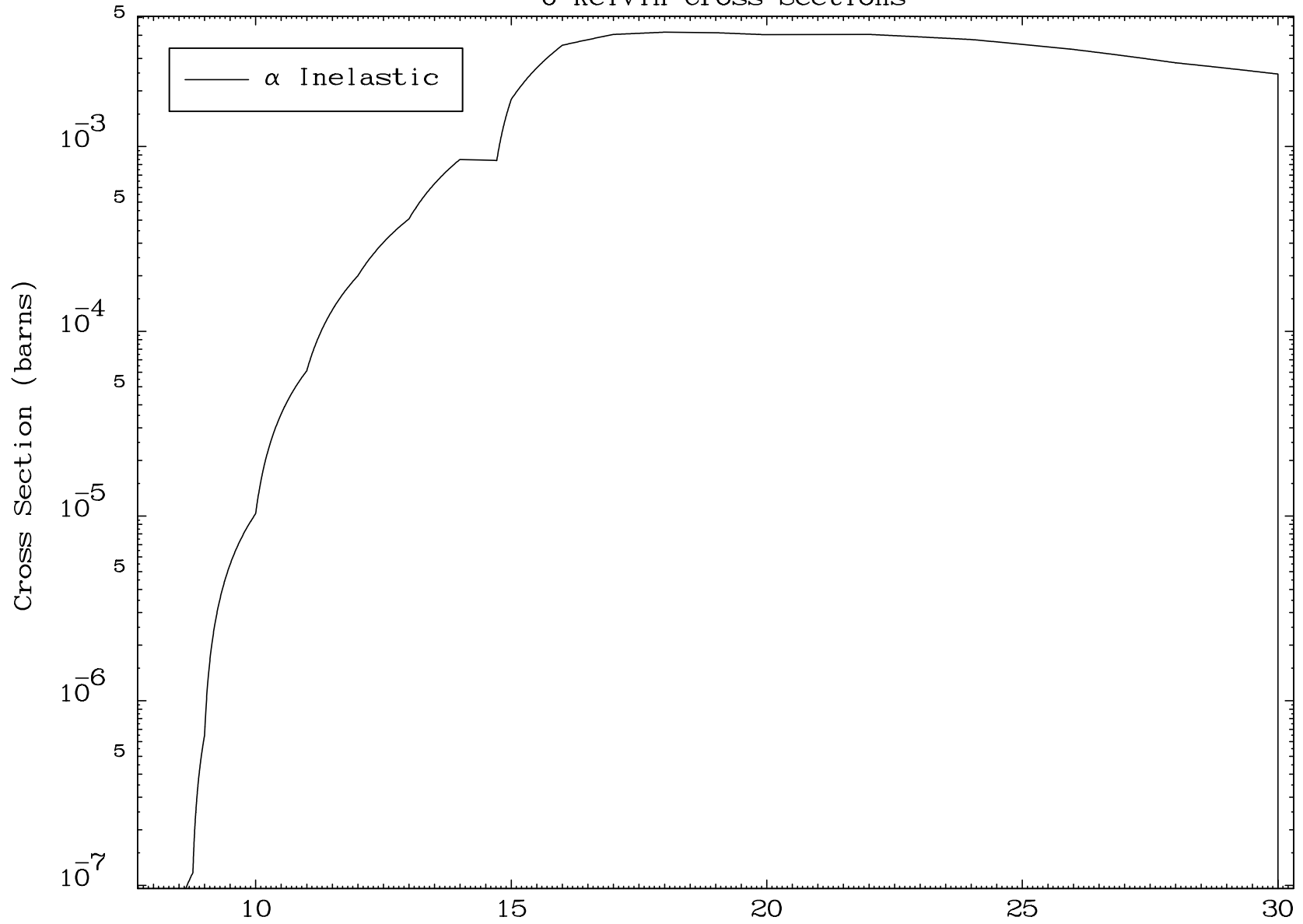


MAT 4210

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

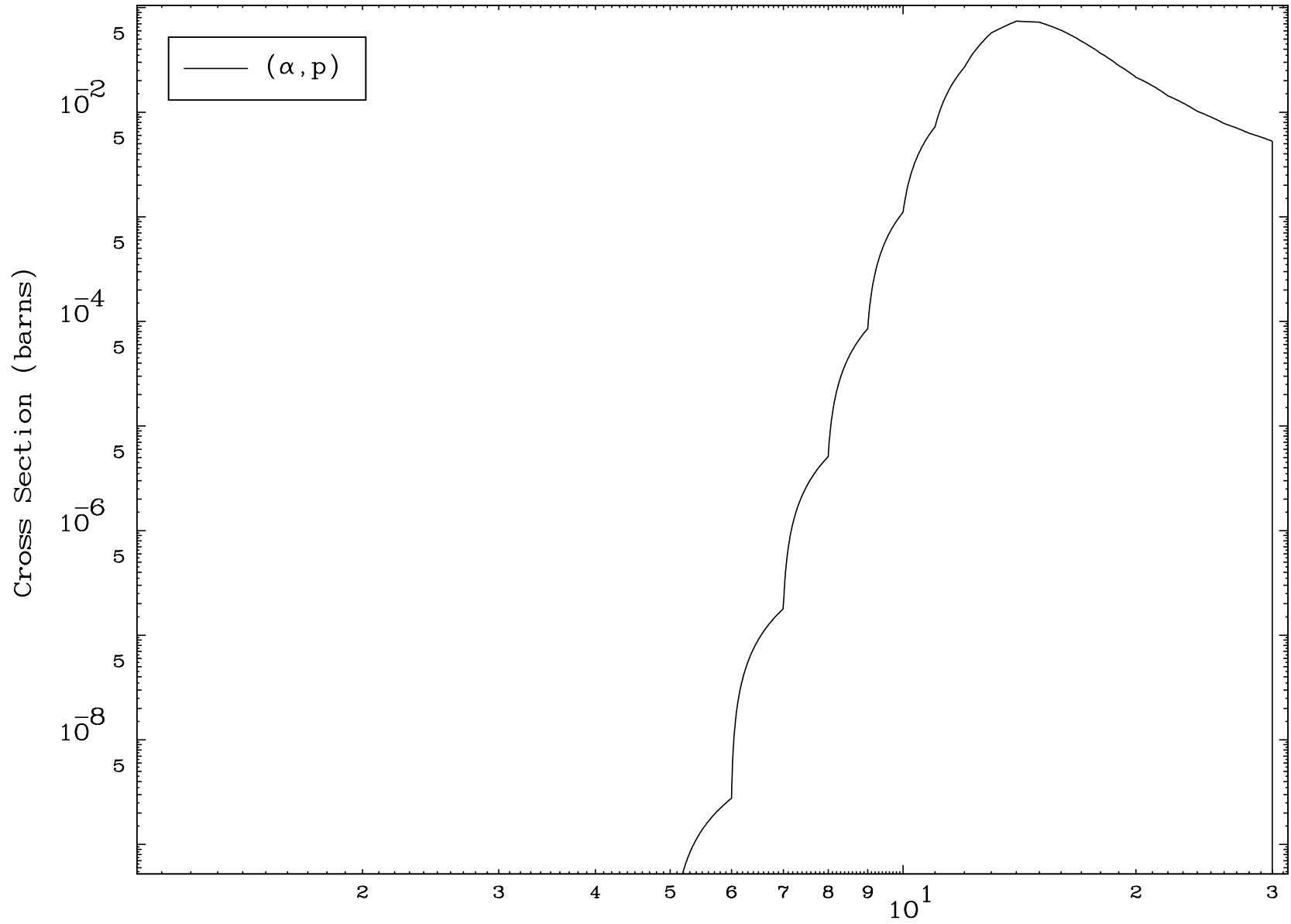
42-Mo-87

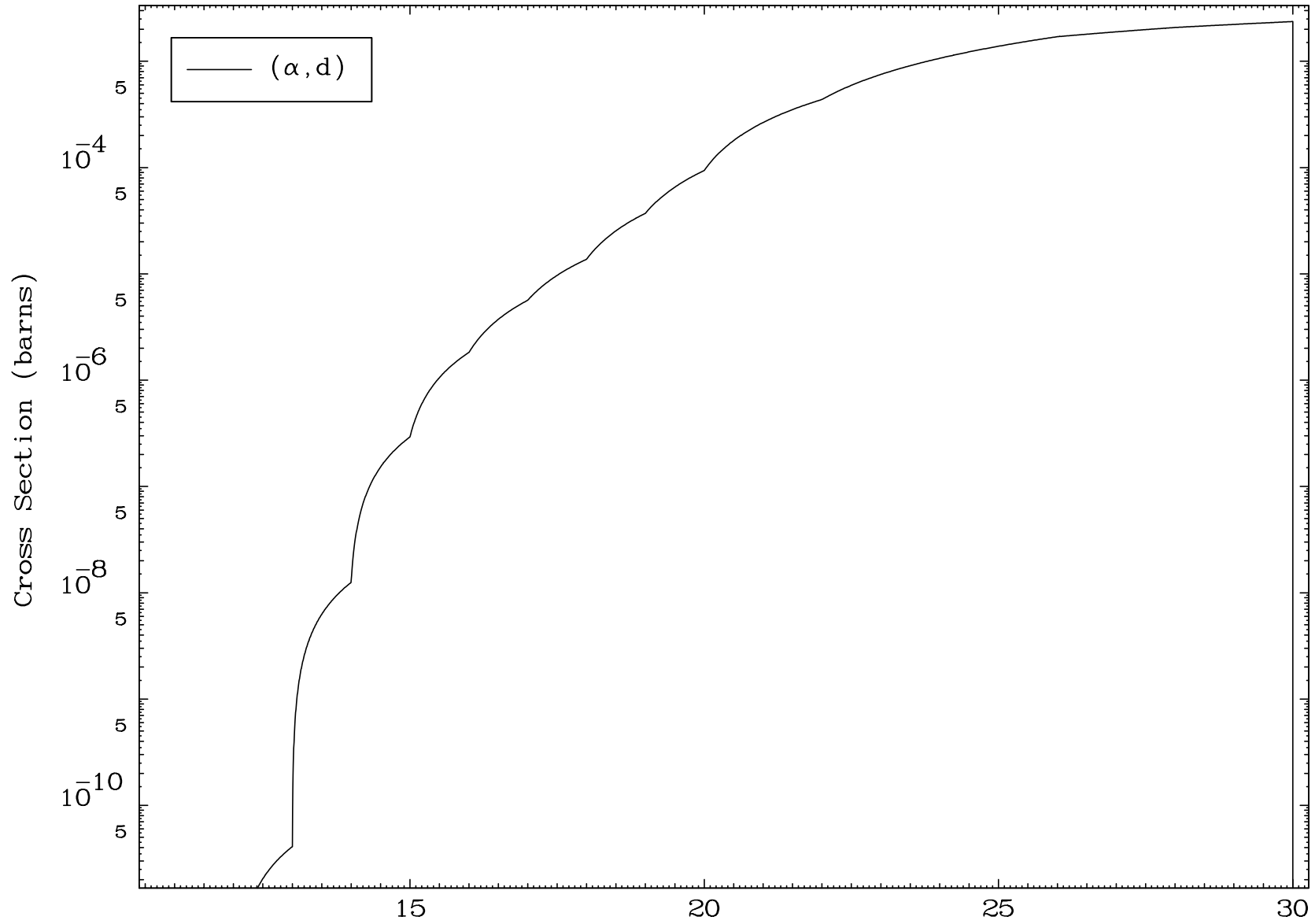


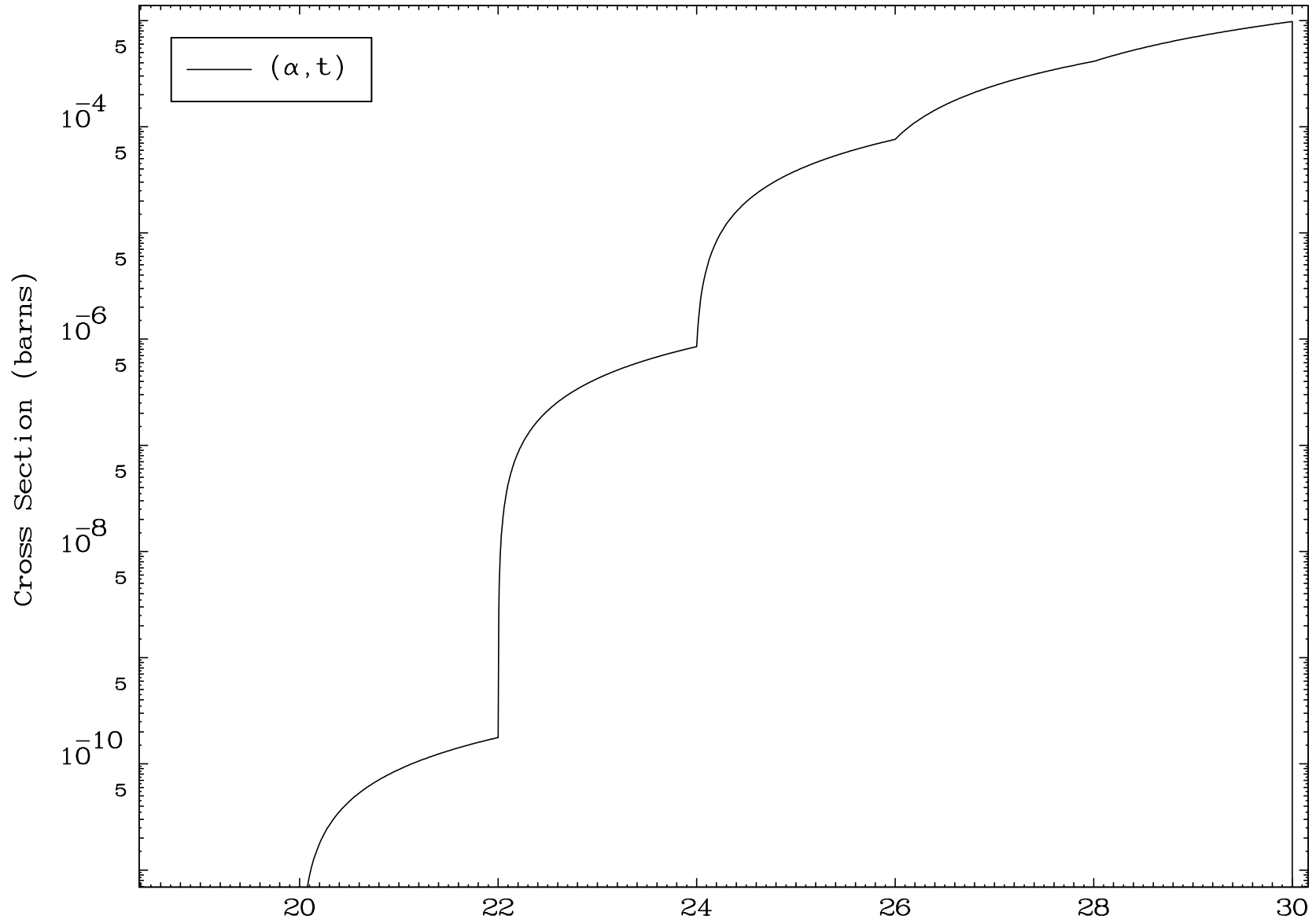
5

Incident Energy (MeV)

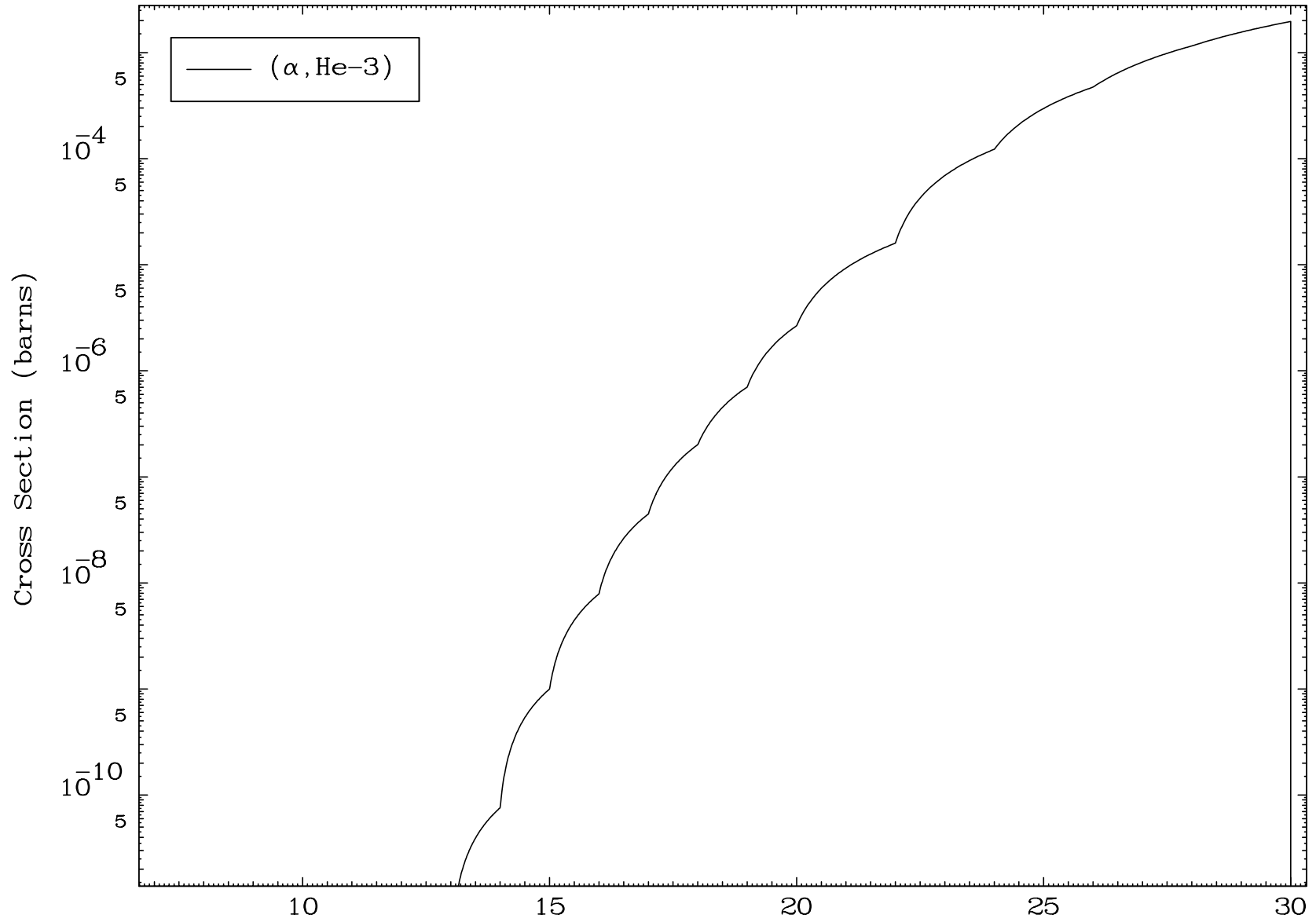
42-Mo-87







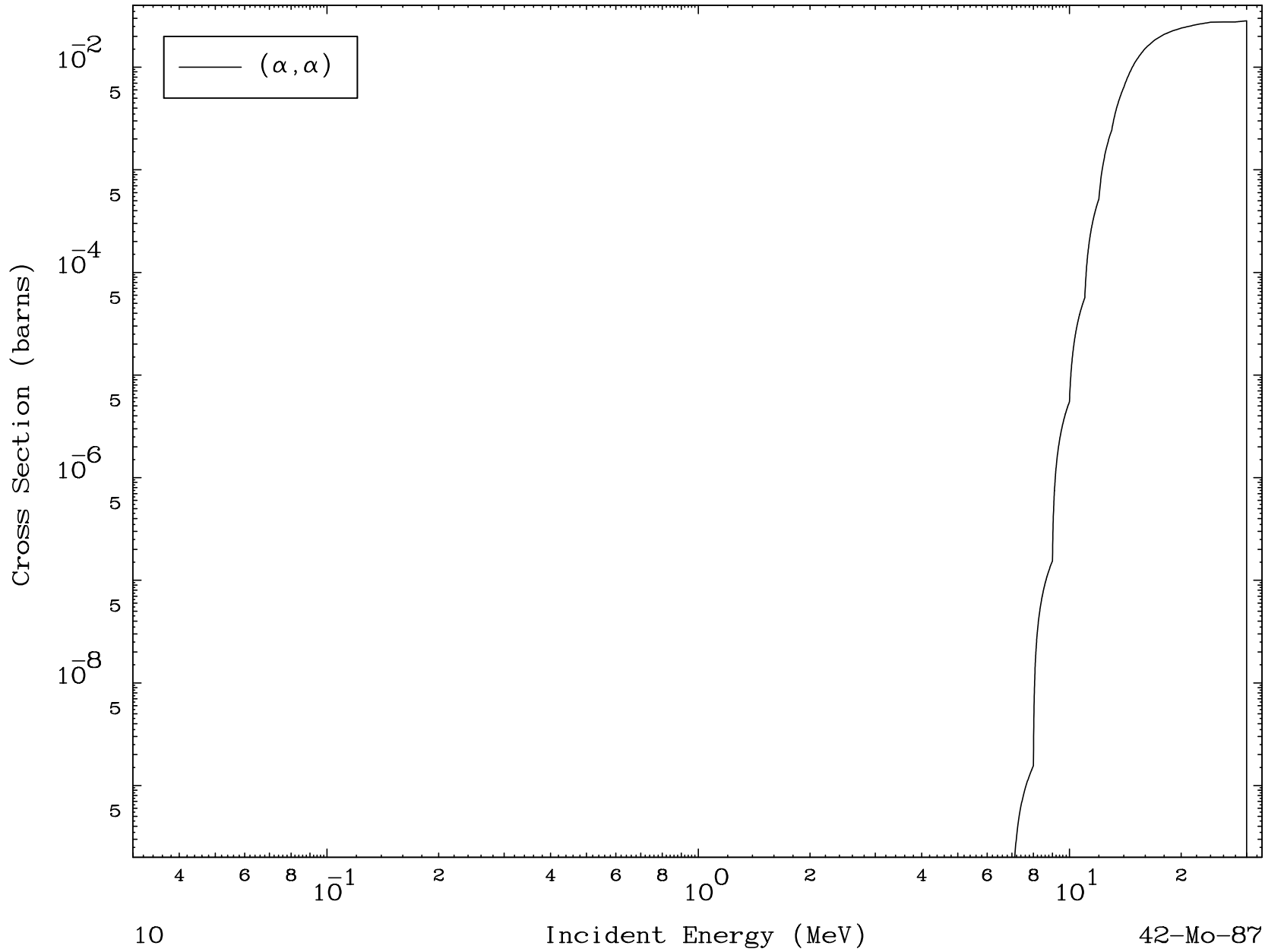




MAT 4210

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

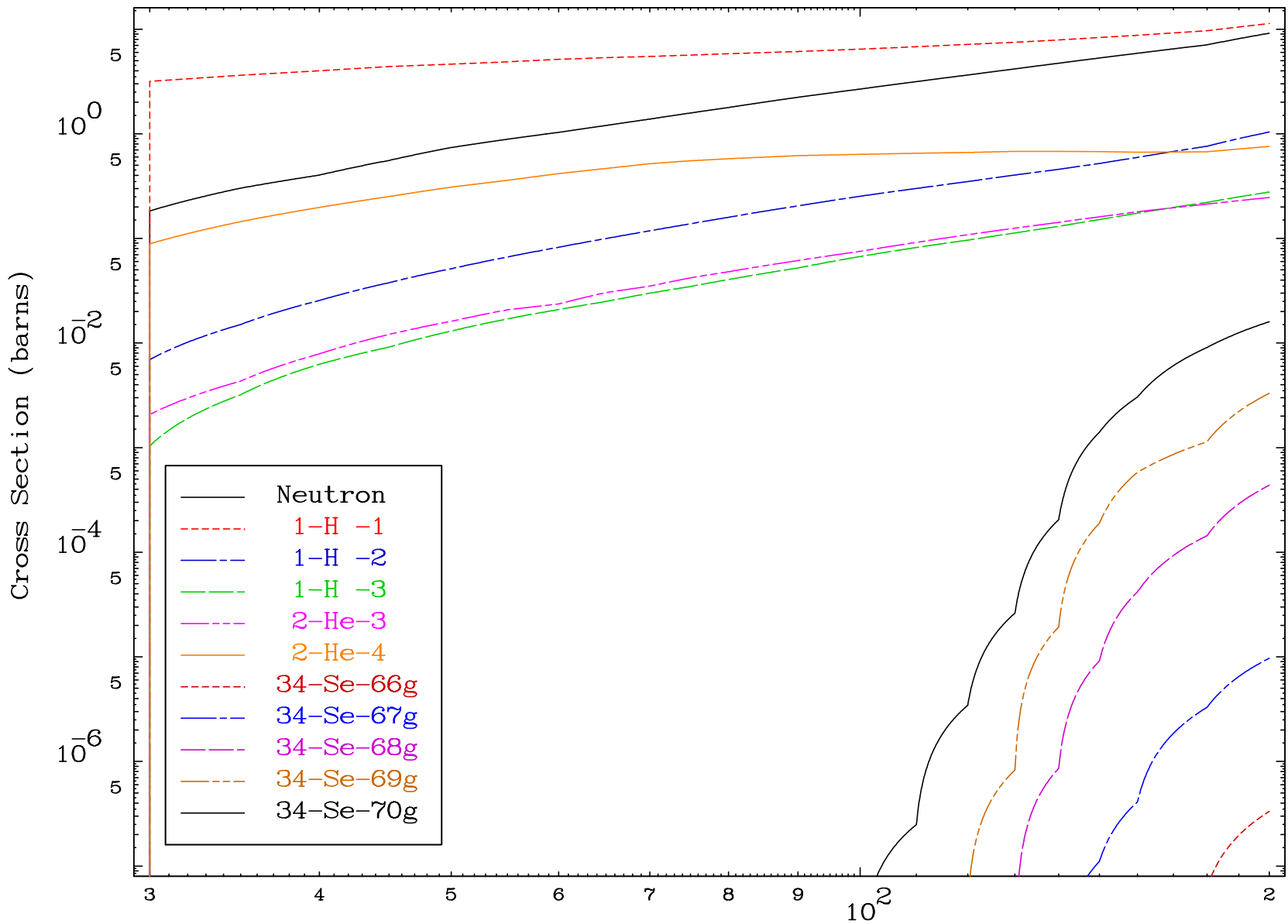
42-Mo-87



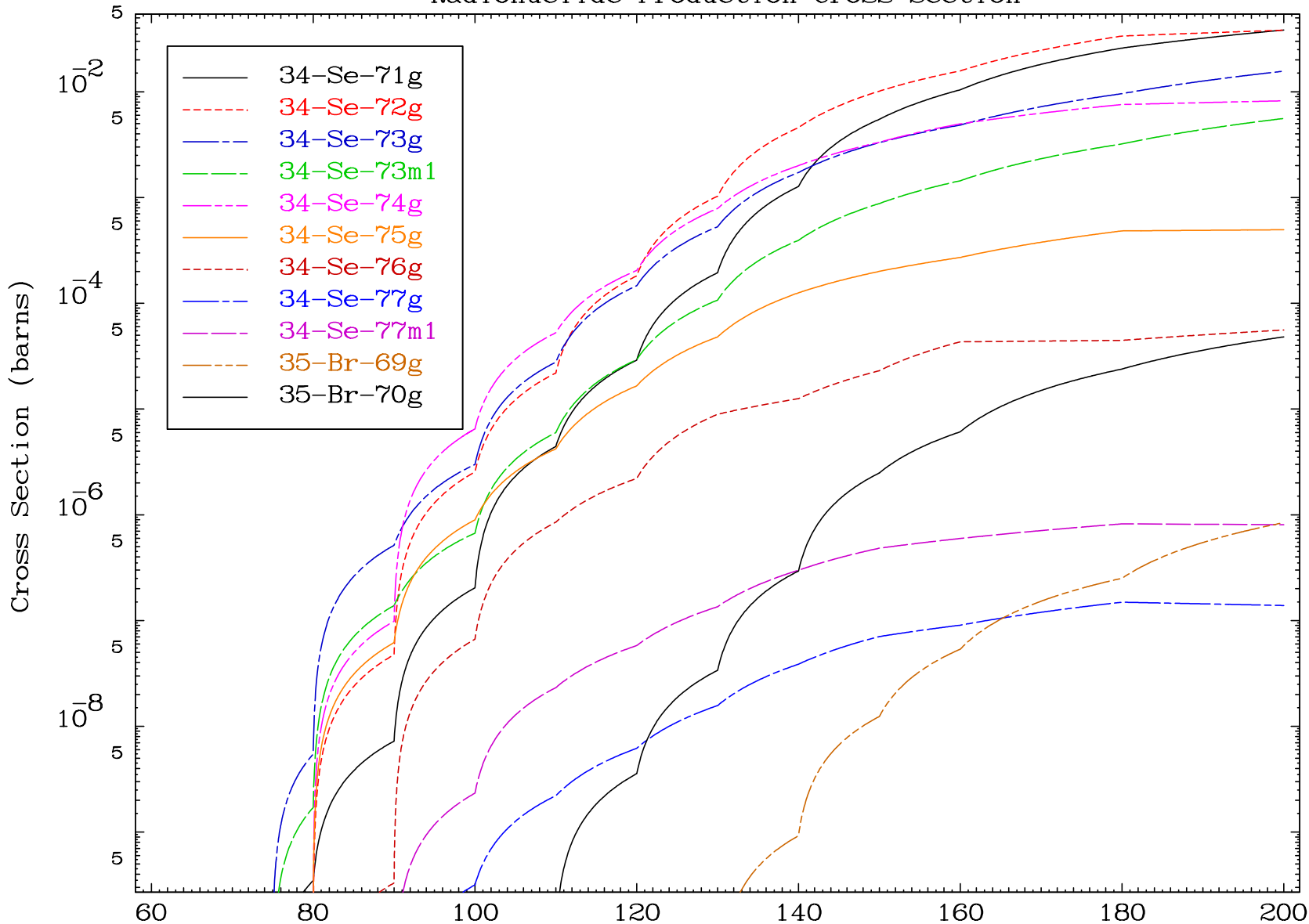
10

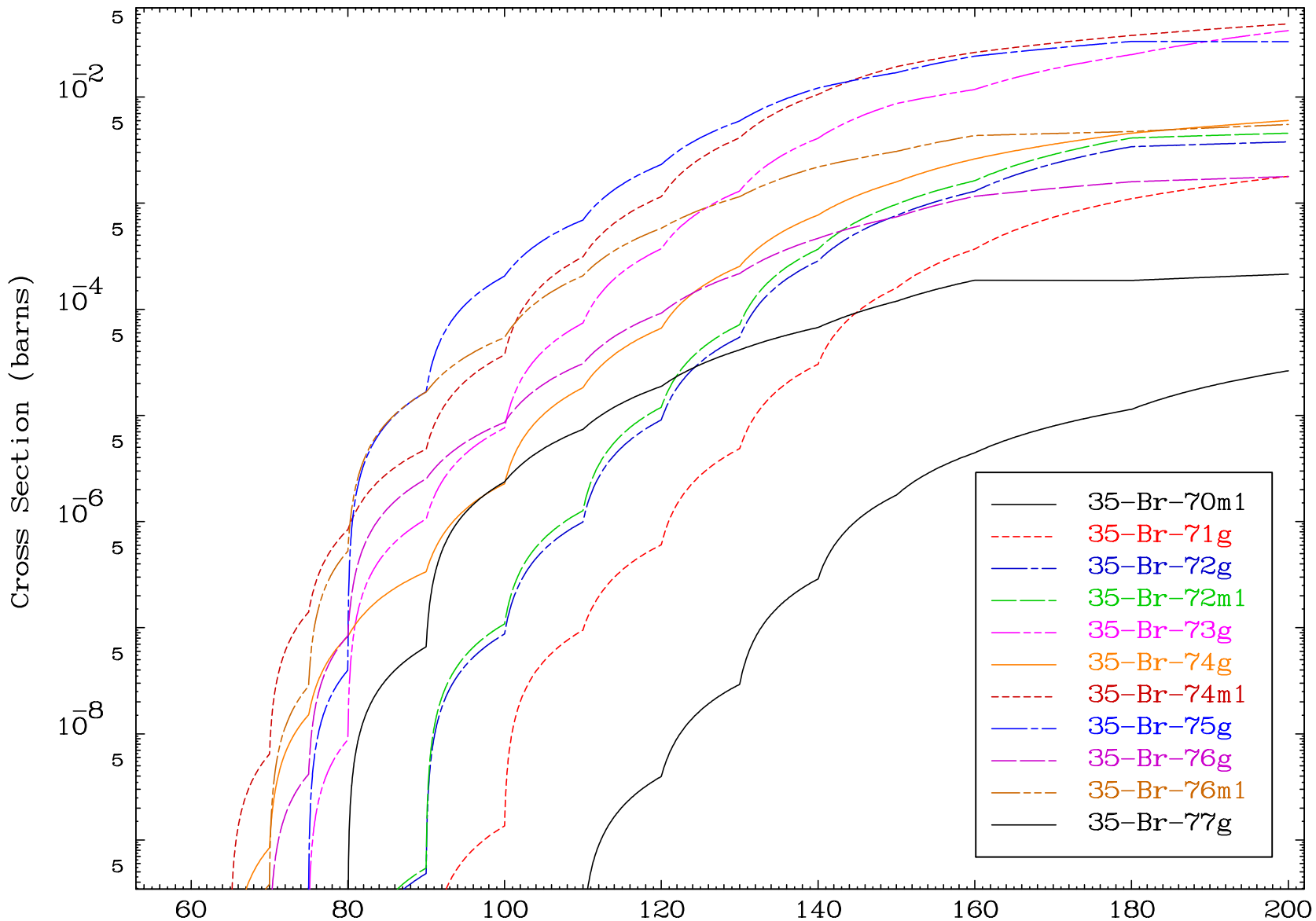
Incident Energy (MeV)

42-Mo-87

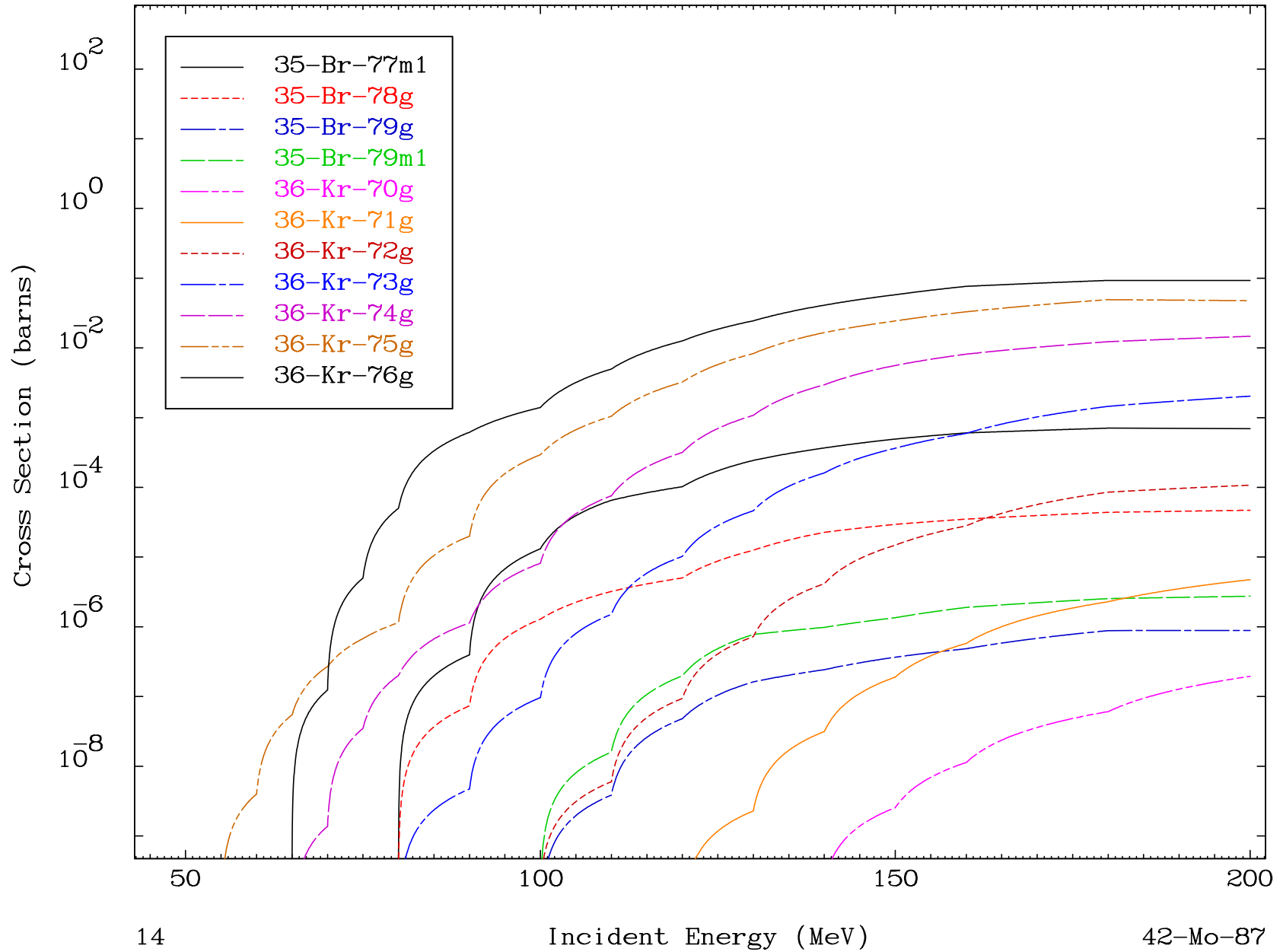


Radionuclide Production Cross Section

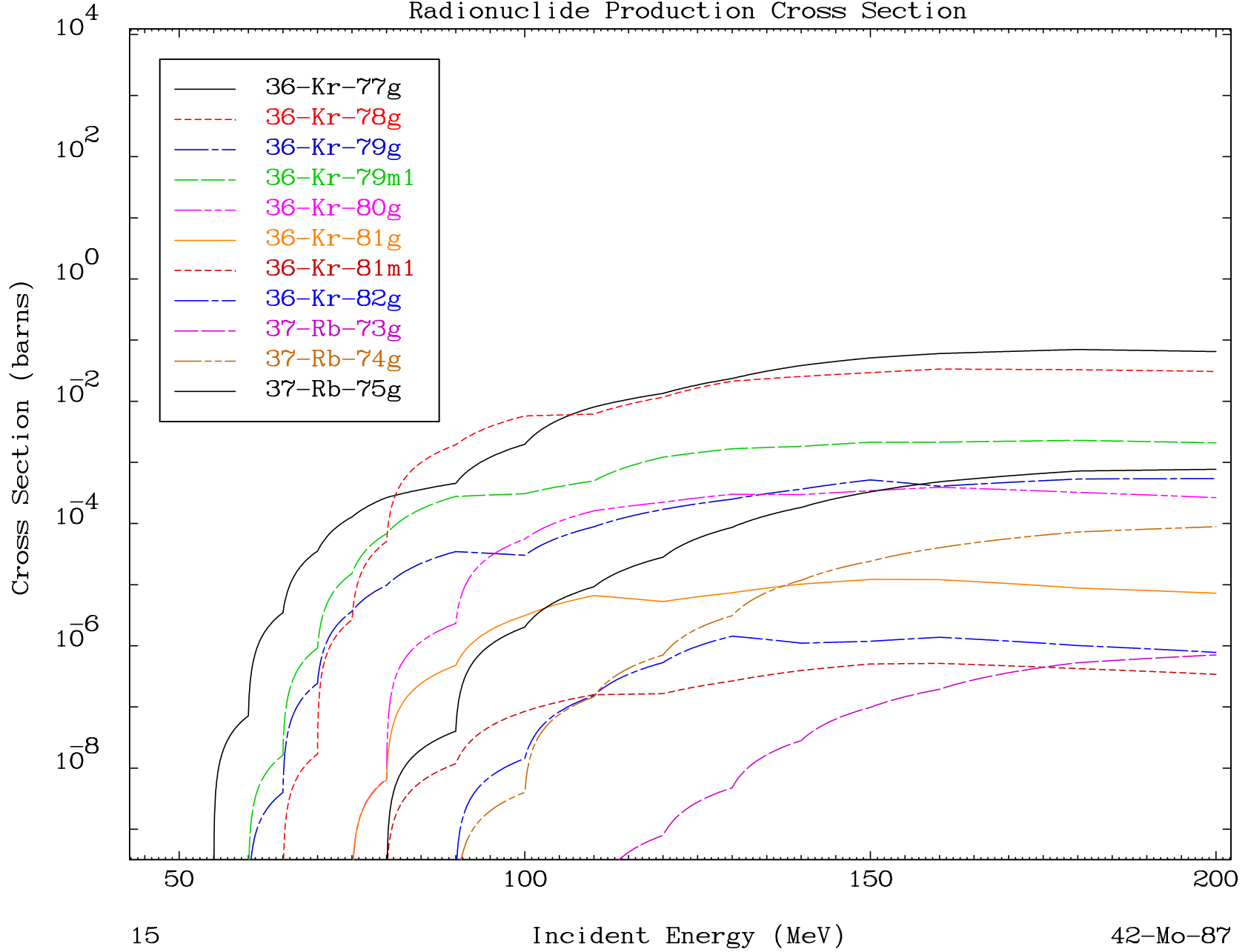




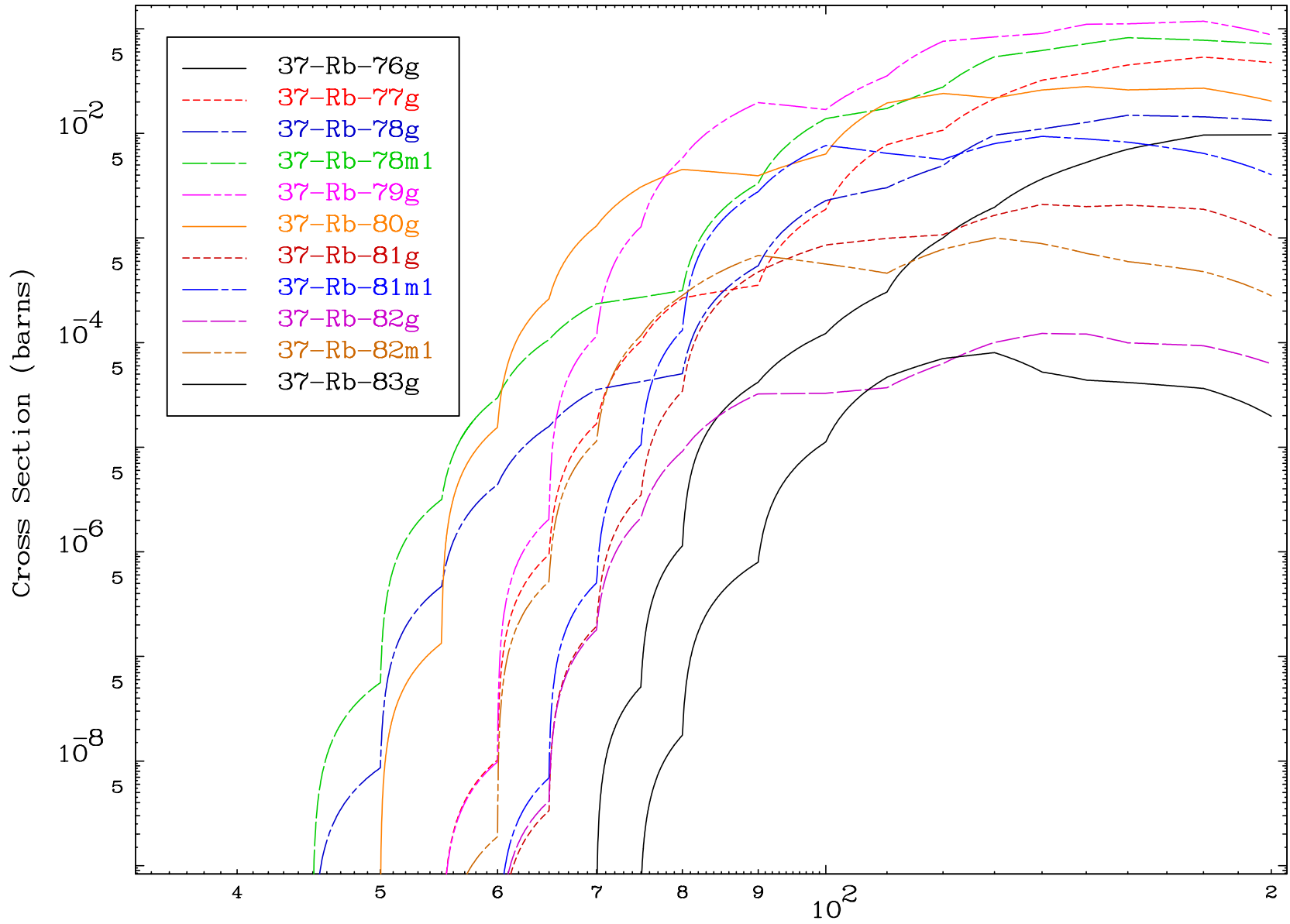
Radionuclide Production Cross Section



Radionuclide Production Cross Section

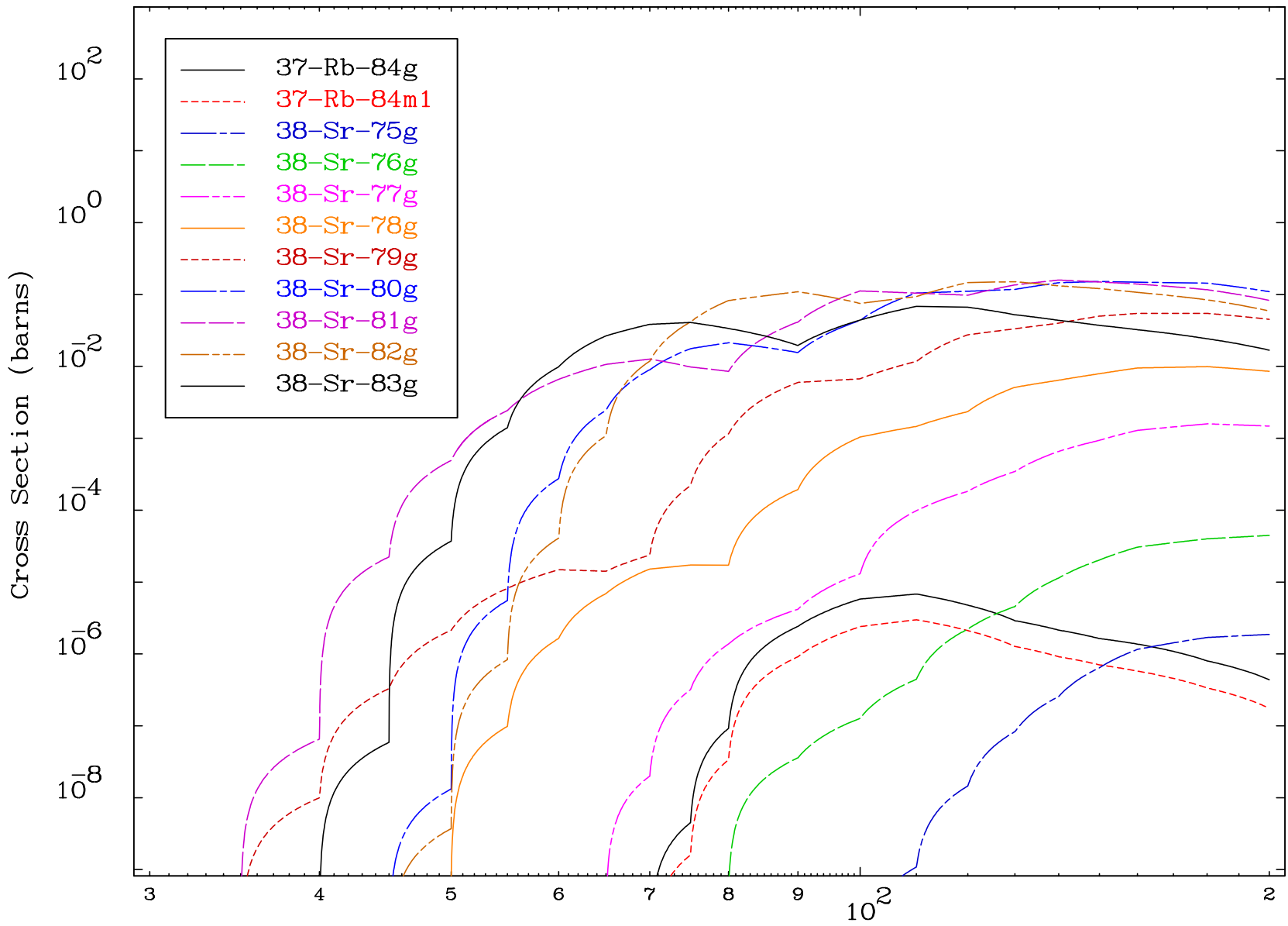


Radionuclide Production Cross Section

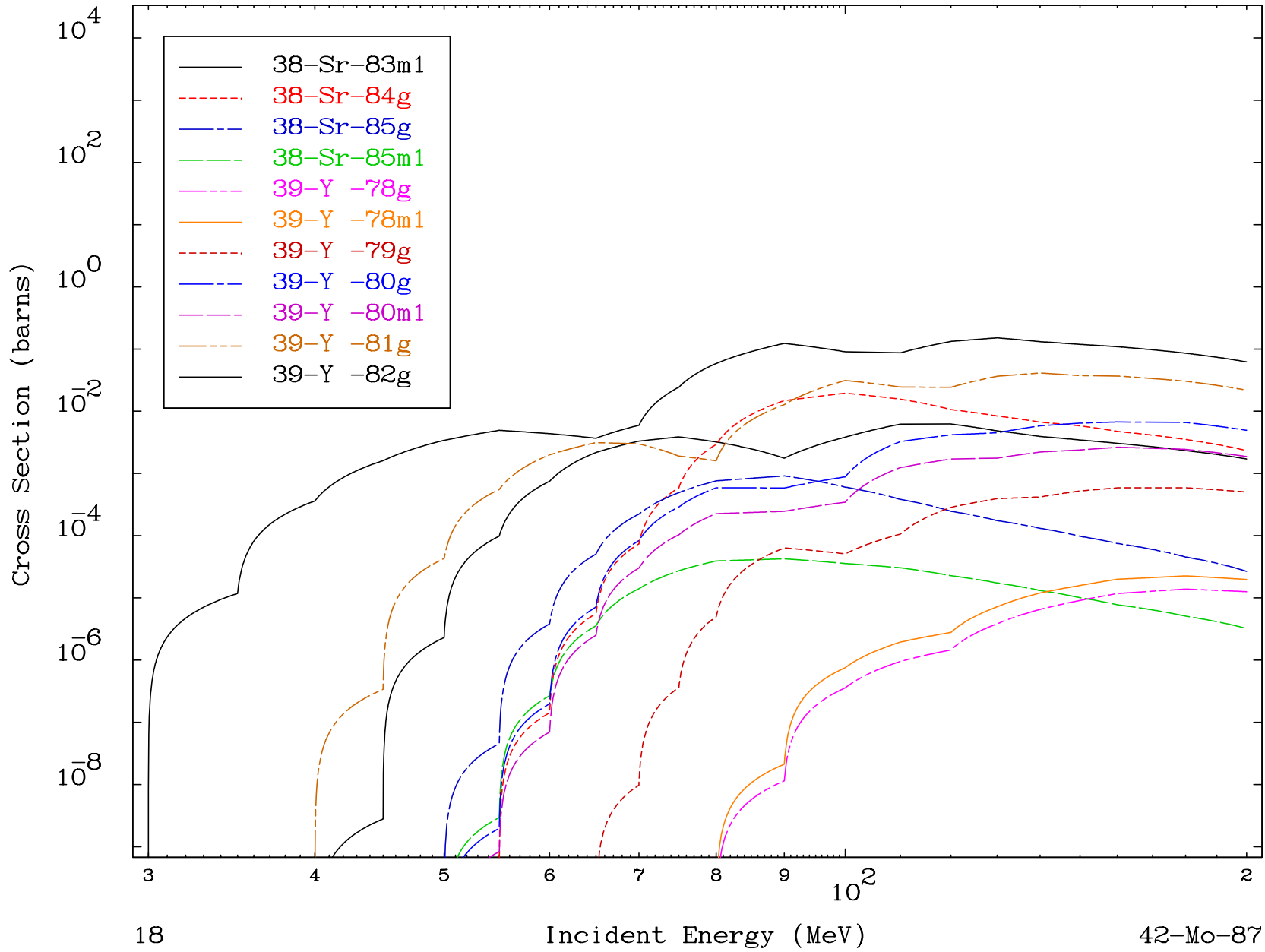




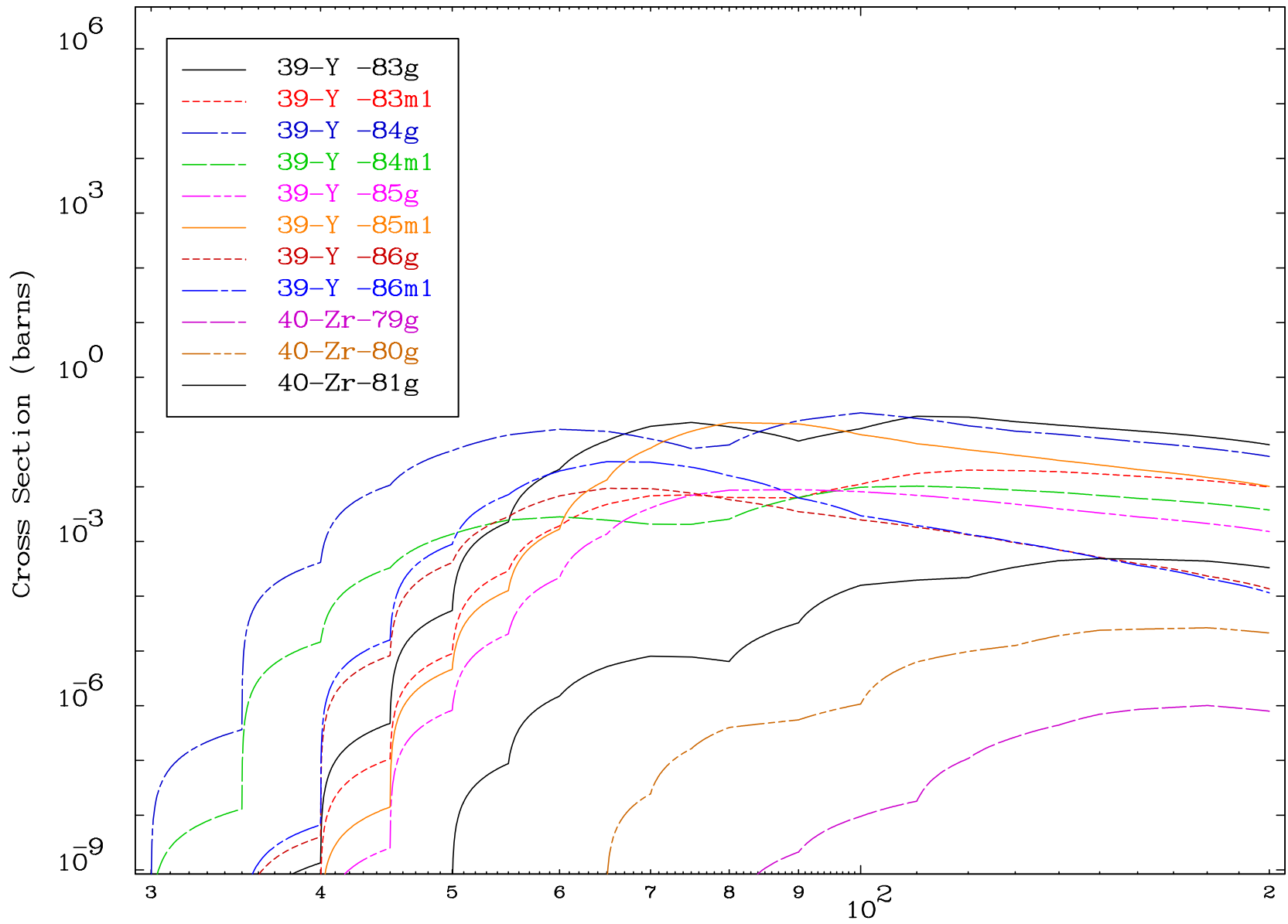
Radionuclide Production Cross Section

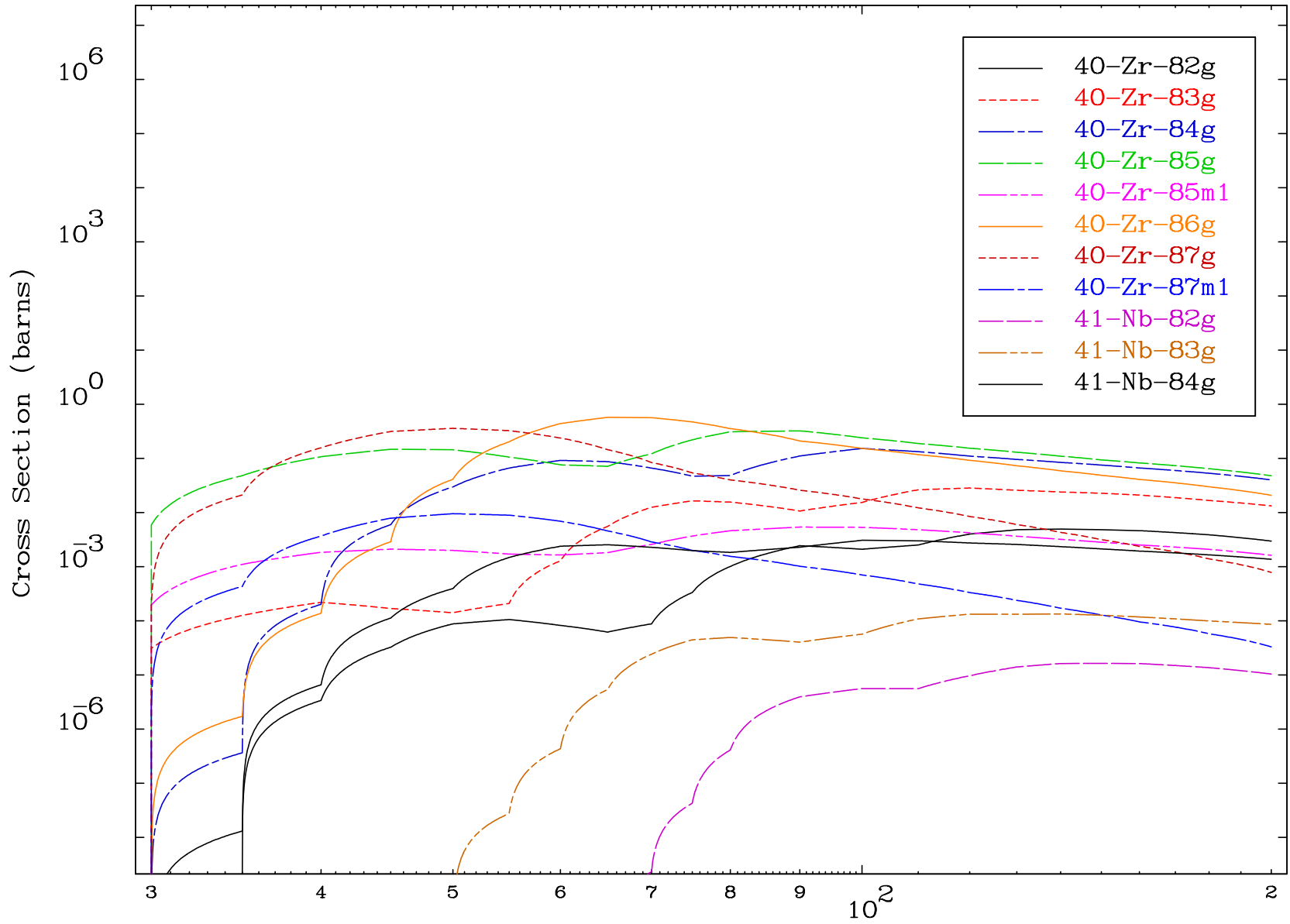


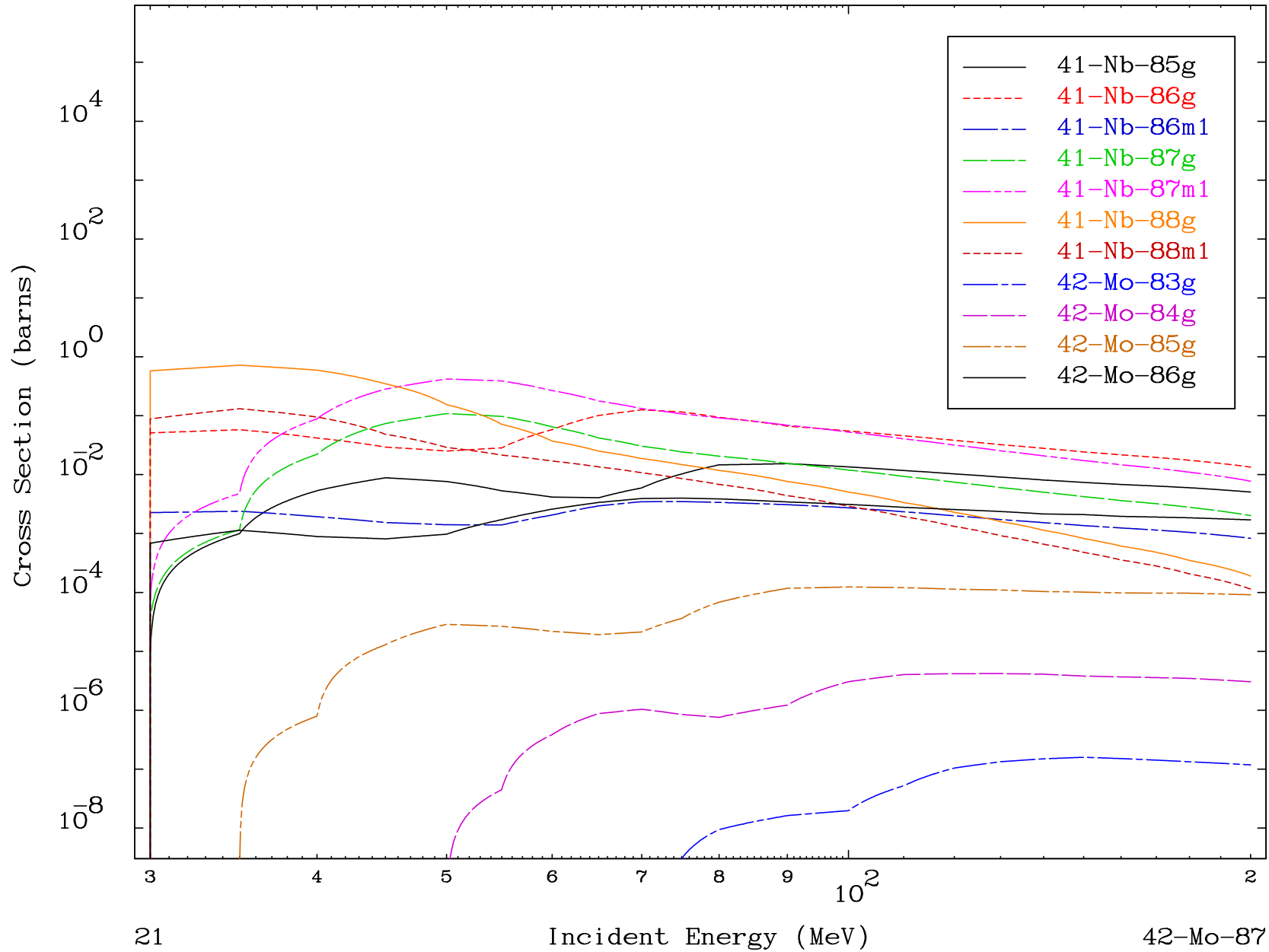
Radionuclide Production Cross Section

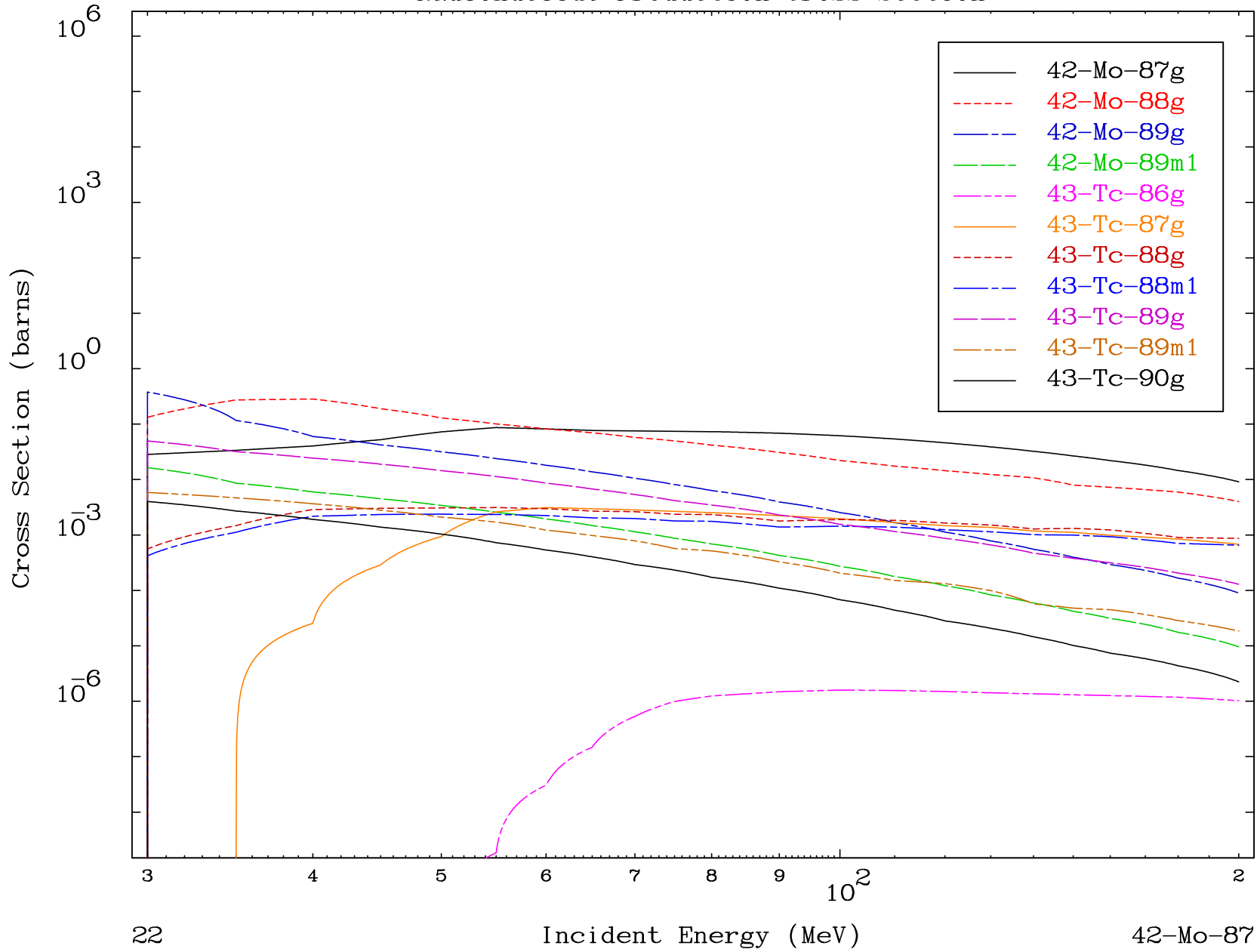


Radionuclide Production Cross Section

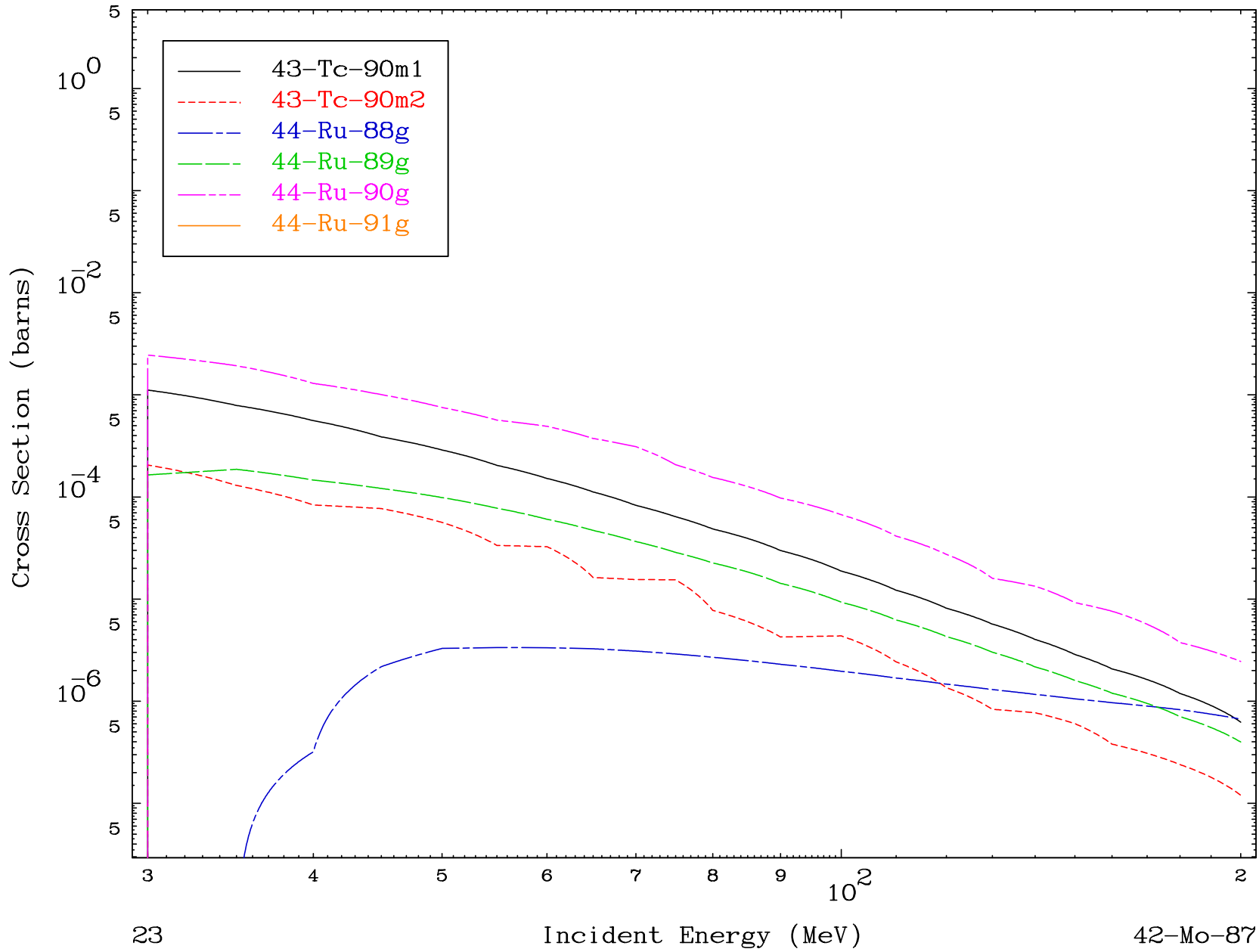








Radionuclide Production Cross Section

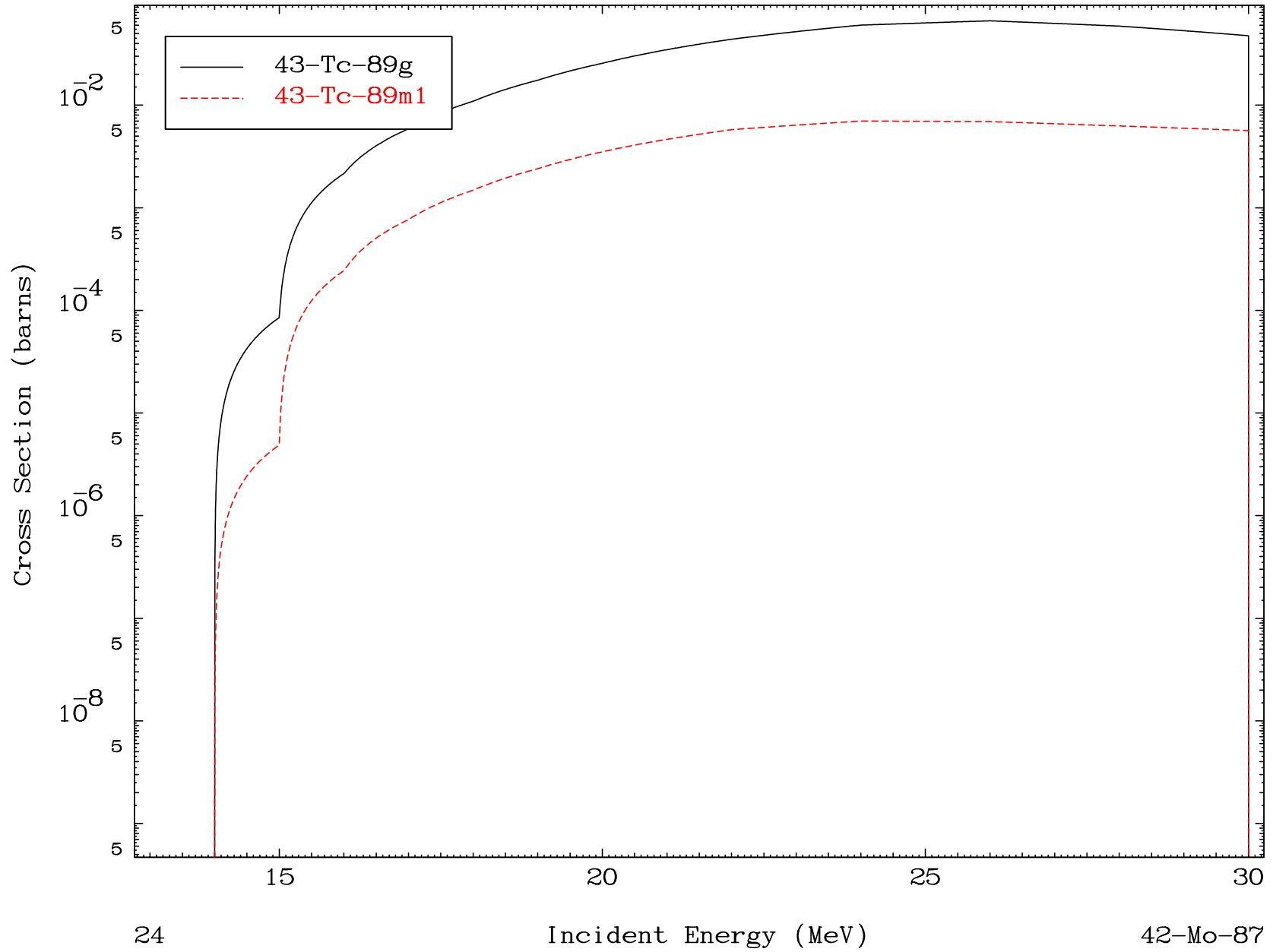


MAT 4210

( $\alpha, n'$ ) p

42-Mo-87

Radionuclide Production Cross Section



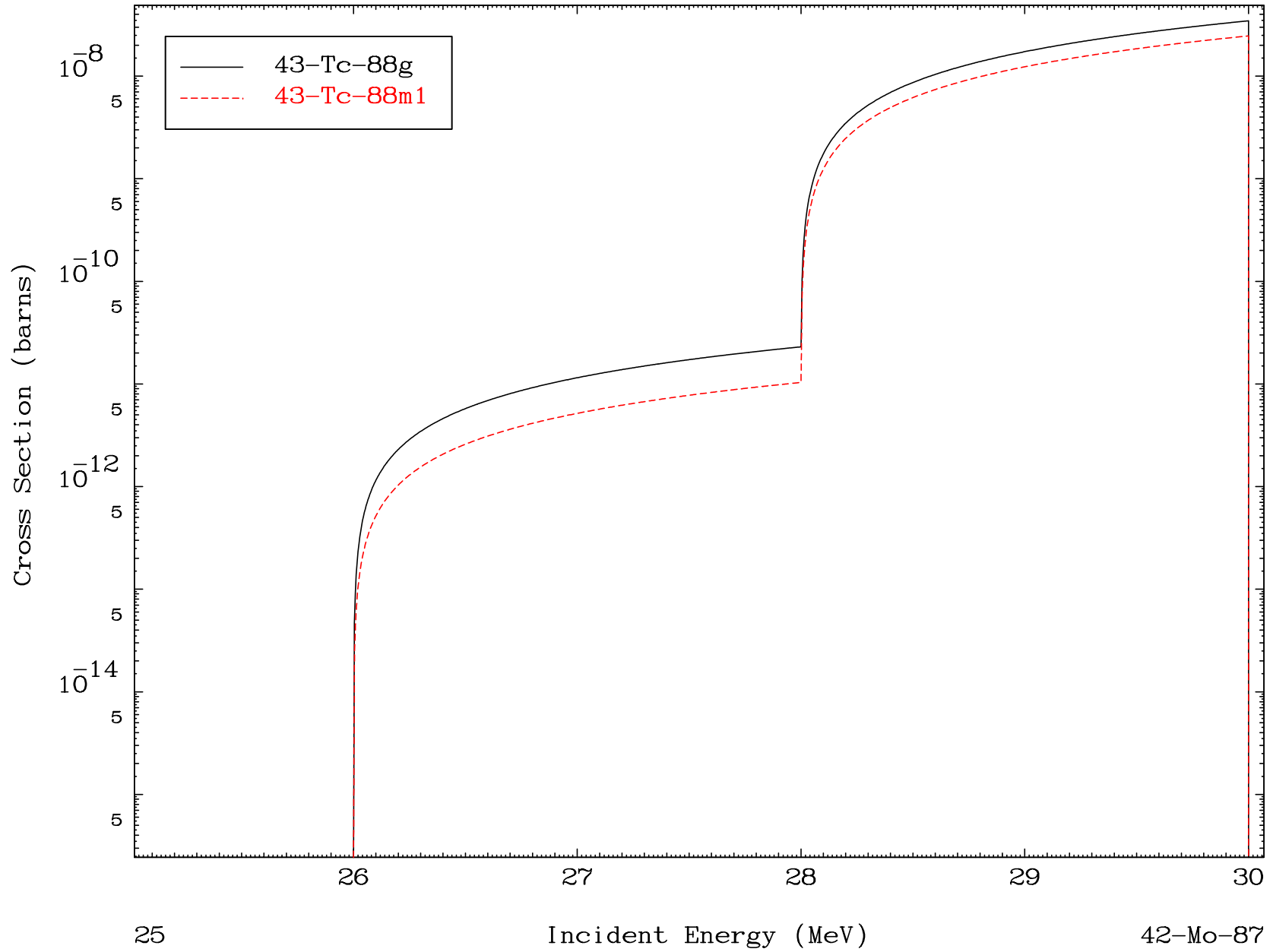


MAT 4210

( $\alpha, n'$ ) d

42-Mo-87

Radionuclide Production Cross Section

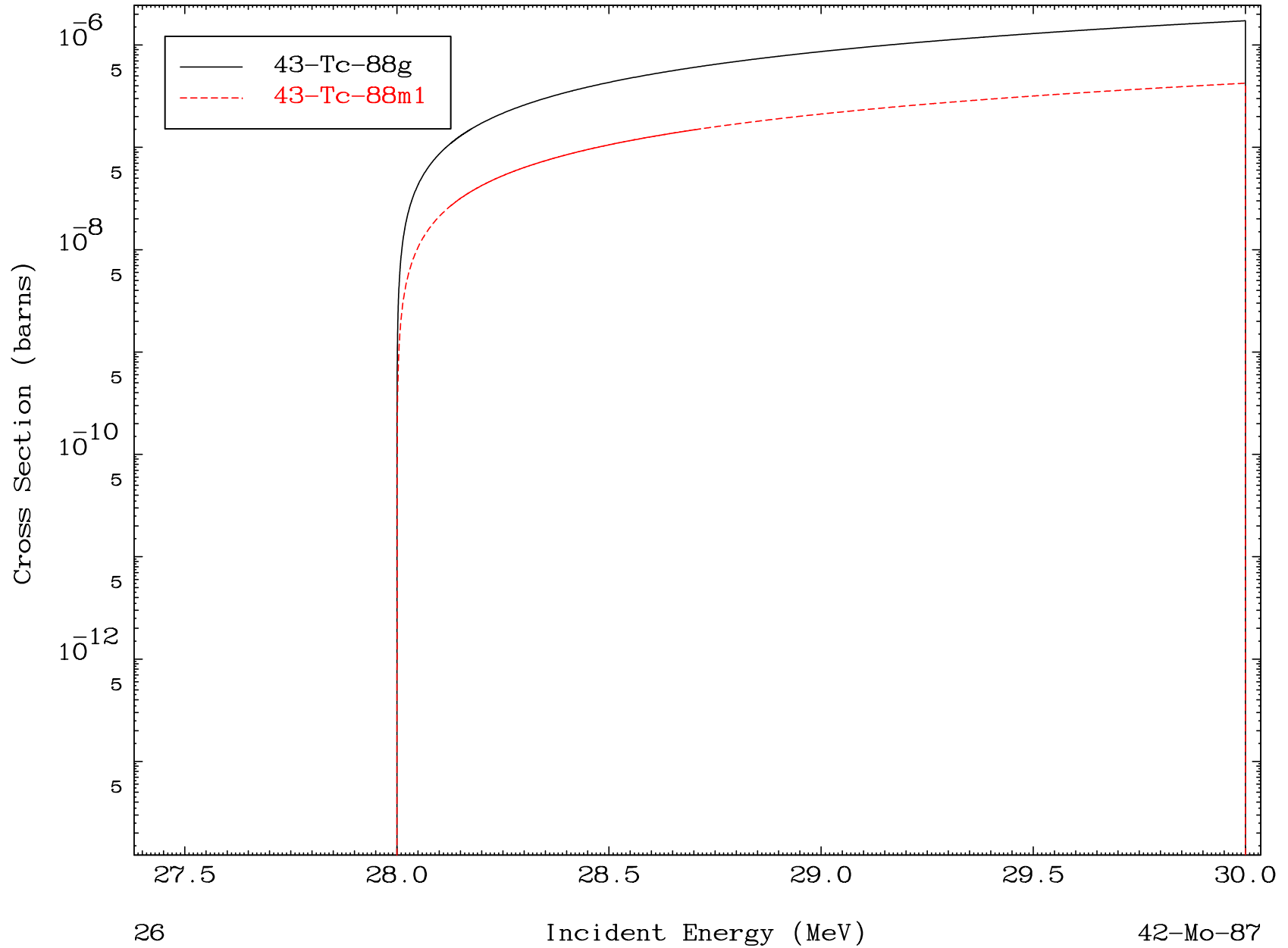


MAT 4210

( $\alpha, 2n$ ) p

42-Mo-87

Radionuclide Production Cross Section

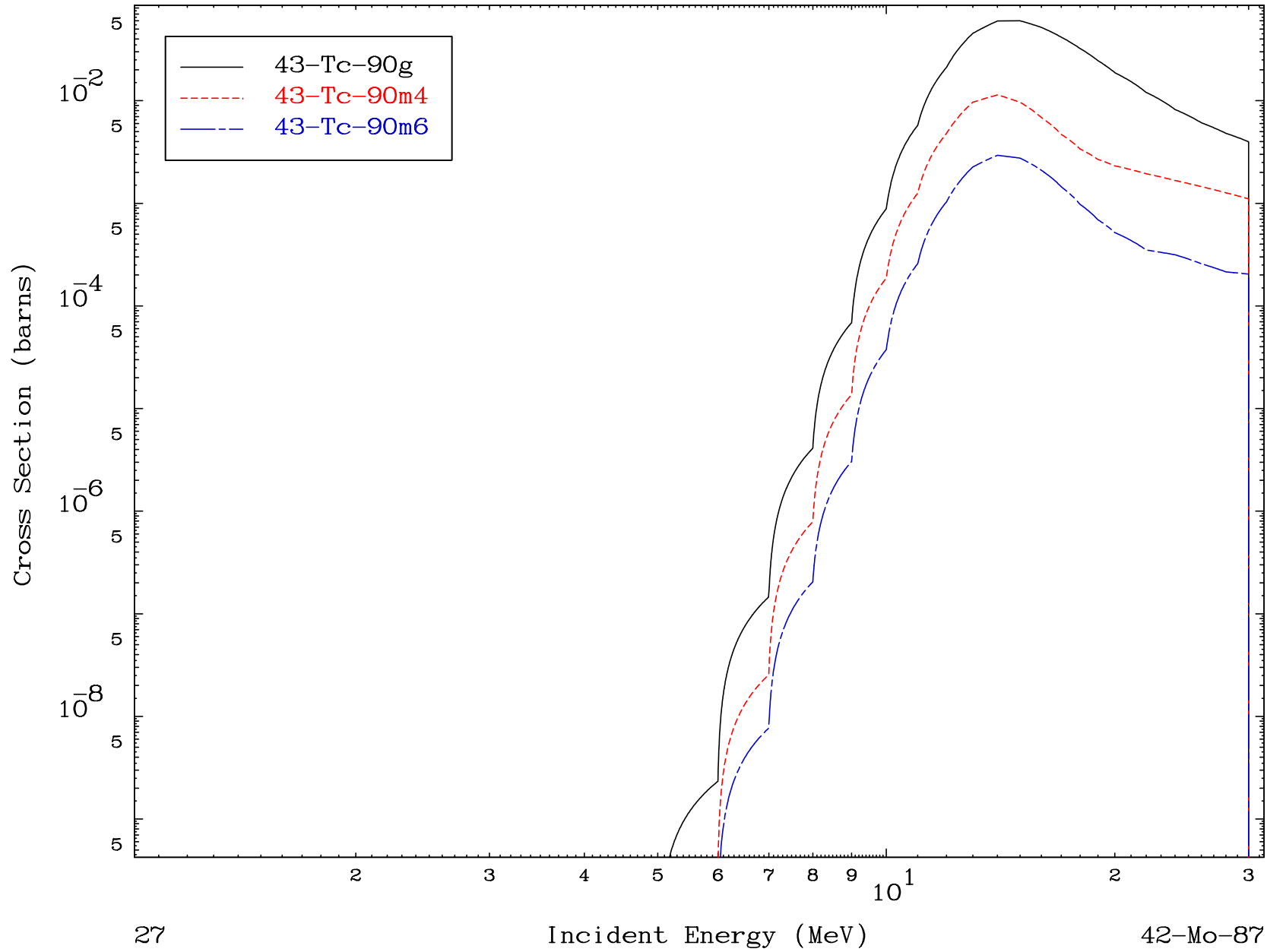


26

Incident Energy (MeV)

42-Mo-87

Radionuclide Production Cross Section

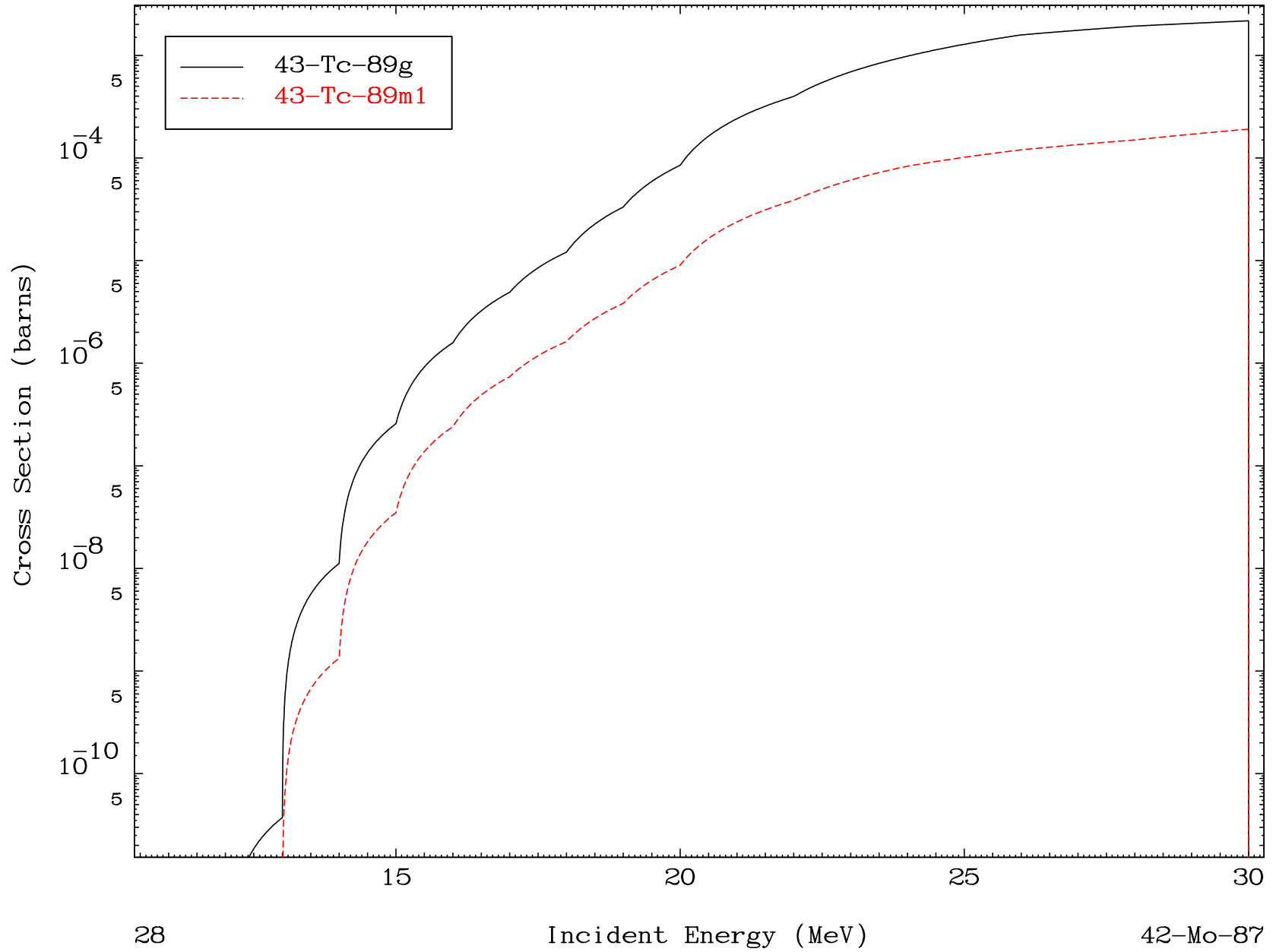


MAT 4210

( $\alpha, d$ )

42-Mo-87

### Radionuclide Production Cross Section



28

Incident Energy (MeV)

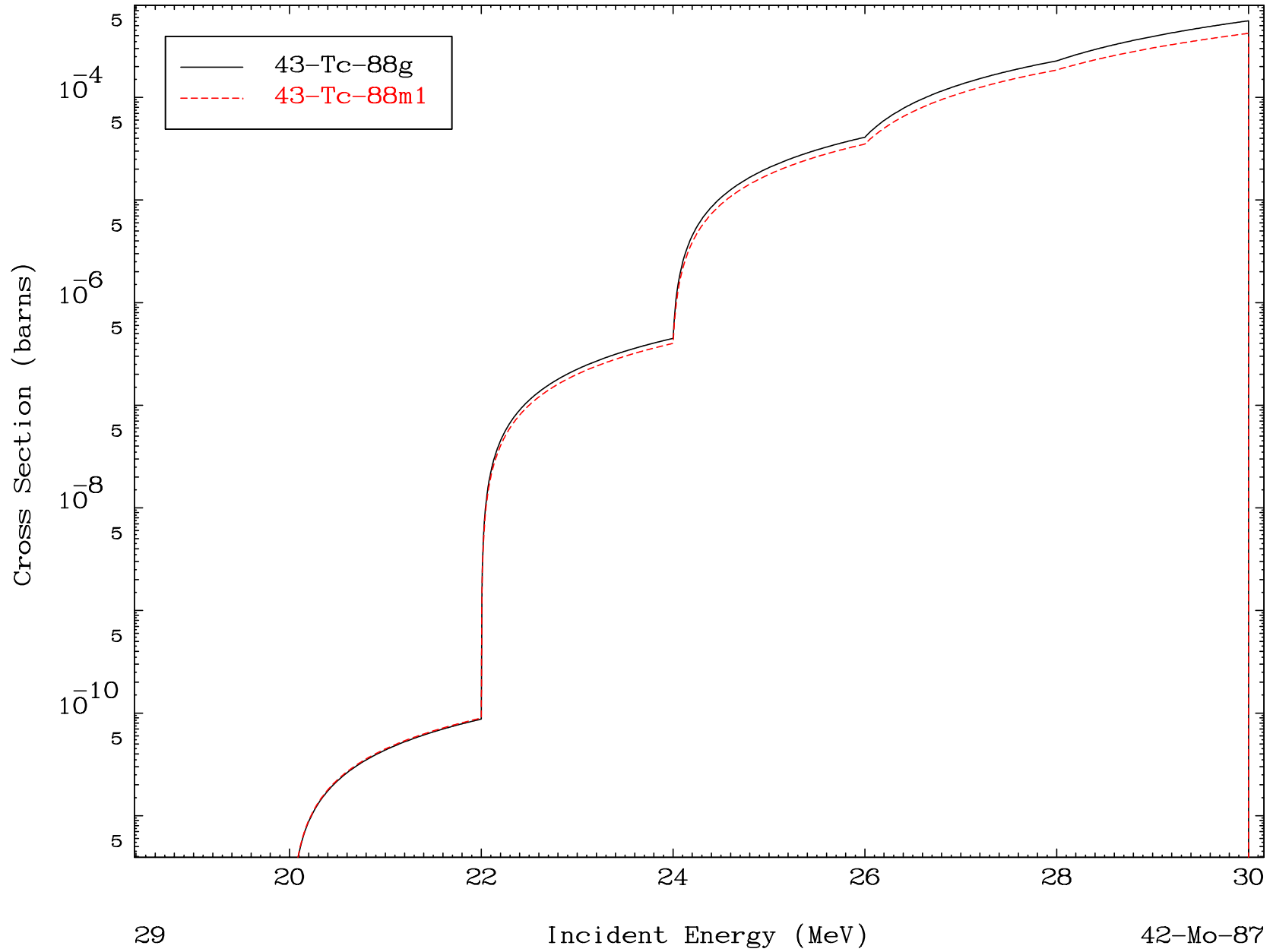
42-Mo-87

MAT 4210

( $\alpha, t$ )

42-Mo-87

Radionuclide Production Cross Section

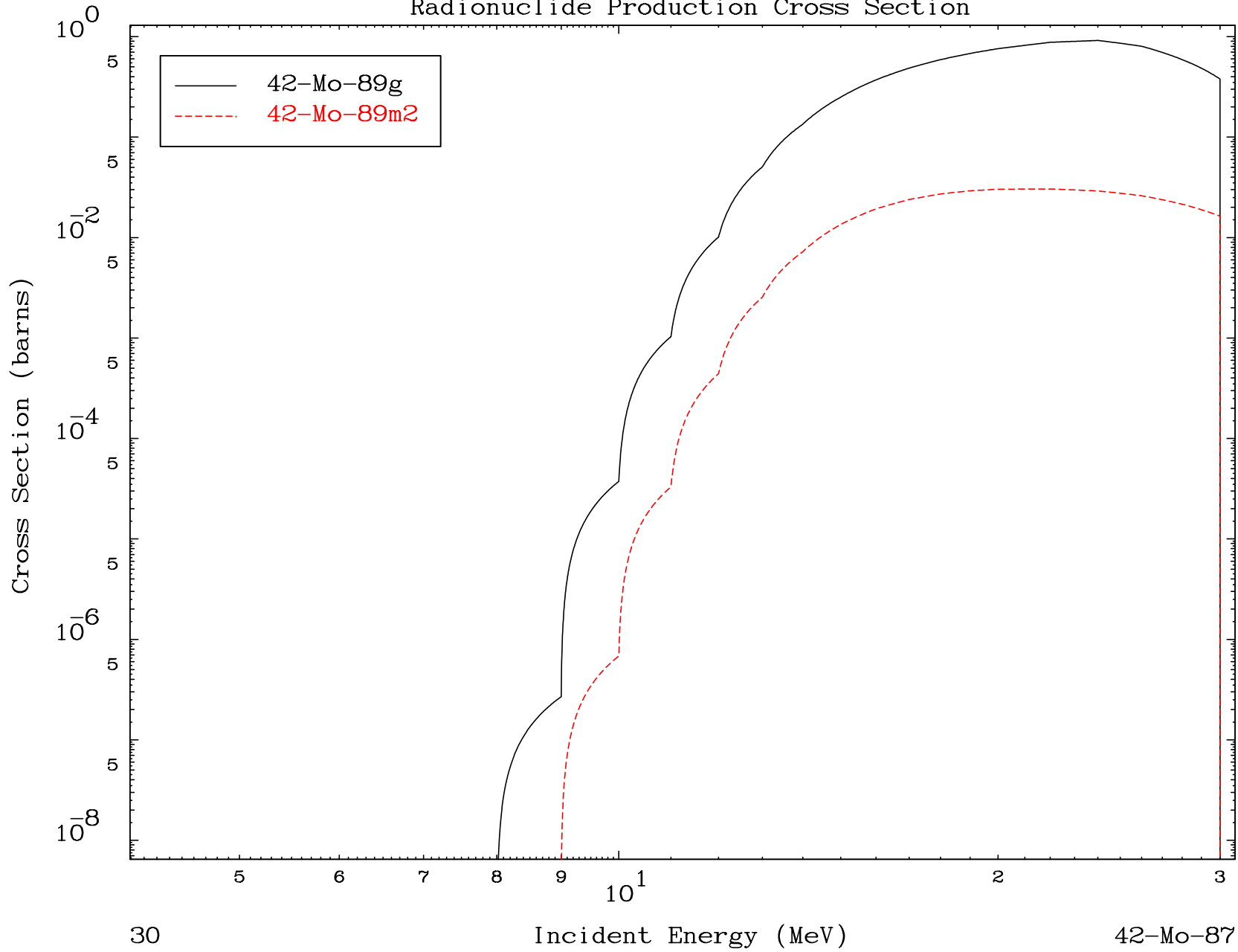


MAT 4210

( $\alpha, 2p$ )

42-Mo-87

Radionuclide Production Cross Section



Radionuclide Production Cross Section

