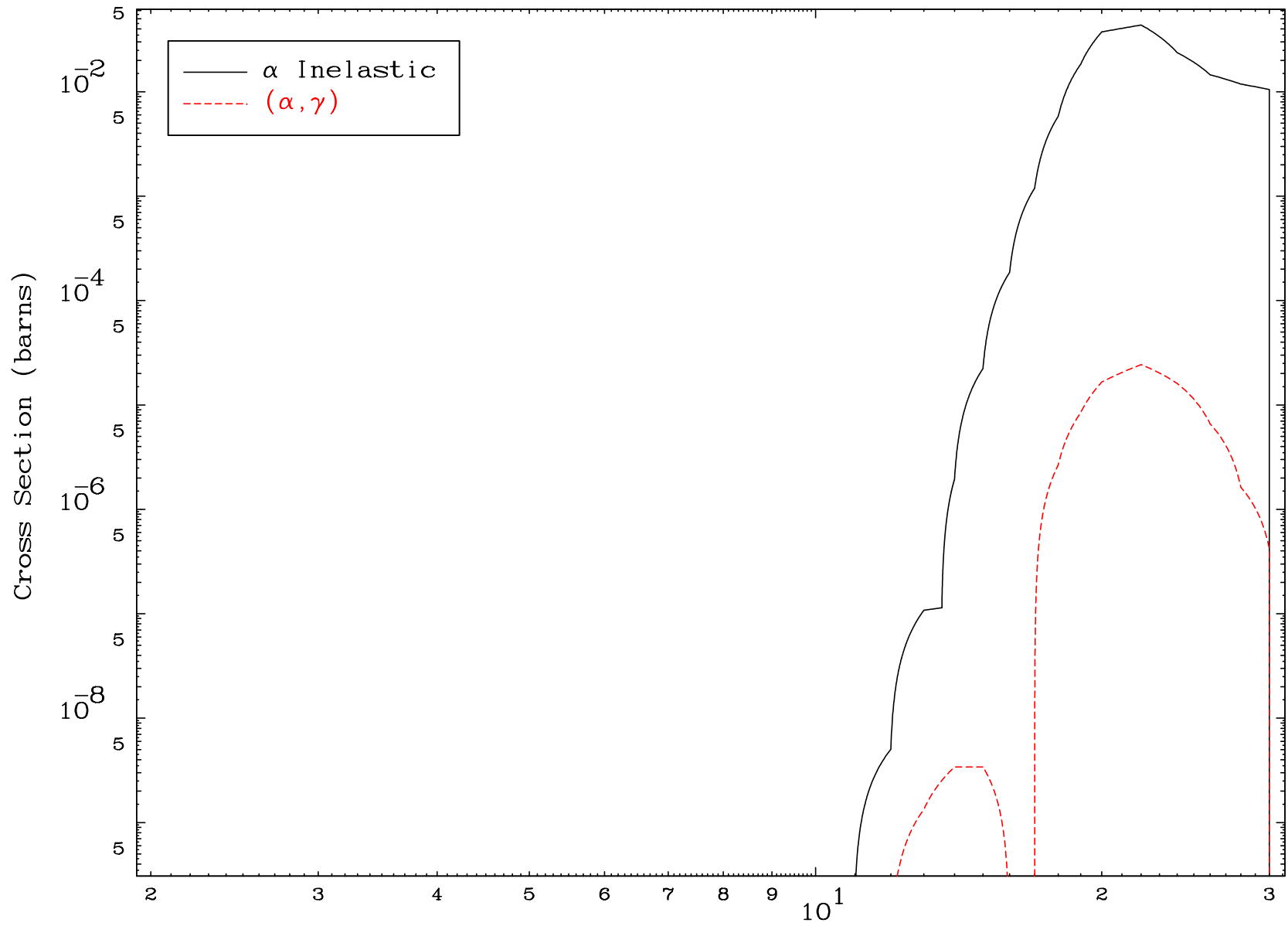


MAT 8037

$\alpha$  Major  
0 Kelvin Cross Sections

80-Hg-200



1

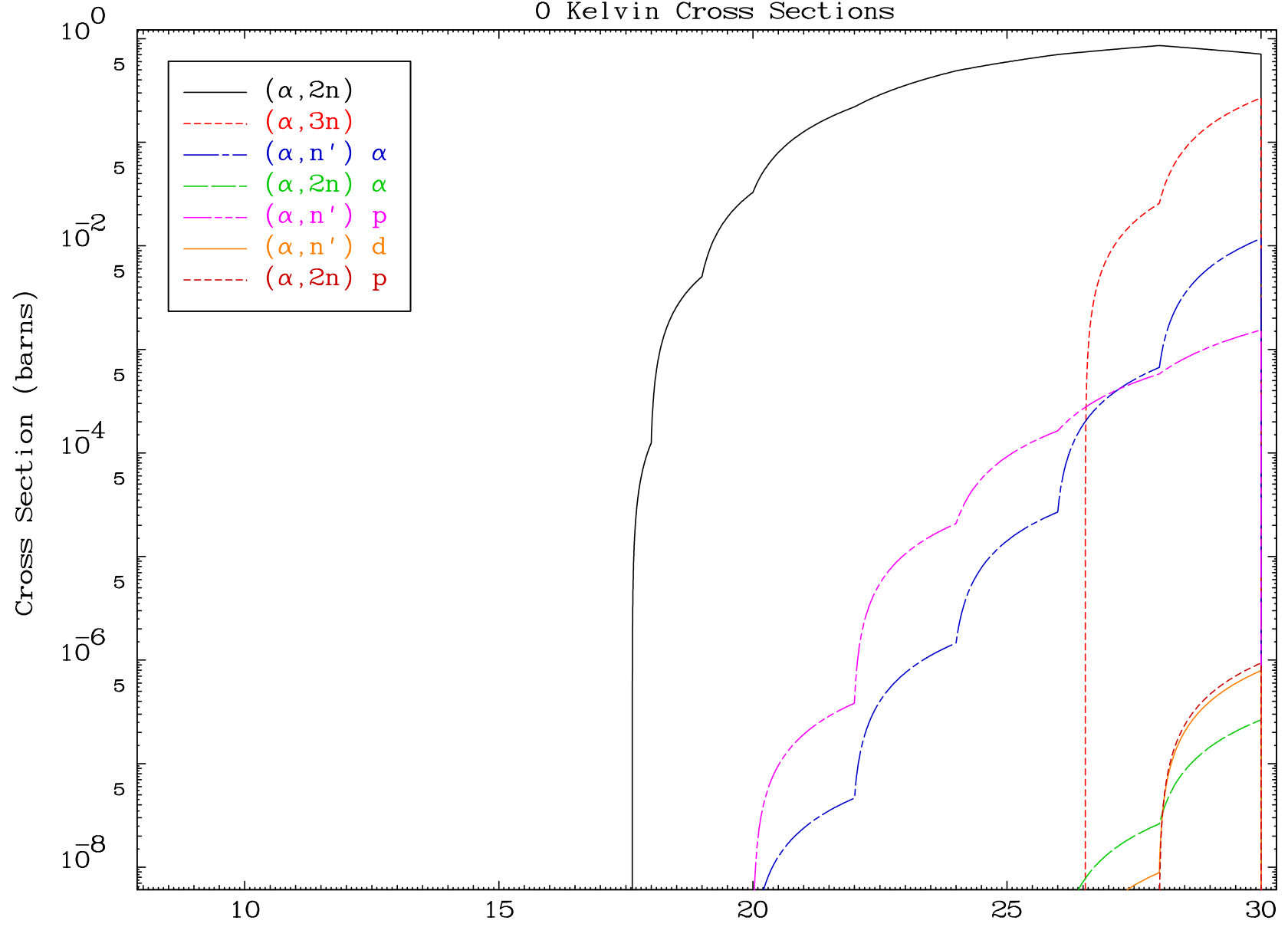
Incident Energy (MeV)

80-Hg-200

MAT 8037

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

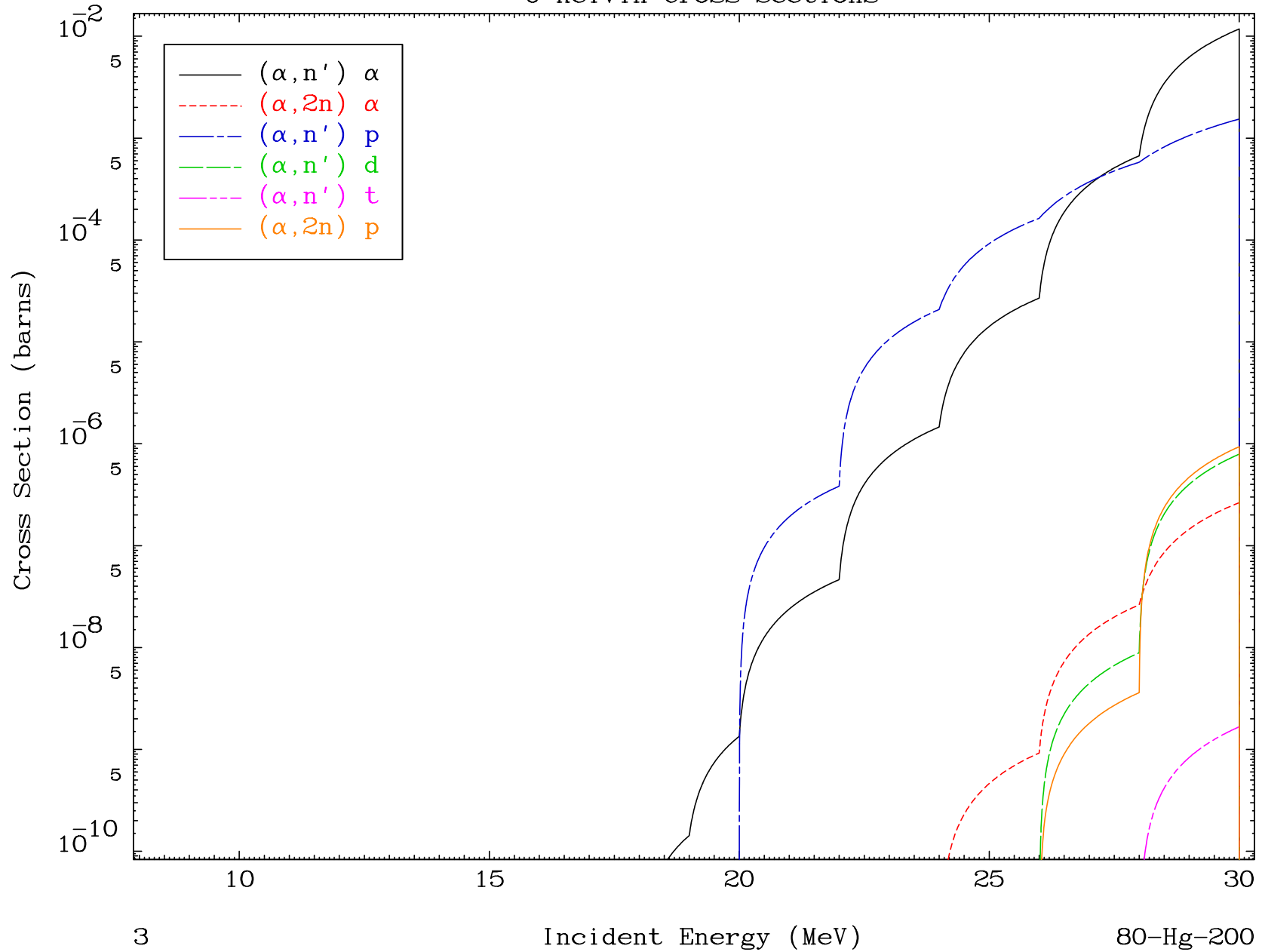
80-Hg-200

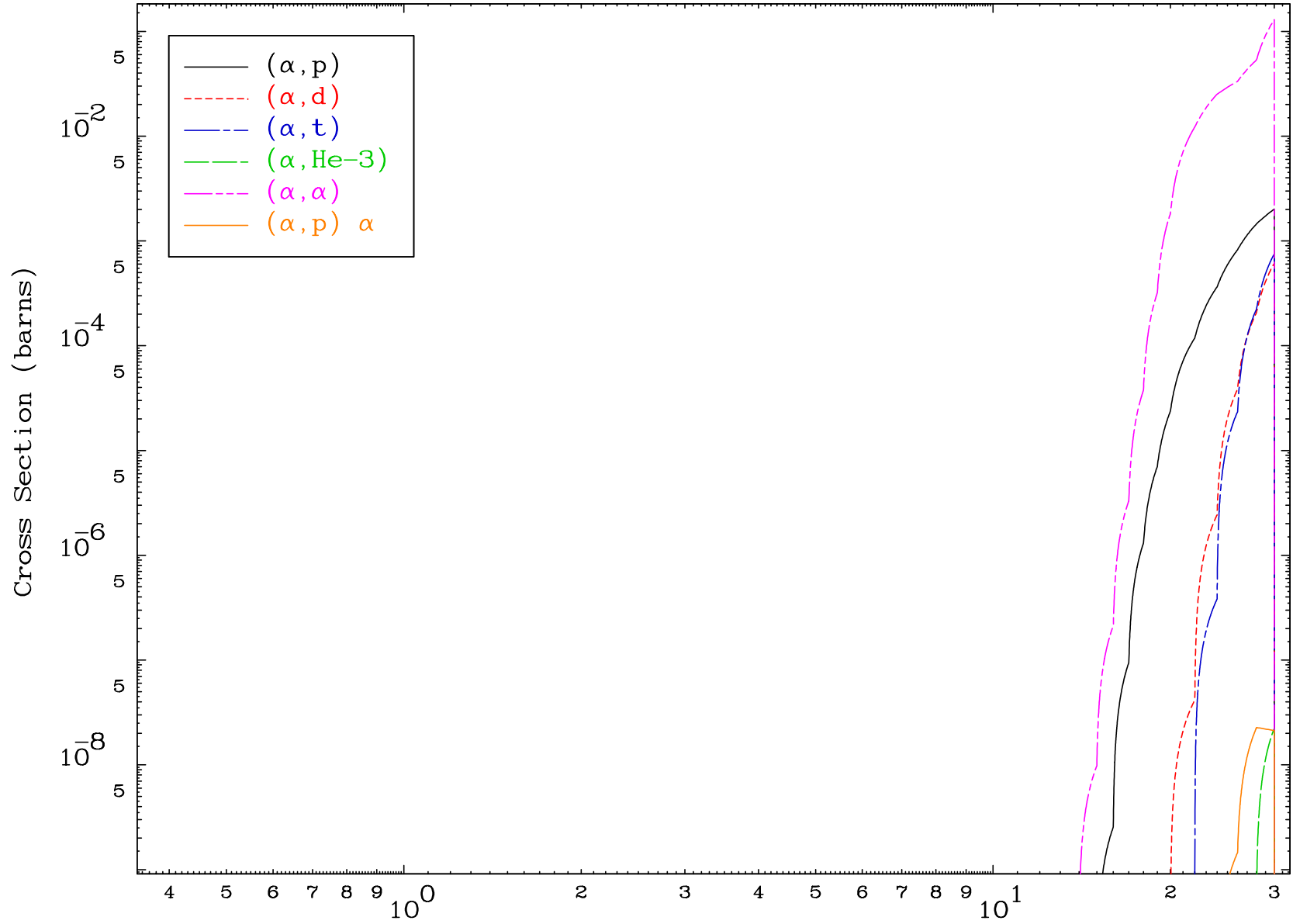


2

Incident Energy (MeV)

80-Hg-200

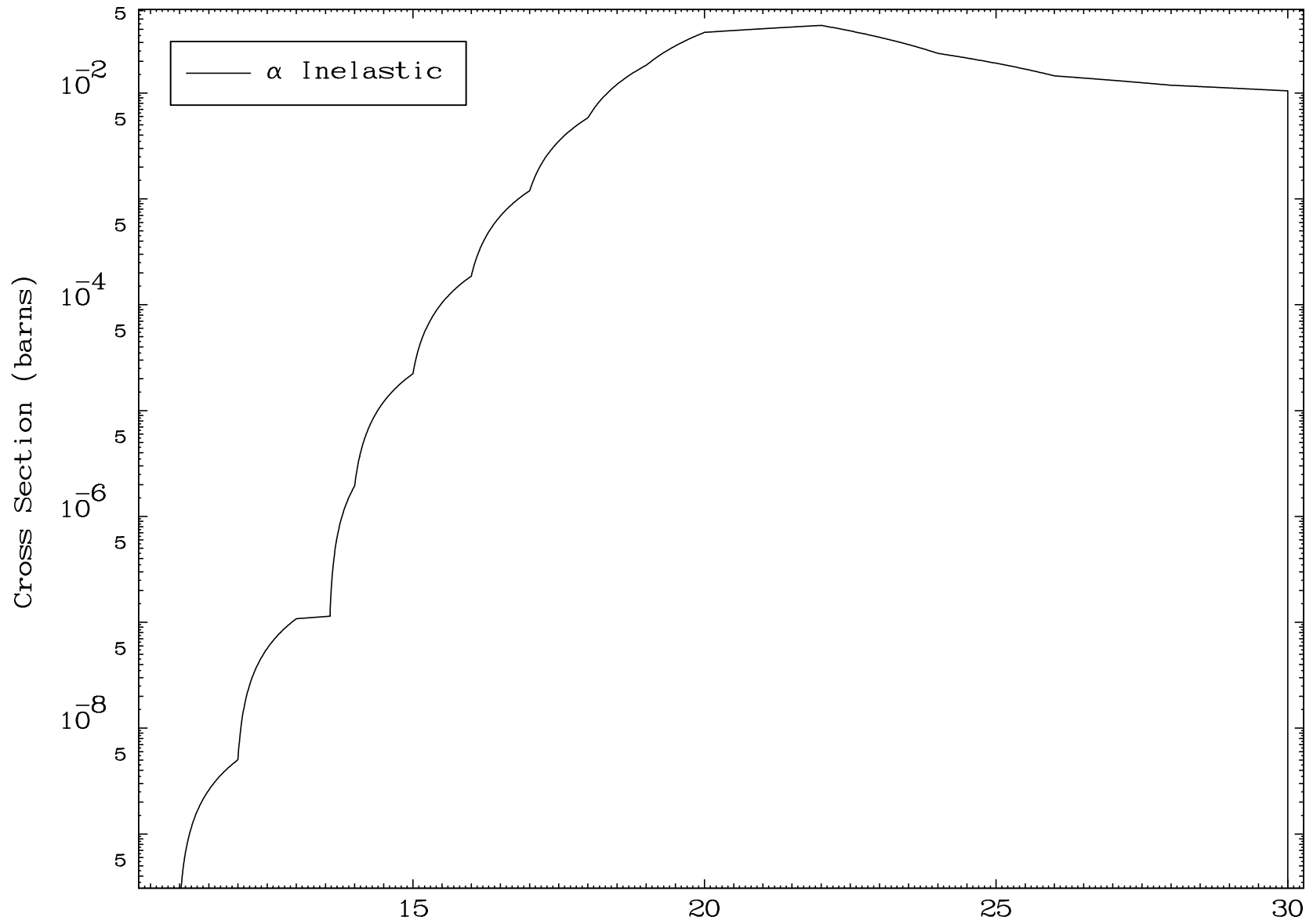




MAT 8037

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

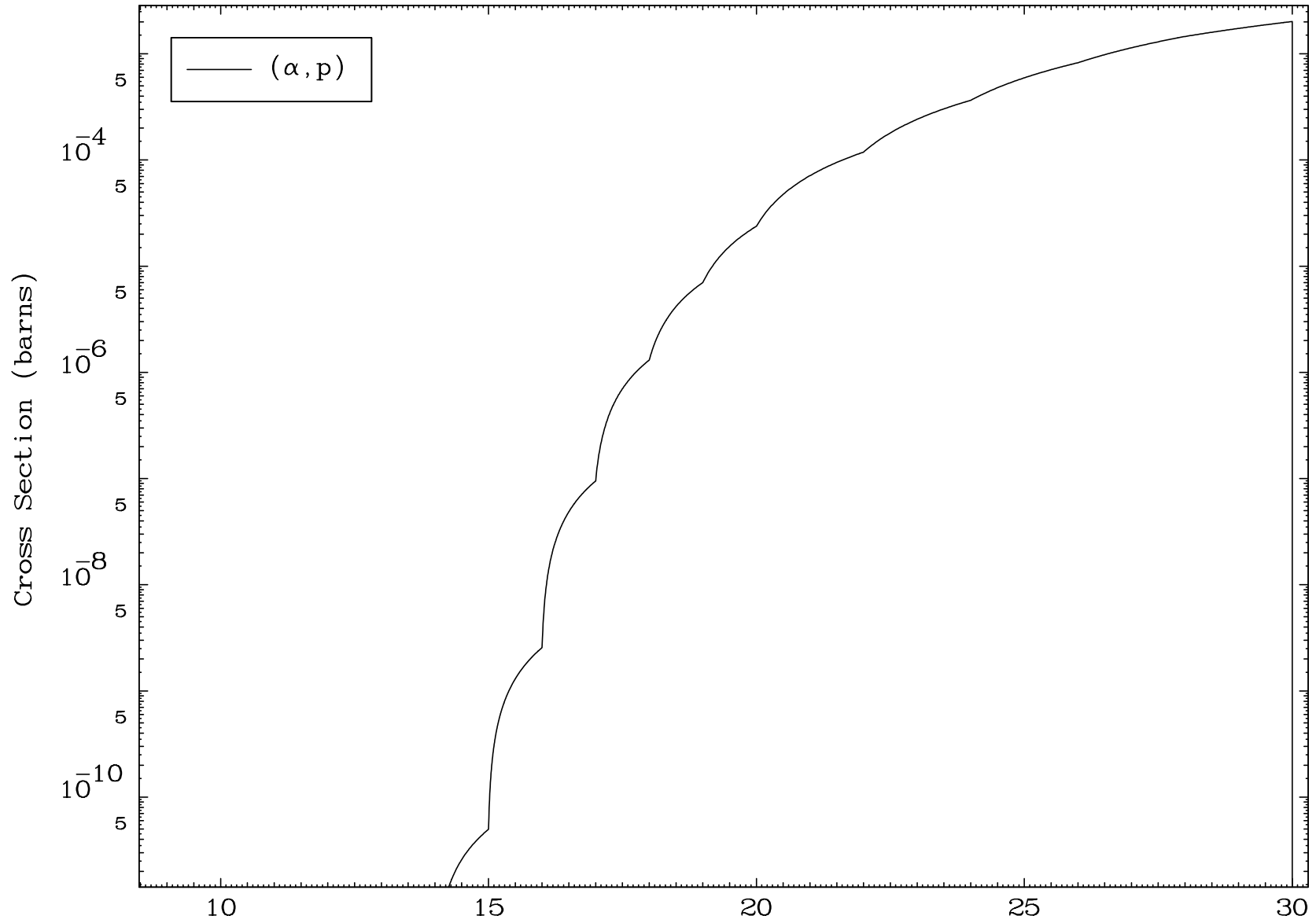
80-Hg-200

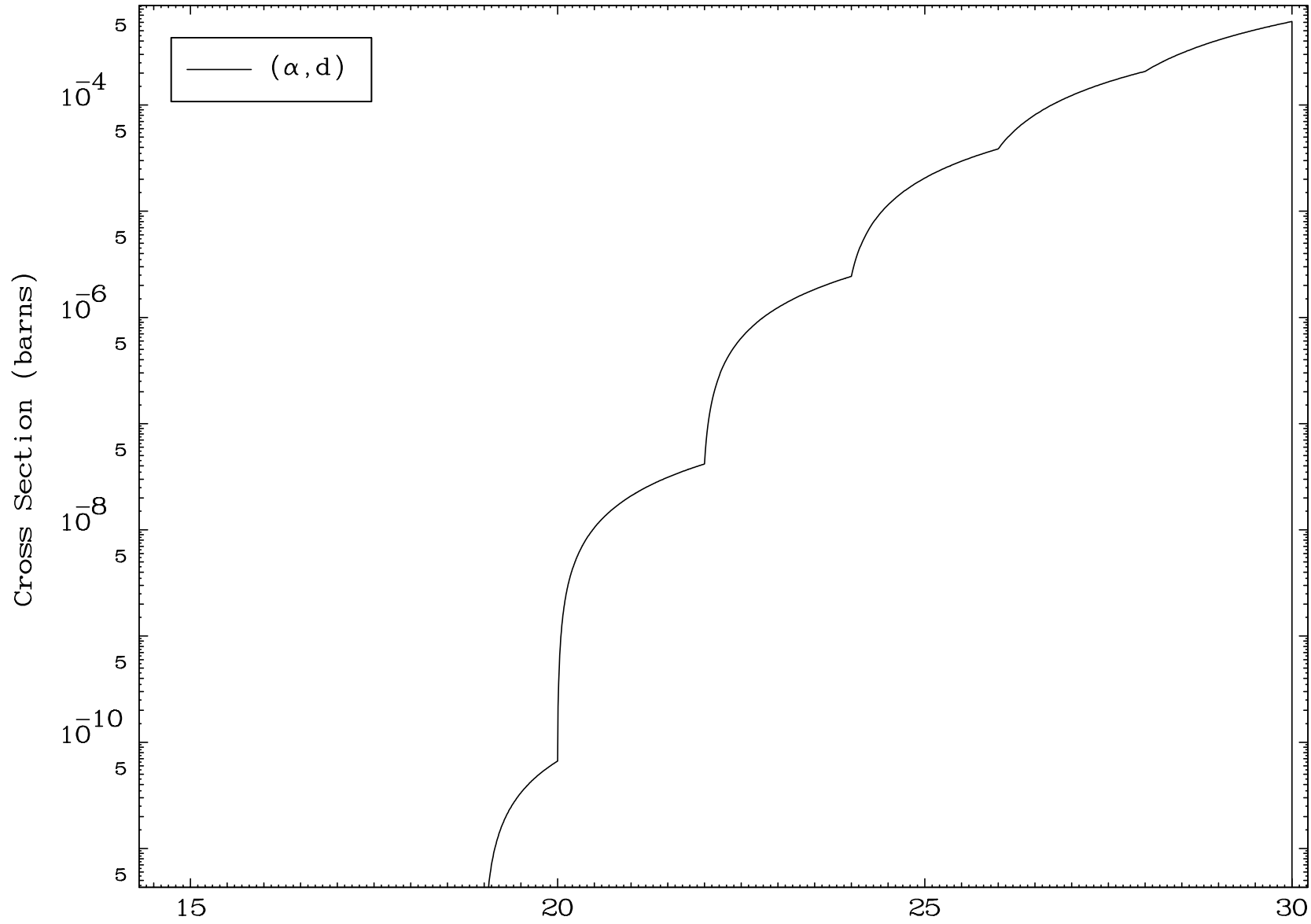


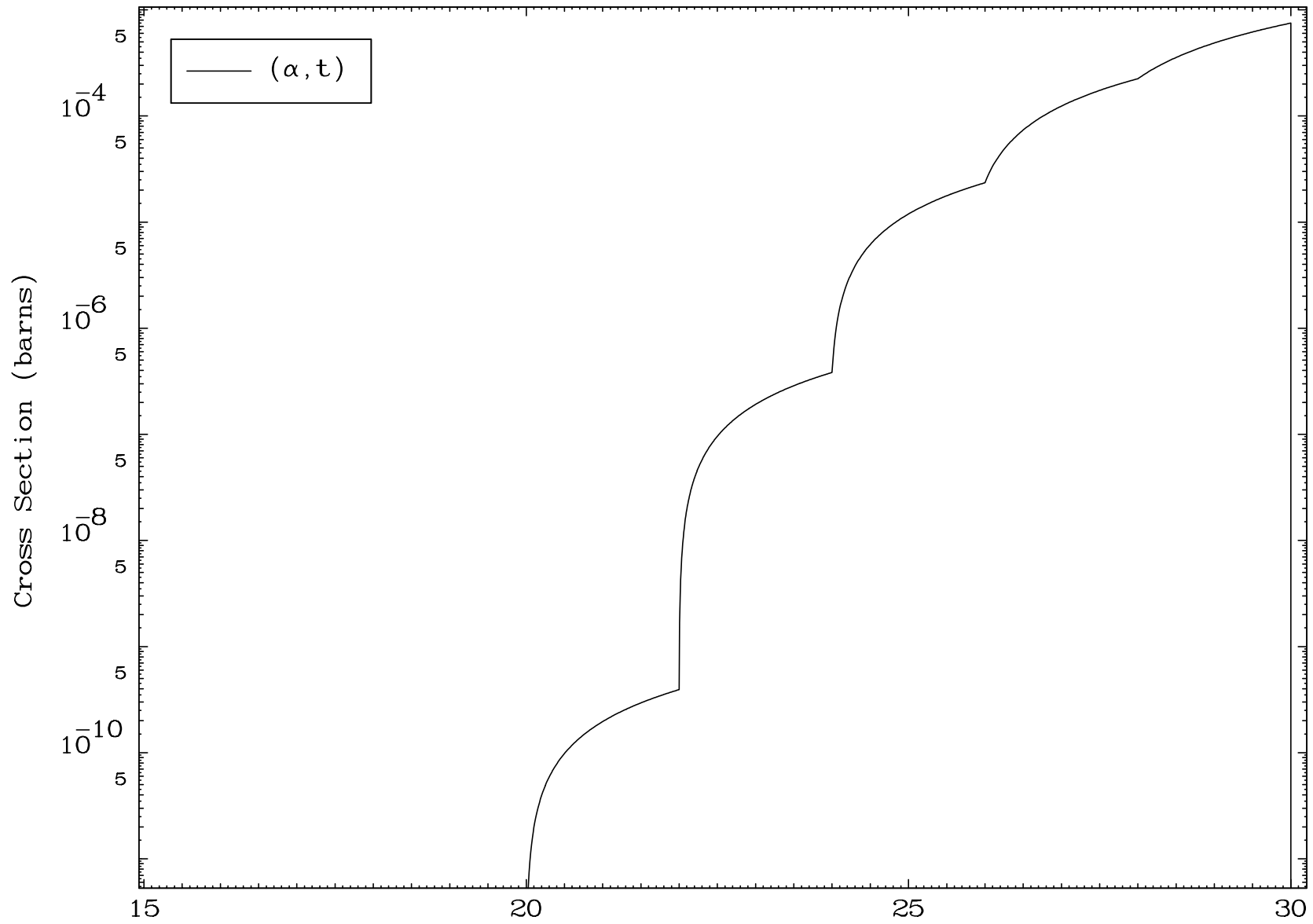
5

Incident Energy (MeV)

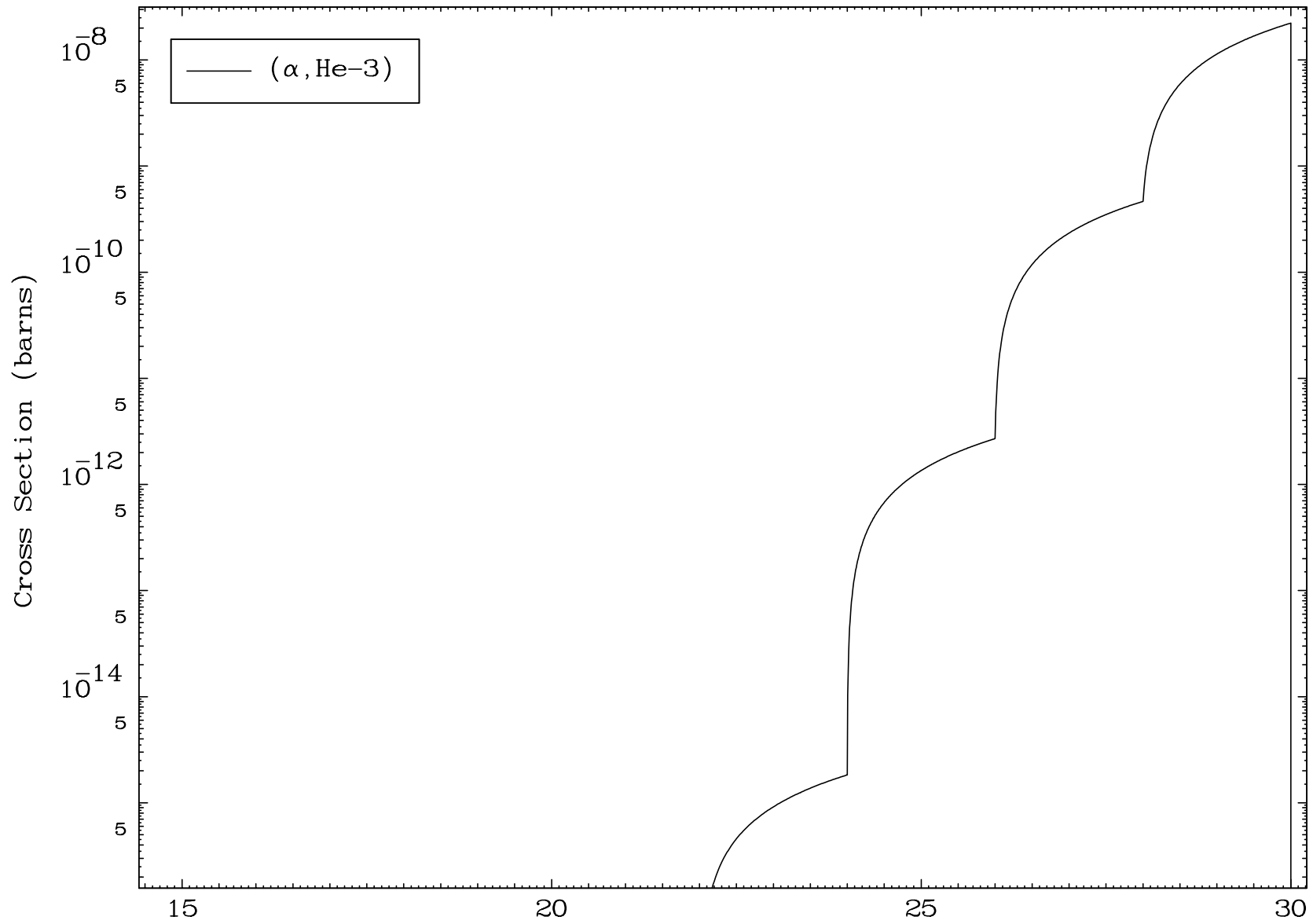
80-Hg-200







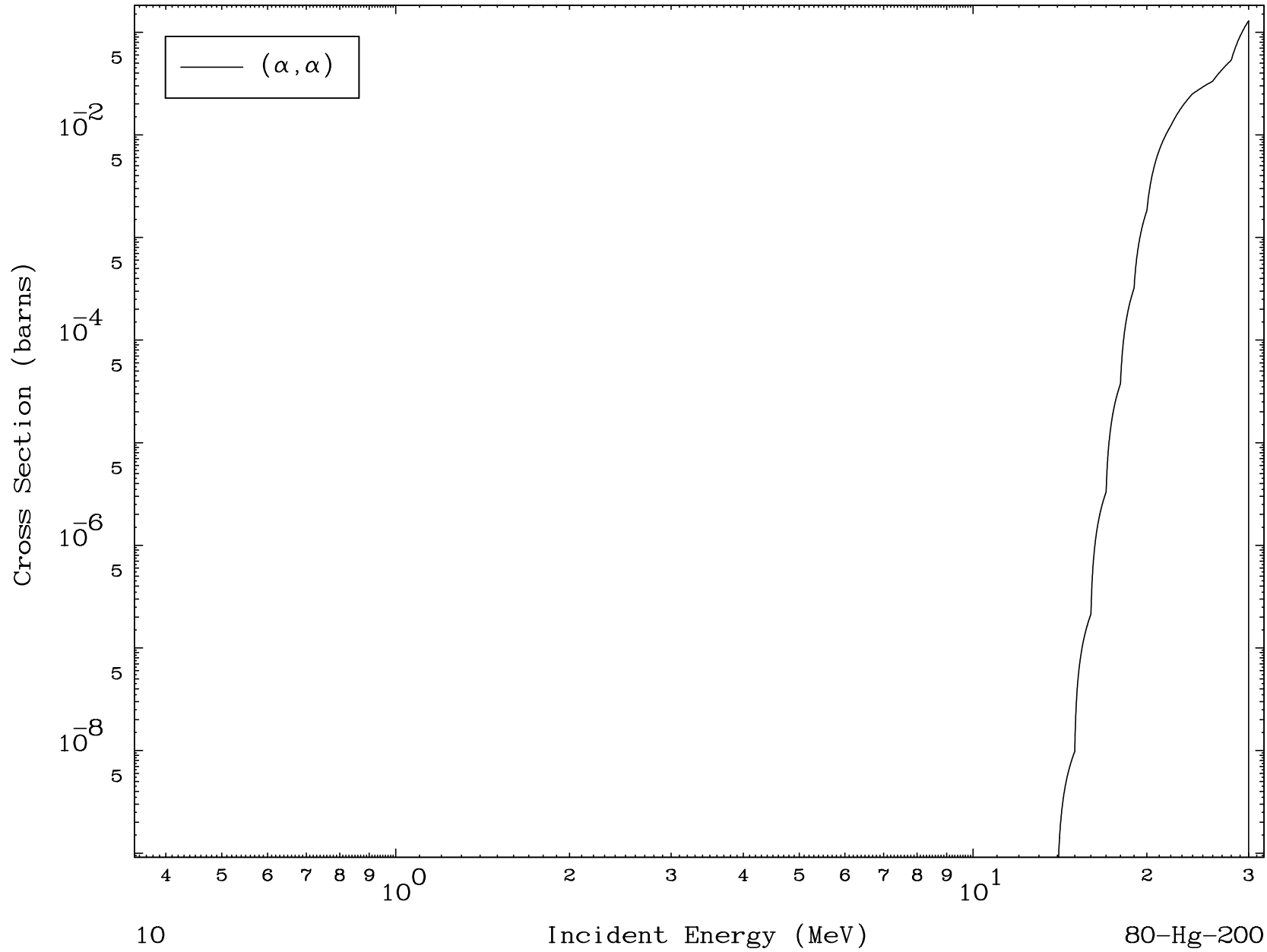


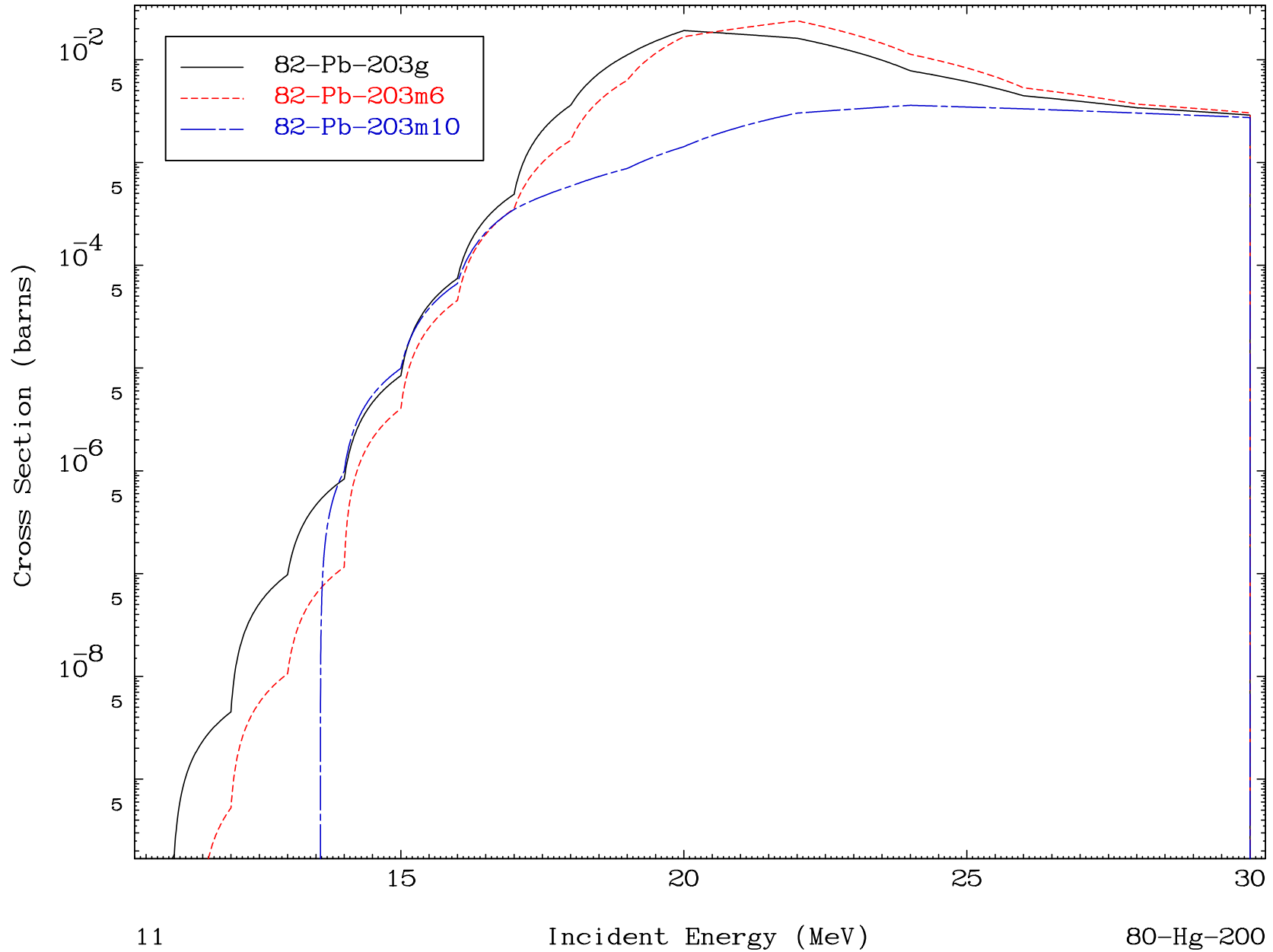


MAT 8037

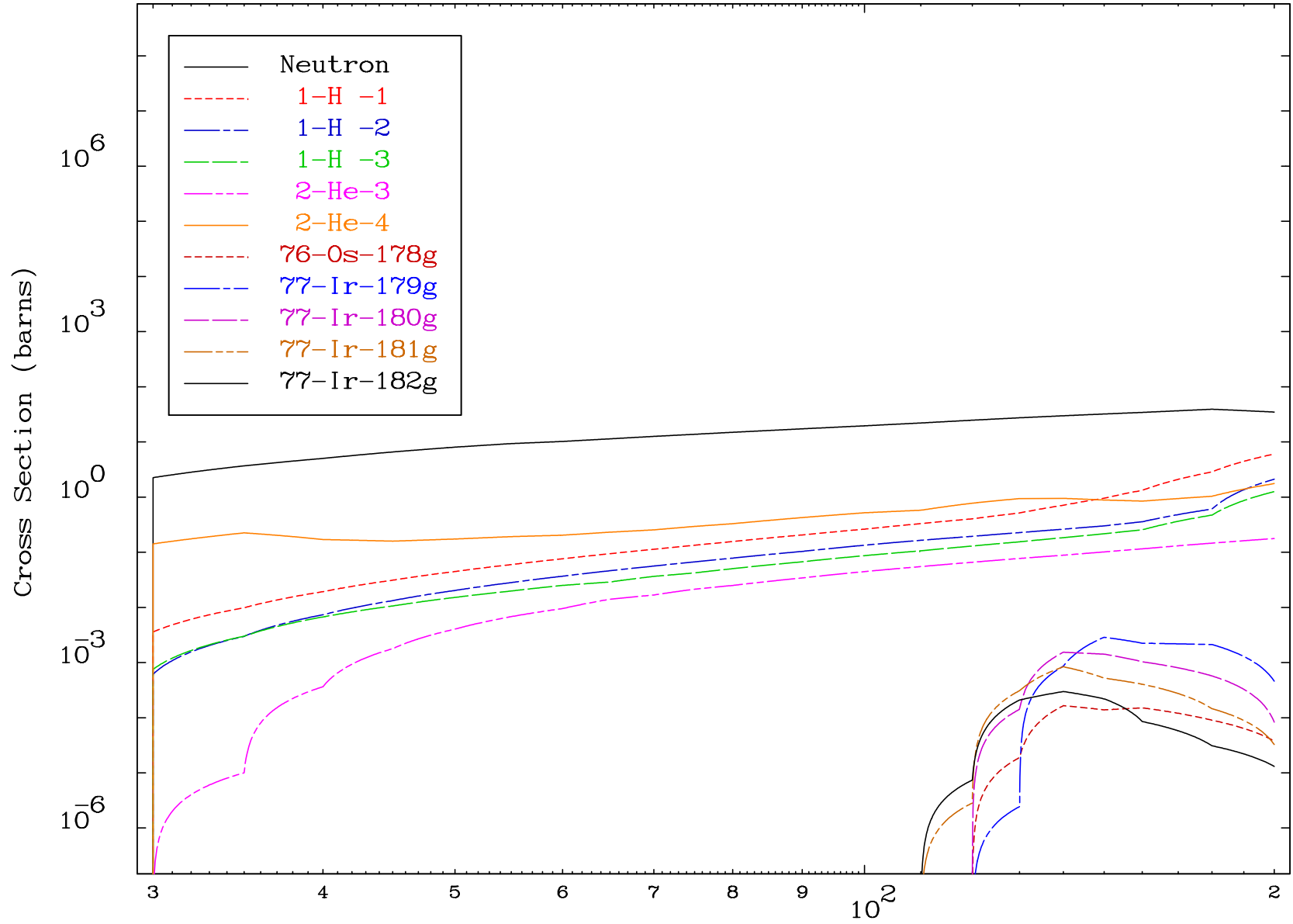
( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

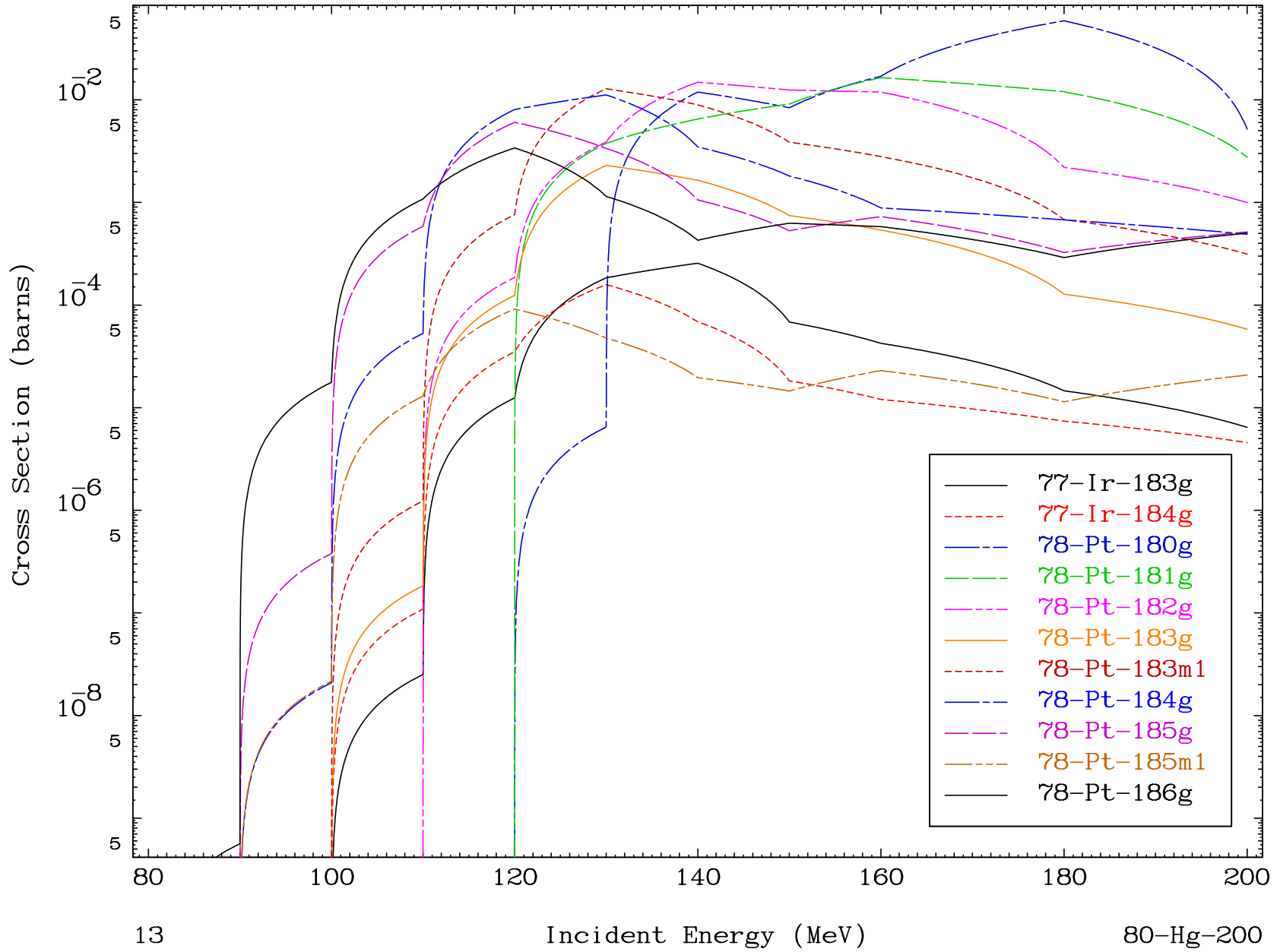
80-Hg-200



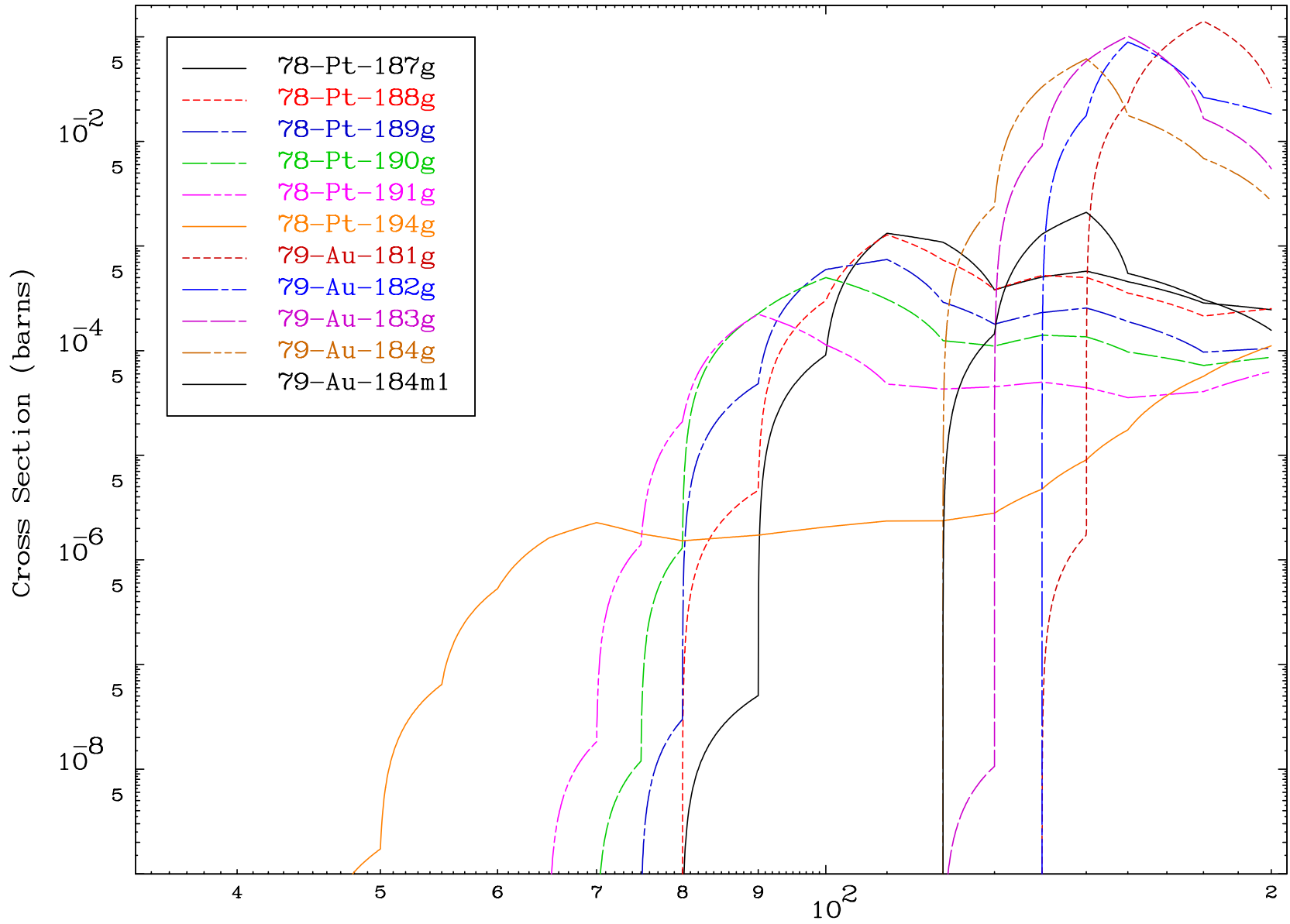


Radionuclide Production Cross Section





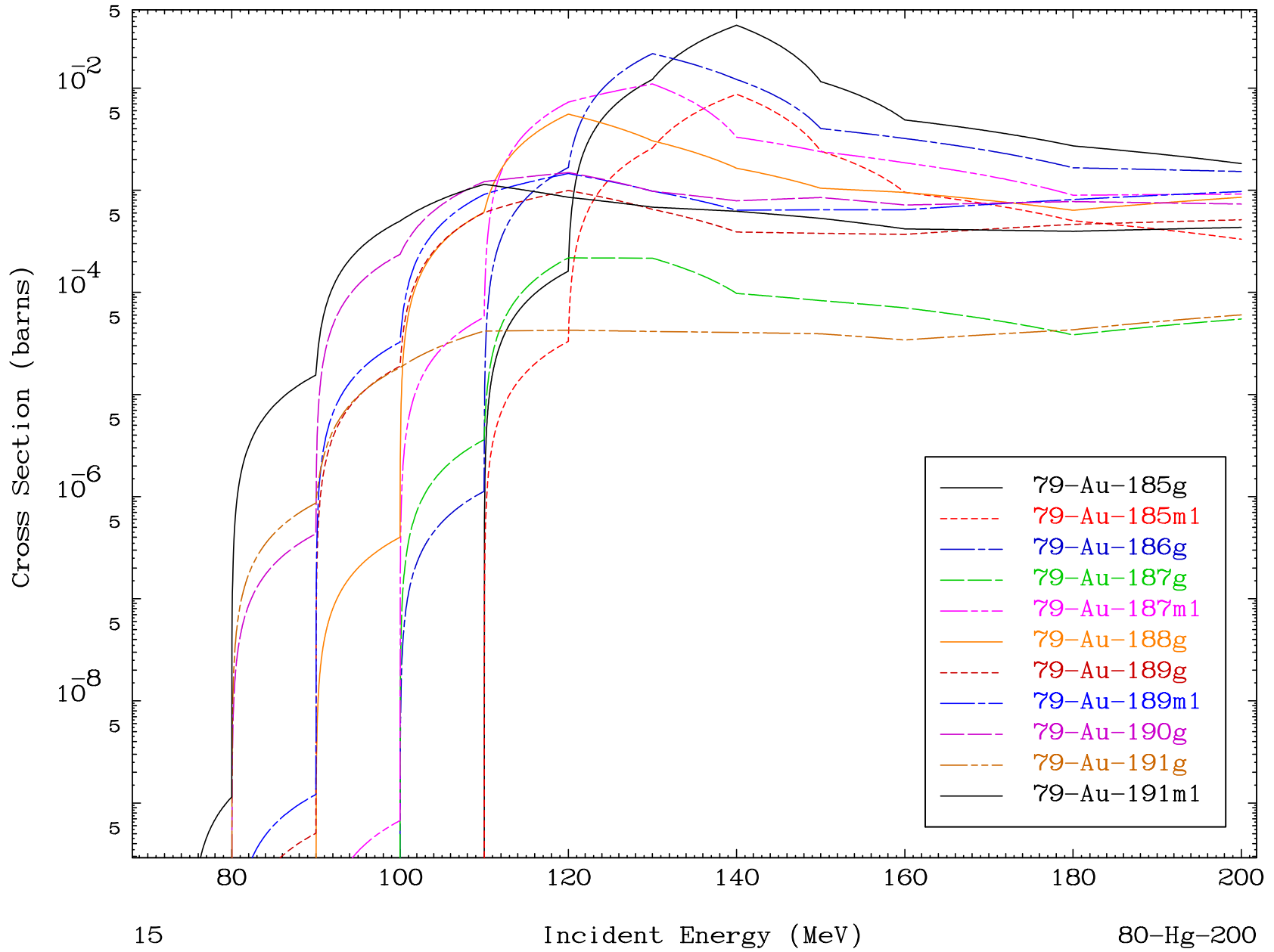
Radionuclide Production Cross Section

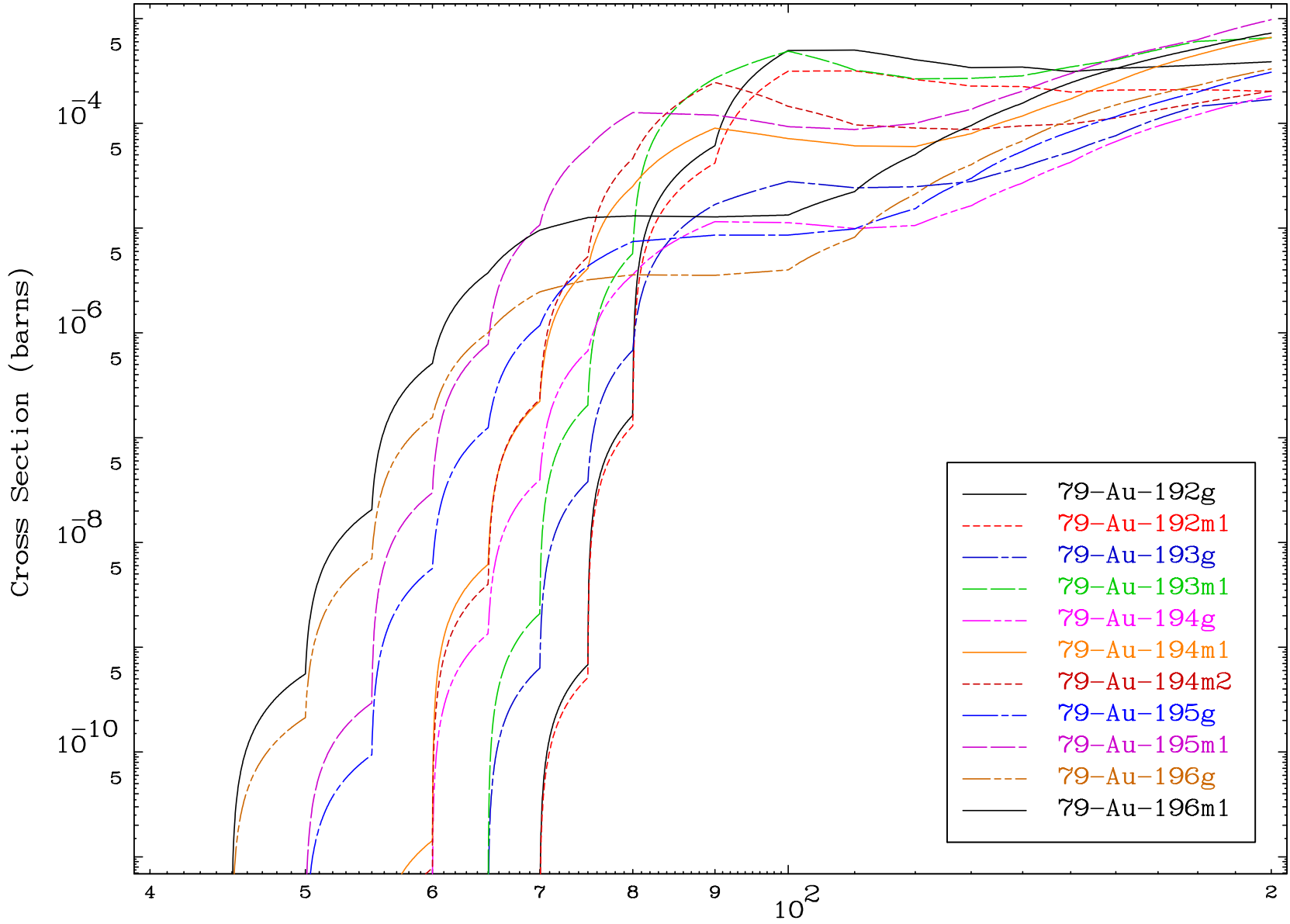


MAT 8037

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

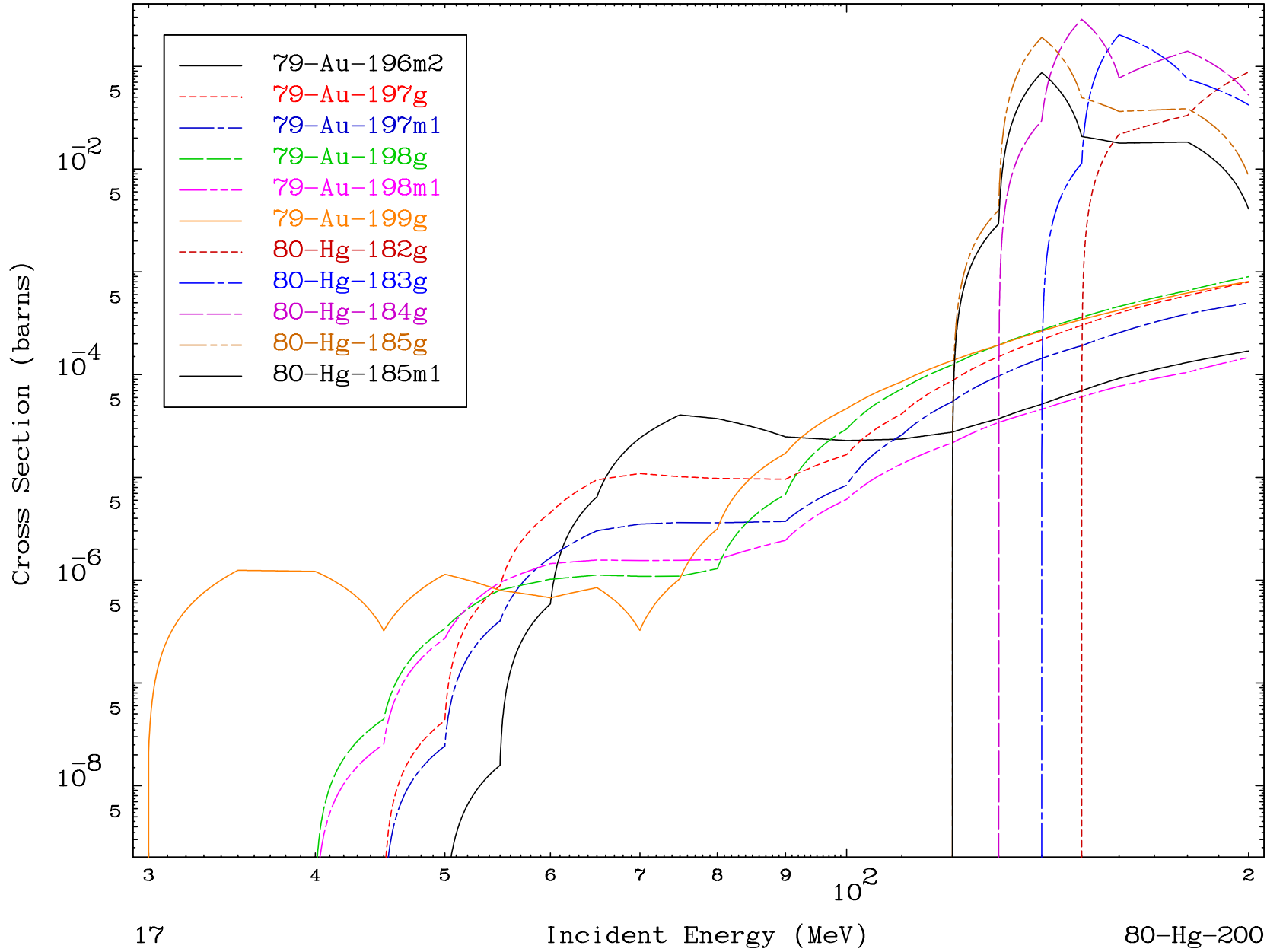
80-Hg-200

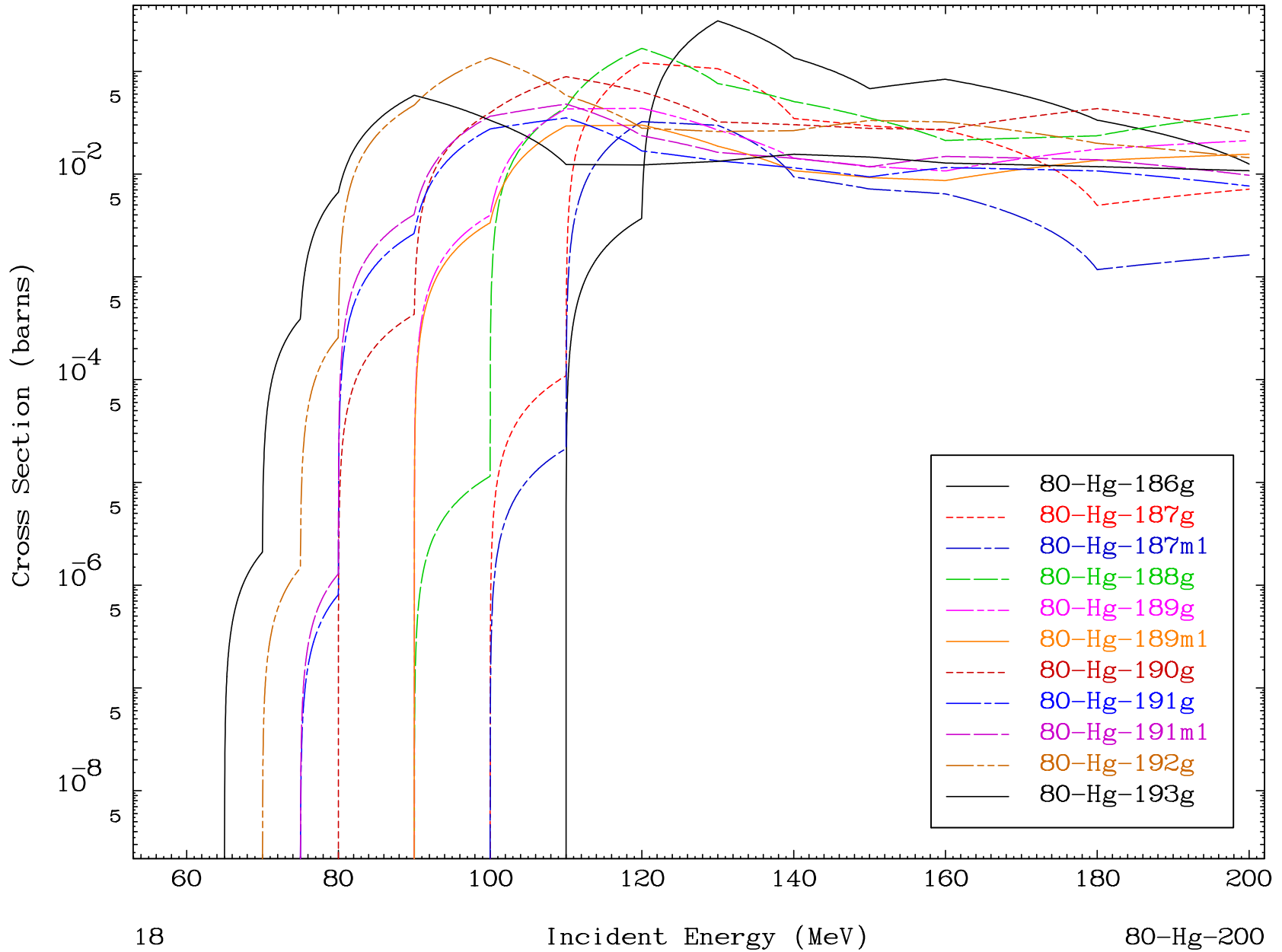


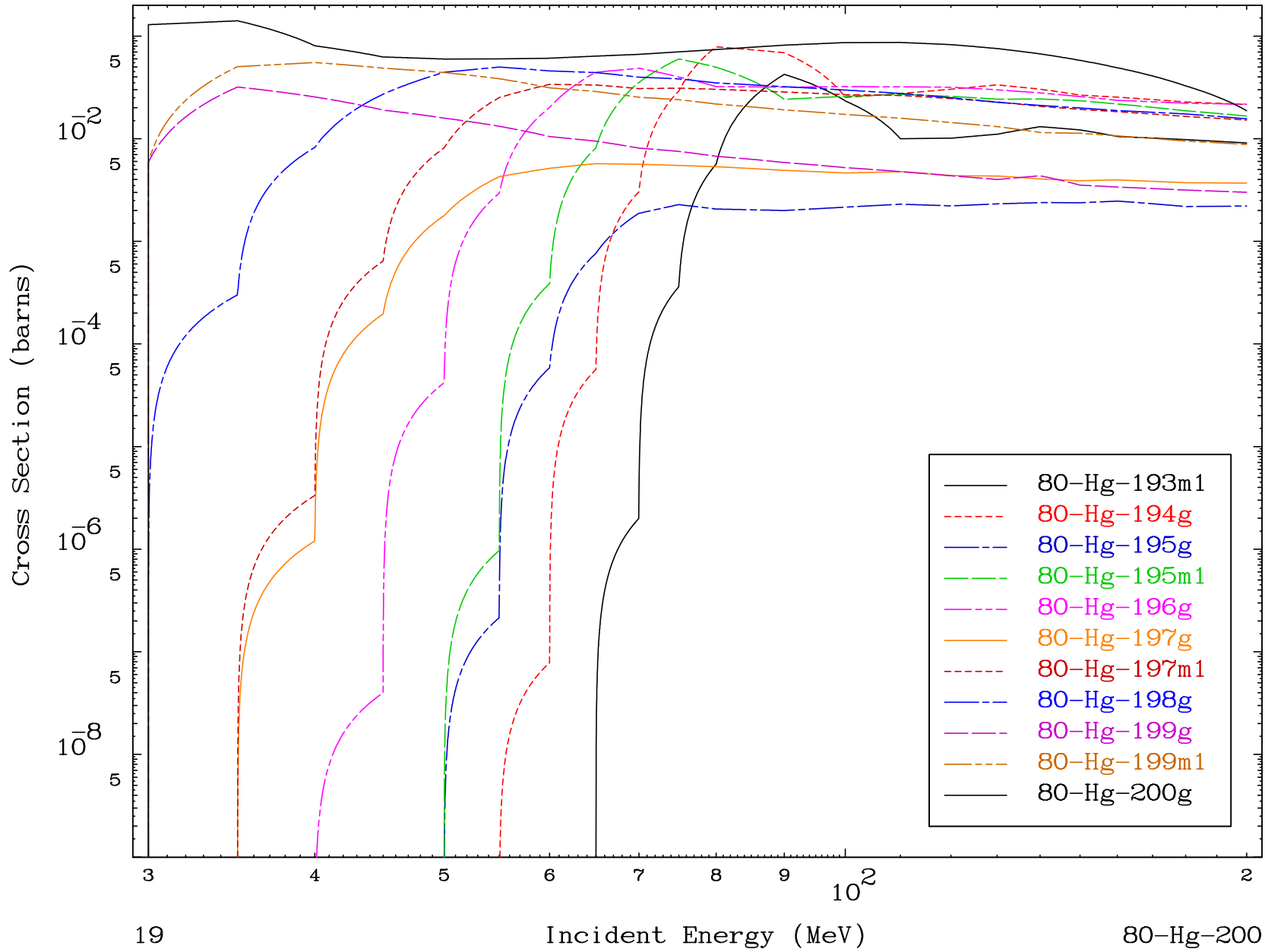




Radionuclide Production Cross Section





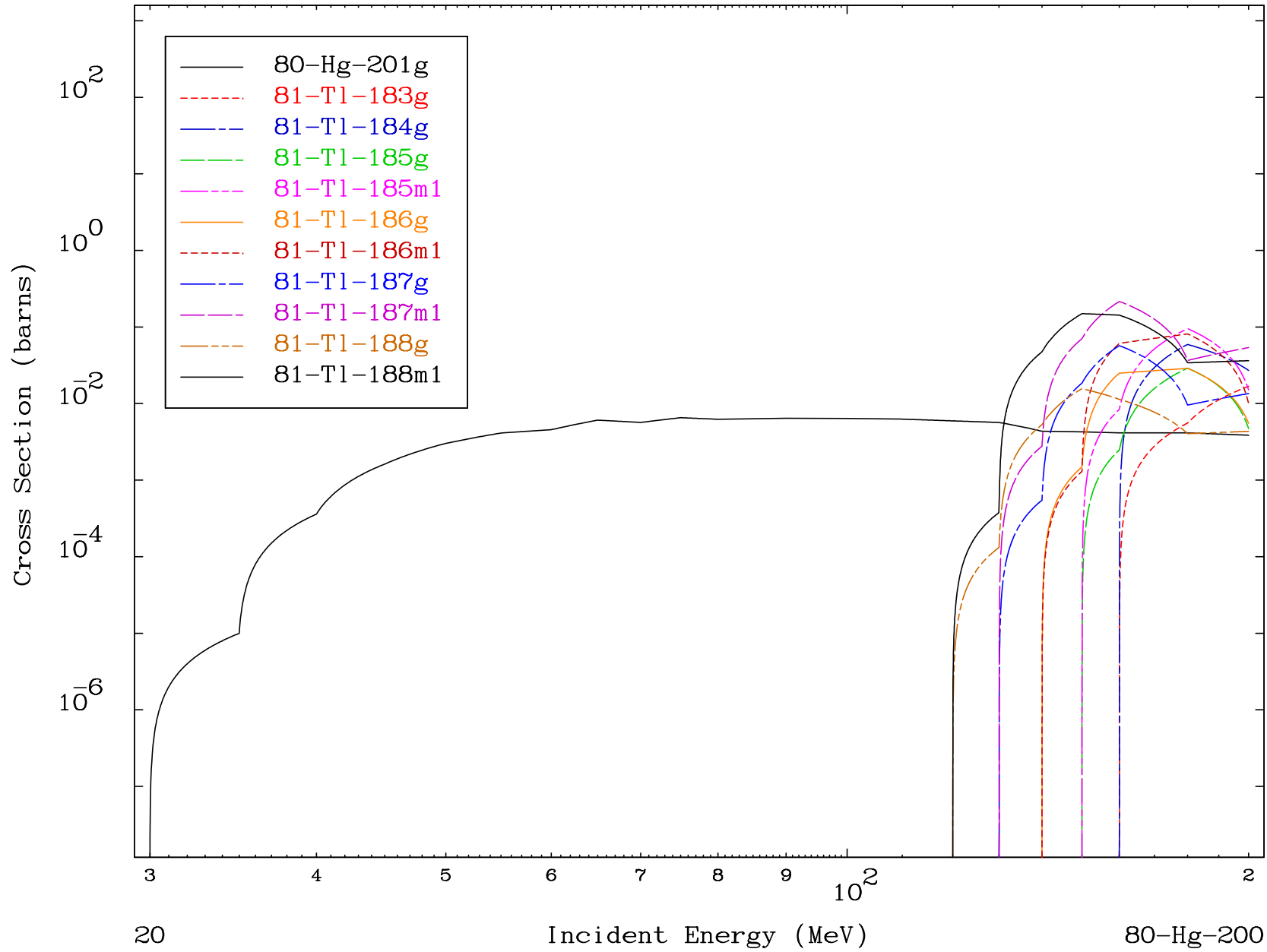


MAT 8037

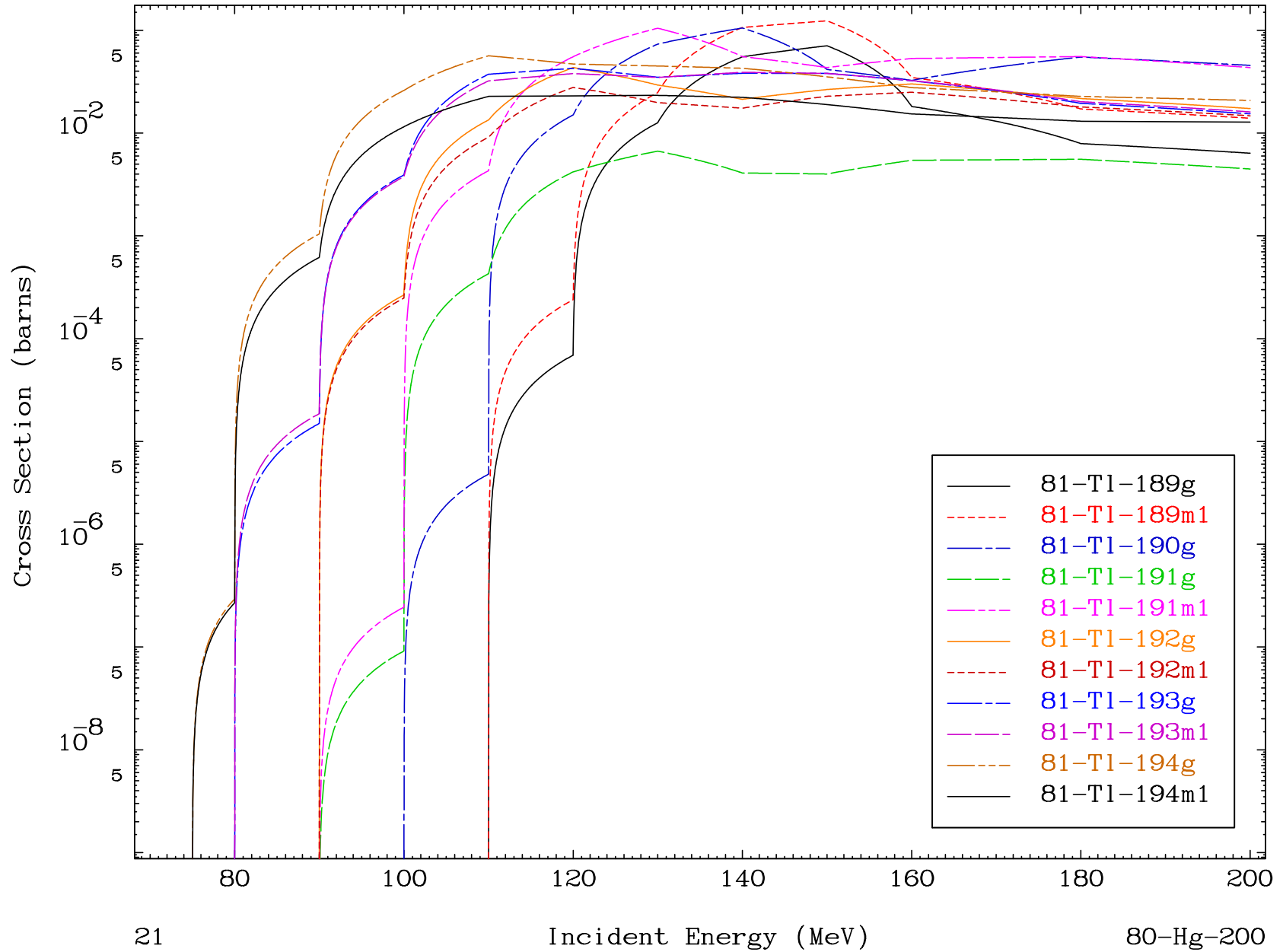
( $\alpha$ , remainder)

80-Hg-200

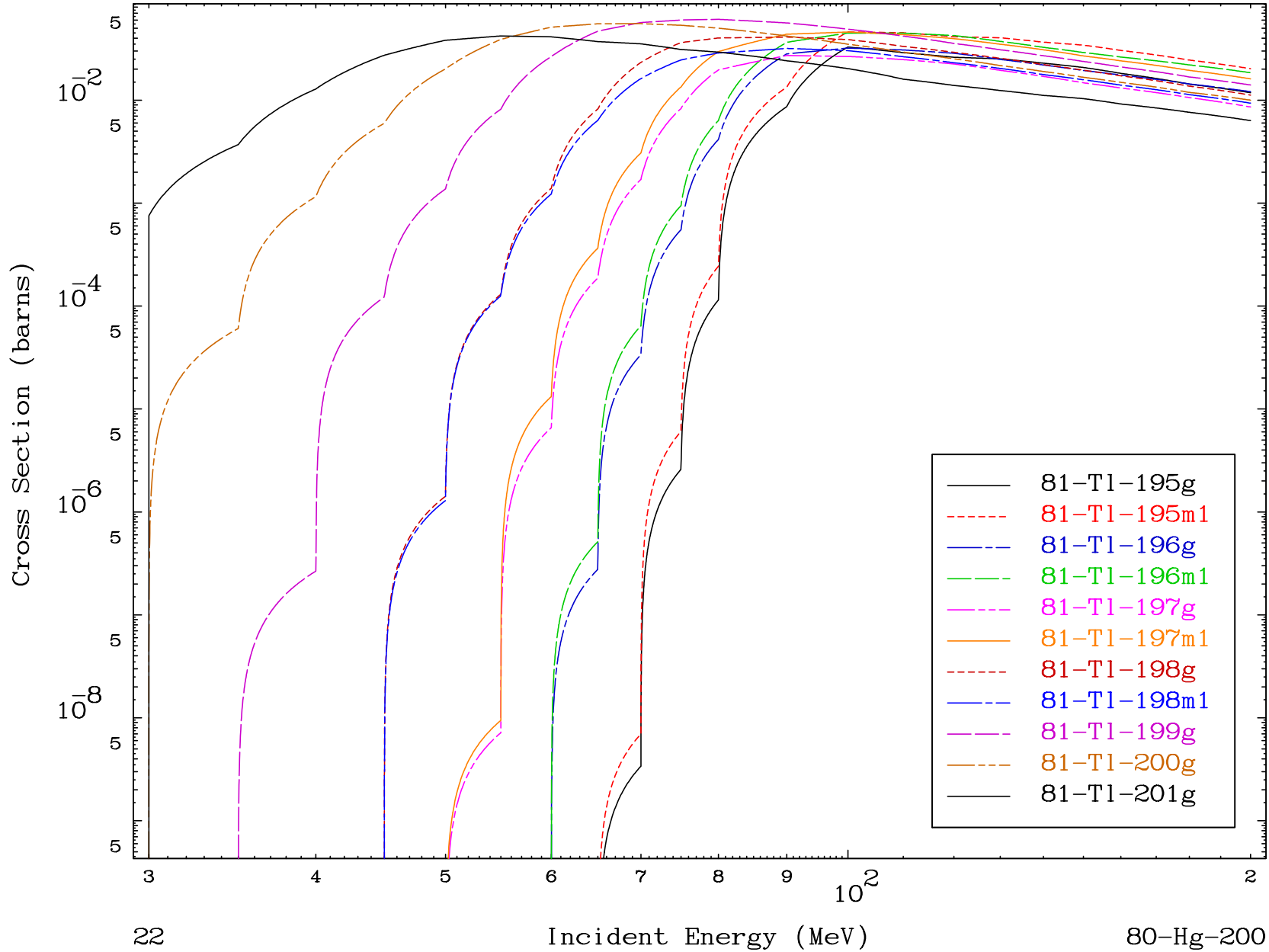
Radionuclide Production Cross Section

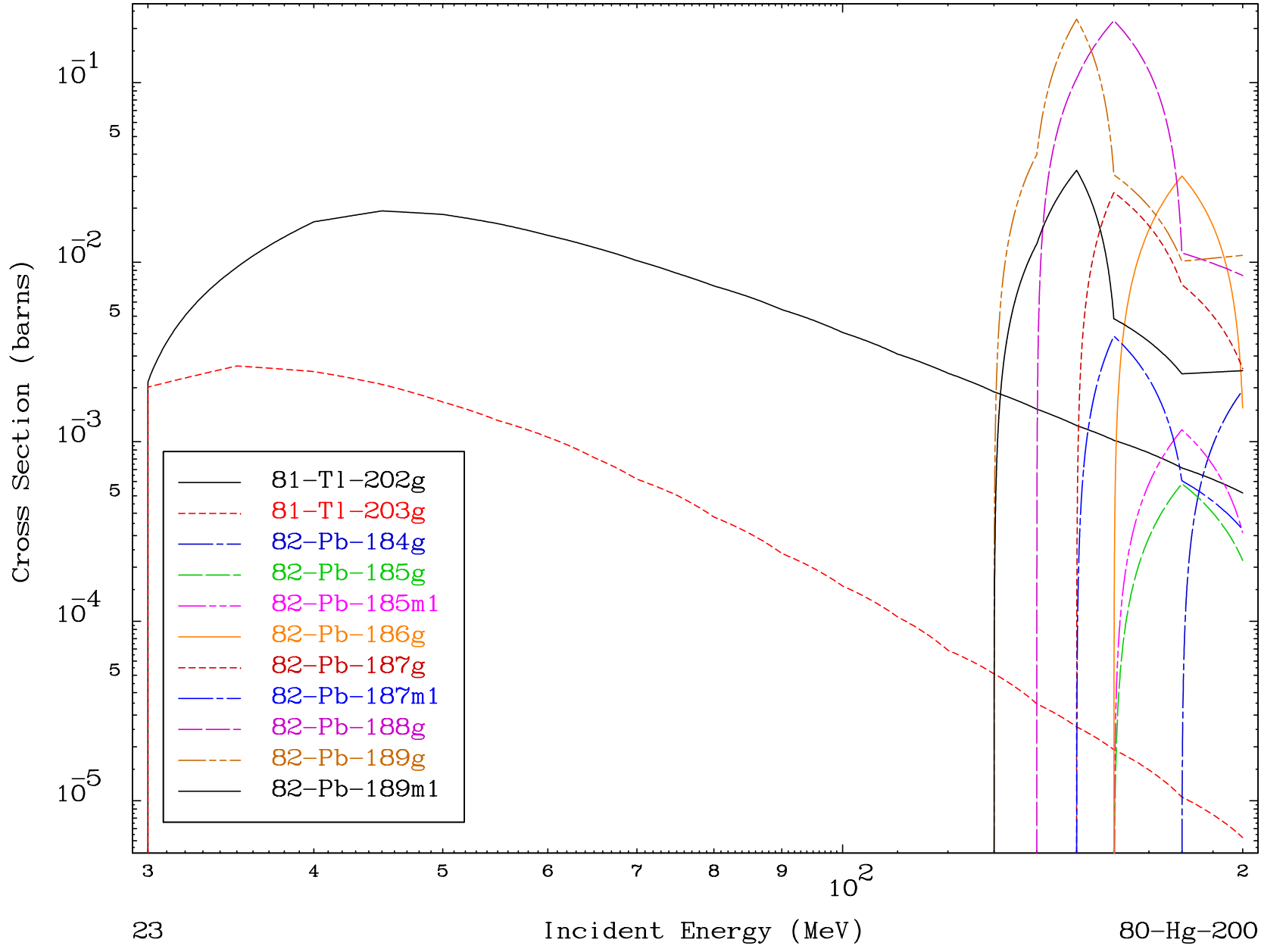


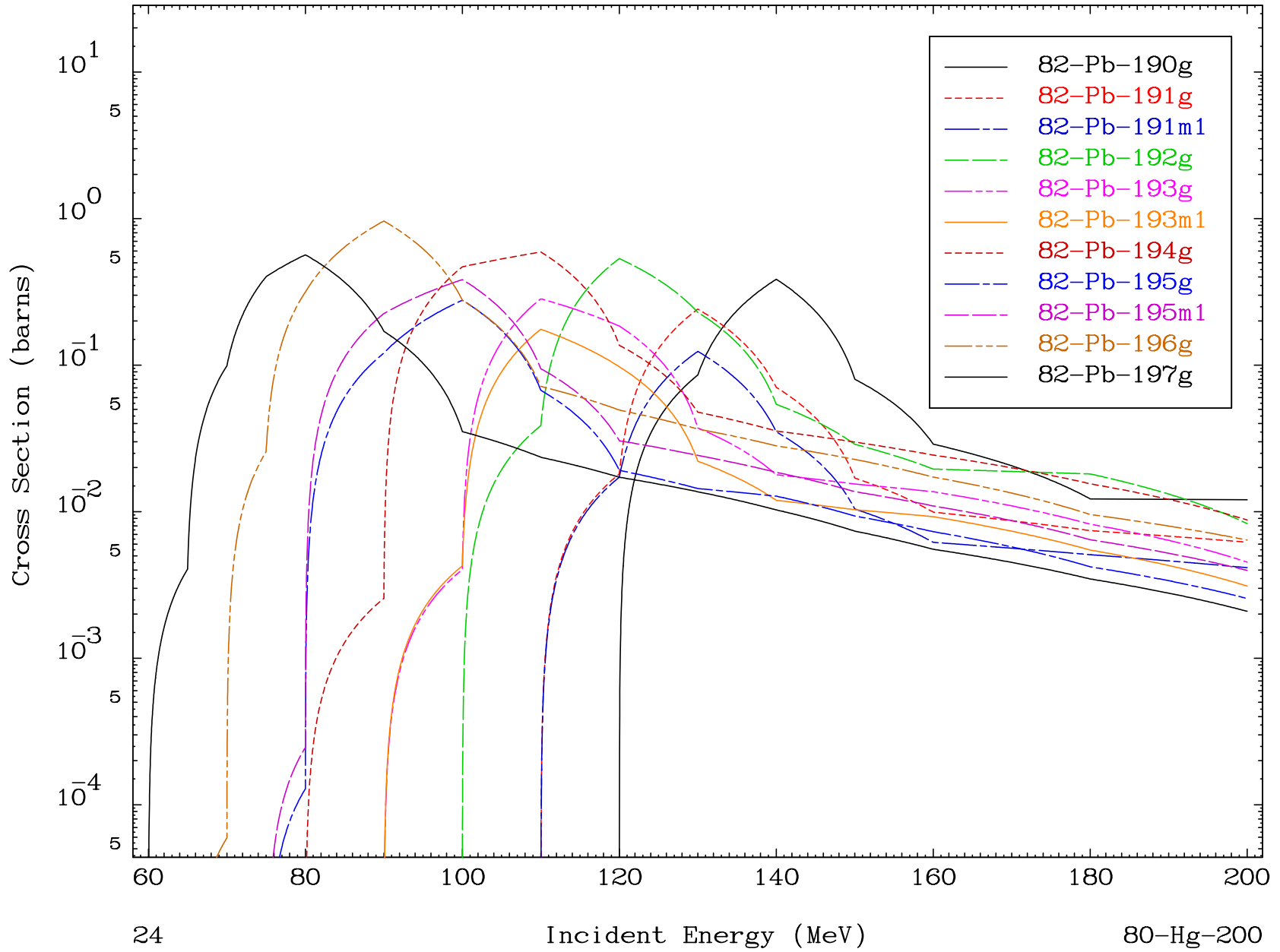
Radionuclide Production Cross Section



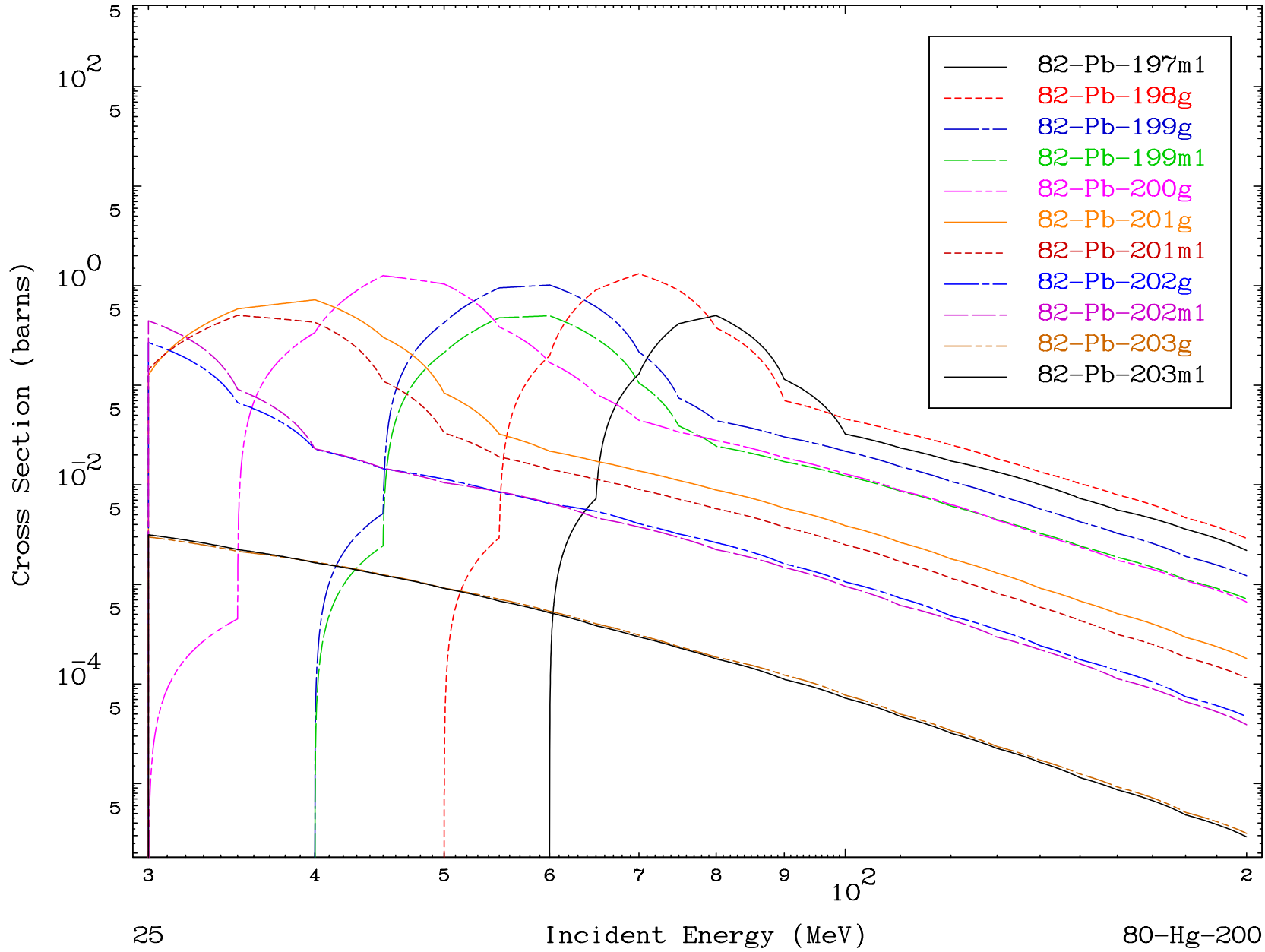
Radionuclide Production Cross Section







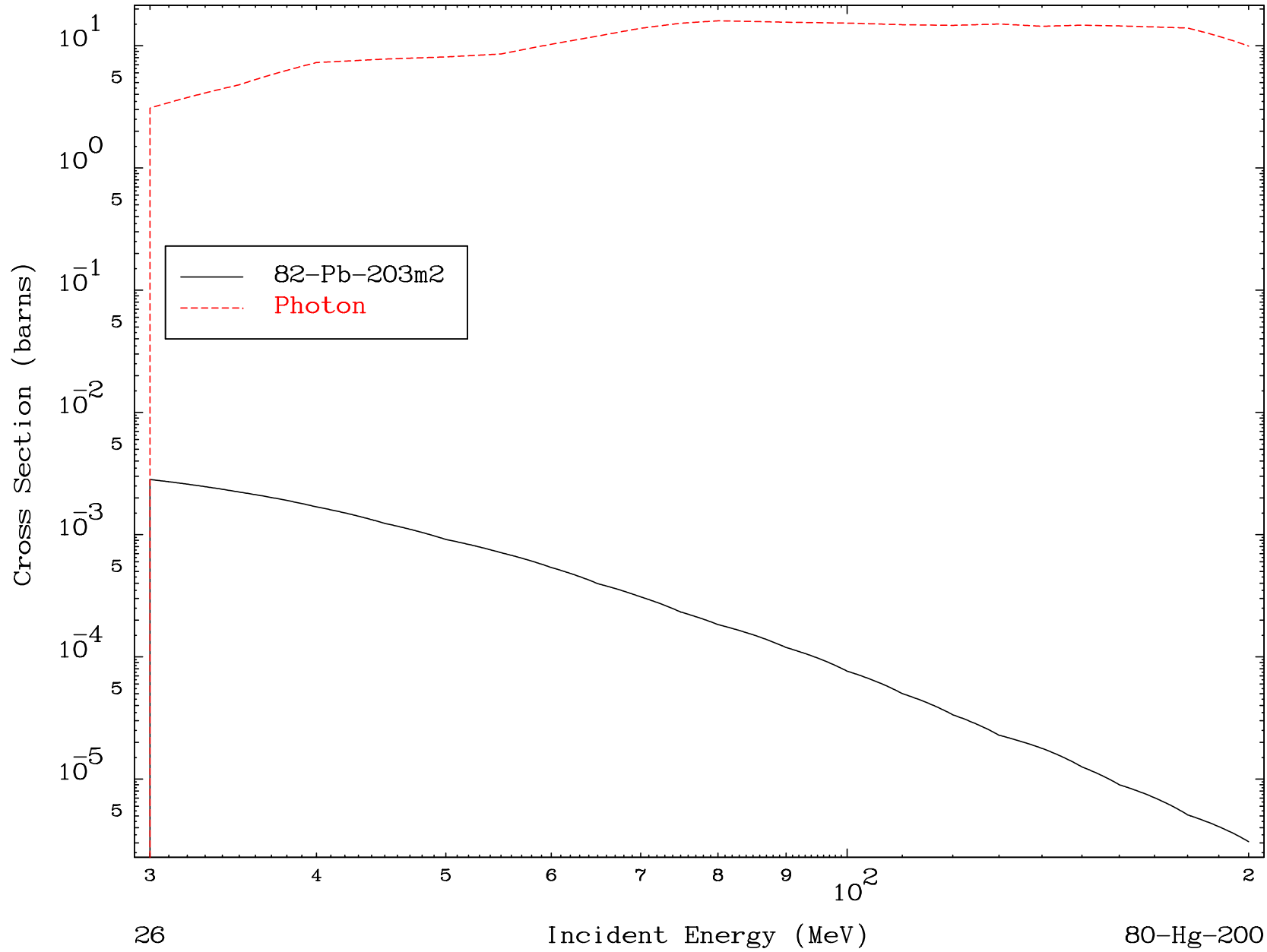




MAT 8037

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

80-Hg-200

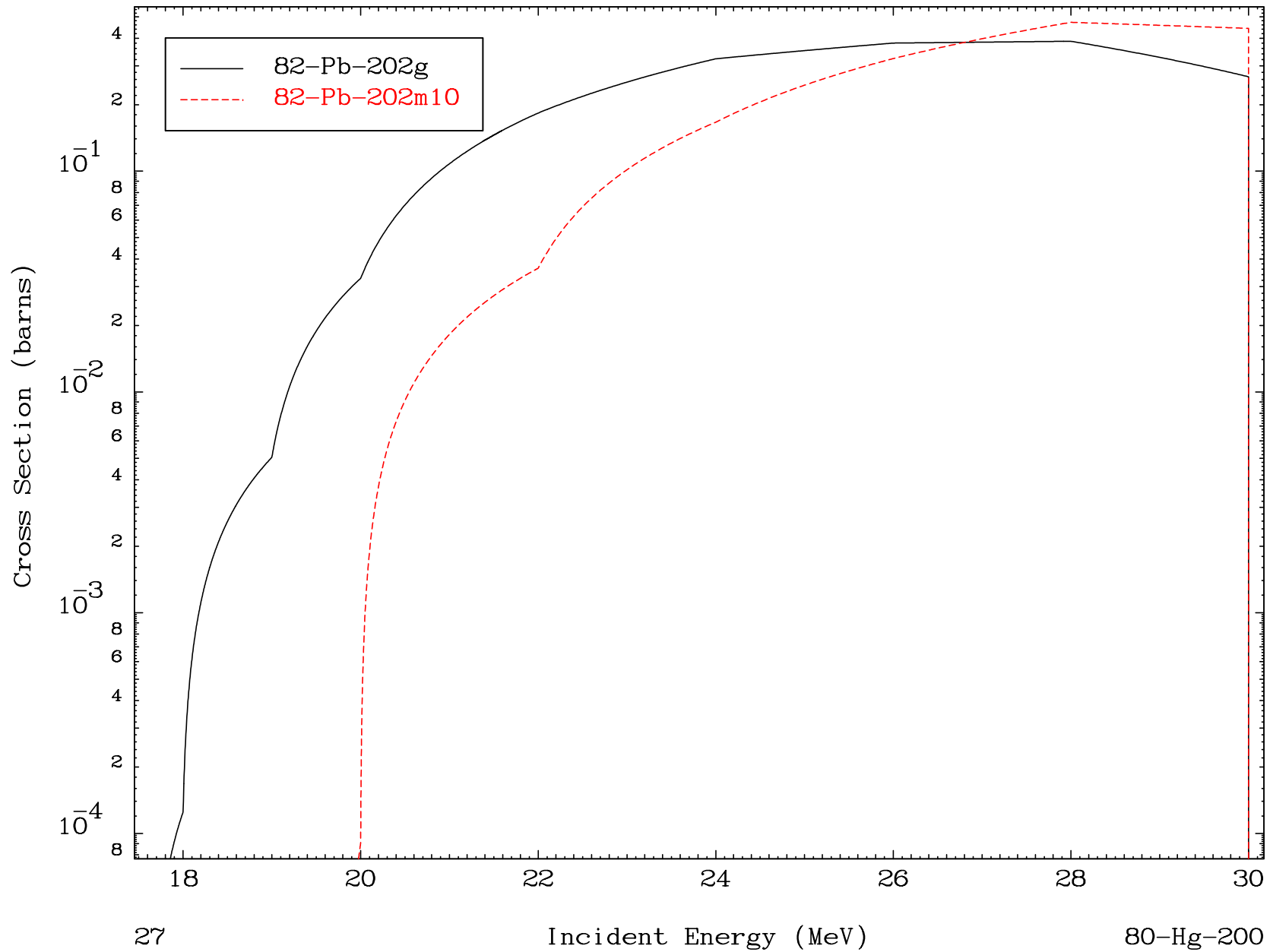


MAT 8037

( $\alpha, 2n$ )

80-Hg-200

Radionuclide Production Cross Section

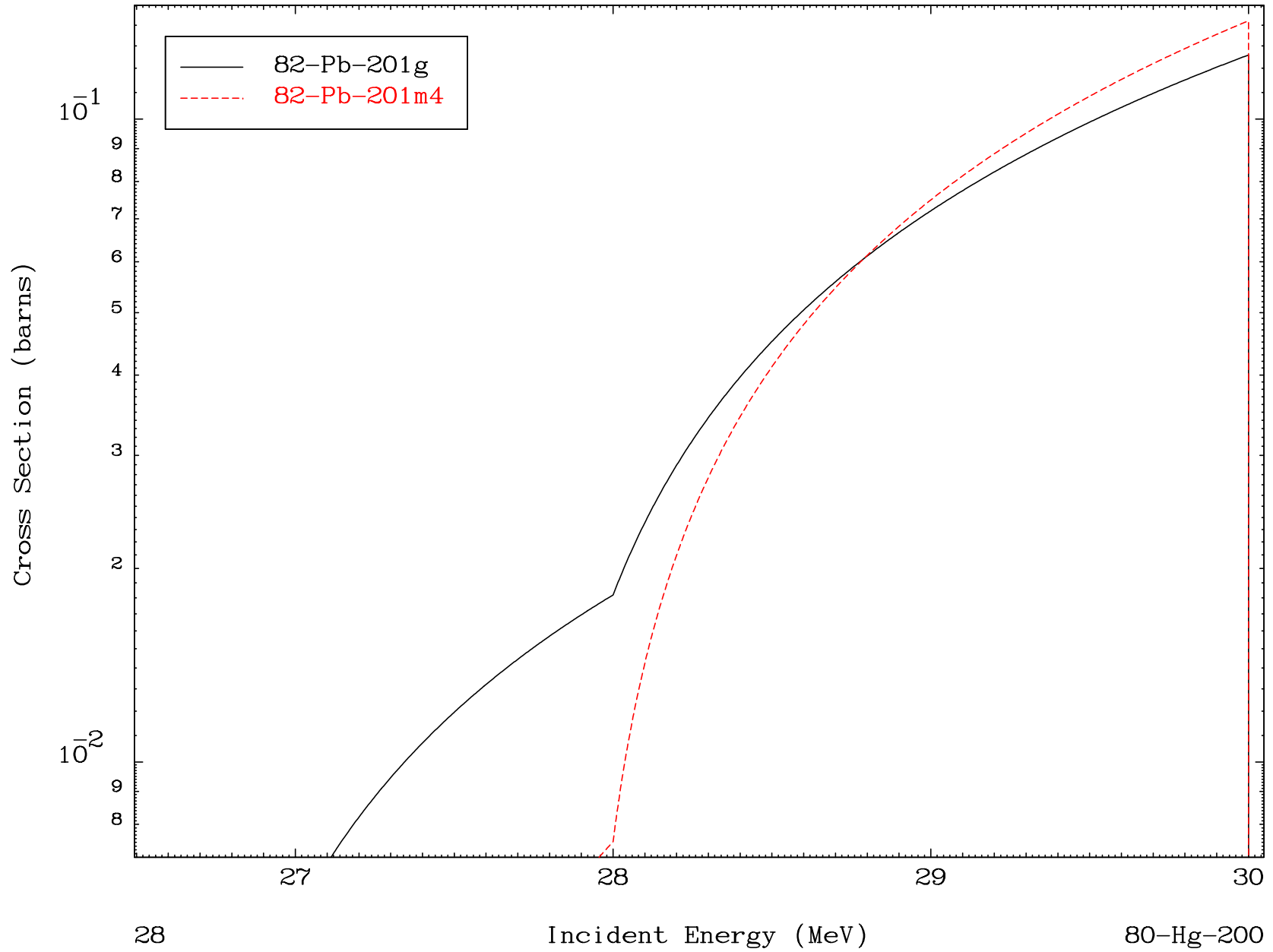


MAT 8037

( $\alpha, 3n$ )

80-Hg-200

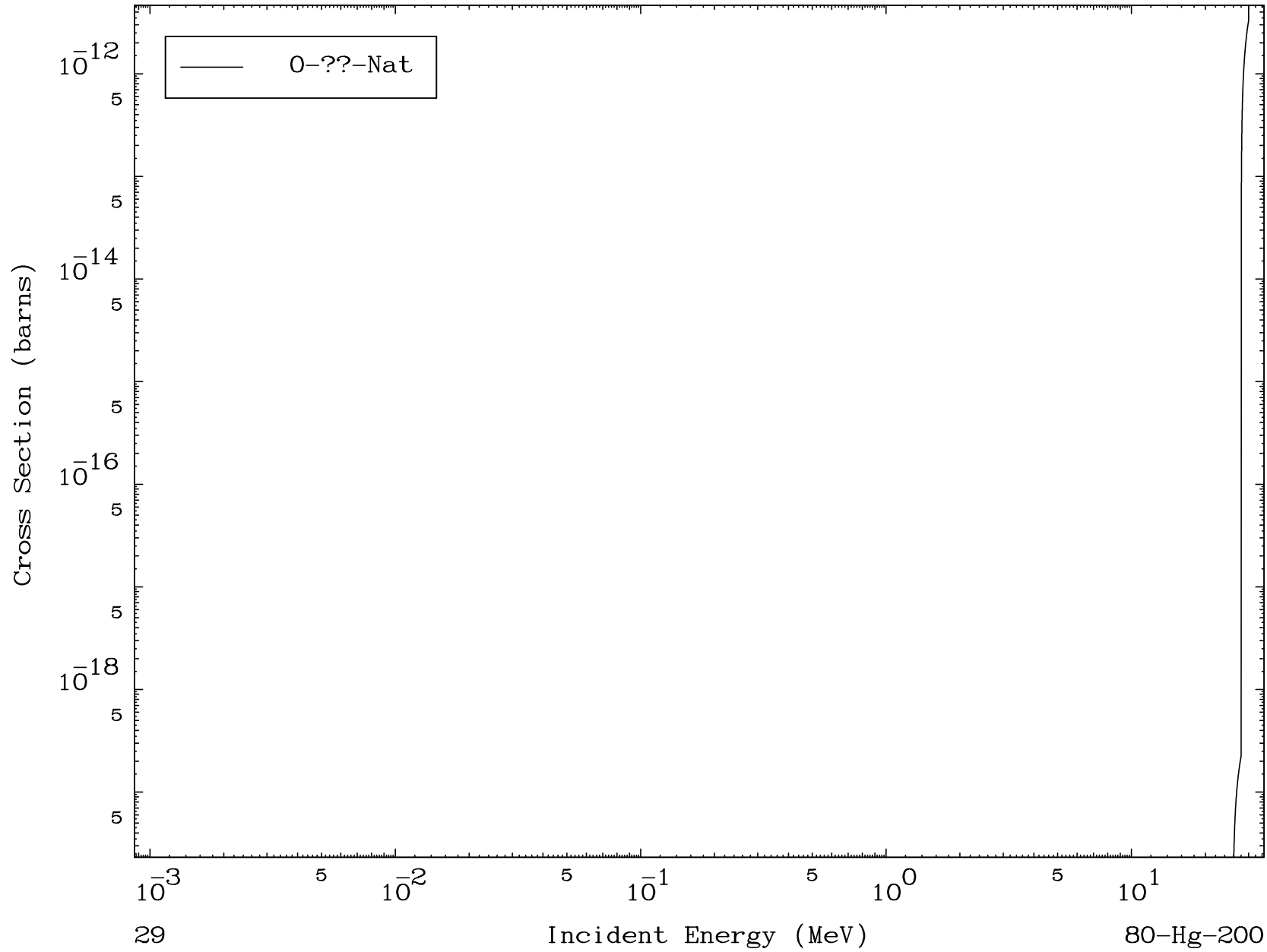
Radionuclide Production Cross Section



MAT 8037

$\alpha$  Fission  
Radionuclide Production Cross Section

80-Hg-200



29

Incident Energy (MeV)

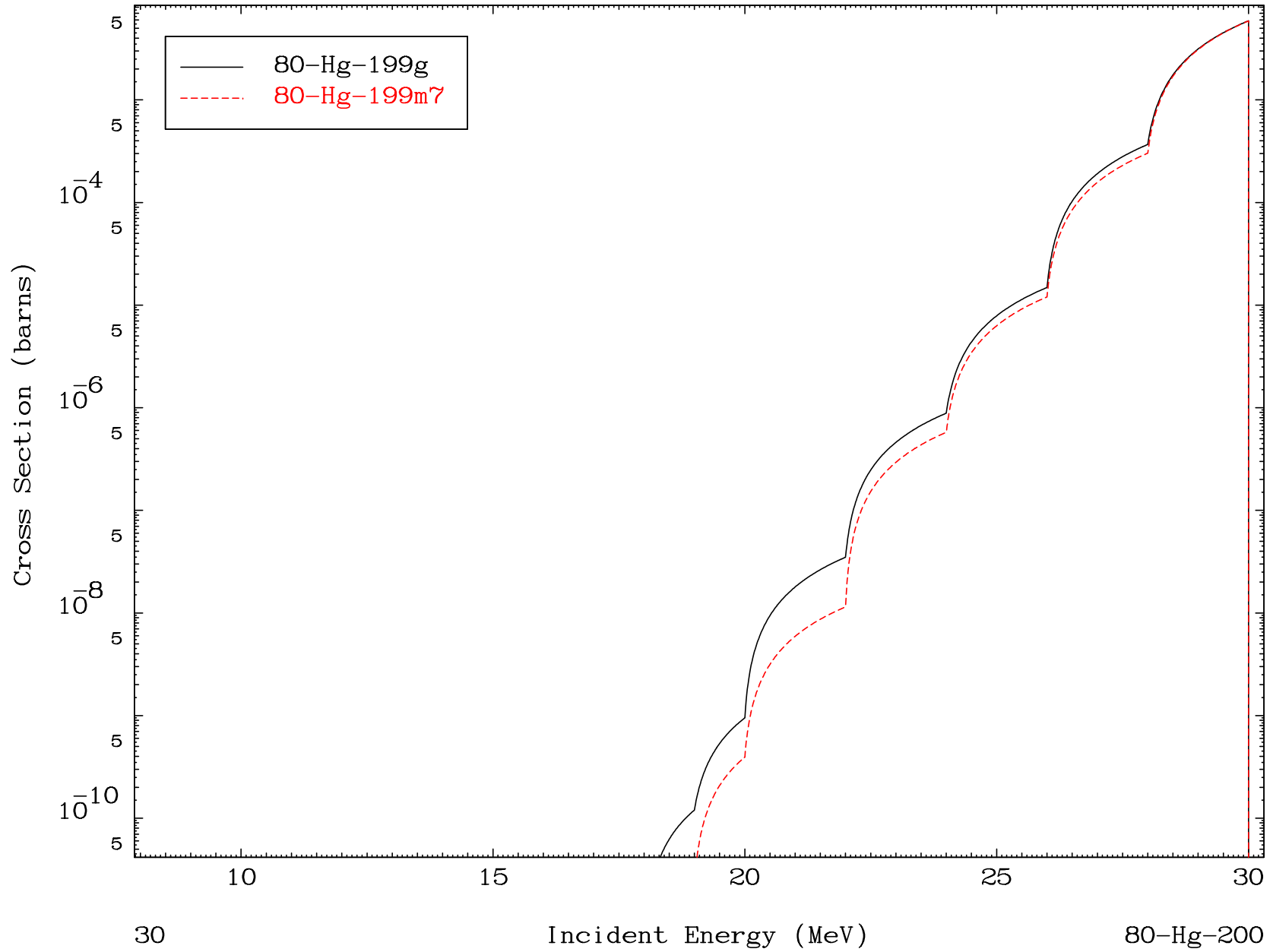
80-Hg-200

MAT 8037

$(\alpha, n')$   $\alpha$

80-Hg-200

Radionuclide Production Cross Section



Radionuclide Production Cross Section

