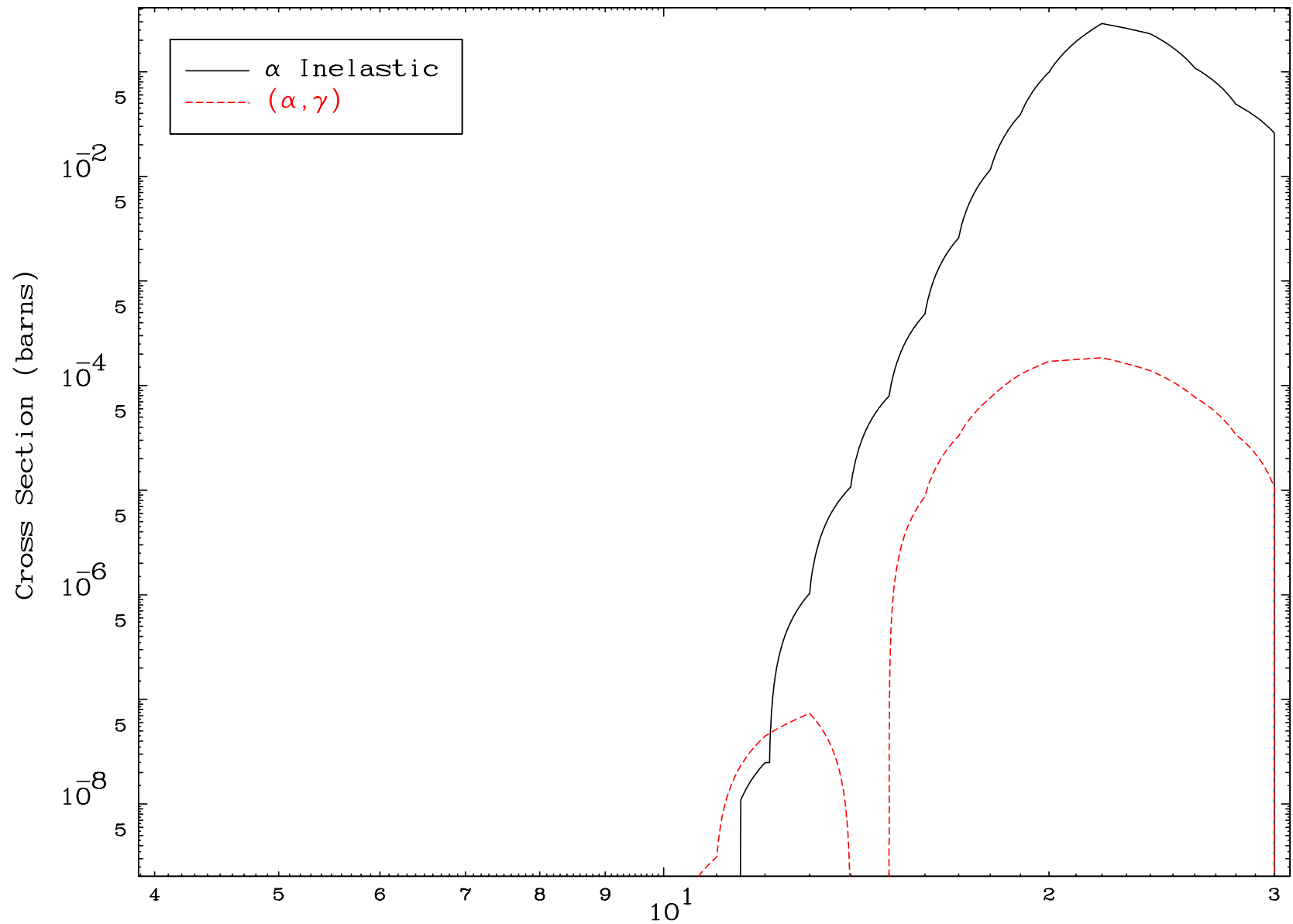
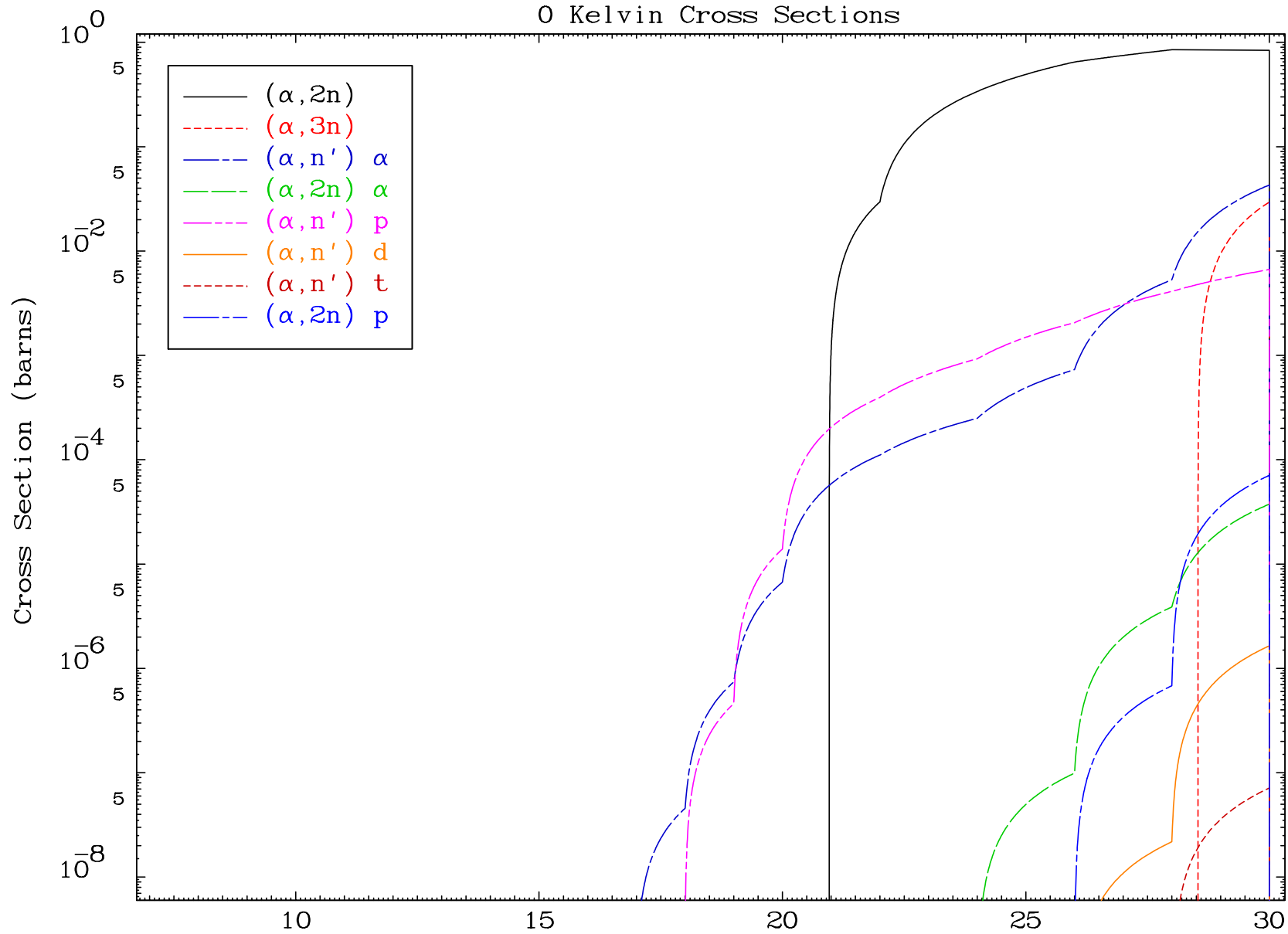




MAT 7710

 α Neutron Production
0 Kelvin Cross Sections

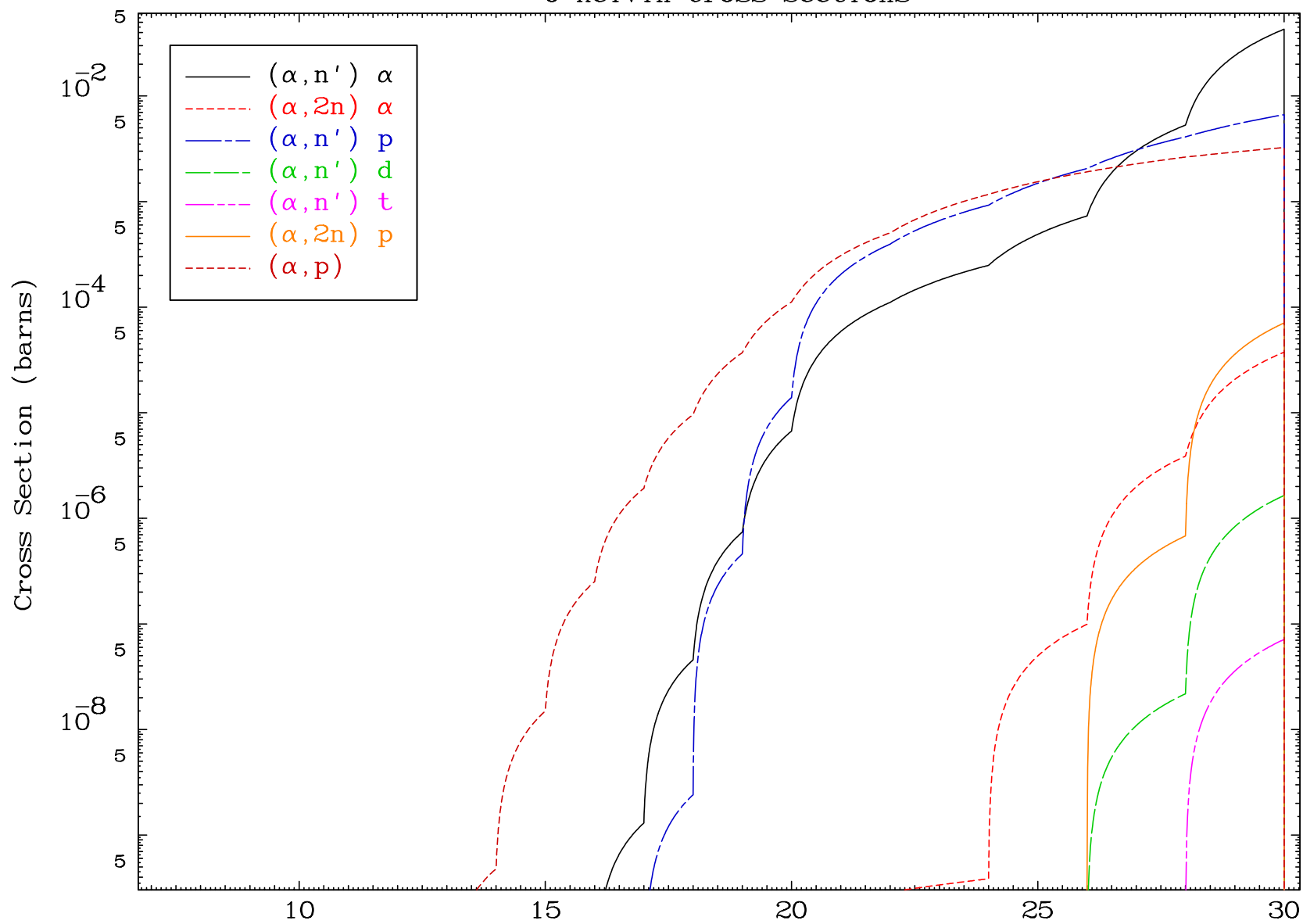
77-Ir-186

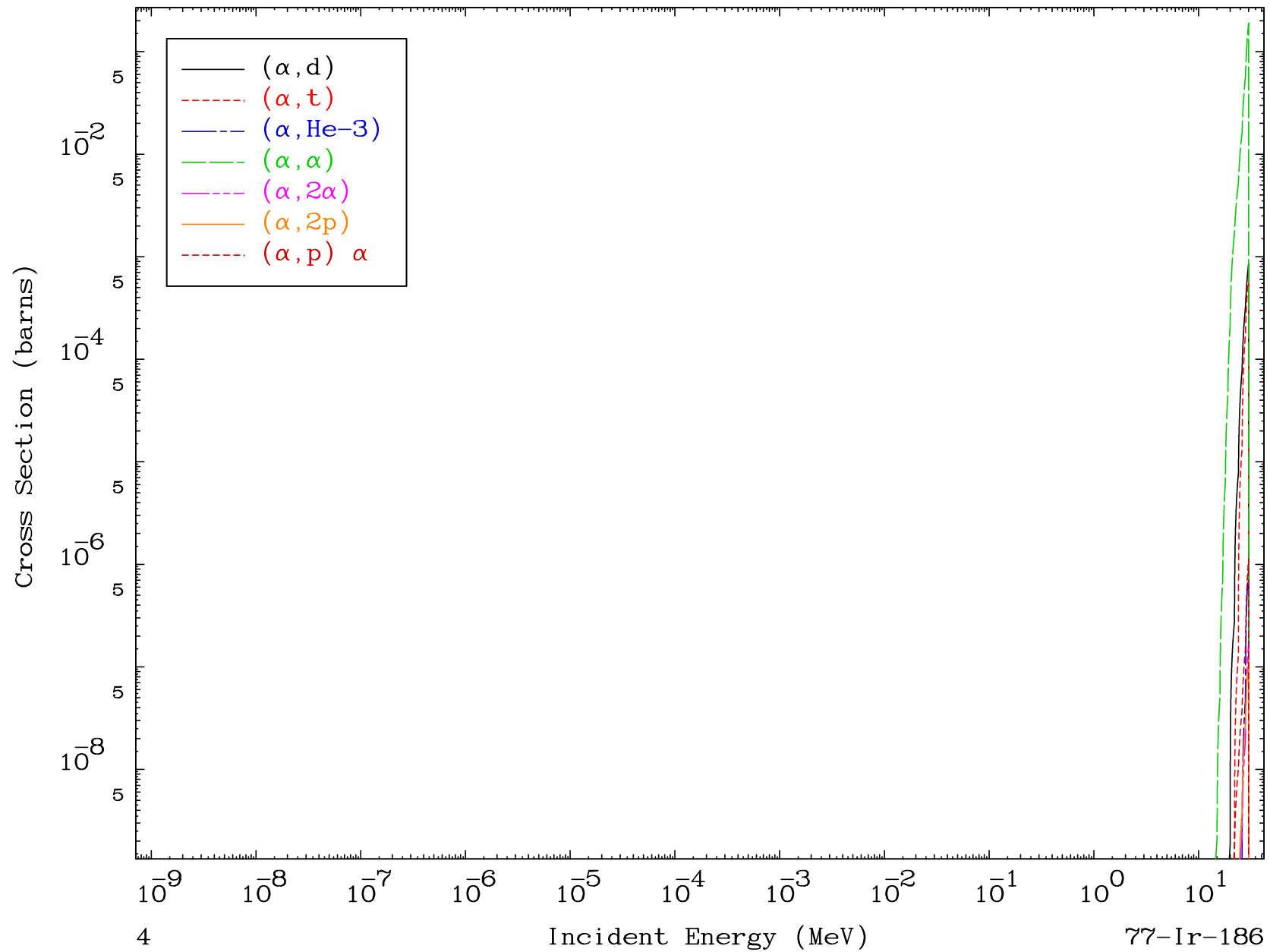


2

Incident Energy (MeV)

77-Ir-186

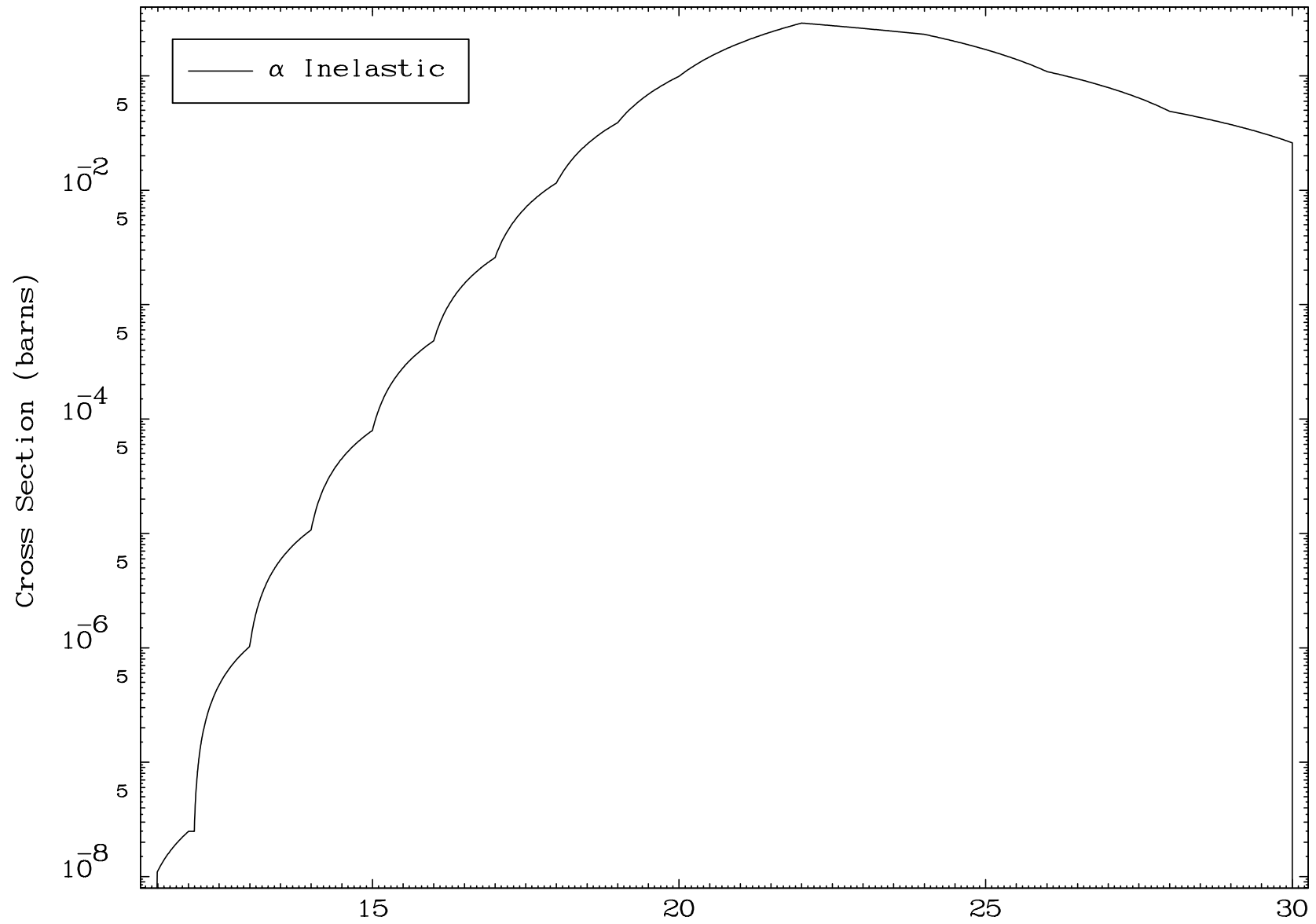




MAT 7710

(α, n') Level
0 Kelvin Cross Sections

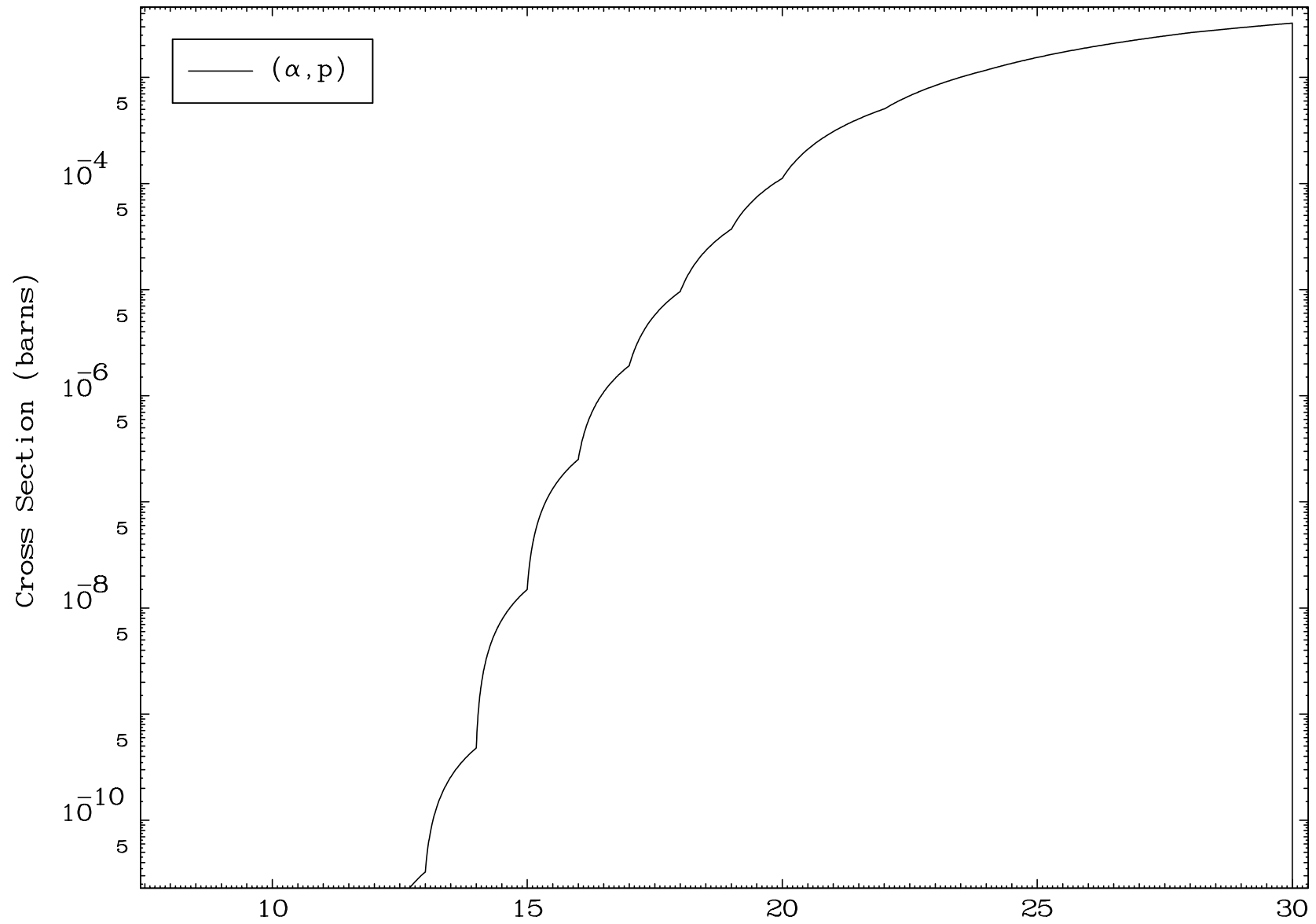
77-Ir-186

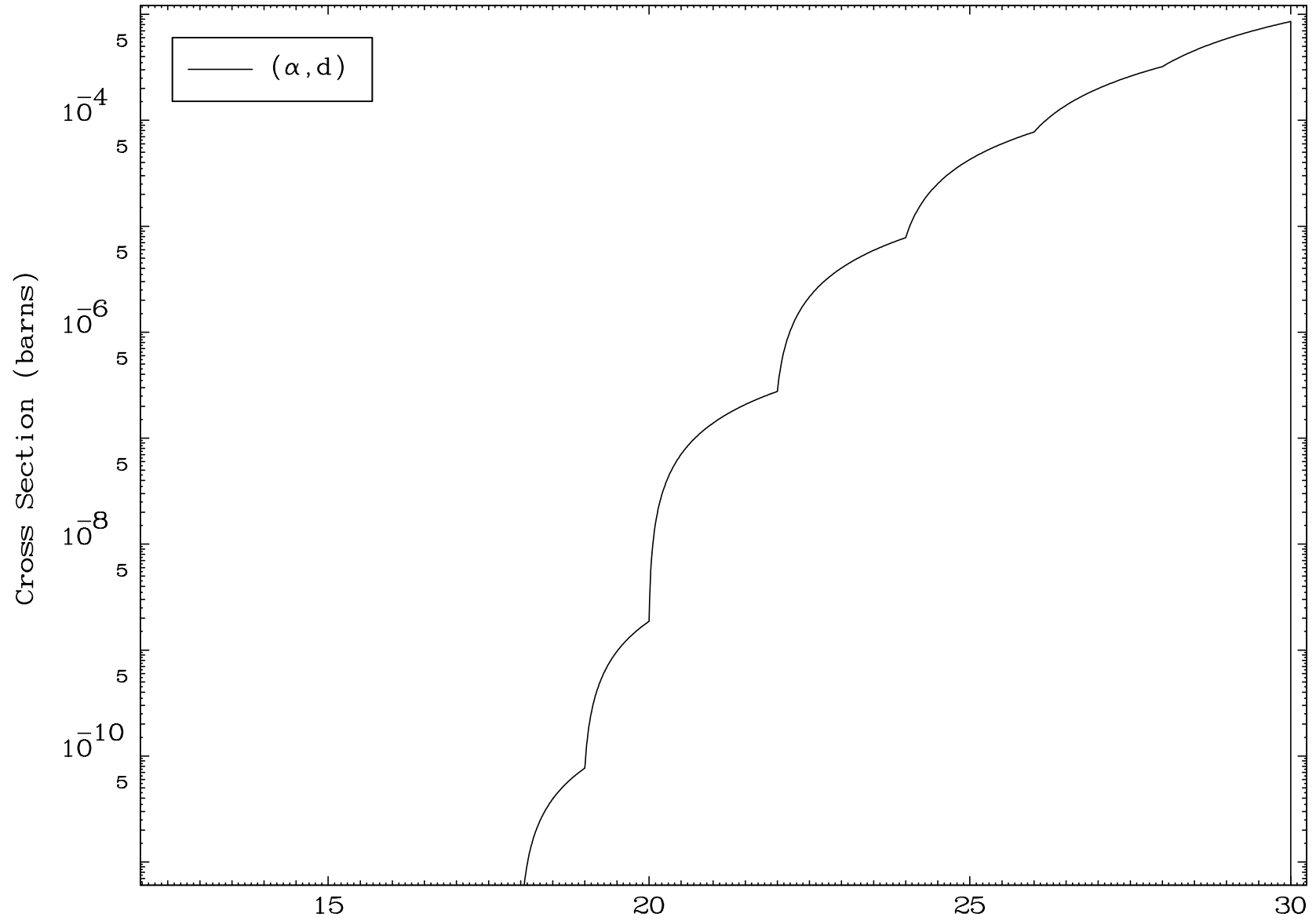


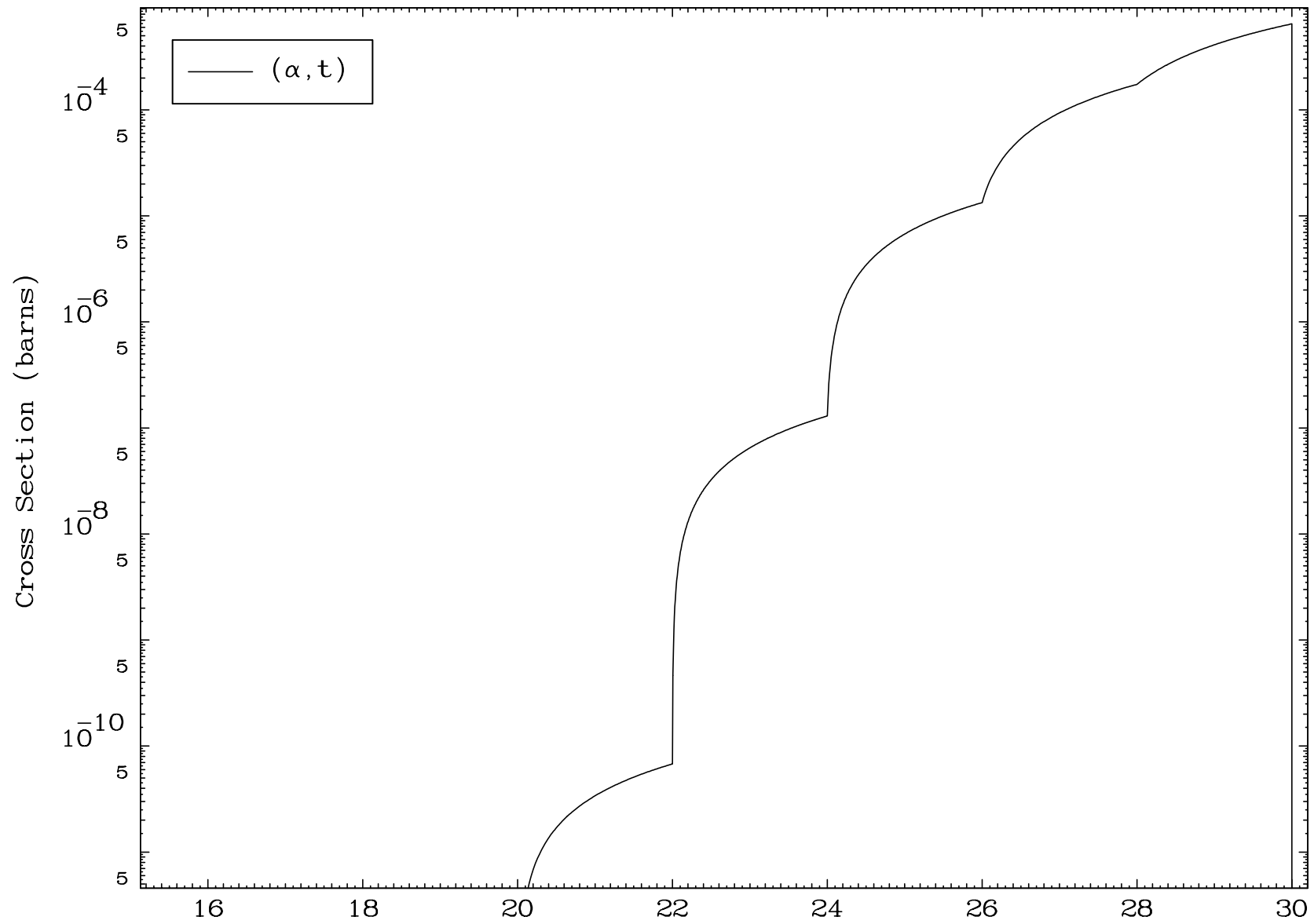
5

Incident Energy (MeV)

77-Ir-186



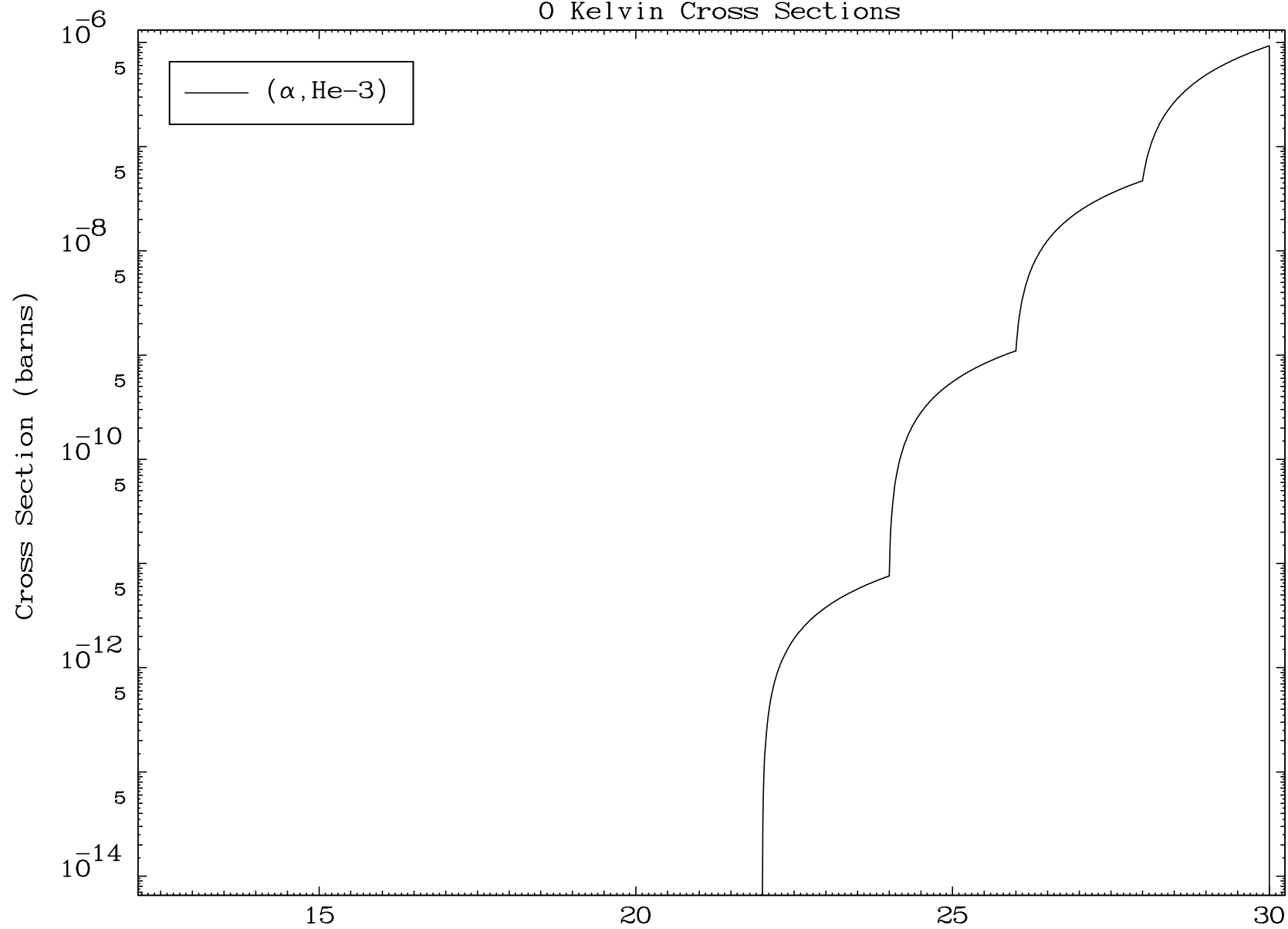




MAT 7710

 $(\alpha, \text{He-3})$ Levels
0 Kelvin Cross Sections

77-Ir-186



9

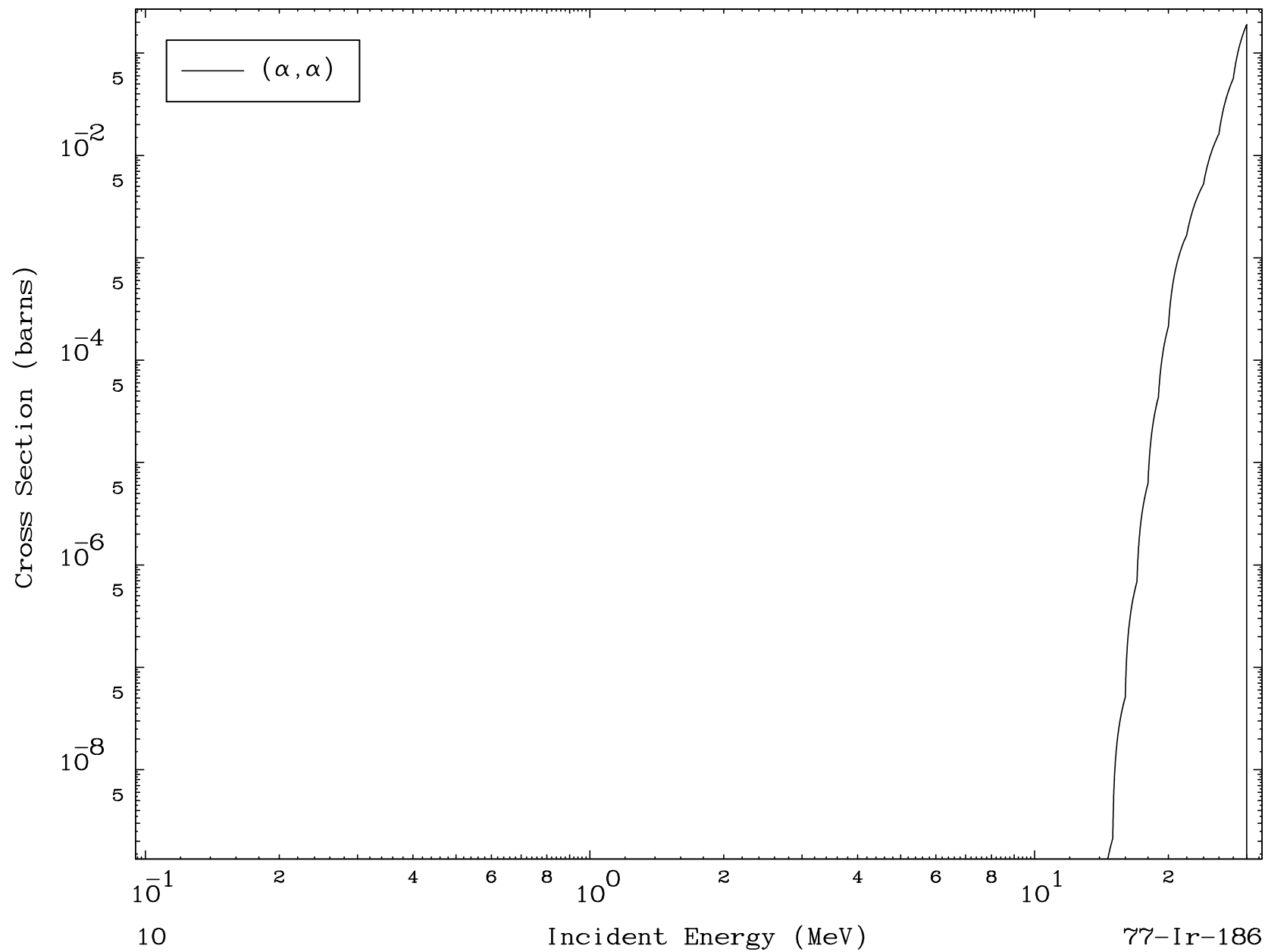
Incident Energy (MeV)

77-Ir-186

MAT 7710

(α, α) Levels
0 Kelvin Cross Sections

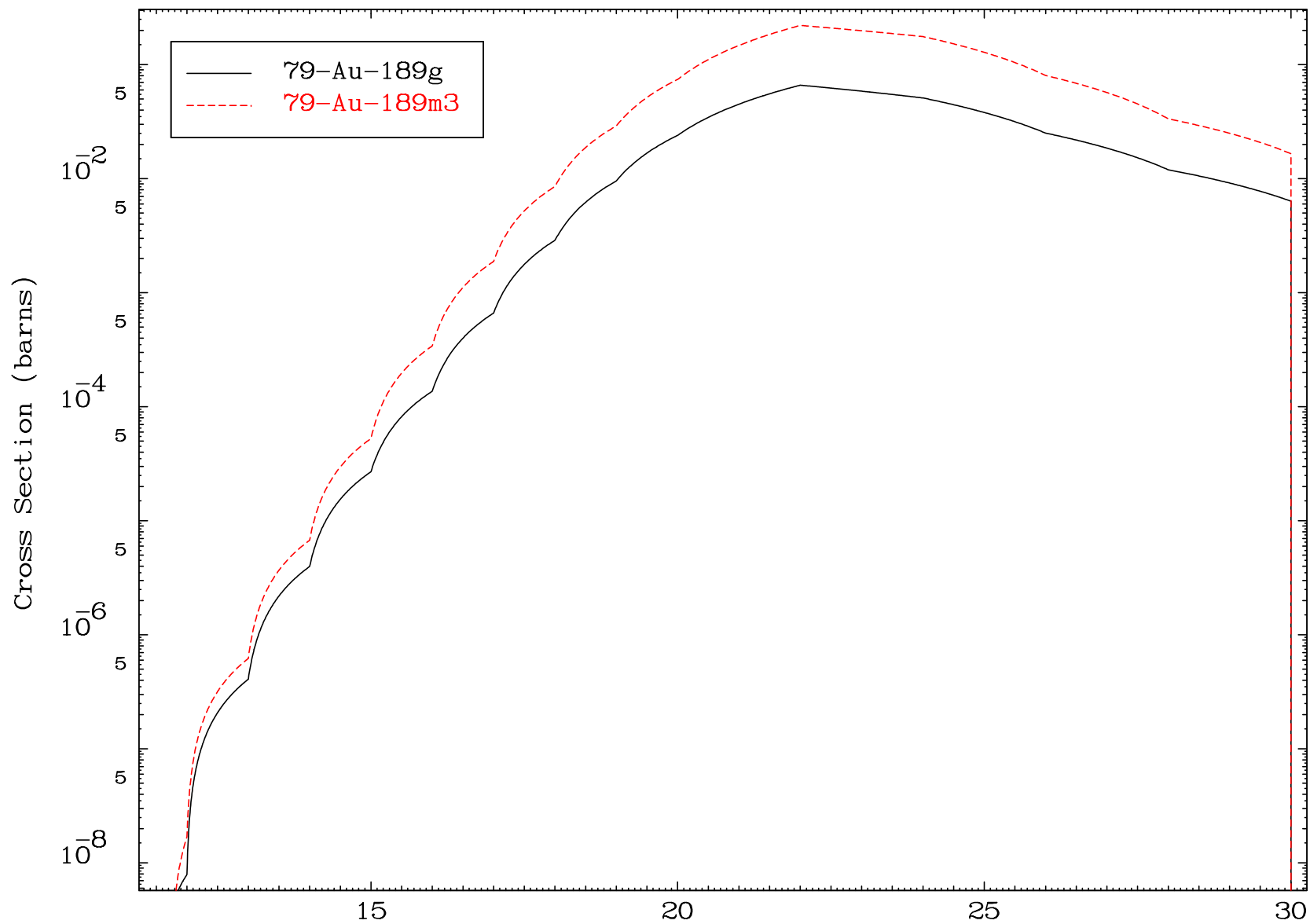
77-Ir-186



MAT 7710

 α Inelastic
Radionuclide Production Cross Section

77-Ir-186



11

Incident Energy (MeV)

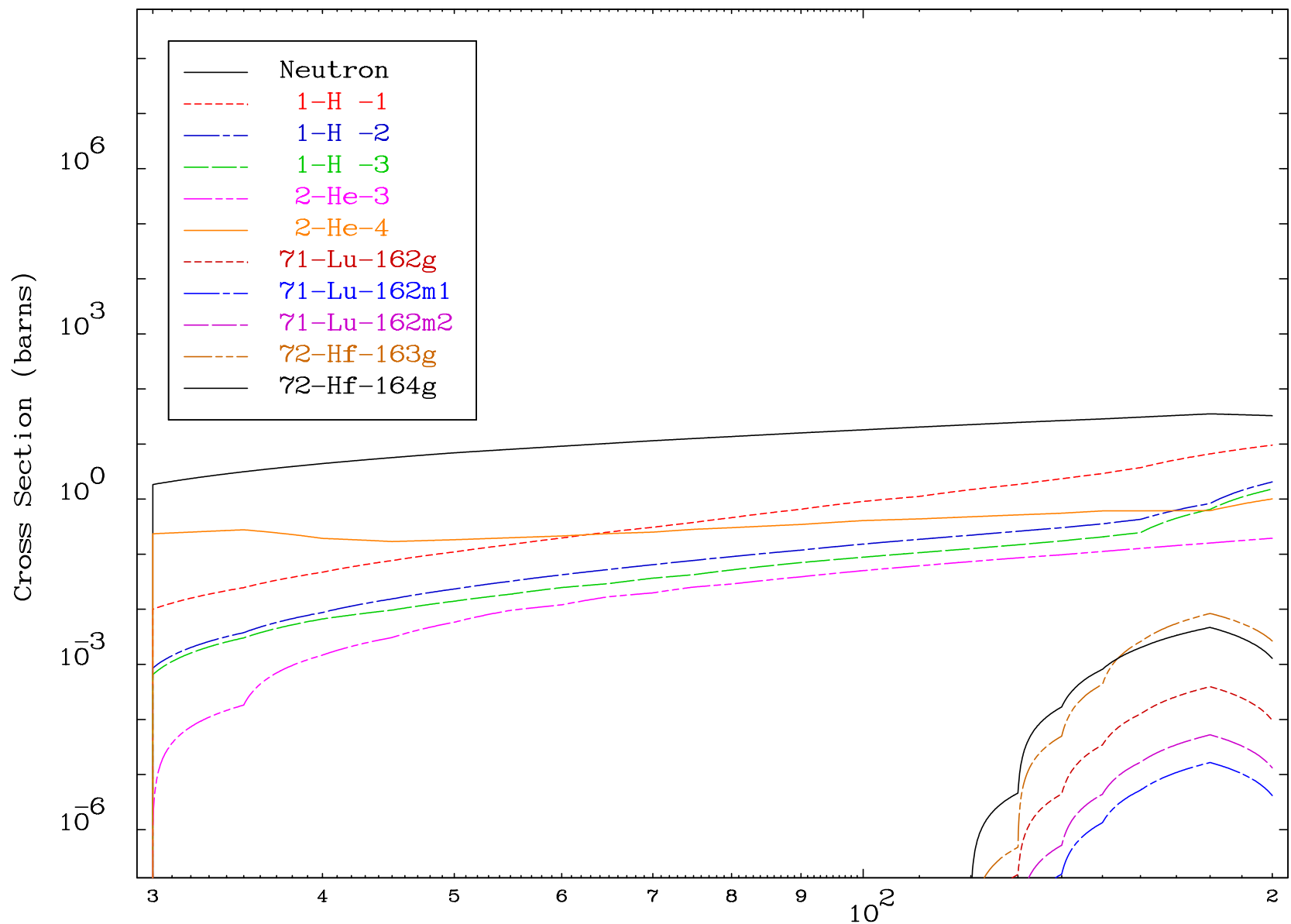
77-Ir-186

MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section



12

Incident Energy (MeV)

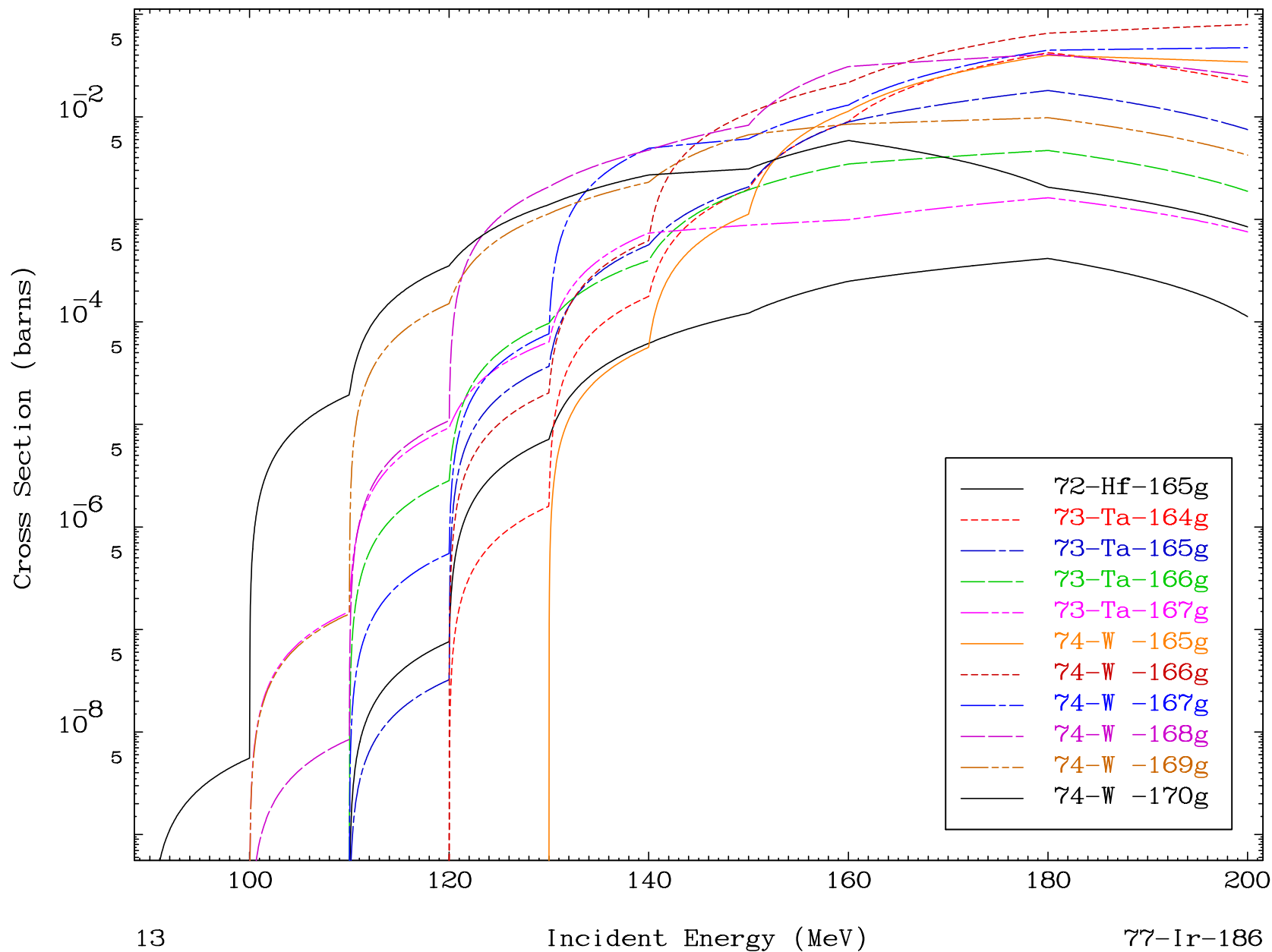
77-Ir-186

MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

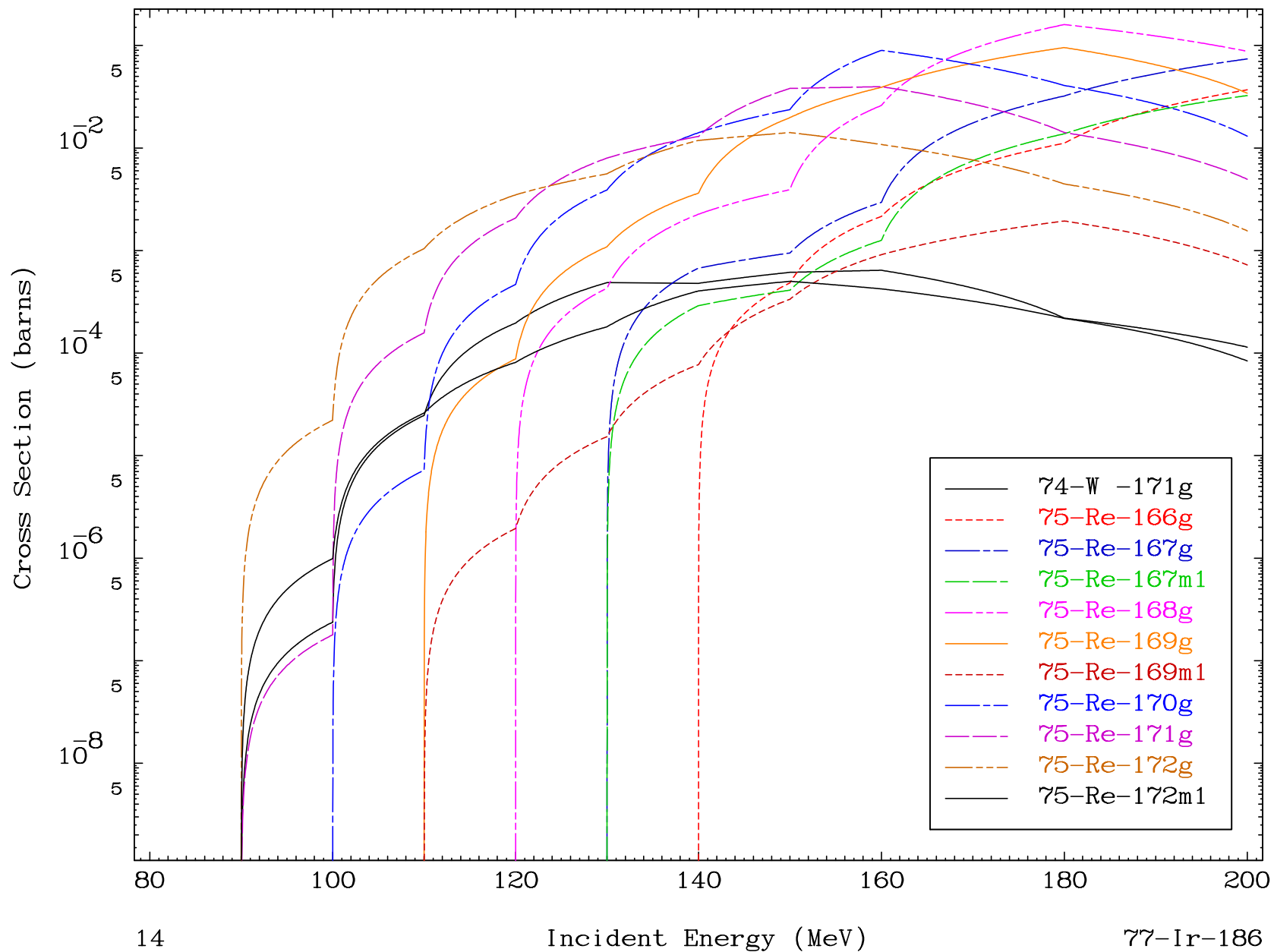


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

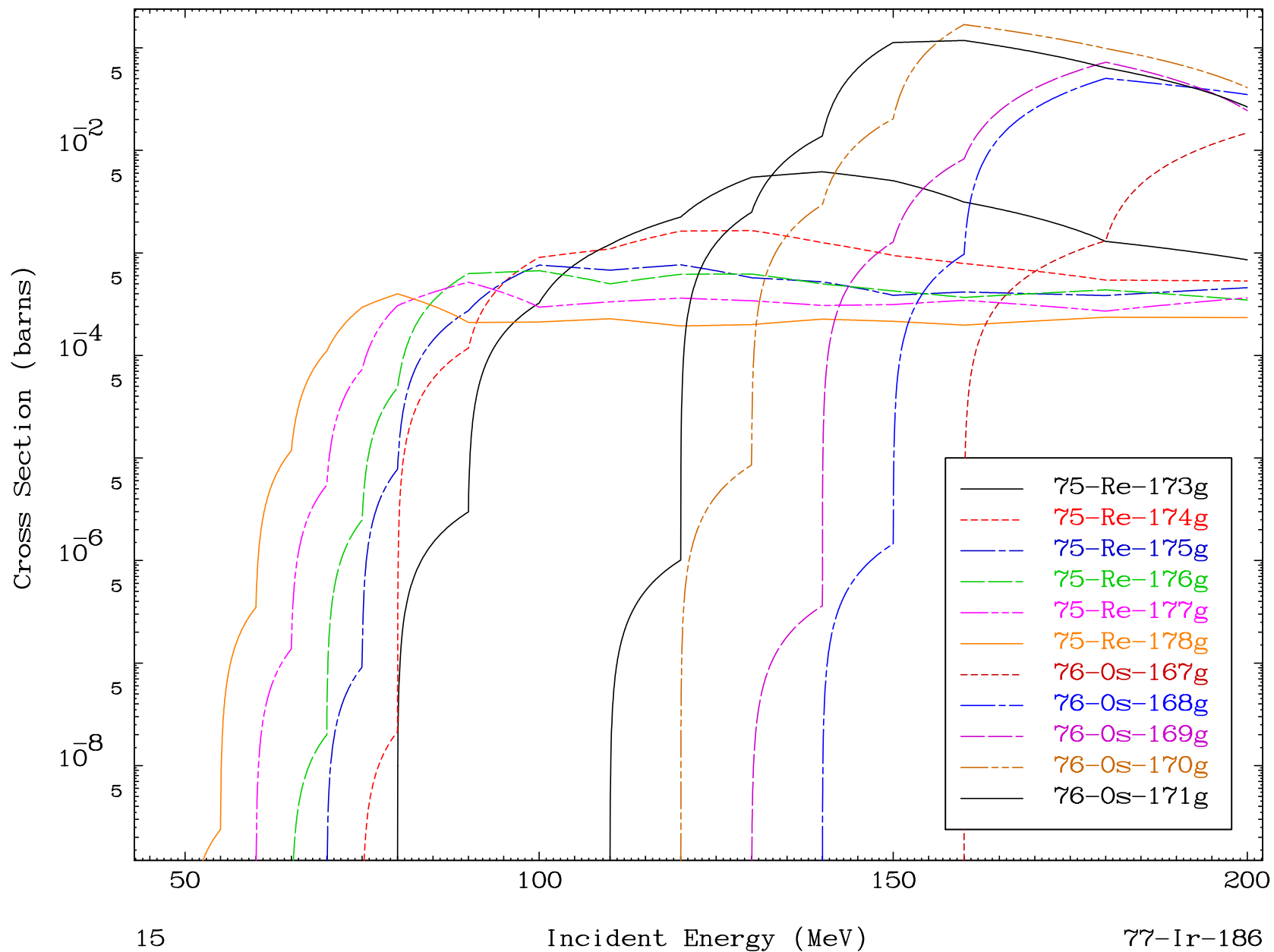


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

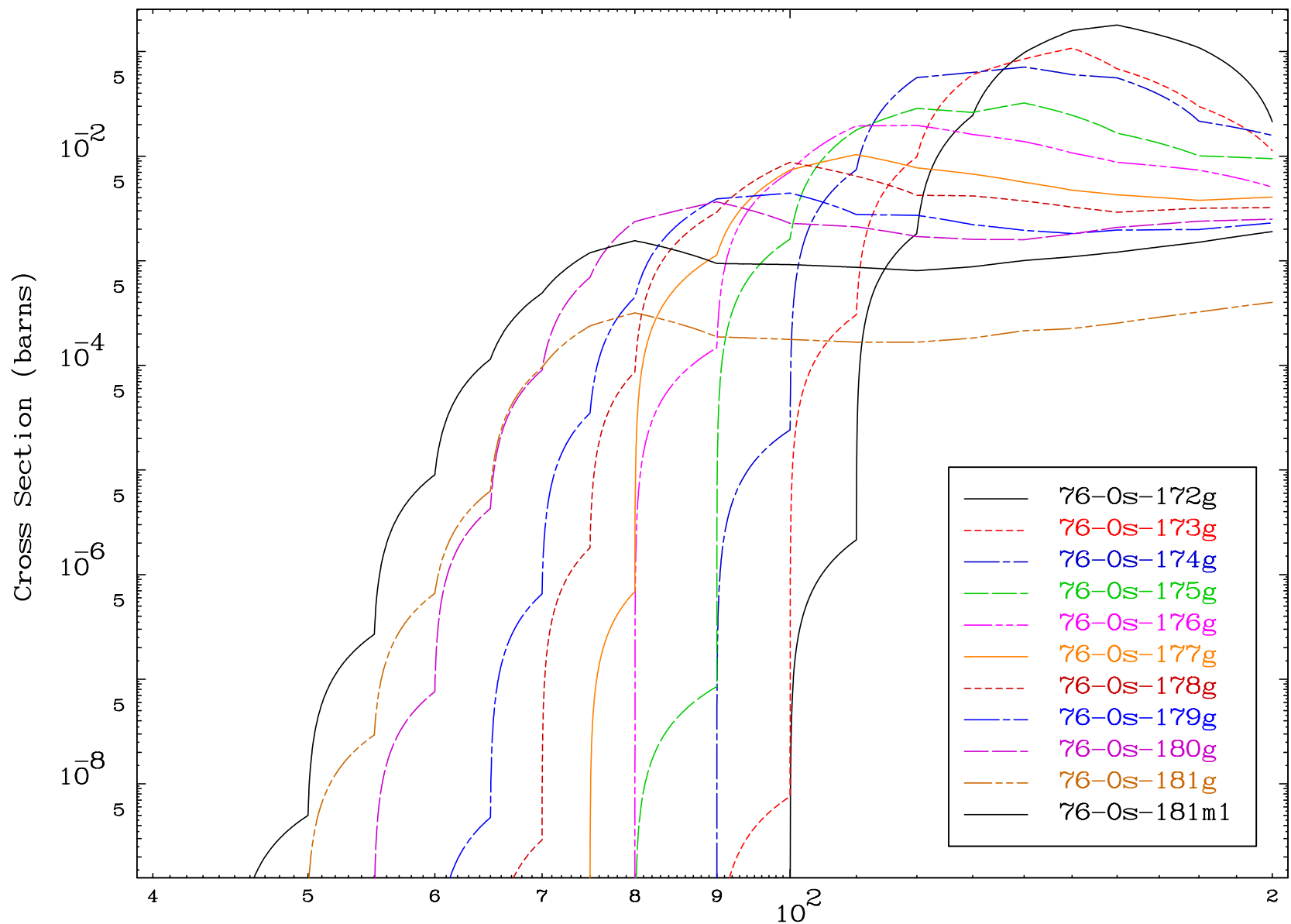


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section



16

Incident Energy (MeV)

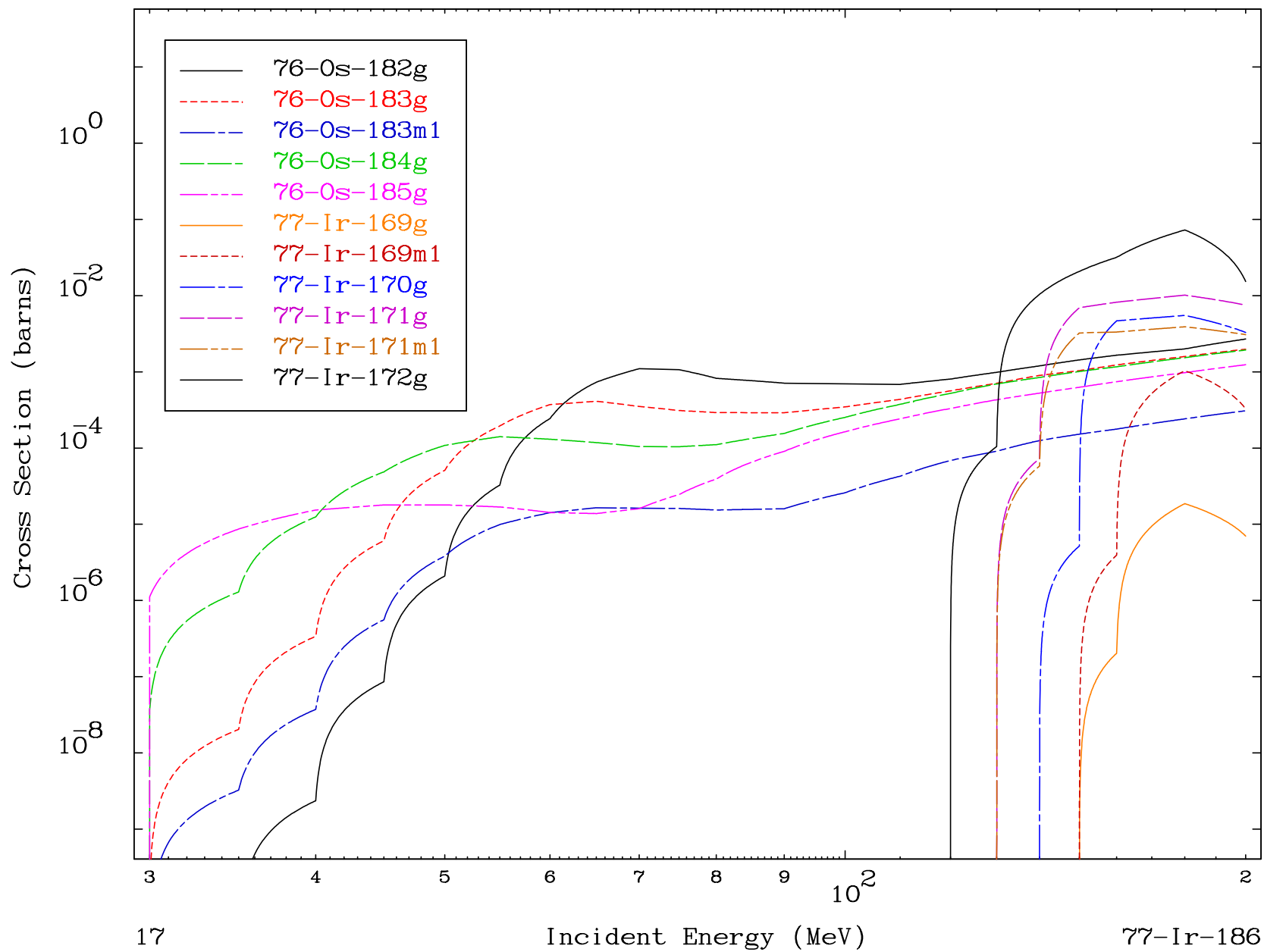
77-Ir-186

MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

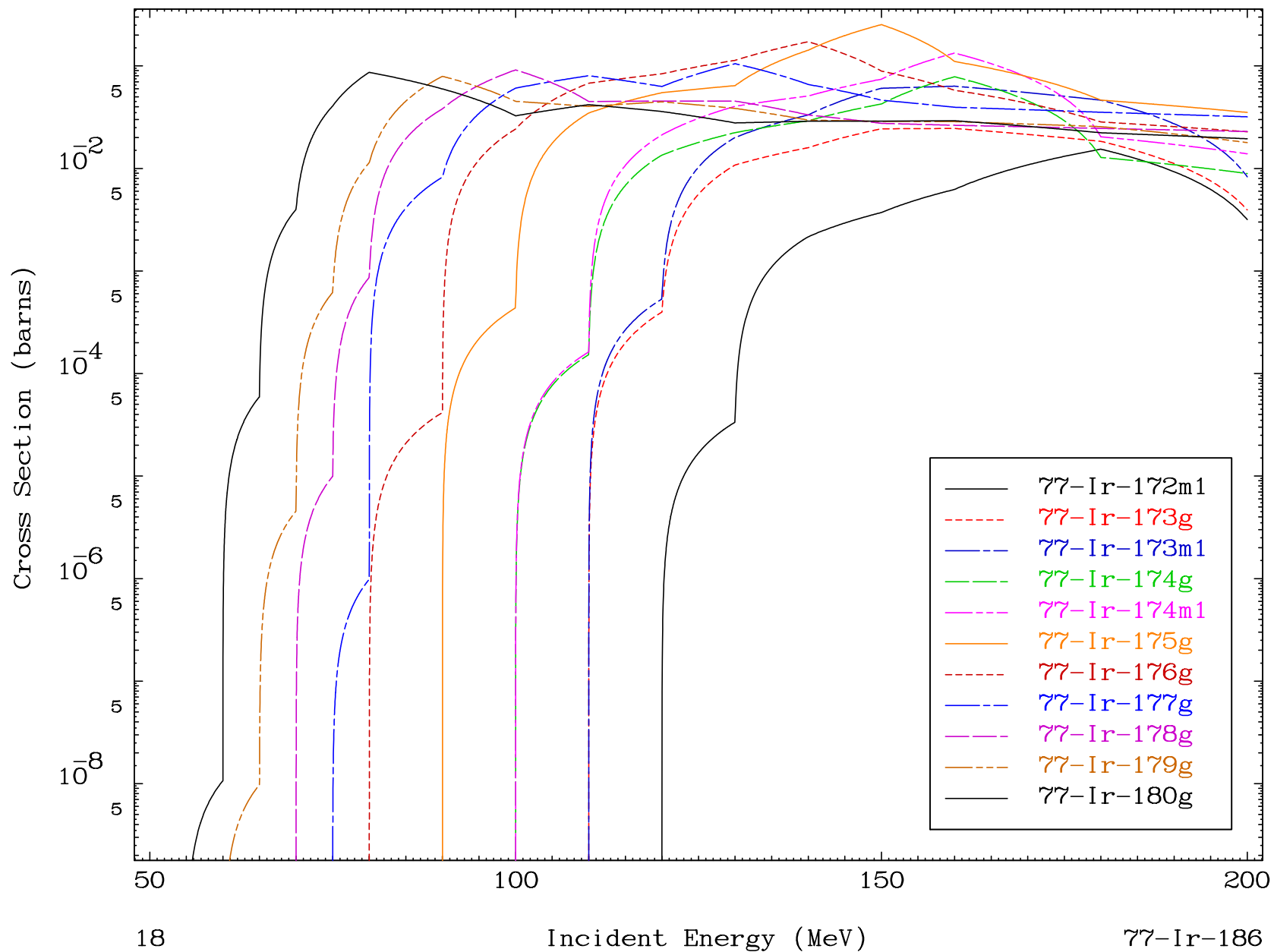


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

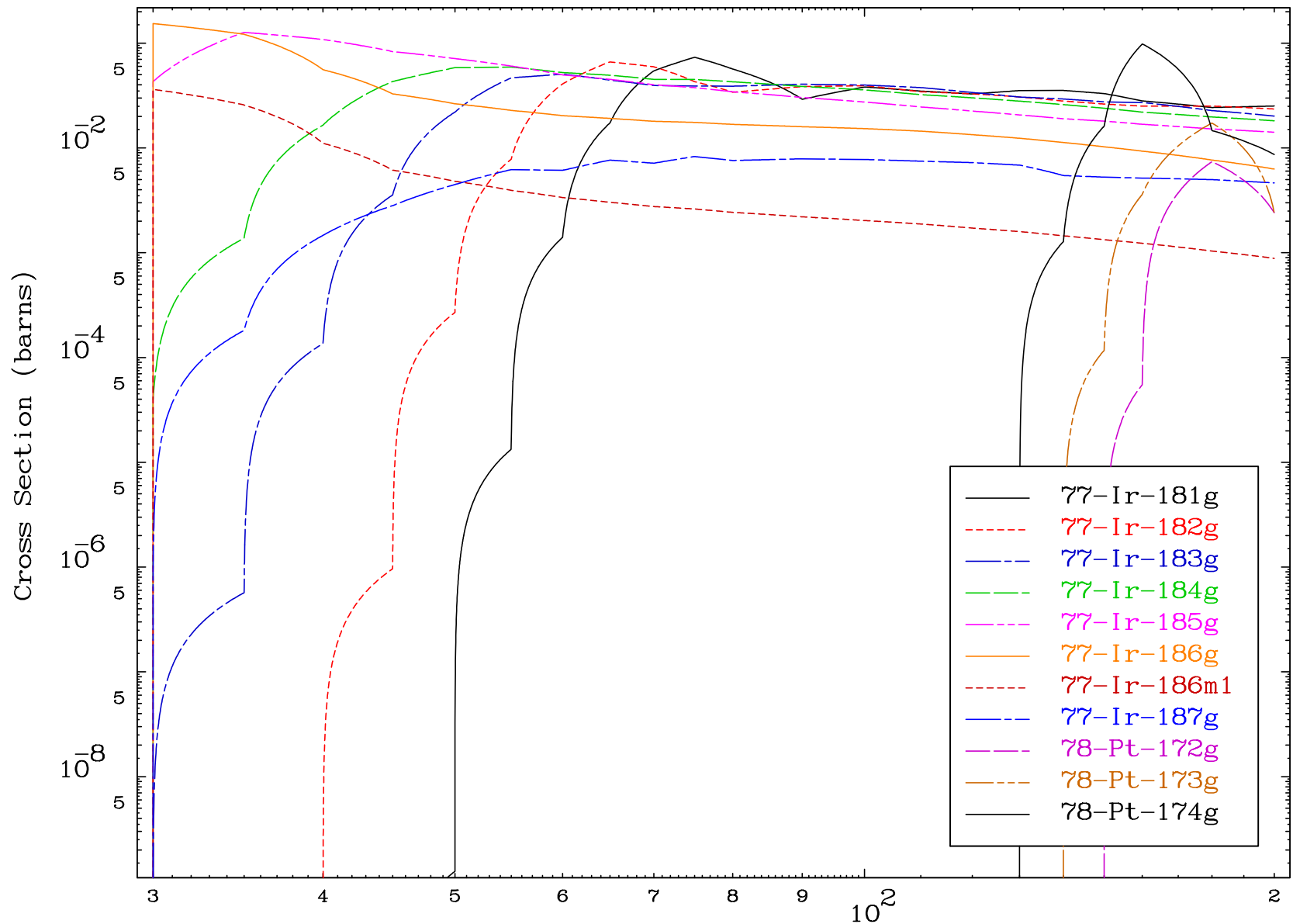


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section



19

Incident Energy (MeV)

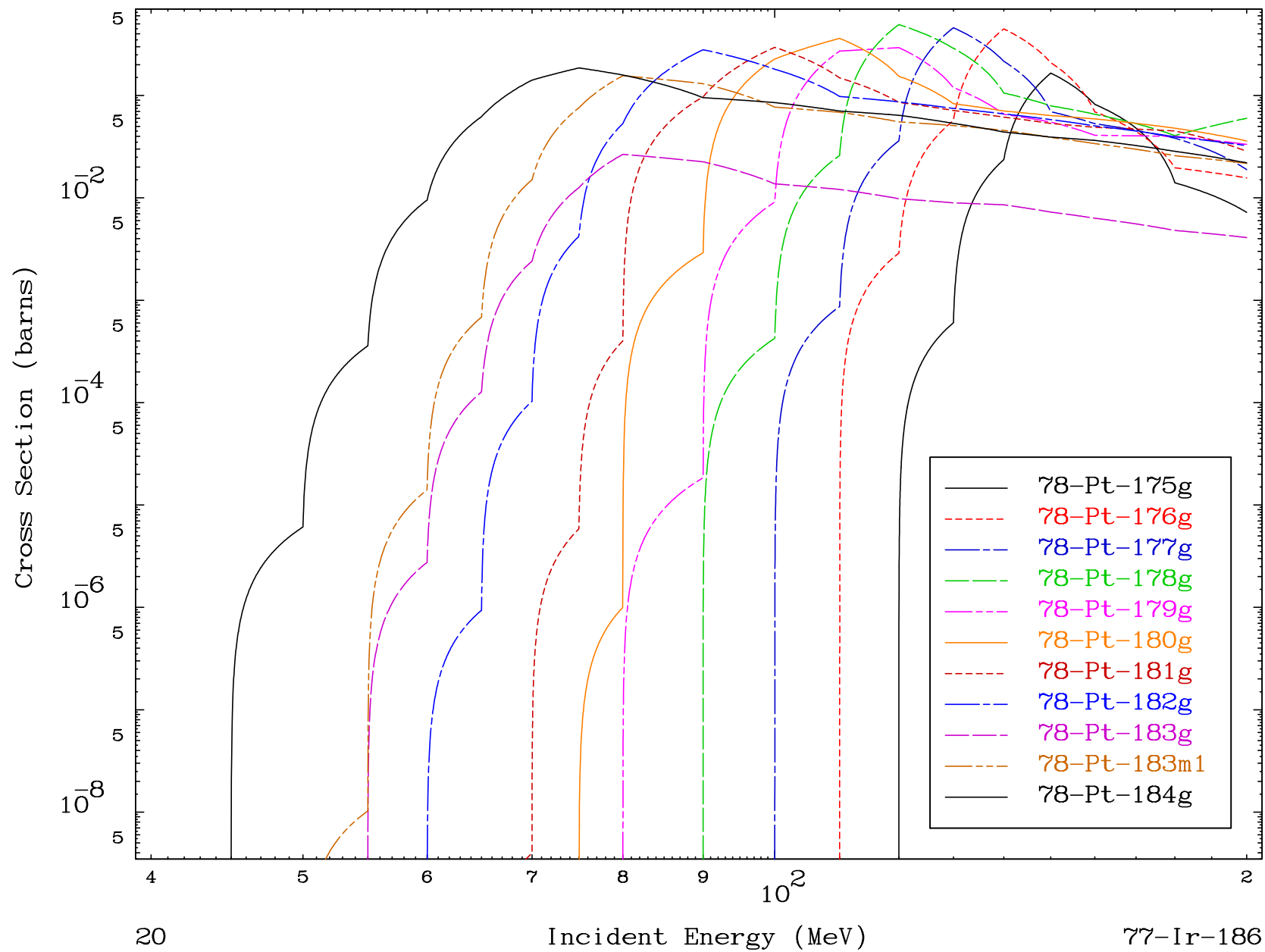
77-Ir-186

MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

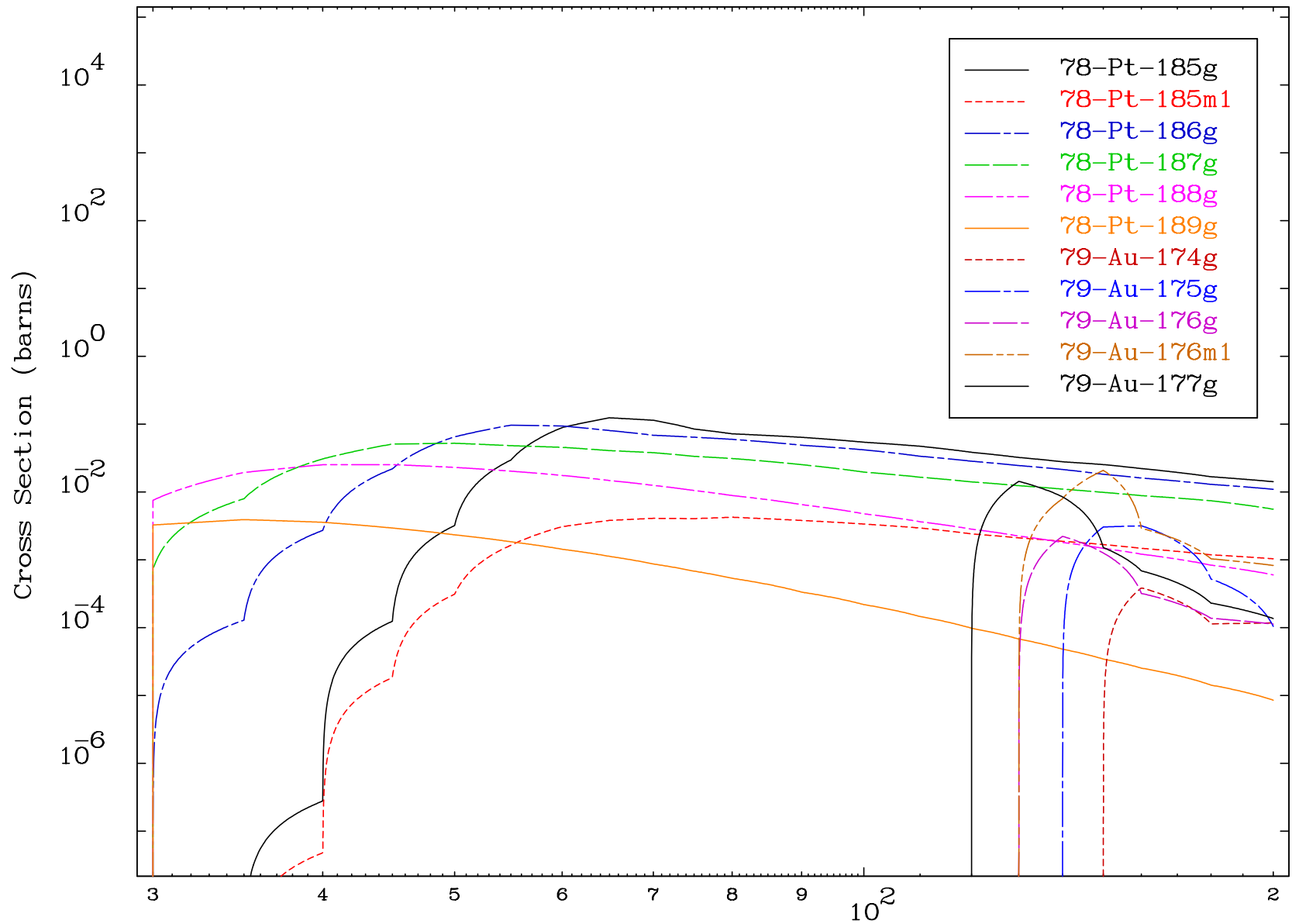


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section



21

Incident Energy (MeV)

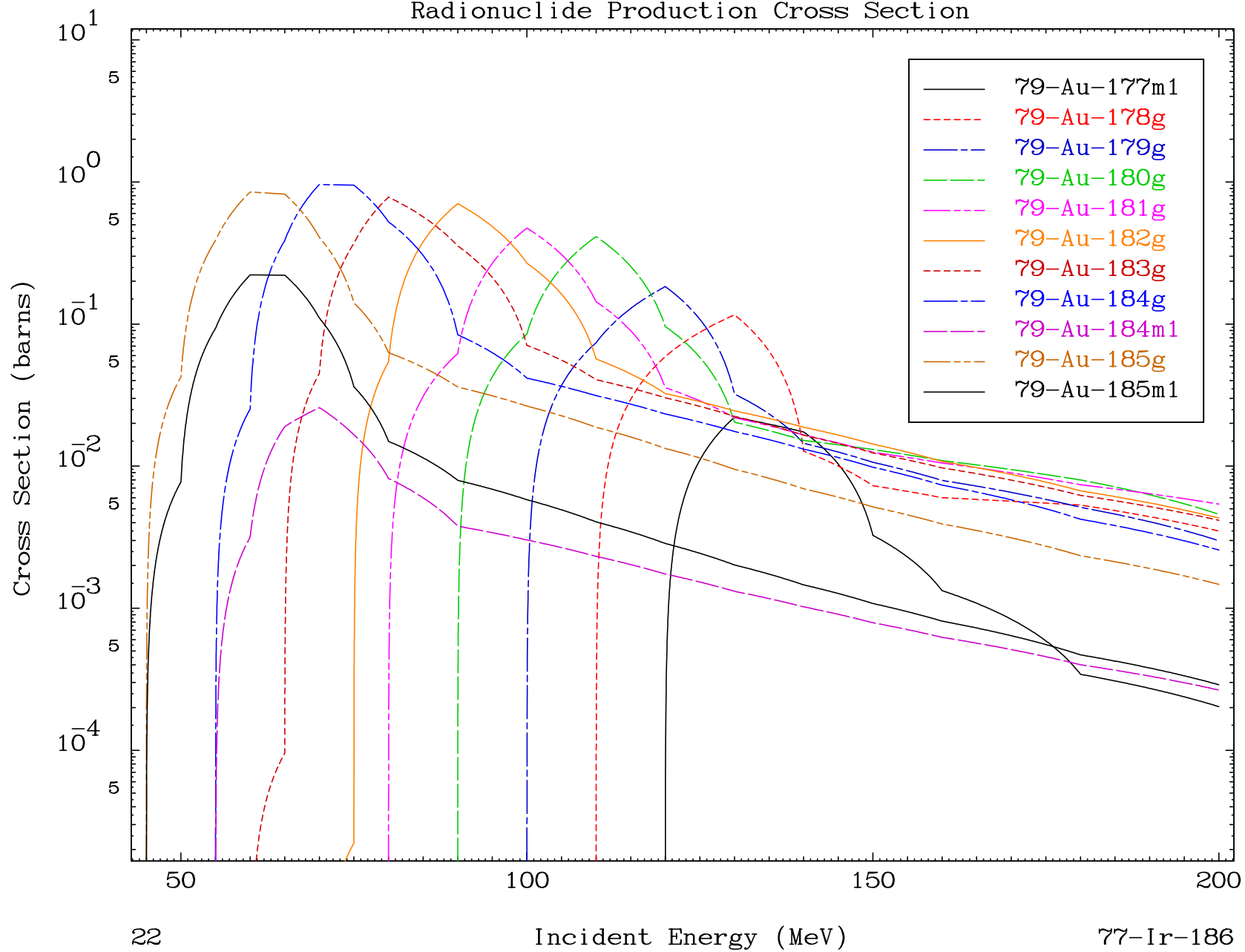
77-Ir-186

MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section

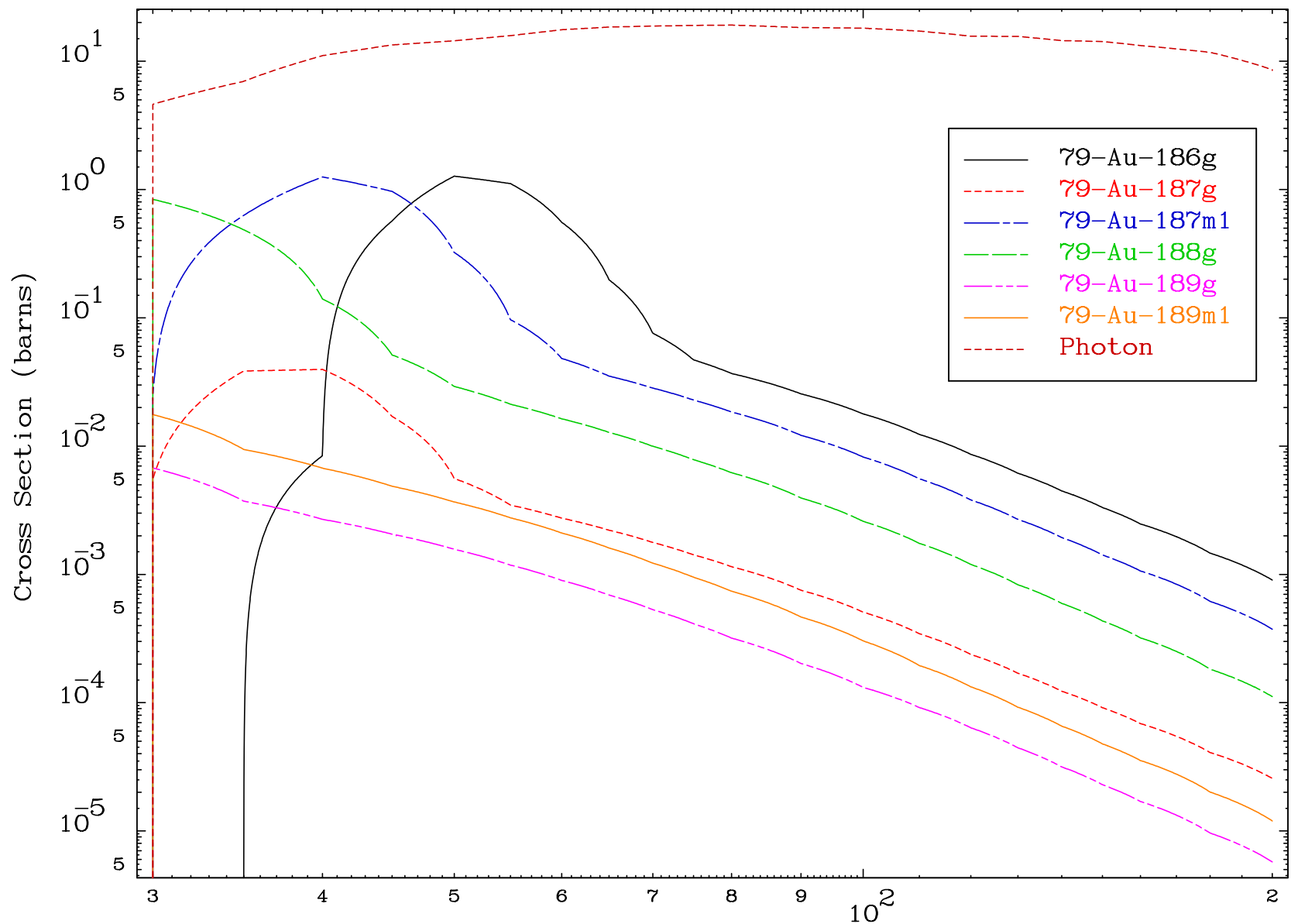


MAT 7710

 $(\alpha, \text{remainder})$

77-Ir-186

Radionuclide Production Cross Section



23

Incident Energy (MeV)

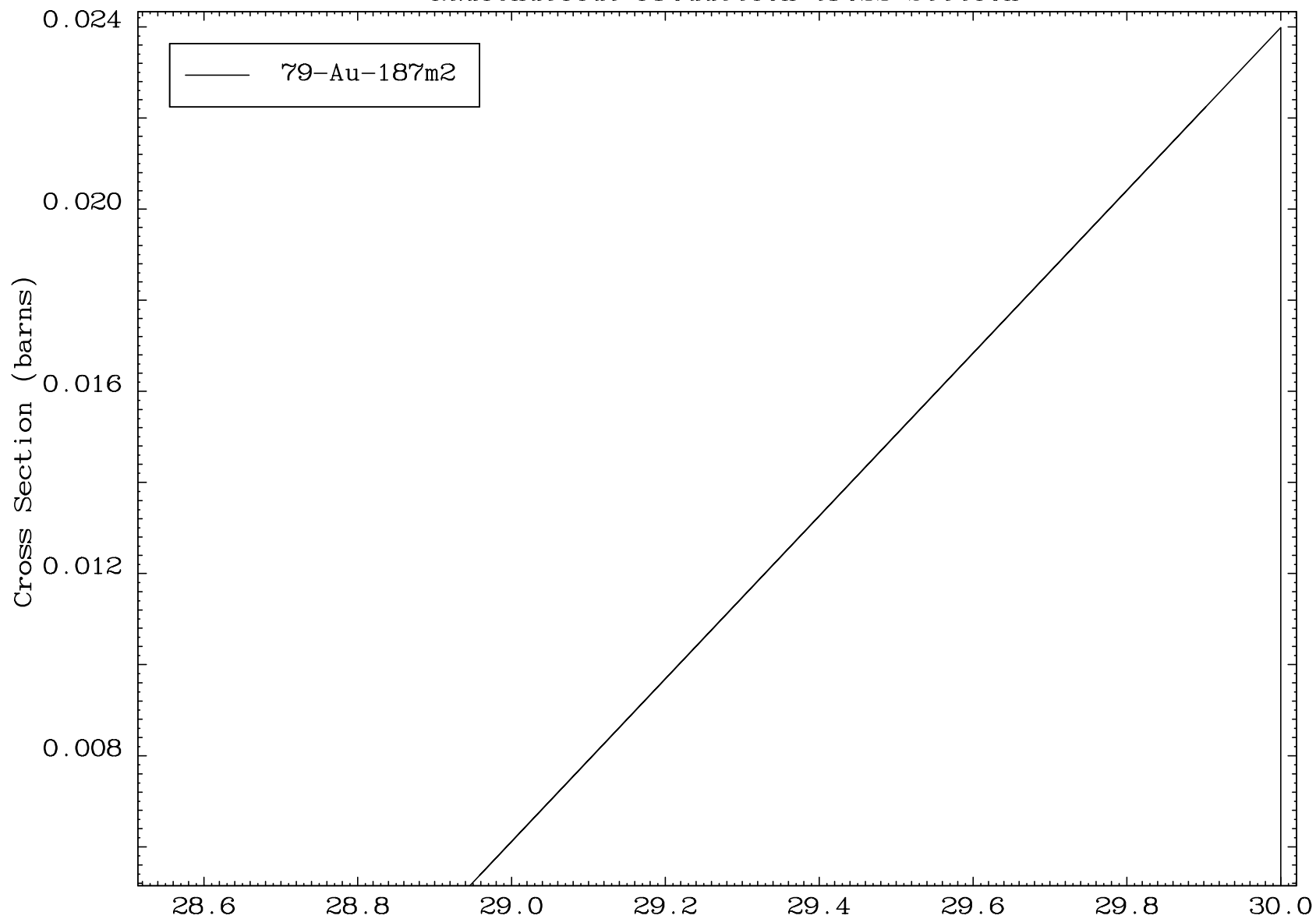
77-Ir-186

MAT 7710

($\alpha, 3n$)

77-Ir-186

Radionuclide Production Cross Section



24

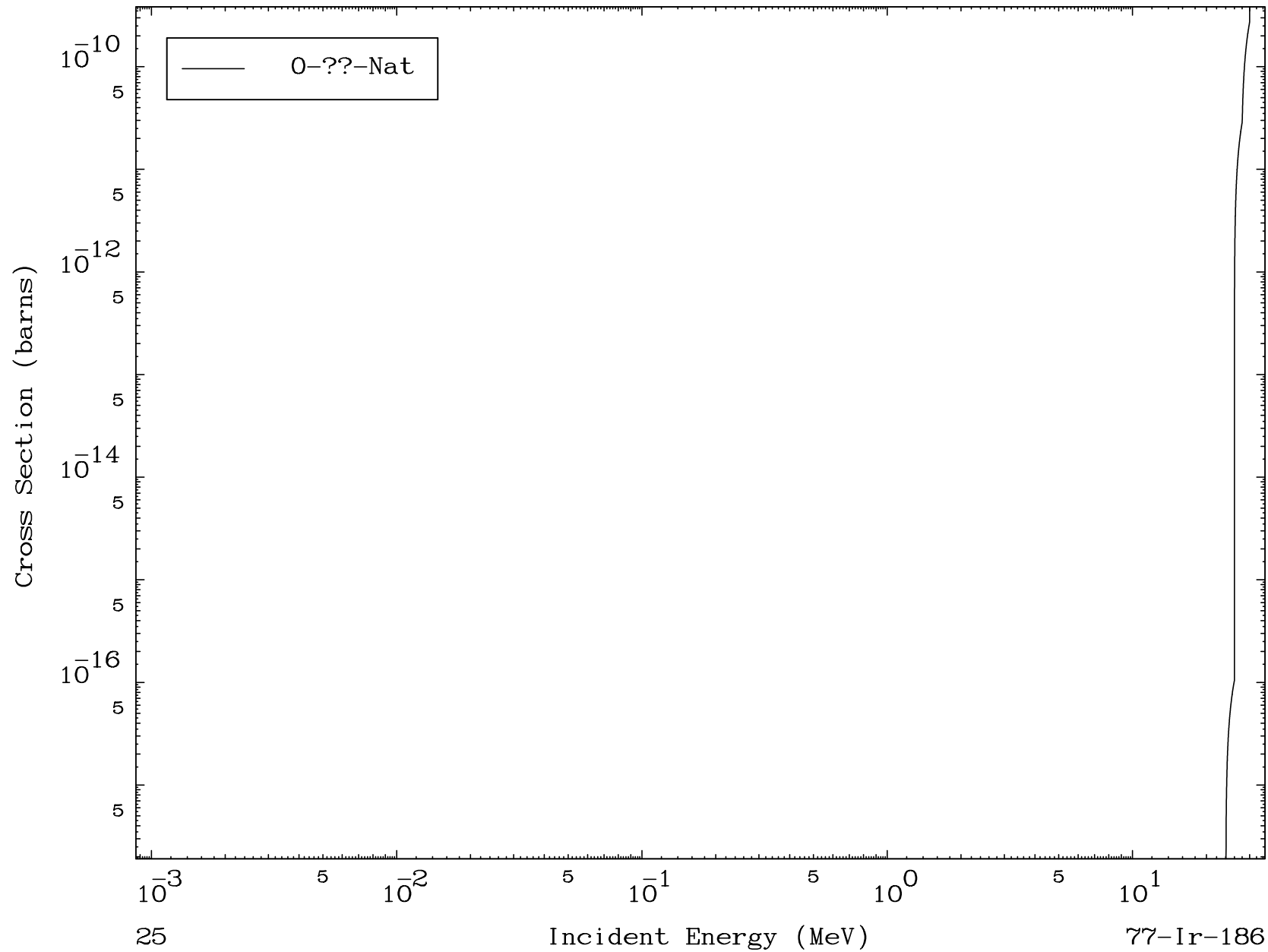
Incident Energy (MeV)

77-Ir-186

MAT 7710

α Fission
Radionuclide Production Cross Section

77-Ir-186

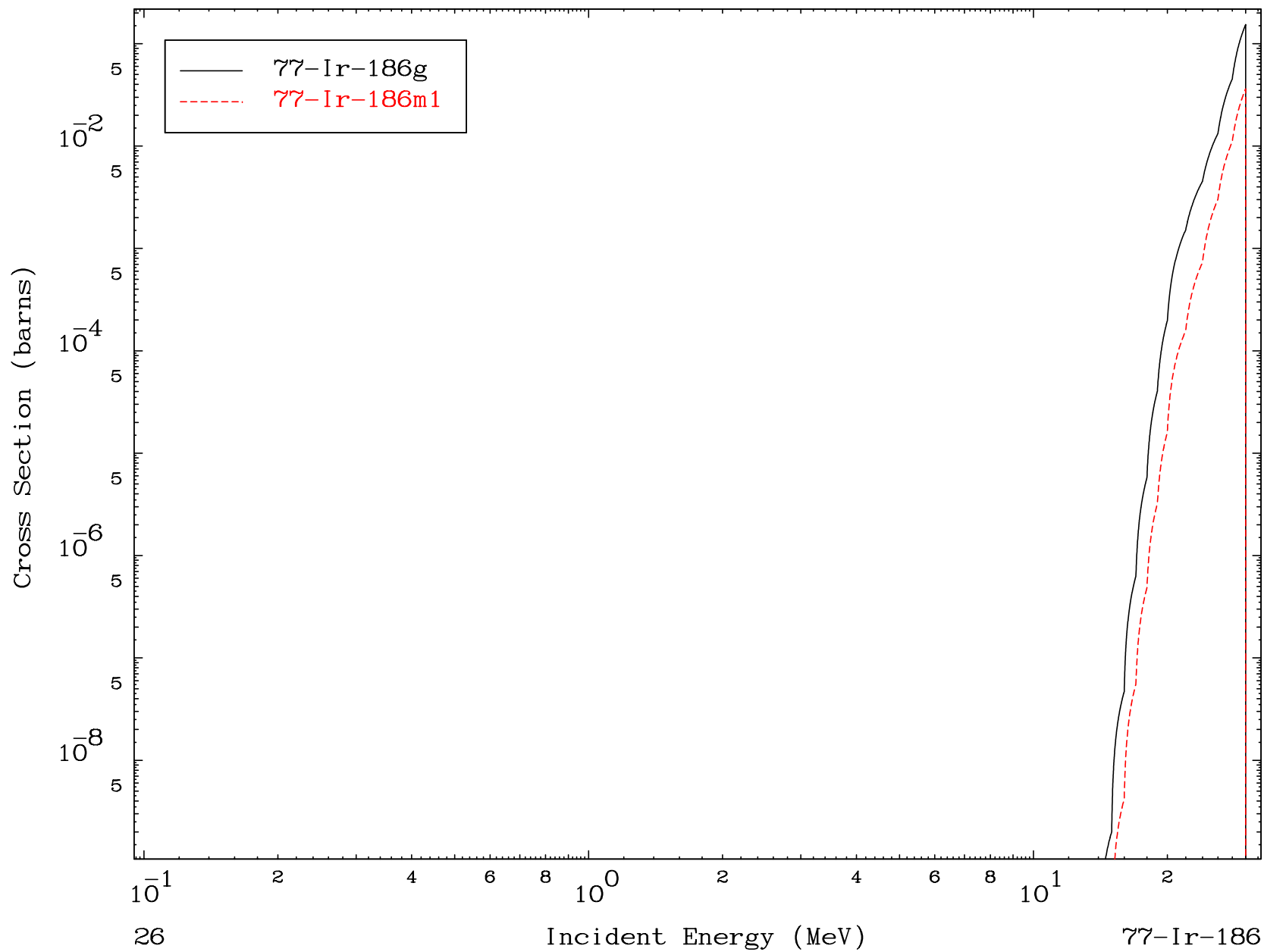


MAT 7710

 (α, α)

77-Ir-186

Radionuclide Production Cross Section



MAT 7710

 $(\alpha, 2\alpha)$

77-Ir-186

Radionuclide Production Cross Section

