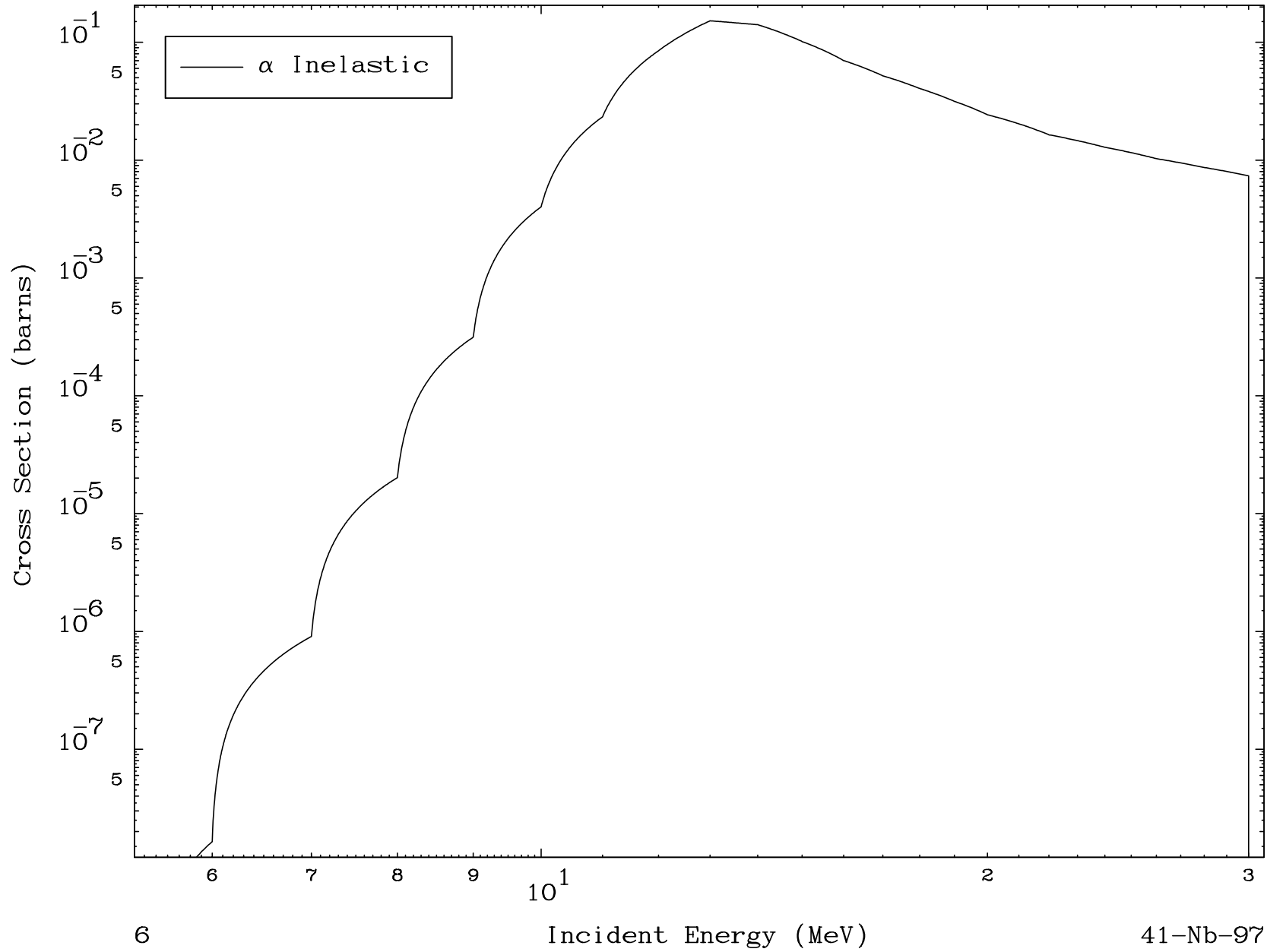


MAT 4137

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

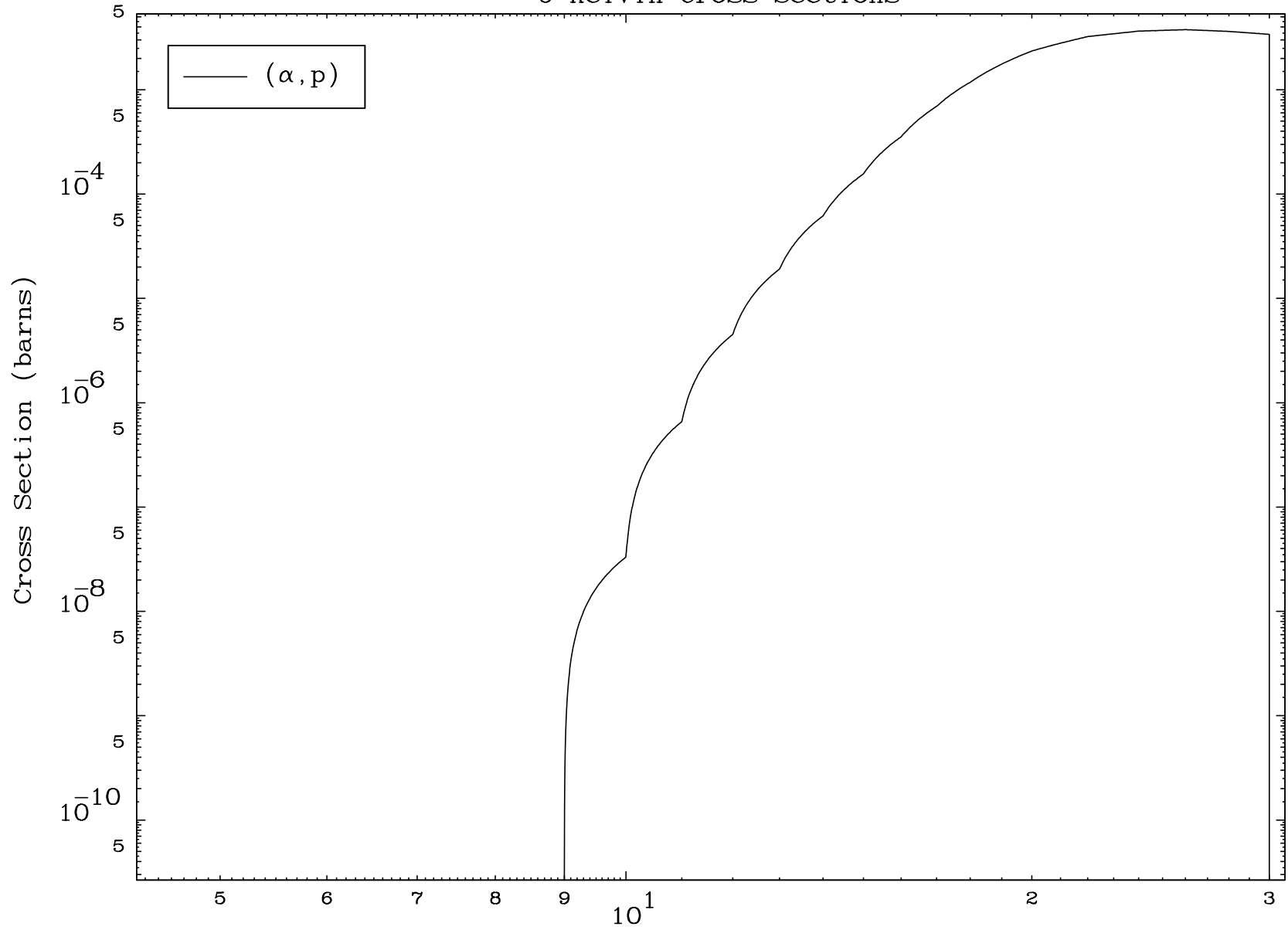
41-Nb-97

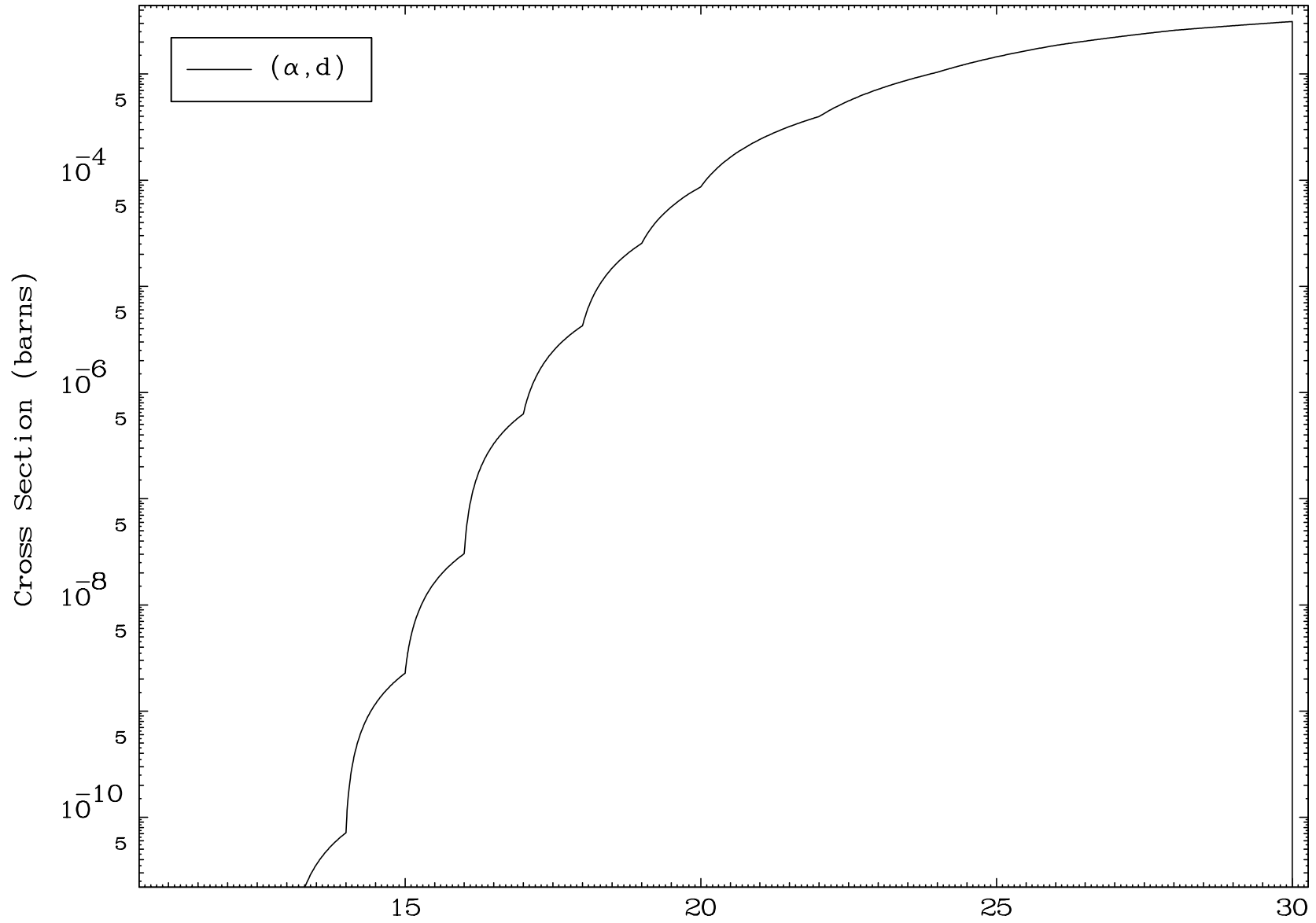


6

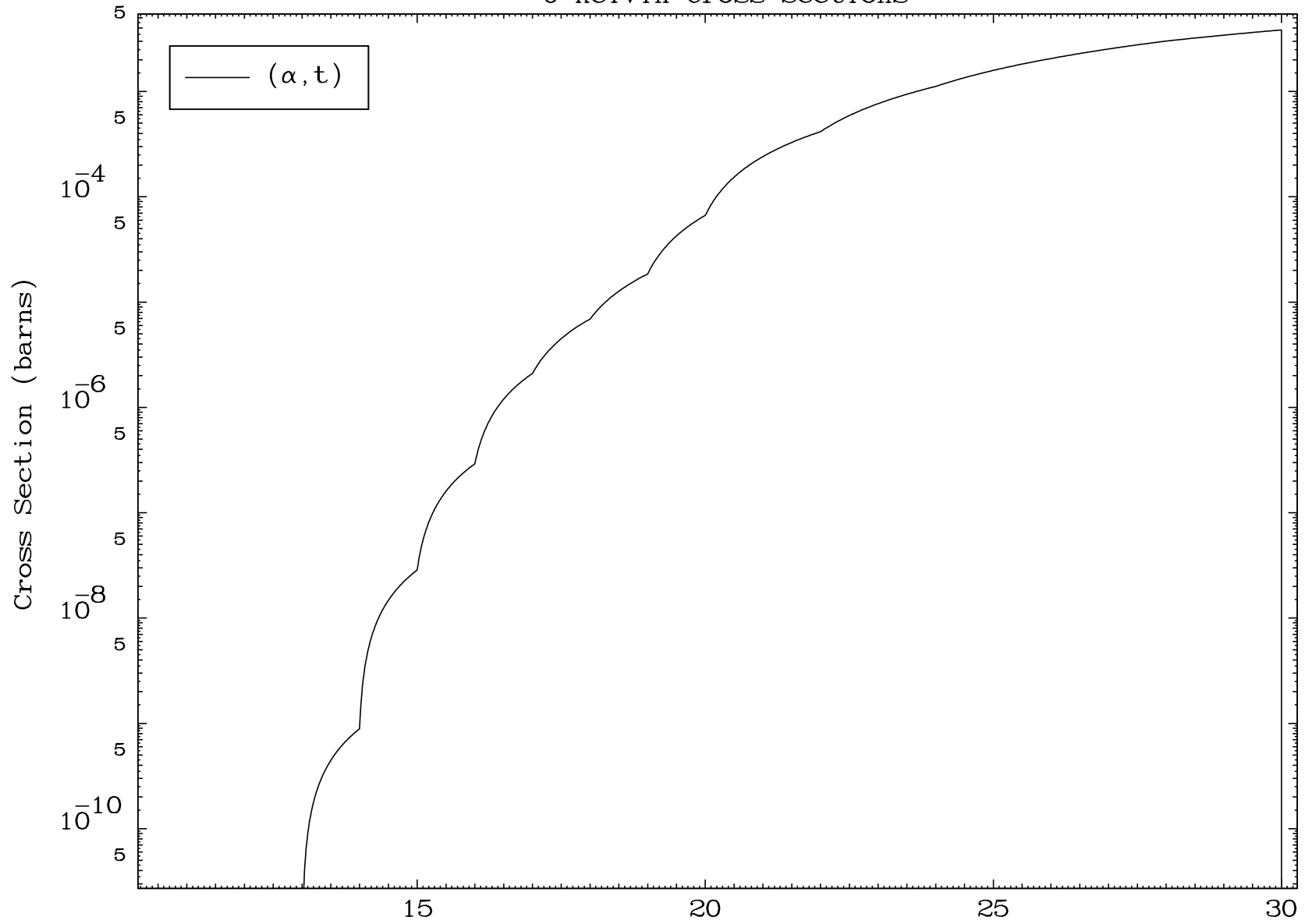
Incident Energy (MeV)

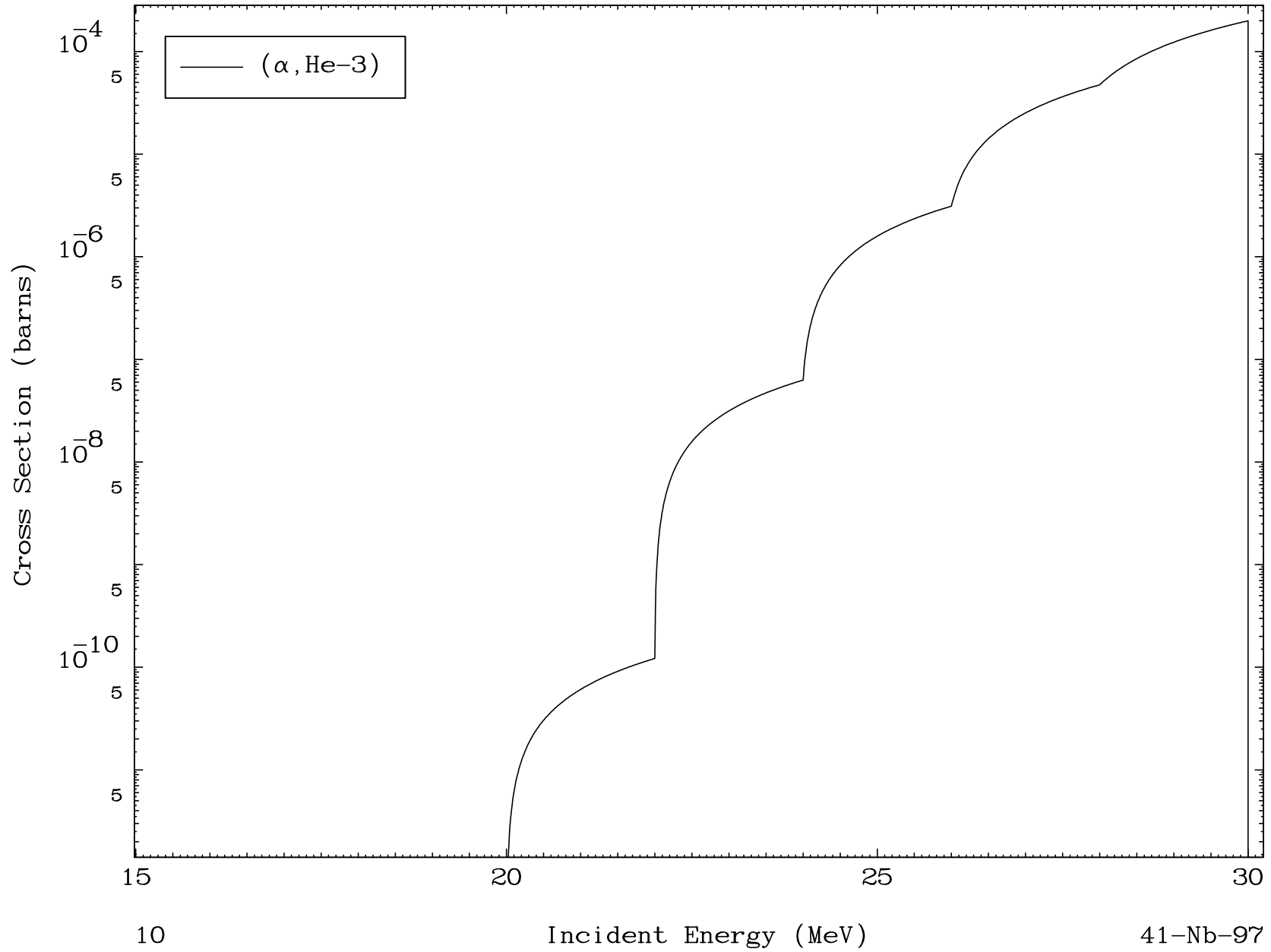
41-Nb-97

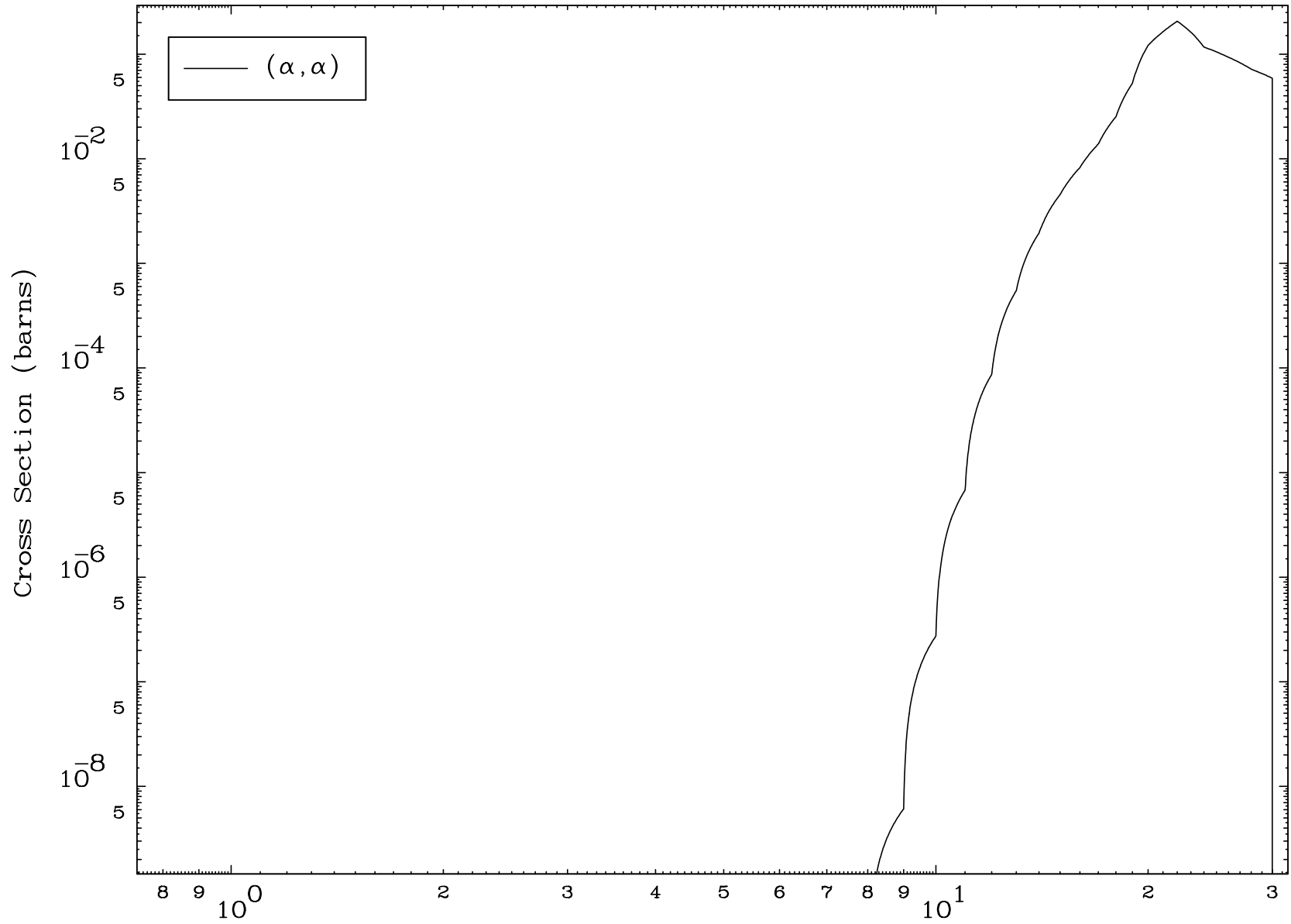




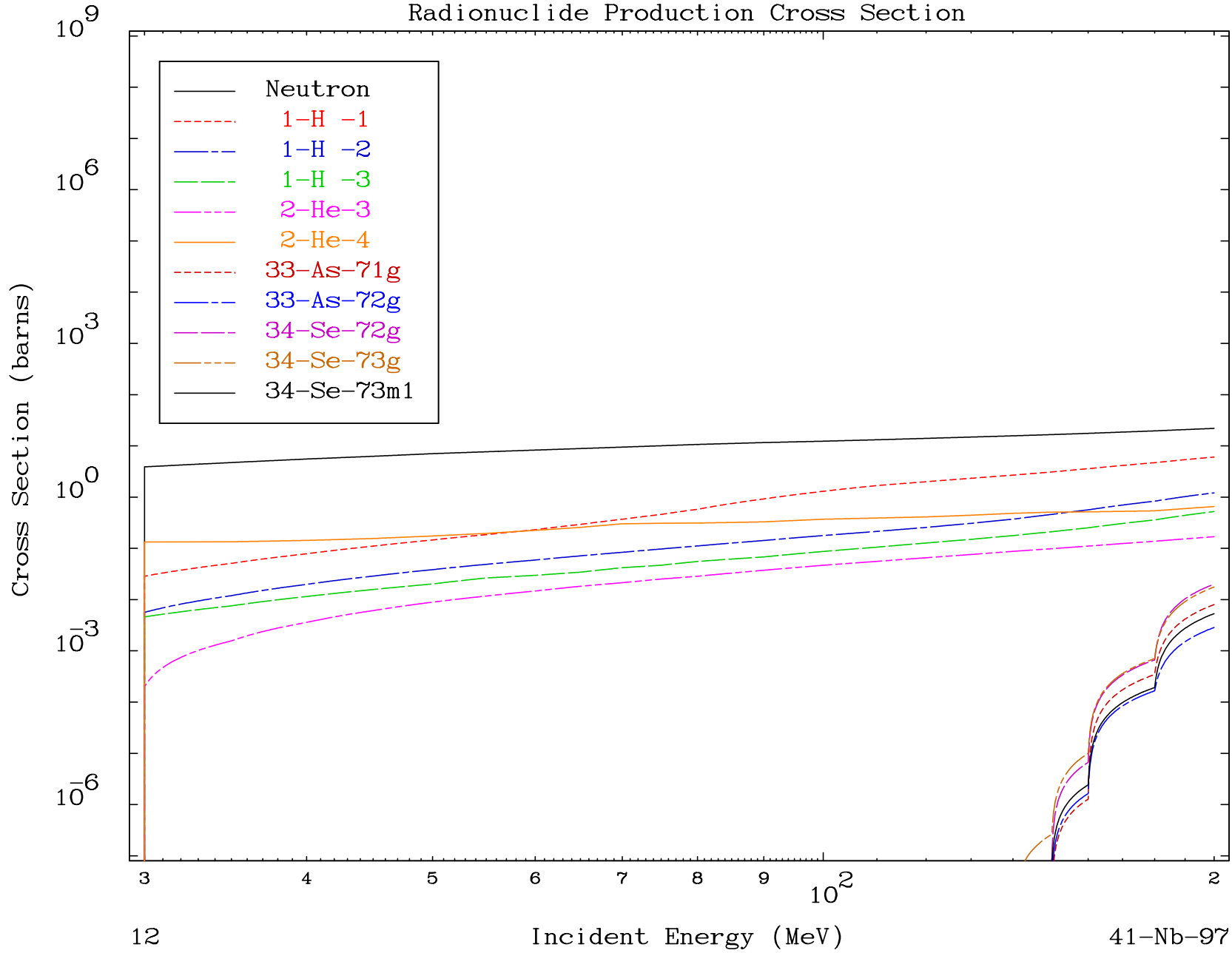


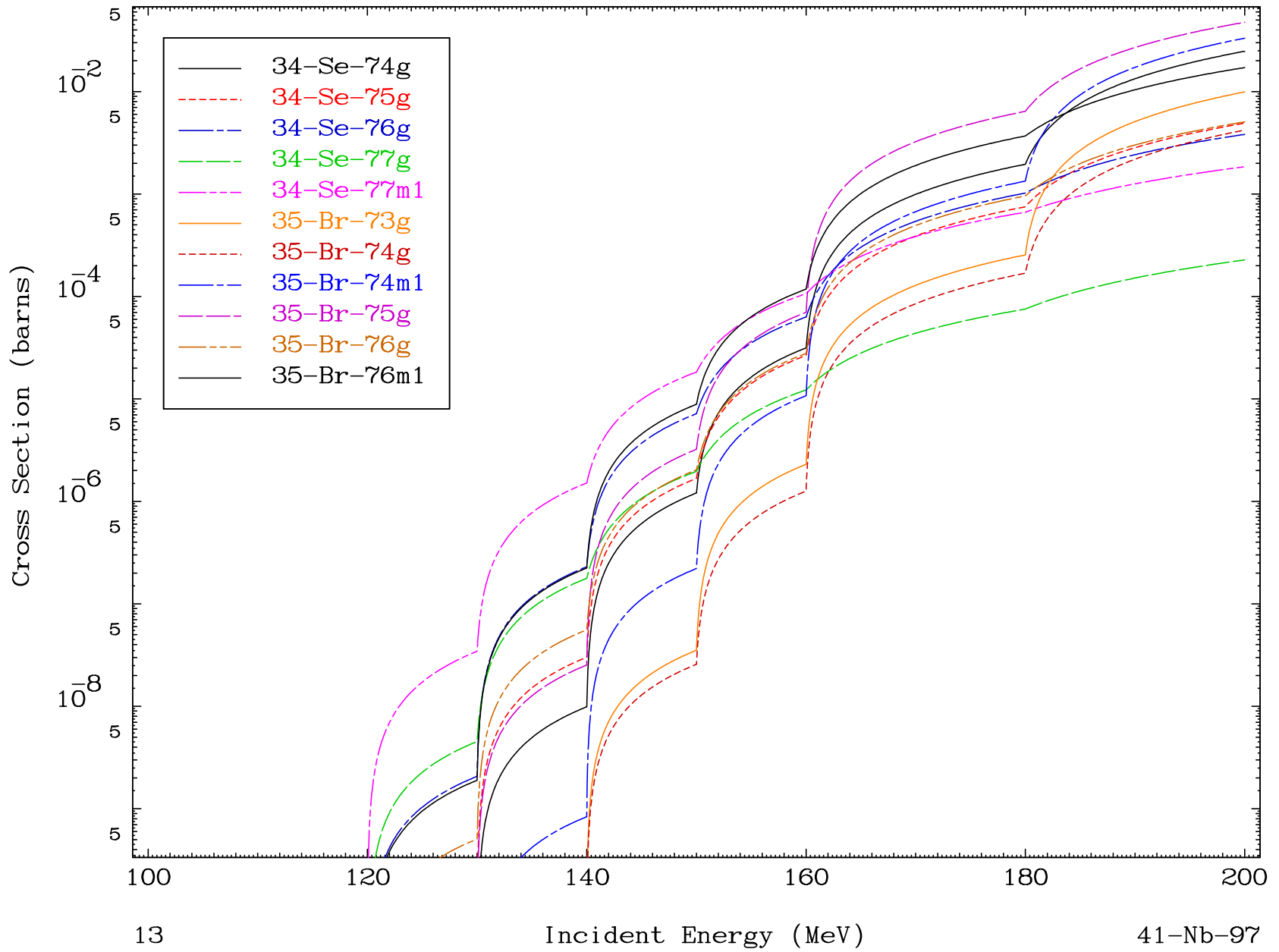




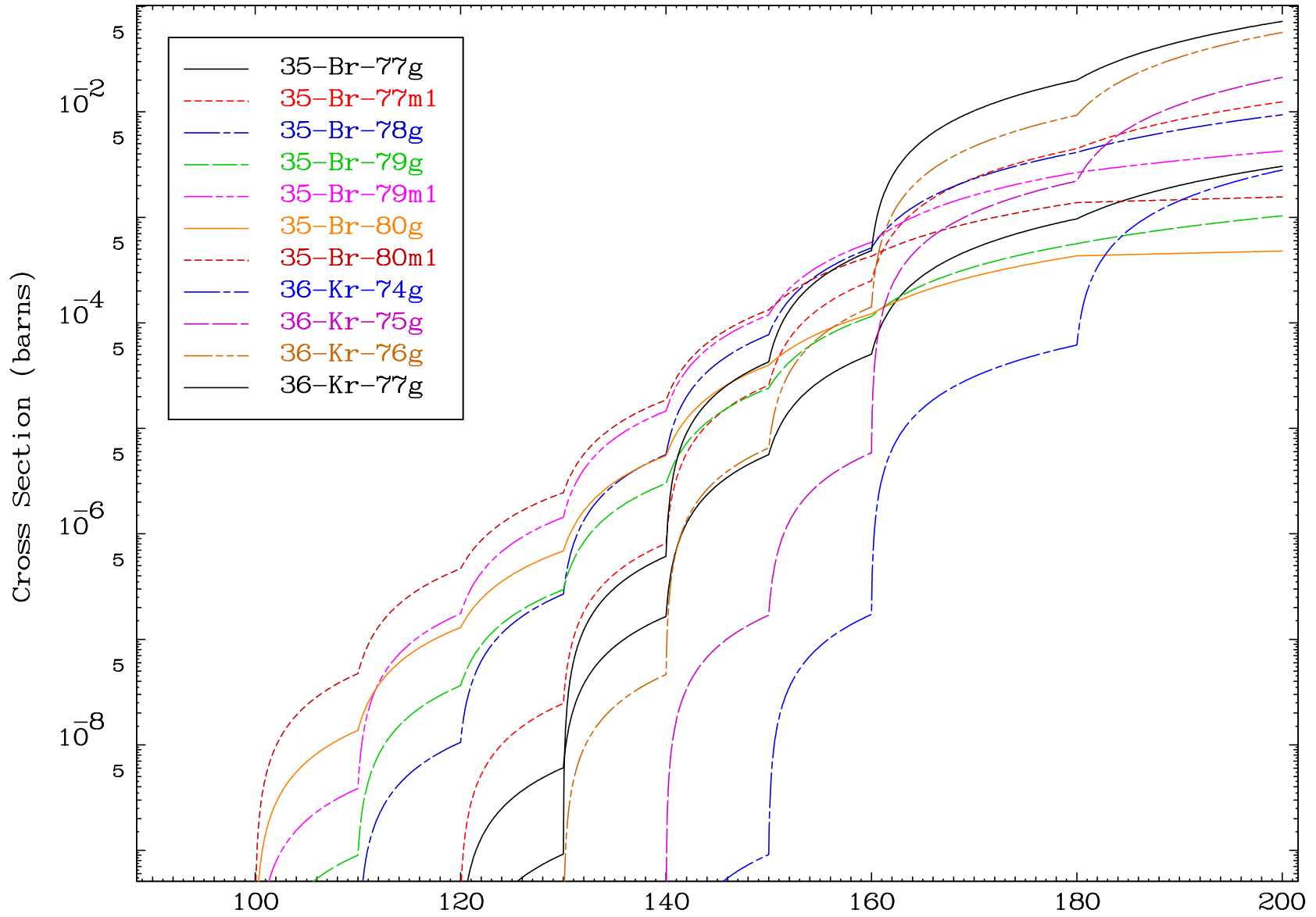


Radionuclide Production Cross Section

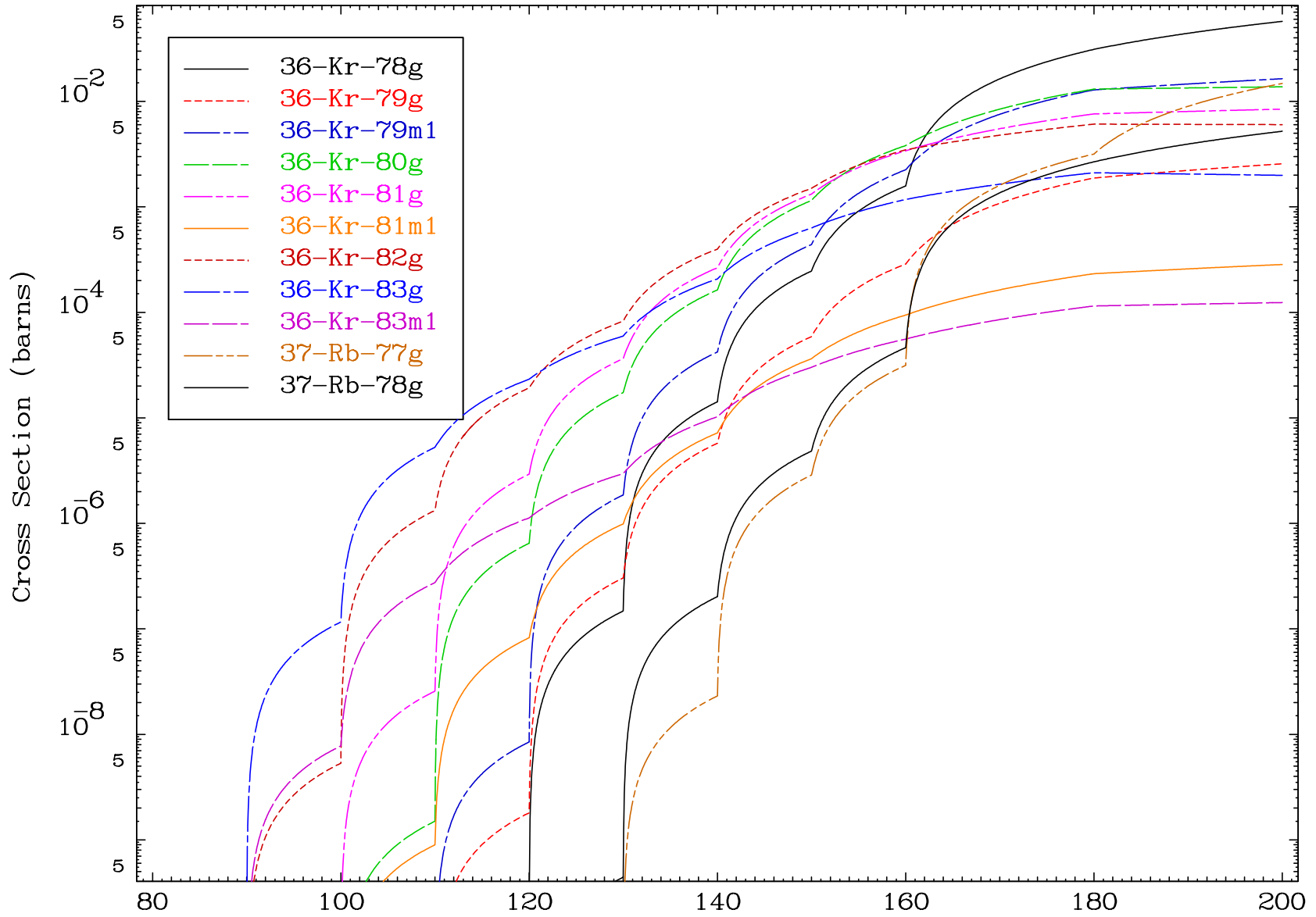


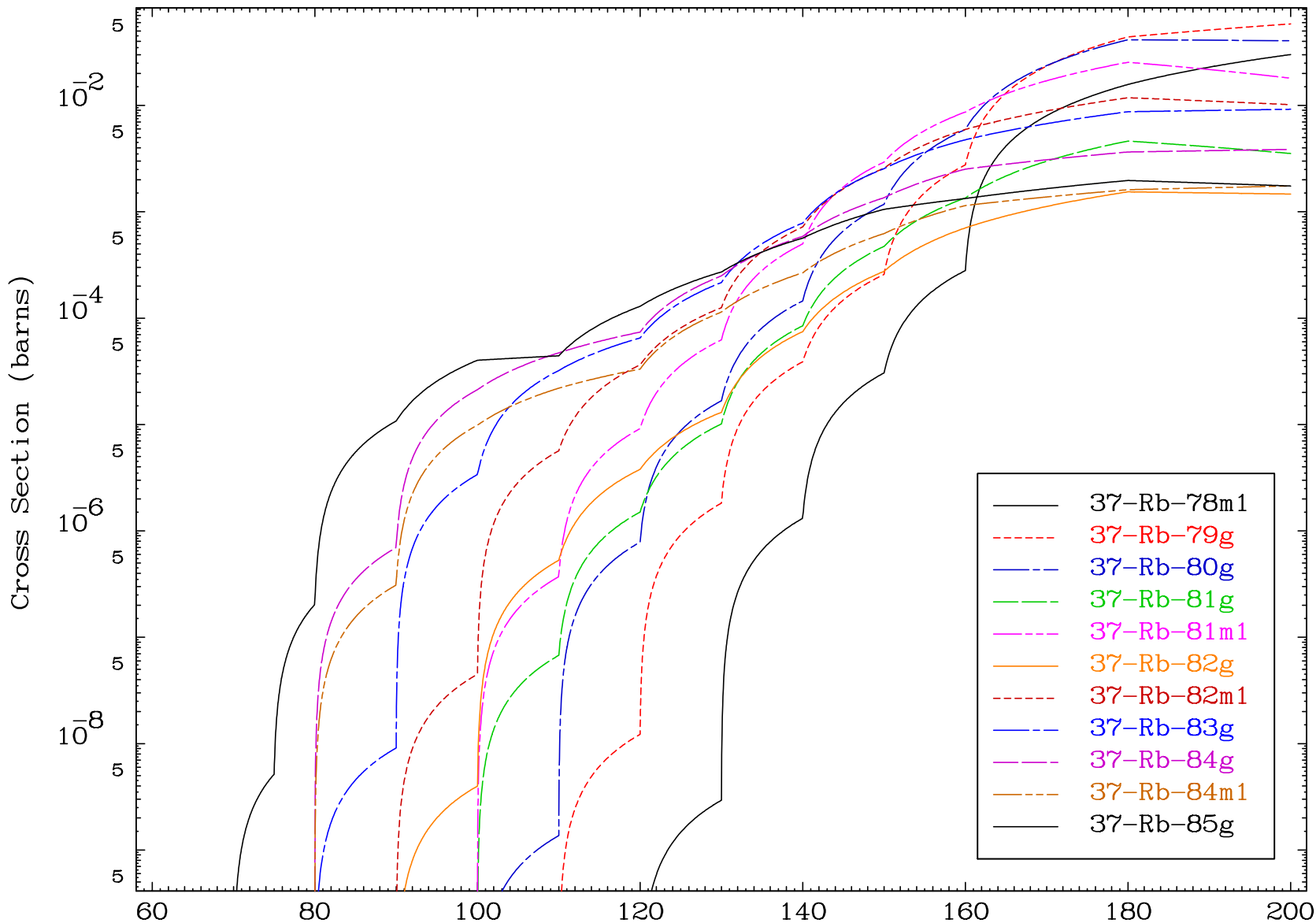


Radionuclide Production Cross Section

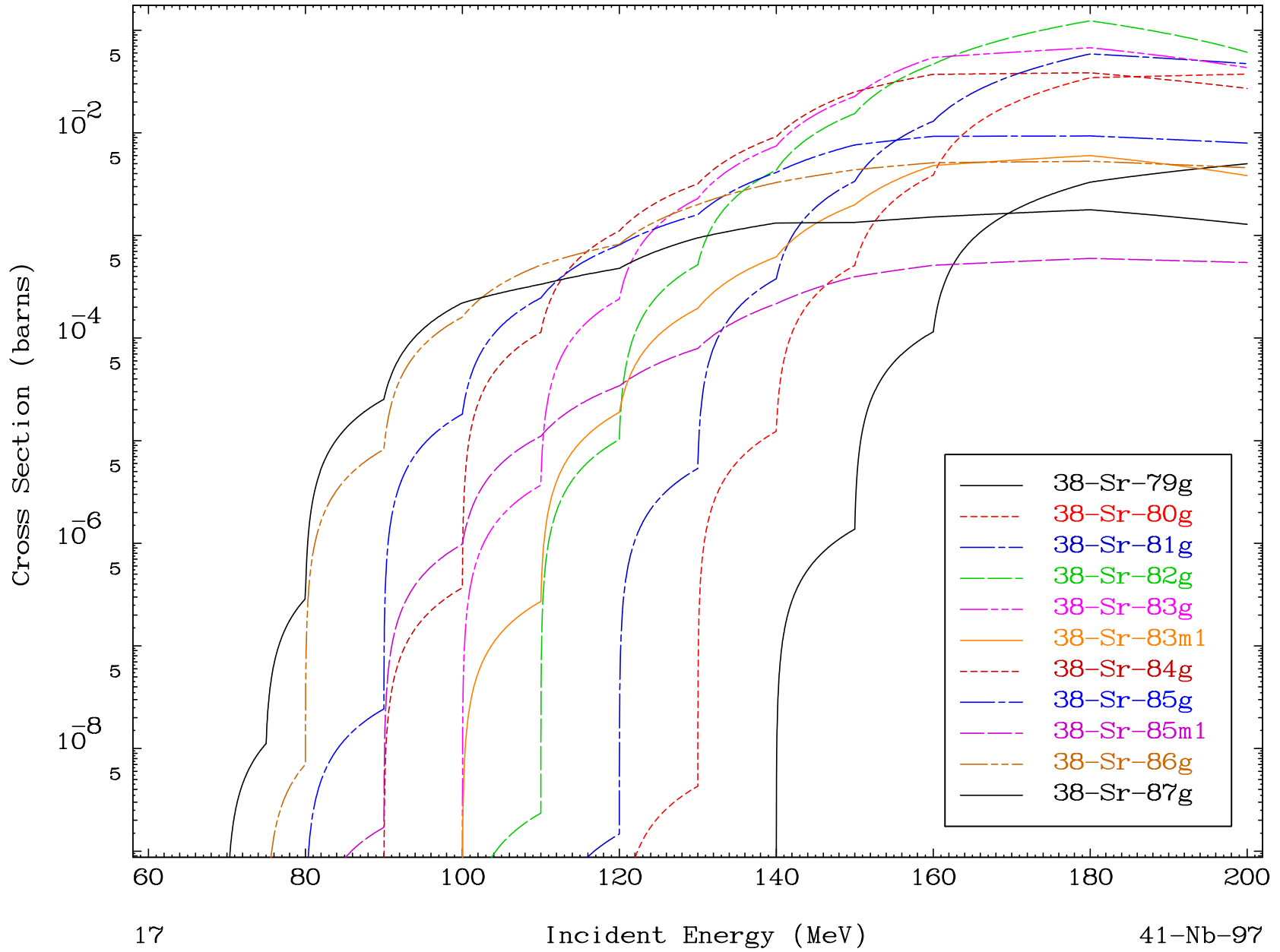


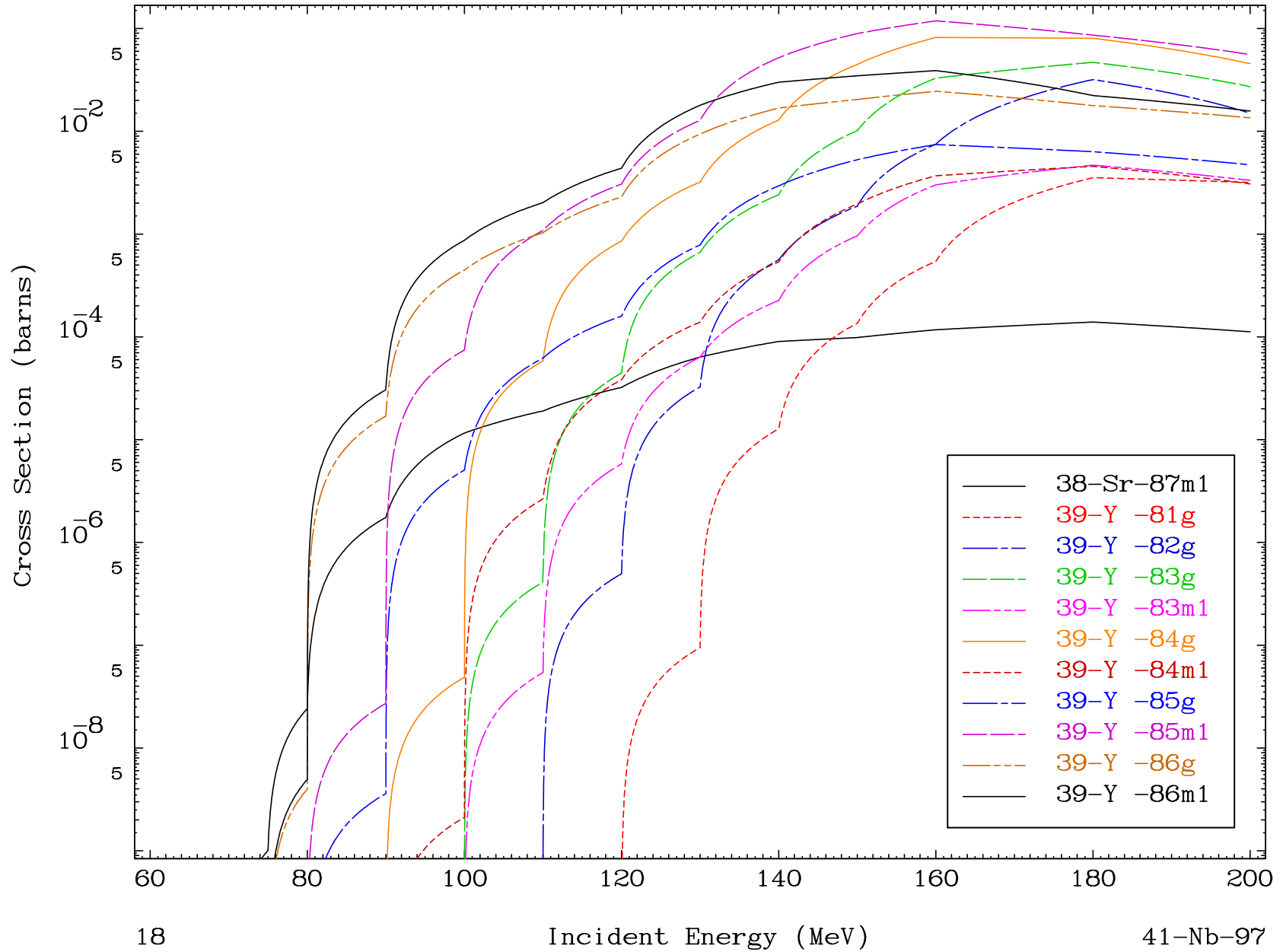
Radionuclide Production Cross Section

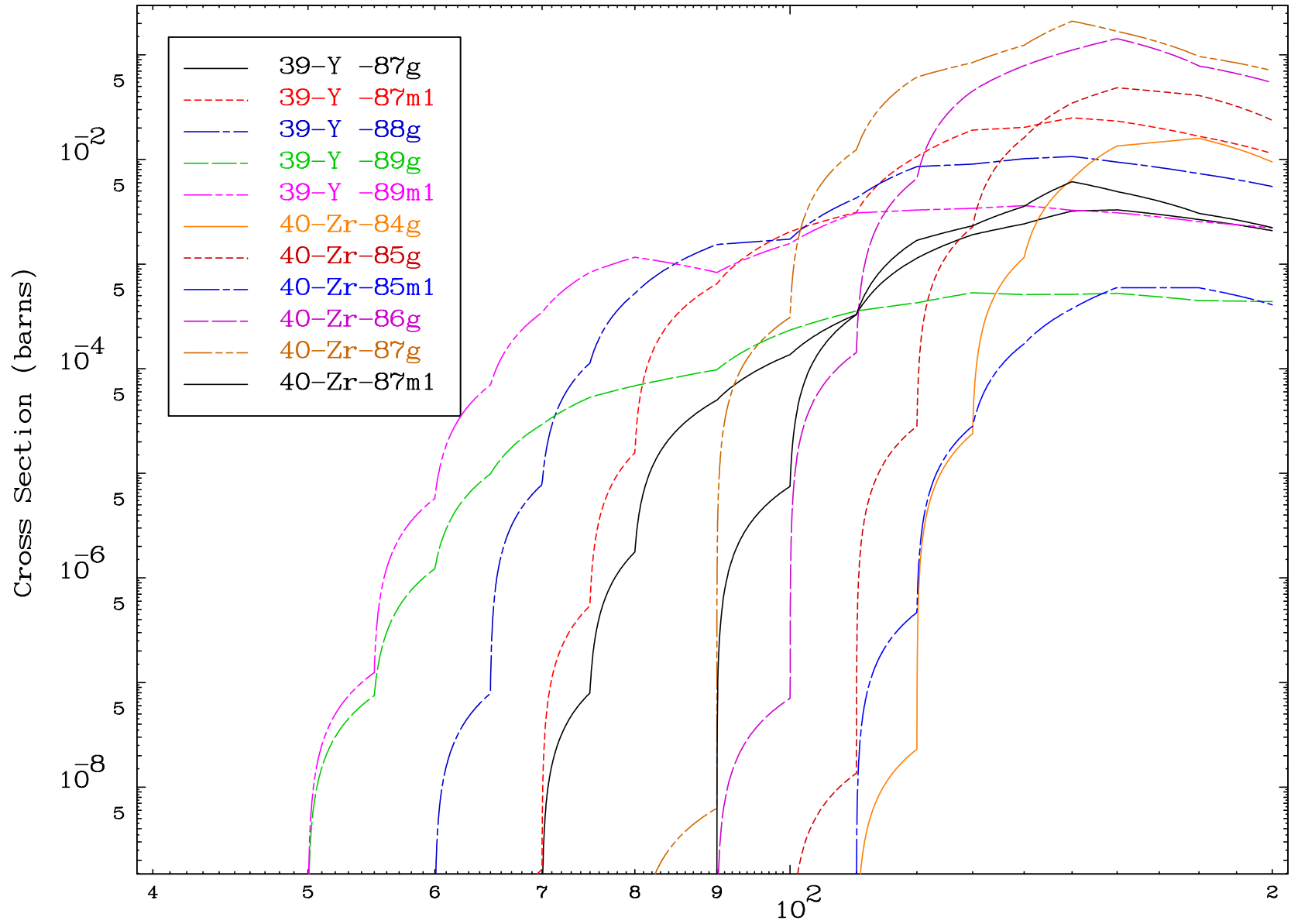


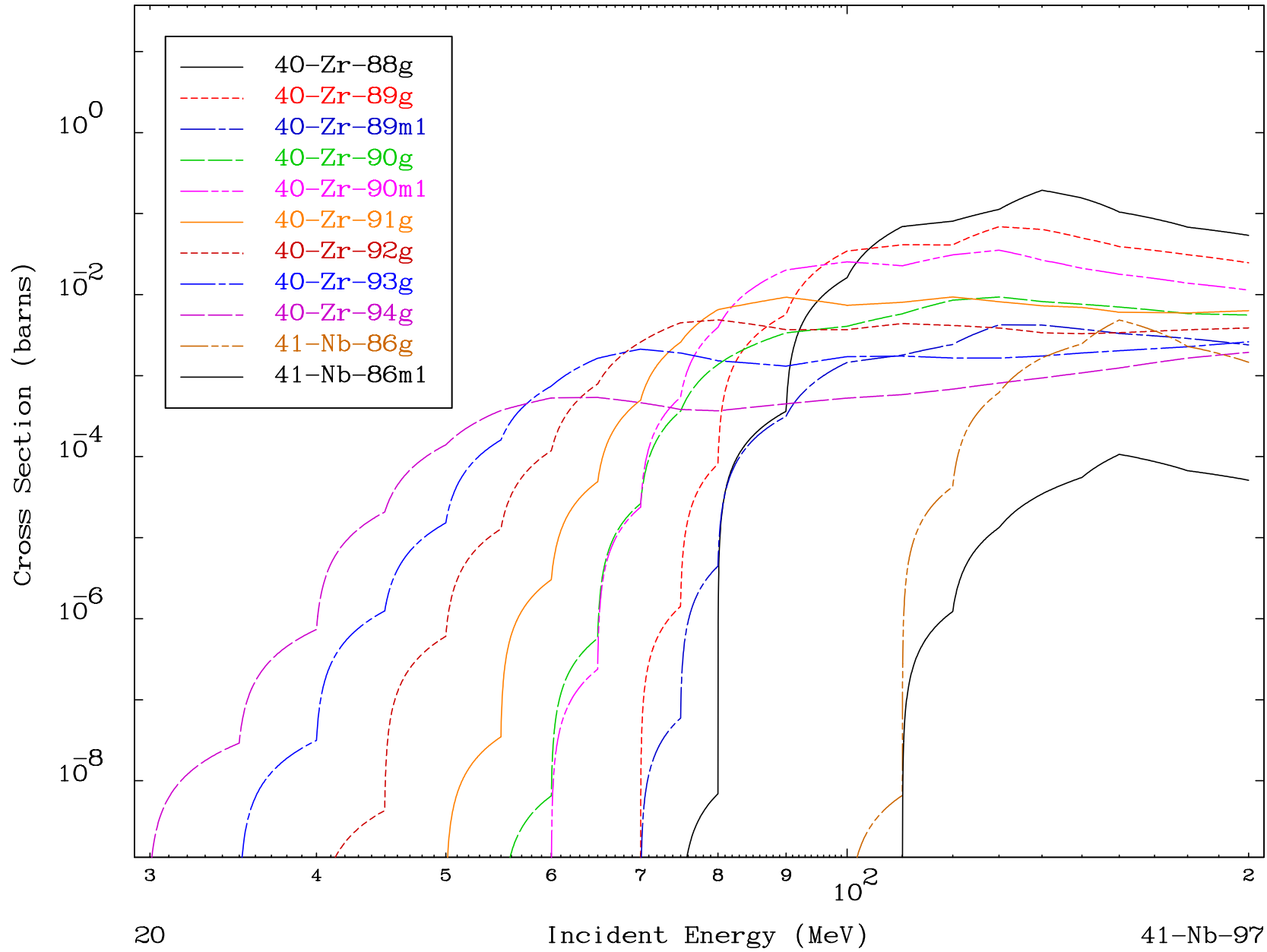


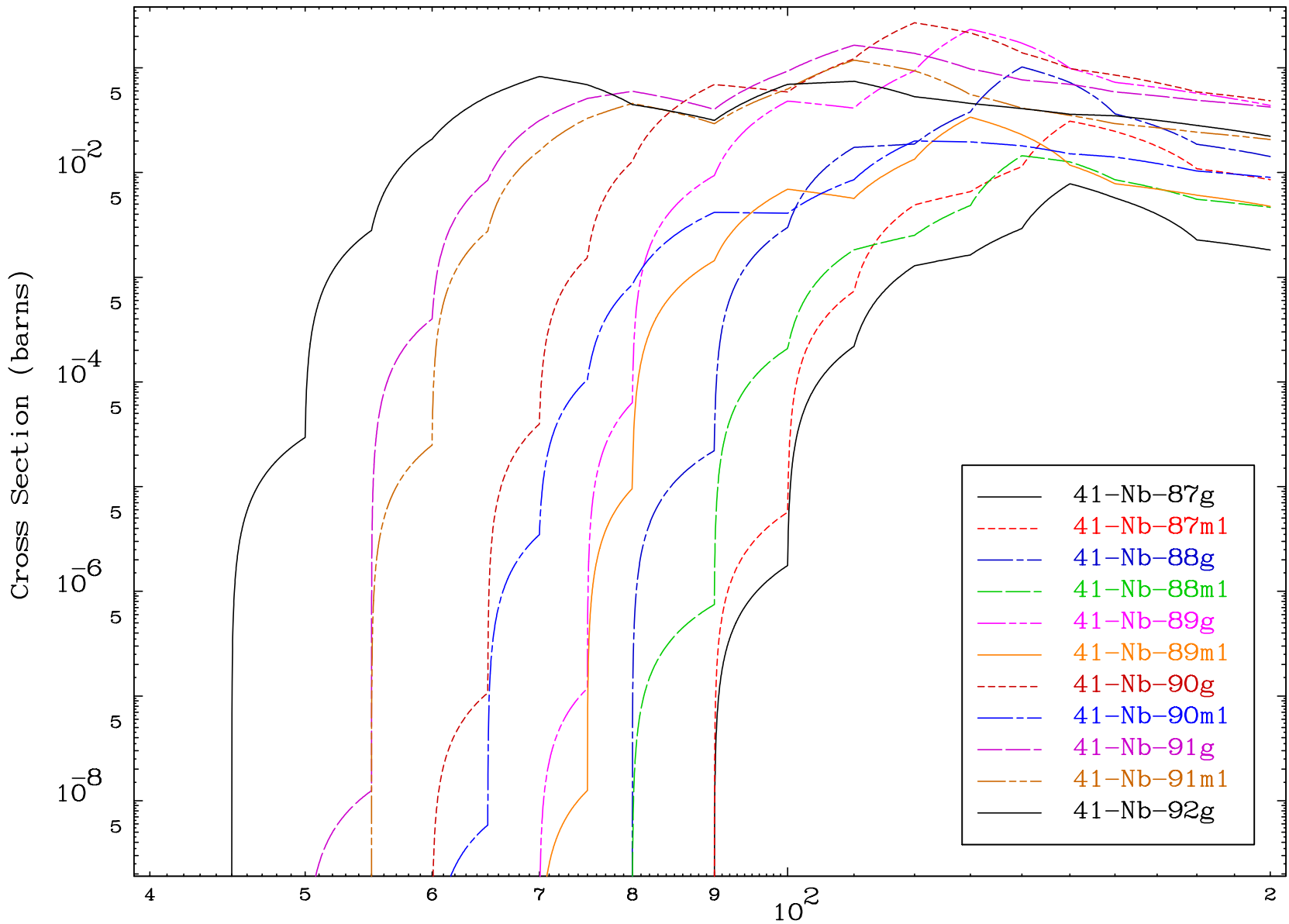




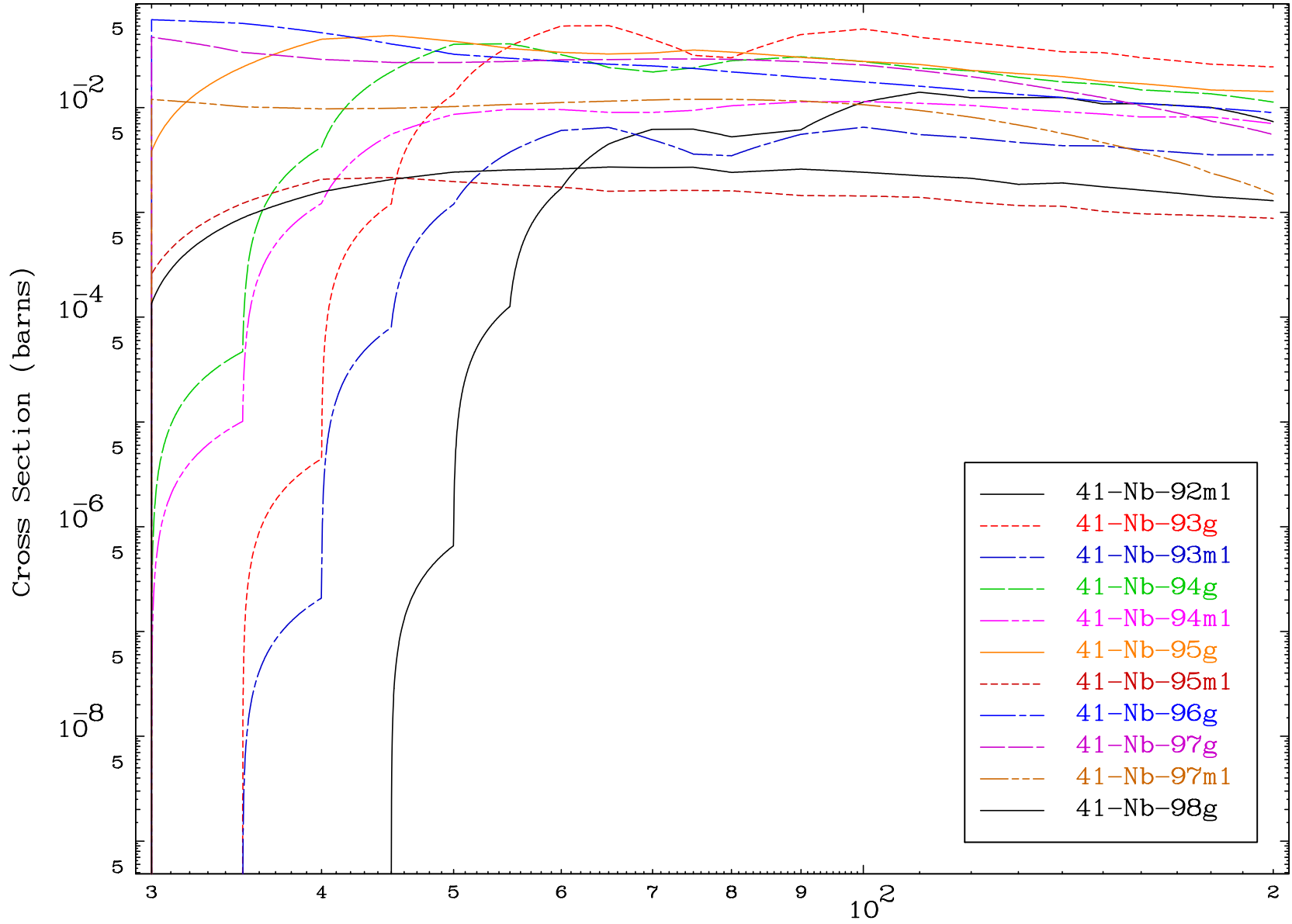


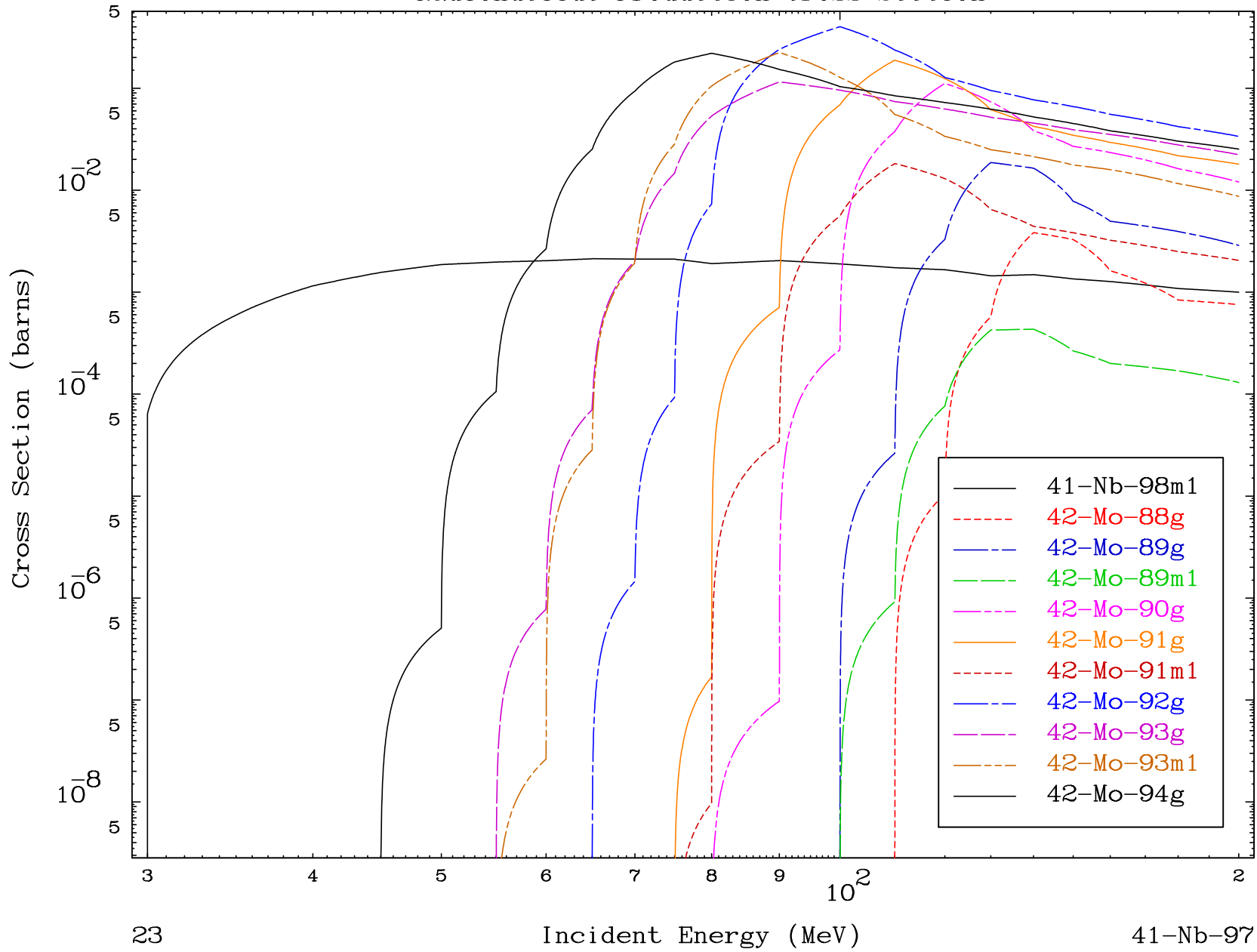


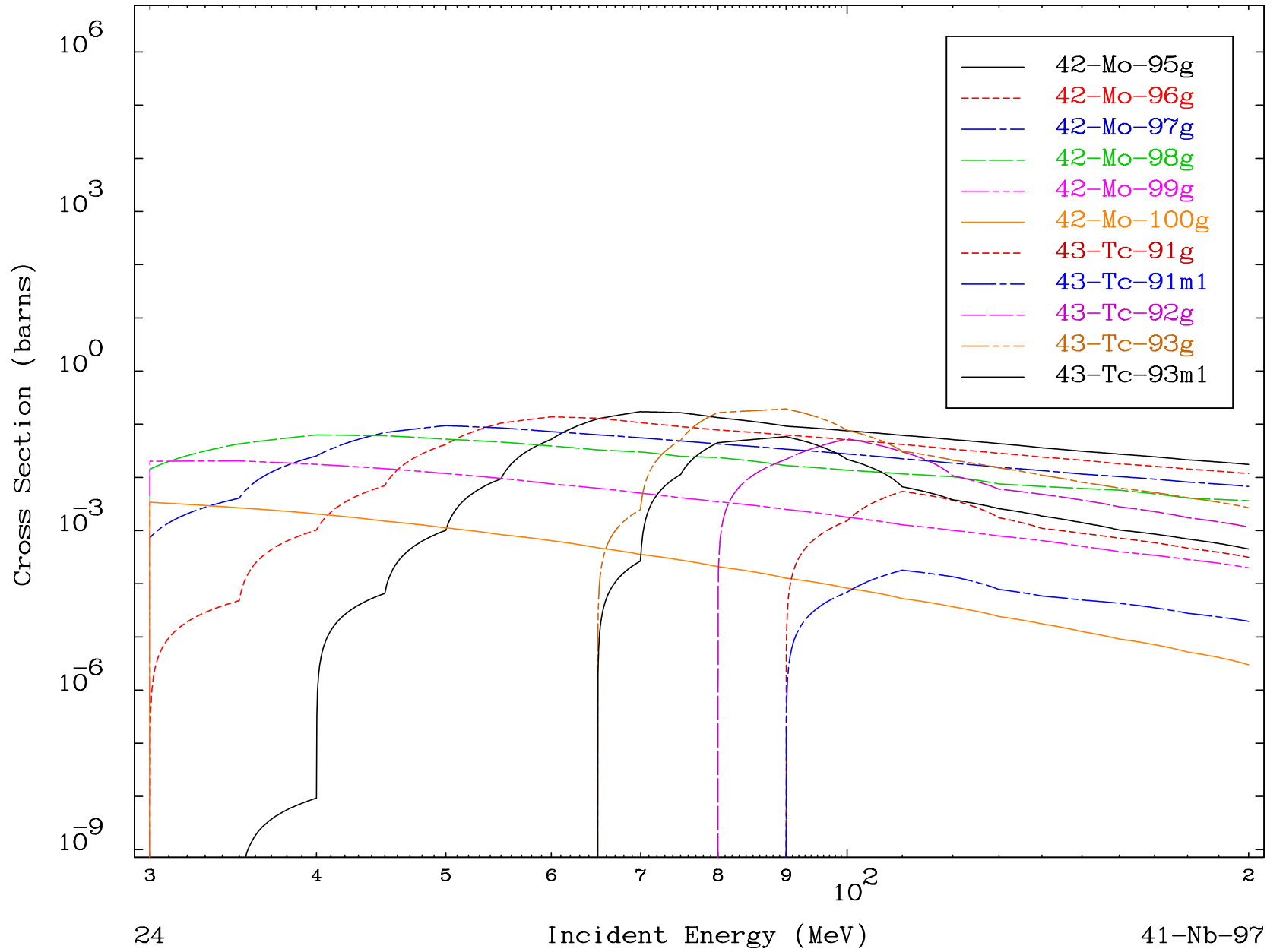




Radionuclide Production Cross Section

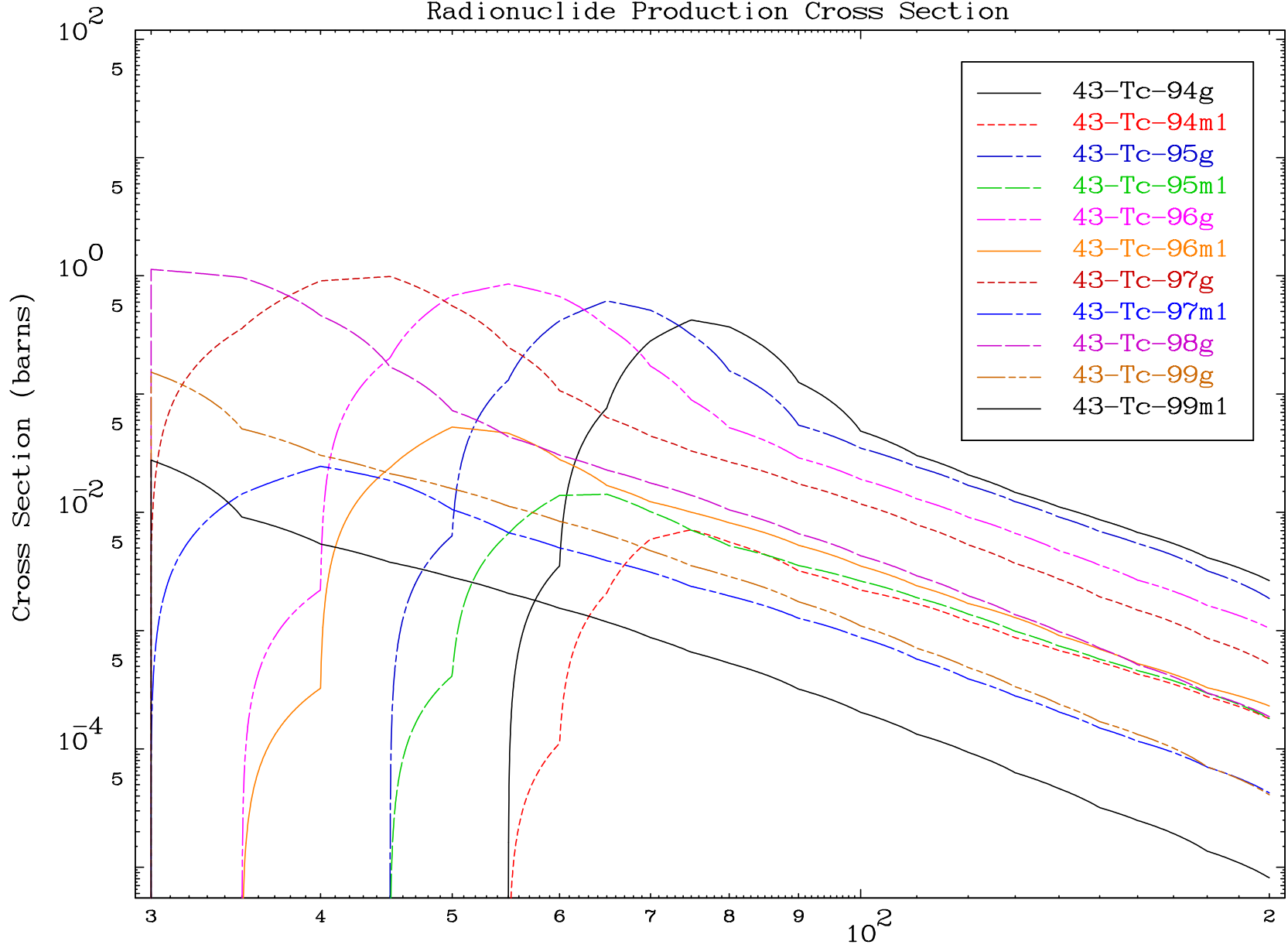


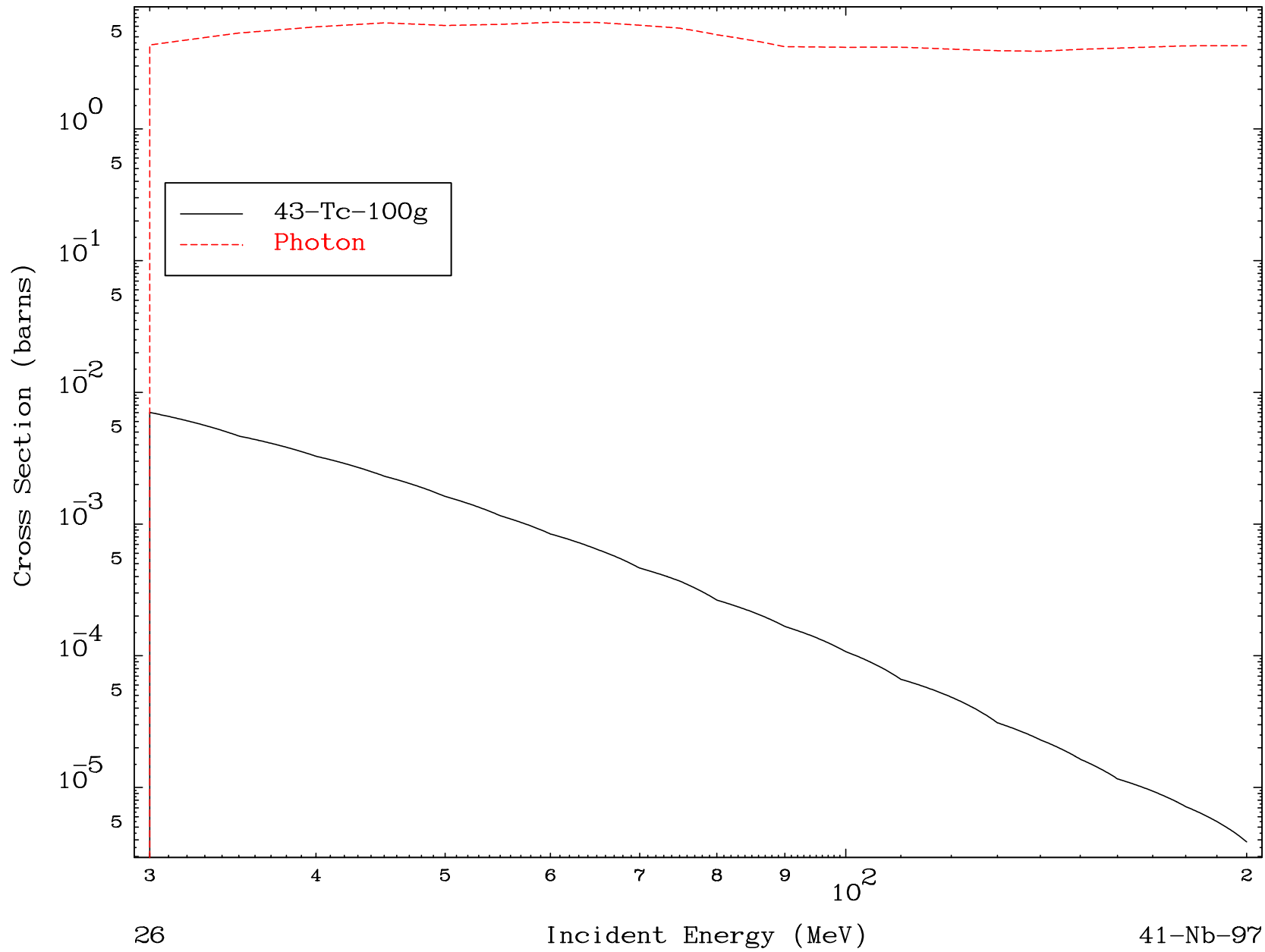






Radionuclide Production Cross Section



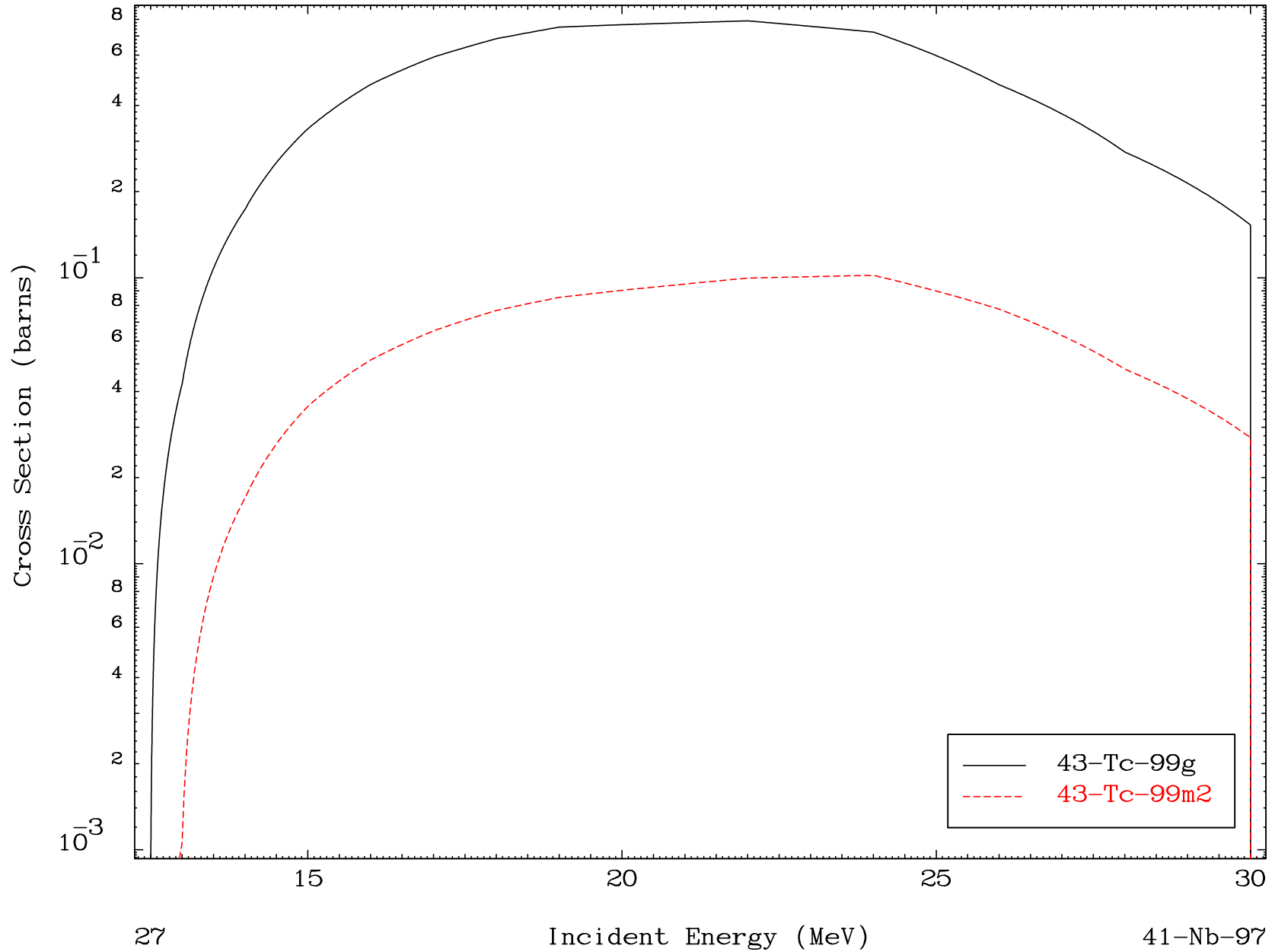


MAT 4137

( $\alpha, 2n$ )

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Radionuclide Production Cross Section

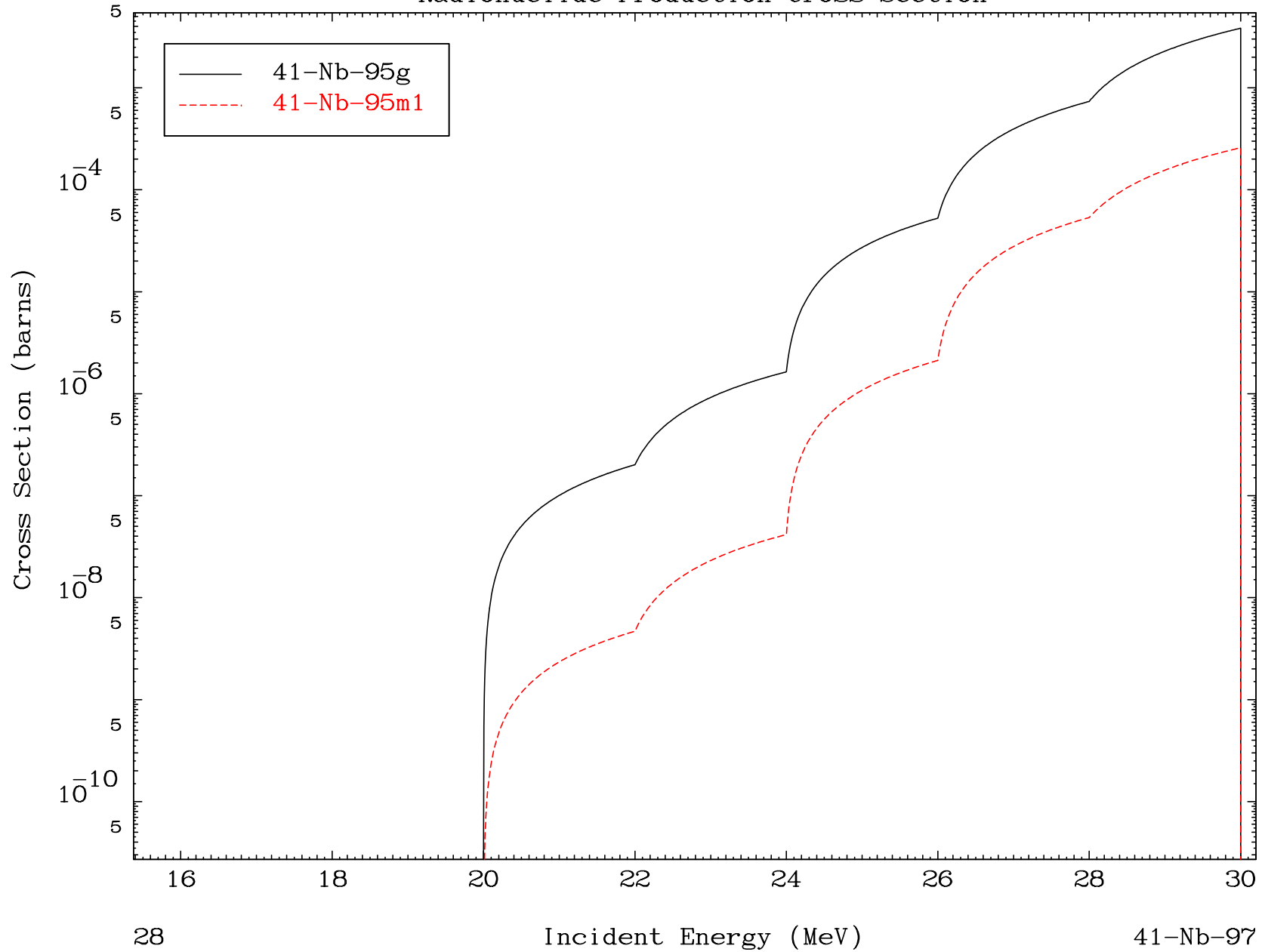


MAT 4137

( $\alpha, 2n$ )  $\alpha$

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Radionuclide Production Cross Section

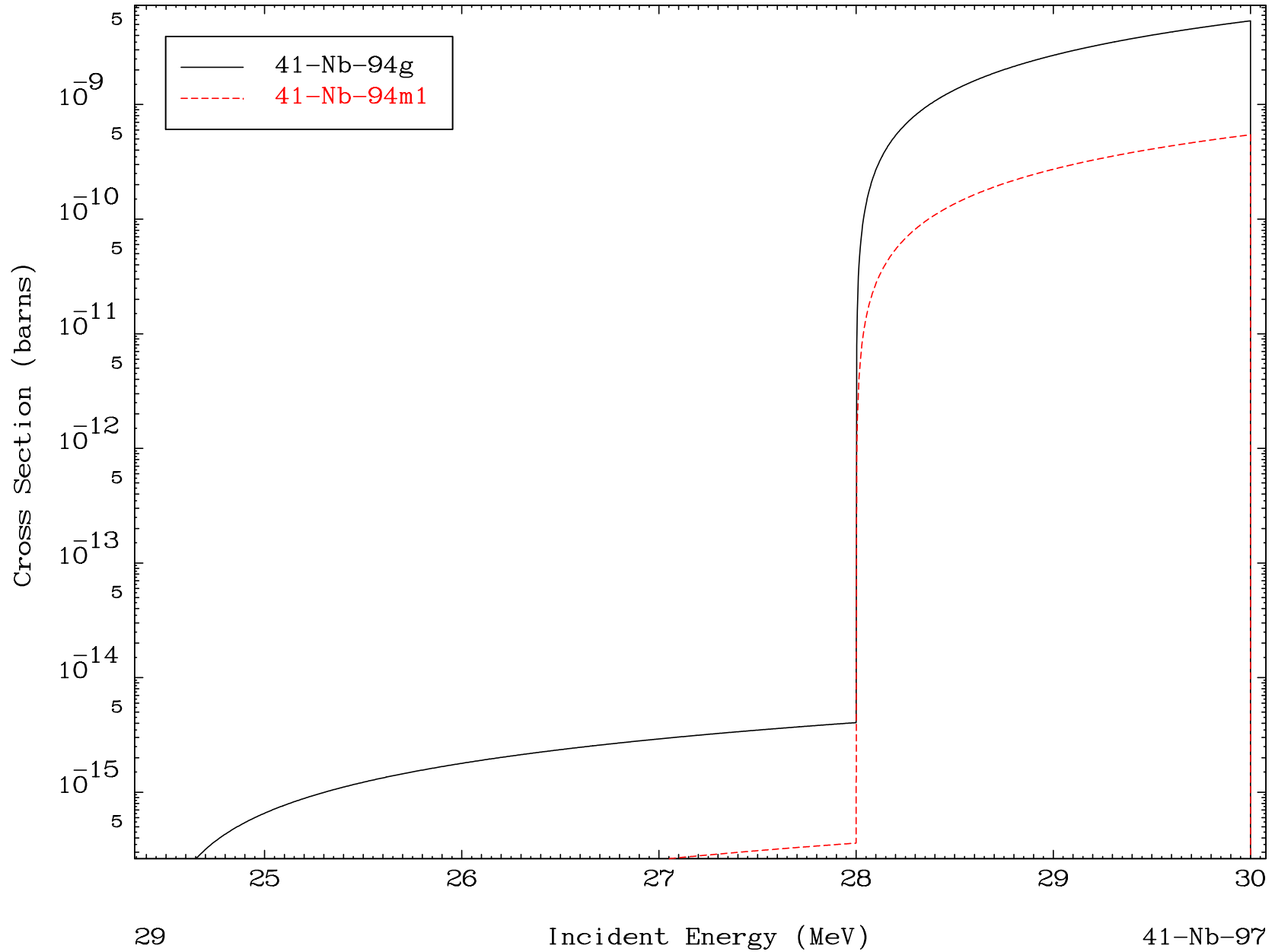


28

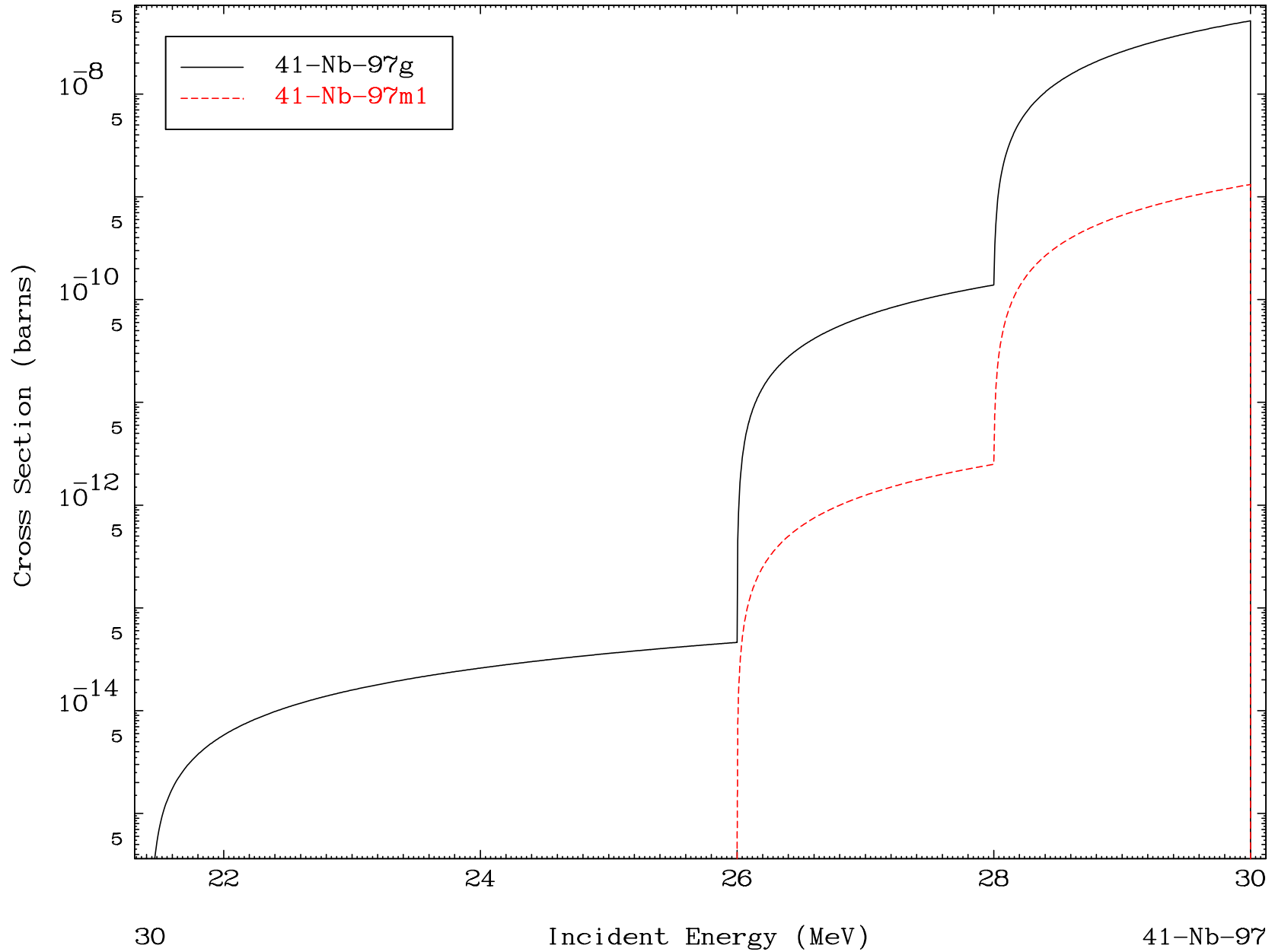
Incident Energy (MeV)

41-Nb-97

Radionuclide Production Cross Section



Radionuclide Production Cross Section

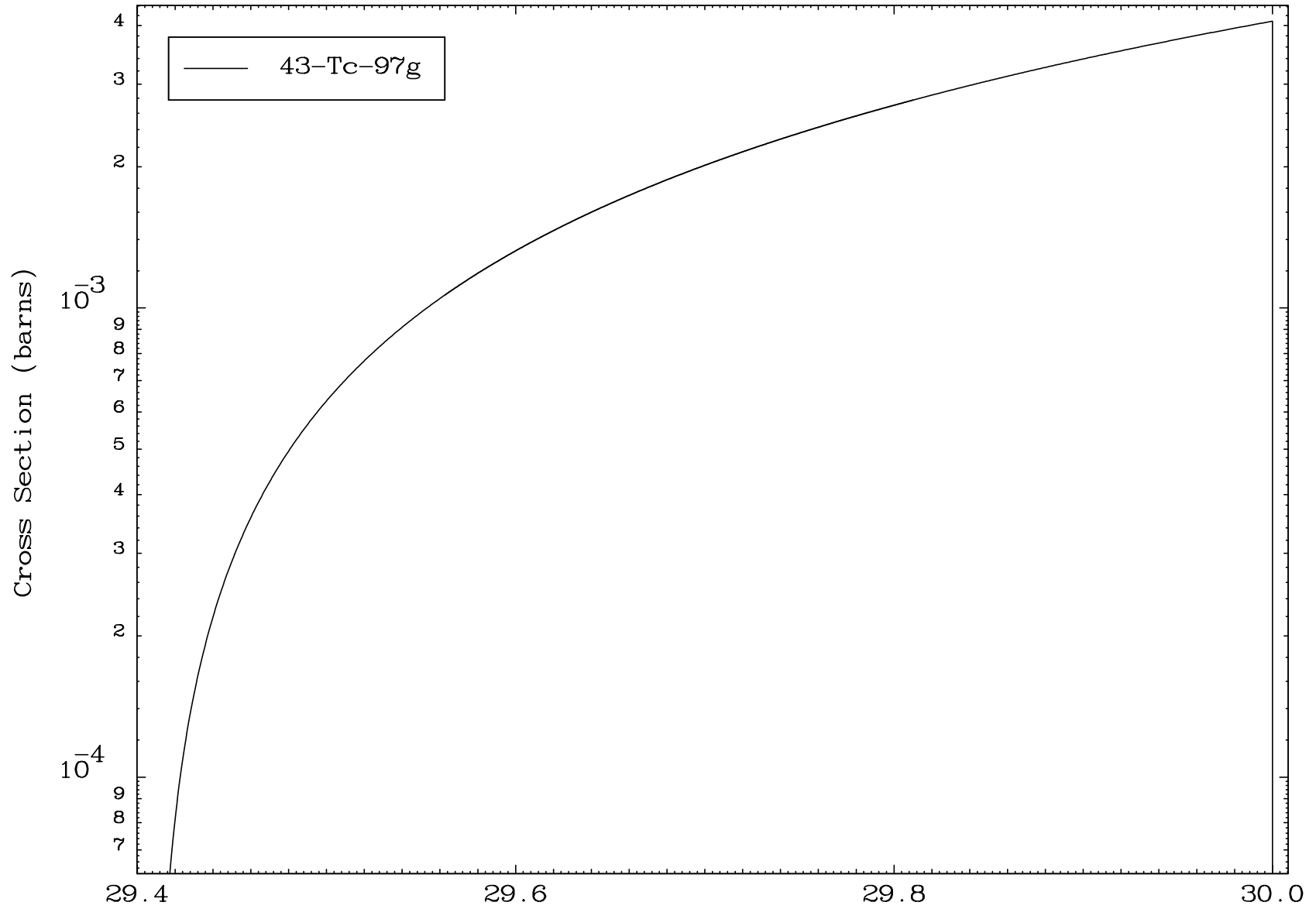


MAT 4137

( $\alpha, 4n$ )

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Radionuclide Production Cross Section



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Incident Energy (MeV)

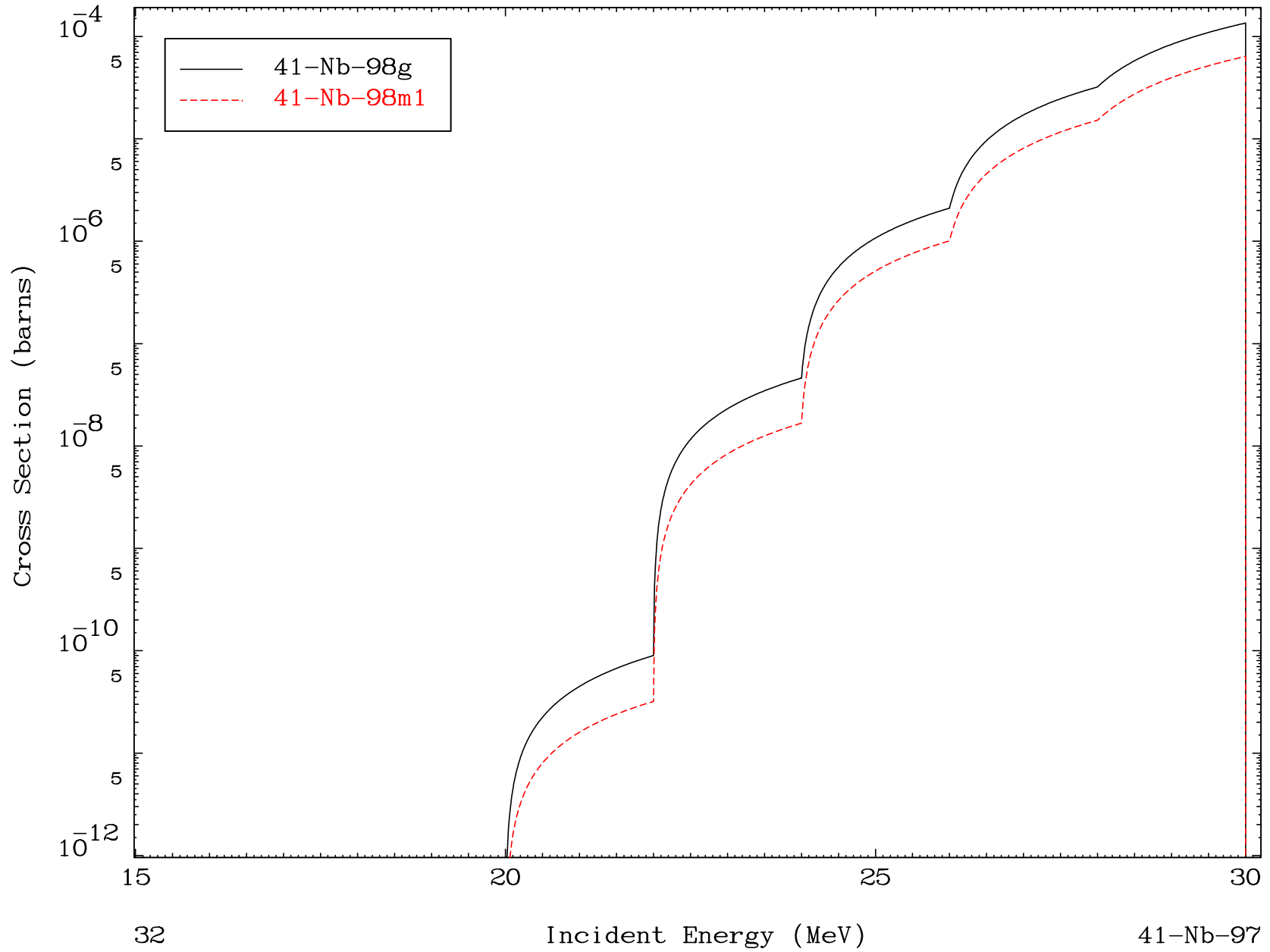
41-Nb-97

MAT 4137

( $\alpha, \text{He-3}$ )

41-Nb-97

Radionuclide Production Cross Section



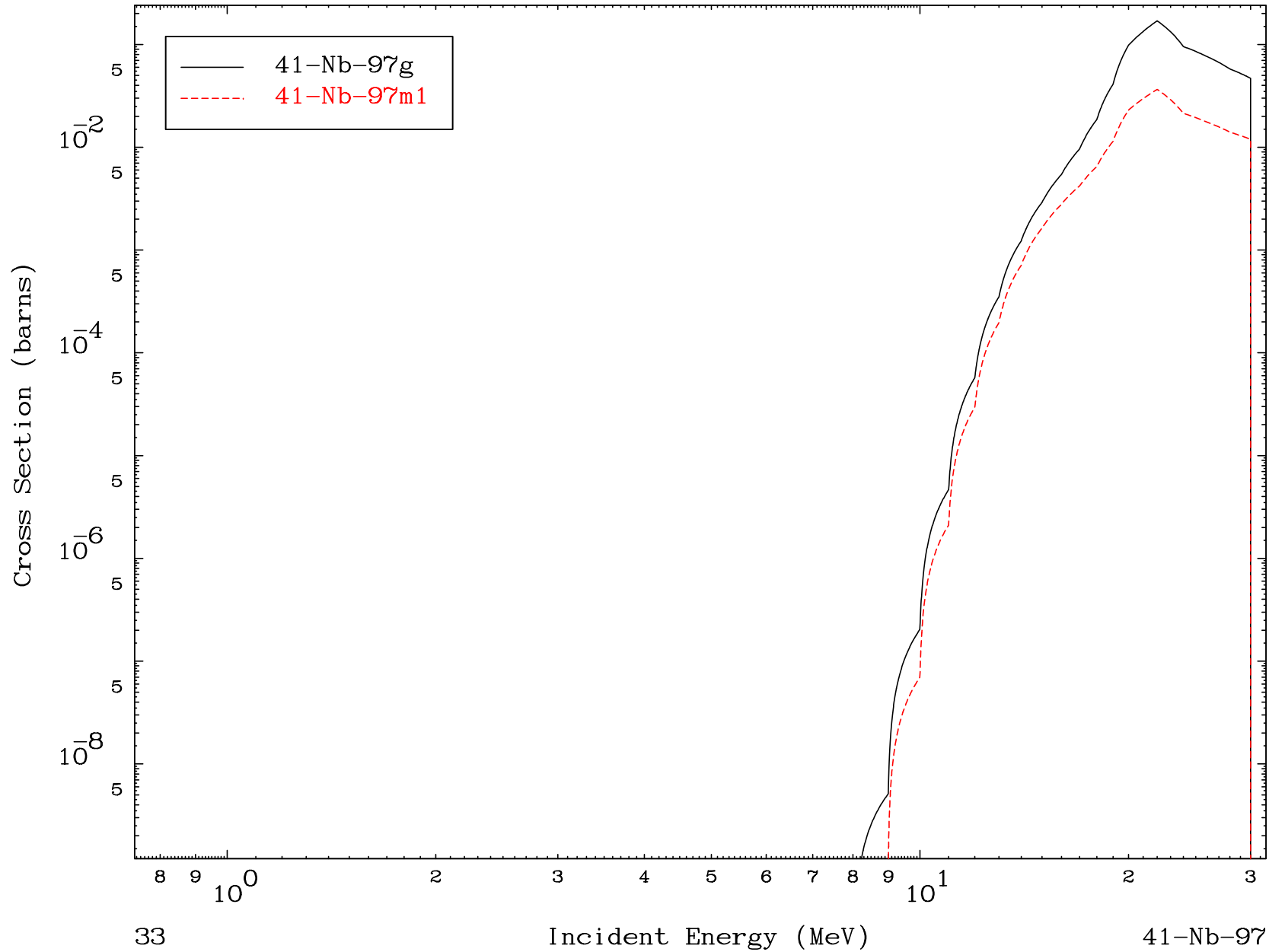
32

Incident Energy (MeV)

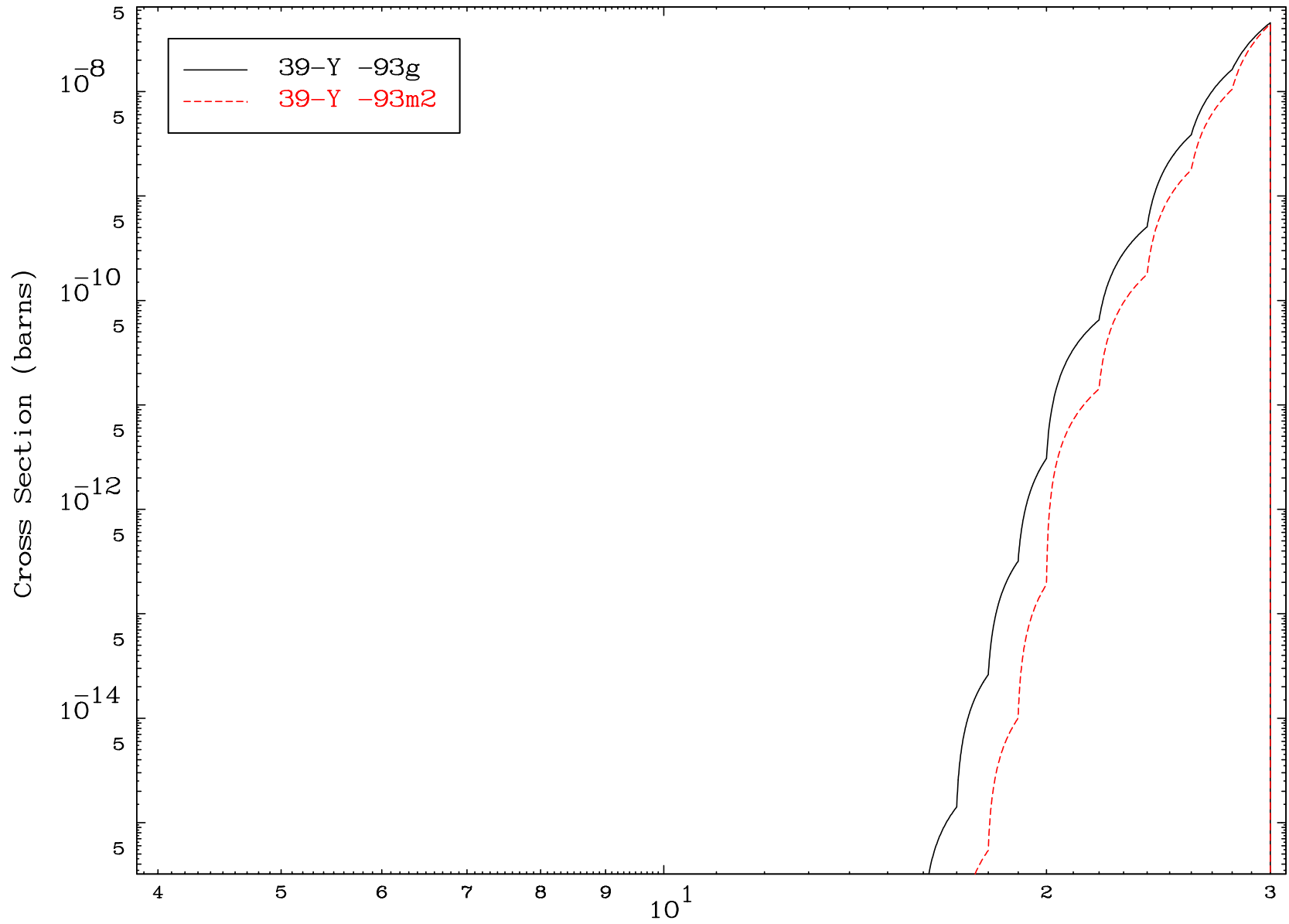
41-Nb-97



Radionuclide Production Cross Section



Radionuclide Production Cross Section



Radionuclide Production Cross Section

