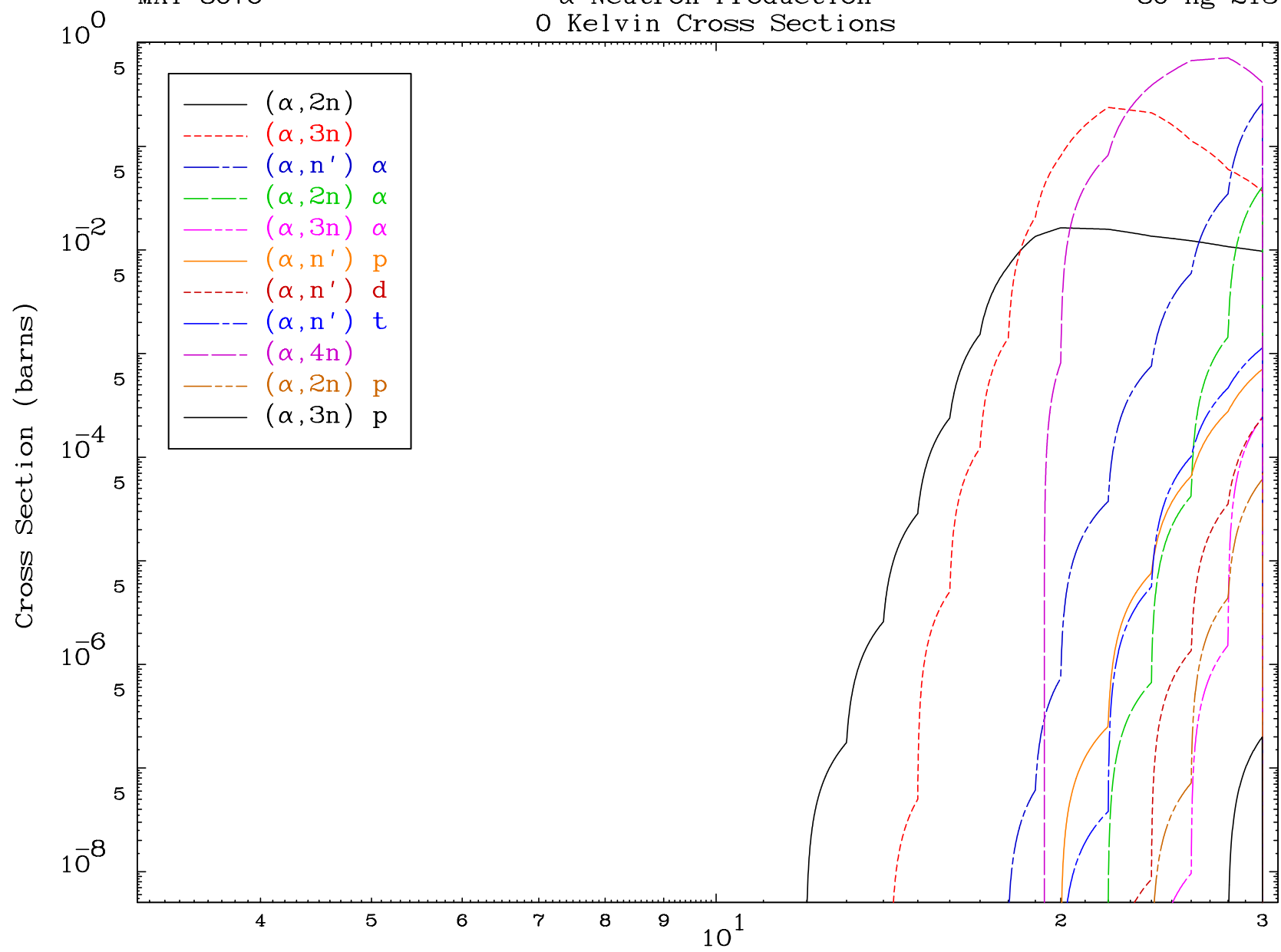


MAT 8076

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

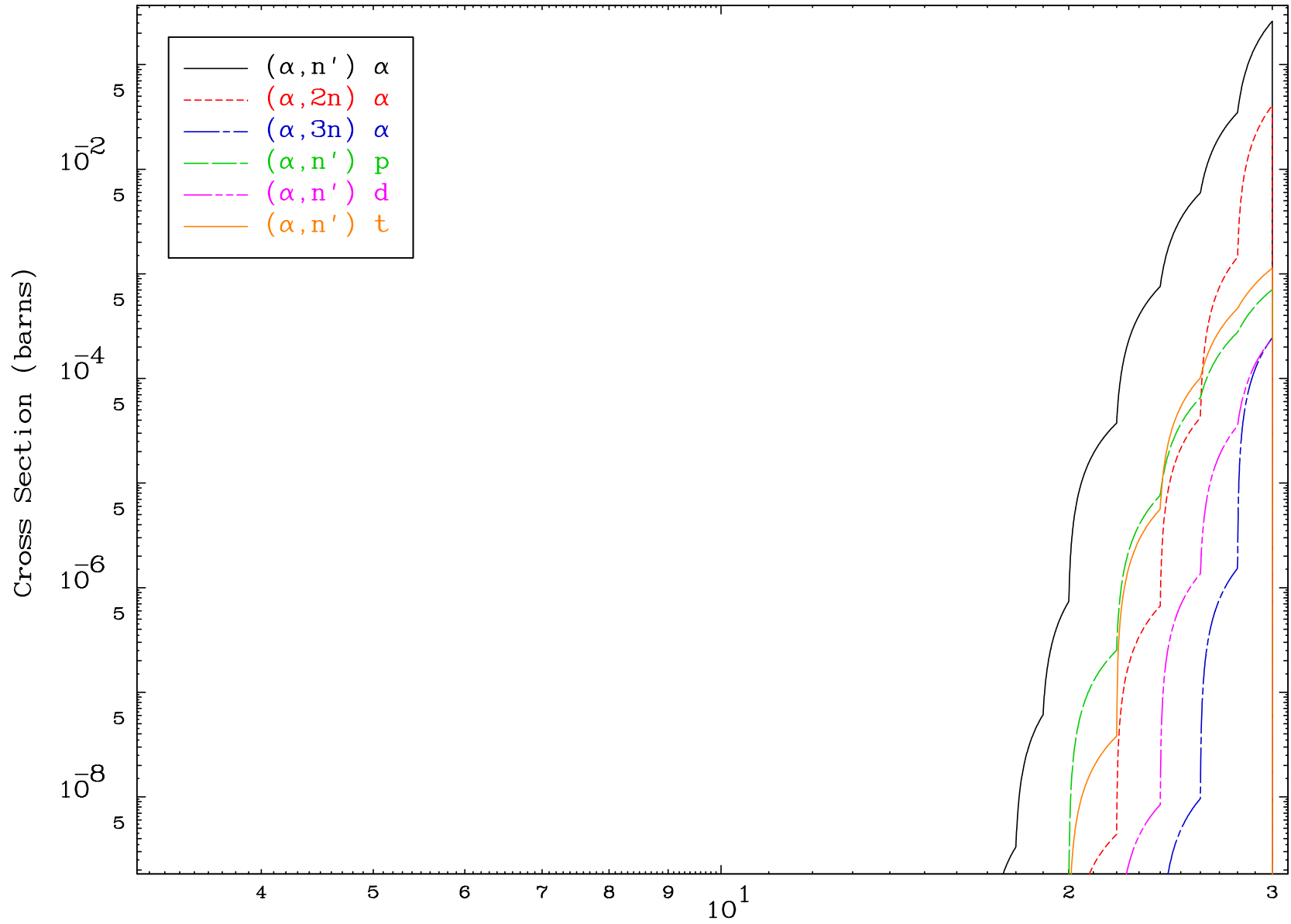
80-Hg-213

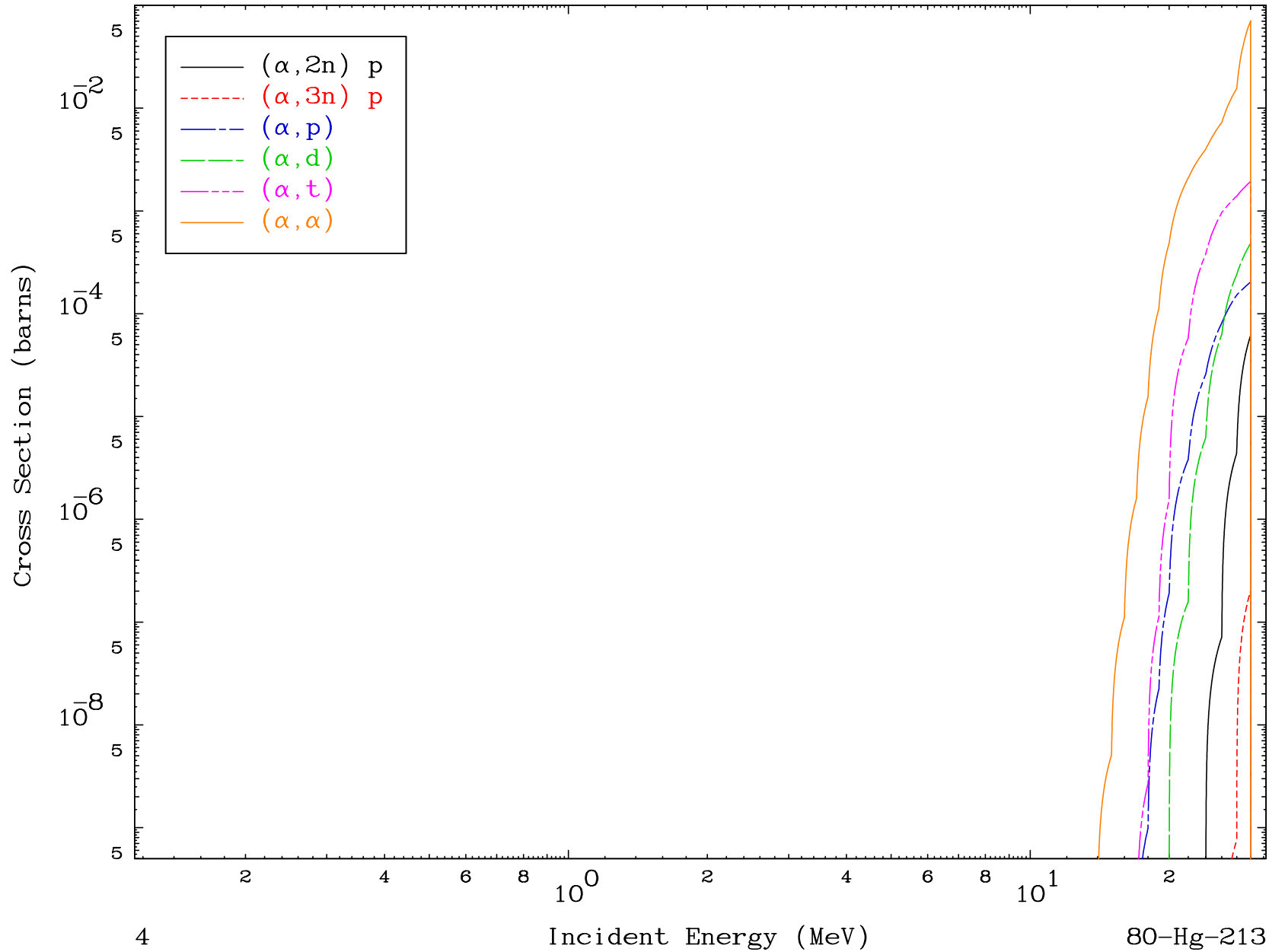


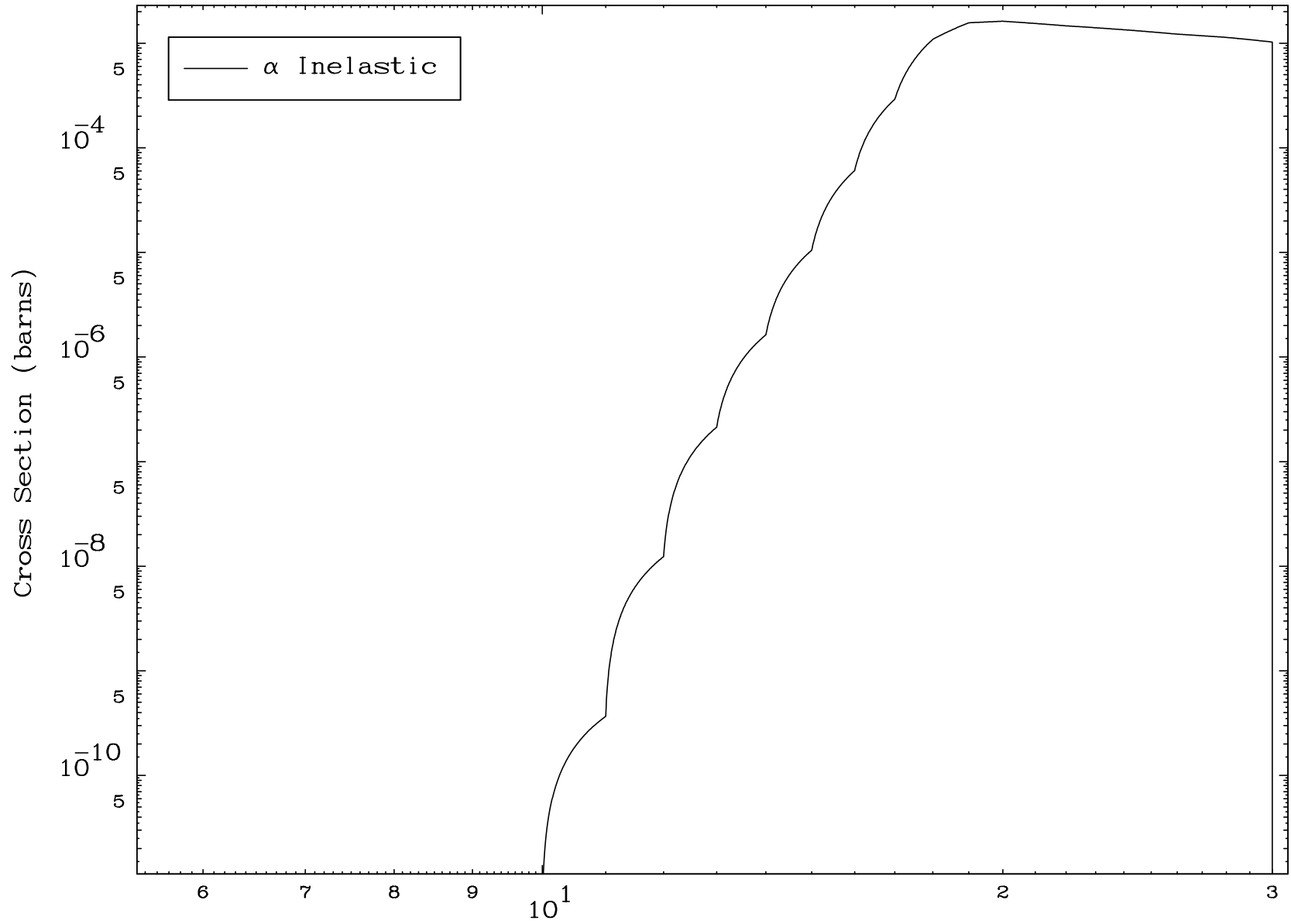
2

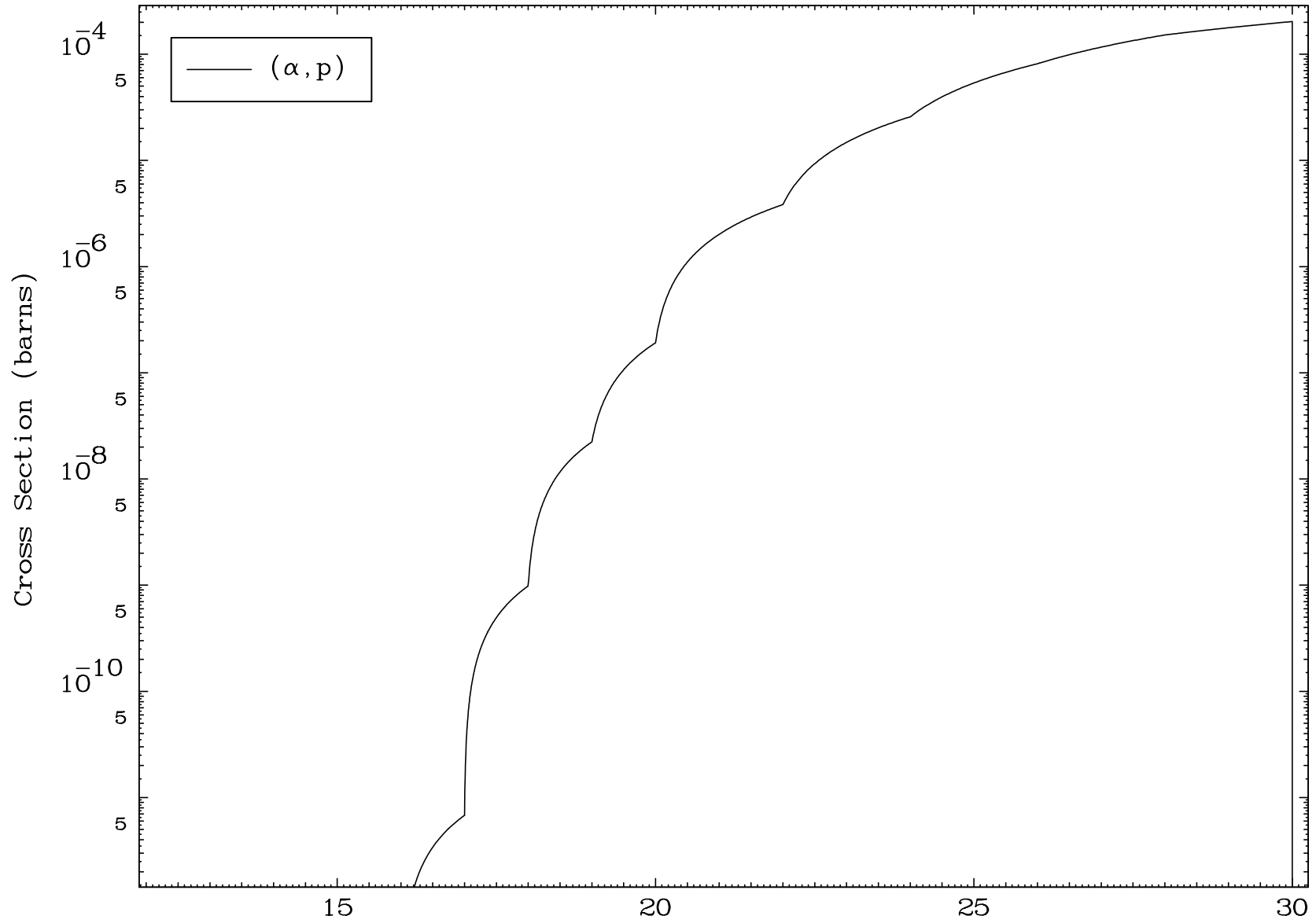
Incident Energy (MeV)

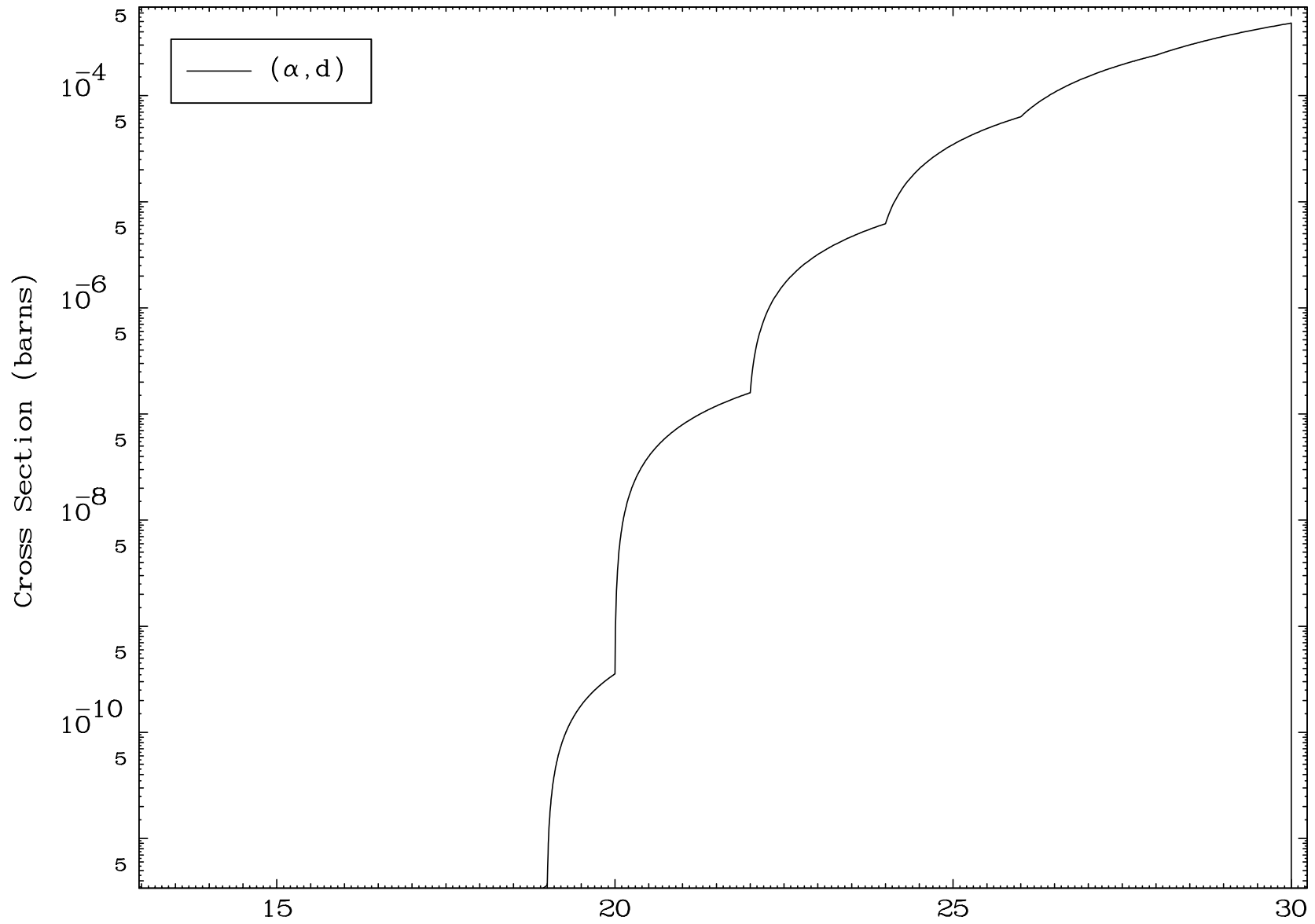
80-Hg-213

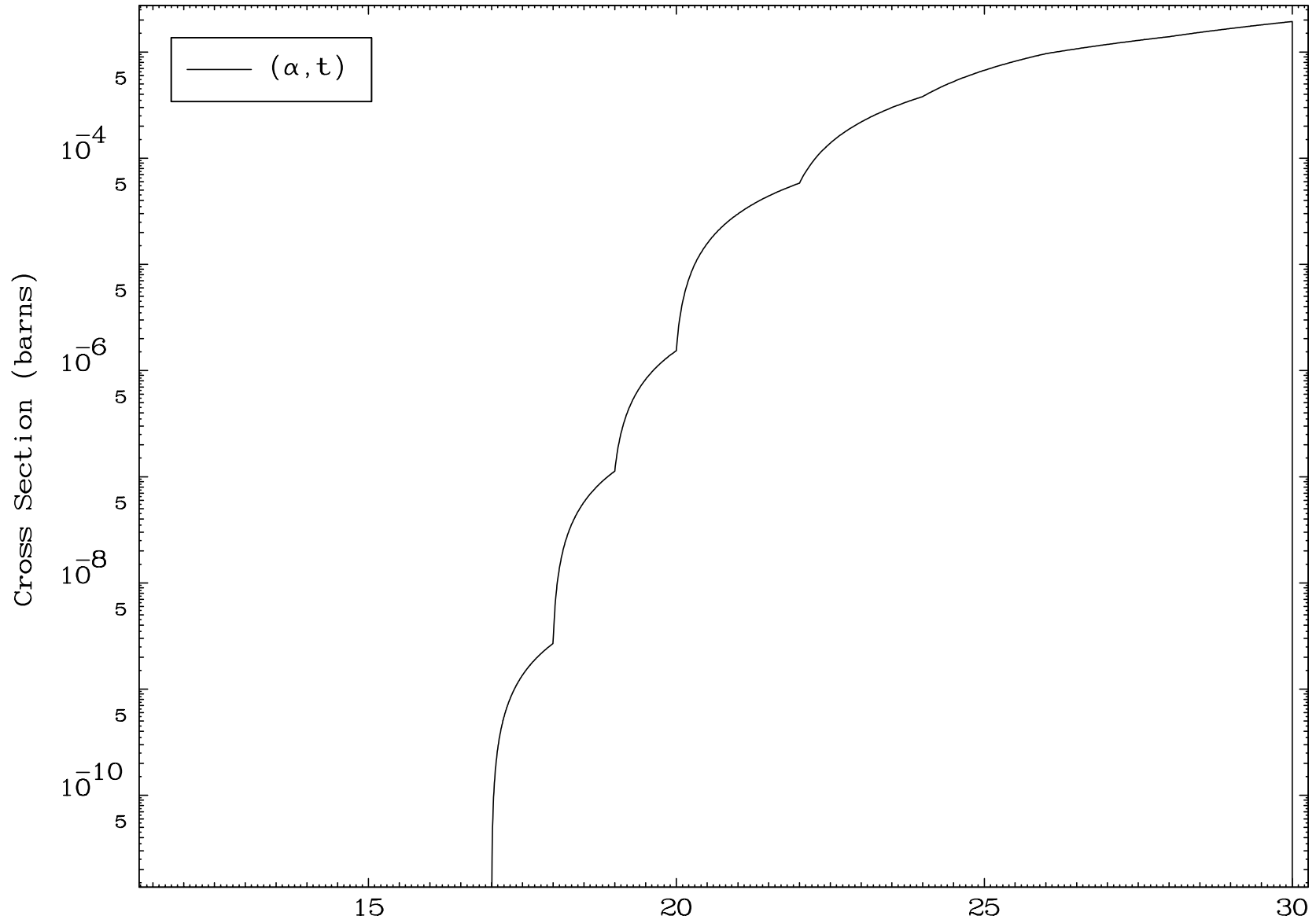




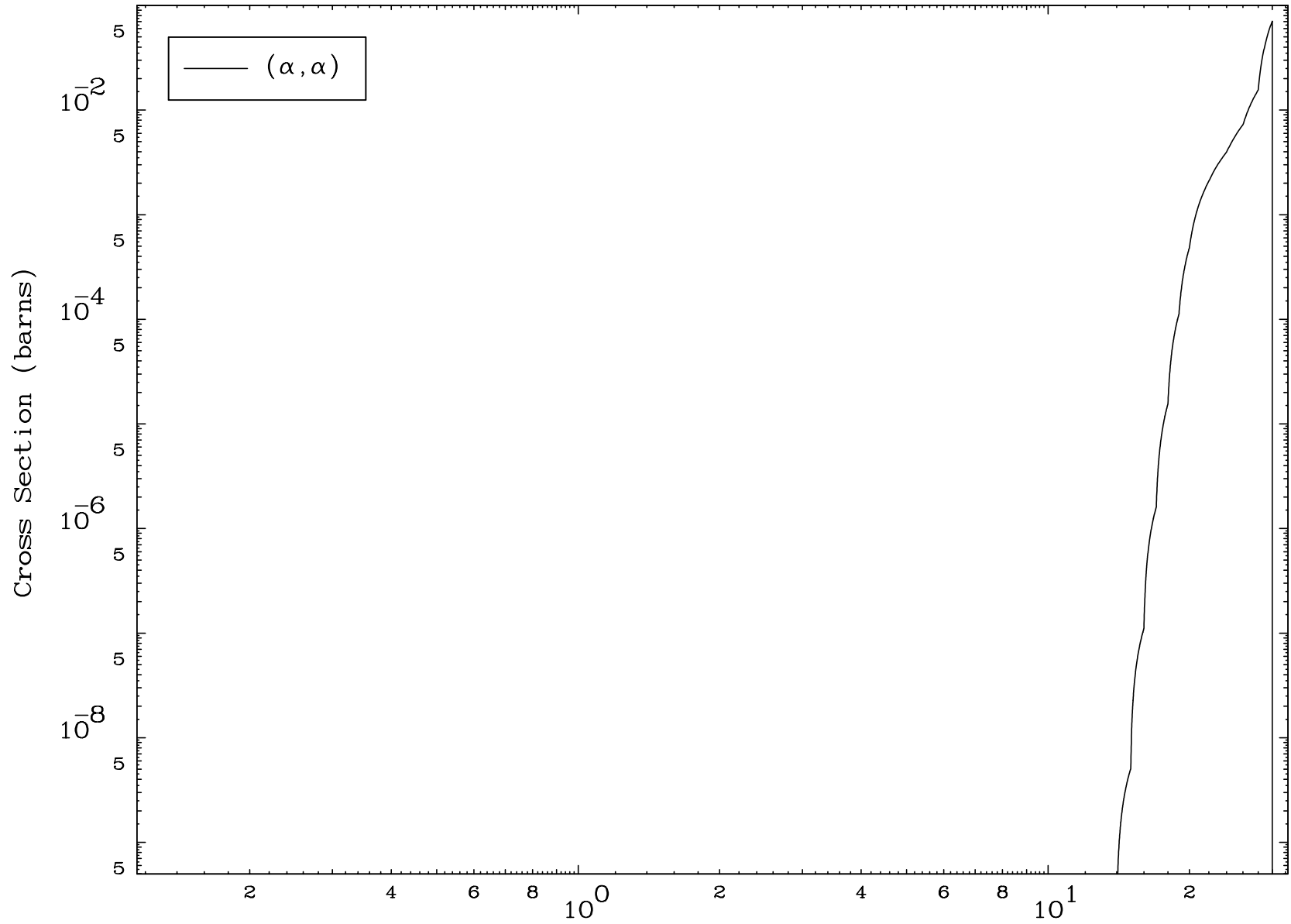




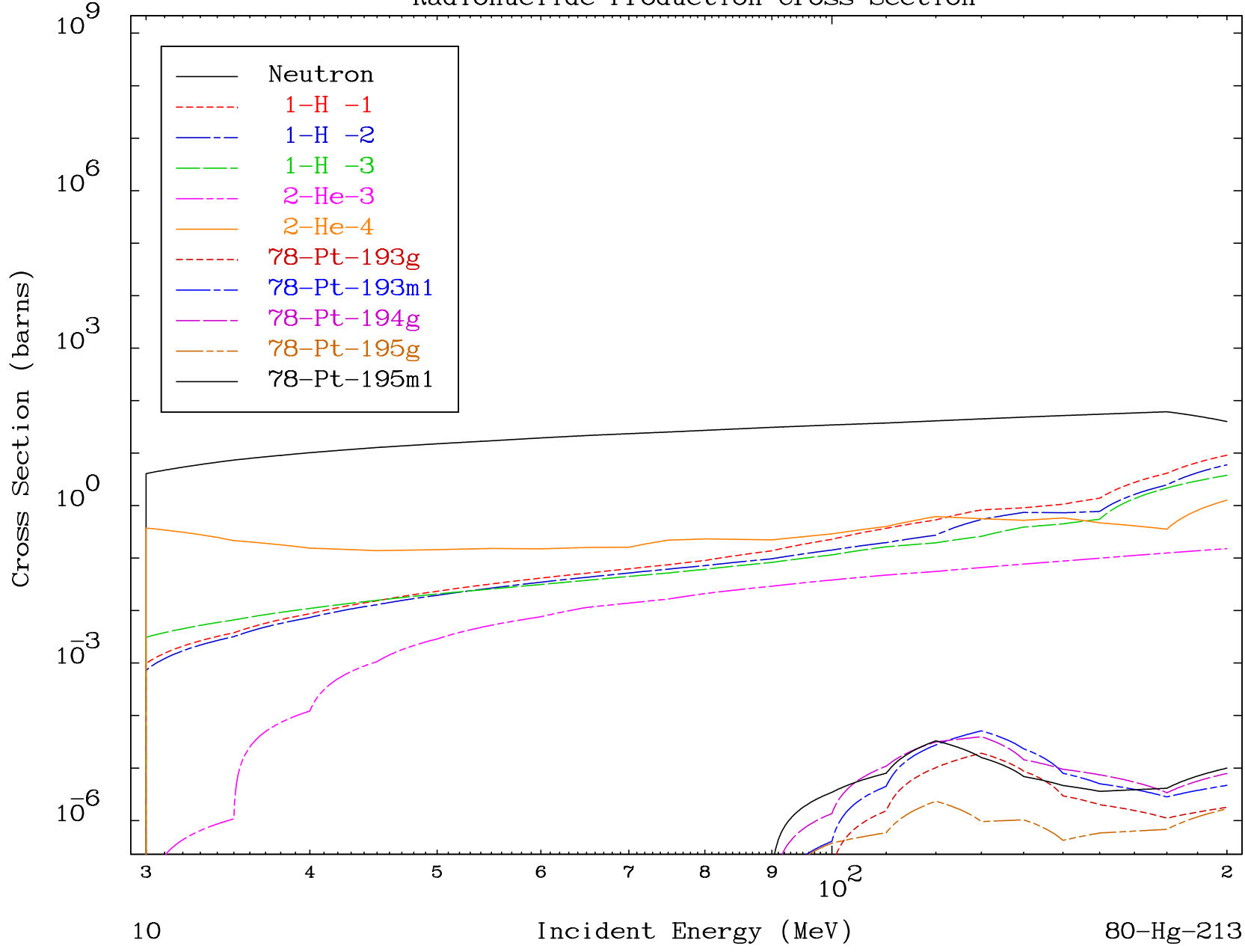


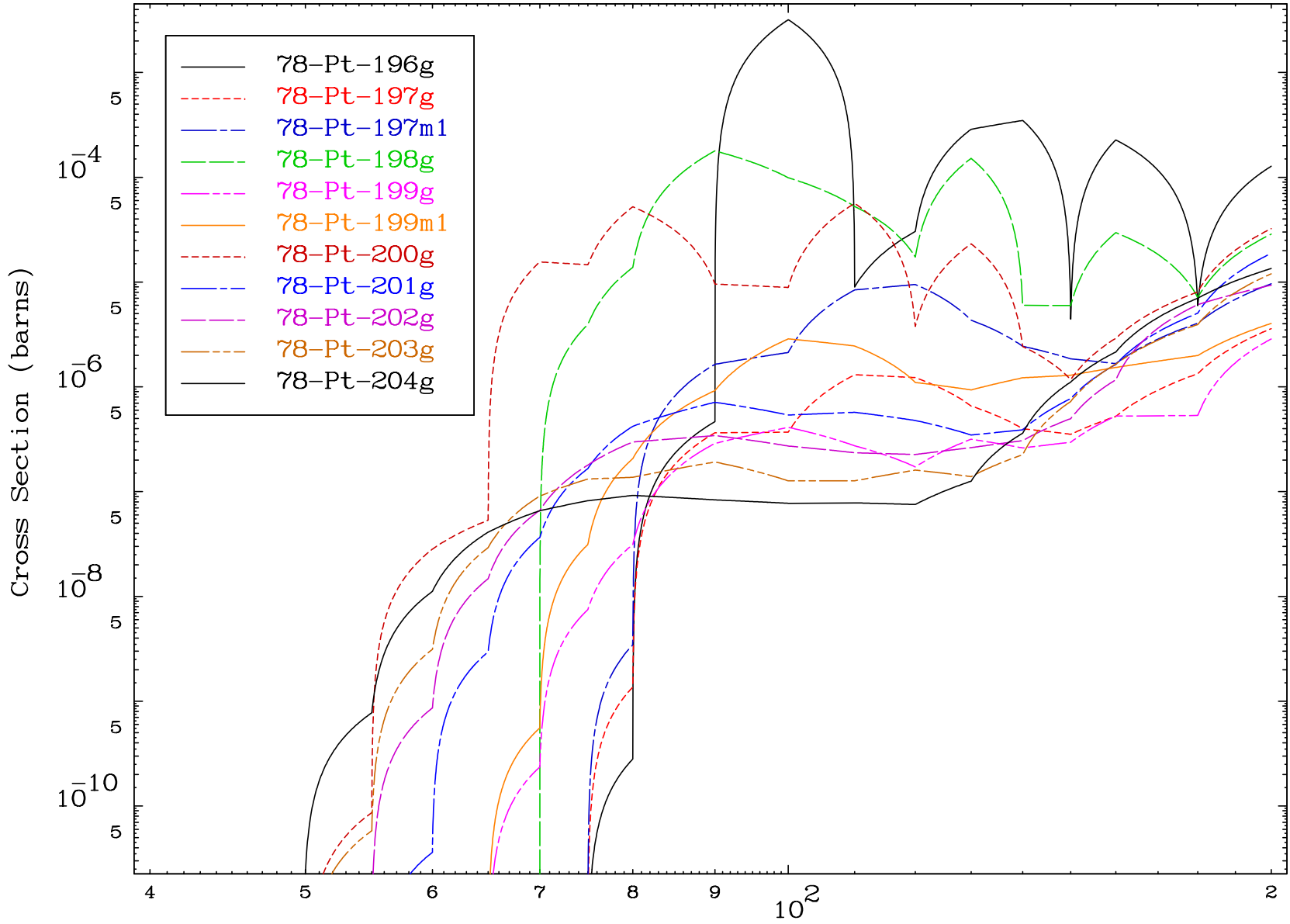




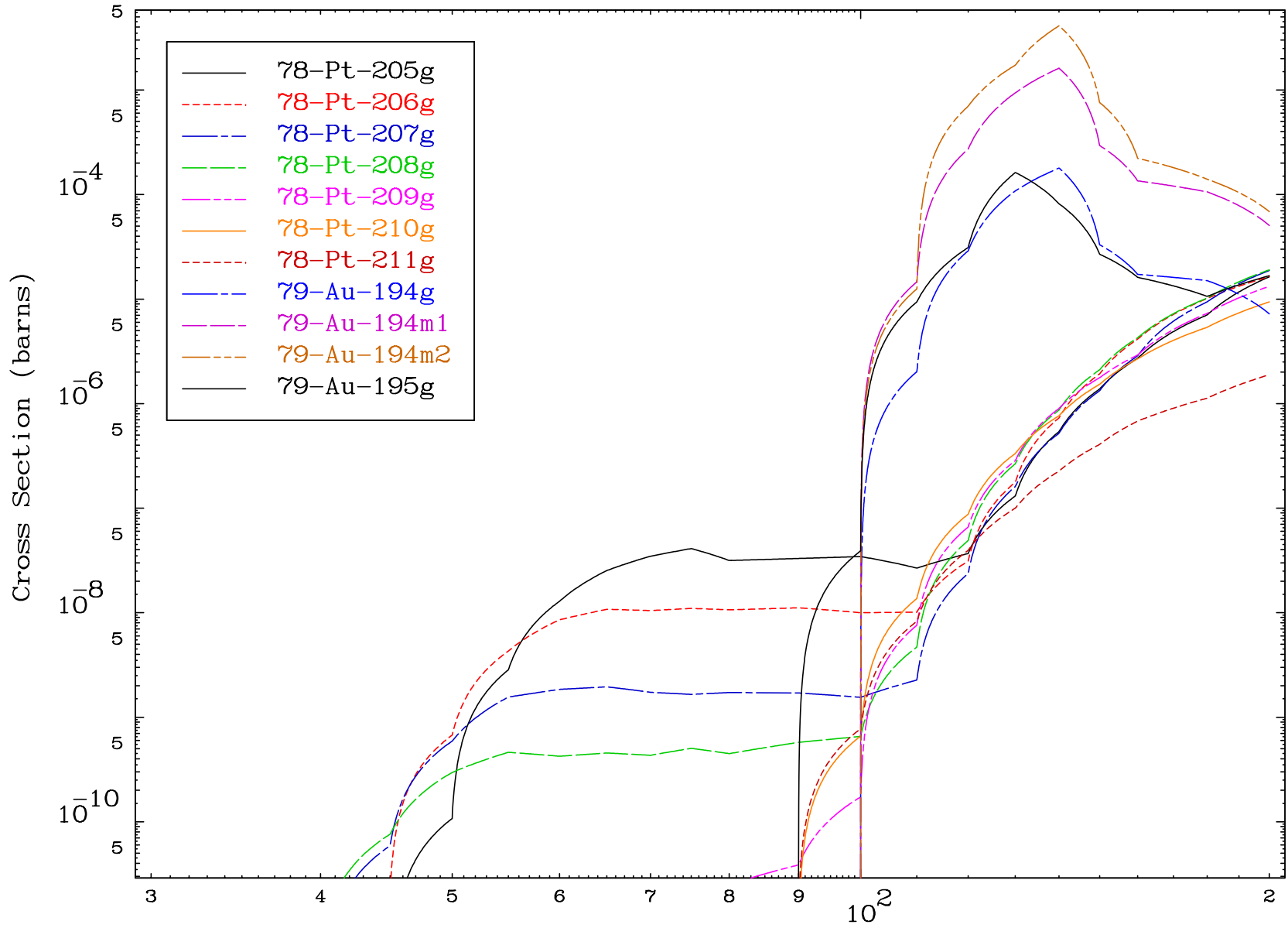


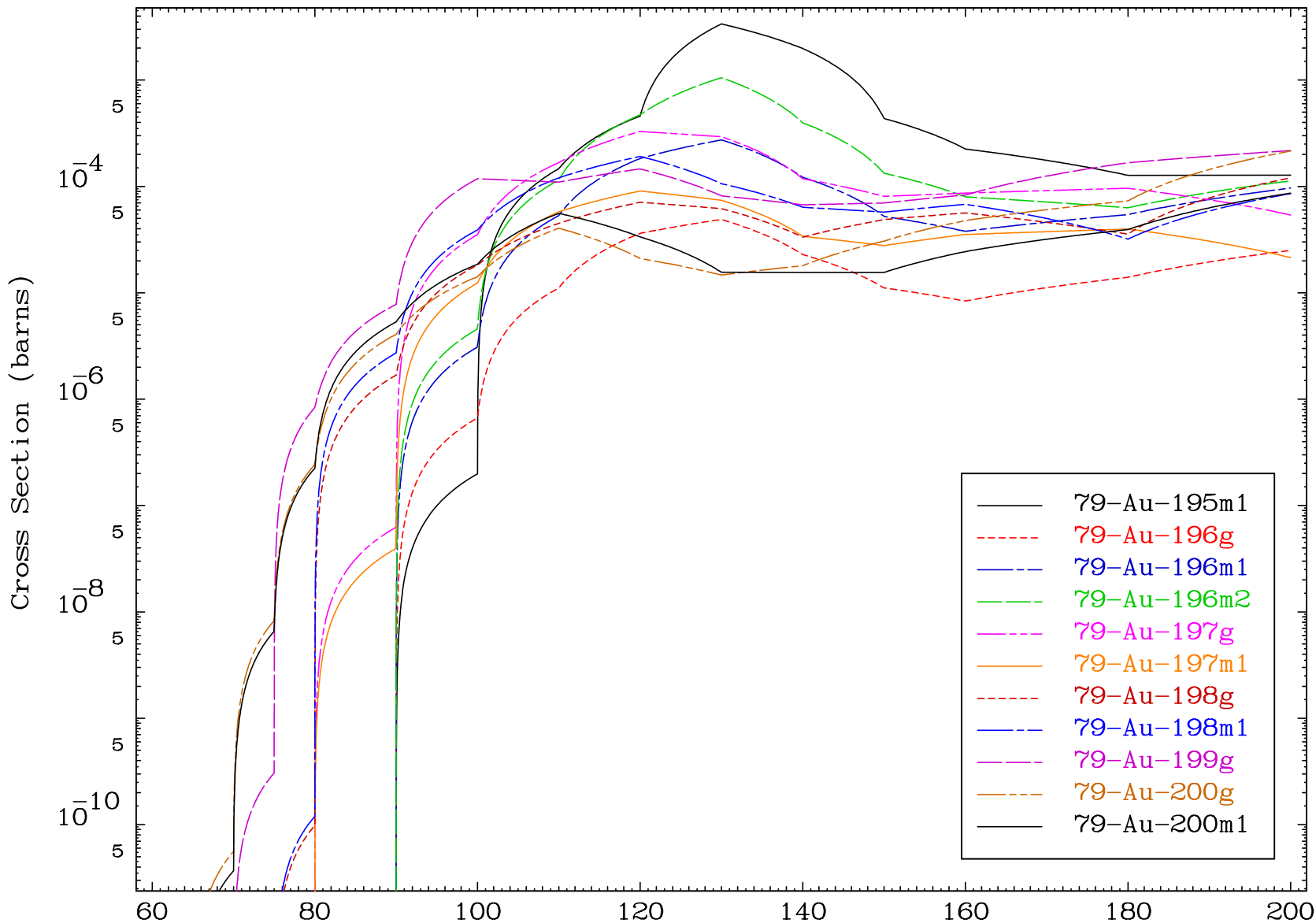
Radionuclide Production Cross Section



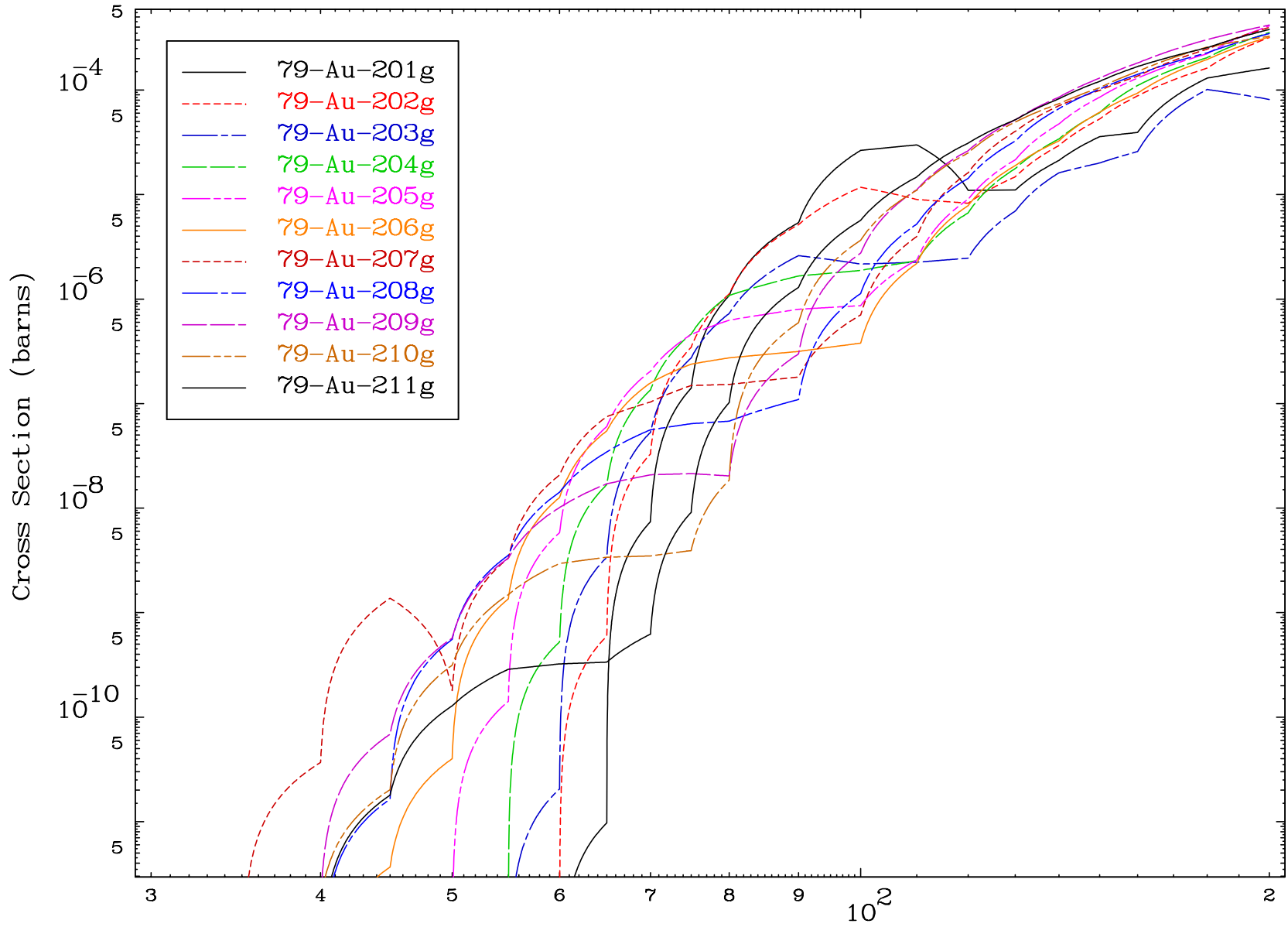


Radionuclide Production Cross Section

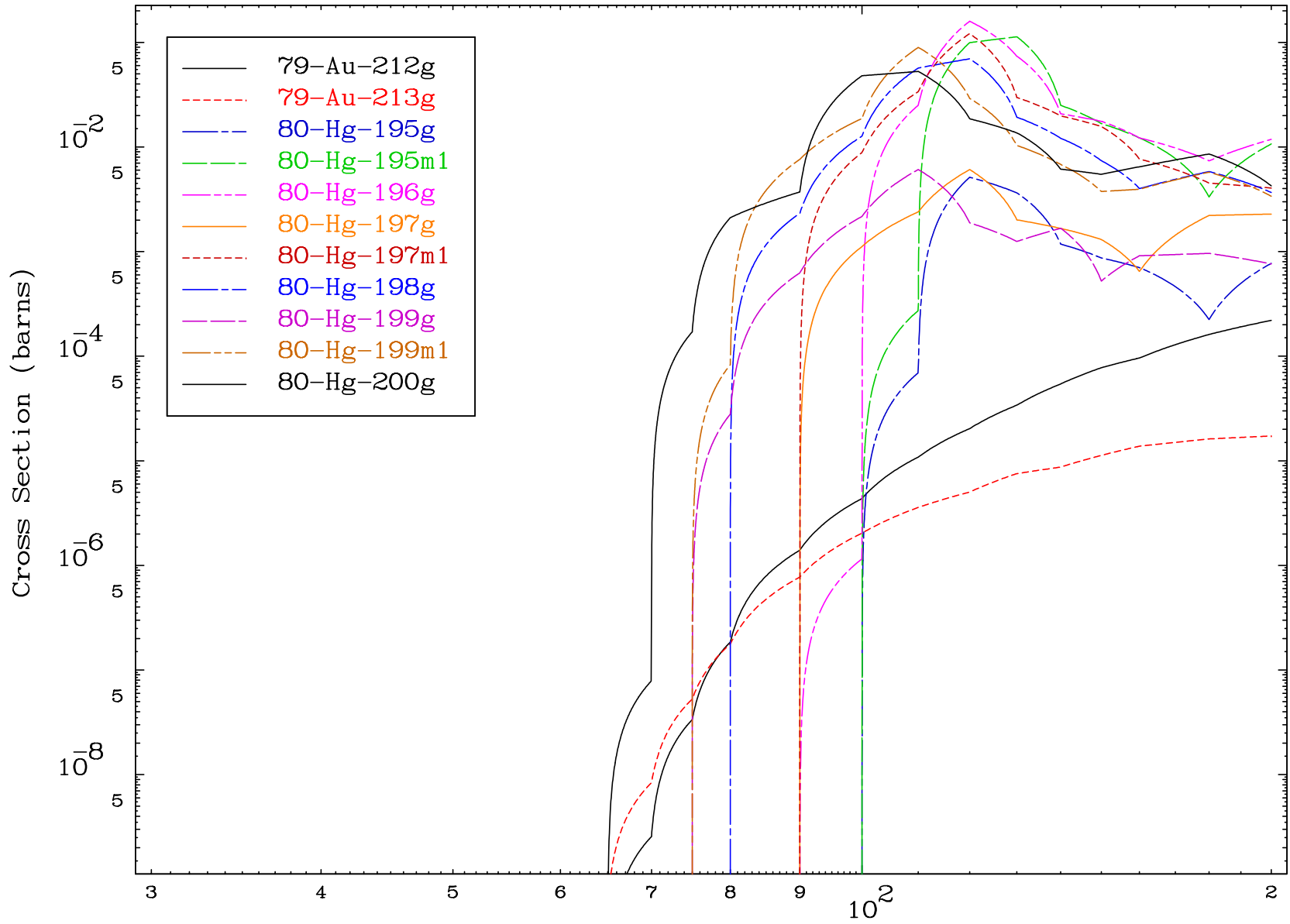


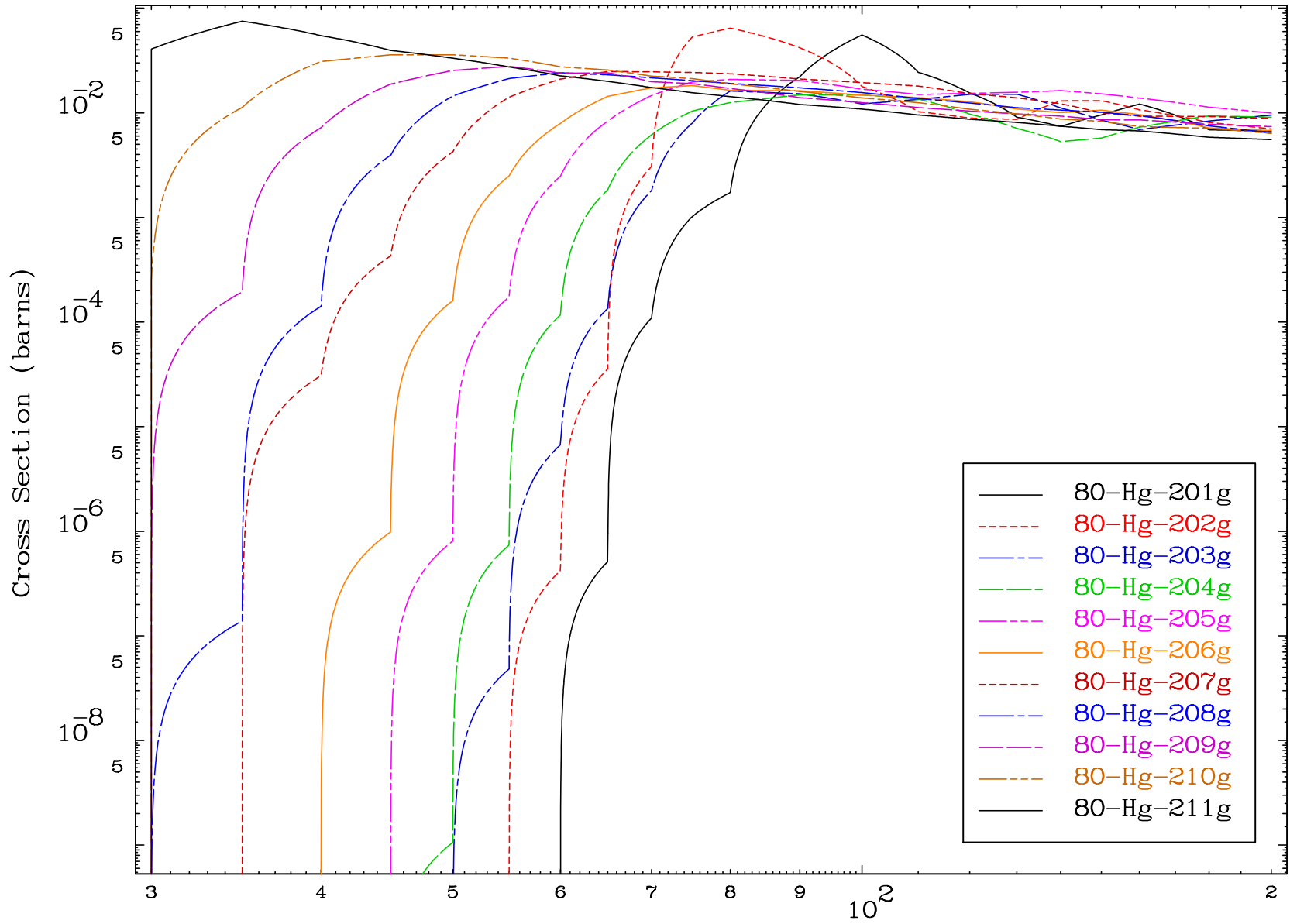


Radionuclide Production Cross Section

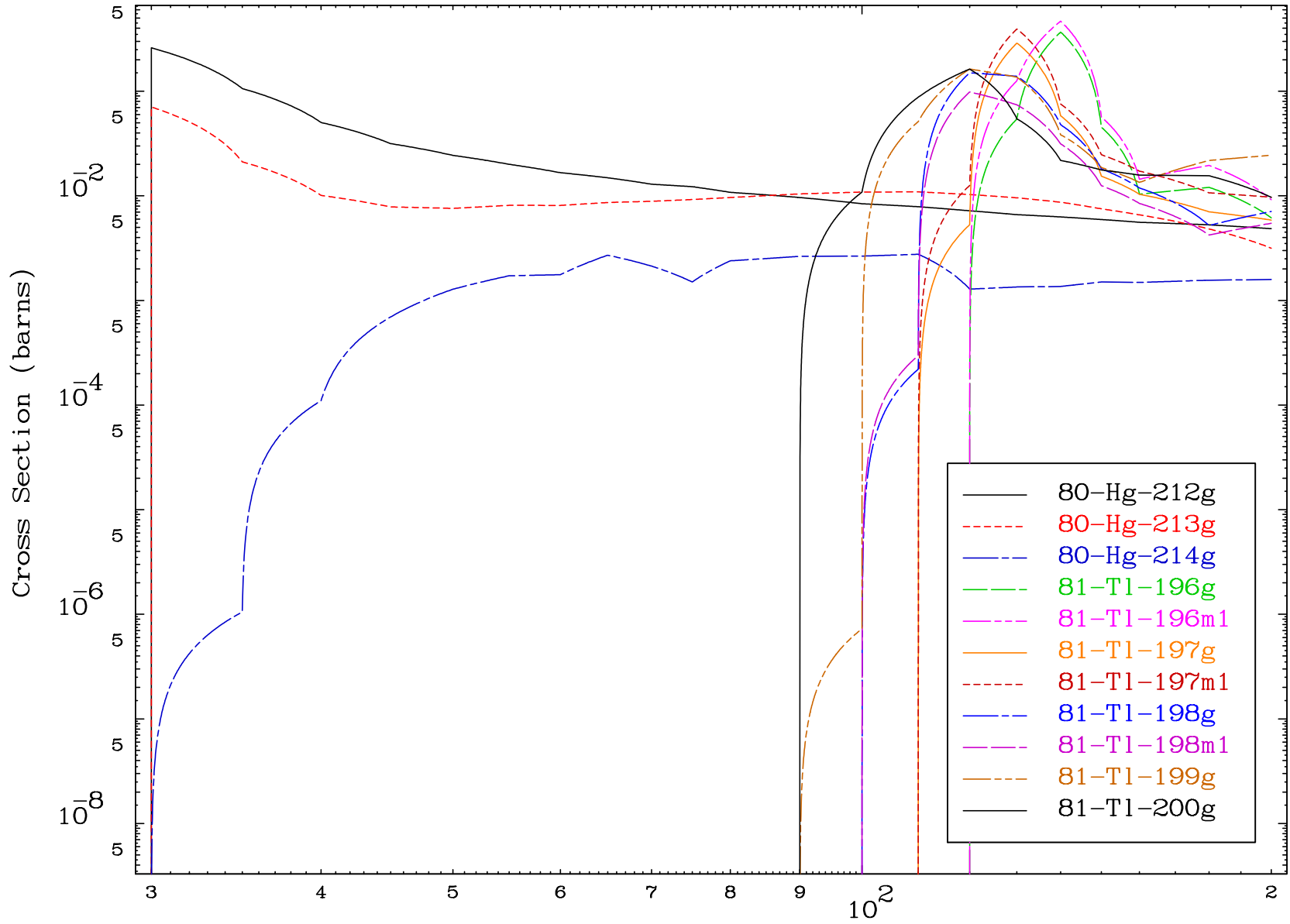


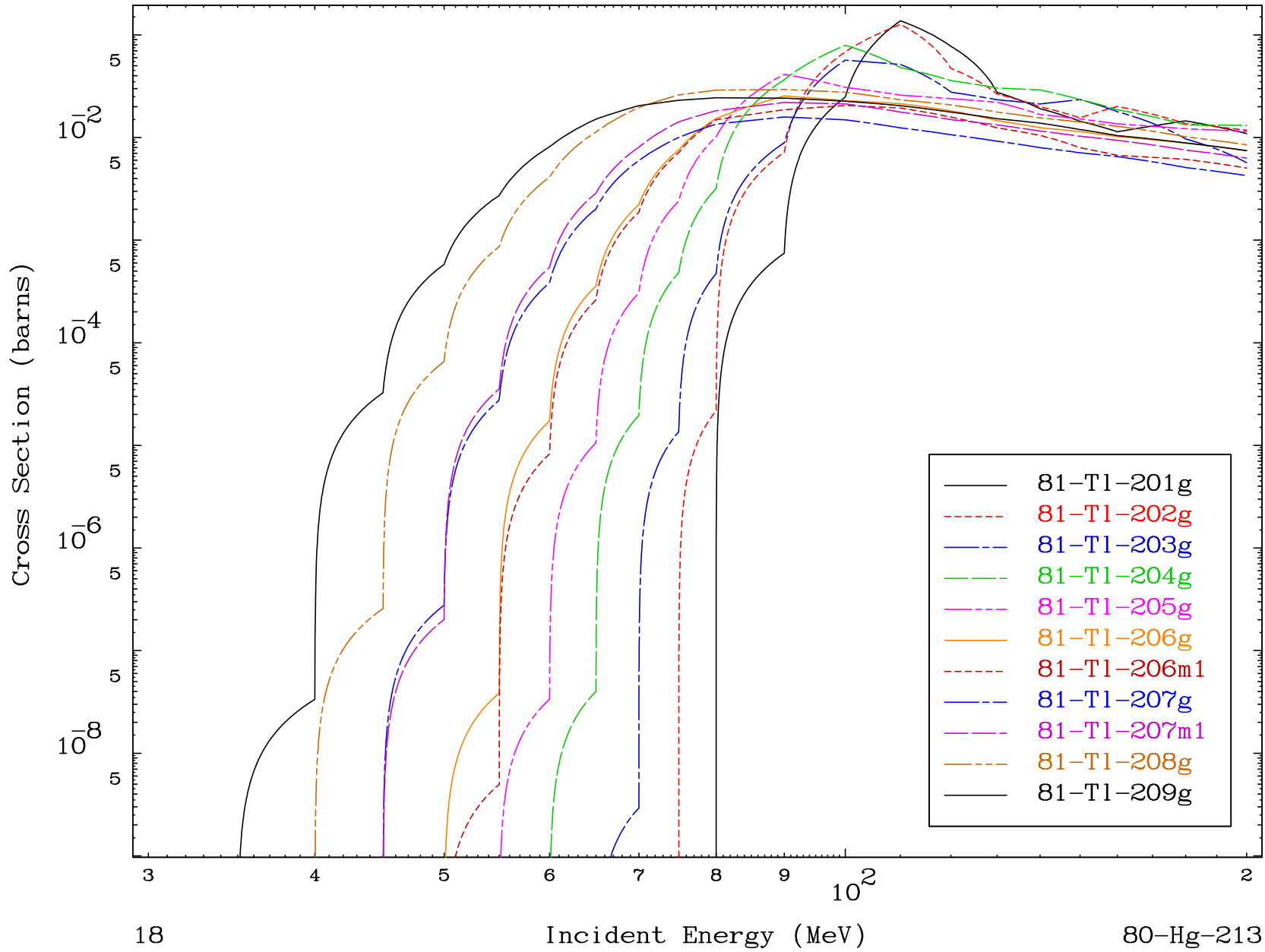
Radionuclide Production Cross Section



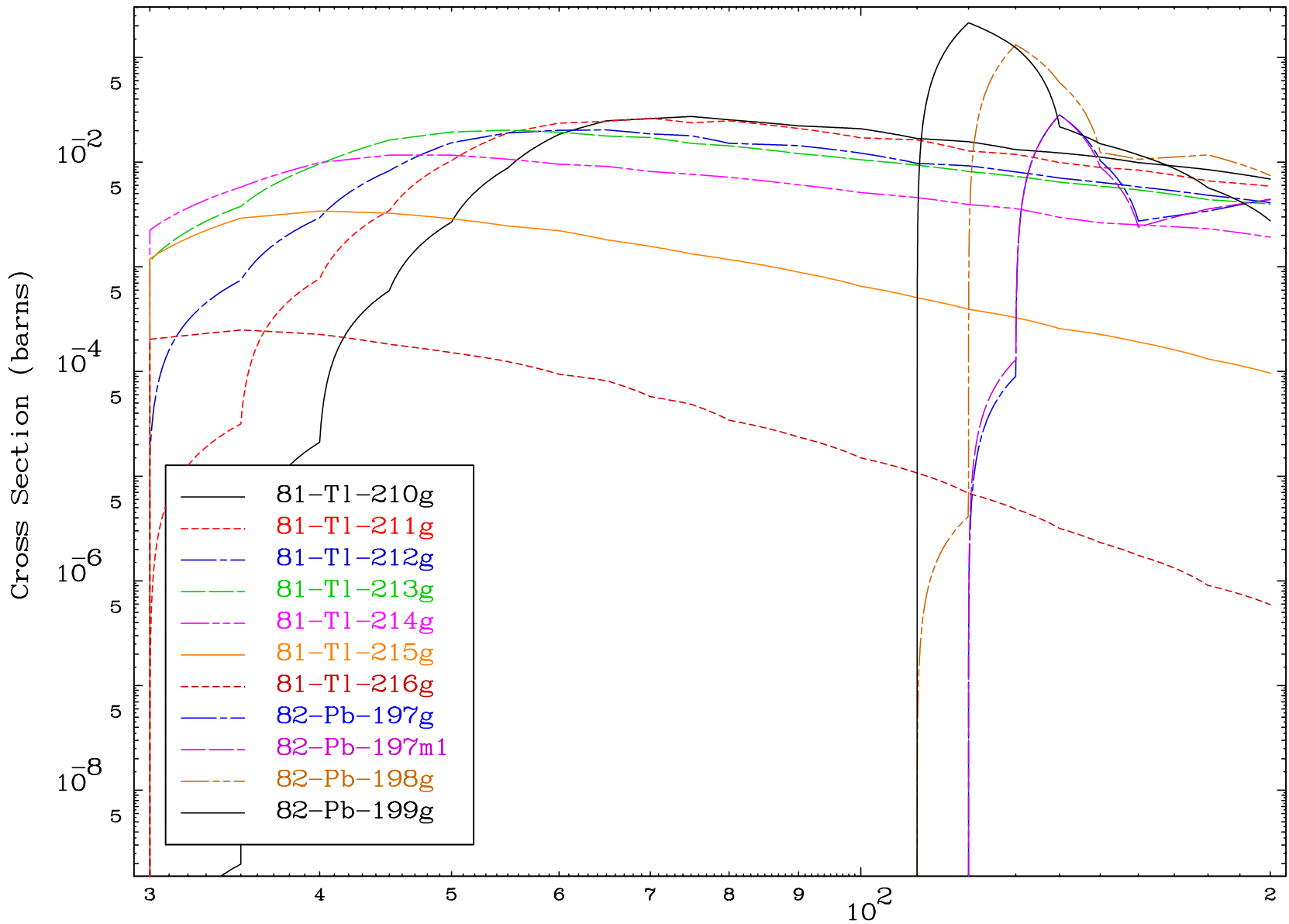




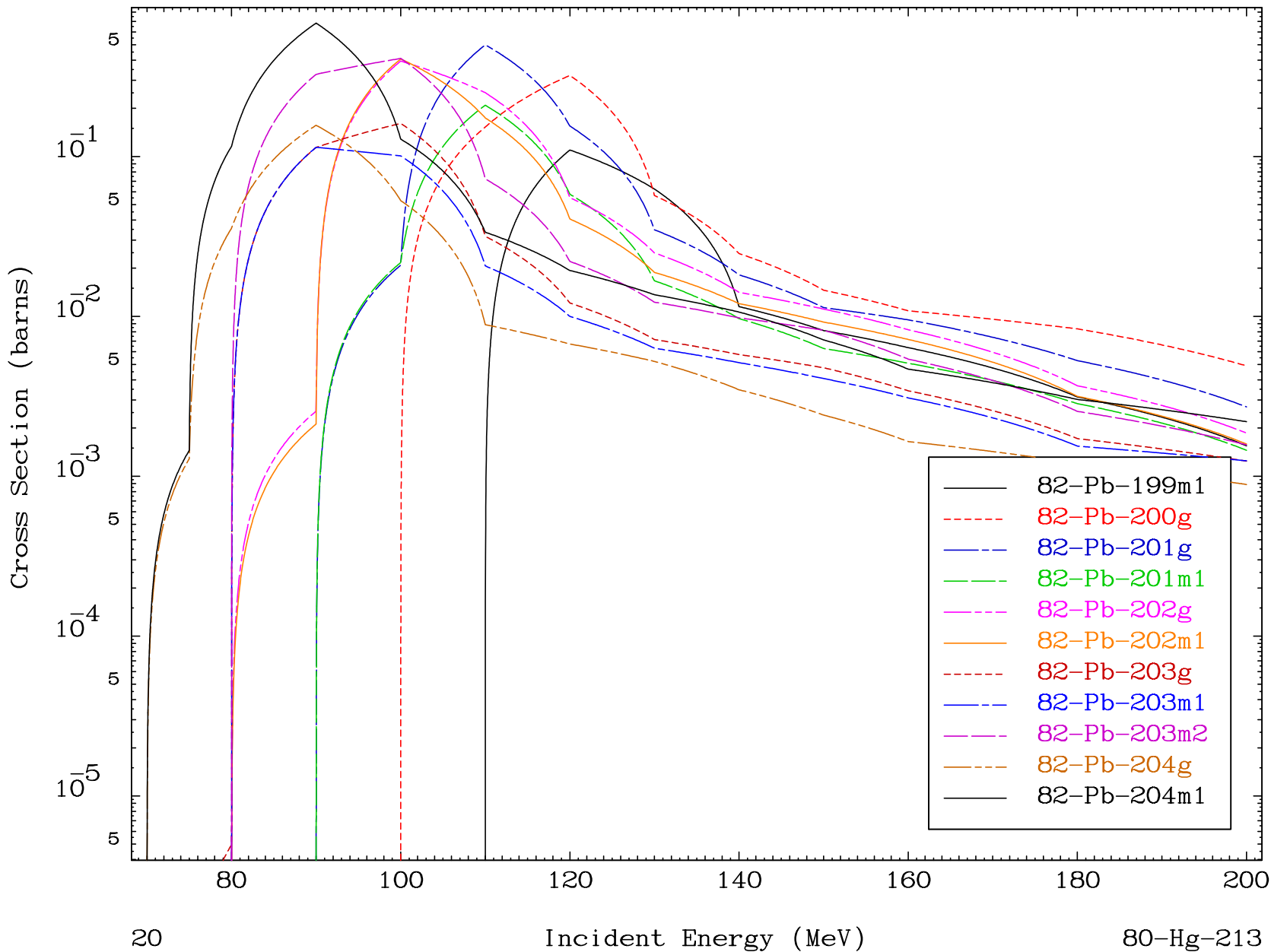


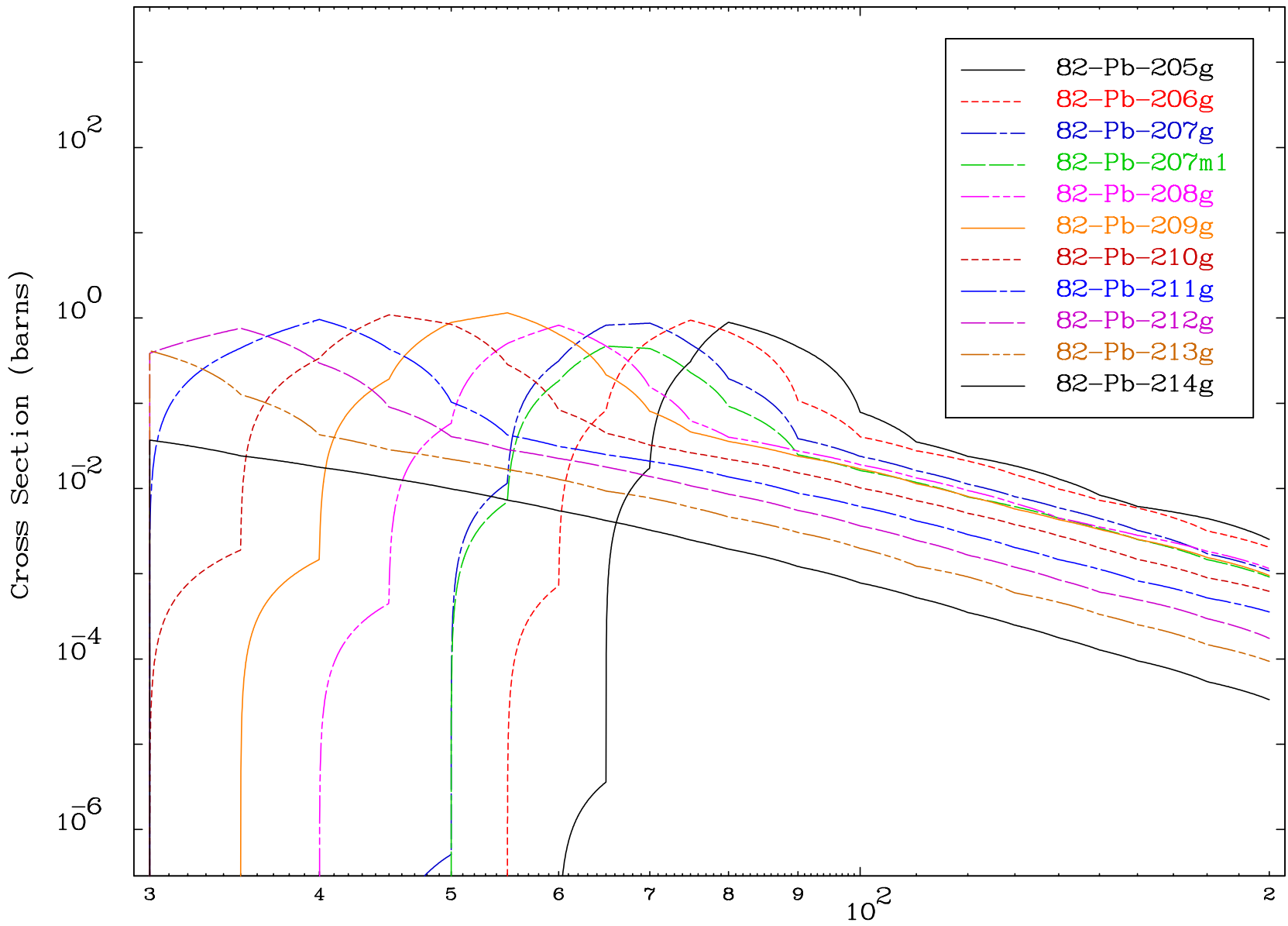


Radionuclide Production Cross Section



Radionuclide Production Cross Section

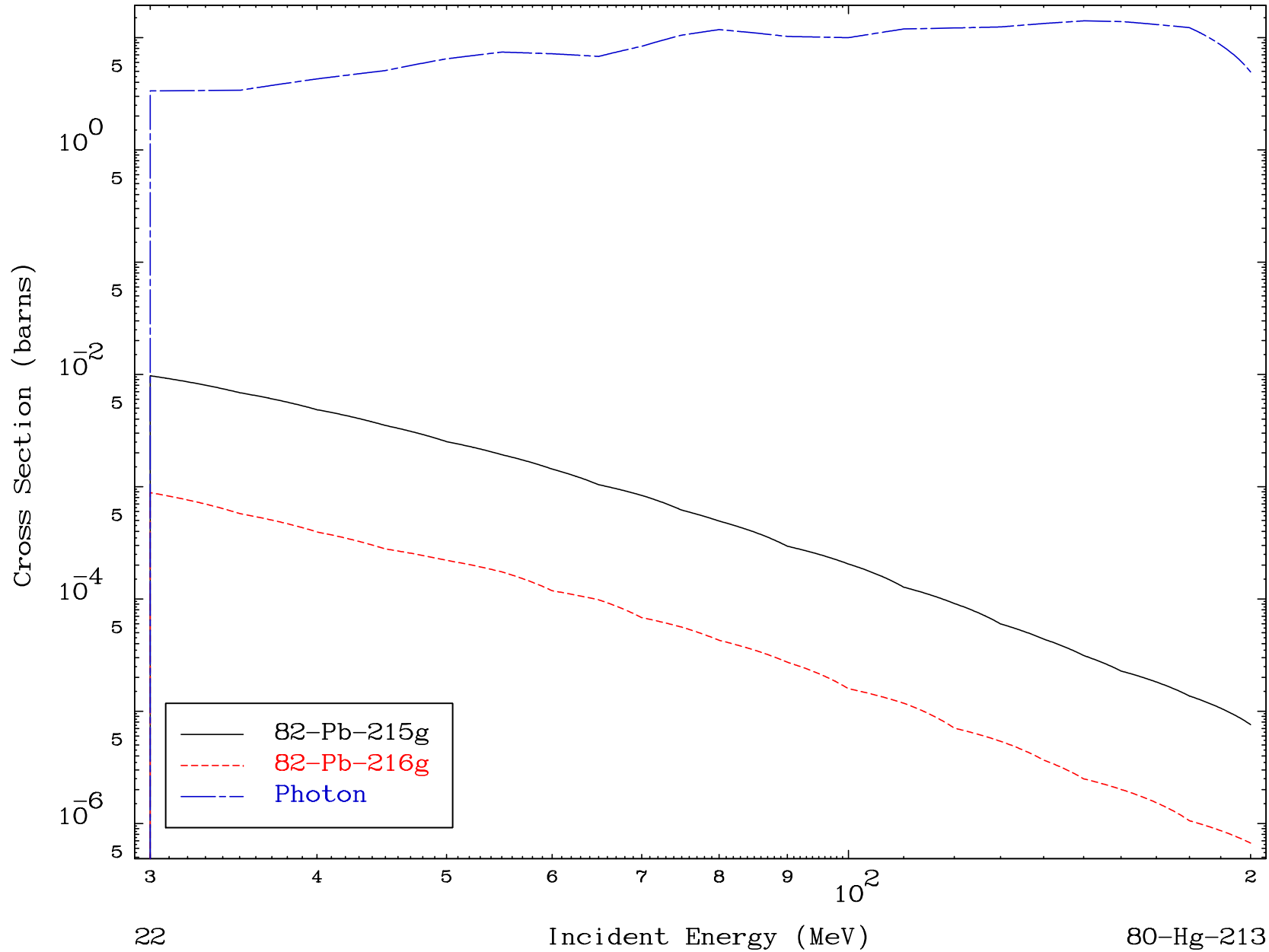




MAT 8076

( $\alpha$ , remainder)  
Radionuclide Production Cross Section

80-Hg-213



MAT 8076

$\alpha$  Fission  
Radionuclide Production Cross Section

80-Hg-213

