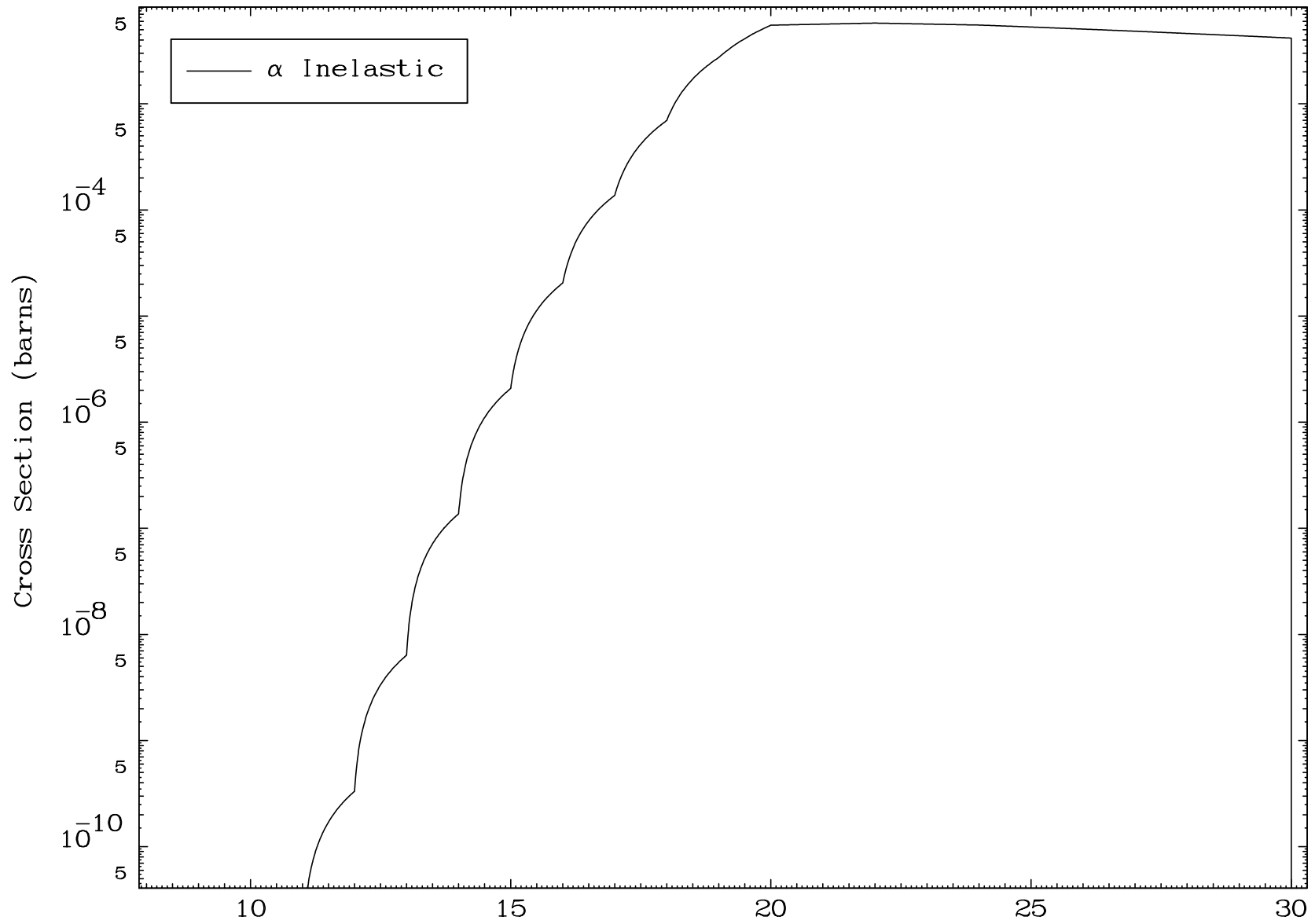




MAT 8773

(α, n') Level
0 Kelvin Cross Sections

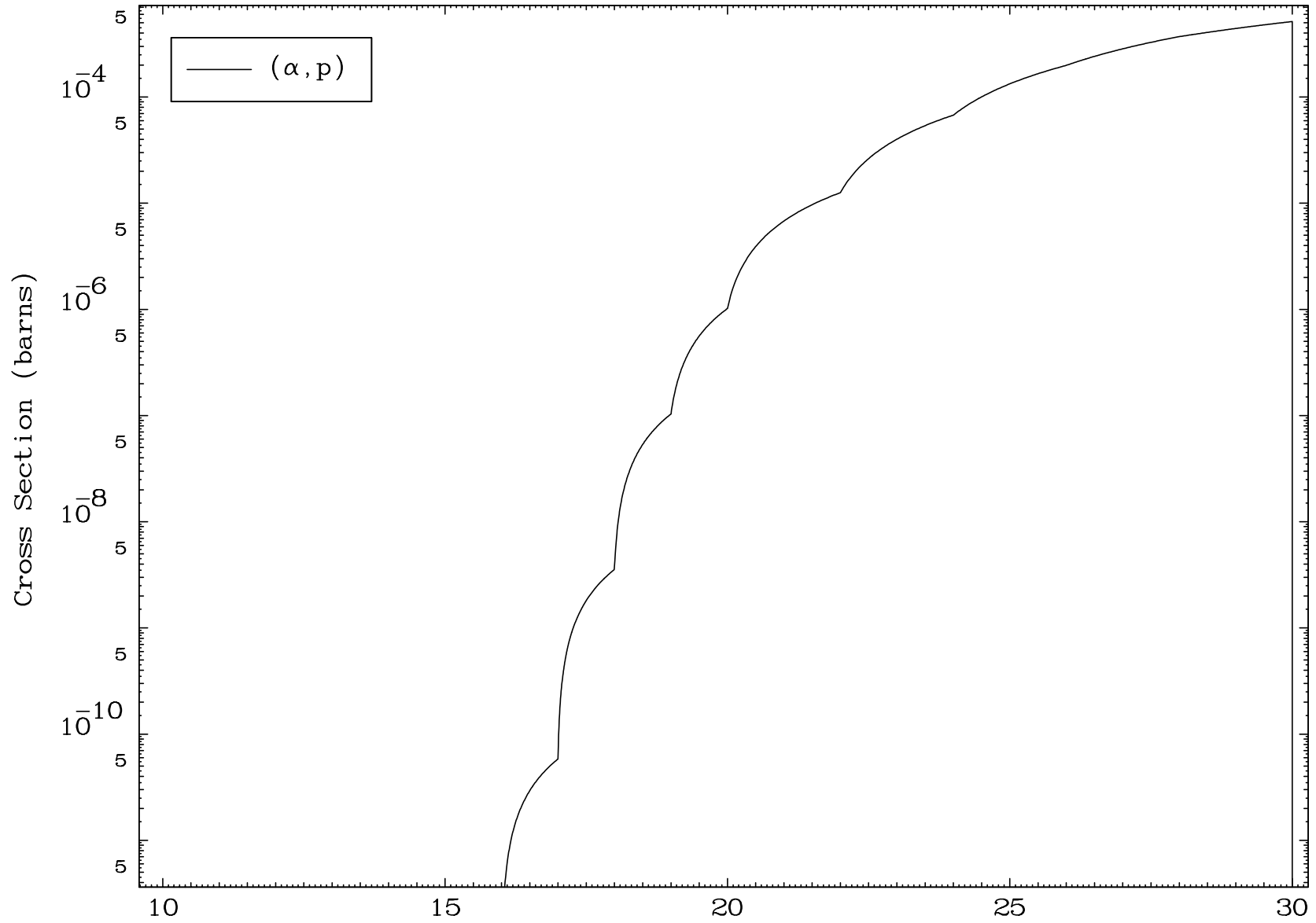
87-Fr-228

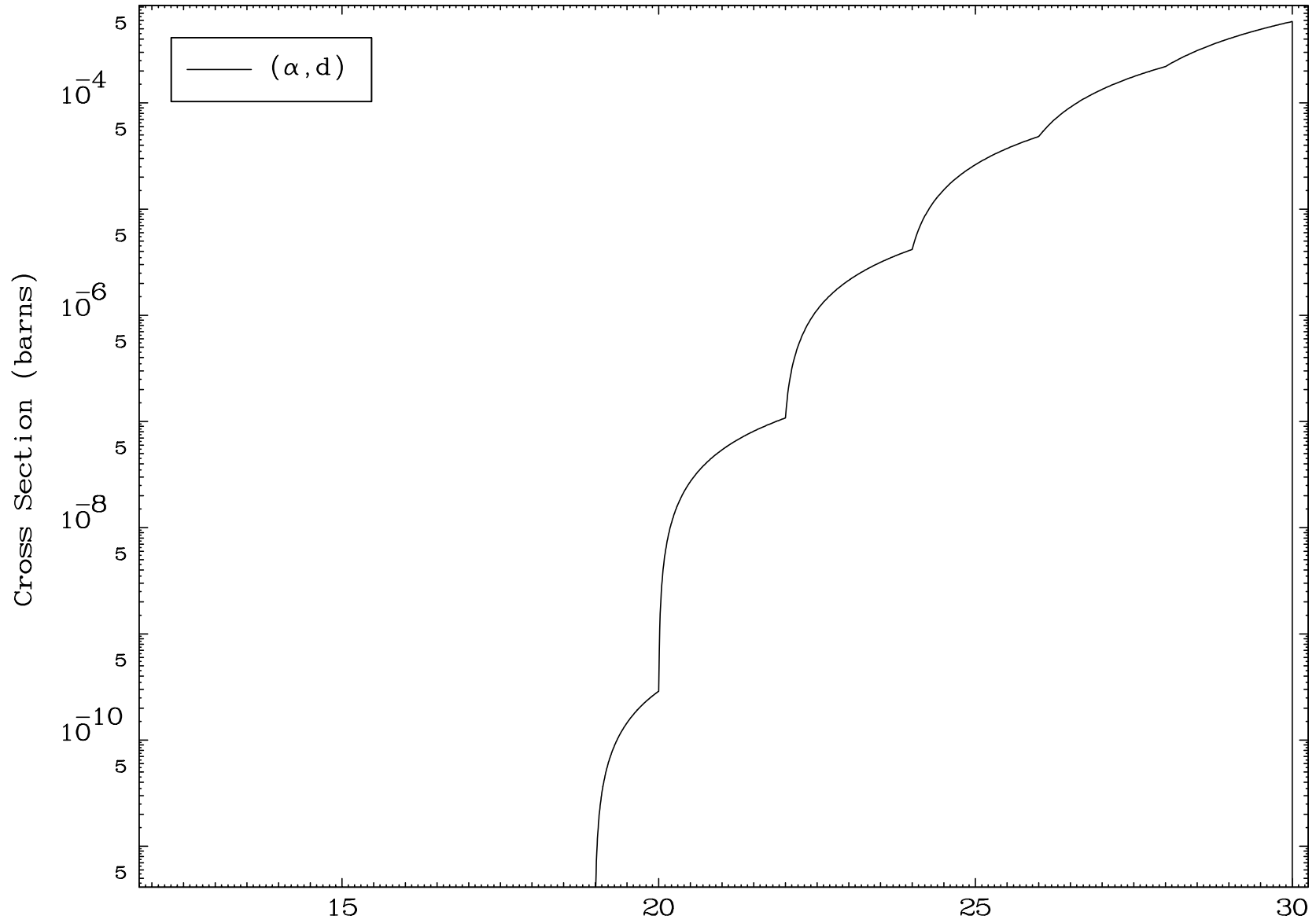


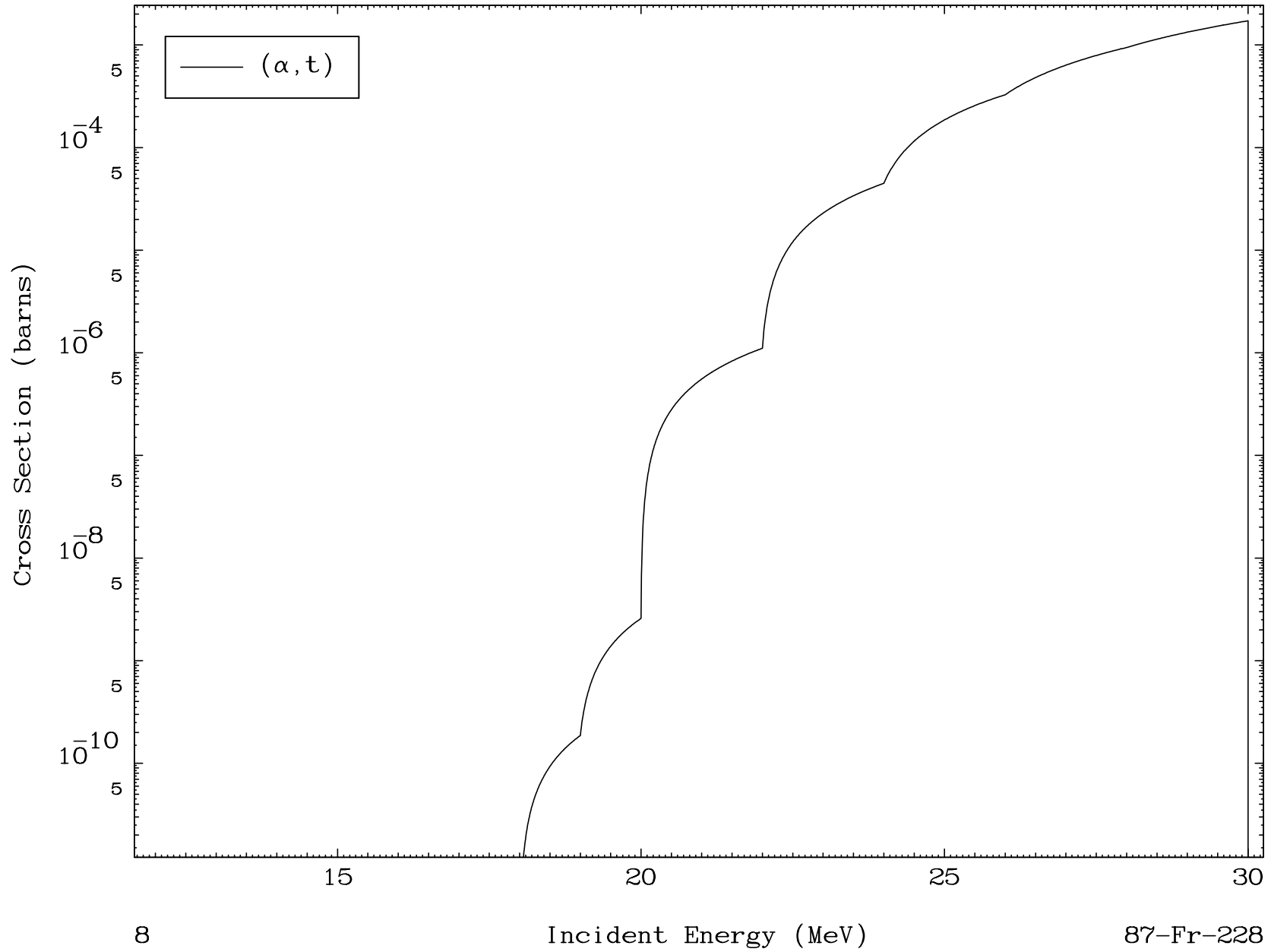
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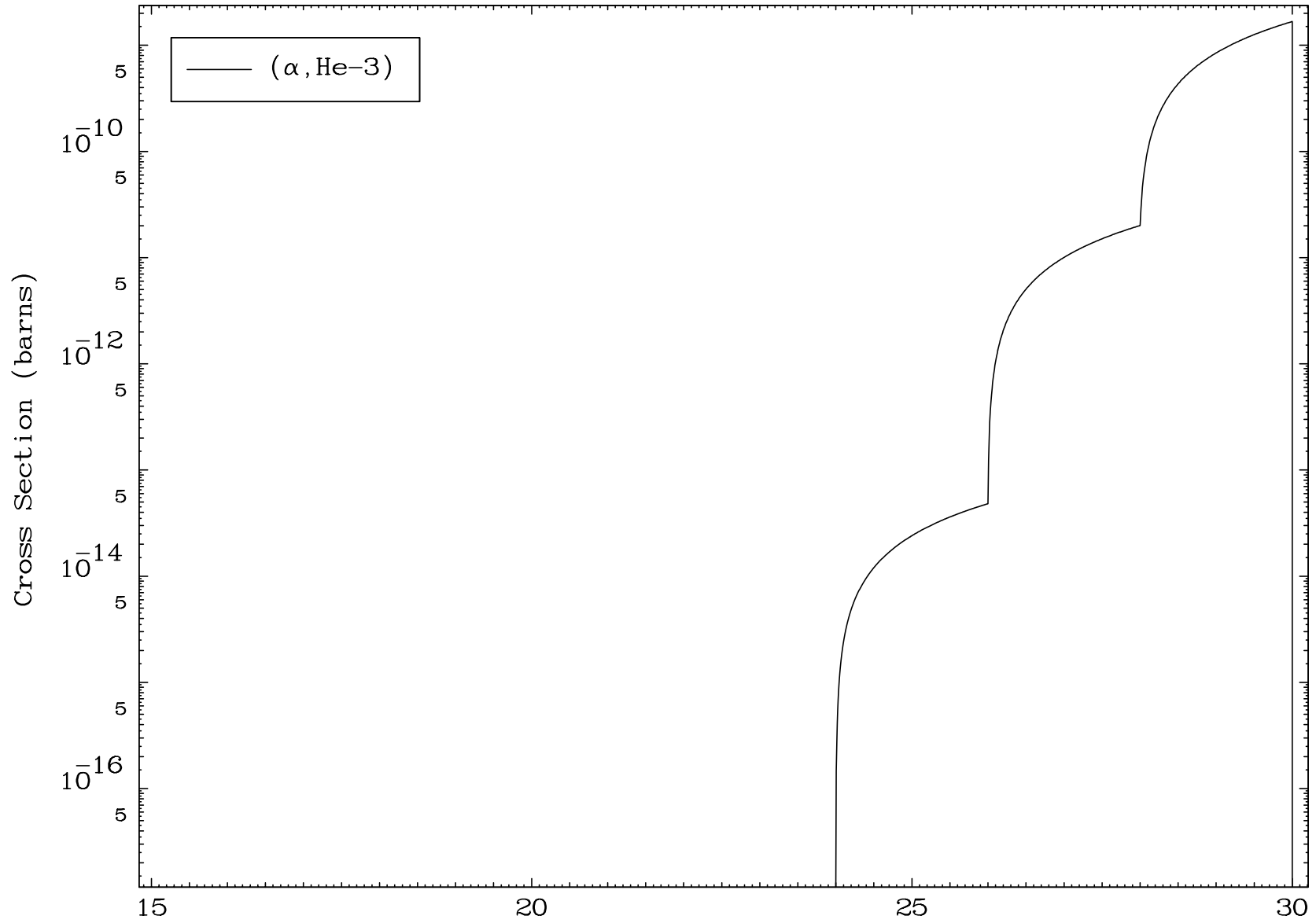
Incident Energy (MeV)

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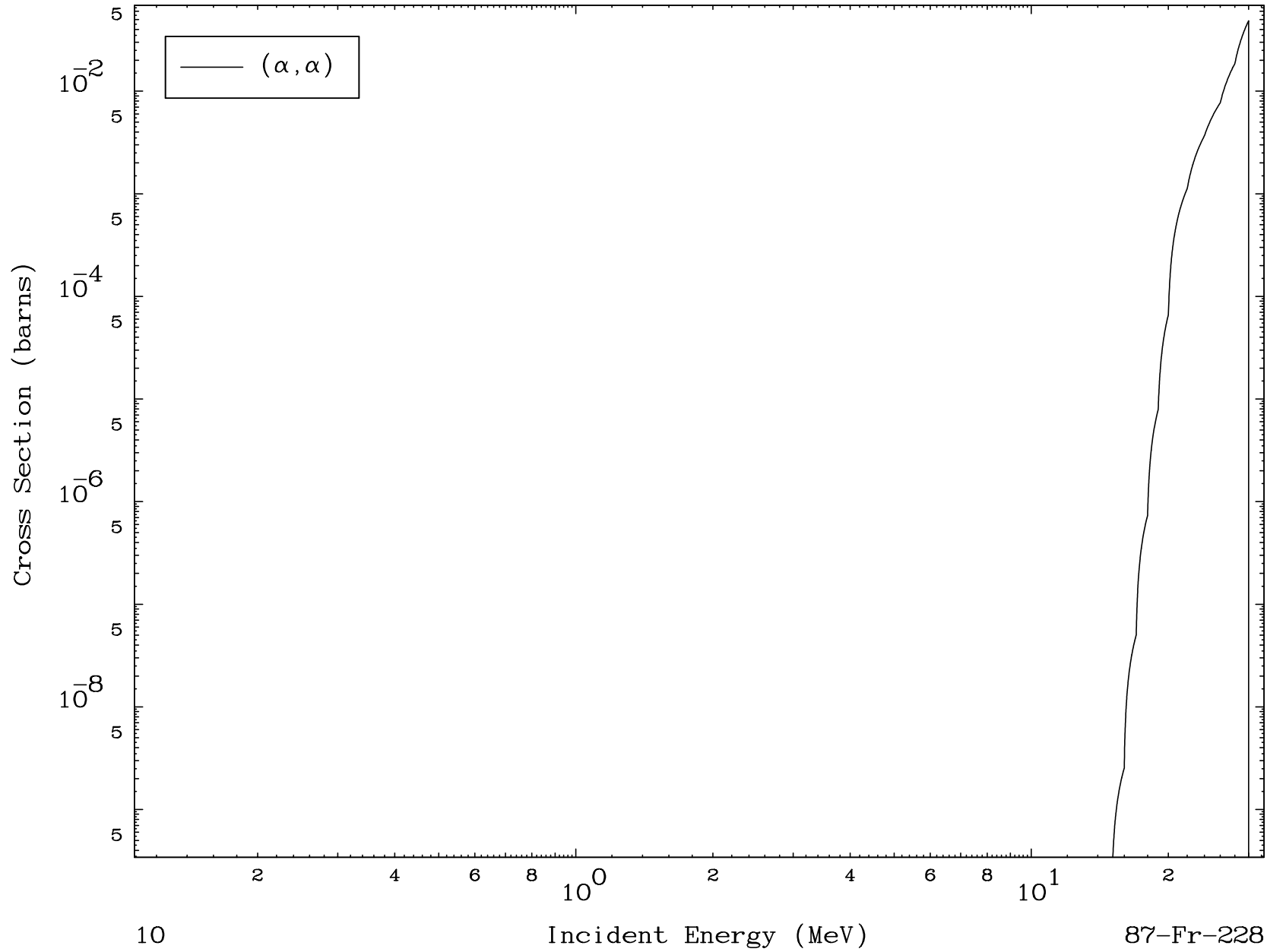




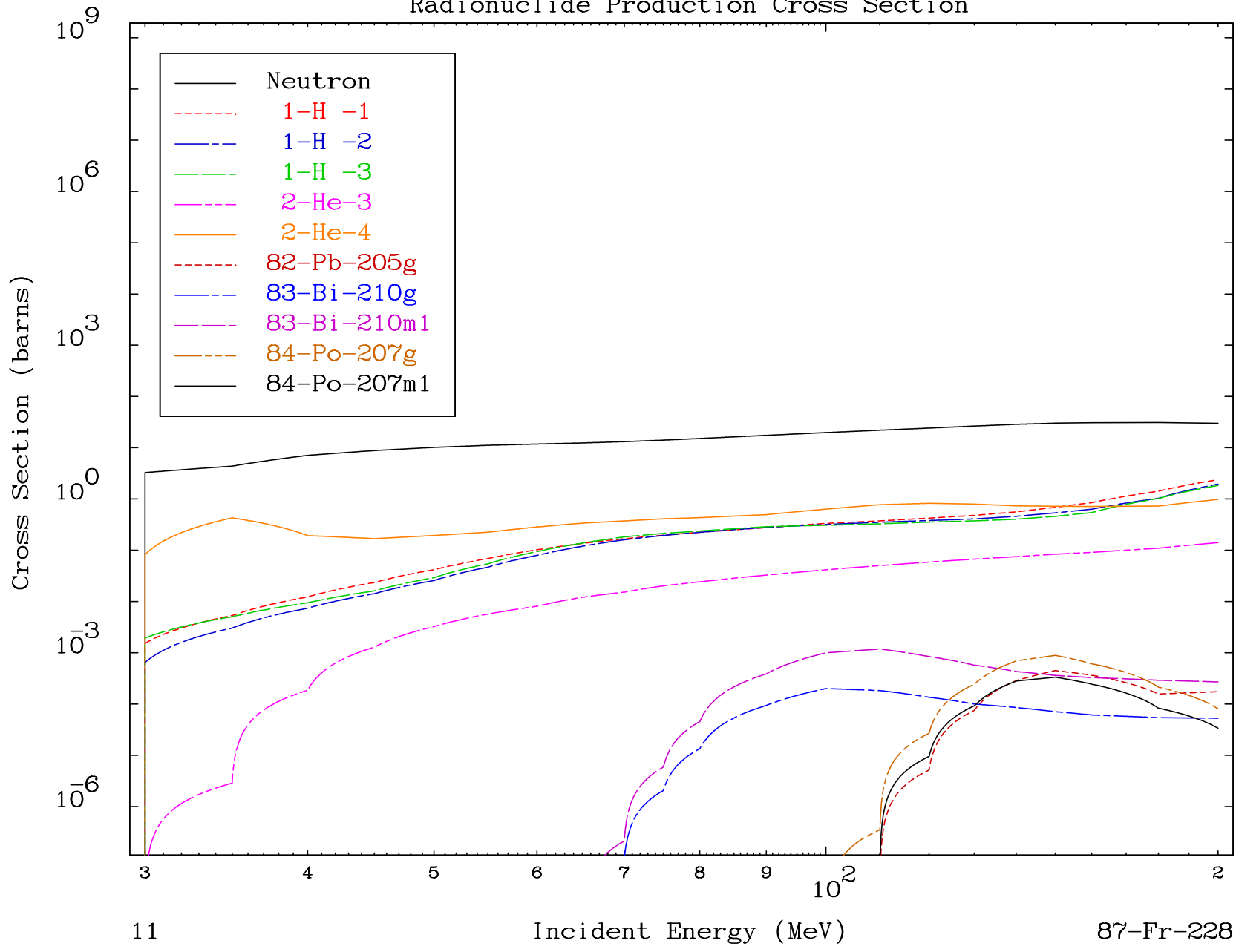
MAT 8773

(α, α) Levels
0 Kelvin Cross Sections

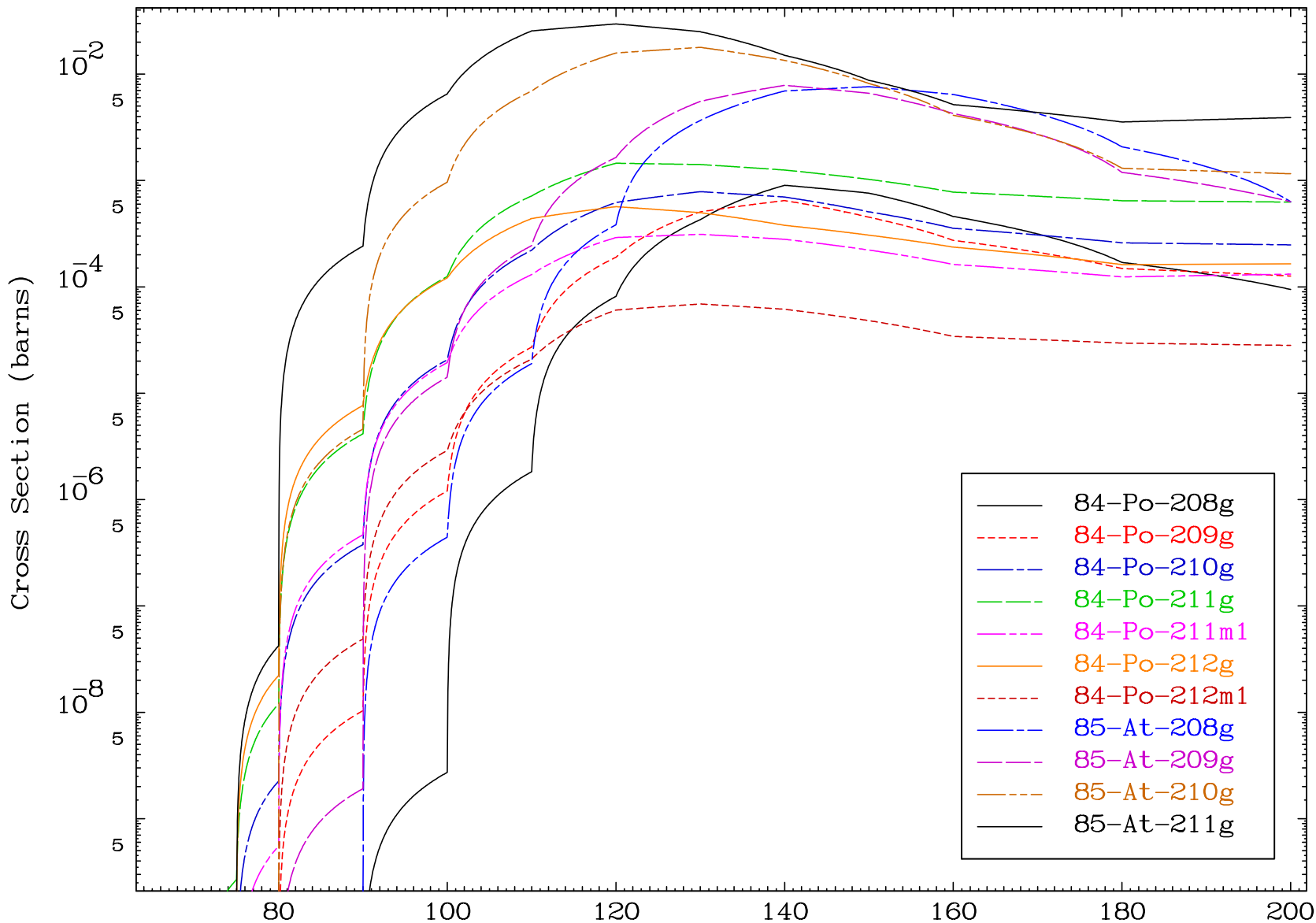
87-Fr-228

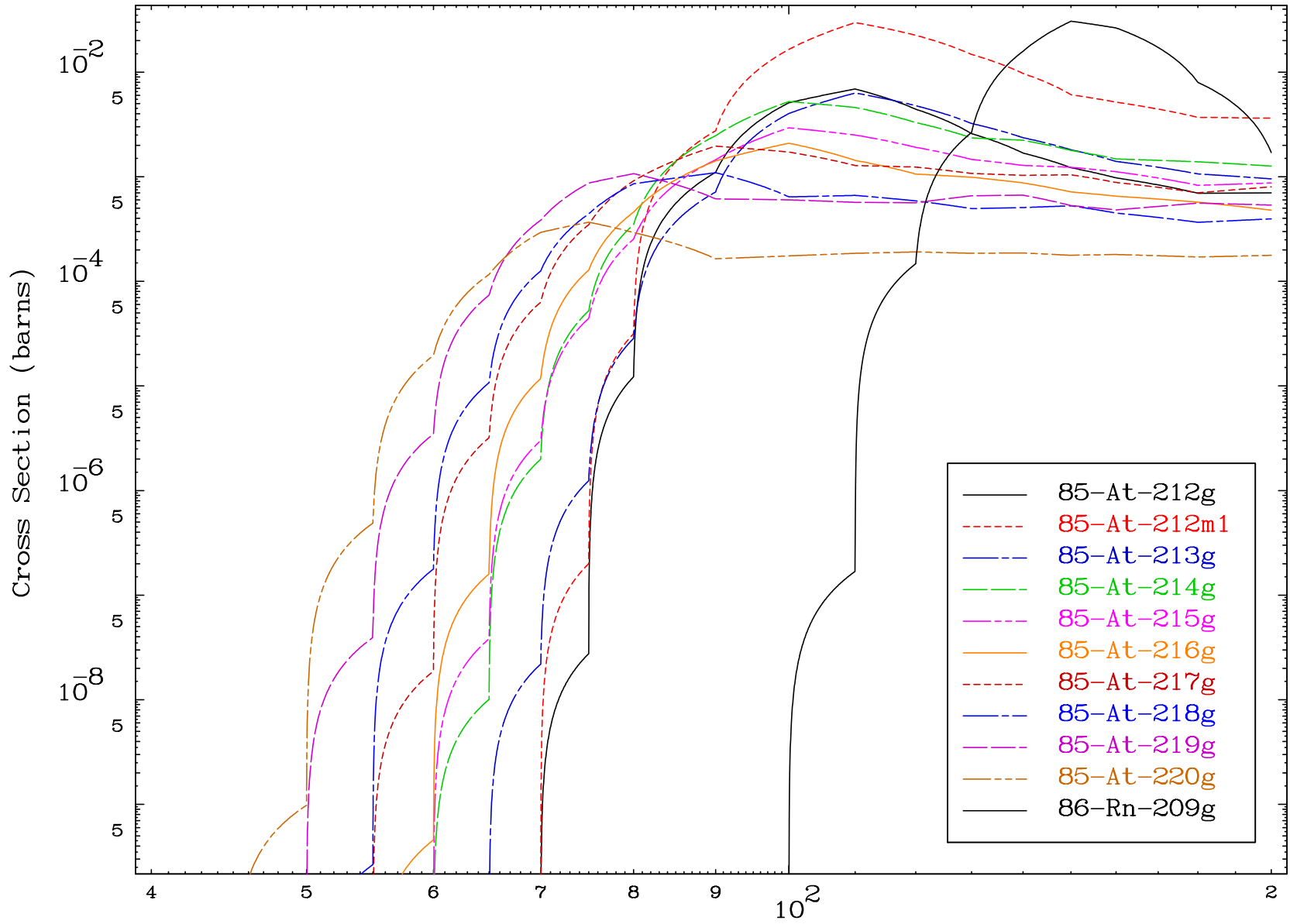


Radionuclide Production Cross Section



Radionuclide Production Cross Section



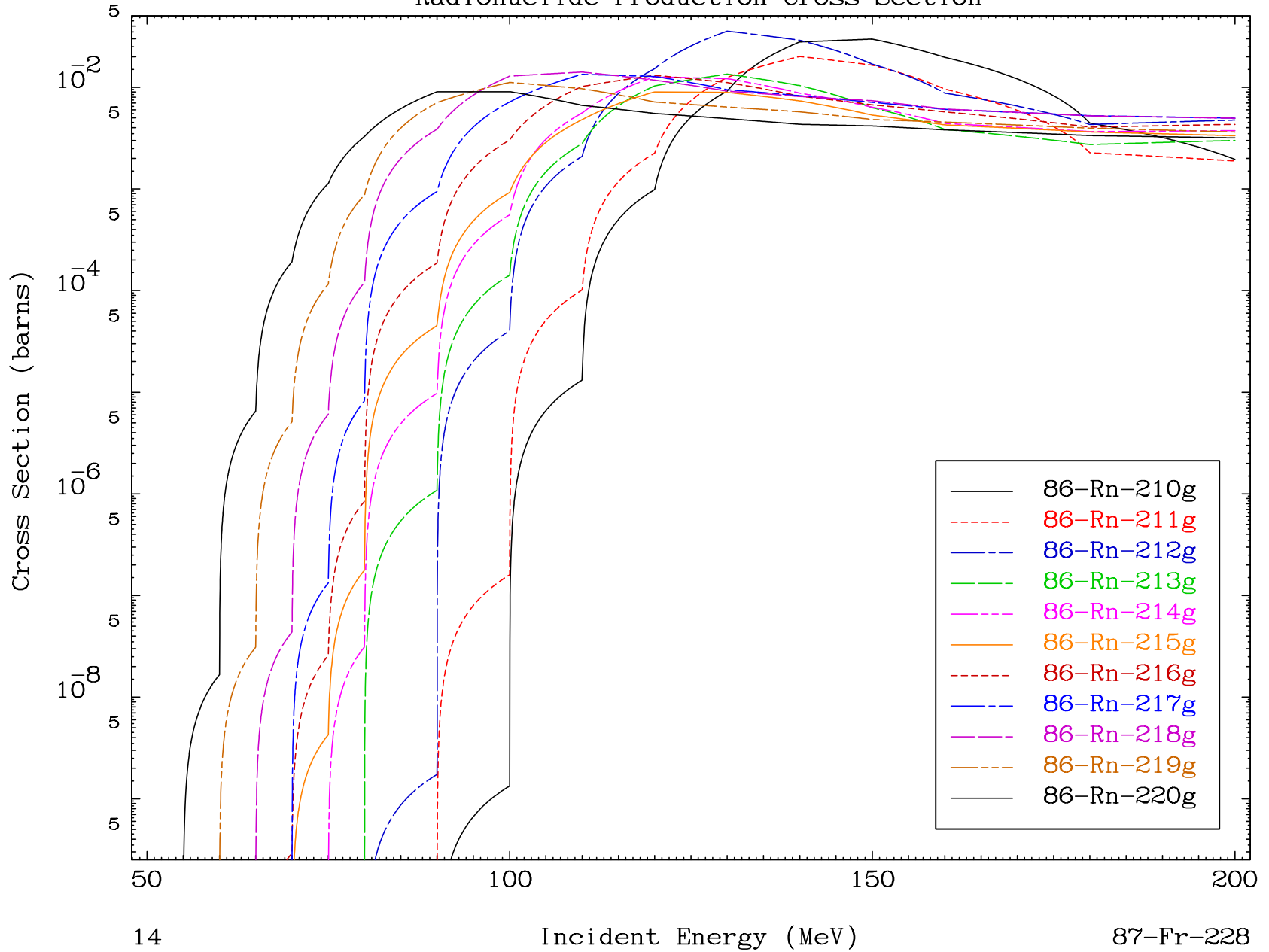


MAT 8773

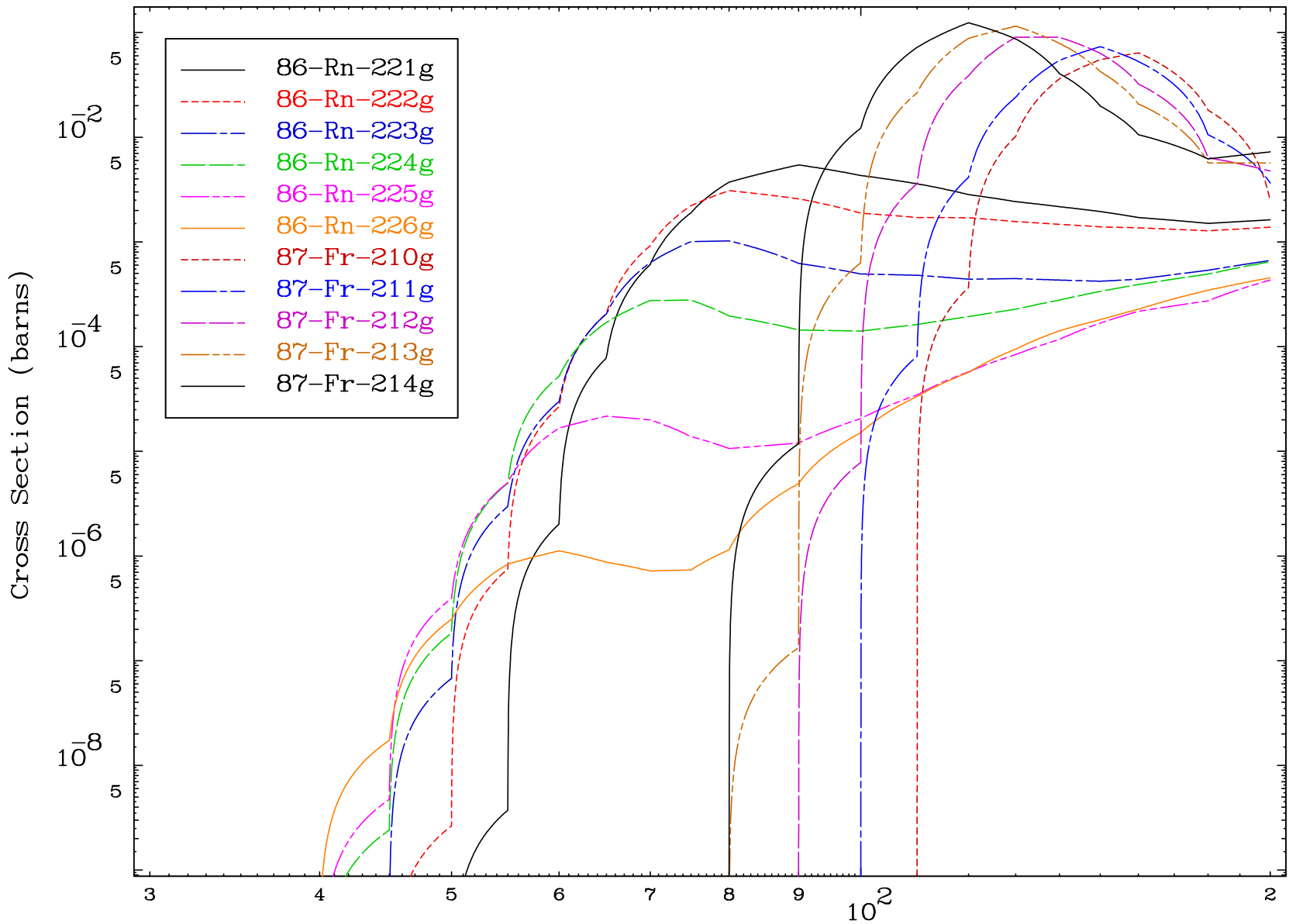
(α , remainder)

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

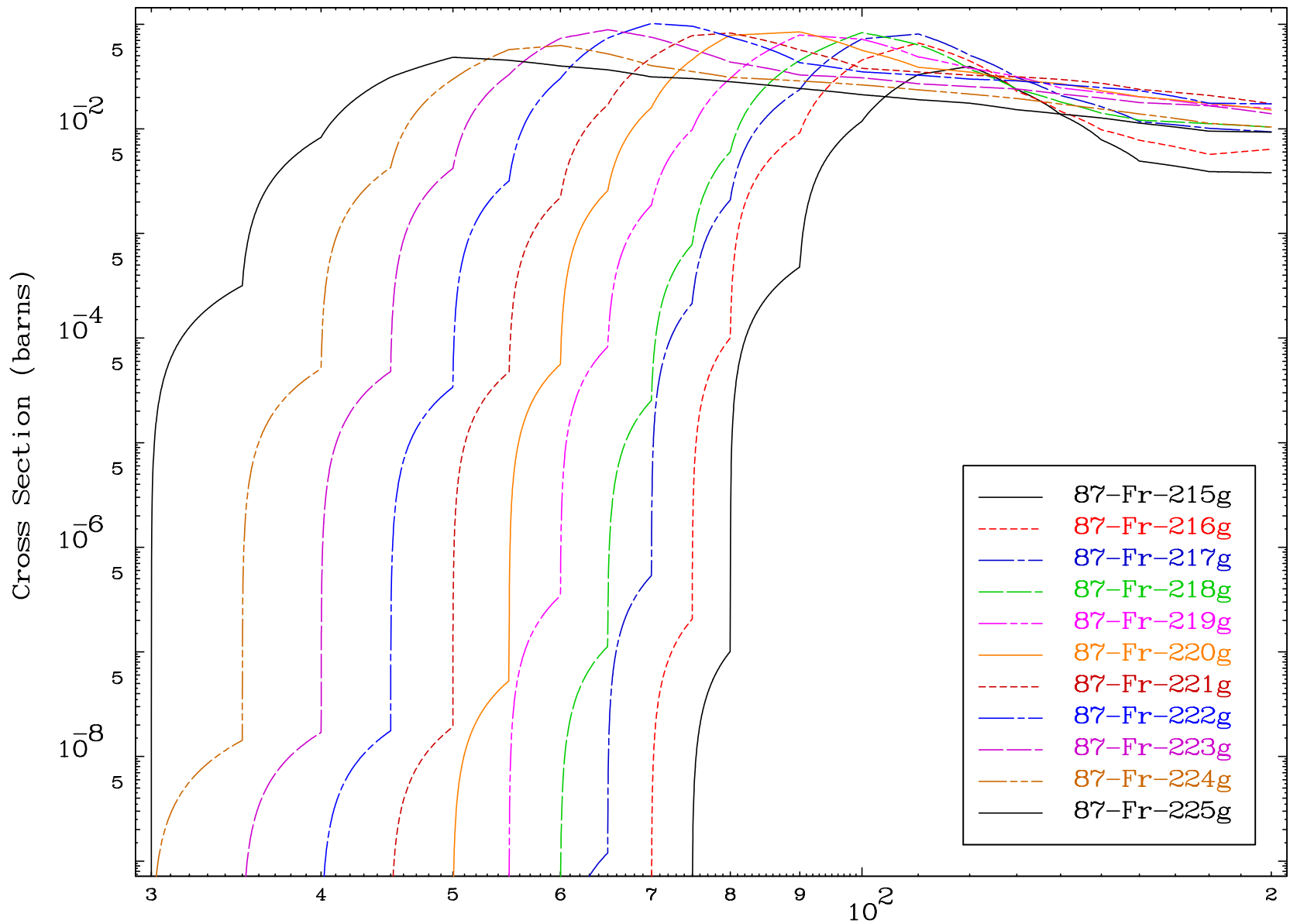


MAT 8773

(α , remainder)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

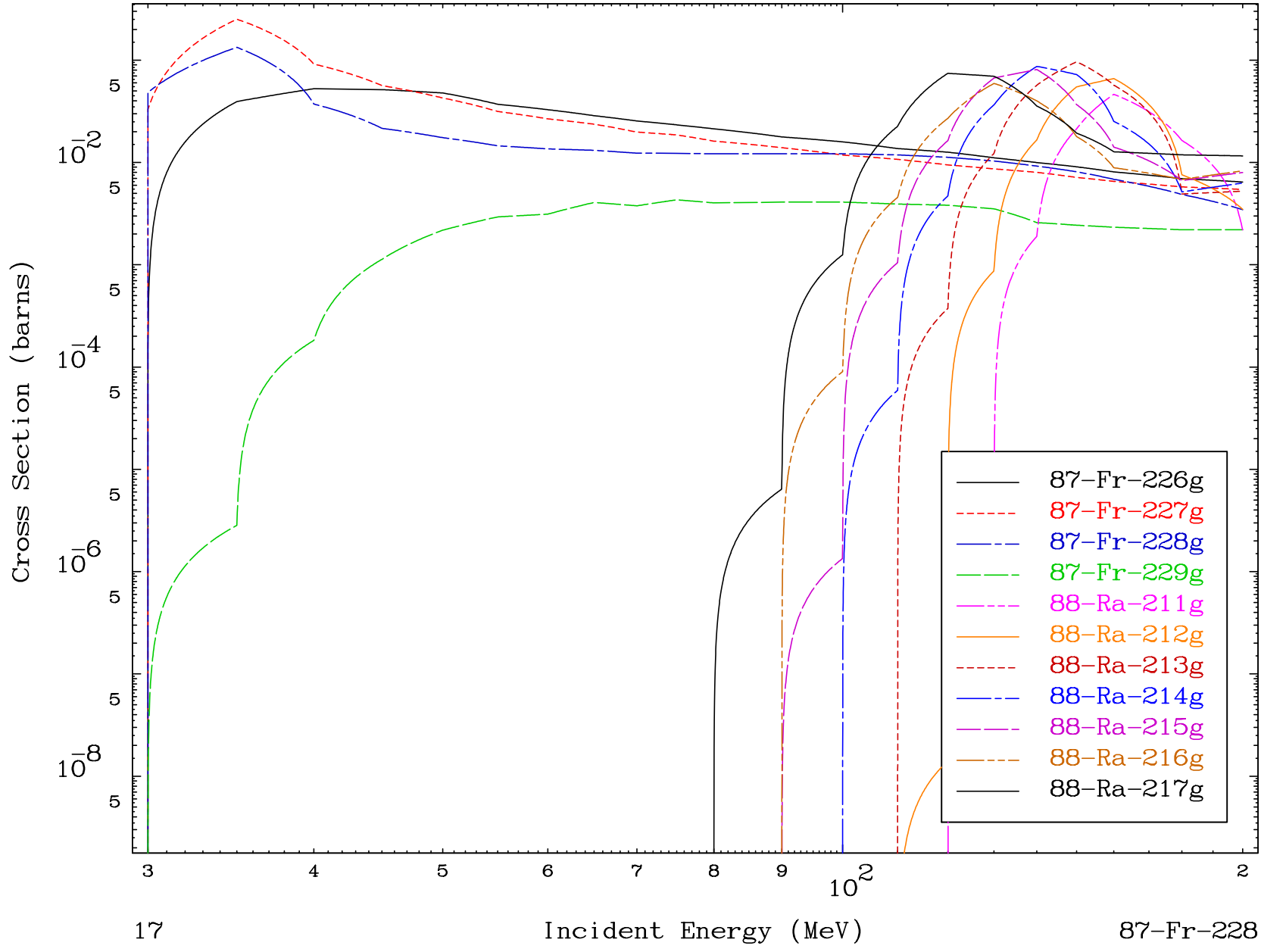
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MAT 8773

(α , remainder)

87-Fr-228

Radionuclide Production Cross Section



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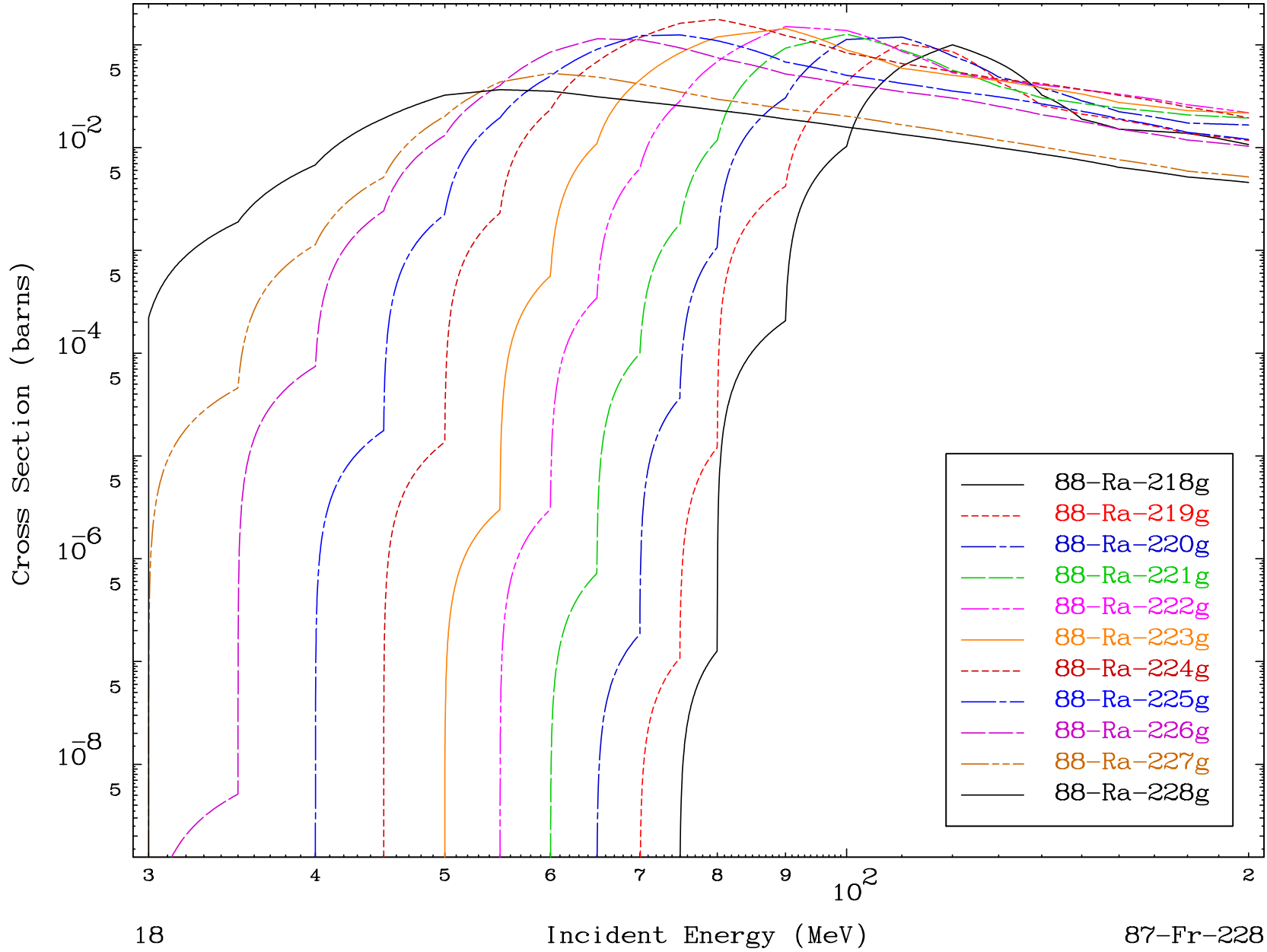
Incident Energy (MeV)

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MAT 8773

(α , remainder)
Radionuclide Production Cross Section

87-Fr-228

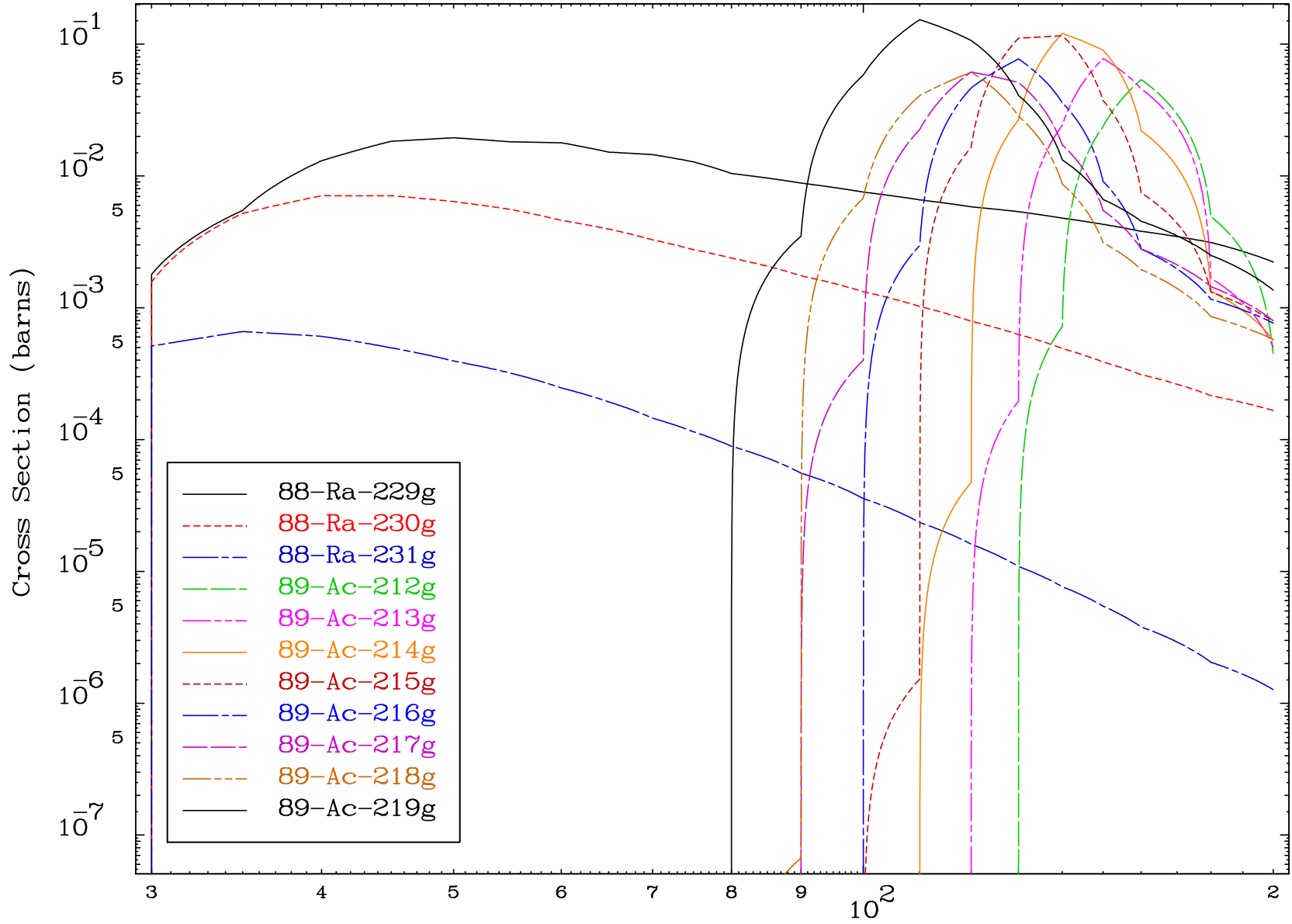


MAT 8773

(α , remainder)

87-Fr-228

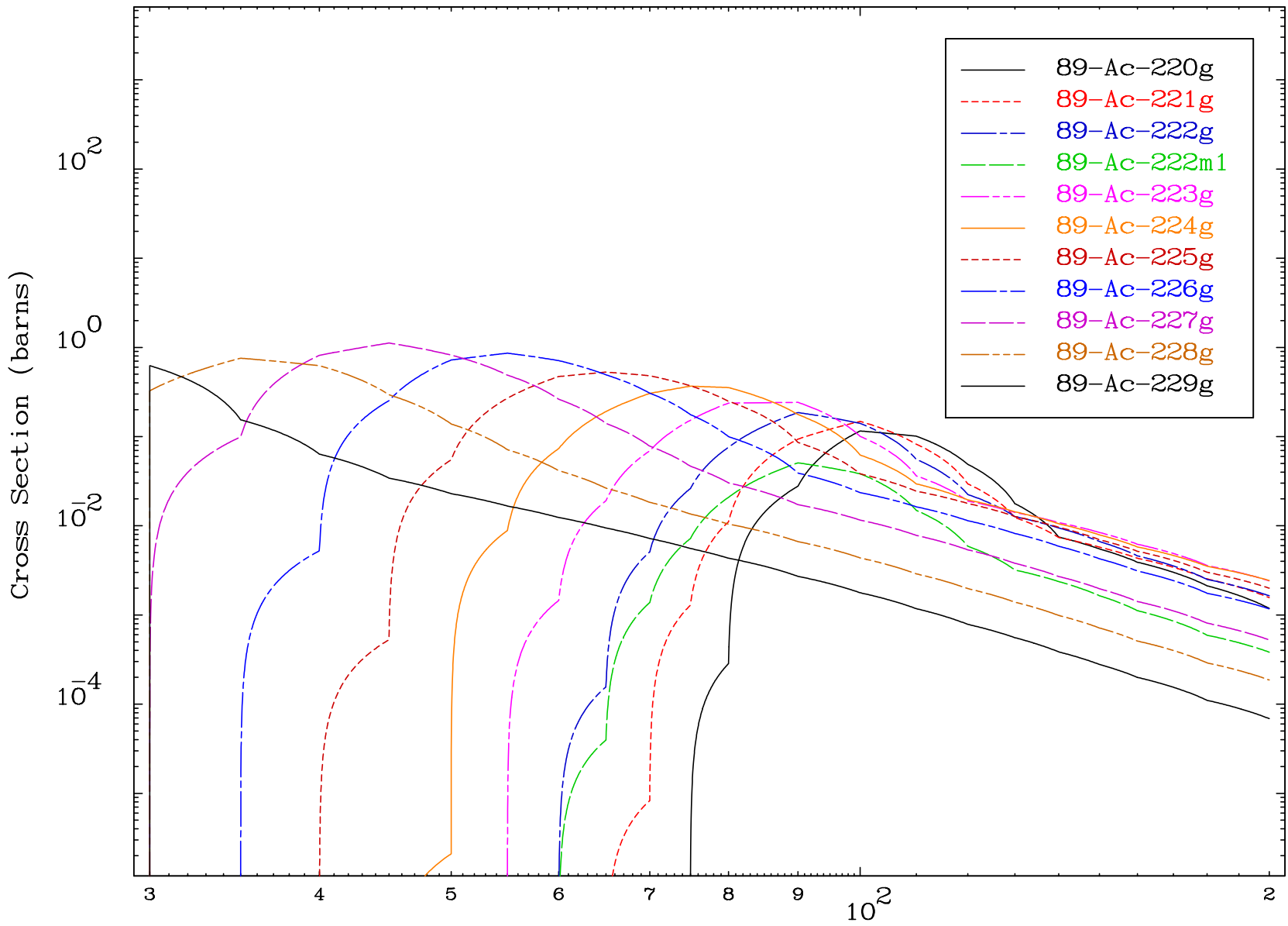
Radionuclide Production Cross Section



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Incident Energy (MeV)

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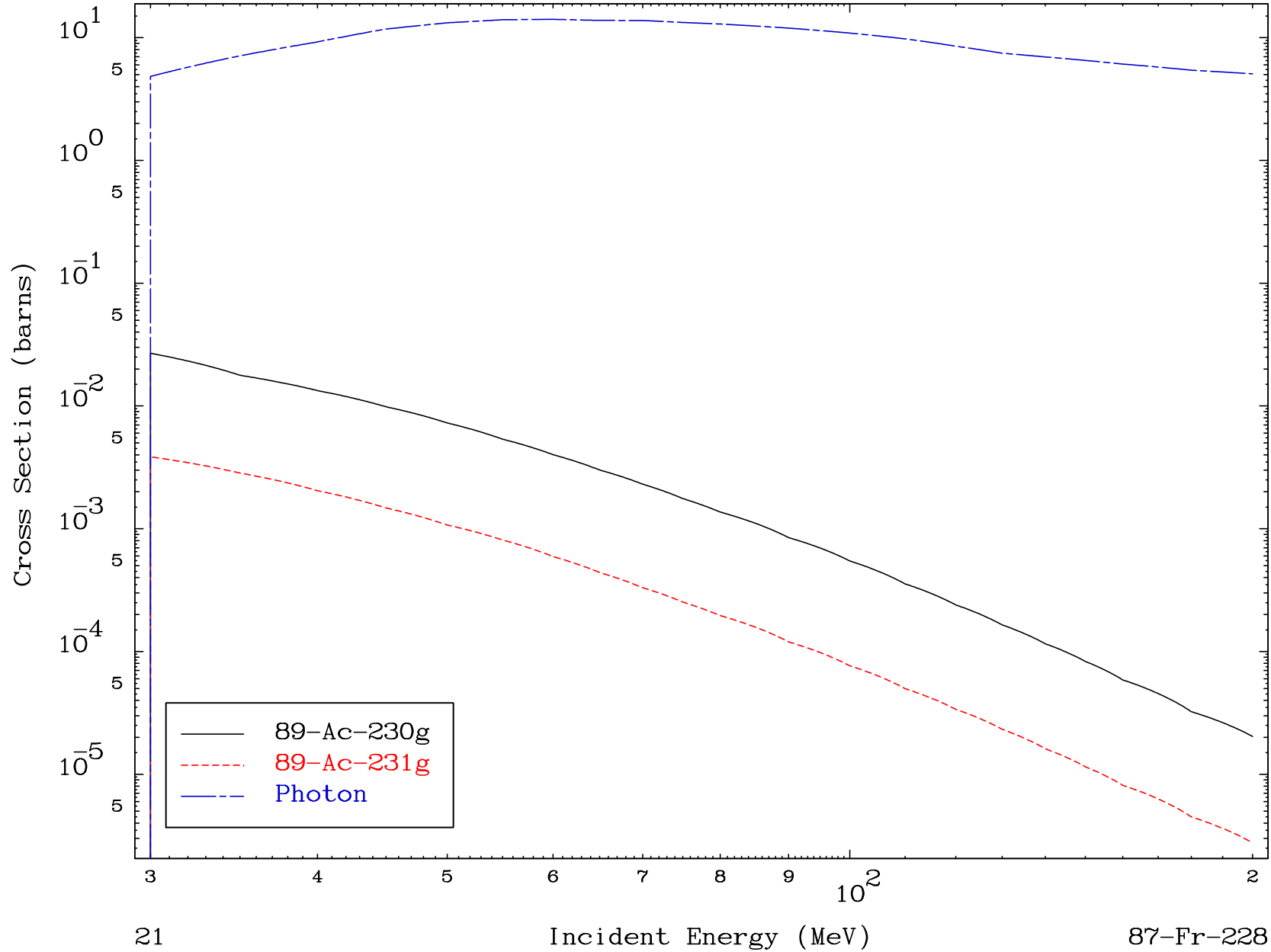


MAT 8773

(α , remainder)

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Radionuclide Production Cross Section



MAT 8773

α Fission
Radionuclide Production Cross Section

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