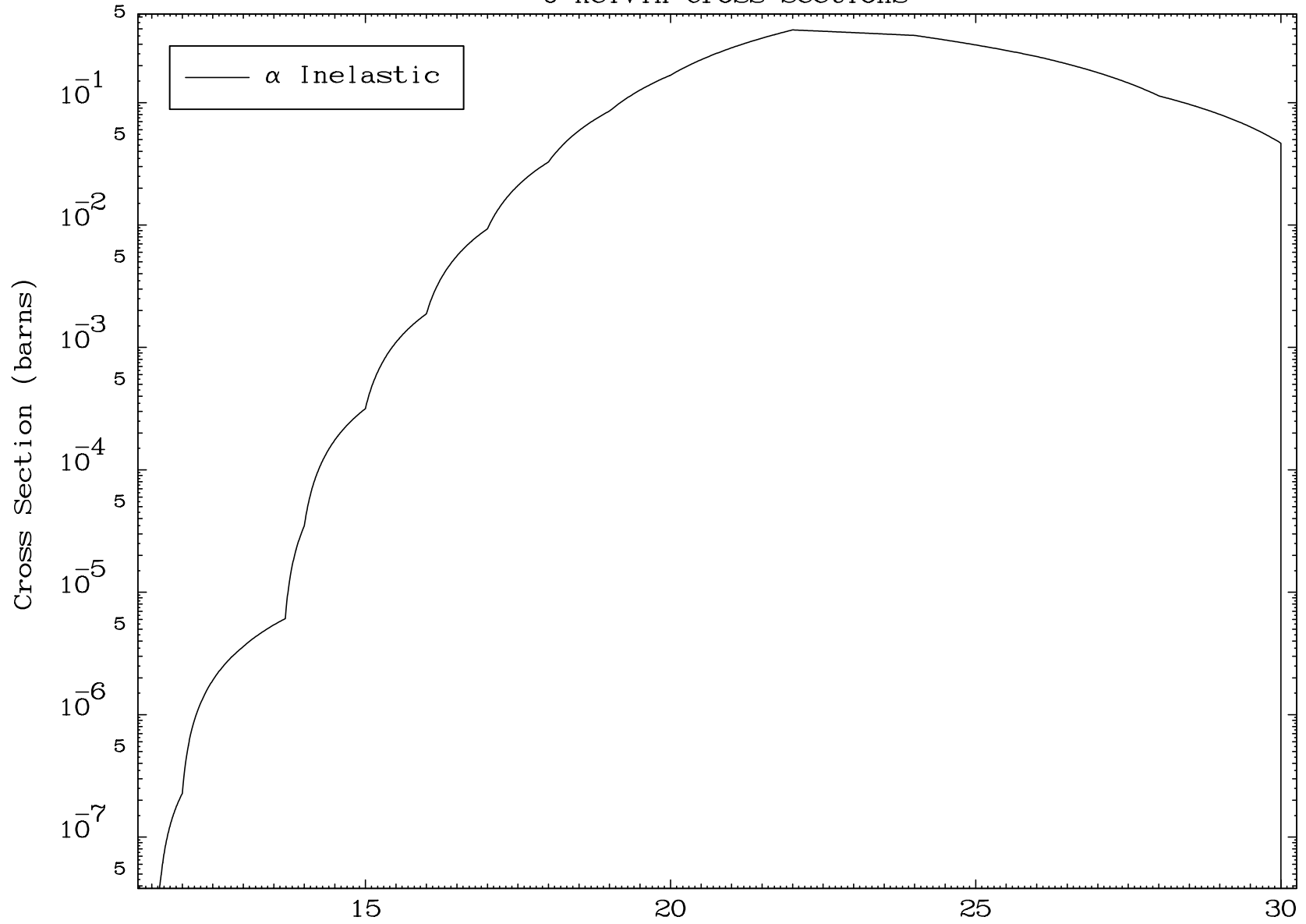


MAT 7210

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

72-Hf-169



5

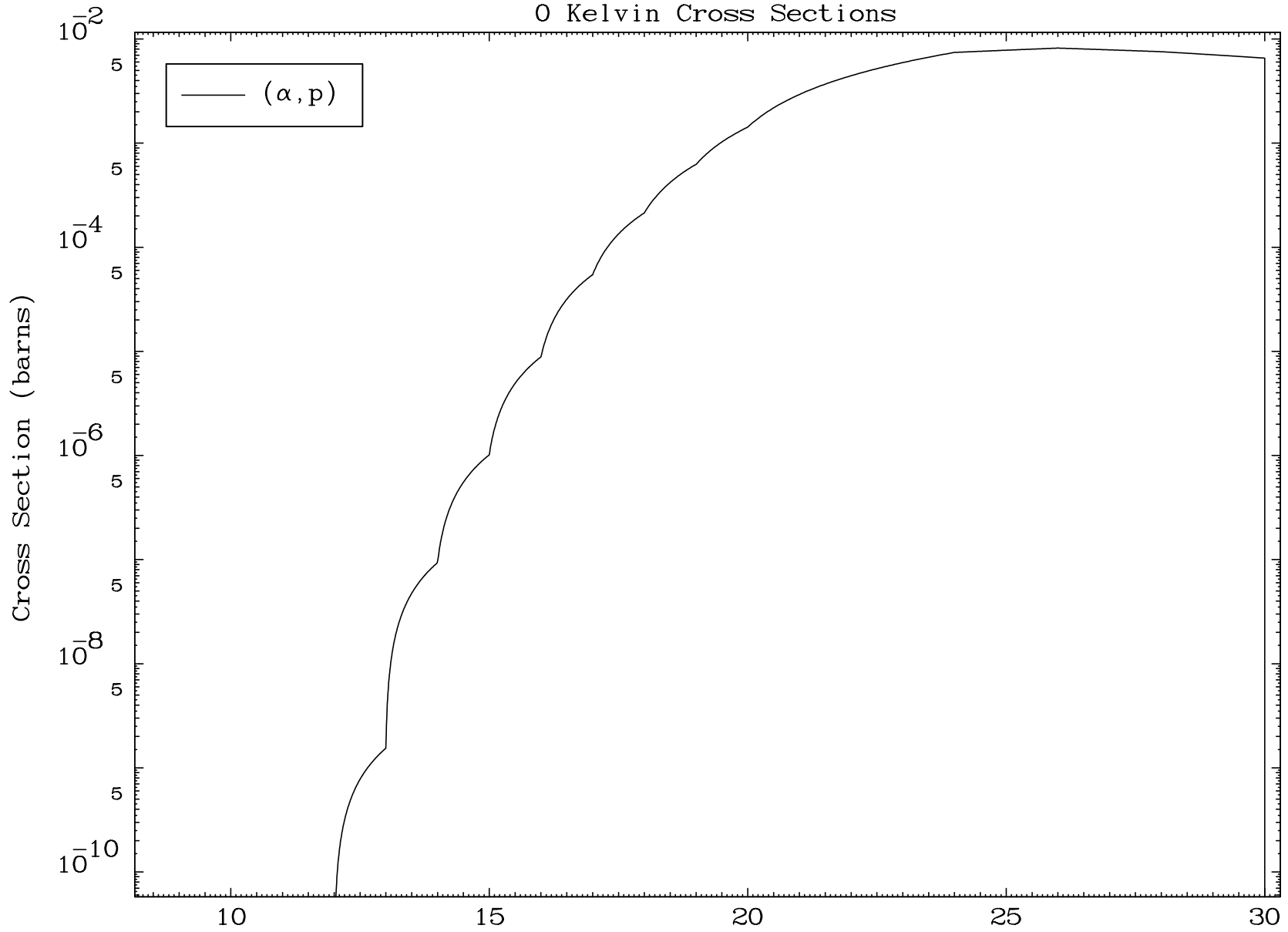
Incident Energy (MeV)

72-Hf-169

MAT 7210

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

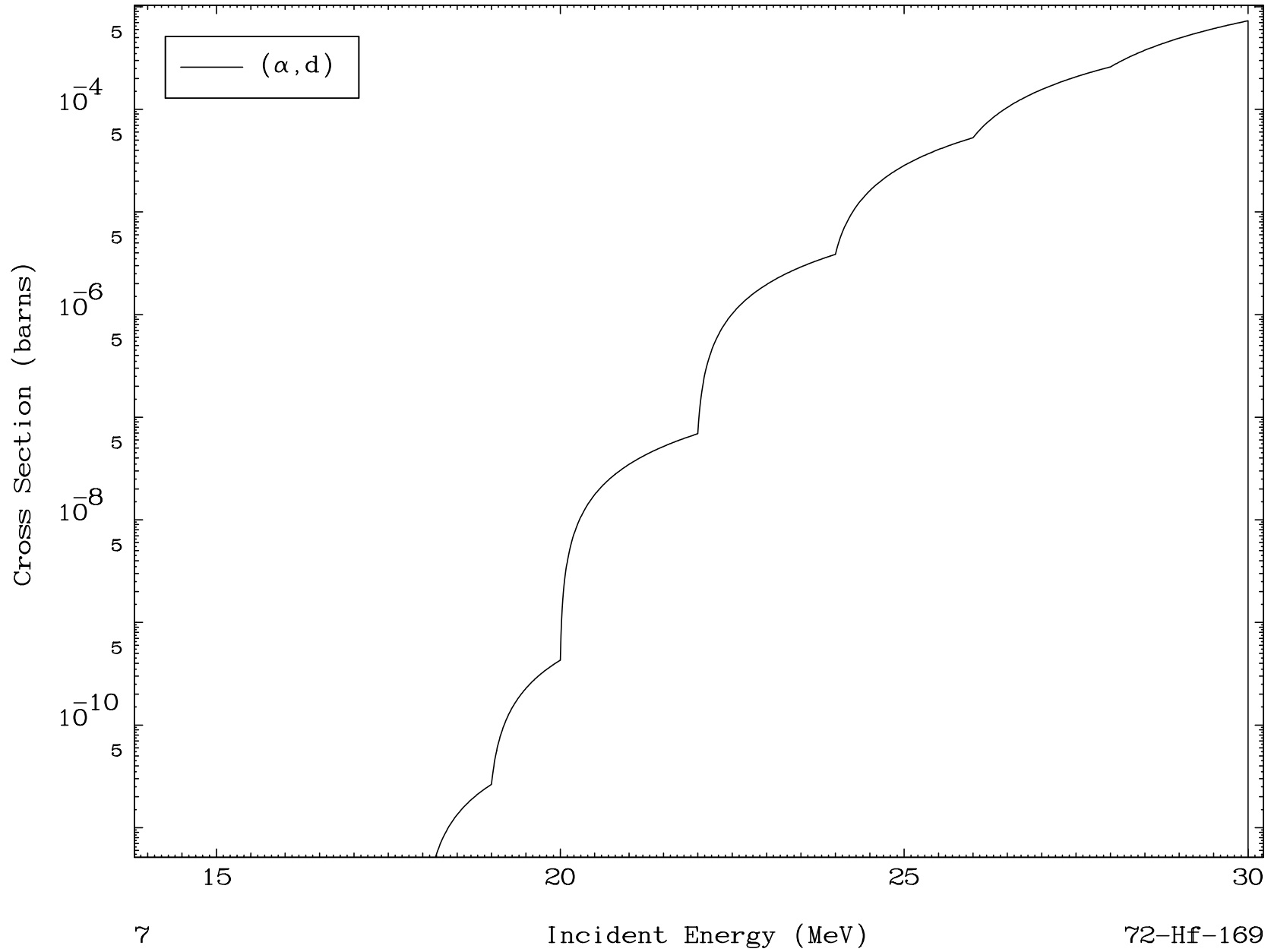
72-Hf-169

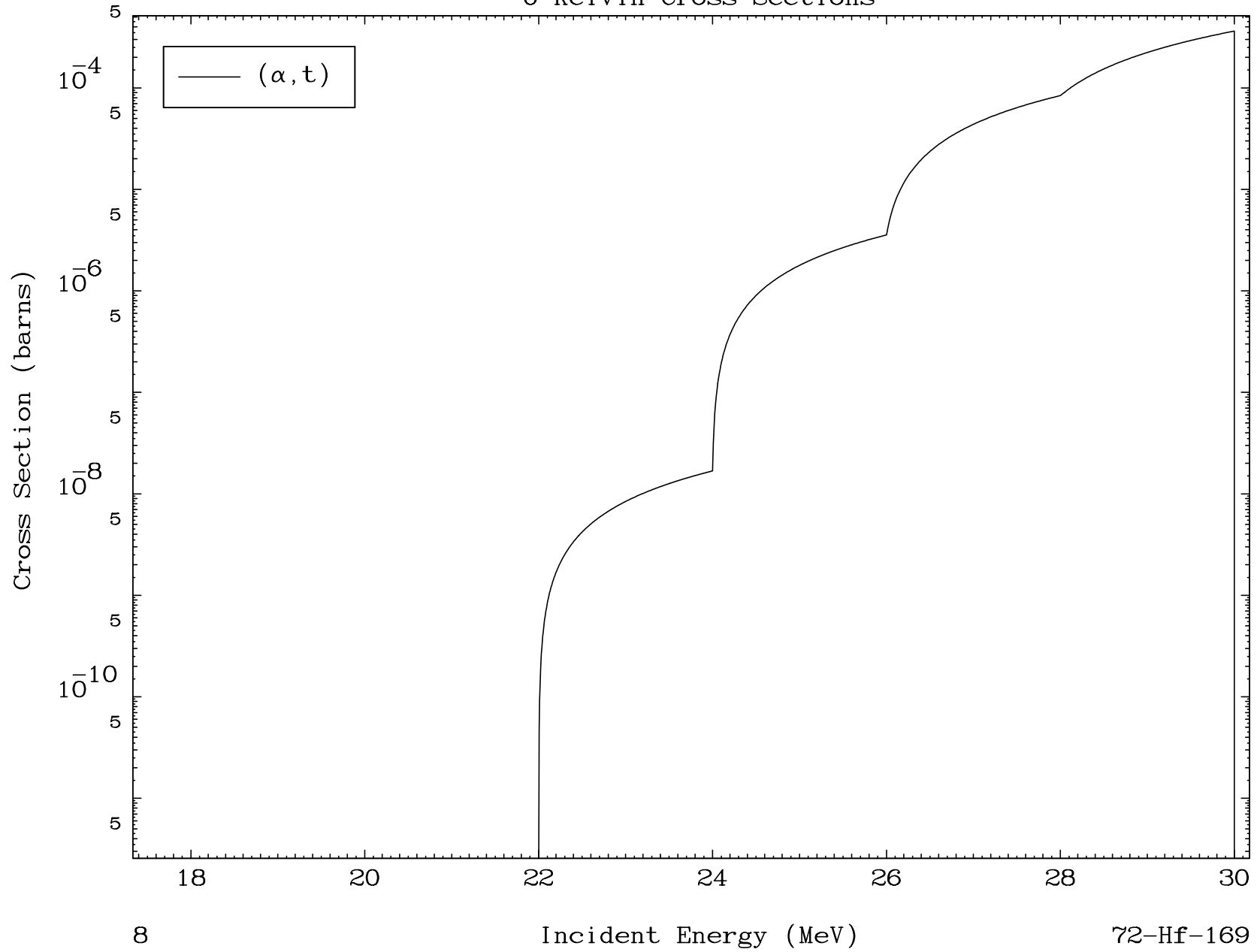


6

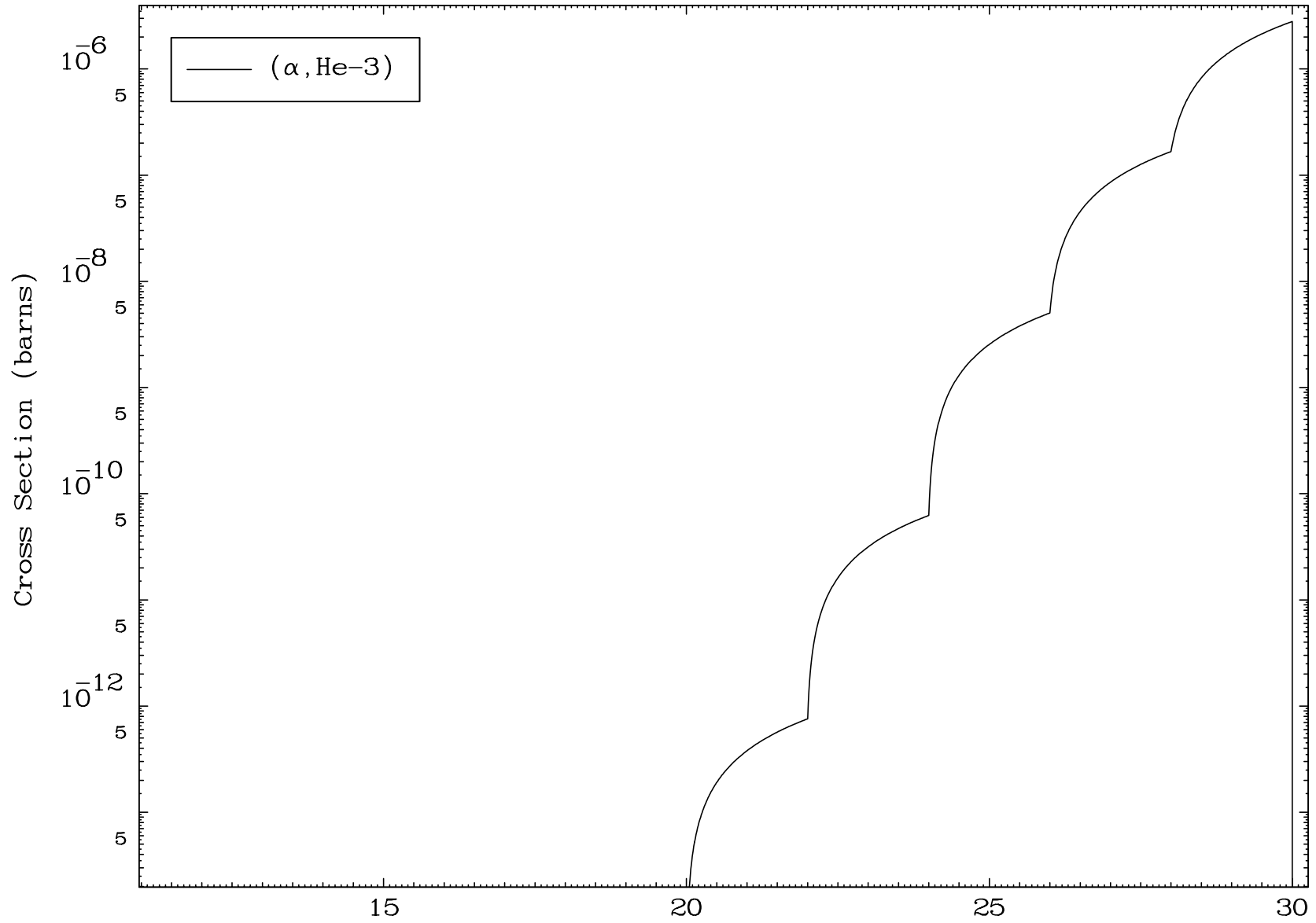
Incident Energy (MeV)

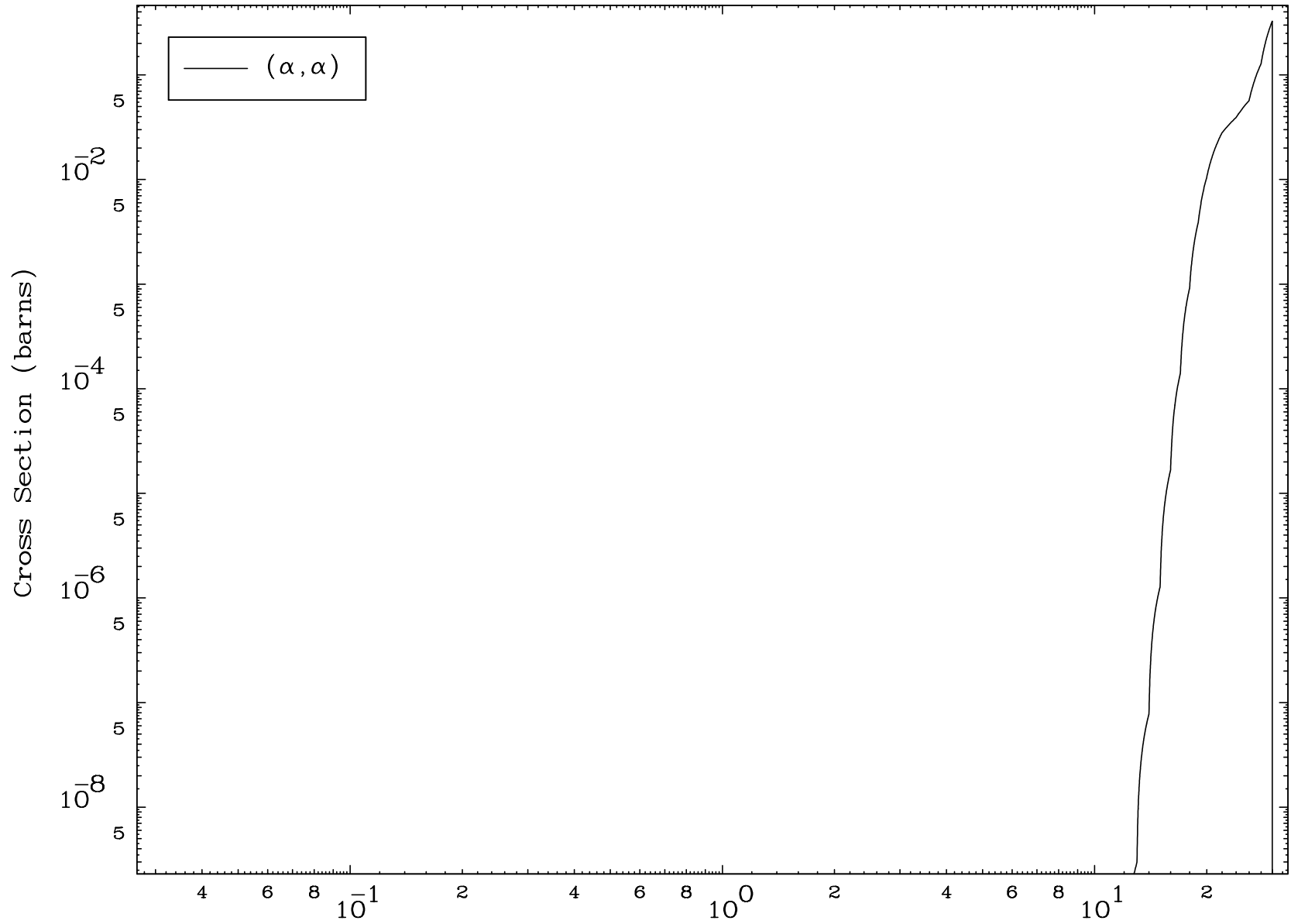
72-Hf-169



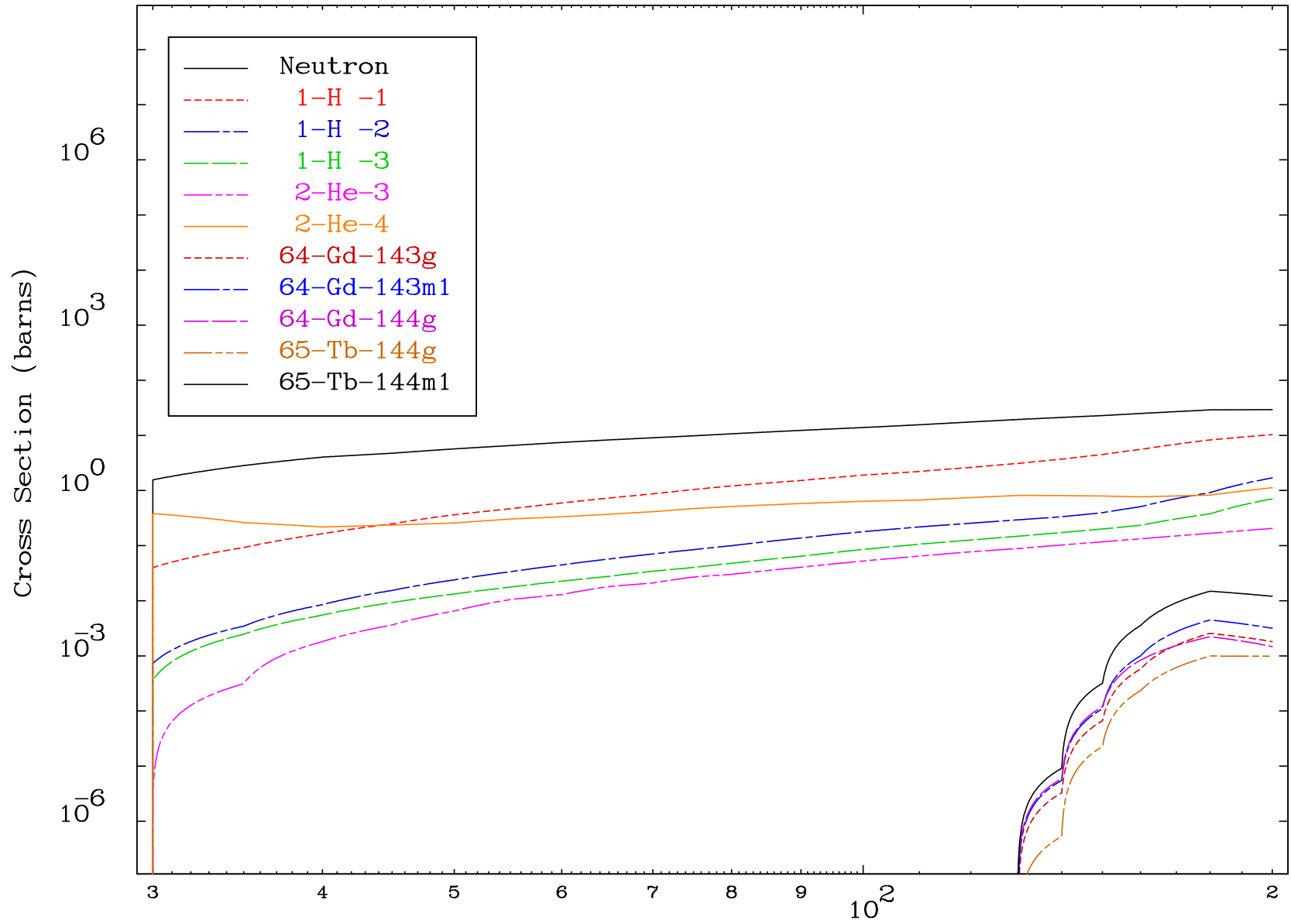




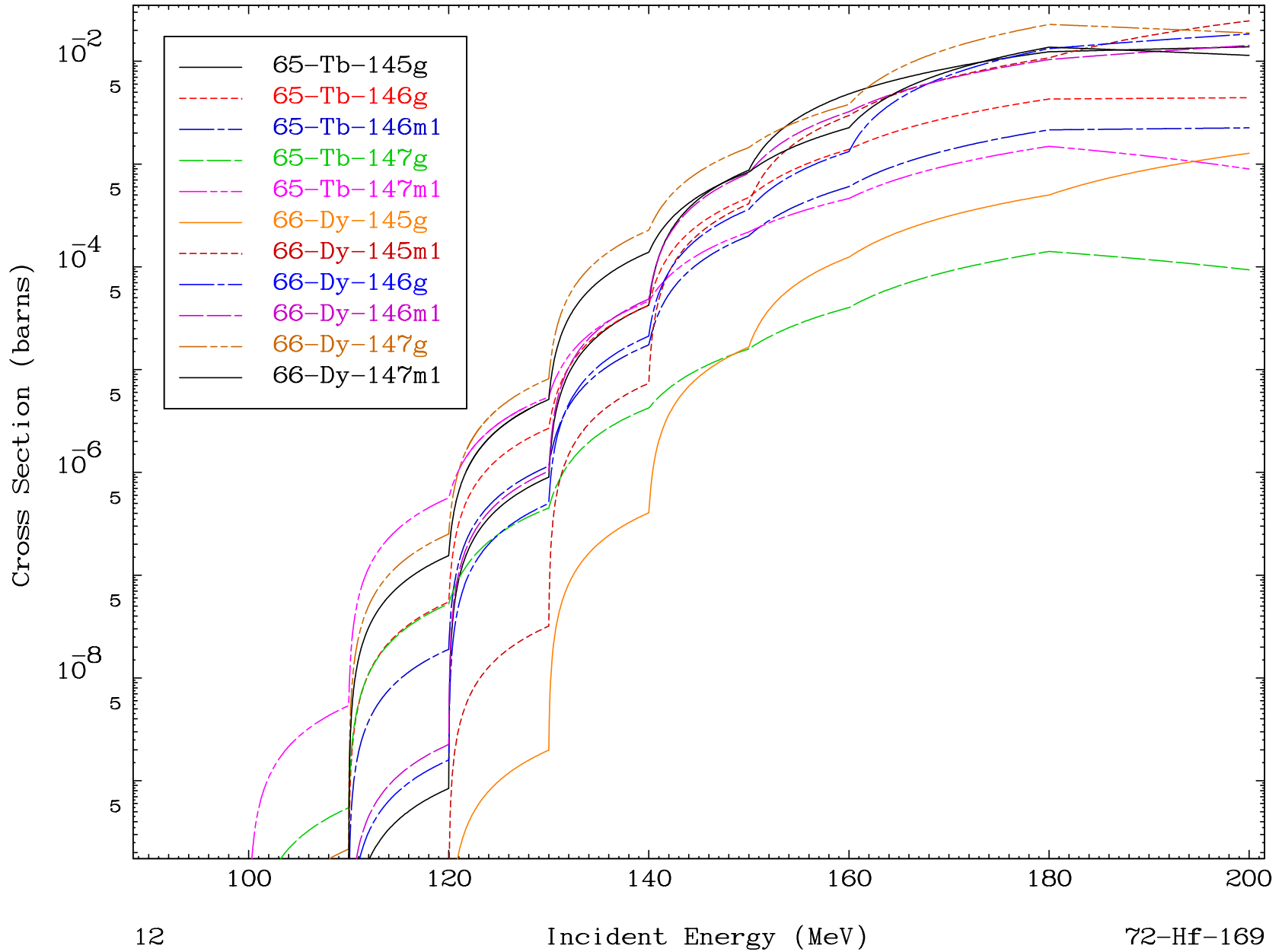


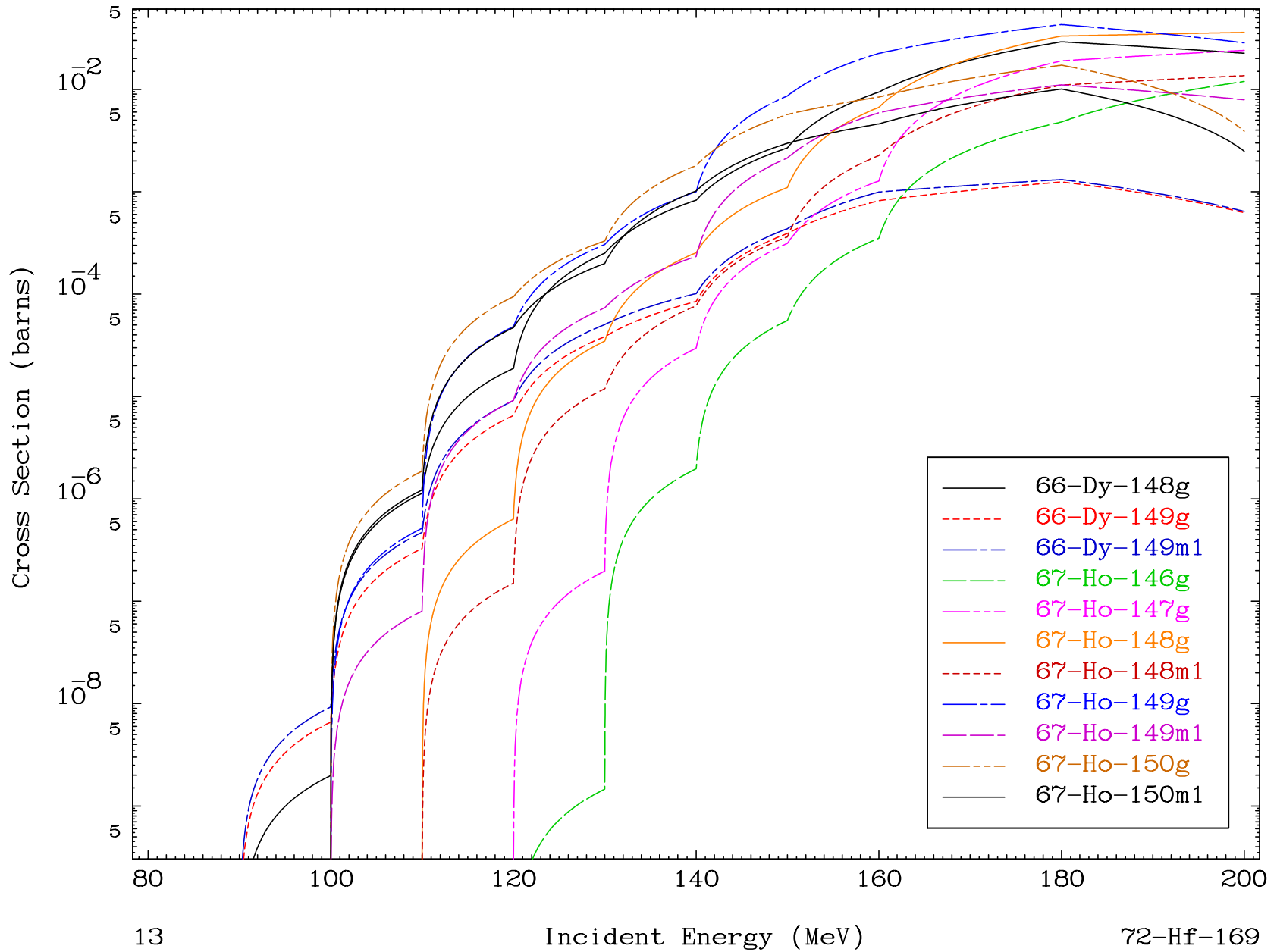


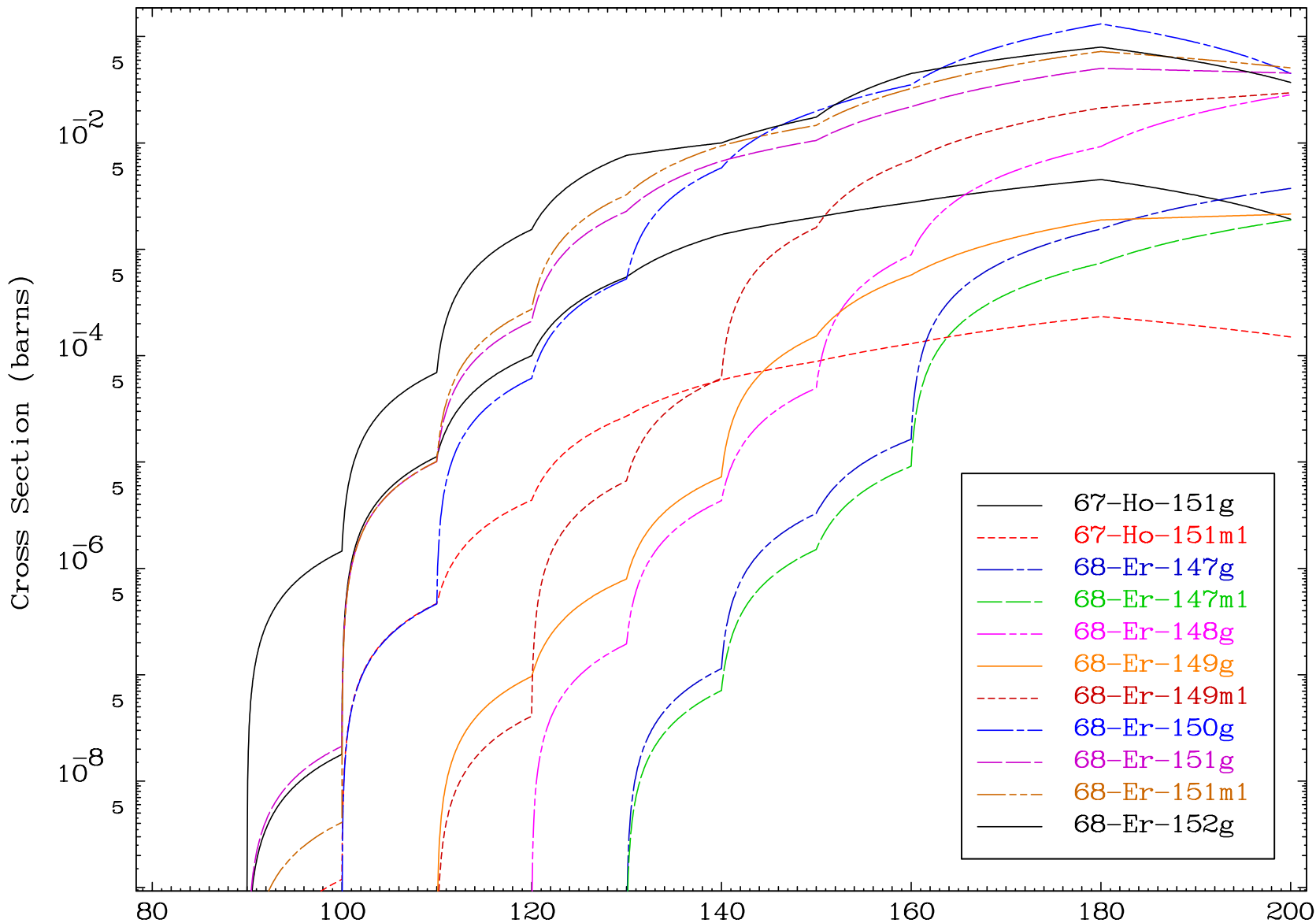
Radionuclide Production Cross Section

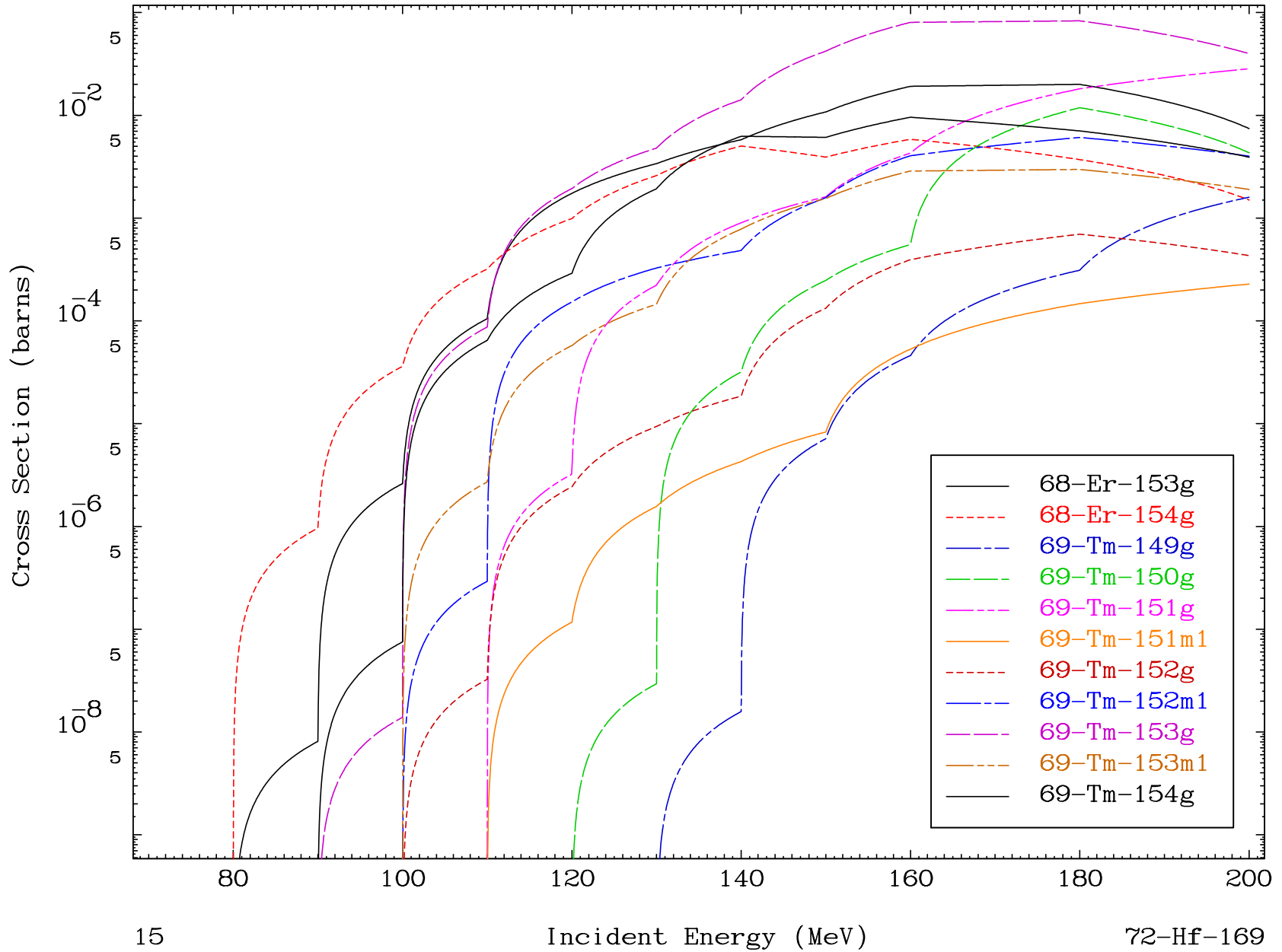


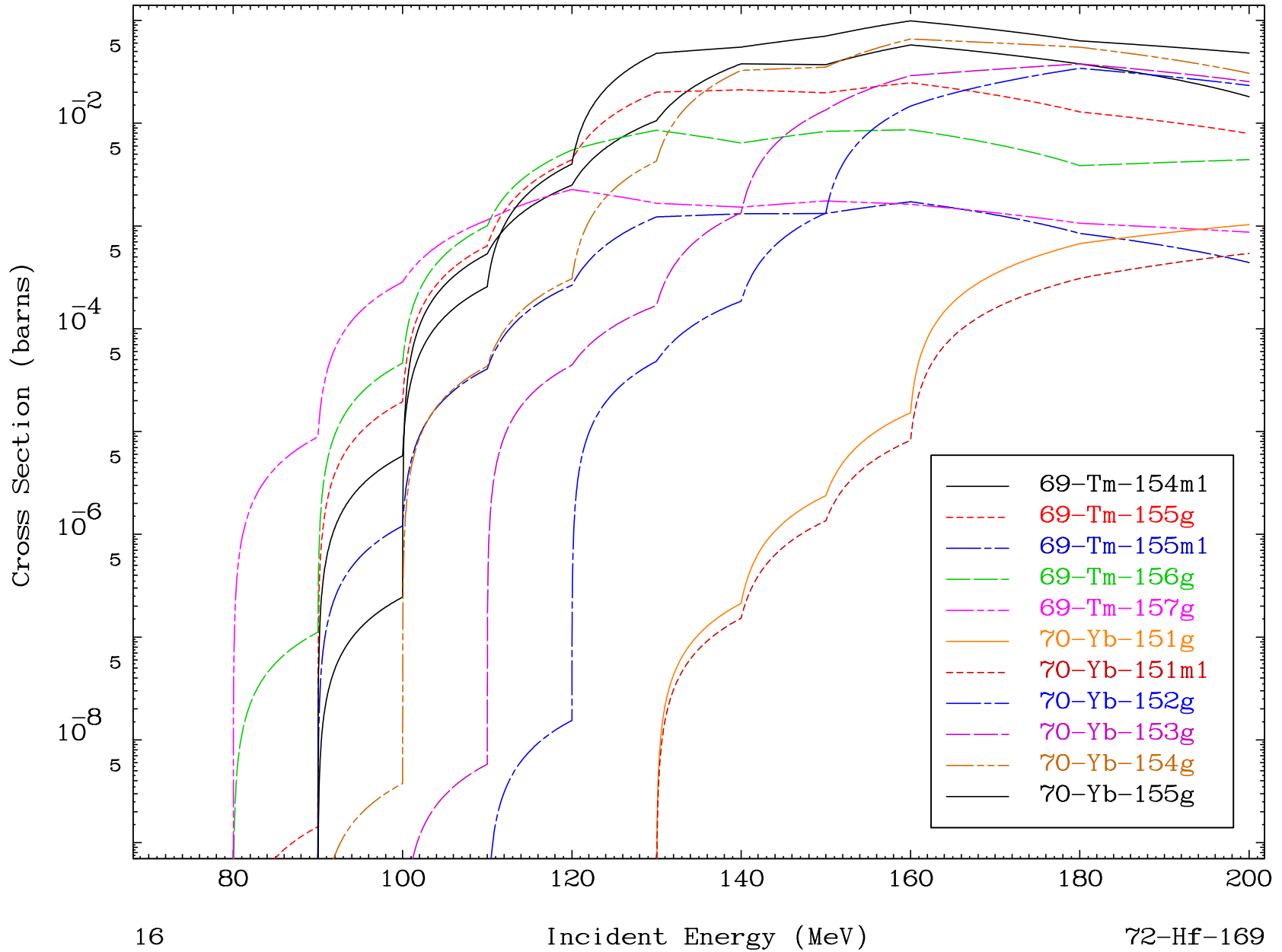
Radionuclide Production Cross Section





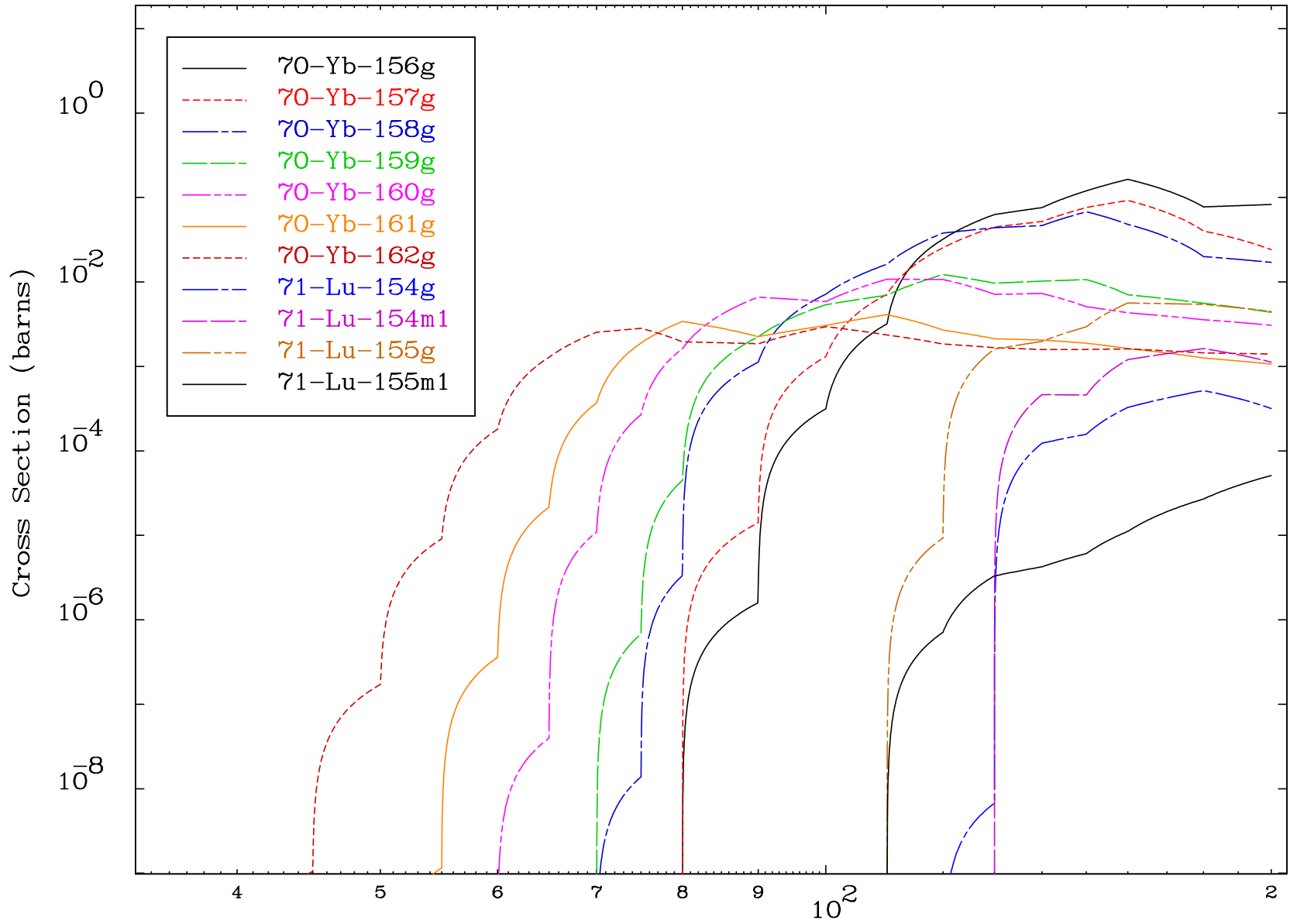




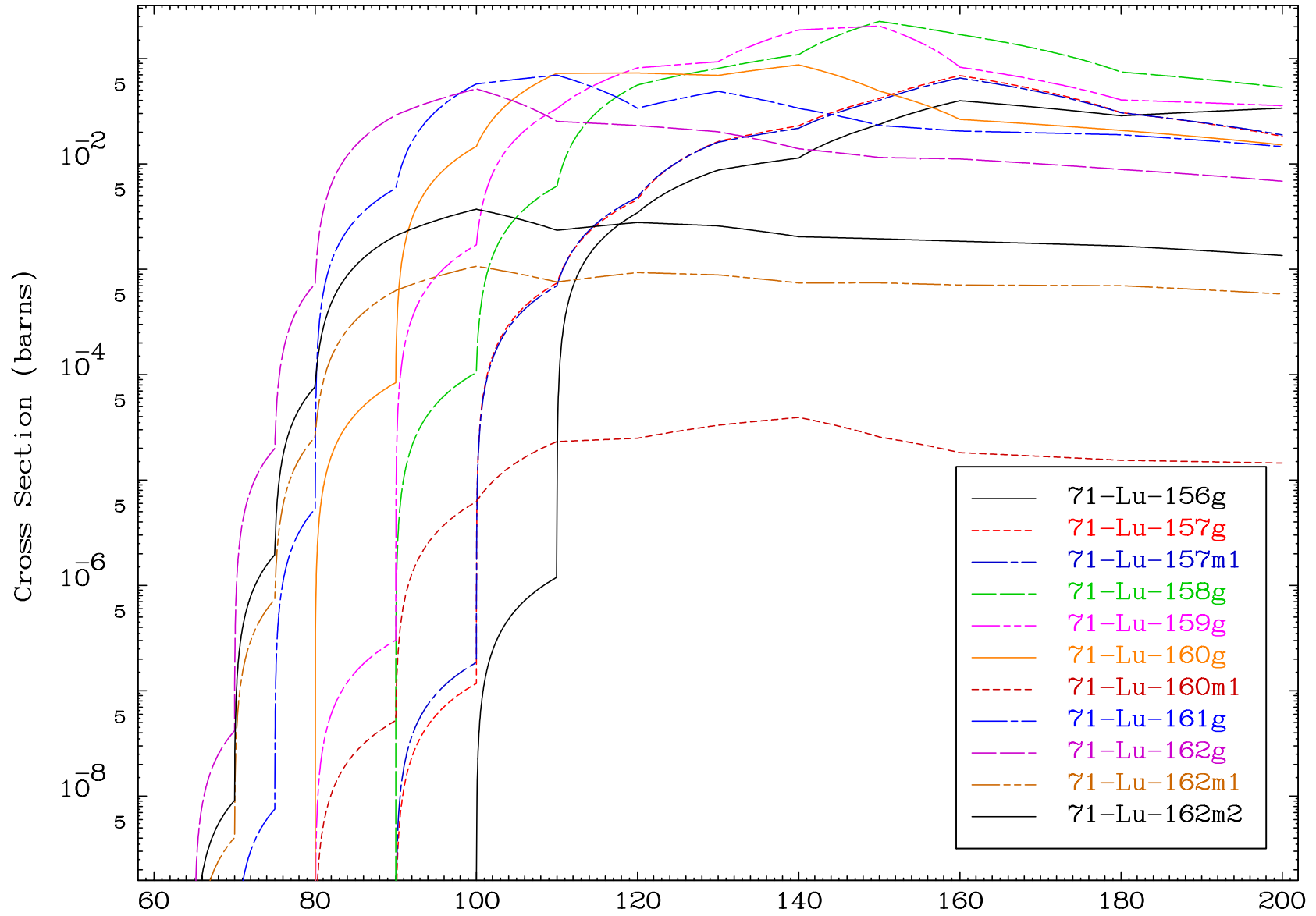




Radionuclide Production Cross Section



Radionuclide Production Cross Section

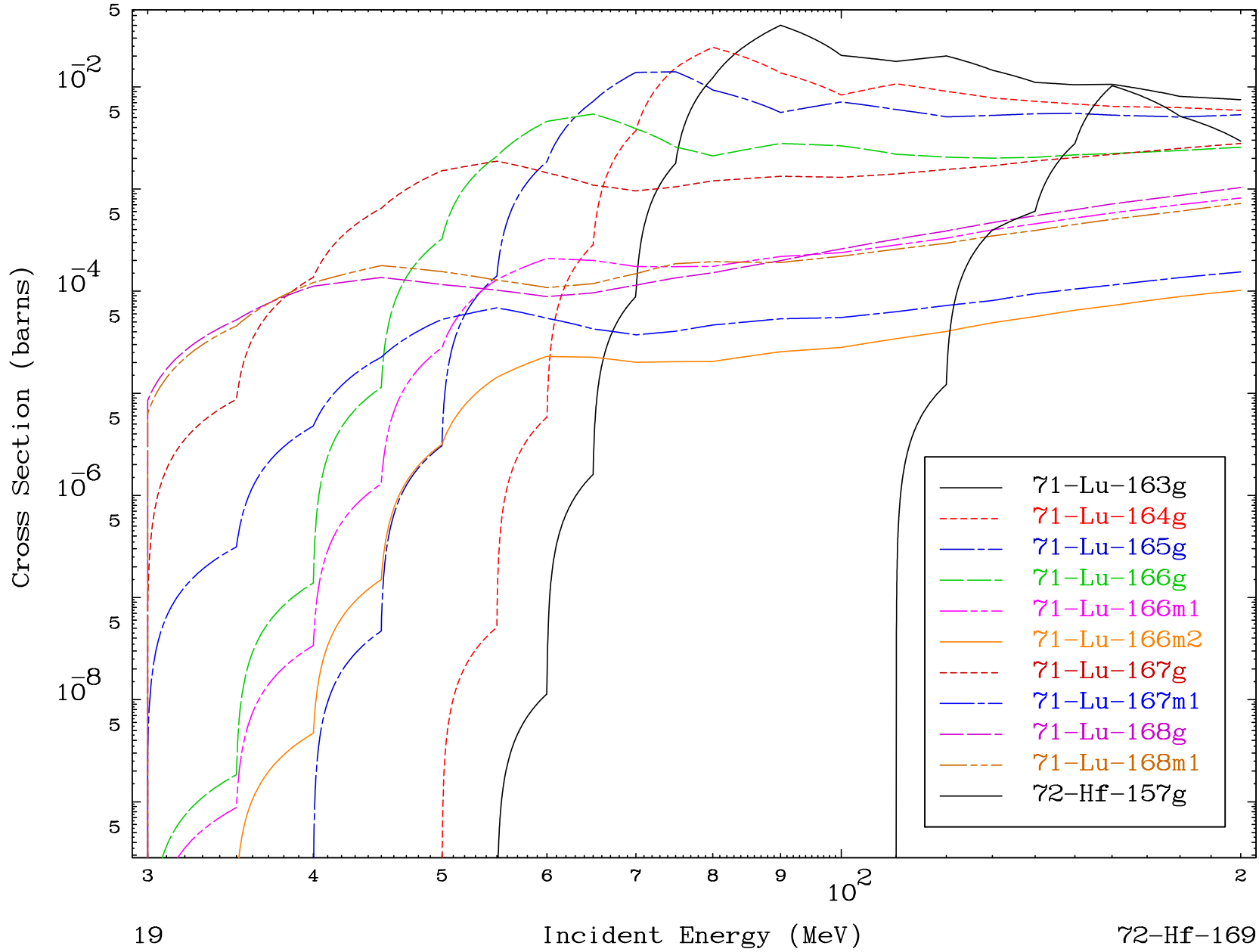


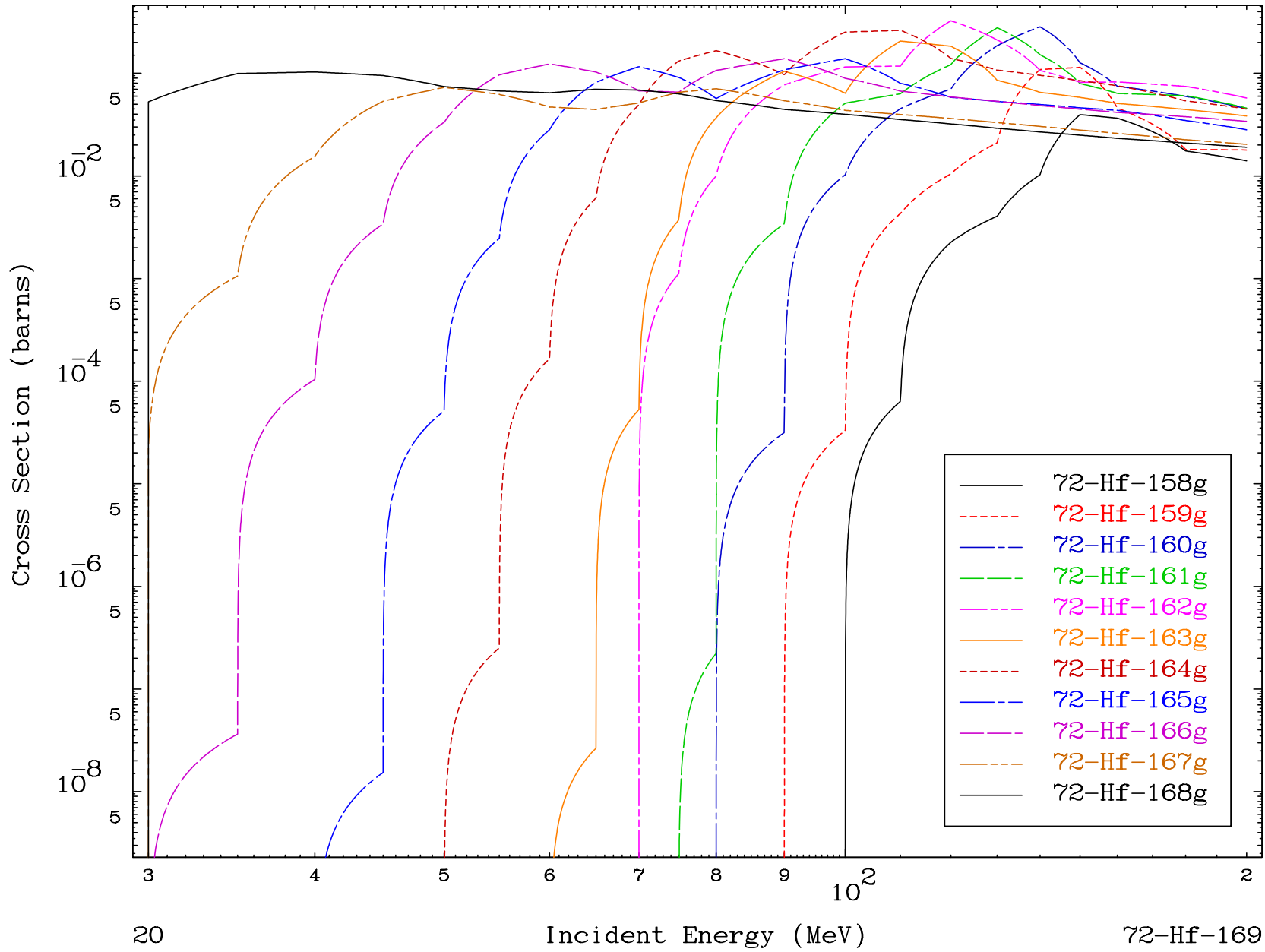
MAT 7210

( $\alpha$ , remainder)

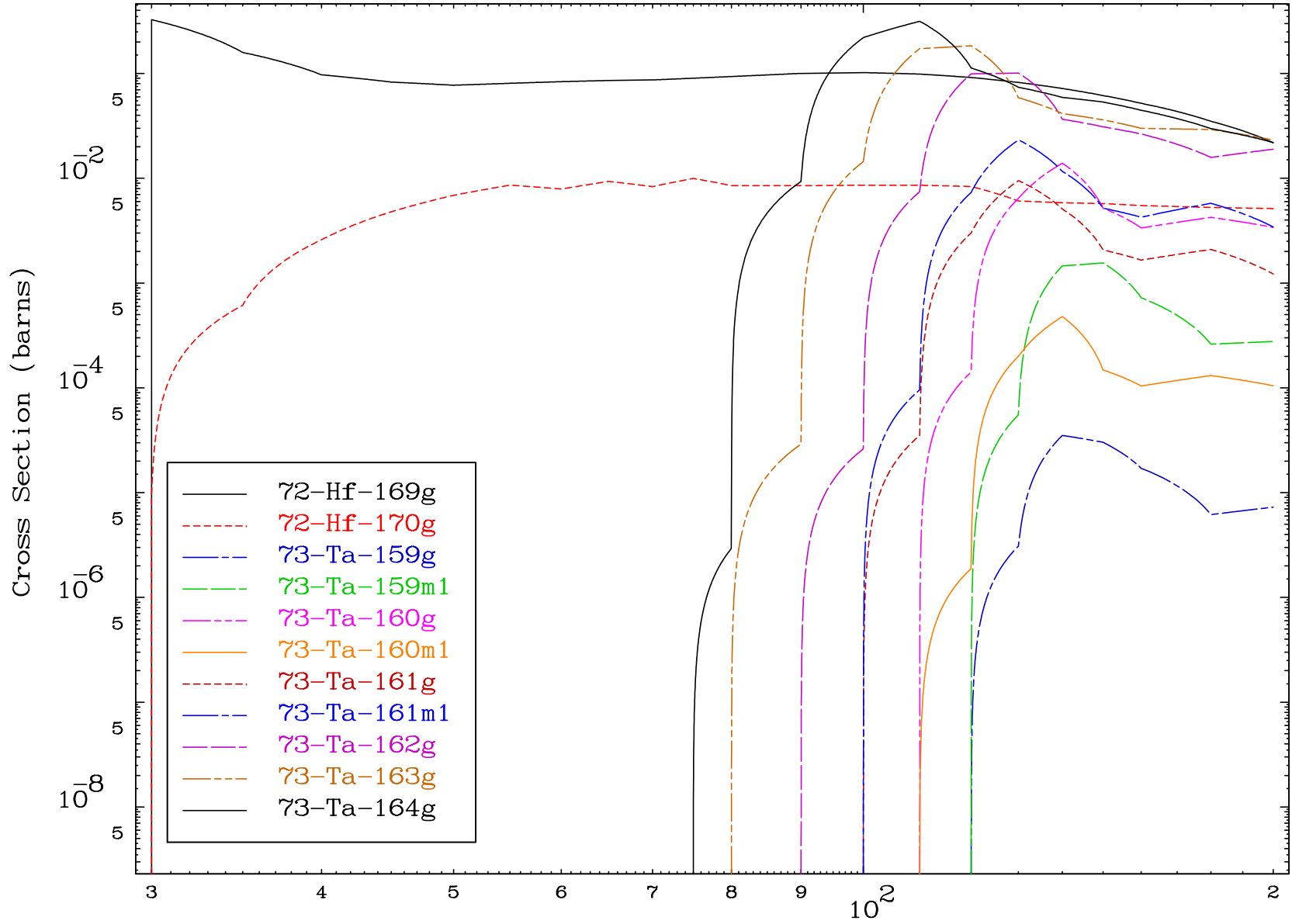
72-Hf-169

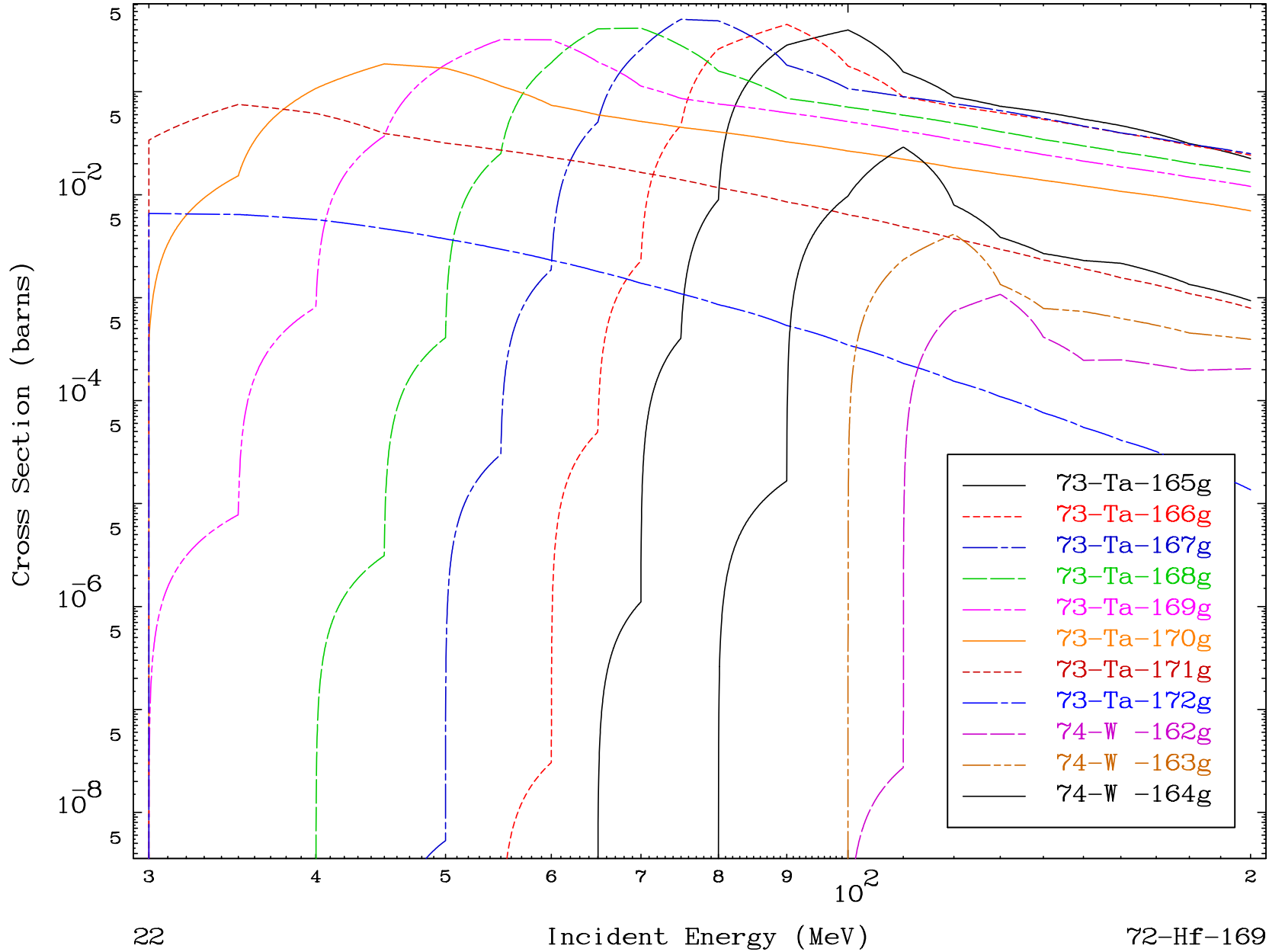
### Radionuclide Production Cross Section



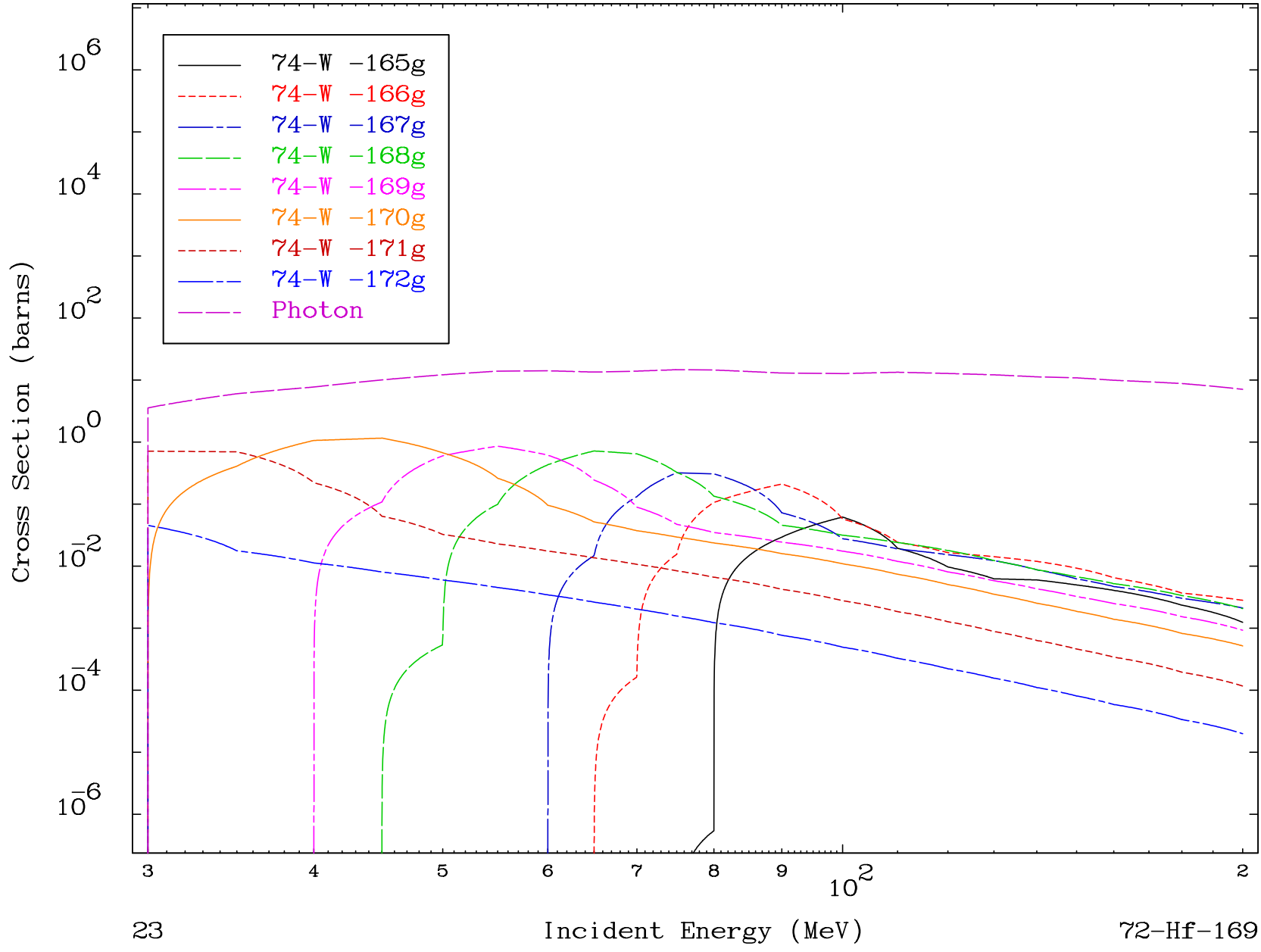


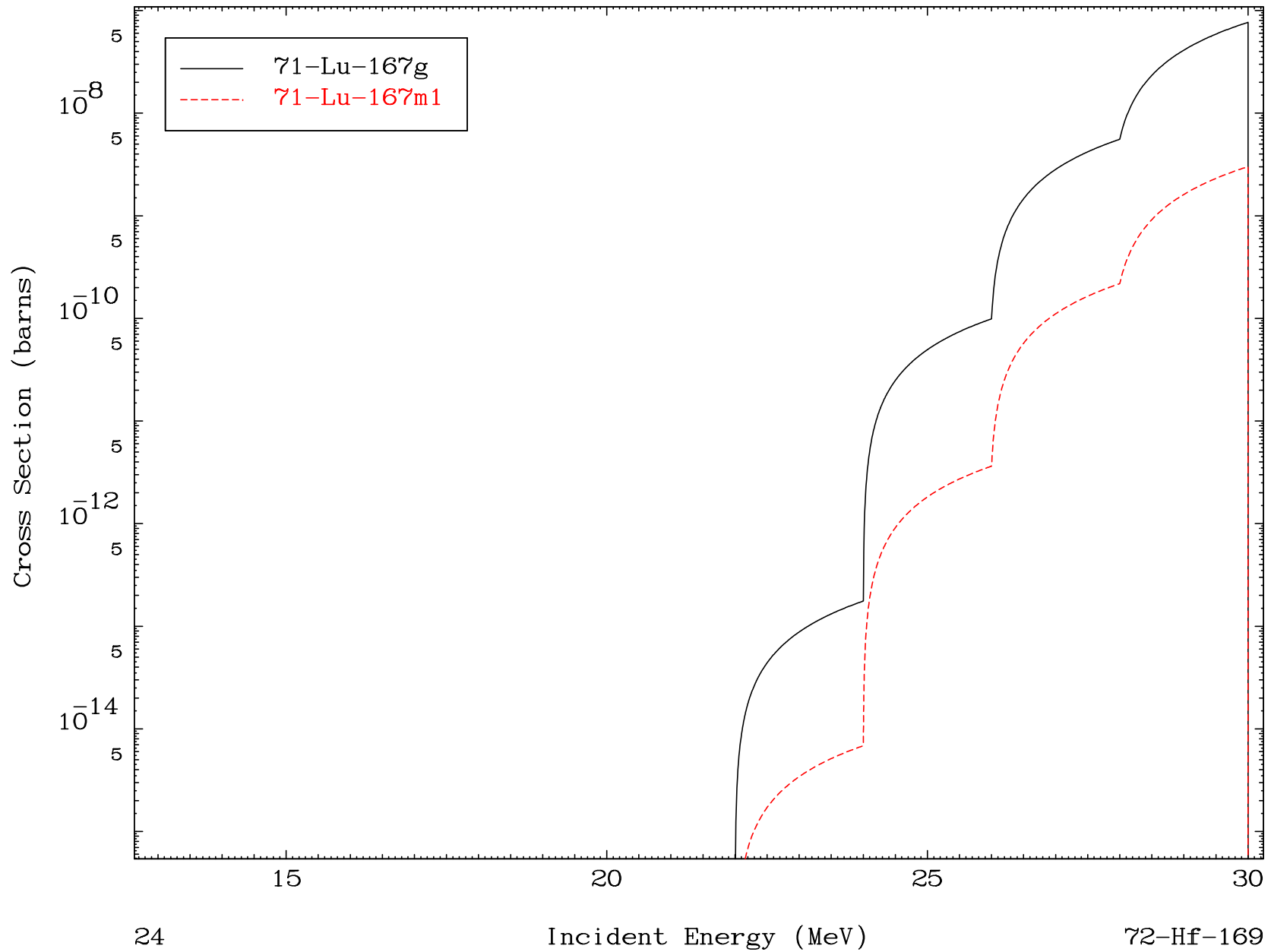
Radionuclide Production Cross Section





Radionuclide Production Cross Section





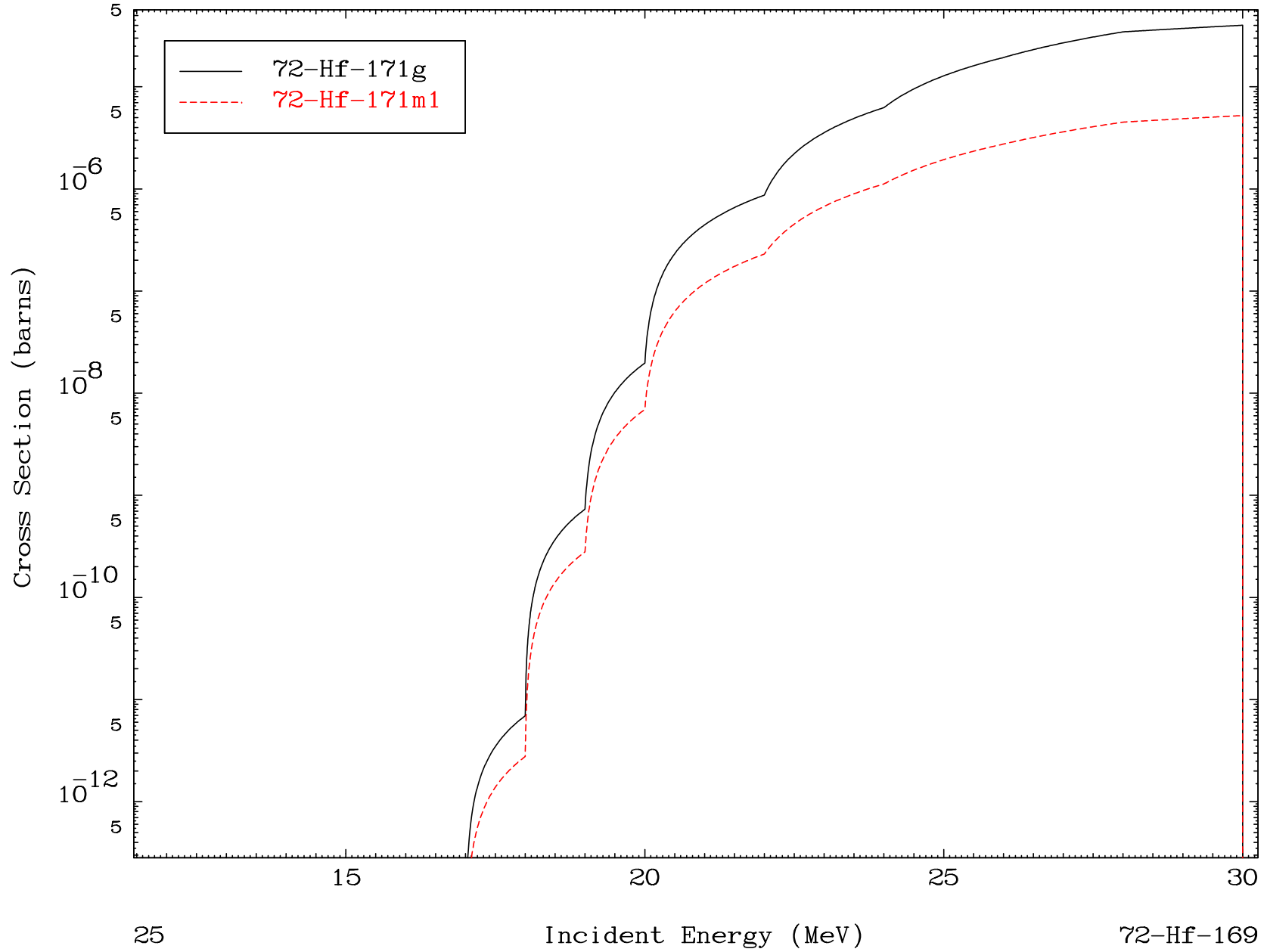


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( $\alpha, 2p$ )

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

